NEXTera ENERGY DUANE ARNOLD

April 26, 2022

NG-22-0049

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555-0001

Duane Arnold Energy Center
Docket No. 50-331
Renewed Op. License No. DPR-49

<u>2021 Annual Radioactive Material Release Report</u>

Please find as the Enclosure to this letter, a copy of NextEra Energy Duane Arnold, LLC's (hereafter, NextEra Energy Duane Arnold) 2021 Annual Radioactive Material Release Report for the Duane Arnold Energy Center (DAEC). This report satisfies the requirements of Defueled Offsite Dose Assessment Manual (DODAM) Section 8.2.1 and Technical Specification Section 5.6.3.

This letter makes no new commitments nor changes to existing commitments.

Should you have any questions regarding this matter, please contact Michael Casey at (319) 851-7606.

Paul Hansen

Paul Hansen
Decommissioning Director
NextEra Energy Duane Arnold, LLC

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A009
NRR

Enclosure

cc: Administrator, Region III, USNRC
Project Manager, DAEC, USNRC
Inspector, DAEC, USNRC

Enclosure to NG-22-0049

Duane Arnold Energy Center
2021 Annual Radioactive Material Release Report

431 pages to follow

NEXTera ENERGY
DUANE ARNOLD

2021
Annual Radioactive Material Release Report

Duane Arnold Energy Center
Cedar Rapids, Iowa
Docket No. 50-331

January 1, 2021 through December 31, 2021

2021
Annual Radioactive Material Release Report

Duane Arnold Energy Center
DOCKET NUMBER. 50-331

Prepared By: J Dierickx Date: 4/11/2022
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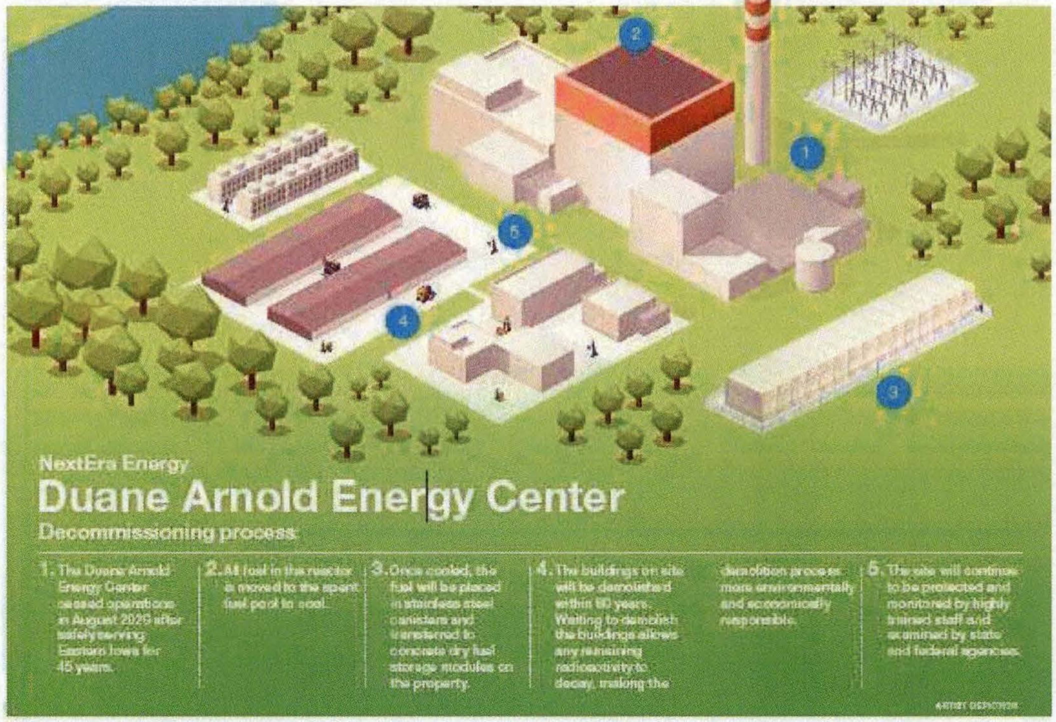
Approved By: Michael Casey Date: 4/11/22
Michael Casey, DAEC Chemistry Manager

Approved By: PHansen Date: 4-11-22
Paul Hansen – Decommissioning Director

NextEraEnergyResources.com

Duane Arnold Energy Center Decommissioning

Decommissioning is a well-defined Nuclear Regulatory Commission (NRC) process for shutting down a nuclear power plant. It includes transferring the used fuel from the Duane Arnold Energy Center into safe, long-term storage. The overall process is gradual, and is expected to be complete by the year 2080.



Emergency preparedness

After the plant shuts down, the possibilities for an emergency decrease dramatically. The Duane Arnold Energy Center will maintain capabilities to address any possibility of an emergency at the plant, no matter how unlikely, until all fuel has been placed in long-term storage. Once all the used fuel has been placed in long-term storage, the risk of an emergency that could affect the community is virtually eliminated. However, a security force will remain to protect the facility.

Funding

NextEra Energy Resources and the joint owners will pay for Duane Arnold's decommissioning. Nuclear power plants are required by the NRC to put aside funds for decommissioning while the plant is operating. The money is invested in dedicated trusts over the lifetime of the plant. The fund distribution is carefully monitored by a third party. NextEra Energy Resources estimates the decommissioning of Duane Arnold is fully funded.

Learn more: **NextEraEnergyResources.com/DuaneArnoldInfo**

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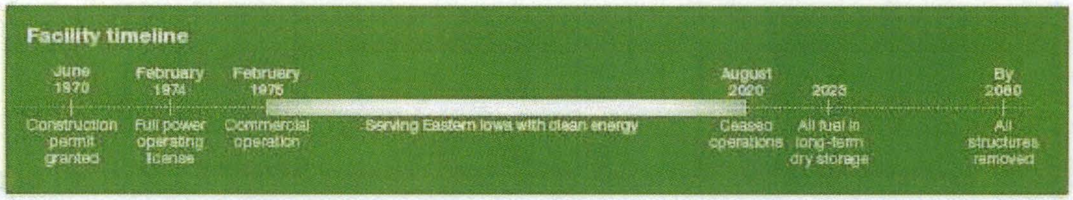

Frequently Asked Questions

What happens to the nuclear material on site?

The used fuel will be placed in long-term dry storage on site within three years of the plant shutting down.

The reactor building has accumulated some residual radioactive particles over the life of the plant. The buildings will remain vacant for 50 years before being demolished. The eventual demolition of the reactor building will be conducted with care and respect for the surrounding environment. Any other buildings remaining on the site will also be demolished at this time.

Why are you waiting so long to demolish the buildings?

While the Duane Arnold Energy Center is a very clean facility, the equipment used over the lifetime of the plant has residual radioactive particles. By maintaining the buildings, we allow time for the natural processes to reduce the intensity of the radioactive particles. When the time comes to take buildings down, it will be easier on the environment and more economical to remove material from the site.

The site will continue to be protected by a security force and monitored for any possible environmental impacts until all spent fuel is removed from the site.

NextEra Energy Resources believes it's important to operate its business in harmony with the environment and our neighbors. We take our responsibility to see that the site is maintained in a way that benefits the long term health of the community very seriously. We will continue to work with the NRC until the decommissioning process is complete.

What will happen to the site? What will they do there?

NextEra Energy Duane Arnold is evaluating redevelopment opportunities at the site, but it would be premature to speculate about what decisions might be made. We will continue to evaluate potential redevelopment opportunities going forward.

What happens to the employees?

Our goal from the very beginning of this process has been to minimize the impact on our employees and their families. We continue to work with each and every team member to help them prepare for the eventual shutdown.

To support Duane Arnold Energy Center employees during the transition, NextEra Energy Resources has developed a comprehensive employee plan that includes an enhanced retirement program for eligible employees, placement in other jobs throughout the company, support services which include dealing with change, preparing for retirement, writing resumes, job searches and career fairs. NextEra Energy Resources also is partnering with Alliant Energy and other companies in the energy industry to identify opportunities for employees.

What does staffing look like in the months and years after shutdown?

The station will still need a group of team members to monitor the plant and provide security for several years after operations cease and throughout the decommissioning process.

EXECUTIVE SUMMARY

The Duane Arnold Energy Center (DAEC) permanently shut down on August 10, 2020, and the reactor fuel has been placed in the spent fuel pool. By letter dated August 27, 2020 (Accession No. ML20240A067), NextEra Energy Duane Arnold (NEDA) certified permanent cessation of power operations at the DAEC. By letter dated October 12, 2020 (ML20286A317), NEDA certified permanent defueling of the reactor at DAEC. Therefore, as specified in 10 CFR 50.82(a)(2), the 10 CFR Part 50 license for DAEC no longer authorizes operations of the reactor or emplacement or retention of fuel into the reactor vessel.

The contribution of dose to a member of the public most likely to be exposed from liquid and gaseous effluent releases was calculated using the Meteorological Information and Dose Assessment System (MIDAS) computer program in accordance with the DODAM. The calculated doses for gaseous and liquid effluent releases are less than the regulatory limits stated in Appendix I to 10 CFR 50 and in 40 CFR 190.

Supportive environmental data for the 2021 dose assessment can be found in the Duane Arnold Energy Center 2021 Annual Radiological Environmental Operating Report.

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REGULATORY LIMITS

Fission and Activation Gases

Dose Rate

- Less than 500 mrem/year to the whole body.
- Less than 3000 mrem/year to the skin.

Gamma Air Dose

- Less than or equal to 5 mrad/quarter.
- Less than or equal to 10 mrad/year.

Beta Air Dose

- Less than or equal to 10 mrad/quarter.
- Less than or equal to 20 mrad/year.

Airborne Particulates, Iodines and Tritium

Dose Rate

- Less than 1500 mrem/year.

Dose

- Less than or equal to 7.5 mrem/quarter to any organ.
- Less than or equal to 15 mrem/year to any organ.

Liquid Effluents

Dose

- Less than or equal to 1.5 mrem to the whole body during any calendar quarter.
- Less than or equal to 5 mrem to any organ during any calendar quarter.
- Less than or equal to 3 mrem to the whole body during any calendar year.
- Less than or equal to 10 mrem to any organ during any calendar year.

Concentration

- Liquid effluents released from the site to unrestricted areas shall not exceed ten times (10x) the concentrations listed in Appendix B, Table 2, Column 2 to 10 CFR 20.1001 – 20.2402.

40CFR190 and 10CFR72

Dose

- Less than or equal to 25 mrem annual whole-body dose.
- Less than or equal to 75 mrem annual thyroid dose.
- Less than or equal to 25 mrem annual dose to any other critical organ.

MAXIMUM PERMISSIBLE CONCENTRATIONS

Dose rates, rather than effluent concentrations, are used to calculate permissible release rates for gaseous effluents. The maximum permissible dose rates for gaseous releases are defined in Duane Arnold Offsite Dose Assessment Manual (ODAM) and the Defueled Offsite Dose Assessment Manual (DODAM). ODAM-DODAM Limiting Condition for Operation (OLCO) 6.2.2. Liquid effluent concentrations are limited per ODAM-DODAM OLCO 6.1.2 to ten times (10x) the concentration specified in 10CFR20 Appendix B, Table 2, Column 2.

AVERAGE ENERGY

The ODAM-DODAM limits dose rates at or beyond the site boundary due to the release of noble gases to less than or equal to 500 mrem per year to the total body and less than or equal to 3,000 mrem per year to the skin. Average energy is not used to determine dose to the public. Compliance with these limits is demonstrated based on dose calculations using measured isotopic concentrations of effluent streams and not based on gross count rate measuring systems.

Therefore, the average beta and gamma energies (E-BAR) for gaseous effluents as described in Regulatory Guide 1.21 "Measuring, Evaluating, and Reporting Radioactivity in Solid Wastes and Releases of Radioactive Materials in Liquid and Gaseous Effluents from Light-Water-Cooled Nuclear Power Plants," are not applicable.

MEASUREMENTS OF TOTAL RADIOACTIVITY

Gaseous Effluents

- With the plant shutdown, DODAM gaseous effluent sampling was revised to reflect current plant conditions. In February 2021, the Offgas Stack was removed from service and subsequently removed from DODAM. In April, flow from the Turbine Building was discontinued, but may be returned to service later.

- With I-131 having an 8-day half-life, the DODAM was revised to remove analysis requirements from the Turbine Building and LLRW. With systems drained from the Turbine Building, the DODAM was revised to remove tritium analysis. With noble gases decayed, the DODAM was revised to remove noble gas sampling from Turbine Building and LLRW.

- Gaseous Effluent particulates from the Reactor Building Vents, and the Low Level Radwaste Storage and Processing Facility (LLRW), when are continuously sampled for particulates. Iodine samples are obtained from Reactor building vents. Sample filters are changed weekly and analyzed by gamma spectroscopy. The gross alpha analyses are performed onsite. The particulate filters are composited quarterly and sent to an outside lab for Sr-89, Sr-90, Fe-55 and Ni-63 analysis. Total error is calculated based on stack flow error, sample flow error, and analytical error.

- Noble gas grab samples from the Reactor Building Vents are obtained monthly and analyzed by gamma spectroscopy. Total error is based on stack flow error, analytical error, and calculated sampling error.

- Tritium samples from Reactor Building Vents and LLRW obtained quarterly and analyzed by liquid scintillation. Total error is based on stack flow error, analytical error, and calculated sampling error.

- A beta sensitive radiation detector provides continuous monitoring at the Reactor Building Vents. For the year 2021, there were no instances where ODAM-DODAM required gaseous radiation monitoring or sampling systems were inoperable on an active release point for a period of 30 (contiguous) days or more. (ODAM-DODAM OLCO 6.2.1.1 A.2)

Liquid Effluents

- Service water systems are sampled once per week for gamma emitting nuclides. Portions of the weekly service water samples are composited for a monthly analysis for tritium and gross alpha. If a positive identification of reactor by-product radioactivity in these samples is made, Sr-89, Sr-90 and Fe-55 analyses are performed. Total error is based on the volume discharge error and analytical error.

- ODAM-DODAM defined "Clean Systems" are sampled prior to batch release or continuous release with composite sampling. Samples are analyzed for gamma emitters and tritium. If reactor by-product gamma emitters are identified, analyses for Sr-89, Sr-90 and Fe-55 are performed. "Clean Systems" include CST Containment Pit, Transformer Pit, Neutralizing Tank 1T022, FRAC tanks, and temporary mitigation of groundwater in a monitoring well.

- NEDA continued discharging liquid radioactive effluent to the Cedar River. In 2021 there were ninety-nine radioactive batch releases of liquids from plant radwaste system using the ALARA filter enhancement in 2021. The primary isotopes released were H-3, Mn-54, Co-60, Zn-65, Cs-134 and Cs-137.

- Liquid radioactive effluents are controlled to keep the Whole-Body dose and Organ dose to less than 10% of the regulatory limits of 3 mRem and 10 mRem respectively.

- Continuous monitoring with gamma sensitive radiation detectors is provided for plant service water systems. For the year 2021, there were no instances where these liquid radiation monitoring systems were inoperable on an active release point for a period of 30 (contiguous) days or more. (ODAM-DODAM OLCO 6.1.1.1 A.2)

- The permanent groundwater mitigation system is designed to remove tritiated groundwater from the shallow aquifer. The composite sample is collected once per week and analyzed for gamma emitters and tritium. If reactor by-product gamma emitters are identified, analyses for Sr-89, Sr-90 and Fe-55 are performed. Total error is based on the volume discharge error and analytical error. For 2021, there were eleven continuous releases from the permanent GWPP mitigation system in which a plant by-product was identified above the lower level of detection. The releases contained only tritium with no plant by-product gamma emitters identified.

- Temporary groundwater mitigation was performed during three quarters of 2021 from select monitoring wells that indicated elevated tritium concentrations. Releases were discontinued in the third quarter due to equipment due to equipment issues and protect the piping from freezing. Releases from the Temporary ground water system were included with the permanent groundwater system. All releases contained only tritium with no plant by-product gamma emitters identified.

- Sewage effluent releases are sampled under the auspices of the Groundwater Protection Program. The sewage sample results from on-site laboratory are included in Attachment 1A. All other Groundwater Protection Program analyses and results can be found in the Duane Arnold Energy Center 2021 Annual Radiological Environmental Operating Report.

SUMMARY OF GASEOUS EFFLUENTS

There were no radioactive gaseous batch releases from the Duane Arnold Energy Center during this report period. All gaseous effluent releases were continuous and resulted in a small fraction of the 10 CFR 50, Appendix I dose limits.

No radioactive noble gas or iodine was detected in gaseous effluents during 2021. For all release points, quarterly average gross alpha concentration of radioactivity measured less than 1.6E-12 µCi/cc.

There were no abnormal releases of gaseous effluents during the period. Gaseous effluent sampling was revised in DODAM, Revision 2. Specific changes are discussed in the Summary of Changes to the Offsite Dose Assessment Manual (DODAM) section.

Table 1A - Gaseous Effluents – Summation of All Releases

	Units	1st Quarter	2nd Quarter	3rd Quarter	4th Quarter	Est. Total Error, %
Fission and Activation Gases						
1. Total Release	Ci	0.00E+00	0.00E+00	0.00E+00	0.00E+00	1.60E+01
2. Average Release Rate for Period	µCi/sec	0.00E+00	0.00E+00	0.00E+00	0.00E+00	
3. Percent of Applicable Limit	%	*	*	*	*	
Iodines						
1. Total I-131	Ci	0.00E+00	0.00E+00	0.00E+00	0.00E+00	1.80E+01
2. Average Release Rate for Period	µCi/sec	0.00E+00	0.00E+00	0.00E+00	0.00E+00	
3. Percent of Applicable Limit	%	*	*	*	*	
Particulates						
1. Total Particulates w/ half-life >8 days	Ci	2.70E-05	2.22E-05	4.09E-05	1.83E-05	1.80E+01
2. Average Release Rate for Period	µCi/sec	3.47E-06	2.81E-06	5.15E-06	2.30E-06	
3. Percent of Applicable Limit	%	*	*	*	*	
Tritium						
1. Total Release	Ci	6.84E-01	3.50E-01	4.93E-01	5.70E-01	1.60E+01
2. Average Release Rate for Period	µCi/sec	8.82E-02	4.45E-02	6.2 E-02	7.18E-02	
3. Percent of Applicable Limit	%	*	*	*	*	
Carbon-14						
1. Total Carbon-14	Ci	0.00E+00	0.00E+00	0.00E+00	0.00E+00	
2. Average Release Rate for Period	µCi/sec	0.00E+00	0.00E+00	0.00E+00	0.00E+00	
3. Percent of Applicable Limit	%	*	*	*	*	

* Applicable limits have been removed from the Technical Specifications. The comparison to DODAM limits is contained in the Radiological Impact on Man section of this report.

Table 1B - Gaseous Effluents by Quarter

ANNUAL RADIOACTIVE MATERIAL RELEASE REPORT (2021)					
GASEOUS EFFLUENTS BY CALENDAR QUARTER (Curies)					
Nuclides Released	Unit	1st Quarter	2nd Quarter	3rd Quarter	4th Quarter
1. Fission gases[2]					
krypton-85	Ci	ND[1]	ND	ND	ND
krypton-85m	Ci	ND	ND	ND	ND
krypton-87	Ci	ND	ND	ND	ND
krypton-88	Ci	ND	ND	ND	ND
xenon-131m	Ci	ND	ND	ND	ND
xenon-133	Ci	ND	ND	ND	ND
xenon-133m	Ci	ND	ND	ND	ND
xenon-135	Ci	ND	ND	ND	ND
xenon-135m	Ci	ND	ND	ND	ND
xenon-138	Ci	ND	ND	ND	ND
argon-41	Ci	ND	ND	ND	ND
Total for period	Ci	**0.00E00**	**0.00E00**	**0.00E00**	**0.00E00**
2. Iodines[2]					
iodine-131	Ci	ND	ND	ND	ND
iodine-133	Ci	ND	ND	ND	ND
iodine-135	Ci	ND	ND	ND	ND
Total for period	Ci	**0.00E00**	**0.00E00**	**0.00E00**	**0.00E00**
3. Particulates					
strontium-89	Ci	1.37E-06	3.46E-08	ND	ND
strontium-90	Ci	ND	ND	ND	ND
cesium-134	Ci	4.29E-08	4.00E-07	ND	ND
cesium-137	Ci	ND	5.81E-6	ND	ND
barium-lanthanum-140	Ci	ND	ND	ND	ND
chromium-51	Ci	ND	ND	ND	ND
cobalt-58	Ci	3.52E-07	ND	ND	ND
cobalt-60	Ci	1.43E-05	4.83E-06	1.79E-05	4.91E-06
manganese-54	Ci	4.64E-06	1.01E-5	5.84E-06	8.79E-06
iron-55	Ci	6.26E-06	8.98E-07	1.04E-05	3.75E-6
iron-59	Ci	ND	ND	ND	ND
nickel-63	Ci	4.85E-06	3.32E-06	9.22E-07	5.11E-07
zinc-65	Ci	ND	ND	6.84E-06	8.36E-07
zinc-69m	Ci	ND	ND	ND	ND
Total for period	Ci	**3.19E-05**	**2.54E-05**	**4.18E-05**	**1.88E-05**
4. Tritium	Ci	6.84E-01	3.50E-01	4.93E-01	5.70E-01
Tritium Total	Ci	**6.84E-01**	**3.50E-01**	**4.93E-01**	**5.70E-01**
5. Carbon-14	Ci	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Carbon-14 Total	Ci	0.00E+00	0.00E+00	0.00E+00	**0.00E+00**

1. ND indicates the radionuclide was not identified in any samples using instrumentation that meets the lower limit of detection as required by the DAEC Offsite Dose Assessment Manual.
2. With the exception of Kr-85, noble gases and iodines have decayed to undetectable concentrations.

Table 1C - Gaseous Effluents by Release Point

ANNUAL RADIOACTIVE MATERIAL RELEASE REPORT (2021)
GASEOUS EFFLUENTS BY RELEASE POINT (Curies)

RELEASE POINT:	OFFGAS STACK[1]	REACTOR BUILDING	TURBINE BUILDING[2]	LLRPSF
RELEASE HEIGHT:	328 FEET	156 FEET	90 FEET	65 FEET
RELEASE MODE:	ELEVATED	WAKE SPLIT	WAKE SPLIT	WAKE SPLIT
barium-140	ND	ND	ND	ND
cesium-134	4.29E-08	4.00E-07	ND	ND
cesium-137	ND	8.26E-07	4.99E-06	ND
chromium-51	ND	ND	ND	ND
cerium-141	ND	ND	ND	ND
cobalt-58	ND	3.52E-07	ND	ND
cobalt-60	ND	4.19E-05	ND	ND
iodine-131	ND	ND	ND	ND
iodine-133	ND	ND	ND	ND
iron-55	ND	2.13E-05	ND	ND
iron-59	ND	ND	ND	ND
krypton-85	ND	ND	ND	ND
manganese-54	ND	2.34E-05	ND	ND
nickel-63	ND	8.95E-06	3.28E-10	5.83E-07
strontium-89	ND	1.20E-06	3.46E-08	1.99E-08
strontium-90	ND	ND	ND	ND
tritium	1.48E-01	1.87E+00	3.74E-02	ND
zinc-65	ND	7.68E-06	ND	ND
zinc-69m	ND	ND	ND	ND

ND means that the radionuclide was not identified in any samples using instrumentation that meets the lower limit of detection as required by the DAEC Offsite Dose Assessment Manual.

1. Offgas effluent flow was discontinued in February 2021.
2. Turbine Building effluent flow was discontinued in April 2021.

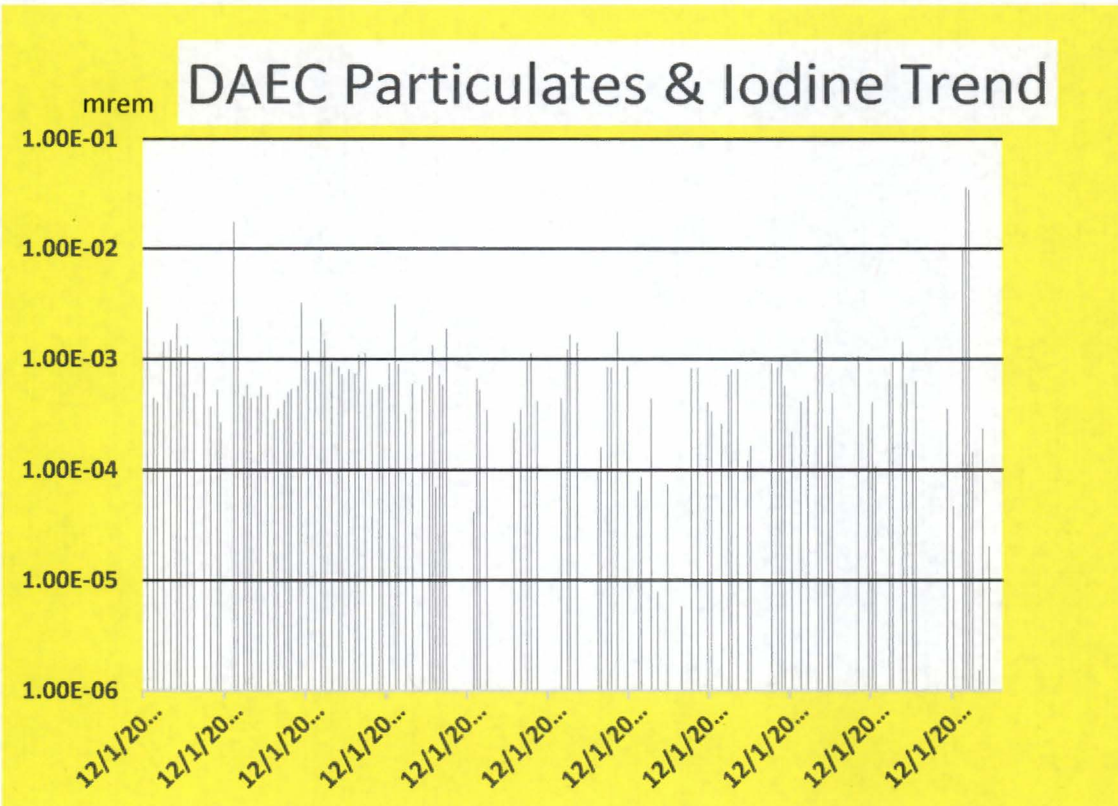
Estimated Release of Gaseous Carbon-14

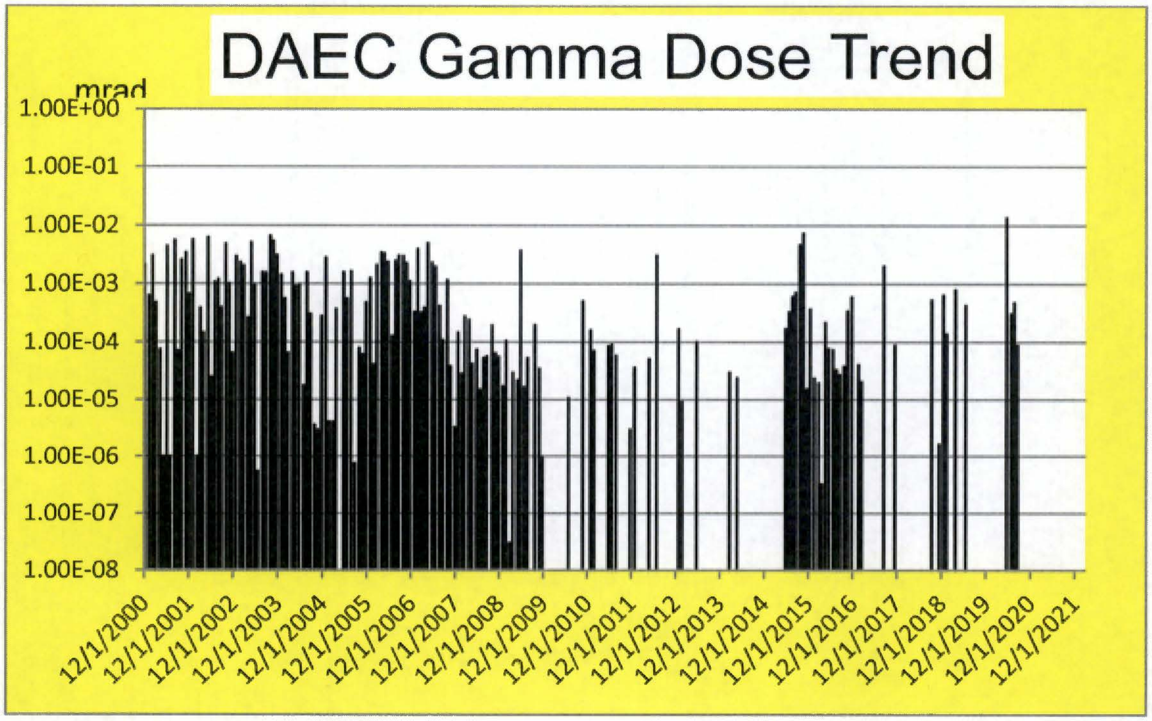
The estimate of gaseous carbon-14 (C-14) released from the Duane Arnold Energy Center was derived using the EPRI document, "Estimation of Carbon-14 in Nuclear Power Plant Gaseous Effluents", Report 1021106, issued December 2010. The site-specific source term for the DAEC was estimated using the proxy generation rate values from Table 3-1 and the actual 2021 power history for the site. Power production permanently ceased on August 10, 2020.

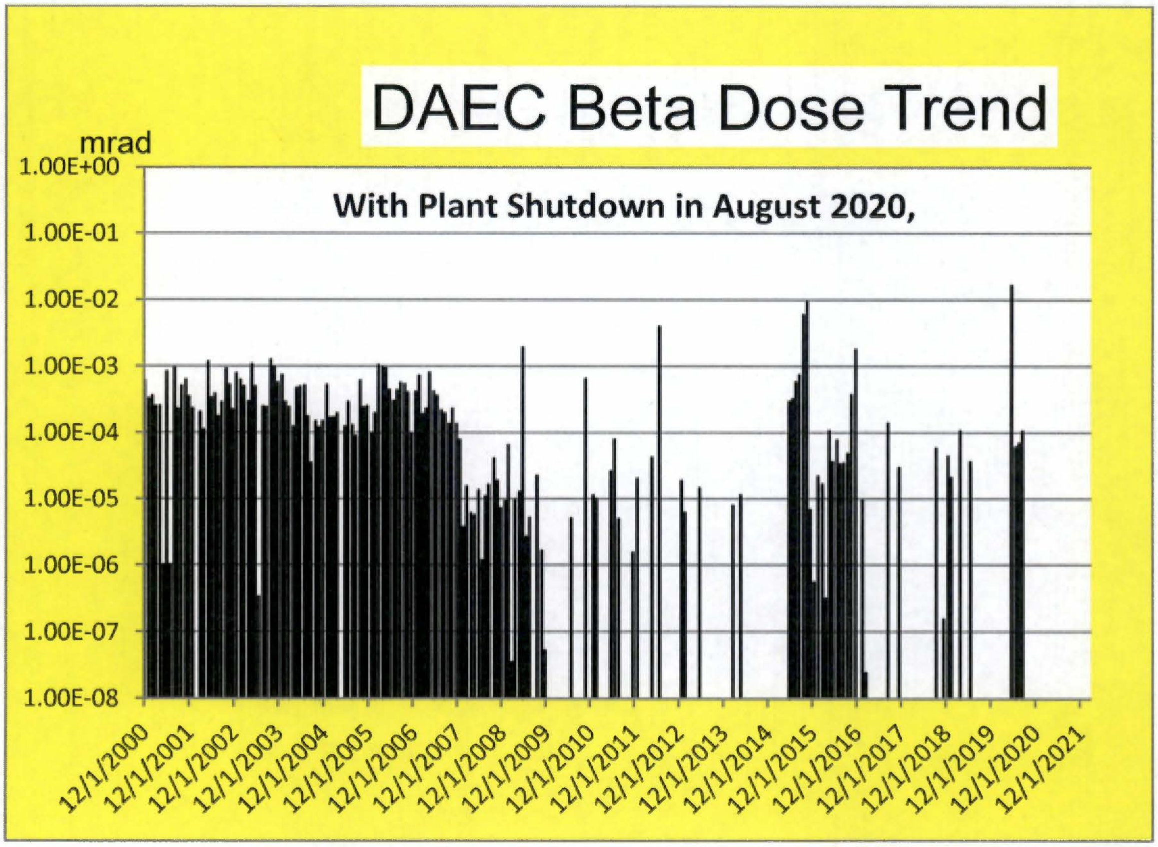
The total amount of C-14 released from the site in 2021 was estimated to be 0.0 curies as no power operation occurred in 2021. Therefore, no dose attributed to C-14 was calculated.

Gaseous Effluents Summary Trend in Curies Per Year Excluding Carbon-14

The following graphs show gaseous effluent dose trends for monthly particulate and iodine, monthly gamma, and monthly beta.







With the plant shutting down in August 2020, most noble gases have decayed and not detected in 2021. Therefore, Beta and Gamma dose was zero.

Comparative analysis of 2000-2021 particulate and iodine data indicates the 2021 gaseous effluent release trends continue to be less than the 10 CFR 50 Appendix I limits.

SUMMARY OF LIQUID EFFLUENTS

Liquid effluent release in the form of service water from the facility was continuous during the period. No reactor by-product radionuclides were identified in samples from service water or sewage effluent. There were ninety-nine liquid batch releases from the radioactive waste system. Batch releases included Tritium, Manganese-54, Cobalt-58, Cobalt-60, Iron-55, Nickel-63, Zinc-65, Cesium-134 and Cesium-137.

While continuous, only eleven groundwater mitigation samples contained reactor by-product material. Tritium was the only reactor by-product identified.

Table 2A - Liquid Effluents – Summation of All Releases

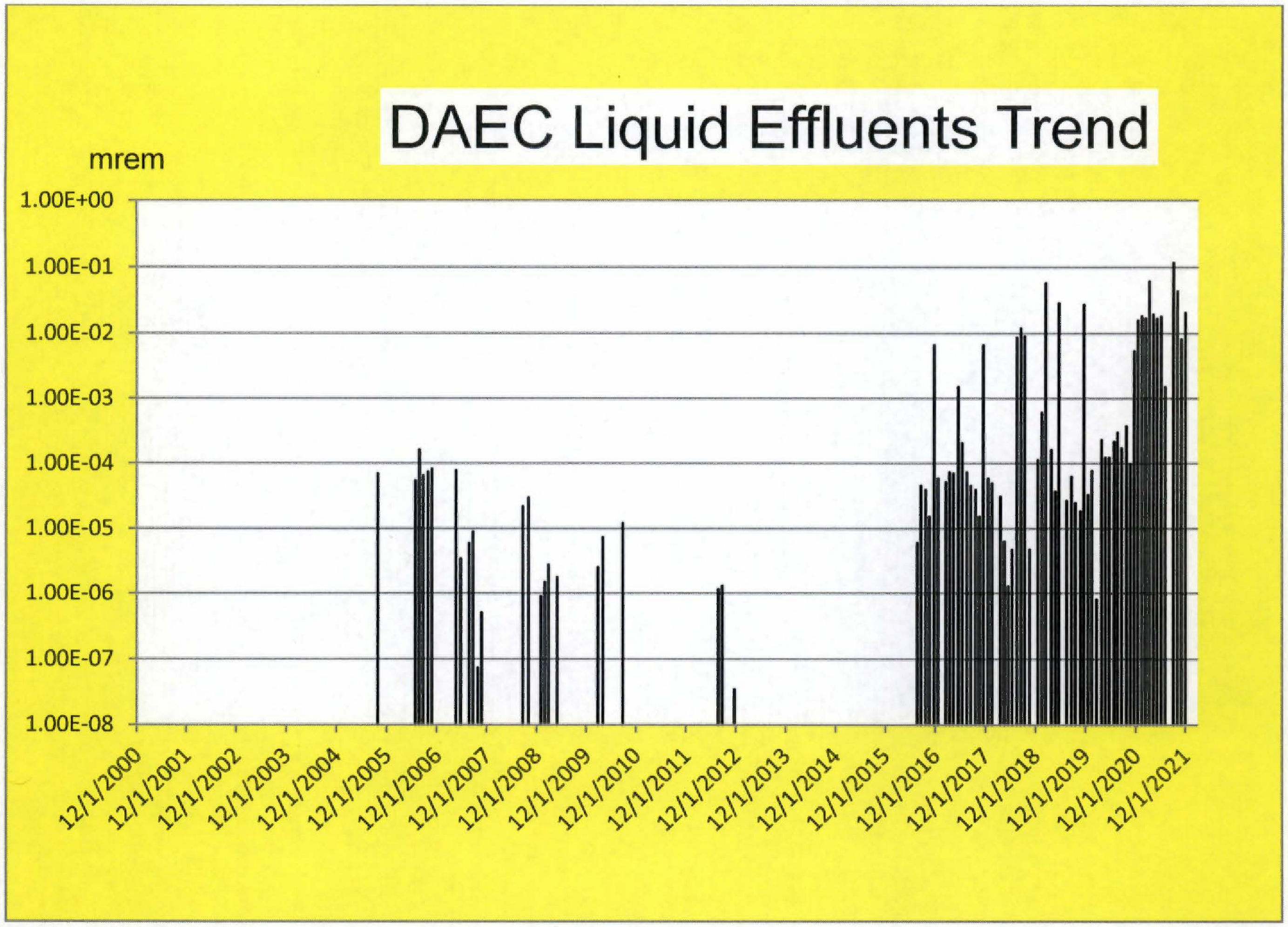
		1st Quarter	2nd Quarter	3rd Quarter	4th Quarter	Est. Total Error, %
Fission and Activation Products						
1. Total Release (not including Tritium, gases, alpha)	Ci	2.4E-05	4.77E-05	2.03E-03	6.11E-03	2.00E+01
2. Average Release Rate for Period	µCi/ml	3.22E-11	8.39E-11	1.71E-08	7.35E-09	
3. Percent of Applicable Limit	%	*	*	*	*	
Tritium						
1. Total Release	Ci	7.75E+00	7.37E-00	6.23E-01	2.79E+00	2.00E+01
2. Average Release Rate for Period	µCi/ml	1.04E-05	1.3E-05	5.25E-06	3.36E-6	
3. Percent of Applicable Limit	%	*	*	*	*	
Dissolved and Entrained Gases						
1. Total Release	Ci	0.00E+0	0.00E+0	0.00E+0	0.00E+0	2.00E+01
2. Average Release Rate for Period	µCi/ml	0.00E+0	0.00E+0	0.00E+0	0.00E+0	
3. Percent of Applicable Limit	%	*	*	*	*	
Gross Alpha Radioactivity						
1 Total Release	CI	1.12E-05	ND	1.13E-06	ND	2.00E+01
Volume of Water Released (Prior To Dilution)	Liters	2.21E+06	3.34E+06	2.74E-05	7.39E+05	
Volume of Dilution Water Used During Period	Liters	9.21E+08	8.90E+08	1.43E+08	8.32E+08	

* Applicable limits have been removed from the Technical Specifications. The comparison to ODAM limits is contained in the Radiological Impact on Man section of this report.
ND – Not Detected

Table 2B - Liquid Effluents

ANNUAL RADIOACTIVE MATERIAL RELEASE REPORT (2021) LIQUID EFFLUENTS (Curies)					
Nuclides Released	Unit	1st Quarter	2nd Quarter	3rd Quarter	4th Quarter
strontium-89	Ci	ND	ND	ND	ND
strontium-90	Ci	ND	ND	ND	ND
cesium-134	Ci	ND	ND	1.44e-04	4.16e-05
cesium-137	Ci	ND	ND	2.42e-04	1.00e-04
iodine-131	Ci	ND	ND	ND	ND
cobalt-58	Ci	ND	ND	ND	2.53E-06
cobalt-60	Ci	7.34E-06	3.38E-05	4.01E-04	1.58E-03
iron-55	Ci	ND	ND	7.38E-04	1.71E-03
iron-59	Ci	ND	ND	ND	ND
zinc-65	Ci	ND	ND	6.13E-04	1.51E-03
manganese-54	Ci	1.67E-05	1.39E-05	1.96E-04	1.06E-03
chromium-51	Ci	ND	ND	ND	ND
zirconium-niobium-95	Ci	ND	ND	ND	ND
molybdenum-99	Ci	ND	ND	ND	ND
technetium-99m	Ci	ND	ND	ND	ND
barium-lanthanum-140	Ci	ND	ND	ND	ND
cerium-141	Ci	ND	ND	ND	ND
other	Ci	ND	ND	5.49E-05	1.15E-04
tritium	Ci	7.75E+00	7.37E+00	6.22E-01	2.79E+00
xenon-133	Ci	ND	ND	ND	ND
xenon-135	Ci	ND	ND	ND	ND

1. ND means that the radionuclide was not identified in any samples using instrumentation that meets lower limit of detection as required by the DAEC Offsite Dose Assessment Manual.


DAEC Liquid Effluents Trend
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1.00E+00
1.00E-01
1.00E-02
1.00E-03
1.00E-04
1.00E-05
1.00E-06
1.00E-07
1.00E-08
12/1/2000
12/1/2001
12/1/2002
12/1/2003
12/1/2004
12/1/2005
12/1/2006
12/1/2007
12/1/2008
12/1/2009
12/1/2010
12/1/2011
12/1/2012
12/1/2013
12/1/2014
12/1/2015
12/1/2016
12/1/2017
12/1/2018
12/1/2019
12/1/2020
12/1/2021

SUMMARY OF RADIOACTIVE SOLID WASTE

Seven solid radioactive waste shipments occurred during 2021.

Two shipments of spent fuel pool artifacts were sent to Andrews, Texas for burial.

Five shipments of Dry Active Waste (DAW) were shipped for processing and subsequent burial in 2021. Shipments were made to Energy Solutions in Bear Creek, Tennessee and Unitech Services Group in Oak Ridge, Tennessee.

There were no shipments of liquid radwaste in 2021.

Shipments in 2021

Listed below are tables summarizing the Duane Arnold Energy Center's generation of Radioactive Solid Waste for the period of January 1, 2021 through December 31, 2021.

Shipments Made To Burial Facilities in 2021

WASTE TYPE	NO. SHIPMENTS	VOLUME (ft^3)	VOLUME (m^3)	ACTIVITY (curies)
Dry Active Waste	2	1.35E+01	3.82E-01	2.20e+04
DESTINATION	Shipments Made to: Waste Control Specialist in Andrews, TX			

See Table 3A below for Major Nuclides

Shipments Made To Processing Facilities in 2021

WASTE TYPE	NO. SHIPMENTS	VOLUME (ft^3)	VOLUME (m^3)	ACTIVITY (curies)
Dry Active Waste	5	9.63E+03	2.73E+02	7.05E-01
DESTINATION	Energy Solutions Bear Creek Facility in Oakridge TN Unitech Services Group in Oak Ridge TN			

See Table 3B below for Major Nuclides

Total Solid Waste Disposition for 2021

WASTE	VOLUME (ft^3)	VOLUME (m^3)	ACTIVITY (curies)
Shipped	9.64E+03	2.73E+02	2.20e+04
Buried	9.64E+03	2.73E+02	2.20e+04

SOLIDIFICATION AGENT: None
MODE OF TRANSPORTATION: Exclusive-Use Vehicle (Trucks).
IRRADIATED COMPONENTS: There were no shipments of irradiated components or nuclear fuel in 2021.
See Table 3C below for Annual Summary of Major Nuclides

Waste Classification per 10 CFR 61	NUMBER OF SHIPMENTS IN 2021
A-Unstable	5
A-Stable	0
B	0
C	2

Site Historical Comparison

Year	Volume Buried(ft3)	Activity (Ci)
2008	5.42E+03	134
2009	1.16E+04	58
2010	1.14E+04	23
2011	7.26E+03	324
2012	2.48E+04	58
2013	7.19E+03	52
2014	2.70E+04	33
2015	6.68E+03	48
2016	1.02E+03	43
2017	1.08E+04	156
2018	1.59E+04	115
2019	2.78E+03	171
2020	4.86E+03	157
2021	9.64E+03	22000

Summary of Buried Radioactive Solid Waste

January 1, 2021 - December 31, 2021

Table 3A Spent Resin Major Nuclide Composition

Principle Nuclide	1st QTR (mCi)	2nd QTR (mCi)	3rd QTR (mCi)	4th QTR (mCi)	Total (mCi)	Percent Abundance
H-3	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
C-14	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
K-40	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cr-51	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Mn-54	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Fe-55	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-57	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-58	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Ni-59	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Fe-59	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-60	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Ni-63	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Zn-65	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Sr-89	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Sr-90	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Sr-91	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Zr-95	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Nb94	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Nb-95	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Mo-93	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Tc-99	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Ag-110m	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Sn-113	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Sb-124	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Sb-125	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
I-125	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
I-129 (LLD)	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-134	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-137	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Ce-144	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
U-235	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
PU-238	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
PU-239	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00

Table 3A Spent Resin Major Nuclide Composition						
Principle Nuclide	1st QTR (mCi)	2nd QTR (mCi)	3rd QTR (mCi)	4th QTR (mCi)	Total (mCi)	Percent Abundance
Pu-240	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Pu-241	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Am-241	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
AM-243	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cm-242	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cm-243	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cm-244	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Totals	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00

Summary of Radioactive Solid Waste - Dry Active Waste

January 1, 2021 - December 31, 2021

Table 3B Dry Active Waste Major Nuclide Composition

Principle Nuclide	1st QTR (mCi)	2nd QTR (mCi)	3rd QTR (mCi)	4th QTR (mCi)	Total (mCi)	Percent Abundance
H-3	7.37E-02	2.60E-01	0.00E+00	6.58E-01	9.92E-01	0.1406%
C-14	7.14E-03	2.52E-02	0.00E+00	6.36E-02	9.59E-02	0.0136%
K-40	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.0000%
Cr-51	1.44E+00	6.05E+00	0.00E+00	1.55E+01	2.30E+01	3.2668%
Mn-54	3.24E+00	1.16E+01	0.00E+00	2.94E+01	4.43E+01	6.2776%
Fe-55	4.00E+01	1.42E+02	0.00E+00	3.59E+02	5.41E+02	76.6809%
Co-57	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.0000%
Co-58	1.71E-01	6.57E-01	0.00E+00	1.67E+00	2.50E+00	0.3542%
Ni-59	3.09E-02	1.09E-01	0.00E+00	2.76E-01	4.16E-01	0.0590%
Fe-59	4.81E-01	1.91E+00	0.00E+00	4.87E+00	7.26E+00	1.0297%
Co-60	4.77E+00	1.69E+01	0.00E+00	4.26E+01	6.43E+01	9.1144%
Ni-63	1.32E+00	4.65E+00	0.00E+00	1.17E+01	1.77E+01	2.5096%
Zn-65	1.42E-01	5.14E-01	0.00E+00	1.30E+00	1.96E+00	0.2777%
Sr-89	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.0000%
Sr-90	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.0000%
Sr-91	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.0000%
Zr-95	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.0000%
Nb94	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.0000%
Nb-95	7.86E-03	3.20E-02	0.00E+00	8.19E-02	1.22E-01	0.0173%
Mo-93	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.0000%
Tc-99 (LLD)	9.28E-03	3.26E-02	0.00E+00	8.23E-02	1.24E-01	0.0176%
Ag-110m	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.0000%
Sn-113	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.0000%
Sb-124	5.47E-02	2.12E-01	0.00E+00	5.40E-01	8.06E-01	0.1144%
Sb-125	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.0000%
I-125	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.0000%
I-129 (LLD)	1.59E-03	5.59E-03	0.00E+00	1.35E-02	2.07E-02	0.0029%
Cs-134	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.0000%
Cs-137	2.53E-02	9.21E-02	0.00E+00	2.25E-01	3.43E-01	0.0486%
Ce-144	3.86E-02	1.37E-01	0.00E+00	3.53E-01	5.29E-01	0.0750%
U-235	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.0000%
PU-238	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.0000%

Table 3B Dry Active Waste Major Nuclide Composition						
Principle	1st QTR	2nd QTR	3rd QTR	4th QTR	Total	Percent
Nuclide	(mCi)	(mCi)	(mCi)	(mCi)	(mCi)	Abundance
PU-239	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.0000%
Pu-240	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.0000%
Pu-241	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.0000%
Am-241	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.0000%
AM-243	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.0000%
Cm-242	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.0000%
Cm-243	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.0000%
Cm-244	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.0000%
Totals	5.18E+01	1.85E+02	0.00E+00	4.68E+02	7.05E+02	100.0%

Summary of Radioactive Solid Waste - Annual Summary

January 1, 2021 - December 31, 2021

Table 3C Radwaste Annual Summary Major Nuclide Composition

Nuclide	Curies
H-3	4.83E-02
C-14	1.00E+00
K-40	0.00E+00
Cr-51	8.56E-02
Mn-54	3.35E+02
Fe-55	1.09E+04
Co-57	0.00E+00
Co-58	2.22E+01
Ni-59	1.22E+01
Fe-59	6.12E-01
Co-60	1.01E+04
Ni-63	6.55E+02
Zn-65	8.05E+00
Sr-89	0.00E+00
Sr-90	1.92E-01
Sr-91	0.00E+00
Zr-95	2.29E-06
Nb94	9.96E-03
Nb-95	1.22E-04
Mo-93	2.41E-05
Tc-99	1.37E-02
Ag-110m	0.00E+00
Sn-113	0.00E+00
Sb-124	8.06E-04
Sb-125	1.08E+01
I-125	0.00E+00
I-129 (LLD)	1.34E-04
Cs-134	0.00E+00
Cs-137	2.16E-01
Ce-144	1.11E+00
U-235	1.31E+01
PU-238	4.29E-03
PU-239	1.94E-06
Pu-240	1.65E-06

Nuclide	Curies
Pu-241	1.46E-04
Am-241	3.18E-06
AM-243	9.18E-09
Cm-242	7.36E-07
Cm-243	1.25E-08
Cm-244	4.48E-07
Total	**2.20E+04**

RADIOLOGICAL IMPACT ON MAN

The annual offsite radiation dose to a member of the public was determined by assessment of environmental dosimetry results, and calculations based on monitored effluent releases.

Dose Contribution from Direct Radiation

Direct radiation dose from the operation of the DAEC was captured by TLDs placed at locations in the surrounding environment as described in the ODAM-DODAM. Pre-operational and 2021 TLD results were evaluated with a paired difference statistical test. The evaluation concluded that there were no significant differences in the TLD populations for the 0.5-mile, 1 mile and Control TLD populations. No measurable dose due to the operation of the DAEC was detected by environmental TLDs in 2021. In addition, compliance with 40 CFR 190 limits of 25 mrem whole body and 75 mrem thyroid is demonstrated in the Duane Arnold Energy Center 2021 Annual Radiological Environmental Operating Report, subsections "Ambient Radiation (TLDs)" and "ISFSI Facility Operations Monitoring".

Estimated Offsite Dose from Effluent Releases

The contribution of dose to a member of the public most likely to be exposed from liquid and gaseous effluent releases was calculated using the MIDAS computer program in accordance with the DODAM. Calculation methods follow those prescribed by Regulatory Guide 1.109, "Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents for the Purpose of Evaluating Compliance with 10 CFR Part 50, Appendix I".

Results of the MIDAS dose calculations are below:

1.) There were ninety-nine batch and continuous releases of radioactive material to liquid effluents in 2021. The maximum dose from liquid releases to an adult total body was 0.202 mrem and maximum organ dose to a teen liver was 0.315 mrem.

2.) As no noble gases were detected in 2021, the maximum gamma air dose from noble gases released was 0.0 mrad located at 481 meters towards the South-Southeast from plant centerline.

3.) As no noble gases were detected in 2021, the maximum beta air dose from noble gases released was 0.0 mrad at 481 meters towards the South-Southeast from the plant centerline.

4.) The whole-body dose equivalent to the hypothetical maximally exposed individual from noble gases was 0.0010 mrem at 805 meters towards the West from the plant centerline.

5.) The skin dose equivalent to the hypothetical maximally exposed individual from noble gases was 0.0 mrem at 805 meters towards the West from the plant centerline.

6.) The hypothetical maximally exposed organ due to airborne iodines and particulates with half-lives greater than eight days (excluding carbon-14) was 0.00103 mrem to the Lung of a child at 805 meters towards the West from the centerline of the plant.

7.) Since the Carbon-14 curie calculation is based on reactor power and the plant shutdown since August 10, 2020, no calculation for C-14 was performed. Zero was entered for dose.

CONCLUSION

The contribution of dose to a member of the public most likely to be exposed from liquid and gaseous effluent releases was calculated using the MIDAS computer program in accordance with the ODAM-DODAM and site procedures. The calculated doses for gaseous and liquid effluent releases are below the regulatory limits stated in 10 CFR 50, Appendix I and in 40 CFR 190.

Estimated Maximum Offsite Individual Doses for 2021

Type	Age Group	Distance (meters)	Direction	Dose or Dose Equivalent (mrem)	Annual 10 CFR 50, Appendix I "Limit"
Direct Radiation (as measured by TLDs)				None	*
Liquid Releases					
Whole Body Dose	Adult	D[1]	SE	0.202 mrem	3 mrem
Organ Dose	Child - Liver	D[1]	SE	0.315 mrem	10 mrem
Noble Gas					
Gamma Air Dose		481	SSE	0.0 mrad	10 mrad
Beta Air Dose		481	SSE	0.0 mrad	20 mrad
Whole Body	All	805	W	0.0 mrem	5 mrem
Skin	Child	805	W	0.0 mrem	15 mrem
Particulates & Iodines					
Organ Dose	Child – Lung	805	W	0.00103 mrem	15 mrem
Carbon 14					
Organ Dose	Child – Bone	1,760	N	0.0 mrem	15 mrem

* There is no Appendix I limit for direct radiation. Compliance with 40 CFR 190 limits of 25 mrem whole body and 75 mrem thyroid is demonstrated in the Duane Arnold Energy Center 2021 Annual Radiological Environmental Operating Report, subsections "Ambient Radiation (TLDs)" and "ISFSI Facility Operations Monitoring".

D[1] Receptor location is aquatic pathway at Cedar River. See ODAM-DODAM, Figure 3-2.

SUMMARY OF METEOROLOGICAL DATA

The following pages are a summation of meteorological data accumulated during the 2021 calendar year by the MIDAS software at the Duane Arnold Energy Center.

Greater than 90% data recovery was obtained for combined wind speed, delta temperature and wind direction. The table below summarizes 2021 data collection results:

Elevation and Sensors	Joint Recovery Valid Data
10-meter Wind Direction Wind Speed Delta Temp	90.4%
50-meter Wind Direction Wind Speed Delta Temp	90.4%

Plant process computer issues caused most of the invalid data.

The following pages contain wind rose plots and stability class summary tables for the specified sensor heights (33 feet or 156 feet). Joint Frequency tables for each of the individual stability classes are maintained on site and are available upon request.

Stability Class Data 33'
Joint Frequency Distribution

Hours at Each Wind Speed and Direction

Period of Record = **Total Period** 01/01/2021 00:00 - 01/01/2022 00:00 **All Hours**

Elevation: **Speed:** WS33 **Direction:** WD33 **Lapse:** DEL T

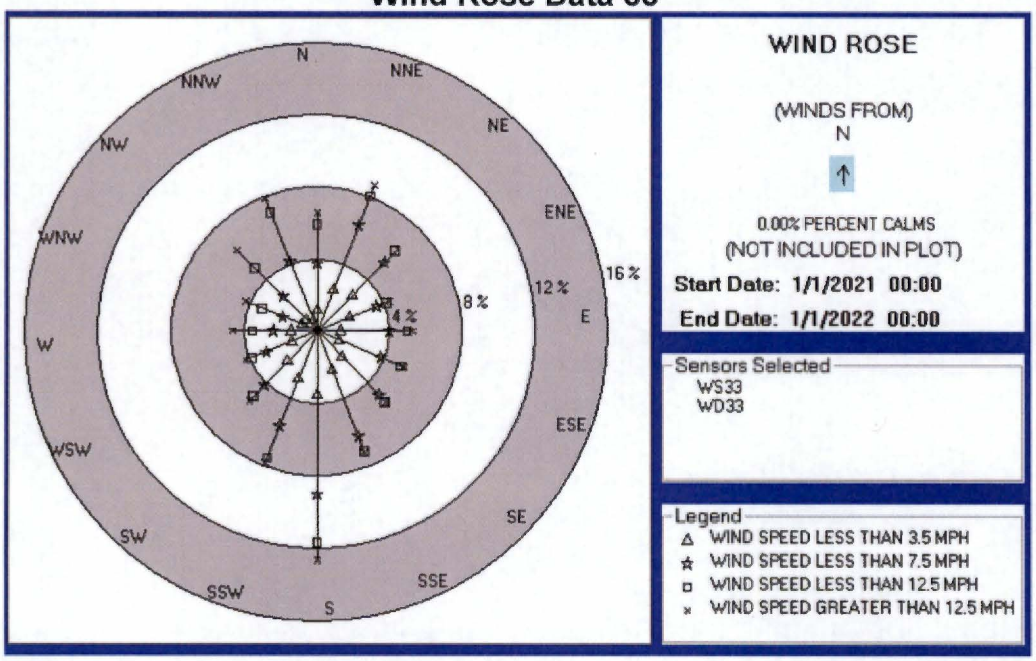
Stability Class: A Delta Temperature Extremely Unstable

Wind Speed (mph)

Wind Direction	1.1-3.5	3.6-7.5	7.6-12.5	12.6-18.5	18.6-24.5	> 24.6	Total
N	2	14	23	2	0	0	41
NNE	2	21	11	0	0	0	34
NE	3	28	2	0	0	0	33
ENE	2	20	1	0	0	0	23
E	3	29	2	0	0	0	34
ESE	5	9	0	0	0	0	14
SE	5	20	3	0	0	0	28
SSE	11	45	6	0	0	0	62
S	11	53	18	0	0	0	82
SSW	3	34	24	0	0	1	62
SW	3	36	11	1	3	2	56
WSW	0	22	14	0	0	0	36
W	0	8	8	3	2	0	21
WNW	5	9	7	4	1	0	26
NW	2	23	15	5	0	0	45
NNW	1	160	13	6	0	0	180
Total	58	531	158	21	6	3	777

Calm Hours not Included above for:	**Total Period**	**All Hours**	295
Variable Direction Hours for:	**Total Period**	**All Hours**	0
Invalid Hours for:	**Total Period**	**All Hours**	913
Number of Valid Hours for this Table	**Total Period**	**All Hours**	777
Total Hours for the Period:			8761

Wind Rose Data 33'



Stability Class Data 156'
Joint Frequency Distribution

Hours at Each Wind Speed and Direction

Total Period **All Hours**

Period of Record = 01/01/2021 00:00 - 01/01/2022 00:00

Elevation: **Speed:** WS156 **Direction:** WD156 **Lapse:** DEL T

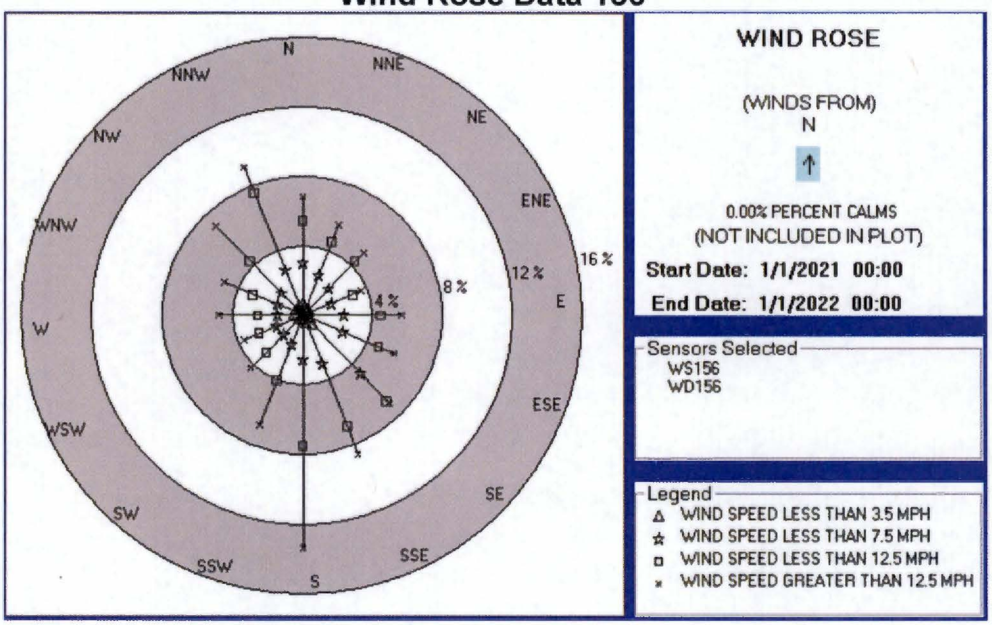
Stability Class: A Delta Temperature Extremely Unstable

Wind Speed (mph)

Wind Direction	1.1-3.5	3.6-7.5	7.6-12.5	12.6-18.5	18.6-24.5	> 24.6	Total
N	0	11	17	12	0	0	40
NNE	0	10	15	4	0	0	29
NE	1	18	20	1	0	0	40
ENE	1	14	14	1	0	0	30
E	0	14	13	0	0	0	27
ESE	1	7	0	0	0	0	8
SE	4	15	11	0	0	0	30
SSE	2	9	35	7	0	0	53
S	1	17	33	40	3	0	94
SSW	2	5	29	29	7	1	73
SW	0	3	32	9	3	6	53
WSW	0	11	16	5	0	0	32
W	0	8	3	7	1	2	21
WNW	2	4	7	6	0	0	19
NW	2	16	24	6	5	0	53
NNW	0	10	153	11	1	0	175
Total	16	172	422	138	20	9	777

Calm Hours not Included above for:	**Total Period**	**All Hours**	20
Variable Direction Hours for:	**Total Period**	**All Hours**	0
Invalid Hours for:	**Total Period**	**All Hours**	908
Number of Valid Hours for this Table	**Total Period**	**All Hours**	777
Total Hours for the Period:			8761

Wind Rose Data 156'



nd Rose (Direction From)

SUMMARY OF GROUNDWATER PROTECTION INITIATIVE ISSUES

The Duane Arnold Energy Center has committed to the Nuclear Energy Institute's Industry Groundwater Protection Initiative - NEI 07-07. Per NEI 07-07, the following information is listed:

- Groundwater Protection Program (GWPP) samples were collected and analyzed in accordance with the requirements and guidance of the site procedures.

- The GWPP is a component of the Duane Arnold Energy Center's Radiological Environmental Monitoring Program (REMP). As such, REMP and GWPP sampling and analysis except for sewage effluent results can be found in the Duane Arnold Energy Center 2021 Annual Radiological Environmental Operating Report.

- Presented in Attachment 1 are only sewage effluent analysis lower level of detection results as determined by the site laboratory. No plant by-products were identified in sewage samples.

- Groundwater Protection Program details can be found in ODAM-DODAM, Table 6.3-2, Attachment 2, and sampling locations can be found in ODAM-DODAM, Table 5-1. Specifically, GWPP sample locations are identified by station number, GWPP station location, and sample type. A simplified map of environmental sample locations can be found in the ODAM-DODAM, Figure 5-1 and Figure 5-2.

- Analysis of GWPP results indicates a tritiated groundwater plume remains confined to the Protected Area and adjacent Owner-Controlled Area at a depth 25 feet or less. Tritiated groundwater has not been identified outside the owner-controlled area. Tritium has not been identified in any drinking water well. In accordance with standards set forth in the ODAM-DODAM, tritiated groundwater is discharged and released as a liquid effluent.

- The vendor laboratory providing results from the Duane Arnold Energy Center's REMP and GWPP sampling is Environmental Inc. Midwest Laboratory of Northbrook, IL. The Environmental Inc. laboratory participates in several cross-check programs. These cross-check program results are presented in the 2021 Duane Arnold Energy Center Annual Radiological Environmental Operating Report or by request.

SUMMARY OF CHANGES TO THE OFFSITE DOSE ASSESSMENT MANUAL (ODAM/DODAM)

Before implementation of the following described changes, a review was performed to validate that the levels of radioactive effluent control required by 10 CFR 20.1302, 40 CFR 190, 10 CFR 50.36a, and 10 CFR 50, Appendix I were not affected. The review also verified that the changes did not adversely impact the accuracy or reliability of effluent dose or set point calculations.

With the plant is a defueled condition the ODAM was renamed to DODAM and reverted to revision 0.

DODAM Revision 0 → DODAM Revision 1

Page 67 – Extended the compensatory action time from 8 hours to 24 hours as most noble gases have decayed.

SUMMARY OF CHANGES TO THE DE-FUELED OFFSITE DOSE ASSESSMENT MANUAL (DODAM)

DODAM Revision 1 → DODAM Revision 2

DODAM Revision
The revision primarily focused on the following:

- Removed the offgas stack as a gaseous discharge point
- Updated REMP sampling based on current plant conditions
- Updated discussions where reactor coolant is referenced
- Extended gaseous sampling compensatory sampling time from 8 to 24 hours.

Below are the specific changes

Page	Change
11	Providing an alternate radiological source for liquid monitor calibrations
17	Removed references to the Offgas Stack
19	Removed references to the Offgas Stack
22	Removed references to the Offgas Stack
23	Removed references to the Offgas Stack
24	Removed references to the Offgas Stack
25	Removed references to the Offgas Stack
26	Removed references to the Offgas Stack
27	Removed references to the Offgas Stack
28	Removed references to the Offgas Stack
29	Removed references to the Offgas Stack
30	Removed references to the Offgas Stack
31	Removed references to the Offgas Stack
32	Removed references to the Offgas Stack
33	Removed references to the Offgas Stack
34	Removed references to the Offgas Stack
35	Removed references to the Offgas Stack
35	Section 3.8.1 content is being deleted as I-131 has decayed and is no longer being detected in gaseous waste streams.
36	Removed references to the Offgas Stack
37	Removed references to the Offgas Stack
39	Removed references to the Offgas Stack
41	Removed references to the Offgas Stack
42	Removed references to the Offgas Stack
43	Removed references to the Offgas Stack
44	Removed references to the Offgas Stack
45	Removed references to the Offgas Stack

Page	Change
47	Irrigated food was removed as a dose pathway in a previous DODAM revision
53 - 59	Retiring environmental sampling locations as effluents from the plant have declined and will be limited in the future.
69	General Service Water (GSW) is no longer a liquid effluent at the plant
73	GSW is no longer a liquid effluent at the plant
82	Extending the compensatory action time from 8 hours to 24 hours as all noble gases except Kr-85 have decayed.
85	Removing references to Offgas Stack
86	Removing noble gas sampling from Turbine Building as there is no longer a noble gas source in the building
89 – 91	Restructuring Table 7.2-2 to remove references of the offgas stack, remove charcoal sampling as I-131 has decayed and is no longer measurable, and there is no source of tritium or noble gas in the turbine building..
101	Table 6.3-1 Reducing air samplers from 5 to 2 based on t-tests of air sample analysis results, reducing the number of ambient sampling from 42 to 20 based on t-test of analysis results, reducing number of drinking water samples from five to two and eliminating the conditional hard to detect analyses, and reducing the vegetation sampling from four to one as the site no longer has an elevated release point (offgas stack).
102	Table 6.3-2 Eliminating the requirement for hard to detect analyses as no hard to detect nuclides have been detected and removing the Electrical Vault sampling and analysis requirement.
111	Removing reference to Offgas Stack
149	Irrigated food was removed as a dose pathway in a previous DODAM revision
100, 113	Revising Land Use Census frequency from annually to biennially.

Recent ODAM and DODAM Revision Dates

Revision:	37	38	39	40	41	42	DODAM-0	DODAM-1	DODAM-2
Date:	2/9/17	1/2/18	4/5/18	3/11/19	4/6/20	10/28/20	4/14/21	6/15/21	11/30/21

ATTACHMENT 1: SEWAGE EFFLUENT-DAEC LABORATORY

Fifty-two sewage effluent samples were collected in 2021. There were no plant by-products identified in the samples. The maximum value for the lower limit of detection (LLD) for environmental sample analysis are noted below and in the ODAM-DODAM, Table 6.3-3.

SEWAGE SAMPLES	
Activity	Maximum Value for the Lower Limit of Detection (pCi/L)
Gross Beta	4
Tritium	2,000[a] 3,000[b]
Mn-54	15
Fe-59	30
Co-58	15
Co-60	15
Zn-65	30
Zr-95	30
Nb-95	15
I-131	1[a] 15[b]
Cs-134	15
Cs-137	18
Ba-140	60
La-140	15
Other [c]	30
[a] Drinking water	
[b] Non-drinking water	
[c] Non-routine, plant by-products	

ATTACHMENT 2: DEFUELED OFFSITE DOSE ASSESSMENT MANUAL (ODAM)

Cover Page

DEFUELED OFFSITE DOSE ASSESSMENT MANUAL

GASEOUS AND LIQUID EFFLUENTS

Duane Arnold Energy Center

Record the following: Date/Time: ____________ / ________ Initials: _______

NOTE: *User shall perform and document a Temp Issue / Rev. Check to ensure revision is current, in accordance with procedure use and adherence requirements.*

Prepared By: DIERICKX (Print)	/ John Dierickx (Signature)	Date: 05/17/21

LICENSING MANAGER CROSS DISCIPLINE REVIEW		
Approved By M. Davis (Print)	/ [signature] (Signature)	Date: 5/18/21

CHEMISTRY MANAGER PROCEDURE APPROVAL		
Approved By [illegible] (Print)	/ [signature] (Signature)	Date: 5/17/21

ORG REVIEW		
Approved By M. Davis (Print)	/ [signature] (Signature)	Date: 5/18/21

SITE DIRECTOR APPROVAL		
Approved By P HANSEN (Print)	/ P Hansen (Signature)	Date: 5.18.2021

DEFUELED OFFSITE DOSE ASSESSMENT MANUAL

GASEOUS AND LIQUID EFFLUENTS

Duane Arnold Energy Center

Record the following: Date/Time: ____________ / _________ Initials: _______

NOTE: *User shall perform and document a Temp Issue / Rev. Check to ensure revision is current, in accordance with procedure use and adherence requirements.*

Prepared By: ____________________ / ____________________________ Date: _________

Print Signature

LICENSING MANAGER CROSS DISCIPLINE REVIEW

Approved By ____________________ / ____________________________ Date: _________

Print Signature

CHEMISTRY MANAGER PROCEDURE APPROVAL

Approved By ____________________ / ____________________________ Date: _________

Print Signature

ORG REVIEW

Approved By ____________________ / ____________________________ Date: _________

Print Signature

SITE DIRECTOR APPROVAL

Approved By ____________________ / ____________________________ Date: _________

Print Signature

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DEFUELED OFFSITE DOSE ASSESSMENT MANUAL
FOR GASEOUS AND LIQUID EFFLUENTS

1.0 INTRODUCTION

This manual provides a description of the facility's Defueled Offsite Dose Assessment Program (DODAM), the Radiological Effluents Controls Program and the Radiological Environmental Monitoring Program (REMP).

CESSATION OF POWER OPERATIONS

In July of 2018 NextEra Energy Duane Arnold L.L.C announced the cessation of power operations planned for the 4th quarter of 2020, however a severe windstorm on August 10, 2020 damaged the plants cooling towers. There were no abnormal releases as all safety systems functioned as designed. The reactor was permanently defueled on October 8, 2020. The decommissioning process has started with system layup plans for long term dormancy period prior to returning the area to greenfield. The plant is being placed in SAFSTOR.

EFFLUENTS CONTROLS PROGRAM

This program, conforming to 10 CFR 50.36a, provides for the control of radioactive effluents and for maintaining the doses to members of the public from radioactive effluents as low as reasonably achievable (ALARA). The program is contained in sections six and seven of the DODAM, is implemented by procedures, and includes remedial actions to be taken whenever the program limits are exceeded.

Section eight of the DODAM includes the requirement for the Annual Radioactive Material Release Report (ARMRR). The ARMRR covers the operation of the unit during the previous calendar year shall be submitted prior to May 1 of each year in accordance with 10 CFR 50.36a. The report includes a summary of the quantities of radioactive liquid and gaseous effluents and solid waste released from the unit. The material provided in the report is consistent with the objectives outlined in the DODAM and Process Control Program and in conformance with 10 CFR 50.36a and 10 CFR Part 50, Appendix I, Section IV.B.1.

RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM

The REMP provides for representative measurements of radioactivity in the highest potential exposure pathways, verification of the accuracy of the effluent monitoring program and modeling of environmental exposure pathways. Additionally, the REMP is designed to demonstrate that radioactive effluents from DAEC are ALARA.

GROUNDWATER PROTECTION PROGRAM

The Groundwater Protection Program (GWPP) provides representative measurements of radioactivity in the highest potentially exposed aquifer, verification of the accuracy of the GWPP monitoring efforts and modeling of the subsurface. Additionally, the GWPP is designed to protect and mitigate radiological contaminates to groundwater. The GWPP is incorporated within REMP and includes standards set forth in Nuclear Energy Institute, NEI 07-07.

The REMP and GWPP programs are described in sections five, six and seven of the DODAM and conforms to the guidance of Appendix I to 10 CFR Part 50 and 10 CFR 72, Section eight of the DODAM includes the requirement for the Annual Radiological Environmental Operating Report (AREOR). The Annual Radiological Environmental Operating Report covers the operation of the unit during the previous calendar year and is submitted by May 15 of each year. The report includes summaries, interpretations, and analyses of trends of the results of the Radiological Environmental Monitoring Program for the reporting period. The material provided is consistent with the objectives outlined in the DODAM, and in 10 CFR 50, Appendix I, Sections IV.B.2, IV.B.3, and IV.C.

DEFUELED OFFSITE DOSE ASSESSMENT PROGRAM

The methodology and parameters used in the calculation of offsite doses resulting from radioactive gaseous and liquid effluents are included in sections two, three and four of the DODAM. These chapters describe acceptable methods of calculating radioactivity concentrations in the environment and the potentially resultant committed doses to a member of the public in the unrestricted area[a] that are associated with LWR liquid and gaseous effluents.

The methodology stated in this Manual is acceptable for use in demonstrating operational compliance with 10 CFR 20.1301, 10 CFR 50 Appendix I, 10 CFR 72.104, and 40 CFR 190. Only the dose attributable to the Duane Arnold Energy Center[b] is considered in demonstrating compliance with 40 CFR 190 since no other nuclear facility exists within 50 miles of the DAEC.

Calculations are made monthly to assess the potential air doses offsite and to a nearby resident in order to guide the management of station effluents. The receptor is described such that the dose to any resident near the Station is unlikely to be underestimated. Calculations made to assess the radioactive noble gas dose to air are based on the location offsite that could be occupied by a person where the maximum air dose is expected. For these monthly-accumulated dose calculations, atmospheric dispersion and deposition of gaseous effluents may be based on reference meteorological conditions.[c] More conservative conditions (i.e., location and/or exposure pathways expected to yield higher computed doses) than appropriate for the maximally exposed person may be assumed in the dose estimated.

Calculations of dose committed from radioactive releases over extended time (3 and 12 months) are also made for the purpose of verifying compliance with regulatory limits on offsite dose. For these calculations the receptor is selected on the basis of the combination of applicable exposure pathways identified in the land use census and the maximum ground level χ/Q at a residence, or on the basis of more conservative conditions such that the dose to any resident near the Station is unlikely to be underestimated.

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[a] Unrestricted area means outside of the boundary of property owned, leased, or controlled by the Company on which DAEC is sited. The DAEC site boundary is identified by UFSAR Figure 1.2-1.

[b] The Duane Arnold Energy Center, also referred to henceforth as the Station, is defined as including BOTH the Nuclear Reactor Facility and the Independent Spent Fuel Storage Installation (ISFSI).

[c] Reference meteorological conditions are 1971, 1974 and 1975 data composited as discussed in "Duane Arnold Energy Center, Evaluation of Liquid and Gaseous Effluent Releases in Accordance With 10 CFR 50 Appendix I," submitted to the NRC June 3, 1976.

2.0 LIQUID EFFLUENT

2.1 Radioactivity In Liquid Waste

The concentration of radionuclides in liquid waste is determined by sampling and analysis in accord with the surveillance requirements of Section 7.1.2, Table 7.1-2. When a radionuclide is identified, it is reported as being present in the sample even if the concentration is below the required LLD for the analysis.

In November 2020 the plant began to discharge liquid radiation waste to the river. The aqueous concentration is managed and tracked batch by batch to maintain margin significantly below 50CFR, Appendix I regulatory limits. The system has a calibrated radiation monitor and an automatic isolation function. Filters were installed prior to the radiation monitor to ensure the process is ALARA; this focus on best practices has significantly lowered the dose to the public.

2.2 Aqueous Concentration

Radioactive material in liquid effluent is diluted successively by water flowing in the discharge pipe and in the Cedar River. The diluted concentration of radionuclide *i* in a receiving stream is estimated with the equation

$$C_{zi} = C_i \frac{F_1}{F_2}$$

where

C_i = concentration of radionuclide *i* in liquid radwaste released (µCi/mL)

C_{zi} = concentration of radionuclide *i* in the receiving stream (µCi/mL)

F_1 = release rate of liquid radwaste (mL/sec)[d]

F_2 = dilution flow of receiving stream of water (mL/sec).

For the purpose of calculating the radioactivity concentration in water at the restricted area boundary (section 2.5), the flow in the discharge pipe, F_c, is assigned to F_2. The water flow in the discharge pipe may include the liquid waste effluent flow, the liquid radwaste dilution water flow, the cooling tower blowdown flow and other streams such as RHR, GWPP discharges, and emergency service water discharged via the dilution structure and discharge pipe. These streams are illustrated in Figure 2-1.

[d] F_1, F_2, and F_c may have any convenient units of flow (i.e., volume/time) provided the units of all are identical.

In the Cedar River immediately beyond the discharge pipe and the restricted area boundary, the effective dilution is

$$F_2 = F_c \, x \, M$$

Where

F_c = discharge pipe flow

M = factor of additional mixing in the River

A near field mixing ratio from the pipe into the near field of the River, M = 5, is assigned when estimating maximum potential individual doses involving exposure by eating fish. Current and historical field surveys of the Cedar River downstream of DAEC do not indicate the presence of irrigation systems withdrawing water for irrigated crops such as strawberries or other produce. In the event water is drawn from the Cedar River downstream of the Station for drinking water or another exposure pathway, F_2 represents the portion of the Cedar River flow into which the liquid effluent from the Station is effectively mixed.

2.3 Basis of Mixing Ratios

Downstream dilution of aqueous discharge from the DAEC has been estimated based on thermal plume studies conducted in 1974 at the DAEC.[e] Measurements of the discharge temperature and river temperature indicated that the 1°F excess temperature isotherm was about 350 feet downstream of the discharge. This 1°F isotherm represented a dilution of the discharge by the Cedar River of about a factor of 12.

In determining additional dilution within the receiving water for evaluating doses from a plant with cooling towers, the NRC guideline[*f] is that the factor should be limited to a number such that the product of the number and the average blowdown flow to the receiving water body is 1000 cfs or less. At the DAEC, the discharge rate can be conservatively approximated by a cooling tower blowdown rate of 4000 gpm, or about 9 cfs. Using the NRC guideline, an additional dilution of 100 in the Cedar River could be assumed for evaluating doses due to liquid effluent. Since the average flow in the Cedar River is about 3,775 cfs, the additional dilution in the Cedar River is achievable.

These results indicate both a dilution factor of 12 at a downstream distance of 350 feet and conformance to the NRC guideline.

Land Use Censuses (field surveys) have shown that the nearest use of river water (from alluvial wells) is 2.2 miles downstream for the City of Palo and more than 8 miles downstream for the City of Cedar Rapids.

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[e] IELP, Cedar River Baseline Ecological Study, DAEC, annual report, Jan. 1974-Jan.1975.

[f] Boegli, J.S., et. al., Preparation of Radiological Effluent Technical Specifications for Nuclear Power Plants, NUREG-0133, p. 16, October, 1978

For the sake of simplification and conservatism in routine liquid dose calculations, a dilution factor of 5 is assumed for fish and a dilution factor of 10 is assumed for drinking water for the evaluation of doses during DAEC operation.

2.4 Method of Establishing Alarm Setpoints

The liquid radwaste effluent line has a monitor which provides automatic isolation when 10 times the water effluent concentration listed in 10 CFR 20 Appendix B, Table 2, is being exceeded in the unrestricted area. The other liquid effluent pathways have monitors which provide alarms when 10 times the 10 CFR 20 water effluent concentrations is being exceeded in the unrestricted area. Given the nature and frequency of discharges, prompt action to reduce radioactive releases following an alarm, will assure the requirements of 10 CFR Part 20. 1301; 10 CFR Part 50 Appendix I, Section IV; and 40 CFR Part 190 are not exceeded.

The alarm setpoint for the liquid effluent radiation monitor is derived from the concentration limit provided in 10 CFR Part 20.1001-20.2402 Appendix B Table 2 Column 2 applied at the unrestricted area boundary where the discharge pipe flows into the river. The alarm setpoint does not consider dilution, dispersion, or decay of radioactive material beyond the site boundary. That is, the alarm setpoint is based on a concentration limit at the end of the discharge pipe. The radiation monitoring and isolation points are located in each line through which radioactive waste effluent is eventually discharged into the discharge pipe. ALARA is achieved by way of ensuring three times tank volume mixing, sample analysis, and filtration prior to release of individual batches.

The alarm setpoint for effluent monitors on batch releases is based on measurements, according to Table 7.1-2. For liquids released in continuous aqueous discharge which are normally radioactively clean, the setpoint is based on the effective Water Effluent Concentration (WEC) for the most likely contaminating source, i.e., the primary coolant water. A measured spectrum from the primary coolant water is used to determine the effective WEC based on WEC fractions according to 10 CFR 20 Appendix B.

2.4.1 Setpoint for a Batch Release

A sample of each batch of liquid radwaste is analyzed for I-131 and other principal gamma emitters, or for total activity concentration prior to release. The ratio, $FWEC_b$, of the activity concentration in the tank to the unrestricted area WEC (10 CFR Part 20, Appendix B, Table 2, Column 2) is calculated with the equation

$$FWEC_b = \sum_i \frac{C_{bi}}{WEC_i}$$

where

$FWEC_b$ = fraction of unrestricted area WEC in batch derived from activity measured prior to release.

C_{bi} = concentration of radionuclide *i* (including I-131 and principal gamma emitters) in batch sample taken prior to release (µCi/mL)

Whether radioiodine and primary gamma emitters are identified prior to a batch release or not, the liquid radwaste effluent line radiation monitor alarm setpoint is determined with the equation

$$S = 10\,x\left[\frac{A}{FWEC_b} \bullet \frac{Fs_2}{Fs_1} \bullet g\right] + Bkg$$

where

S = radiation monitor alarm setpoint (cpm)

10 = factor to account for fact that DAEC's instantaneous release limit is ten times the listed WECs

A = counting rate (cpm/mL) or activity concentration (µCi/mL) of sample in laboratory analysis. A equals $\sum_i C_{bi}$ if an isotopic analysis were performed or C_b if a gross activity analysis was performed.

g = ratio of effluent radiation monitor counting rate to laboratory counting rate or activity concentration in a given batch of liquid (cpm per cpm/mL or cpm per µCi/mL)

Bkg = monitoring instrument background (cpm)

Fs_1 = flow in the batch release line (gal/min).* Value not greater than the discharge line flow alarm maximum setpoint.

Fs_2 = minimum flow in the discharge pipe (gal/min).[g] Value not less than the discharge pipe flow alarm minimum setpoint.

Note that $A/FWEC_b$ represents the counting rate of a solution having the same radionuclide distribution as the sample and having the water effluent concentration of that mixture.

2.4.2 Setpoint for a Continuous Release

[g] Any suitable but identical units of flow (volume/time)

Continuous aqueous discharges are sampled and analyzed according to the schedule in Table 7.1-2. The ratio, $FWEC_c$, of the activity concentration in each of the continuous release streams to the unrestricted area WEC is calculated with the equations

$$FWEC_c = \sum_i \frac{C_{ci}}{WEC_i}$$

where

$FWEC_c$ = fraction of unrestricted area WEC in continuous release based upon activity measured in primary coolant sample(s)

C_{ci} = concentration of radionuclide *i* in sample(s) (µCi/mL)

The alarm setpoint of the radiation monitor on a continuous radioactive discharge line is determined with the equation

$$S = 10\,x\left[\frac{A}{FWEC_c} \bullet \frac{Fs_2}{Fs_1} \bullet g\right] + Bkg$$

where

10 = factor to account for fact that DAEC's instantaneous release limit is ten times the listed WECs

A = activity concentration (µCi/mL) or counting rate (cpm/mL) in laboratory of monthly reactor primary coolant sample.

Fs_1 = Flow in the liquid discharge line (mL/sec).[h] Value not greater than off discharge line flow alarm maximum setpoint.

Fs_2 = flow in the discharge pipe (mL/sec).[h] Value not less than discharge pipe flow alarm minimum setpoint.

g = ratio of effluent radiation monitor counting rate to laboratory counting rate or activity concentration in a given batch of liquid (cpm per cpm/mL or cpm per µCi/mL)

The radioactivity concentration in continuous aqueous effluent is usually so low that measurement of a representative radionuclide in a sample of the water is uncertain. Thus, the ratio, $A/FWEC_c$, which represents the WEC of a given spectrum of radionuclides, is usually derived from reactor primary coolant analysis(es). Alternatively, it may be determined from analyses of the continuous effluent itself.

In the event the concentration of radioactive material in the sample from the continuous release is below measurable levels (i.e., less than the lower limit of detection), the value

h Any suitable but identical units of flow (volume/time) of 1 x 10^{-8} µCi/mL or the equivalent counting rate (cpm/mL) may be substituted for the factor

$$\frac{A}{FWEC_c} \; (i.e., \frac{A}{FWEC_c} = 1 \bullet 10^{-8}).$$

2.5 Radioactivity Concentration in Water at the Restricted Area Boundary

Section 6.1.2 provides limits on instantaneous radioactivity concentration in the unrestricted area due to aqueous effluents from DAEC. Compliance is assessed by monitoring, sampling, analyzing and establishing setpoints according to Section 7.1.2. As long as a liquid effluent monitor named in Table 6.1-1 does not exceed an alarm or trip setpoint, determined in accordance with section 2.3, or as long as the total or gross activity concentration, measured as required in Section 7.1.2, does not exceed 1 x 10^{-7} µCi/mL after dilution in the discharge pipe, Section 6.1.2 is satisfied.

In the event of an alarm, indicating concentrations in the unrestricted area in excess of section 6.1.2 limits, the release shall be terminated and dose calculations will be performed to assure the limits specified in sections 6.1.3 and 6.3.1 are not exceeded.

Compliance with 10 CFR 20.1301 shall not be demonstrated on the basis of determining the average annual liquid effluent concentration. But rather by demonstrating compliance with 40 CFR 190 (i.e., section 6.3.1). Such a practice was deemed acceptable by the NRC in their preamble to the revised 10 CFR 20.[i]

2.6 Accumulated Personal Maximum Dose

Section 7.1.3 requires an assessment to be performed at least once every 30 days in any quarter in which radioactive effluent is discharged which determines whether the dose or dose commitment to a person offsite due to radioactive material released in liquid effluent calculated on a <u>cumulative</u> basis exceeds the limits of Section 6.1.3. The requirement is satisfied by computing the accumulated dose commitment to the most exposed organ and to the whole body of a hypothetical person exposed by eating fish and drinking water taken from the river offsite downstream of the discharge pipe.

The pathway(s) and or age group(s) selected may vary by season. For instance, fishing near the DAEC is practically non-existent during the winter; thus, a dose evaluation of the fish pathway is not required for aqueous effluent discharged during the winter months of January, February, or March.

The accumulated dose commitment is computed at least once every 30 days but may be computed as analyses become available.

Normally, DAEC employs the MIDAS computer program to calculate dose to a member of the public from aqueous effluent that uses the equations in Reg Guide 1.109[j] and standard values therein for maximally exposed people.

[i] Federal Register Volume 56, #98, Tuesday, May 21, 1991, p. 23360

[j] USNRC, Regulatory Guide 1.109, revision 1, Position C.1, pp. 1.109-2 thru 1.109-4, Oct. 1977

Alternatively, the dose may be calculated in the following way, for instance, in the event calculations by hand were necessary.

$$\Delta D_{ank} = 3.785 \bullet x\, 10^{-3} \sum_i C_{ik} \bullet \Delta T_k \sum_e \frac{F\, 1_k}{F\, 2_{ek}} \bullet A_{eani}$$

$$D_{an} = \sum_k \Delta D_{ank}$$

where

ΔD_{ank} = the dose commitment (mrem) to organ *n* of age group *a* due to the isotopes identified in analysis *k*, where

the analyses are those required by Table 7.1-2. Thus the contribution to the dose from gamma emitters become available on a batch basis for batch releases and on a weekly basis for continuous releases. Similarly the contributions from H-3 is available on a monthly basis and the contributions from Fe-55, Sr-89, and Sr-90 become available on a quarterly basis.

D_{an} = the dose commitment during the quarter-to-date to organ *n*, including whole body, of the maximally exposed person in age group *a* (mrem)

A_{eani} = transfer factor relating a unit release of radionuclide *i* (Ci) in a unit stream flow (gal/min) to dose commitment to organ *n*, or whole body, of an exposed person in age group *a* $\left[\frac{mrem\ gal}{Ci\ \min}\right]$ via environmental pathway *e*.

C_{ik} = the concentration of radionuclide *i* in the undiluted liquid waste represented by sample *k* to be discharged (µCi/mL)

Δt_k = duration of radioactive release represented by sample *k* which occurs within time boundaries TB and TE and during which concentration C_{ik} and flows $F1_k$ and $F2_k$ exist. (min.)

$3.785 \bullet 10^{-3}$ = conversion constant (3785 mL/gal • 10^{-6} Ci/µCi)

$F\, 1_k$ = flow in the radioactive waste release line (gal/min)* represented by sample *k*.

$F\, 2_{ke}$ = flow into which radioactive release represented by sample *k* is mixed in the river at the point of exposure or withdrawal of water for use (same units as $F1_k$)*

$$= \quad M \times F_{ck}$$

where

F_{ck} = discharge pipe flow (gal/min)[k] during release represented by sample k

M = factor of additional mixing in the Cedar River

Pathway-to-dose transfer factors, A_{eani}, for use in calculating the dose commitment arising from radioactive material released in aqueous effluents are tabulated in Appendix C. These dose transfer factors were derived using LADTAP II and standard values from Regulatory Guide 1.109, revision 1, except where corrections have been incorporated in LADTAP II. Appropriate tables representing applicable environmental pathways of exposure and most exposed age group(s) are selected and used in calculating the dose commitment. The pathway(s) and/or age group(s) selected may vary by season.

Pathways of Maximum Exposure to a Member of the Public:

- Ingestion of Fish taken from the river near the discharge pipe
  - Age Group: Adult
  - Dilution: $F_2 = 5F_c$
  - Food: Sport Fish

- Drinking Water Pathway
  - Age Group: Infant
  - Dilution: $F_2 = 10F_c$
  - Dilution: $F_2 = 5F_c$ only when Land Use Census identifies this pathway within 3 miles of the plant

Variables F_1, F_2, and F_c are also defined in section 2.2.

2.7 Projected Maximum Dose to a Person Offsite

The dose commitment to a person offsite due to radioactive material released in liquid effluent may be projected by calculating the extrapolated whole body and most exposed organ dose commitments to a hypothetical person exposed via the same pathways evaluated in section 2.6. The potential dose commitments to organs and to the whole body are computed separately.

The dose commitment to a maximally exposed hypothetical person will be projected by calculating the doses accumulated during the most recent three months (according to the method described in section 2.6) and by assuming the result represents the projected doses during the current quarter. Alternatively, the quarterly dose commitment may be projected by using the equation:

$$P_{an} = \frac{92\ D_{an}}{X}$$

[k] Any suitable, identical units of flow (volume/time)

where

P_{an} = projected dose commitment (mrem) to organ *n* (including whole body) of age group *a* for the current quarter

92 = number of days in a quarter

X = number of days to date in current quarter

D_{an} = dose commitment to organ *n*, including whole body, of the maximally exposed person in age group *a* based on available aqueous effluent measurements during the quarter to date (mrem)

2.8 Groundwater Pathway

Low levels of radioactive contamination have been identified in sub-surface water on site. There is the possibility that this water could represent potential exposure to the public. Hydrogeology studies indicate that this water will migrate towards the Cedar River in sectors ranging from south-southeast to the southeast. The only potential exposure pathways are described in Section 2.6.

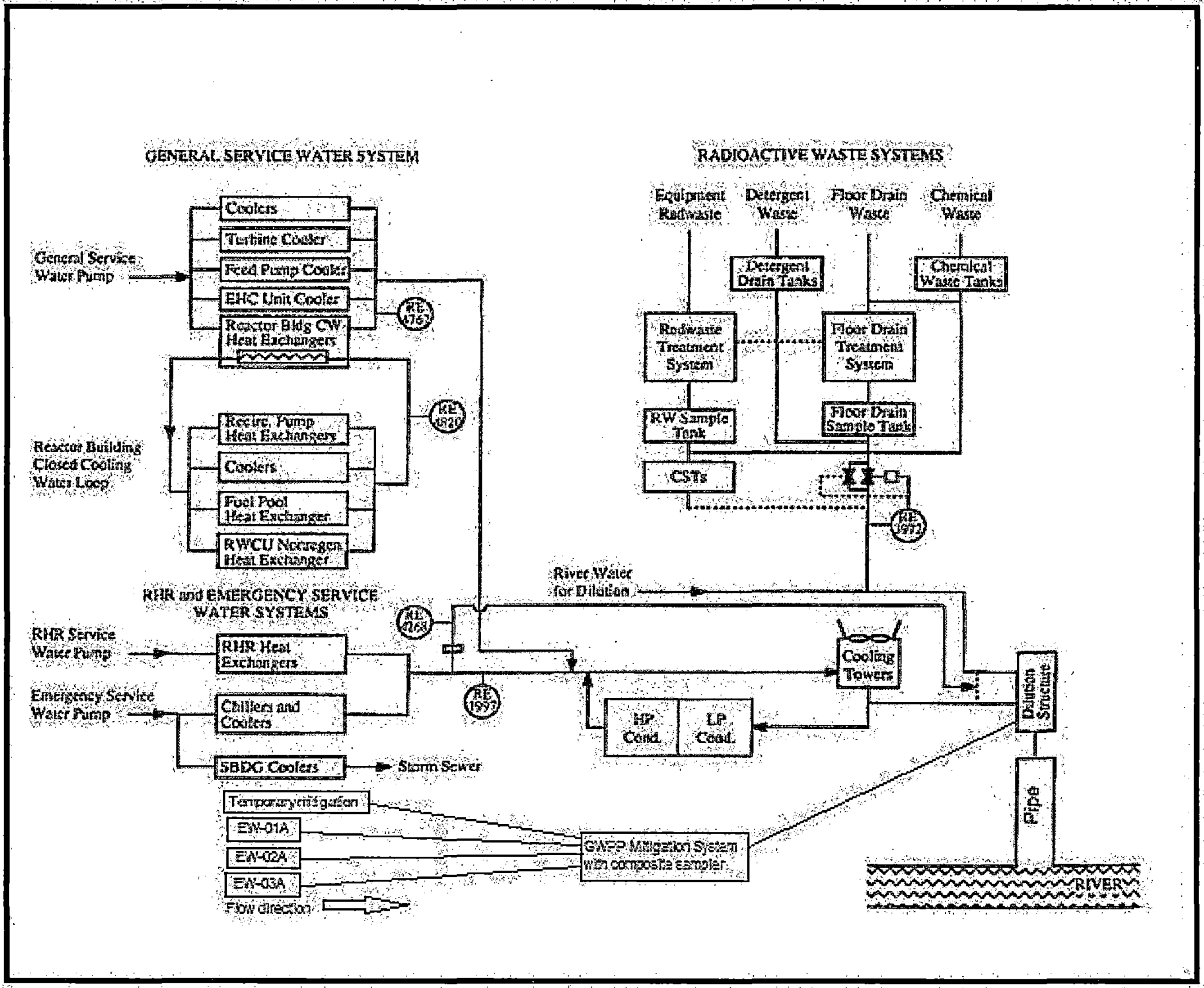
To monitor for the migration of any radioactive contamination beyond the owner controlled area, sampling of on-site ground water, Cedar River water, and down gradient drinking water is performed as a function of the REMP and GWPP. These programs are described in Section 5.0.

Release standards of contaminated groundwater are set forth in Section 6.0, Radiological Liquid Effluent Release O.6.1.2, Table 7.1-2. In accordance with GWPP Administrative Control Procedure (ACP) 1411.35 and Environmental Protection Agency (EPA) drinking water standards for tritium, 20,000 pCi/L, groundwater batch and continuous releases are expected less than (<) 20,000 pCi/L. Groundwater samples are analyzed on-site and validated by off-site secondary laboratory. Off-site laboratory results are published in Annual Radiological Environmental Operating Report.

In February 2016, routine GWPP sampling identified a contaminate plume in the shallow aquifer (less than 25 feet deep). By February 2017, three Extraction Wells were installed in the shallow aquifer to facilitate continuous groundwater withdrawal to mitigate the contaminated plume. Extraction well EW01A (Table 5-1, D-68) located inside the Protected Area, is the primary well designed to remove tritiated groundwater at less than 17.3 gallons per minute from a narrow plume less than 25 feet below the surface. Extraction wells EW02A (D-69) and EW03A (D-70) are located southeast of EW01A and were similarly designed to withdraw the same volume of contaminated or uncontaminated groundwater from the shallow aquifer. These Extraction Wells, their AC-powered pumps, and the associated piping system are considered to be tools that may be used by Chemistry for groundwater mitigation.

Based on calibration of pump speed, flow rate, dilution flow rates, and groundwater tritium concentrations, tritiated groundwater is released with anticipated concentrations less than the EPA drinking water limit of 20,000 pCi/L. Liquid effluent releases from groundwater are published in the Annual Radiological Material Release Report.

Figure 2-1
Duane Arnold Energy Center
Radioactive Water Effluent Diagram



3.0 GASEOUS EFFLUENT

3.1 Introduction

The Station discharges gaseous effluent through a stack and discharges ventilation air from the reactor and radwaste building through the reactor building vents. Ventilation air from the Turbine Building is discharged through the Turbine Building vent and through the Reactor Building vent. Ventilation from the LLRPSF is discharged through the LLRPSF vent. These gaseous effluent streams, radioactivity monitoring points, and effluent discharge points are shown schematically in Figure 3-1. Gaseous release point locations and elevations at the Station are described in Table 3-1. Gaseous discharges from the stack are treated as an elevated release while discharges via the building vents are assumed to be ground-level, building wake, or split wake releases.

3.2 Radioactivity in Gaseous Effluent

For the purpose of estimating offsite radionuclide concentrations and radiation doses, measured radionuclide concentrations in gaseous effluent and in ventilation air exhausted from the Station are relied upon.

The gross radioactivity of noble gases discharged is measured by the radioactive noble gas effluent monitors according to Tables 6.2-1 and 7.2-2. Radionuclides other than noble gases in gaseous effluents are measured by sampling and analyses in accordance with Table 7.2-2. Each radionuclide measured in an effluent may be assumed to be discharged uniformly during the sampling period. When radioactivity is identified at a concentration below the LLD for the analysis, that concentration is reported. When radioactivity is not identified in a sample, it is not reported as being present in that sample.

The quantity of radioactive noble gas discharged via the offgas stack or a vent during an interval of time is determined by integrating the release rate measurement of each effluent noble gas monitor. An hourly interval is normally used for dose rate assessments and a daily or longer interval is used for dose assessments. If ΔQ_j represents the gross activity of noble gas discharged via the offgas stack or a vent and g_i represents the fraction of radionuclide *i* in the distribution of radioactive gases in that effluent stream, then the quantity of radionuclide *i* released in the gaseous effluent stream during counting interval *j* is estimated by the relation:

$$\Delta Q_{ij} = \Delta Q_j \ x \ g_i$$

The distribution of radioactive noble gases in gaseous effluent streams is determined by gamma spectrum analysis of gaseous effluent samples in accordance with Table 7.2-2. Results of one or more previous analyses may be averaged to obtain a representative spectrum. In the event a representative distribution is not available or is unobtainable from sample(s) of an effluent stream taken during the current quarter, it will be derived from past measurements, e.g., earlier sample results or annual radioactive material release reports. Alternatively, a noble gas spectrum for a given effluent stream in Table 3-2 herein, may be assumed.

An airborne discharge of radionuclides other than noble gases may be represented by multiple samples with each sample providing a measure of the concentration of specific radionuclides, C_i, in gaseous effluent discharged at flow, F_a, during a time increment Δt. Thus, each release is quantified according to the relation:

$$Q_{ik} = \sum_j C_{ik} \; F_{aj} \; \Delta t_j$$

where

Q_{ik} = the quantity of radionuclide *i* released in a given effluent stream based on analysis *k* (Ci)

C_{ik} = concentration of radionuclide *i* in gaseous effluent identified by analysis *k* (μCi/ML or Ci/m^3)

F_{aj} = effluent stream discharge rate during the increment Dt_j (m^3/sec)

Δt_j = time increment during which radionuclide *i* at concentration C_{ik} is being discharged (sec)

The analysis index *k* may represent either a grab sample, integrated sample, or a composite sample required by the effluent sampling and analysis program specified in Table 7.2-2.

3.3 Not Used

3.4 Effluent Noble Gas Monitor Alarm Setpoint

Section 6.2.2 provides limits on dose equivalent rates associated with airborne radioactive materials concentrations in the unrestricted area due to airborne effluents from the Station. Instrumentation is provided to monitor gamma radiation in the airborne effluents according to Table 6.2-1. Each effluent noble gas monitor includes an alarm that can be set to activate when the dose rate off site or the noble gas concentration at ground level offsite is expected (calculated) to exceed a specified level. Compliance with the limits on dose rate from noble gases is demonstrated by setting each gaseous effluent monitor alarm setpoint so that an alarm will occur at or before the dose rate limit for noble gases is reached. If an alarm occurs with the setpoint at the limit, compliance with Section 6.2.2.a is assessed as described in section 3.5.

On the basis of effluent noble gases from the DAEC during recent years, the gamma dose rate to a person's whole body is expected to be a larger fraction of the limit, 500 mrem/yr, than is the beta plus gamma dose rate to skin is to its limit, 3000 mrem/yr. As a result, a gaseous effluent monitor setpoint may be derived on the basis of the gamma dose rate to a person's whole body alone such that an alarm is set to occur at or before the whole body dose rate offsite exceeds 500 mrem/yr.

A noble gas monitor may be set to activate an alarm at a lower setting than the derived setpoint corresponding to the dose rate limit (or corresponding concentration limit). In the event an alarm occurs at the lower setting, the monitor record is compared with the derived setpoint. If the derived setpoint is exceeded, compliance with Section 6.2.2.a is assessed as described in section 3.5.

Each radioactive noble gas effluent monitor setpoint is derived on the basis of whole body dose equivalent rate in the unrestricted area. Setpoints for gaseous effluent monitors may be set independently because excessive effluent release via the turbine building vent is unlikely. Other releases are likely to be initiated independently and released from different levels, i.e., vent and stack releases with points of maximum concentration offsite not likely to coincide.

For the purpose of deriving a setpoint, the distribution of radioactive noble gases in an effluent stream is determined as described in section 3.2.

<u>Setpoint Determination</u>

The alarm setpoint of a radioactive noble gas effluent monitor may be calculated on the basis of whole body dose equivalent rate offsite, 500 mrem/yr. A setpoint of a monitor of an elevated release, e.g., from the stack, may be calculated with the equation:

$$S = \left[1.06 \bullet \frac{h}{f} \bullet \frac{\sum_i C_i}{\sum_i C_i \, x \, DF_i^s} \right] + Bkg$$

The setpoint of a monitor of a ground-level or building release, e.g., from the turbine building vent or 3 reactor building vents or LLRPSF vent may be calculated with the equation:

$$S = \left[1.06 \bullet \frac{h}{f\left(\frac{\chi}{Q}\right)} \frac{\sum_i C_i}{\sum_i C_i \bullet DF_i^v} \right] + Bkg$$

where

s = the alarm setpoint (cpm) or (MR/hr)

h = monitor response to activity concentration of effluent being monitored,

$$\left(\frac{cpm}{\mu Ci / cm^3} \right) \quad or \quad \left(\frac{mR / hr}{\mu Ci / cm^3} \right)$$

C_i = relative concentration of noble gas radionuclide *i* in effluent at the point of monitoring (µCi/cm³)

χ / Q = atmospheric dispersion from point of ground-level or building wake release to the location of potential exposure (sec/m³)

DF_i^s = factor converting elevated release rate of radionuclide *i* to whole body dose equivalent rate at the location of potential exposure

$$\left(\frac{mrem}{yr \bullet \frac{\mu Ci}{\sec}} \right)$$

DF_i^v = factor converting ground-level of split-wake release of radionuclide *i* to the whole body dose equivalent rate at the location of potential exposure

$$\left(\frac{mrem}{yr \bullet \frac{\mu Ci}{m^3}} \right)$$

f = flow of gaseous effluent stream, i.e., flow past the monitor (ft³/min)

Bkg = monitoring instrument background (cpm) or (MR/hr)

$$1.06 = 500\frac{mrem}{yr} \bullet 60\frac{\sec}{\min} \bullet 35.3\frac{ft^3}{m^3} \bullet \frac{1\,m^3}{10^6\,cm^3}$$

Kaman Industries performed primary calibrations using Xe-133 and Kr-85. The average response for Xe-133 was 4.83e+07, and for Kr-085 was 7.65e+07 cpm/µCi/cc. Because Xe-133 response factor is more conservative, *h* will be set at 4.83e+07 cpm/µCi/cc. Response factor *h* for Kr-85 may be considered if conditions are more appropriate.

The concentration of each noble gas radionuclide, C_i, in a gaseous effluent is determined as discussed earlier in this section.

Since the dose rate limits for airborne effluents apply everywhere offsite, alarm setpoints are determined and compliance is assessed at the site boundary where the minimum atmospheric dispersion (maximum χ / Q) occurs. The atmospheric dispersion factor and the dose conversion factor DF_i^s depend on local conditions. The value of χ / Q adopted in a setpoint calculation will be based either on prevailing meteorological conditions or on reference meteorological conditions at the DAEC. The minimum atmospheric dispersion offsite from a ground-level or building wake release derived from reference meteorological conditions is at the site boundary 1260 meters NNW of the Station where:

$$\left(\frac{\chi}{Q}\right)_{vent} = 4.3 \bullet 10^{-6} \sec / m^3$$

The dose conversion factors, DF_i^s, used in setpoint calculations for gaseous effluent monitors are in Table 3-4. In the event DF_i^s is derived on the basis of prevailing meteorology, it will be calculated in accordance with Regulatory Guide 1.109, Appendix B.

3.5 Dose Equivalent Rate Offsite

Section 6.2.2 provides limits on dose equivalent rates associated with airborne radioactive materials concentrations in the unrestricted area due to airborne effluents from the Station. Compliance is assessed on the basis of measurements specified in Table 7.2-2.

3.5.1 Noble Gas

Limits on radioactive noble gas in the unrestricted area are provided in Section 6.2.2.a. Each radioactive noble gas effluent monitor is set to alarm when, or below when, the noble gas in airborne effluent from a monitored stack or vent is expected to cause either dose rate limit in Section 6.2.2.a to be exceeded. In the event an airborne effluent release from the Station exceeds the derived setpoint (limit) for an effluent noble gas monitor (except when caused by the performance of a Surveillance Test Procedure), an assessment of compliance is performed as described herein.

The quantity of radioactive noble gas released in a increment of time is measured by the radioactive noble gas effluent monitors and the distribution of radioactive noble gases in a gaseous effluent stream is determined as described in section 3.2 herein.

Compliance with Section 6.2.2.a may be assessed by calculating the dose equivalent rate as described hereafter and by comparing it with the limiting dose rate in the Specification.

3.5.1.1 Total Body Dose Rate

For evaluating compliance with Effluent Control, Section 6.2.2.a, the total body dose equivalent rate due to noble gas gamma radiation is calculated with the equation:

$$\bar{D}_\gamma = \frac{1}{3600}\left[\sum_i\left(\frac{Qg_i}{t} \bullet DF_i^s\right)_s + \sum_v\sum_i\left(\frac{Qg_i}{t} \bullet \left(\frac{\chi}{Q}\right) \bullet DF_i^v\right)_v\right]$$

where

$\bar{D}_\gamma$ = noble gas gamma dose rate to total body (mrem/yr)

Qg_i/t = quantity of noble gas radionuclide *i* discharged (µCi) during time increment *t* (hr[TD1][TD2])

DF_i^s = factor converting unit noble gas nuclide *i* stack release to total body dose at ground level received from the overhead plume

$$\left[\frac{mrem}{(\mu Ci\ yr)/\sec}\right]$$

DF_i^γ = factor converting time integrated, ground level concentration of noble gas nuclide *i* to total body dose from gamma dose from gamma radiation $\left(\frac{mrem}{\mu Ci\ yr / m^3}\right)$

$\frac{1}{3600}$ = conversion (hr/sec)

Dose factor DF_i^s, for exposure to noble gases released from an elevated stack are calculated by a finite plume model implemented in the MIDAS program.[la] It assumes decay of short-lived nuclides, an air dose-to-tissue dose conversion of 1.11, and that the dose to internal organs is equivalent to the tissue dose. In this model, meteorological data concurrent with the period of release are used to evaluate the dose rate.

The dose from noble gases released from a vent (near ground level) or to skin from an elevated release is derived from a semi-infinite cloud model. Noble gas semi-infinite cloud gamma-to-total body dose factors, DF_i^γ, are listed in Table 3-4.

When the total body and organ doses from noble gases are computed as required by Section 7.3.1, the nearby resident exposed to the maximal ground-level noble gas concentration (maximum χ/Q) is selected as the receptor.

Alternatively, the dose from noble gases may be computed at 1260 meters NNW of the reactor, a location identified in Figure 3-2, where the nearby resident who may be exposed to maximal ground-level noble gas concentrations (maximum χ/Q) is selected as the receptor.[c] In that case, values of the dose factors DF_i^s and DF_i^γ in Table 3-4 are employed.[m] Total body dose factors for exposure from a plume from an elevated, stack release, DF_i^s, in Table 3-4 were computed with the aid of an NRC code, RABFIN. In those computations, reference meteorology was assumed as the basis of atmospheric dispersion. For discharge from a vent, reference meteorological dispersion at the 1260 m NNW, $(\chi/Q)_v$, is 4.3 x 10^{-6} sec/m^3.

[a] David Slade, ed., Meteorology and Atomic Energy 1968, TID-24190,pp. 350-355.

[m] Dose transfer factors DF_i^γ in Table 3.4 are a units conversion of values in Regulatory Guide 1.109, Table B-1, column 5.

[c] Due to limitations of the Midas© software used to perform these calculations, a conservative higher calculated dose may be reported. When using Annualized Average Meteorological data, the Midas system uses known X/Q values from the 1 mile radius to extrapolate X/Q values at the site boundary. The resulting maximum site boundary X/Q value is greater than 4.3 x 10^{-6} sec/m^3 and is located on the site boundary toward the SSE.

3.5.1.2 Skin Dose Rate

The skin dose equivalent rate due to radioactive noble gas is calculated with the equation:

$$\bar{D}\beta = \frac{1}{3600}\left[\sum_i S_{\beta i}\left(\frac{Q_i}{t} \bullet \left(\frac{\chi}{Q}\right)\right)_s + \sum_v \sum_i S_{\beta i}\left(\frac{Q_i}{t} \bullet \left(\frac{\chi}{Q}\right)\right)_v \right]$$

where

$\bar{D}_\beta$ = noble gas beta dose rate to skin (mrem/yr)

$\frac{Q_i}{t}$ = quantity of noble gas radionuclide *i* (µCi) discharged during time increment *t* (hr)

$S_{\beta i}$ = factor converting time integrated ground level concentration of noble gas to skin dose from beta radiation $\left[\frac{mrem\ m3}{\mu Ci\ yr}\right]$

$\frac{1}{3600}$ = conversion (hr/sec)

Compliance with Section 6.2.2.a dose rate to skin is evaluated by calculating the noble gas beta dose equivalent rate offsite at a location 1260 meters NNW of the Station, which is also identified in Figure 3-2.[d] At that location, the reference atmospheric dispersion factors to be used in the calculations are:

$$\left(\frac{\chi}{Q}\right)_v = 4.3 \bullet 10^{-6} \sec/m^3 \quad and \quad \left(\frac{\chi}{Q}\right)_s = 2.8 \bullet 10^{-7} \sec/m^3$$

Alternatively, averaged meteorological dispersion data coincident with the period of release may be used to evaluate the dose rate. The semi-infinite noble gas cloud-to-skin dose equivalent factors are in Table 3-4. They are also derived from Regulatory Guide 1.109, Table B-1.

[d] Due to limitations of the Midas© software used to perform these calculations, a conservative higher calculated dose may be reported. When using Annualized Average Meteorological data, the Midas system uses known X/Q values from the 1 mile radius to extrapolate X/Q values at the site boundary. The resulting maximum site boundary X/Q value is greater than 4.3×10^{-6} sec/m^3 and is located on the site boundary toward the SSE.

3.5.2 Iodine, Tritium, and Particulates

Section 6.2.2.b provides a limit on iodine-131, iodine-133, H-3, and on radioactive particulates having 8 day or longer half-lives in air in the unrestricted area around the Station. In the event airborne effluent from the Station causes a radioactive noble gas effluent monitor to alarm (except when alarm is due to the performance of a Surveillance Test Procedure) or if the assessment required by Section 7.2.4 shows Section 6.2.4 to have been exceeded, an assessment of compliance with Section 6.2.2.b will be performed using a method described in this section.

3.5.2.1 Organ Dose Rate[e]

Compliance with Section 6.2.2.b is assessed by calculating the dose rate* to the most exposed organ of an assumed adult member of the public inhaling airborne I-131, I-133, H-3, and inhaling radioactive particulates having half-lives of 8 days or longer at the location in the unrestricted area having the maximum potential concentration of the effluents (i.e., the location at which reference meteorological data indicates minimum atmospheric dispersion from the Station (max χ / Q).

The organ dose rate is calculated with the following equations:

For a vent discharge:

$$\overline{D}_{anv} = \frac{8.766E-3}{TE-TB} \sum_i \sum_k Q_{ikv} \ TA_{ani} \left(\frac{\chi_i}{Q}\right)_v$$

For an offgas stack discharge:

$$\overline{D}_{ans} = \frac{8.766E-3}{TE-TB} \sum_i \sum_k Q_{iks} \ TA_{ani} \left(\frac{\chi_i}{Q}\right)_s$$

Combining separate release points gives

$$\overline{D}_{an} = \overline{D}_{ans} + \sum_v \overline{D}_{anv}$$

where

$\overline{D}_{an}$ = the dose equivalent rate to organ *n* of a person in age group *a* due to radionuclides discharged in airborne effluents during time interval TB to TE (mrem/yr)

$\overline{D}_{ans}$ = dose equivalent rate from a stack discharge (mrem/yr)

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[e]For inhaled or ingested radioactive material, the consequent "dose" means the committed dose equivalent. The "dose rate" is the committed dose equivalent per unit of time of exposure to the radioactive material in the environment.

$\overline{D}_{anv}$ = dose equivalent rate from a vent discharge (mrem/yr)

Q_{iks}, Q_{ikv} = quantity of radionuclide *i* released in a given effluent stream based on analysis *k* (µCi) during discharged time increment TB to TE (hr) of interest

Q_{iks}, Q_{ikv} = quantity of radionuclide *i* released in a given effluent stream based on analysis *k* (µCi) during discharged time increment TB to TE (hr) of interest

TA_{ani} = factor converting airborne concentration of radionuclide *i* to dose commitment to organ *n* of a person in age group *a* where exposure is directly to airborne material

$$\left(\frac{mrem}{(Ci \sec)/m3}\right)$$

$\left(\frac{\chi i}{Q}\right)_s, \left(\frac{\chi i}{Q}\right)_v$ = atmospheric dispersion from stack and vent, respectively, to ground level at location of interest (sec/m^3)

$8.766E-3$ = Conversion (1 Ci/^{1}E6 µCi)(8766 hr/yr)

Radionuclides other than noble gases airborne effluent are measured and quantified as described in section 3.2. Normally, radioactive material measured in effluent is assumed to be discharged uniformly over the period represented by the sample.

The averaging time of the measured releases used to evaluate compliance will not exceed 92 days for Sr-89 and Sr-90 and will not exceed 31 days for the other radionuclides.

The maximum offsite exposure potential is expected to occur at 1260 meters NNW of the Station where the reference atmospheric dispersion, to be used in the calculation is

$\left(\frac{\chi}{Q}\right)_v$ = $4.3 \bullet 10^{-6}$ sec/m^3 and

$\left(\frac{\chi}{Q}\right)_s$ = $2.8 \bullet 10^{-7}$ sec/m^3

Currently, compliance with Section 6.2.2.b is evaluated by calculating an adult inhalation dose rate at 1260 meters NNW of the Station[nf]. The dose transfer factors, TA_{ani}, used in the computation are tabulated in Appendix A.

[f]Due to limitations of the Midas© software used to perform these calculations, a conservative higher calculated dose may be reported. When using Annualized Average Meteorological data, the Midas system uses known X/Q values from the 1 mile radius to extrapolate X/Q values at the site boundary. The resulting maximum site boundary X/Q value is greater than 4.3×10^{-6} sec/m^3 and is located on the site boundary toward the SSE.

3.5.2.2 Rainwater Recapture of Tritium

The phenomenon of rainwater recapture of the tritium present in gaseous effluents has been observed on site. A liquid pathway analysis of the activity present in the rainwater runoff has been performed. The maximum dose to a member of the public from this liquid has been conservatively estimated to be less than 0.01 mrem per year.

3.6 Noble Gas Gamma Radiation Dose Accumulated in Air

Section 6.2.3 requires that the offsite air dose during any calendar quarter not exceed 5 mrad and the annual air dose not exceed 10 mrad from noble gas gamma radiation. Section 7.2.3.1 requires a monthly calculation assessment to verify that the cumulative air dose due to gamma radiation from radioactive noble gas released in gaseous effluents during the quarter and year do not exceed Section 6.2.3.

The distribution of radioactive noble gases in gaseous releases and the quantity of radioactive noble gas discharged during an interval of time are determined as described in section 3.2 herein.

The gamma radiation dose to air offsite as a consequence of noble gas discharge from DAEC is calculated with the

$$D\gamma = \sum_i \sum_j \left(\Delta Q_j \bullet g_i \bullet A\gamma_i^s \right) + \sum_v \sum_i \left(A\gamma_i^v \sum_j \Delta Q_j \bullet gi \bullet \left(\frac{\chi}{Q} \right)_v \right)$$

where

D_γ = noble gas gamma dose to air due to effluent from stack and vent (mrad)

ΔQ_j = total measured radioactivity release via stack or vent measured by noble gas effluent monitor during counting interval *j* (µCi)

g_i = the fraction of radioactive gas in a given effluent stream attributable to noble gas radionuclide *i*.

$A\gamma_i^s$ = factor converting unit release of noble gas radionuclide *i* from the stack to air dose at ground-level received from gamma radiation from the overhead plume (mrad/µCi)

$A\gamma_i^v$ = factor converting time integrated, ground-level concentration of noble gas to air dose from gamma radiation

$$\left(\frac{mrad}{(\mu Ci \sec)/m^3} \right)$$

$\left(\frac{\chi}{Q}\right)_v$ = atmospheric dispersion factor for a vent (ground-level or building wake) discharge (sec/m^3)

Section 7.2.3.1 is satisfied by calculating the noble gas gamma radiation dose to air at the offsite location identified in Figure 3-2. At that location, 1260 meters NNW of the Station[h], the reference*[g] atmospheric dispersion factor to be used is

$$\left(\frac{\chi}{Q}\right)_v = 4.3 \bullet 10^{-6} \sec/m^3$$

Values of $A\gamma^s$ and $A\gamma^v$ appropriate for use at that location, assuming reference meteorological conditions, are listed in Table 3-3. An NRC code, RABFIN, modified for sector width averaged meteorology was used to calculate the air dose transfer factors, $A\gamma_i^s$ for a stack discharge, in DODAM Table 3-3. They represent air dose at ground level 1260 meters NNW of the station from a unit release of each radionuclide *i* from the stack. Dose transfer factors for vent discharges, $A\gamma_i^v$, are equivalent to factors in Regulatory Guide 1.109, Table B-1, g-air expressed in different units.

[g] Due to limitations of the Midas© software used to perform these calculations, a conservative higher calculated dose may be reported. When using Annualized Average Meteorological data, the Midas system uses known X/Q values from the 1 mile radius to extrapolate X/Q values at the site boundary. The resulting maximum site boundary X/Q value is greater than 4.3 x 10^{-6} sec/m^3 and is located on the site boundary toward the SSE.

[h] Reference atmospheric conditions are summarized and discussed in "Duane Arnold Energy Center, Evaluation of Liquid and Gaseous Effluent Releases in Accordance with 10 CFR 50 Appendix I," submitted to NRC June 3, 1976, Reference atmospheric dispersion factors tabulated therein, also appear in Appendix B herein.

3.6.1 Alternate Method of Evaluating Compliance with Gamma Air Dose Limits

Alternatively, the gamma radiation dose to air offsite may be calculated with the equation

$$D_\gamma = \frac{1}{0.8}\sum_j \left(\Delta Q_j \bullet A_{\gamma_{seff}}\right)_s + \frac{1}{0.8}\sum_v \sum_j \left(\Delta Q_j \bullet \left(\frac{\chi}{Q}\right) \bullet A_{\gamma_{veff}}\right)_v$$

where

$A_{\gamma_{seff}}$ = an effective dose conversion factor based on the typical radionuclide distribution in stack releases converting unit release of radioactive noble gases from the stack to air dose at ground level at a specific location (mrad/µCi).

$A_{\gamma_{veff}}$ = an effective dose conversion factor based on the typical radioactive distribution in vent releases converting a time integrated, ground level concentration of noble gases to air dose from gamma radiation $\left(\frac{mrad}{(\mu Ci \ \sec/m^3)}\right)$

0.8 = a factor of conservatism which compensates for variability in radionuclide distribution

The derivation and basis of the effective gamma air dose conversion factor are provided in Appendix B. Values of the effective factors are tabulated in Table 3-3. By inserting the appropriate values for $D\gamma$ (5 mrad/quarter γ-air dose) and for $A\gamma_{seff}$ (1.6×10^{-11} mrad/µCi) or Ag_{veff} (6.4×10^{-5} mrad/(µCi sec/m^3)) into the equation above and solving for either (ΔQ_j)s or (ΔQ_j)v, respectively, release quantities of noble gases from either the stack or vent corresponding to the technical specification limit of 5 mrad/quarter (total for all release points) may be determined. The limit of 5 mrad/quarter is described in section 6.2.3 and is for the total of all release points. At the location, 1260 meters NNW of the station, (which is the controlling location based on reference meteorology) the release limits are individually

Release Point	Quarterly Limit (Ci)	Annual Limit (Ci)
Stack	2.5×10^5	5.0×10^5
Vent	1.25×10^4	2.5×10^4

The following equations may be used to assess both the stack and vent discharges for compliance with the quarterly release limits on noble gas gamma dose to air.

$$\frac{\sum_j (\Delta Q_j)_s}{250{,}000} + \frac{\sum_v \sum_j (\Delta Q_j)_v}{12{,}500} \leq 1$$

or, on a monthly rate basis (although not a requirement)

$$\frac{\sum_j (\Delta Q_j)_s}{250{,}000} + \frac{\sum_v \sum_j (\Delta Q_j)_v}{12{,}500} \leq \frac{1}{3}$$

The equations which may be used to assess both the stack and vent discharges for compliance with the annual air dose limits from noble gas are:

$$\frac{\sum_j (\Delta Q_j)_s}{500{,}000} + \frac{\sum_v \sum_j (\Delta Q_j)_v}{25{,}000} \leq 1$$

or, on a monthly rate basis (although not a requirement)

$$\frac{\sum_j (\Delta Q_j)_s}{500{,}000} + \frac{\sum_v \sum_j (\Delta Q_j)_v}{25{,}000} \leq \frac{1}{12}$$

As long as these relations are satisfied for both stack and vent releases of noble gases, no additional calculations are needed to verify compliance with the gamma-air dose limits of Section 6.2.3. Calculations of beta air doses per Section 3.6 may be omitted as discussed in Appendix B.

3.7 Noble Gas Beta Radiation Dose Accumulated in Air

Section 6.2.3 requires that the offsite air dose during any calendar quarter not exceed 10 mrad from noble gas beta radiation and not exceed 20 mrad during any calendar year. Section 7.2.3.1 requires a monthly assessment to verify that the cumulative air dose due to beta radiation from radioactive noble gas released in gaseous effluents not exceed either limit of Section 6.2.3.

The radioactive noble gas distribution and activity discharged are determined as described in paragraph 3.6 herein.

The beta radiation dose to air offsite as a consequence of noble gas released from the Station is calculated with the equation:

$$D\beta = \sum_i A\beta_i \left[\Delta Q_j g_i \frac{\chi}{Q} \right]_s + \sum_v \sum_i A\beta_i \sum_j \left[\Delta Q_j g_i \frac{\chi}{Q} \right]_v$$

where

D_β = noble gas beta dose to air due to stack and vent releases (mrad)

A_{β_i} = factor converting time-integrated, ground-level concentration of noble gas radionuclide *i* to air dose from beta radiation $\left(\frac{mrad - m^3}{\mu Ci - \sec} \right)$

$\left(\frac{\chi}{Q} \right)_s$ = atmospheric dispersion factor for a discharge via the stack (sec/m^3)

$\left(\frac{\chi}{Q} \right)_v$ = atmospheric dispersion factor for a vent (ground level or building wake) discharge (sec/m^3).

Specification 7.2.3.1 is satisfied by calculating the noble gas beta radiation dose to air at the location identified on Figure 3-2. At that location, 1260 meters NNW of the reactor[i], the reference atmospheric dispersion factors to be used are

$\left(\frac{\chi}{Q} \right)_s$ = 2.8×10^{-7}

$\left(\frac{\chi}{Q} \right)_v$ = 4.3×10^{-6}

[i]Due to limitations of the Midas© software used to perform these calculations, a conservative higher calculated dose may be reported. When using Annualized Average Meteorological data, the Midas system uses known X/Q values from the 1 mile radius to extrapolate X/Q values at the site boundary. The resulting maximum site boundary X/Q value is greater than 4.3×10^{-6} sec/m^3 and is located on the site boundary toward the SSE.

3.8 Dose Due to Iodine and Particulates in Gaseous Effluents[j]

Section 6.2.4 requires that I-131, I-133, H-3, and radioactive material in particulate form having half-lives greater than 8 days in gaseous effluents released to the area offsite cause no more than 7.5 mrem to any organ of a member of the public during a calendar quarter and no more than 15 mrem during any calendar year. Section 7.2.4 requires an assessment at least once every month to verify that the cumulative dose commitment does not exceed either limit of Section 6.2.4.

Airborne releases are discharged either via the offgas stack as an elevated release or via building vents and treated as a ground-level, building wake, or split wake release. Radionuclides mentioned above in airborne effluents that are measured by the sampling and analysis schedule in Table 7.2-2 are included in the release term used to calculate doses. Section 3.2 describes the quantification of these radionuclides other than noble gases.

A person may be exposed directly to an airborne concentration of radioactive material discharged in effluent and indirectly via pathways involving deposition of radioactive material onto the ground. Dose estimates account for the separate exposure pathways. The dose commitment to a person offsite associated with a gaseous release, Q_{ik}, of radioactive material other than noble gas is calculated with the appropriate one(s) of the following equations

for a stack release:

$$D_{anske} = \sum_i Q_{iks} \left[TA_{anie} \left(\frac{\chi}{Q}\right)_s + TG_{anie} \left(\frac{D}{Q}\right)_s \right]$$

for a vent release:

$$D_{anvke} = \sum_i Q_{ikv} \left[TA_{anie} \left(\frac{\chi}{Q}\right)_v + TG_{anie} \left(\frac{D}{Q}\right)_v \right]$$

[j]The dose to any organ of a person arising from radioactive iodine-131, iodine-133, tritium, and radioactive material in particulate form having half-lives greater than 8 days. Noble gases not considered.

where

D_{anske} = the dose commitment (mrem) to organ *n* of a person in age group *a* via exposure pathway *e* due to radionuclides identified in analysis *k* of a stack release where the analysis is required by Table 7.2-2.

D_{anvke} = the dose commitment via pathway *e* from a vent release (mrem)

TA_{anie} = factor converting airborne concentration of radionuclide *i* to dose commitment to organ *n* of a person in any group *a* where exposure is directly to airborne material via exposure pathway *e*.

$$\left(\frac{mrem}{(Ci\ \sec)/m^3}\right)$$

TG_{anie} = factor converting ground deposition of radionuclide *i* to dose commitment organ *n* of a person in age group *a* where exposure is directly or indirectly to radioactive material that has been deposited on the ground via exposure pathway *e*. $\left(\frac{mrem}{Ci/m^2}\right)$

Q_{ik} = quantity of radionuclide *i* released in a given effluent stream based on analysis *k* (Ci)

$\left(\frac{D}{Q}\right)_s, \left(\frac{D}{Q}\right)_v$ = relative deposition factor, i.e., factor converting airborne effluent discharge from stack or vent respectively, to a real deposition on land (m^{-2}).

The analysis index *k* may represent either an analysis of a grab sample, a weekly composite analysis, a monthly composite analysis, or a quarterly composite analysis.

Since tritium in water vapor is absorbed directly by vegetation, the tritium concentration in growing vegetation is proportional to the airborne concentration rather than to relative deposition as in the case of particulates. Thus the dose commitment from airborne tritium via vegetation (fruit and vegetables), air-grass-cow-milk, or air-grass-cow-meat pathways is calculated with the appropriate one(s) of the equations:

for a stack release

$$D_{anse} = \left(\frac{\chi}{Q}\right)_s \sum_i \sum_k Q_{iks}\ TA_{anie}$$

for a vent release

$$D_{anve} = \left(\frac{\chi}{Q}\right)_v \sum_i \sum_k Q_{ikv} \ TA_{anie}$$

The dose commitment accumulated by a person offsite is computed at least every 30 days to satisfy Section 7.2.4.1 but may be calculated as analytical results of effluent measurements, performed as specified in Table 7.2-2, become available.

The dose accumulated as a result of stack discharge is computed with

$$D_{ans} = \sum_e D_{anse}$$

and the dose accumulated as a result of vent discharge is computed with

$$D_{anv} = \sum_e D_{anve}$$

Doses committed during the same time period due to discharges from the stack and vents are additive, thus

$$D_{an} = D_{ans} + \sum_v D_{anv}$$

where

D_{an} = the dose commitment accumulated during the quarter to date as a result of all measured radioactive gaseous discharges except noble gases to any organ n, including total body, of a person offsite in age group a (mrem).

When the dose to a person from iodine and particulates discharged in gaseous effluents is calculated as required by Section 7.2.4, appropriate environmental pathways (from among those for which dose transfer factors are provided in Appendix A) will be evaluated. The dose calculated is to a receptor at the location of the nearby residence experiencing the minimum atmospheric dispersion at ground-level from the station, i.e., maximum $\frac{X}{Q}$, concurrent with the effluent discharge. Alternatively, the dose may be calculated to a receptor at the location identified in Figure 3-2 where reference atmospheric dispersion and deposition factors are:

$$\left(\frac{\chi}{Q}\right)_s = 3.1 \bullet 10^{-7} \sec / m^3 \quad \left(\frac{D}{Q}\right)_s = 7.2 \bullet 10^{-9} m^{-2}$$

$$\left(\frac{\chi}{Q}\right)_v = 3.9 \bullet 10^{-6} \sec / m^3 \quad \left(\frac{D}{Q}\right)_v = 1.3 \bullet 10^{-8} m^{-2}$$

Food pathways are evaluated at the location of food production based on minimum atmospheric dispersion at ground-level concurrent with the effluent discharge or, alternatively, with reference meteorology applicable at that location. Seasonal appropriateness of pathways is considered. The air-grass (fresh or stored)-cow-milk-man pathway is evaluated** where a cow is located, 2650 meters WNW of DAEC, reference atmospheric deposition factors are:

$$\left(\frac{D}{Q}\right)_s = 2.1 \bullet 10^{-9}\, m^{-2} \quad \left(\frac{D}{Q}\right)_v = 4.28 \bullet 10^{-9}\, m^{-2}$$

**This receptor is historical in nature. See the current land use census to verify this point as the most conservative receptor location for the pathway.

3.8.1 Alternate Method of Evaluating Doses Due to Iodine and Particulates in Gaseous Effluents

Alternatively, the dose commitment to a maximally exposed, hypothetical individual may be calculated by the equation

$$D_{inf\text{-}thy} = \frac{1}{0.8} \bullet \sum_i \left[Q_i \bullet TG_{inf\text{-}thy\text{-}I\text{-}131} \bullet (D/Q) \right]_s$$

$$= \frac{1}{0.8} \bullet \sum_i \left[Q_i \bullet TG_{inf\text{-}thy\text{-}I\text{-}131} \bullet (D/Q) \right]_v$$

where:

$D_{inf\text{-}thy}$ = the dose commitment accumulated during the quarter to date to a hypothetical infant's thyroid as a result of the releases of I-131 (mrem).

Q_i = the measured quantity of I-131 released in a given effluent stream, stack or vent (Ci)

$TG_{inf\text{-}thy\text{-}I\text{-}131}$ = the dose transfer factor for the infant thyroid from the cow-milk pathway for I-131 measured in the effluent stream

$$\left(\frac{mrem}{Ci / m^2}\right)$$

$\frac{1}{0.8}$ = a factor of conservatism which accounts for the dose contribution for releases of particulate radioactive material other than I-131

When the maximum organ dose is evaluated by using the equation above, analyses of other organ doses via other pathways are not needed to demonstrate compliance within the dose limits of Section 6.2.4.

The rationale for only evaluating the dose contribution of I-131 is derived from an evaluation of the radioactive material releases and the environmental pathways. The air-grass-cow-milk-man pathway is by far the controlling pathway and the infant's thyroid is the limiting organ. This pathway typically contributes greater than 90% of the total calculated dose to the infant's thyroid and I-131 contributes essentially all of the dose (~95%). Therefore, it is possible to demonstrate compliance with the dose limits of Section 6.2.4 by the conservative calculation method presented above.

3.9 Dose to a Person from Noble Gases

Section 7.3.1 requires the calculation of the dose or dose commitment to a person offsite exposed to 12 consecutive months of radioactive liquid and gaseous effluents from the Station. One component of personal dose is total body irradiation by gamma rays from noble gases. Another is irradiation of skin by beta and gamma radiation from noble gases. The methods of calculating these doses are presented in sections 3.9.1 and 3.9.2.

The amount of radioactive noble gas discharged is determined in the manner described in section 3.2.

3.9.1 Gamma Dose to Total Body

The gamma radiation dose to the whole body of a member of the public as a consequence of noble gas released from the Station is calculated with the equation:

$$D\gamma_i = \sum_i \left(Q_i \bullet P\gamma_i^s\right)_s + \sum_v \sum_i \left(Q_i \bullet \left(\frac{\chi}{Q}\right) \bullet P\gamma_i^v\right)_v$$

where:

D_γ = noble gas gamma dose to total body (mrem)

Q_i = quantity of noble gas nuclide *i* released via stack or vent (Ci)

$P\gamma_i^s$ = factor converting unit noble gas nuclide *i* stack release to total body dose at ground level received from the overhead plume (mrem/Ci)

$P\gamma_i^v$ = factor converting time integrated, ground level concentration of noble gas nuclide *i* to total body dose from gamma radiation

$$\left(\frac{mrem}{Ci\ \sec/m^3}\right)$$

The dose to total body and organs other than skin from noble gases released from an elevated stack are calculated by a finite plume model.[k] It assumes decay of short-lived nuclides, a residential shielding factor of 0.7, an air dose-to-tissue dose conversion of 1.11, and that the dose to internal organs is equivalent to the tissue dose. Dose factors for exposure to noble gas plume gamma radiation, $P\gamma_i^s$, are computed by the MIDAS program.

The dose from noble gases released from a vent (near ground level) or to skin from an elevated release is derived from a semi-infinite cloud model. Noble gas semi-infinite cloud gamma-to-total body dose factors, $P\gamma_i^v$, are listed in Appendix A under the plume pathway.

When the total body and organ doses from noble gases are computed as required by Section 7.3.1, the nearby resident exposed to the maximal ground-level noble gas concentration (minimum χ/Q) is selected as the receptor.

Alternatively, the dose from noble gases may be computed at 1260 meters NNW of the reactor where the nearby resident who may be exposed to maximal ground-level noble gas concentrations (minimum χ/Q) is selected as the receptor. In that case, values of the dose factors $P\gamma_i^s$ and $P\gamma_i^v$ in Table 3-5 are employed.[l] Reference meteorological dispersion for vent discharges at that location, $(\chi/Q)_v$, is $4.3 \times 10^{-6}\ \sec/m^3$.

3.9.2 Dose to Skin

The beta radiation dose to the skin of a member of the public due to beta radiation from noble gas released from the Station may be calculated with the equation

$$D_\beta = \sum_i S_{\beta i}\left[Q_i \bullet \left(\frac{\chi}{Q}\right)\right]_s + \sum_v \sum_i S_{\beta i} \bullet \left[Q_i \bullet \left(\frac{\chi}{Q}\right)\right]_v$$

where

D_β = noble as dose to skin (mrem)

$S_{\beta i}$ = factor converting time integrated ground level concentration of noble gas to skin dose from beta radiation

$$\left(\frac{mrem}{Ci\ \sec/m^3}\right)$$

[k]David Slade, ed., Meteorology and Atomic Energy 1968, TID-24190, pp. 350-355.

Semi-infinite cloud noble gas beta-to-skin dose factors, $S_{\beta i}$ appear in Table 3-5.

The total dose to the skin from noble gases is approximately equal to the beta radiation dose to the skin plus the gamma radiation dose to the total body.

When the skin dose due to noble gas beta radiation is computed as required by Section 7.3.1, the receptor selected is the nearby resident exposed to maximal ground-level concentrations (maximum χ / Q).

Alternatively, the skin dose to a postulated receptor (resident) at 1260 meters NNW of the reactor may be calculated.

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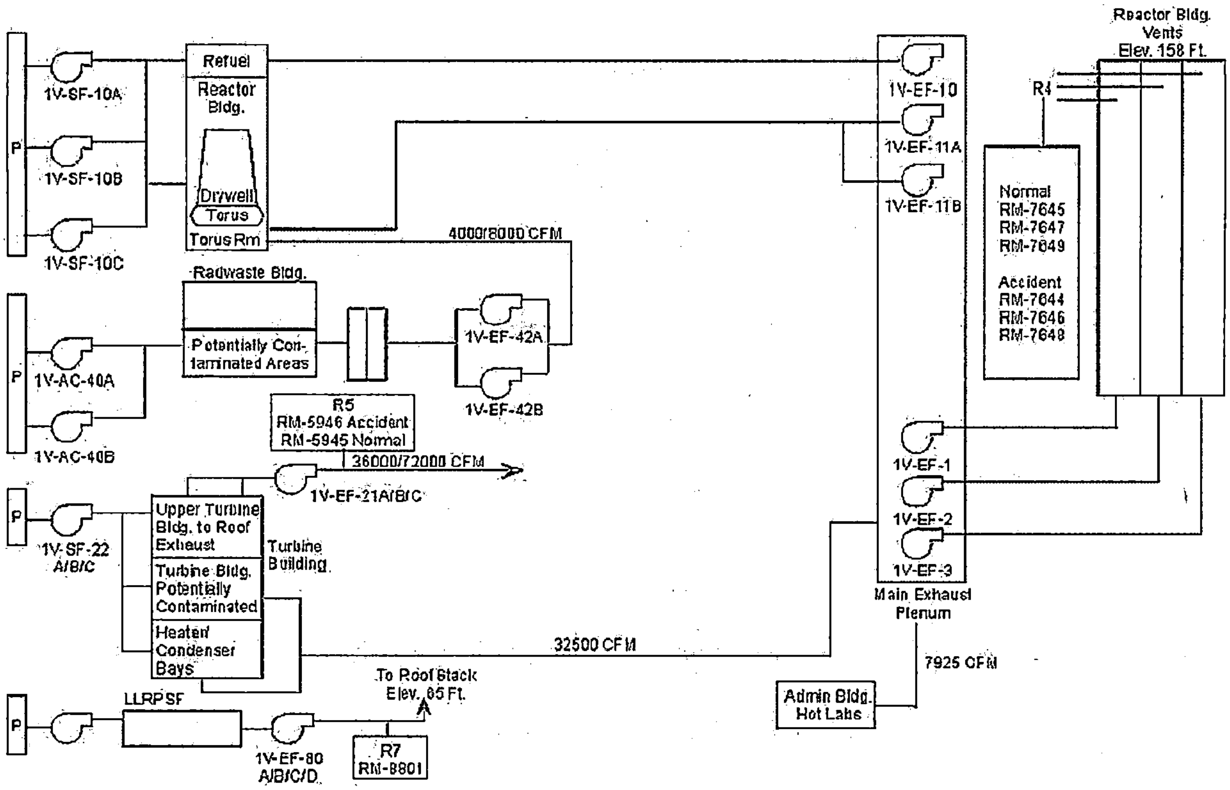
[1]Dose transfer factors $P\gamma_i^v$ in Table 3.5 and in Appendix A under the plume pathway are the same.

Figure 3.1
Gaseous Radioactive Waste Flow Diagram



R3 Offgas Stack Radiation Monitoring System

R4 Reactor Building Exhaust Vent Monitoring System

R5 Turbine Building Exhaust Vent Monitoring System

R7 LLRPSF Exhaust Vent Monitoring System

Figure 3-2
Dose Calculation Sites

[TD3]

Site	Description	Distance from Plant
A	Noble gas gamma air dose:	481 meters South-Southeast
B	Noble gas beta air dose:	481 meters South-Southeast
C	Iodines and Particulates:	805 meters West
D	Aquatic pathways:	Cedar River
E	Most Exposed resident:	805 meters West

Table 3-1

Atmospheric Gaseous Release Points
at the Duane Arnold Energy Center

Parameter	RELEASE POINT			
	Offgas Stack	Reactor Building Vent	Turbine Building Vent	LLRPSF Building Vent
Release Height	328 feet	156 feet	90 feet	65 feet
Release Mode	Elevated	Wake-split	Wake-split	Wake-split
Effluent Source	Waste Gas System Standby Gas Treatment System	Reactor Building Radwaste Building Lower Turbine Building	Upper Turbine Building	LLRPSF Building and Storage Facility

Table 3-2
Computed Releases of Radioactive Noble Gases in Gaseous Effluent from Duane Arnold Energy Center

	STACK RELEASE		PLANT VENTS RELEASE	
Nuclide	(Ci/yr)[a]	Fraction[b]	(Ci/yr)[a]	Fraction[b]
Kr-83m	4.90E+01	2.53E-03	0	0
Kr-85m	2.34E+03	1.21E-01	7.40E+01	1.98E-02
Kr-85	1.40E+02	7.23E-03	0	0
Kr-87	1.56E+02	8.06E-03	1.36E+02	3.64E-02
Kr-88	1.65E+03	8.52E-02	2.36E+02	6.32E-02
Kr-89	6.40E+02	3.31E-02	0	0
Xe-131m	4.80E+01	2.48E-03	0	0
Xe-133m	3.50E+01	1.81E-03	0	0
Xe-133	1.24E+04	6.41E-01	3.92E+02	1.05E-01
Xe-135m	1.80E+01	9.30E-04	7.42E+02	1.99E-01
Xe-135	5.10E+02	2.63E-02	7.43E+02	1.99E-01
Xe-137	7.80E+02	4.03E-02	0	0
Xe-138	5.90E+02	3.05E-02	1.41E+03	3.78E-01
	19356	1.0	3733.	1.0

TABLE NOTATION

[a] Releases computed by BWR-GALE for DAEC Base Case gaseous radwaste treatment. Computed releases are included only to show the basis of the radionuclide distribution.

[b] This is the calculated distribution of radionuclides in gaseous effluents in each release pathway. To estimate radionuclide concentrations in a sample in which only the total activity concentration has been measured, multiply the total activity concentration by the fraction of respective radionuclides listed above.

Table 3-3
Transfer Factors for Maximum Offsite Air Dose

Radionuclide	Air Dose Transfer Factors		
	$A\gamma_i^s$ [a] $\left(\frac{mrad}{\mu Ci}\right)$	$A\gamma_i^v$ [b] $\left(\frac{mrad}{\mu Ci\ \sec/m^3}\right)$	A_{β_i} [b] $\left(\frac{mrad}{\mu Ci \sec/m^3}\right)$
Kr-83m	4.3E-14	6.1E-7	9.1E-6
Kr-85m	6.0E-12	3.9E-5	6.2E-5
Kr-85	8.4E-14	5.4E-7	6.2E-5
Kr-87	2.3E-11	2.0E-4	3.3E-4
Kr-88	6.4E-11	4.8E-4	9.3E-5
Kr-89	3.0E-11	5.5E-4	3.4E-4
Kr-90	---	5.2E-4	2.5E-4
Xe-131m	1.8E-12	4.9E-6	3.5E-5
Xe-133m	1.4E-12	1.0E-5	4.7E-5
Xe-133	1.5E-12	1.1E-5	3.3E-5
Xe-135m	1.1E-11	1.1E-4	2.3E-5
Xe-135	9.5E-12	6.1E-5	7.8E-5
Xe-137	2.6E-12	4.8E-5	4.0E-4
Xe-138	3.6E-11	2.9E-4	1.5E-4
Ar-41	4.4E-11	2.9E-4	1.0E-4

TABLE NOTATIONS

a An NRC code, RABFIN, modified for sector width averaged meteorology, was used to calculate the air dose transfer factors, $A\gamma_i^s$ for a stack discharge. They represent air dose at ground level 1260 meters NNW of the station from a unit release of each radionuclide *i* from the stack and dispersed by reference meteorology at the DAEC.

b Dose transfer factors for vent discharges, $A\gamma_i^v$ and $A_{\beta i}$ are equivalent to factors in Regulatory Guide 1.109, Table B-1, g-air and b-air, expressed in different units.

Table 3-4
Transfer Factors for Dose Equivalent Rate to A Person Offsite Due to Radioactive Noble Gases

Radionuclide	Dose Transfer Factors		
	DF_i^s [a, b, c] $\left[\frac{mrem}{(\mu Ci\ yr)/\sec}\right]$	DF_i^v [d] $\left[\frac{mrem}{(\mu Ci\ yr)/m^3}\right]$	$S_{\beta i}$ $\left[\frac{mrem}{(\mu Ci\ yr)/m^3}\right]$
Kr-83m	6.27E-9	7.56E-2	0
Kr-85m	1.81E-4	1.17E+3	1.46E+3
Kr-85	2.51E-6	1.61E+1	9.73E+3
Kr-87	6.97E-4	5.92E+3	9.73E+3
Kr-88	1.91E-3	1.47E+4	2.37E+3
Kr-89	9.14E-4	1.66E+4	1.01E+4
Kr-90	---	1.56E+4	7.29E+3
Xe-131m	4.83E-5	9.15E+1	4.76E+2
Xe-133m	3.61E-5	2.51E+2	9.94E+2
Xe-133	4.09E-5	2.94E+2	3.06E+2
Xe-135m	3.39E-4	3.12E+3	7.11E+2
Xe-135	2.84E-4	1.81E+3	1.86E+3
Xe-137	7.90E-5	1.42E+3	1.22E+4
Xe-138	1.08E-3	8.83E+3	4.13E+3
Ar-41	1.32E-3	8.84E+3	2.69E+3

TABLE NOTATIONS

a Receptor located 1260 meters NNW of Station
b Based on reference meteorology at DAEC
c Factors DF_i^s computed by computer code RABFIN, modified for sector width averaged meteorology
d Factors DF_i^v from Regulatory Guide 1.109, revision 1, Table B-1, column 5

Table 3-5
Transfer Factors for Dose Equivalent Rate to A Person Offsite Due to Radioactive Noble Gases

Radionuclide	Dose Transfer Factors		
	$P\gamma_i^s$ [a, b, c] $\left[\frac{mrem}{Ci}\right]$	$P\gamma_i^v$ [c, d] $\left[\frac{mrem}{(Ci\sec)/m^3}\right]$	$S_{\beta i}$ [d] $\left[\frac{mrem}{(Ci\sec)/m^3}\right]$
Kr-83m	1.39E-10	1.68E-3	---
Kr-85m	4.03E-6	2.60E+1	4.63E+1
Kr-85	5.58E-8	3.58E-1	4.25E+1
Kr-87	1.55E-5	1.31E+2	3.08E+2
Kr-88	4.25E-5	3.26E+2	7.51E+1
Kr-89	2.03E-5	3.68E+2	3.20E+2
Kr-90	---	3.46E+2	2.31E+2
Xe-131m	1.07E-6	2.03	1.51E+1
Xe-133m	8.05E-7	5.57	3.15E+1
Xe-133	9.03E-7	6.52	9.70
Xe-135m	7.49E-6	6.92E+1	2.25E+1
Xe-135	6.31E-6	4.01E+1	5.89E+1
Xe-137	1.75E-6	3.15E+1	3.87E+2
Xe-138	2.39E-5	1.96E+2	1.31E+2
Ar-41	2.92E-5	1.96E+2	8.52E+1

TABLE NOTATIONS

[a] Receptor located 1260 meters NNW of Station

[b] Based on reference meteorology at DAEC

[c] Factors $P\gamma_i^s$ and $P\gamma_i^v$ already account for a 0.7 residential shielding factor. Factors $P\gamma_i^s$ were computed by computer code RABFIN, modified for sector width averaged meteorology

[d] Factors $P\gamma_i^v$ and $S_{\beta i}$ from Regulatory Guide 1.109, revision 1, Table B-1

4.0 DOSE COMMITMENT FROM RELEASE OVER EXTENDED TIME

4.1 Dose Assessment for 10 CFR Part 50, Appendix I

Sections 7.1.3, 7.2.3, and 7.2.4 require quarterly and annual assessments to demonstrate compliance with Appendix I dose limits. The assessment includes the following calculations of dose as described by equations for:

1. total body and maximally exposed organ doses due to liquid effluent via drinking water and eating fish from the River and from consuming food irrigated with river water as in paragraph 2.5.

2. total body and maximally exposed organ doses due to gaseous effluents[a] other than noble gases as in paragraph 3.8.

3. doses to air offsite due to noble gas g as in paragraph 3.6 and due to noble gas β as in paragraph 3.7.

The dose calculations are based on liquid and gaseous effluents from the Station during each calendar quarter and for a calendar year, determined in accord with Tables 7.1-2 and 7.2-2.

Environmental concentrations depend on dispersion and dilution of the effluent. For aqueous effluents over extended time, the aquatic concentration is estimated according to section 2.2. Atmospheric dispersion and deposition factors used to estimate the dose commitment due to gaseous effluents are ordinarily derived from reference meteorological data. Otherwise, quarterly averaged or annual averaged meteorological conditions concurrent with the gaseous release being evaluated will be used to estimate atmospheric dispersion and deposition.

The receptor of the dose is described such that the dose to any resident near the Station is unlikely to be underestimated. That is, the receptor is selected on the basis of the combination of applicable pathways of exposure to gaseous effluent identified in the annual land use census and maximum ground level χ / Q at the residence. Conditions (i.e., location, χ / Q, and/or pathways) more conservative (i.e., expected to yield higher calculated doses) than appropriate for the maximally exposed individual may be assumed in the dose assessment.

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a Radioactive iodine-131, iodine-133, tritium, and radioactive material in particulate form having half-lives greater than 8 days.

Seasonal appropriateness of exposure pathways may be considered. Exposure by eating fresh vegetation or drinking milk from cows or goats fed fresh forage is an inappropriate assumption during the first or fourth calendar quarter; rather consumption of stored vegetation and stored forage is assumed during those quarters. Otherwise, during the second and third calendar quarters, exposure by eating fresh vegetation and/or drinking milk from cows or goats fed fresh forage is assumed where those pathways exist. Similarly, the liquid effluent-river-fish-man pathway is not assumed during the winter quarter.

Factors converting stack-released noble gas to gamma radiation dose from the overhead plume are calculated on the basis of reference meteorological data for the receptor location or alternatively, by the MIDAS program for a residential location offsite where maximum χ / Q at ground level occurs.

Other environmental pathway-to-dose transfer factors used in the dose calculations are provided in Appendix A.

4.2 Dose Assessment for 40 CFR Part 190 and 10 CFR 72.104

The regulations governing the maximum allowable dose or dose commitment to a member of the public from all uranium fuel cycle-sources of radiation and radioactive material in the environment is stated in 40 CFR Part 190 and 10 CFR 72.104. It requires that the dose or dose commitment to a member of the public from all sources not exceed 25 mrem/yr to the whole body or 25 mrem/yr to any organ or 75 mrem/yr to the thyroid. Section 7.3.1 requires calculation of the dose at least once every year to assess compliance with the regulation. More frequent calculations may be performed if higher than normal releases are experienced (twice the design objective rates in a single quarter).

Fuel cycle sources or nuclear power reactors other than the Station[b] itself do not measurably or significantly increase the radioactivity concentration in the vicinity of the Station; therefore, only radiation and radioactivity in the environment attributable to the Station itself are considered in the assessment of compliance with 40 CFR Part 190 and 10 CFR 72.104.

Contributions to the dose due to liquid and gaseous effluent are calculated as described by the equations for:

1. total body and maximally exposed organ doses due to liquid effluent via drinking water, consuming irrigated food and from eating fish from the River as in paragraph 2.5

2. total body dose due to noble gas γ as in paragraph 3.9.1

3. skin dose due to noble gas β as in paragraph 3.9.2

[b] The Station is defined as BOTH the Nuclear Reactor Facility and the Independent Spent Fuel Storage Installation (ISFSI).

4. total body and maximally exposed organ doses due to gaseous effluents[c] other than noble gases as in Paragraph 3.8.

Additionally, the contribution to total dose from direct radiation is assessed annually by using environmental TLDs.

The doses are calculated on the basis of liquid and gaseous effluents from the Station during 12 consecutive months, determined in accord with Tables 7.1-2 and 7.2-2. For the purpose of the Annual Radiological Environmental Report, doses are based upon release during a calendar year.

Aqueous radioactive material concentrations are estimated according to paragraph 2.2 on the basis of annual averaged stream flow. Annual averaged meteorological conditions concurrent with gaseous releases being evaluated are used to estimate atmospheric dispersion, deposition, and elevated plume gamma exposure.

The receptor of the dose is described such that the dose to any resident near the Station is not likely to be underestimated, although conditions more conservative than appropriate for the maximally exposed person may be assumed in the dose assessment.

Ordinarily, the receptor is selected on the basis of the applicable combination of existing pathways of exposure to gaseous effluent identified in the annual land use census and the maximum ground level χ / Q at the residence.

When assessing compliance with 40 CFR 190, Radiological Environmental Monitoring Program results may be used to indicate actual radioactivity levels in the environment attributable to the DAEC. These measured levels may be used to supplement the evaluation of doses to members of the public for assessing compliance with 40 CFR 190.

Factors converting stack-released noble gas to gamma radiation dose from the overhead plume are calculated on the basis of annual averaged meteorological data for the receptor location. Other environmental pathway-to-dose transfer factors are listed in Appendices A, B and C.

[c] Radioactive iodine-131, iodine-133, tritium, and radioactive material in particulate form having half-lives greater than 8 days.

5.0 ENVIRONMENTAL MONITORING PROGRAMS

Section 5.0 of the DODAM provides a description of the Radiological Environmental Monitoring Program (REMP) and the Ground Water Protection Program (GWPP). This section also contains descriptions of the Environmental Sampling Program Station Locations.

5.1 Radiological Environmental Monitoring Program

A program shall be provided to monitor the radiation and radionuclides in the environs of the station.

The program shall provide:

(1) representative measurements of radioactivity in the highest potential exposure pathways,

(2) verification of the accuracy of the effluent monitoring program and modeling of environmental exposure pathways.

The program shall:

(1) be contained in the DODAM,

(2) conform to the guidance of Appendix I to 10 CFR Part 50 and 10 CFR 72,

(3) include the following:

   (a) Monitoring, sampling, analysis, and reporting of radiation and radionuclides in the environment in accordance with the methodology and parameters in the DODAM.

   (b) A Land Use Census to ensure that changes in the use of areas at and beyond the SITE BOUNDARY are identified and that modifications to the monitoring program are made if required by the results of this census.

   (c) Participation in an Interlaboratory Comparison Program to ensure that independent checks on the precision and accuracy of the measurements of radioactive materials in environmental sample matrices are performed as part of the quality assurance program for environmental monitoring.

5.2 Ground Water Protection Program (GWPP)

A program shall be provided to prevent, detect and respond to inadvertent and radiological releases with the potential to reach ground water.

The program shall provide:

(1) for the prevention of inadvertent/unplanned radiological releases from plant systems, structures and components (SSCs) or during plant evolutions or work practices that represent an elevated risk of experiencing a release of licensed radiological material into the environment.

(2) aspects for monitoring, detecting and responding to unplanned/unmonitored releases of licensed radioactive material to the environment and a communications/notification plan that addresses internal notifications to management and communications/reporting to State and local stakeholders and regulators for specified events or conditions.

The program shall:

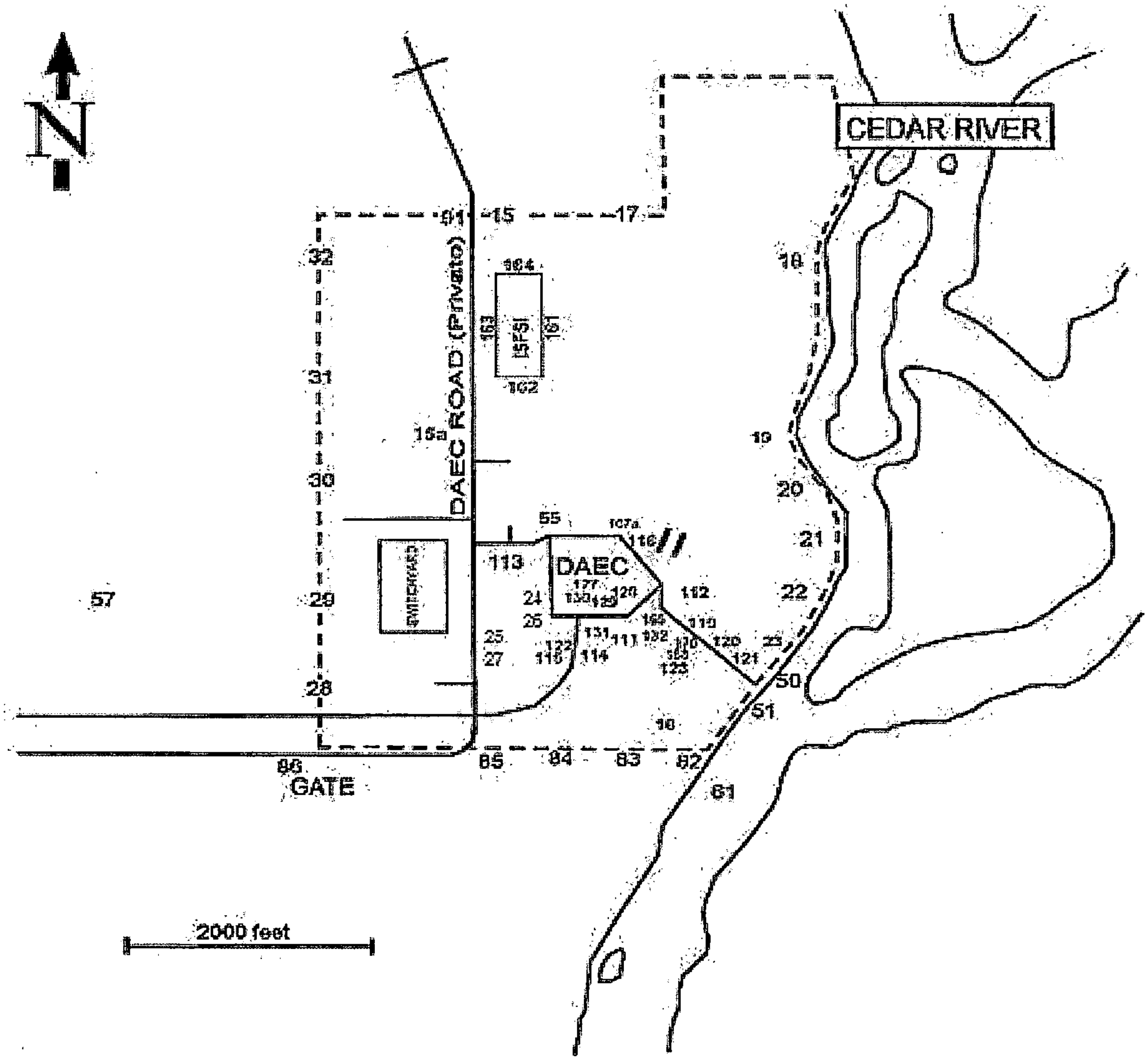
(1) be contained in the site administrative control procedure ACP 1411.35, "The DAEC Groundwater Protection Program".

(2) implement the Ground Water Protection Initiative Final Guidance Document, NEI-07-07[Final].

5.3 Sampling Station Locations

DODAM Table 5-1 "ENVIRONMENTAL SAMPLE STATIONS" is a list of locations where samples may be collected and does not represent a list of required samples. Environmental monitoring locations are shown on Figures 5-1 and 5-2. DAEC may conduct additional environmental monitoring exclusive of the requirements of Specifications 6.3.2 and ACP 1411.35.

Figure 5-1
Environmental Monitoring Programs
Sampling Near the Duane Arnold Energy Center



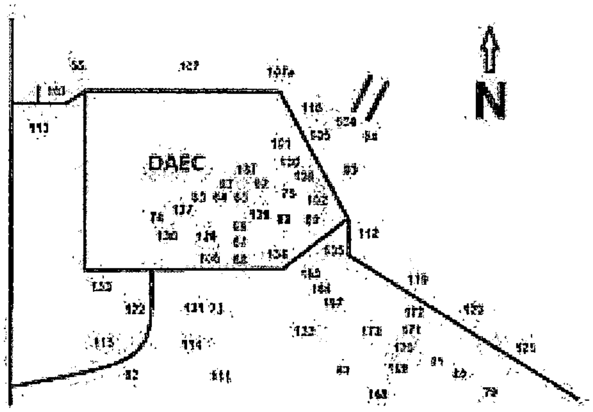


Figure 5-2
Radiological Environmental Monitoring Program
Sampling Stations Outside 0.5 Miles from DAEC

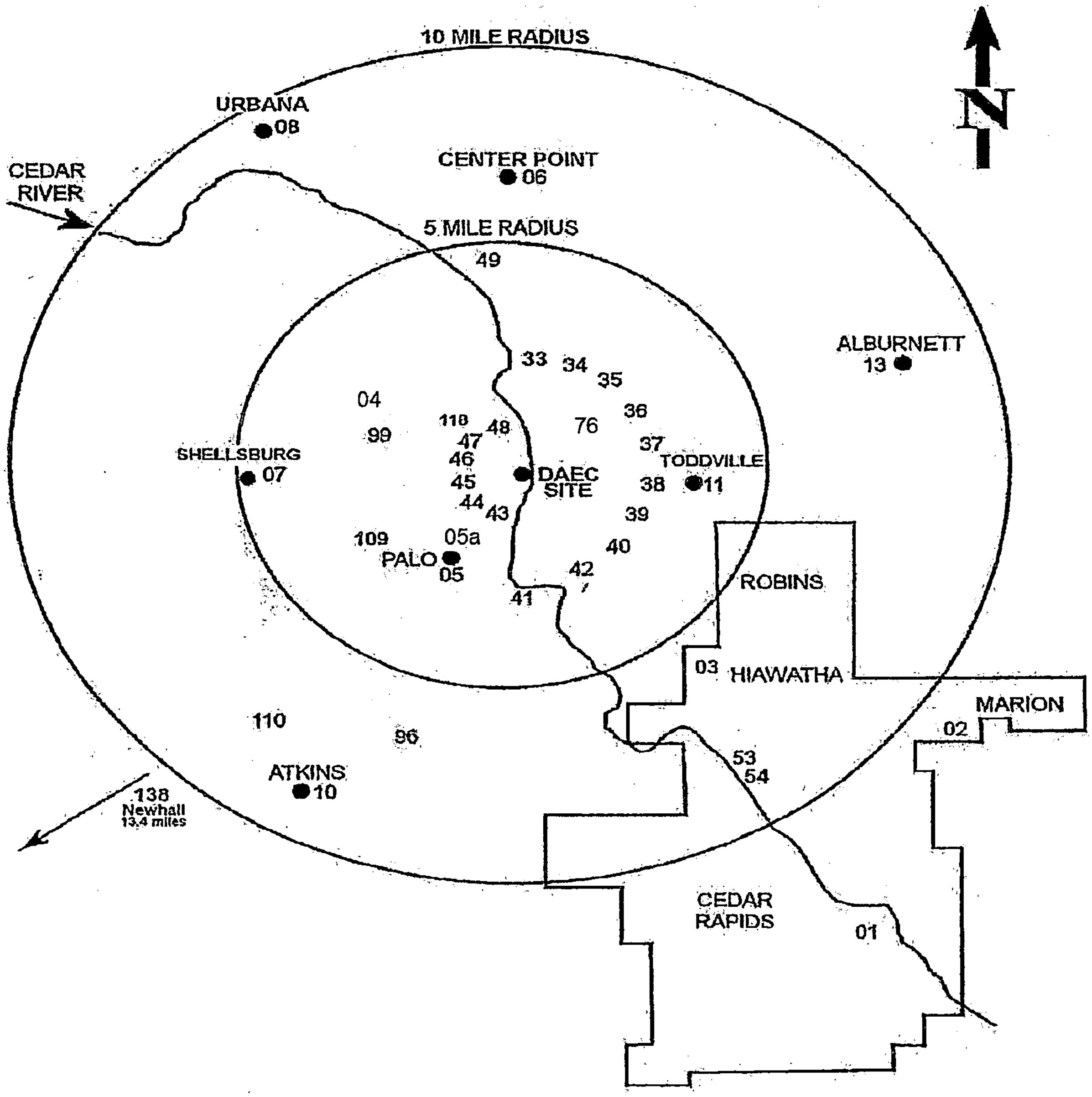


Table 5-1
ENVIRONMENTAL SAMPLE STATIONS

Station Number	Program	Station Location and Sample Type	
1	REMP	Location: Type:	Cedar Rapids, 20,800 meters SE TLD Control Airborne Particulate and Iodine Control – Not Currently Active
2	REMP	Location: Type:	Marion, 16,900 meters ESE TLD Control Airborne Particulate and Iodine Control – Not Currently Active
3	REMP	Location: Type:	Hiawatha, 10,800 meters SE TLD Control Airborne Particulate and Iodine – Not Currently Active
4	REMP	Location: Type:	Pleasant Creek SRA, 4960 meters NW TLD Control Airborne Particulate and Iodine
5	REMP	Location: Type:	Palo, 4,500 meters SSW TLD Control
5a	REMP	Location: Type:	Palo, 3,470 meters SSW TLD Control Airborne Particulate and Iodine
6	REMP	Location: Type:	Center Point, 9,660 meters N TLD Control Airborne Particulate and Iodine – Not Currently Active
7	REMP	Location: Type:	Shellsburg, 7,950 meters W TLD Control Airborne Particulate and Iodine – Not Currently Active
8	REMP	Location: Type:	Urbana, 15,000 meters NNW TLD Control Airborne Particulate and Iodine Control – Not Currently Active
9		Not Used	
10	REMP	Location: Type:	Atkins, 13,600 meters SSW TLD Control Airborne Particulate and Iodine Control – Not Currently Active
11	REMP	Location: Type:	Toddville, 4,980 meters E TLD Control Airborne Particulate and Iodine – Not Currently Active
12		Not Used	
13	REMP	Location: Type:	Alburnett, 14,500 meters ENE TLD Control Airborne Particulate and Iodine Control
14		Not Used	
15	REMP	Location: Type:	On-site, 1,050 meters NNW TLD Airborne Particulate and Iodine
15a	GWPP	Location: Type:	On-site, 505 meters NNW Soil – discontinued
16	REMP	Location: Type:	On-site, 520 meters SSE TLD Airborne Particulate and Iodine Vegetation Meteorology Tower
16a	GWPP	Location: Type:	On-site, 520 meters SSE Soil – discontinued Precipitation – discontinued
17	REMP	Location: Type:	On-site, 1,050 meters N TLD

Table 5-1
ENVIRONMENTAL SAMPLE STATIONS

Station Number	Program	Station Location and Sample Type	
18	REMP	Location: Type:	630 meters NNE TLD
19	REMP	Location: Type:	On-site, 590 meters NE TLD
20	REMP	Location: Type:	On-site, 550 meters ENE TLD
21	REMP	Location: Type:	On-site, 515 meters ENE TLD
22	REMP	Location: Type:	On-site, 535 meters ESE TLD
23	REMP	Location: Type:	On-site, 490 meters SE TLD
24	GWPP	Location: Type:	MW104 electrical vault W of lower level RadWaste Building inside Protected Area Ground Water
25	GWPP	Location: Type:	MW105 electrical vault N of contractor parking lot along DAEC Road Ground Water
26	GWPP	Location: Type:	MW106 electrical vault W of Lower Level RadWaste Building inside Protected Area Ground Water
27	GWPP	Location: Type:	MW107 electrical vault N of contractor parking lot along DAEC Road Ground Water
28	REMP	Location: Type:	On-site, 730 meters WSW TLD
29	REMP	Location: Type:	On-site, 630 meters W TLD
30	REMP	Location: Type :	On-site, 640 meters WNW TLD
31	REMP	Location: Type:	On-site, 1,020 meters NW TLD
32	REMP	Location: Type:	On-site, 1,110 meters NNW TLD
33	REMP	Location: Type:	4,340 meters N TLD
34	REMP	Location: Type:	3,930 meters NNE TLD
35	REMP	Location: Type:	2,800 meters NE TLD
36	REMP	Location: Type:	3,500 meters ENE TLD
37	REMP	Location: Type:	2,960 meters E TLD
38	REMP	Location: Type:	3,180 meters ESE TLD
39	REMP	Location: Type:	2,510 meters SE TLD
40	REMP	Location: Type:	Wickiup Hill Learning Center 2,430 meters SSE TLD Airborne Particulate and Iodine – Not Currently Active
41	REMP	Location: Type:	5,680 meters S TLD

Table 5-1
ENVIRONMENTAL SAMPLE STATIONS

Station Number	Program	Station Location and Sample Type	
42	REMP	Location: Type:	4,380 meters SSE TLD
43	REMP	Location: Type:	1,590 meters SSW TLD
44	REMP	Location: Type:	1,580 meters WSW TLD
45	REMP	Location: Type:	1,420 meters W TLD
46	REMP	Location: Type:	1,580 meters WNW TLD
47	REMP	Location: Type:	1,760 meters NW TLD
48	REMP	Location: Type:	1,680 meters NNW TLD
49	REMP	Location: Type:	Lewis Access, upstream of DAEC 6,750 meters NNW Fish Control Surface Water Control Bottom Sediment Control
50	REMP	Location: Type:	Plant Intake, 560 meters SE Bottom Sediment Control – discontinued Surface Water – discontinued
51	REMP	Location: Type:	Plant Discharge, 600 meters SE Bottom Sediment Surface Water
52	REMP	Location: Type:	Plant potable water supply Drinking Water
53	REMP	Location: Type:	Treated Municipal Water, 13,900 meters SE Drinking Water
54	REMP	Location: Type:	Inlet to Municipal Water Treatment System, 13,900 meters SE Drinking Water
55	REMP	Location: Type:	On-site, Production Wells Ground Water
56	REMP GWPP	Location: Type:	Generic control sample for sampling quality assurance Water, soil, vegetation, milk, sediment, fish, special samples
57	REMP	Location: Type:	Farm, 805 meters W Drinking Water Vegetation
58	REMP	Location: Type:	Farm, 974 meters WSW-SW Drinking Water Vegetation Meat sample
59	REMP	Location: Type:	Farm, 2,615 meters SE Vegetation
60		Not Used	
61	REMP	Location: Type:	Cedar River, 670 meters SSE Fish Surface Water
62	GWPP	Location: Type:	Onsite Monitoring Well MW-18A near SE corner of the turbine building Ground Water
63	GWPP	Location: Type:	Onsite Monitoring Well MW-19A near S wall of the turbine building Ground Water – discontinued (well sealed)

Table 5-1
ENVIRONMENTAL SAMPLE STATIONS

Station Number	Program	Station Location and Sample Type
64	GWPP	Location: Onsite Monitoring Well MW-20A near S wall of the turbine building Type: Ground Water
65	GWPP	Location: Onsite Monitoring Well MW-21A near S wall of the turbine building Type: Ground Water
66	GWPP	Location: Onsite Monitoring Well MW-22A S turbine building wall and W of MW-08A Type: Ground Water
67	GWPP	Location: Onsite Monitoring Well MW-23A S of MW-22A and NW of MW-14A Type: Ground Water
68	GWPP	Location: Extraction Well EW-01A SE of MW-23A and W of MW-14A Type: Ground Water
69	GWPP	Location: Extraction Well EW-02A SSE of MW-12A in the Owner Controlled Area Type: Ground Water
70	GWPP	Location: Extraction Well EW-03A SSE between MW-24A and MW-26A. Formerly D-166. Type: Ground water
71		Not Used
72	REMP	Location: Farm, 3,200 meters SSW Type: Drinking Water
73	GWPP	Location: 2MH216/1MH117 electrical vault, West of the Offgas Stack Type: Surface Water
74	GWPP	Location: 2MH215/1MH116 electrical vault, E. of RB RR, W. of Condensate Storage Tanks Type: Surface Water
75	GWPP	Location: 2MH207 electrical vault, East of South End of Pump House Type: Surface Water
76	REMP	Location: Farm, 2,888 meters East Northeast Type: Milk – discontinued
77	REMP	Location: Farm on Palo Marsh Road, 2288 meters SW Type: Vegetation – discontinued
78		Not Used
79	GWPP	Location: On-site monitoring well MW-33A SSE in Owner Controlled Area Type: Ground Water
80	GWPP	Location: On-site monitoring well MW-34A SSE in Owner Controlled Area Type: Ground Water
81	GWPP	Location: On-site monitoring well MW-35A SSE in Owner Controlled Area Type: Ground Water Precipitation – discontinued
82	REMP	Location: On-site, 660 meters SSE Type: TLD
83	REMP	Location: On-site, 620 meters SSE Type: TLD
84	REMP	Location: On-site, 610 meters S Type: TLD
85	REMP	Location: On-site, 660 meters SSW Type: TLD
86	REMP	Location: On-site, 850 meters SW Type: TLD
87	GWPP	Location: MH218 electrical vault, East of the Turbine Building Type: Surface Water
88	GWPP	Location: MH219 electrical vault, North of Air Compressor Building Type: Surface Water

Table 5-1
ENVIRONMENTAL SAMPLE STATIONS

Station Number	Program	Station Location and Sample Type	
89	GWPP	Location: Type:	MH221 electrical vault, Between Air Compressor Building and H2 Skid Surface Water
90	GWPP	Location: Type:	MH222 electrical vault, North of "B" Well Surface Water
91	REMP	Location: Type:	On-site, 1,090 meters NNW TLD
92	GWPP	Location: Type:	MH202 electrical vault, West of XR1/XR2 Transformers Surface Water
93	GWPP	Location: Type:	MH213 electrical vault, East of the Pump House Surface Water
94	GWPP	Location: Type:	MH214 electrical vault, South of "B" Cooling Tower Surface Water
95	GWPP	Location: Type:	MH217 electrical vault, Southwest of Intake Building Surface Water
96	REMP	Location: Type:	Farm, 11,400 meters SSW Vegetation – discontinued
97	GWPP	Location: Type:	MH102 electrical vault, "A" Well Road North of the Substation Surface water
98	GWPP	Location: Type:	MH103 electrical vault, "A" Well Road West of the Substation Surface Water
99	REMP	Location: Type:	Pleasant Creek Lake, 3,880 meters WNW Surface Water and deer meat sample – discontinued
100	GWPP	Location: Type:	MH206 electrical vault, West of Pump House Surface Water
101	GWPP	Location: Type:	1MH109 electrical vault, East of the North End of the Pump House Surface Water
102	GWPP	Location: Type:	2MH208/1MH110 electrical vault, Road to Intake Building Surface Water
103	GWPP	Location: Type:	MH101 electrical vault, North of PAB, East of Security Fence Surface Water
104	GWPP	Location: Type:	MH115 electrical vault, South of "A" Cooling Tower Surface Water
105	GWPP	Location: Type:	MH114 electrical vault, South of "A" Cooling Tower Surface Water
106	GWPP	Location: Type:	MH201 electrical vault, South of Turbine Building Surface Water
107	GWPP	Location: Type:	DAEC Sewage Plant Effluent Surface Water
107 a	REMP	Location: Type:	On-Site: North Drainage Ditch Bottom Sediments – discontinued
108	REMP	Location: Type:	4,590 meters SSW Vegetation – discontinued
109	REMP	Location: Type:	5,890 meters SW Vegetation – discontinued
110	REMP	Location: Type:	12,700 meters SW Milk – discontinued
111	GWPP	Location: Type:	Onsite Monitoring Wells MW01A and MW01B (SSE) Ground Water – discontinued Precipitation – discontinued

Table 5-1
ENVIRONMENTAL SAMPLE STATIONS

Station Number	Program	Station Location and Sample Type	
112	GWPP	Location: Type:	Onsite Monitoring Wells MW02A and MW02B (ESE) Ground Water – discontinued Precipitation – discontinued
113	GWPP	Location: Type:	Onsite Monitoring Wells MW03A and MW03B (ESE) Ground Water – discontinued
114	GWPP	Location: Type:	Onsite Monitoring Wells MW04A and MW04B (South) Ground Water – discontinued Precipitation - discontinued
115	GWPP	Location: Type:	Onsite Monitoring Wells MW05A and MW05B (SSW) Ground Water – discontinued Precipitation – discontinued
116	GWPP	Location: Type:	Onsite Monitoring Wells MW06A and MW06B (NE) Ground Water – discontinued Precipitation - discontinued
117		Not Used	
118	REMP	Location: Type:	2,230 meters NW Vegetation – discontinued
119	GWPP	Location: Type:	2MH209/1MH111 electrical vault, Road to Intake Building Surface Water
120	GWPP	Location: Type:	2MH210/1MH112 electrical vault, Road to Intake Building Surface Water
121	GWPP	Location: Type:	2MH211/1MH113 electrical vault, Road to Intake Building Surface Water
122	GWPP	Location: Type:	Onsite Sluice Pond, in OCA Surface Water
123	GWPP	Location: Type:	Onsite South Drainage Ditch Surface Water
124	GWPP	Location: Type:	Onsite North Drainage Ditch Surface Water
125	GWPP	Location: Type:	Onsite South Storm Drain Outfall Surface Water
127	GWPP	Location: Type:	Protected Area, North of CST Pit, Monitoring Wells MW07A and MW07B Ground Water Precipitation – discontinued
128	GWPP	Location: Type:	Protected Area, CAD Shack, Monitoring Wells MW08A and MW08B Ground Water Precipitation – discontinued
129	GWPP	Location: Type:	Protected Area, SE Corner of CST Pit, Monitoring Wells MW09A and MW09B Ground Water
130	GWPP	Location: Type:	Protected Area, SW Corner of CST Pit, Monitoring Wells MW10A and MW10B Ground Water – discontinued
131	GWPP	Location: Type:	On-site Monitoring Wells MW11A and 11B SE in Owner Controlled Area Ground Water
132	GWPP	Location: Type:	On-site Monitoring Wells MW12A and MW12B SE in Owner Controlled Area Ground Water
133	GWPP	Location: Type:	On-site Monitoring Wells MW13A and MW13B SW in Owner Controlled Area Ground Water – discontinued
134	GWPP	Location: Type:	On-site Monitoring Wells MW14A and MW14B SSE in Protected Area Ground Water
135	GWPP	Location: Type:	On-site Monitoring Wells MW15A and MW15B SE in Owner Controlled Area Ground Water – discontinued

Table 5-1
ENVIRONMENTAL SAMPLE STATIONS

Station Number	Program	Station Location and Sample Type	
136	GWPP	Location: Type:	On-site Monitoring Wells MW16A and MW16B ENE in Protected Area Ground Water – discontinued
137	GWPP	Location: Type:	On-site Monitoring Well MW17C NE in Protected Area Ground Water – discontinued
138	REMP	Location: Type:	21,600 meters WSW Milk Control – discontinued Vegetation Control – discontinued
161	REMP	Location: Type:	On-site, ISFSI, East Fence Line TLD
162	REMP	Location: Type:	On-site, ISFSI, South Fence Line TLD
163	REMP	Location: Type:	On-site, ISFSI, West Fence Line TLD
164	REMP	Location: Type:	On-site, ISFSI, North Fence Line TLD
165	GWPP	Location: Type:	On-site Monitoring Well MW24A ESE in Owner Controlled Area Ground Water
166	GWPP	Location: Type:	On-site Monitoring Well MW25A ESE in Owner Controlled Area Ground Water – discontinued
167	GWPP	Location: Type:	On-site Monitoring Well MW26A ESE in Owner Controlled Area Ground Water
168	GWPP	Location: Type:	On-site Monitoring Wells MW27A and MW27B SE in Owner Controlled Area Ground Water – discontinued
169	GWPP	Location: Type:	On-site Monitoring Wells MW28A and MW28B SE in Owner Controlled Area Ground Water
170	GWPP	Location: Type:	On-site Monitoring Wells MW29A and MW29B SE in Owner Controlled Area Ground Water
171	GWPP	Location: Type:	On-site Monitoring Wells MW30A and MW30B SE in Owner Controlled Area Ground Water
172	GWPP	Location: Type:	On-site Monitoring Wells MW31A and MW31B SE in Owner Controlled Area Ground Water – discontinued
173	GWPP	Location: Type:	On-site Monitoring Wells MW32A and MW32B SE in Owner Controlled Area Ground Water

6.0 RADIOLOGICAL EFFLUENT CONTROLS AND SURVEILLANCE REQUIREMENTS

O 6.0.1 Use and Application

O 6.0.1.1 Definitions

NOTE
The defined terms of this section appear in capitalized type and are applicable throughout these OLCOs and Bases

Term	Definition
ACTIONS	ACTIONS shall be that part of an DODAM Specification that prescribes Required Actions to be taken under designated Conditions within specified Completion Times.
CHANNEL CALIBRATION	A CHANNEL CALIBRATION shall be the adjustment, as necessary, of the channel output such that it responds within the necessary range and accuracy to known values of the parameter that the channel monitors. The CHANNEL CALIBRATION shall encompass the entire channel, including the required sensor, alarm, display, and trip functions, and shall include the CHANNEL FUNCTIONAL TEST. The CHANNEL CALIBRATION may be performed by means of any series of sequential, overlapping, or total channel steps so that the entire channel is calibrated.
CHANNEL	A channel is an arrangement of a sensor and associated components used to evaluate plant variables and produce discrete outputs used in logic. A channel terminates and loses its identity where individual channel outputs are combined in logic.
CHANNEL CHECK	A CHANNEL CHECK shall be the qualitative assessment, by observation, of channel behavior during operation. This determination shall include, where possible, comparison of the channel indication and status to other indications or status derived from independent instrument channels measuring the same parameter.
CHANNEL FUNCTIONAL TEST	A CHANNEL FUNCTIONAL TEST shall be the injection of a simulated or actual signal into the channel as close to the sensor as practicable to verify FUNCTIONALITY, including required alarm, interlock, display, and trip functions, and channel failure trips. The CHANNEL FUNCTIONAL TEST may be performed by means of any series of sequential, overlapping, or total channel steps.

(continued)

GASEOUS RADWASTE TREATMENT SYSTEM	A GASEOUS RADWASTE TREATMENT SYSTEM is any system designed and installed to reduce radioactive gaseous effluents by collecting primary coolant system offgases from the primary system and providing delay or holdup for the purpose of reducing radioactivity prior to release to the environment.
MEMBER(S) OF THE PUBLIC	MEMBER(S) OF THE PUBLIC are persons who are not occupationally associated with the Company and who do not normally frequent the DAEC site. The category does not include contractors, contractor employees, vendors, or persons who enter the site to make deliveries or to service equipment.
FUNCTIONAL-FUNCTIONALITY	An SSC is functional or has functionality when it is capable of performing its function(s), as set forth in the CLB.
SITE BOUNDARY	The SITE BOUNDARY is that line beyond which the land is neither owned, nor leased, nor otherwise controlled by the Company. UFSAR Figure 1.2-1 identifies the DAEC SITE BOUNDARY.
SOURCE CHECK	A SOURCE CHECK is the assessment of channel response when the channel sensor is exposed to a source of radiation.
UNRESTRICTED AREA	The UNRESTRICTED AREA is that land (offsite) beyond the SITE BOUNDARY.

O 6.0.1 Use and Application

O 6.0.1.2 Logical Connectors

PURPOSE	The purpose of this section is to explain the meaning of logical connectors. Logical connectors are used in Defueled Offsite Dose Assessment Manual (DODAM) to discriminate between, and yet connect, discrete Conditions, Required Actions, Completion Times, Surveillances, and Frequencies. The only logical connectors that appear in the DODAM are **AND;** and **OR**. The physical arrangement of these connectors constitutes logical conventions with specific meanings.
BACKGROUND	Several levels of logic may be used to state Required Actions. These levels are identified by the placement (or nesting) of the logical connectors and by the number assigned to each Required Action. The first level of logic is identified by the first digit of the number assigned to a Required Action and the placement of the logical connector in the first level of nesting (i.e., left justified with the number of the Required Action). The successive levels of logic are identified by additional digits of the Required Action number and by successive indentions of the logical connectors. When logical connectors are used to state a Condition, Completion Time, Surveillance, or Frequency, only the first level of logic is used, and the logical connector is left justified with the statement of the Condition, Completion Time, Surveillance, or Frequency.

O 6.0.1 Use and Application

O 6.0.1.3 Completion Times

PURPOSE	The purpose of this section is to establish the Completion Time convention and to provide guidance for its use.
BACKGROUND	OLCOs specify minimum requirements for ensuring safe operation of the unit. The ACTIONS associated with an OLCO state Conditions that typically describe the ways in which the requirements of the OLCO can fail to be met. Specified with each stated Condition are Required Action(s) and Completion Times(s).
DESCRIPTION	The Completion Time is the amount of time allowed for completing a Required Action. It is referenced to the time of discovery of a situation (e.g., non-functional equipment or variable not within limits) that requires entering an ACTIONS Condition unless otherwise specified, providing the unit is in a specified condition stated in the Applicability of the OLCO. Required Actions must be completed prior to the expiration of the specified Completion Time. An ACTIONS Condition remains in effect and the Required Actions apply until the Condition no longer exists or the unit is not within the OLCO Applicability. If situations are discovered that require entry into more than one Condition at a time within a single OLCO (multiple Conditions), the Required Actions for each Condition must be performed within the associated Completion Time. When in multiple Conditions, separate Completion Times are tracked for each Condition starting from the time of discovery of the situation that required entry into the Condition.
	When "Immediately" is used as a Completion Time, the Required Action should be pursued without delay and in a controlled manner. The DODAM implements Completion Times in precisely the same manner they are applied in the Technical Requirements Manual (TRM) and the Technical Specifications (TS).

O 6.0.1 Use and Application

O 6.0.1.4 Frequency

PURPOSE	The purpose of this section is to define the proper use and application of Frequency Requirements.
DESCRIPTION	Each DODAM Surveillance Requirement (OSR) has a specified Frequency in which the Surveillance must be met in order to meet the associated OLCO. An understanding of the correct application of the specified Frequency is necessary for compliance with the OSR. The "specified Frequency" is referred to throughout this section and each of the Specifications of Section 6.0.3.0, Surveillance Requirement (SR) Applicability. The "specified Frequency" consists of the requirements of the Frequency column of each OSR, as well as certain Notes in the Surveillance column that modify performance requirements. Sometimes special situations dictate when the requirements of a Surveillance are to be met. They are "otherwise stated" conditions allowed by OSR 7.0.3.0.1. They may be stated as clarifying Notes in the Surveillance, as part of the Surveillance or both. Situations where a Surveillance could be required (i.e., its Frequency could expire), but where it is not possible or not desired that it be performed until sometime after the associated OLCO is within its Applicability, represent potential OSR 7.0.3.0.4 conflicts. To avoid these conflicts, the OSR (i.e., the Surveillance or the Frequency) is stated such that it is only "required" when it can be and should be performed. With an OSR satisfied, OSR 6.0.3.0.4 imposes no restriction. The use of "met" or "performed" in these instances conveys specific meanings. A Surveillance is "met" only when the acceptance criteria are satisfied. Known failure of the requirements of a Surveillance, even without a Surveillance specifically being "performed," constitutes a Surveillance not "met." "Performance" refers only to the requirement to specifically determine the ability to meet the acceptance criteria. OSR 7.0.3.04 restrictions would not apply if both the following conditions are satisfied: a. The surveillance is not required to be performed; and b. The Surveillance is not required to be met or, even if required to be met, is not known to be failed.

6.0.3 Limiting Conditions for Operation (OLCO) Applicability

OLCO 6.0.3.0.1	OLCOs shall be met during the specified conditions in the Applicability, except as provided in OLCO 6.0.3.0.2.
OLCO 6.0.3.0.2	Upon discovery of a failure to meet an OLCO, the Required Actions of the associated Conditions shall be met, except as provided in OLCO 6.0.3.0.5. If the OLCO is met or is no longer applicable prior to expiration of the specified Completion Time(s), completion of the Required Action(s) is not required, unless otherwise stated.
OLCO 6.0.3.0.3	Not Used.
OLCO 6.0.3.0.4	When an OLCO is not met, entry into a specified condition in the Applicability shall not be made except when the associated ACTIONS to be entered permit continued operation in the specified condition in the Applicability for an unlimited period of time.
OLCO 6.0.3.0.5	Equipment removed from service or declared non-functional to comply with ACTIONS may be returned to service under administrative control solely to perform testing required to demonstrate its FUNCTIONALITY or the FUNCTIONALITY of other equipment. This is an exception to OLCO 6.0.3.0.2 for the system returned to service under administrative control to perform the testing required to demonstrate FUNCTIONALITY.

O 7.0.3 Surveillance Requirement (OSR) Applicability

OSR 7.0.3.0.1	OSRs shall be met during the specified conditions in the Applicability for individual OLCOS, unless otherwise stated in the OSR. Failure to meet a Surveillance, whether such failure is experienced during the performance of the Surveillance or between performances of the Surveillance, shall be failure to meet the OLCO. Failure to perform a Surveillance within the specified Frequency shall be failure to meet the OLCO except as provided in OSR 7.0.3.0.3. Surveillances do not have to be performed on non-functional equipment or variables outside specified limits.
OSR 7.0.3.0.2	The specified Frequency for each OSR is met if the Surveillance is performed within 1.25 times the interval specified in the Frequency, as measured from the previous performance or as measured from the time a specified condition of the Frequency is met. For Frequencies specified as "once," the above interval extension does not apply. If a Completion Time requires periodic performance on a "once per . . ." basis, the above Frequency extension applies to each performance after the initial performance. Exceptions to this OSR are stated in the individual OLCOs.
OSR 7.0.3.0.3	If it is discovered that a Surveillance was not performed within its specified Frequency, then compliance with the requirement to declare the OLCO not met may be delayed, from the time of discovery, up to 24 hours or up to the limit of the specified Frequency, whichever is more. This delay period is permitted to allow performance of the Surveillance. If the Surveillance is not performed within the delay period, the OLCO must immediately be declared not met, and the applicable Condition(s) must be entered. When the Surveillance is performed within the delay period and the Surveillance is not met, the OLCO must immediately be declared not met, and the applicable Condition(s) must be entered.
OSR 7.0.3.0.4	Entry into a MODE or other specified condition in the Applicability of an OLCO shall not be made unless the OLCO's Surveillances have been met within their specified Frequency.

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O[CC4] 6.1	Radioactive Liquid Effluent Controls and Surveillance Requirements
O 6.1.1	Radioactive Liquid Instrumentation
OLCO 6.1.1	The Radioactive Liquid Instrumentation for each function in Table 6.1-1 shall be FUNCTIONAL with their alarm and trip setpoints set to ensure that the limits of Section 6.1.2 are not exceeded.
APPLICABILITY:	As shown in Table 6.1-1

ACTIONS

CONDITION		REQUIRED ACTION		COMPLETION TIME
A.	One or more channels non-functional	A.1	Enter the condition(s) referenced in Table 6.1-1 for the channel(s).	Immediately
		AND		
		A.2	Restore required channel(s) to FUNCTIONAL status.	30 days
B.	As required by Required Action A.1 and referenced in Table 6.1-1	B.1	Obtain at least two samples and analyze in accordance with Section 6.1.2 and Table 7.1-2	Prior to release
		AND		
		B.1	Independently verify release rate calculations and discharge valving by technically qualified Facility Staff member.	Prior to release
C.	As required by Required Action A.1 and referenced in Table 6.1-1	C.1	Obtain sample and analyze for either gross beta/gamma or gamma radioactivity.	Once per 24 Hours
D.	Less than the minimum required channels FUNCTIONAL.	D.1	Estimate flow rate at least once per batch.	Immediately
E.	Required Action and associated Completion Time of Condition A not met.	E.1	Explain in next Annual Radioactive Material Release Report why the instrument was not made functional in a timely manner.	Next submittal of the Annual Radioactive Material Release Report

SURVEILLANCE REQUIREMENTS

------------------------------NOTE------------------------------
Instrumentation shall be FUNCTIONAL and in service except that channels out of service are permitted for the purpose of required tests, checks, and calibrations without declaring the channel to be non-functional.

SURVEILLANCE		FREQUENCY
OSR 7.1.1.1	Perform CHANNEL CHECK	24 Hours
OSR 7.1.1.2	Perform SOURCE CHECK	Prior to release
OSR 7.1.1.3	Perform SOURCE CHECK	30 Days
OSR 7.1.1.4	Perform CHANNEL FUNCTIONAL TEST	92 Days
OSR 7.1.1.5	Perform CHANNEL CALIBRATION	18 Months
OSR 7.1.1.6	Perform Monitor Setpoint Calculation.	18 Months
OSR 7.1.1.7	Perform CHANNEL FUNCTIONAL TEST	92 Days
OSR 7.1.1.8	Perform CHANNEL CALIBRATION	18 Months
OSR 7.1.1.9	Perform Monitor Setpoint Calculation.	Prior to release

TABLE 6.1-1 / 7.1-1
RADIOACTIVE LIQUID EFFLUENT MONITORING INSTRUMENTATION

Instrument	Applicability	Minimum Channels Functional	Conditions Referenced from Required Action A.1	Surveillance Requirements
1. Gross Radioactive Monitors Providing Automatic Termination of Release				
a. Liquid Radwaste Effluent Line	During Releases	1	B	OSR 7.1.1.1 [a] OSR 7.1.1.2 [b] OSR 7.1.1.7 [d] [e] OSR 7.1.1.8 [c] OSR 7.1.1.9
2. Gross Radioactive Monitors Not Providing Automatic Termination of Release				
a. RHR Service Water System	During Releases	1	C	OSR 7.1.1.1 [a] OSR 7.1.1.3 OSR 7.1.1.4 [e] OSR 7.1.1.5 [c] OSR 7.1.1.6
b. General Service Water System	During Releases	1		
c. RHRSW/ESW Rupture Disc Effluent Line	During Releases	1		
3. Flow Rate Measurement Devices				
a. Liquid Radwaste Effluent Line	During Releases	1	D	OSR 7.1.1.1 [a] [f] OSR 7.1.1.7 OSR 7.1.1.8
b. Liquid Radwaste Dilution Line	During Releases	1		

TABLE NOTATIONS

(a) During releases via this pathway.

(b) On any day on which a release is made, a SOURCE CHECK shall be made at least once, prior to the first release.

(c) The CHANNEL CALIBRATION shall include the use of a known radioactive source (traceable to the NIST radiation measurement system or acceptable non-NIST standards) positioned in a reproducible geometry with respect to the sensor and emitting beta or gamma radiation in the range measured by the channel. CHANNEL CALIBRATION may normally be done during refueling outages.

(d) The CHANNEL FUNCTIONAL TEST shall also demonstrate that automatic isolation of this pathway if the following condition exists:

1. Instrument indicates measured levels above the alarm/trip setpoint.

(e) The CHANNEL FUNCTIONAL TEST shall also demonstrate that control room alarm annunciation occurs if any of the following conditions exist:

1. Instrument indicates measured levels above the alarm/trip setpoint.
2. Circuit failure.
3. Instrument indicates a downscale failure.
4. Instrument controls not set in operate mode.

(f) CHANNEL CHECK shall consist of verifying indication of flow during periods of release. CHANNEL CHECK shall be made at least once daily on any day on which continuous, periodic, or batch releases are made.

O 6.1 **Radioactive Liquid Effluent Controls and Surveillance Requirements**

O 6.1.2 Liquid Effluent Concentration

OLCO 6.1.2 The concentration of radioactive material in liquid effluent released from the site to the UNRESTRICTED AREA shall not exceed ten times the concentrations specified in 10 CFR 20, Appendix B, Table 2, Column 2 to 10 CFR 20.1001 - 20.2402.

APPLICABILITY: At all times.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Concentration of radioactive material released from site to UNRESTRICTED AREAS not within the limit.	A.1 Restore concentration to within limits.	Immediately

SURVEILLANCE REQUIREMENTS

----------NOTE----------
Refer to DODAM Table 7.1-2 for details concerning Sampling and Analysis
----

	SURVEILLANCE	FREQUENCY
OSR 7.1.2.1	----------NOTE---------- Portions of each sample are "split out" and added to the Monthly and Quarterly Composite sample. -------------------- Sample the Batch Release.	PRIOR TO EACH RELEASE
OSR 7.1.2.2	The results of pre-release analyses shall be used with the calculation methods in the DODAM to establish trip setpoints for batch releases to assure that the concentration at the UNRESTRICTED AREA boundary does not exceed the limit in OLCO 6.1.2.	PRIOR TO EACH RELEASE
OSR 7.1.2.3	----------NOTE---------- Portions of each sample are split out and added to the Monthly and Quarterly Composite sample. -------------------- ----------NOTE---------- In the event of a positive identification of reactor by-product radioactivity in the service water, additional sampling and analysis shall be performed as specified in Table 7.1-2, Table Items B.2 through B.5. -------------------- Sample the Continuous Release during releases to the environment via this pathway. AND Analyze the sample from the release for Gross Beta/Gamma. OR Analyze the sample from the release for Principal Gamma Emitters and I-131.	7 Days

(continued)

OSR 7.1.2.4	Analyze the sample from the release for Principal Gamma Emitters and I-131.	PRIOR TO EACH RELEASE
OSR 7.1.2.5	Analyze the "Monthly" Composite sample for Tritium.	30 Days
OSR 7.1.2.6	------------------------NOTE------------------------ For Service Water, this OSR is only required following a positive identification of reactor by-product radioactivity in the sample as determined during OSR 7.1.2.3 or OSR 7.1.2.5. -- ------------------------NOTE------------------------ For "clean" water systems, this OSR is only required following a positive identification of reactor by-product radioactivity in the sample as determined during OSR 7.1.2.4. -- ------------------------NOTE------------------------ For GWPP Mitigation system, this OSR is only required following a positive identification of a radioactive reactor by-product in the sample as determined during OSR 7.1.2.3. -- Analyze the Quarterly Composite sample for Sr-89, Sr-90, Fe-55	92 Days
OSR 7.1.2.7	Analyze the "Monthly" Composite sample for Gross Alpha.	30 Days

TABLE 7.1-2

RADIOACTIVE LIQUID WASTE SAMPLING AND ANALYSIS PROGRAM

Liquid Release Type	Surveillance Requirements	Table item/note	Type of Activity Analysis	Lower Limit of Detection (LLD)[(a)] (μCi/mL)
A. Radwaste Tanks Batch Release	OSR 7.1.2.1 OSR 7.1.2.4	**A.1:** Each Batch	Principal Gamma Emitters	5×10^{-7}
	OSR 7.1.2.2		I-131[c]	1×10^{-6}
	OSR 7.1.2.1 OSR 7.1.2.5	**A.2:** (b), (d)	H-3	1×10^{-5}
	OSR 7.1.2.7		Gross alpha	1×10^{-7}
	OSR 7.1.2.1 OSR 7.1.2.6	**A.3:** (b), (d)	Sr-89, Sr-90	5×10^{-8}
			Fe-55	1×10^{-6}
B. Continuous Service Water Release	OSR 7.1.2.3	**B.1:** (d)	Gross beta/gamma	1×10^{-7}
Sample Points:	OSR 7.1.2.3	**B.2:** (d)	Principal Gamma Emitters	5×10^{-7}
• General Services Water • RHR Service Water System A • RHR Service Water System B			I-131	1×10^{-6}
	OSR 7.1.2.5	**B.4:** (b), (d)	H-3	1×10^{-5}
	OSR 7.1.2.7		Gross alpha	1×10^{-7}
	OSR 7.1.2.6	**B.5:** (b), (d)	Sr-89, Sr-90	5×10^{-8}
			Fe-55	1×10^{-6}
C. "Clean" Systems Batch Release	OSR 7.1.2.1 OSR 7.1.2.4	**C.1:**	Principal Gamma Emitters[(f)]	5×10^{-7}
Sample Points:			I-131[(f)]	1×10^{-6}
	OSR 7.1.2.5	**C.2:** (b) (d) (e)	H-3[(f)]	1×10^{-5}
• CST Pit • Transformer Pit • Neutralizing Tank 1T022 • FRAC tanks • Temporary groundwater	OSR 7.1.2.6	**C.3:** (b) (d)	Sr-89, Sr-90	5×10^{-8}
			Fe-55	1×10^{-6}
D. GWPP Mitigation System	OSR 7.1.2.3	D.1:	Principal Gamma Emitters	5.0×10^{-7}
Continuous Release			I-131[(c)]	1.0×10^{-6}
	OSR 7.1.2.5	D.2: (b)(d)(e)	H-3[(f)]	1.0×10^{-5}
	OSR 7.1.2.6	D.3: (b)(d)	Sr-89, Sr-90	5.0×10^{-8}
			Fe-55	1.0×10^{-6}

TABLE NOTATIONS

[a]The LLD is defined, for purposes of these controls, as the smallest concentration of radioactive material in a sample that will yield a net count, above system background, that will be detected with 95% probability with only 5% probability of falsely concluding that a blank observation represents a "real" signal.

For a particular measurement, which may include radiochemical separation:

$$LLD = \frac{4.66\ S_b}{E\,x\ V\,x\ 2.22E6\ x\ e^{-\lambda\Delta t}}$$

where:

LLD is the "a priori" lower limit of detection as defined above (microcuries per unit mass or volume)

and where:

S_b is the standard deviation of the background counting rate or of the counting rate of a blank sample as appropriate (counts per minute)
E is the counting efficiency (counts per disintegration)
V is the sample size (units of mass or volume)
2.22E6 is the number of disintegrations per minute per microcurie,
Y is the fractional radiochemical yield, when applicable,
λ is the radioactive decay constant for the particular radionuclide (sec^{-1}), and
Δ t for effluents is the elapsed time between the midpoint of sample collection and the time of counting (sec^{-1}).

Alternatively, exp may be replaced by

$$\frac{\lambda t_1 e^{-\lambda t_2}}{1 - e^{-\lambda t_1}} \bullet \cdot\ e^{-\lambda t_2}$$

Where:

t_1 is the total sampling time or sample compositing time
t_2 is the elapsed time between the end of sample collection and the time of counting.

It should be recognized that the LLD is defined as an a priori (before the fact) limit representing the capability of a measurement system and not as an a posteriori (after the fact) limit for a particular measurement.

Analyses shall be performed in such a manner that the stated LLDs will be achieved under routine conditions with typical values of E, V, Y, and Dt for the radionuclides Mn-54, Fe-59, Co-58, Co-60, Zn-65, Mo-99, Cs-134, Cs-137, Ce-141, and Ce-144. Occasionally background fluctuations, unavoidably small sample sizes, interfering radionuclides, or other uncontrollable circumstances may render these LLDs unachievable.

When calculating the LLD for a radionuclide determined by gamma ray spectrometry, the background may include the typical contributions of other radionuclides normally present in the samples. The background count rate of a Ge(Li) detector is determined from background counts that are determined to be within the full width of the specific energy band used for the quantitative analysis for that radionuclide.

The principal gamma emitters for which the LLD specification will apply are exclusively the following radionuclides: Mn-54, Fe-59, Co-58, Co-60, Zn-65, Mo-99, Cs-134, Cs-137 and Ce-141. Ce-144 shall be measured, but with an LLD of 5E-6. This list does not mean that only these nuclides are to be detected and reported. Other peaks which are measurable and identifiable, together with the above nuclides, shall also be identified and reported. Nuclides which are below the LLD for the analyses should not be reported as being present at the LLD level. When unusual circumstances result in LLDs higher than required, the reasons shall be documented in the Annual Radioactive Material Release Report.

(b) A composite sample is one in which the quantity of liquid sampled is proportional to the quantity of liquid waste discharged and in which the method of sampling employed results in a specimen which is representative of the liquids released.

(c) In the event a gross β or γ analysis is performed in lieu of an isotopic analysis before a batch is discharged, a sample shall be analyzed for principal gamma emitters afterward.

(d) Analysis may be performed after release.

(e) Analysis at a frequency of less than 30 days is allowed.

(f) If liquids from these systems are released from the site via a pathway that is NOT directly to the Cedar River, the required LLDs of Table 6.3.2 are applicable.

O 6.1 Radioactive Liquid Effluent Controls and Surveillance Requirements

O 6.1.3 Dose Due to Liquid Radioactive Effluents

OLCO 6.1.3 The dose or dose commitment to a MEMBER OF THE PUBLIC from radioactive materials in liquid effluents released to the UNRESTRICTED AREA shall not exceed:

1.5 mrem to the total body during any calendar quarter,
5.0 mrem to any organ during any calendar quarter,
3.0 mrem to the total body during any calendar year, or
10.0 mrem to any organ during any calendar year.

APPLICABILITY: At All Times

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Calculated dose from the release of radioactive materials in liquid effluents exceeds limits.	A.1 Prepare and submit a Special Report to the Commission which identifies the cause(s) for exceeding the limit and defines the action to be taken.	30 days

SURVEILLANCE REQUIREMENTS

	SURVEILLANCE	FREQUENCY
OSR 7.1.3.1	In any quarter in which radioactive liquid effluent is discharged, an assessment shall be performed in accordance with the DODAM in order to verify that the cumulative dose commitment does not exceed the limits in OLCO 6.1.3.	30 Days

O 6.1 Radioactive Liquid Effluent Controls and Surveillance Requirements

O 6.1.4 Liquid Waste Treatment

OLCO 6.1.4 Appropriate liquid radwaste equipment shall be used to treat any untreated batch of liquid waste prior to discharge when a pre-released analysis indicates a radioactivity concentration (exclusive of tritium and dissolved gases) of 0.01 μCi/ml or higher.

APPLICABILITY: At all times.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Radioactive liquid waste being discharge without treatment and in excess of limits.	A.1 Prepare and submit a Special Report to the Commission which includes identification of non-functional equipment or subsystems and the reason, actions taken to restore the non-functional equipment, and description of action(s) taken to prevent recurrence.	30 days

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
OSR 7.1.4.1	Each radioactive liquid waste batch shall be sampled and analyzed in accordance with Section 7.1.2 and Table 7.1-2.	PRIOR TO EACH RELEASE

O 6.1 Radioactive Liquid Effluent Controls and Surveillance Requirements

O 6.1.5 Liquid Holdup Tanks

OLCO 6.1.5 The quantity of radioactive material contained in the unprotected outdoor tanks shall be limited to less than or equal to 50 Curies, excluding tritium and dissolved or entrained noble gases.

APPLICABILITY: At all times.

ACTIONS

--NOTE--

Tanks included in this specification are those outdoor tanks that are not surrounded by liners, dikes, or walls capable of holding the tanks' contents and that do not have tank overflows and surrounding area drains connected to the liquid radwaste treatment system. (The liquid radwaste storage tanks (1T-88 and 1T-269) located in the Low-Level Radwaste Processing and Storage Facility are considered unprotected outdoor tanks.)

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Quantity of radioactive material in the tanks exceeding the limit.	A.1 Suspend all additions of the radioactive material to the tanks.	Immediately
	<u>AND</u>	
	A.2 Reduce tank contents to within the limit.	48 hours
	<u>AND</u>	
	A.3 Describe in next Annual Radioactive Effluent Release Report the events leading to this condition.	1 year
	<u>AND</u>	
	A.4 Prepare and submit a Special Report to the Commission which identifies the cause(s) for exceeding the limit and defines the action to be taken. (10CFR20.2203)	30 Days

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
OSR 7.1.5.1	When radioactive materials are being added to a tank, the quantity of radioactive material contained in all tanks shall be determined to be within the 50Curie limit by analyzing a representative sample of the tanks' contents.	7 Days

O 6.2 Radioactive Gaseous Effluent Controls and Surveillance Requirements

O 6.2.1.1 Radioactive Gaseous Effluent Instrumentation

OLCO 6.2.1.1 The Radioactive Gaseous Effluent Instrumentation for each function in Table 6.2-1 shall be FUNCTIONAL

APPLICABILITY: During releases via this pathway.

ACTIONS

CONDITION	REQUIRED ACTION		COMPLETION TIME
A. One or more channels non-functional.	A.1	Enter the condition referenced in Table 6.2-1 for the channel.	Immediately
	AND		
	A.2	Restore required channel(s) to FUNCTIONAL status.	30 days
B. As required by Required Action A.1 and referenced in Table 6.2-1	B.1	Collect gaseous grab sample.	once per 8 hours
	AND		
	B.2	Analyze sample for radioactivity.	Within 24 hours of sample collection
C. As required by Required Action A.1 and referenced in Table 6.2-1	C.1	Establish preplanned alternate monitoring method.	8 hours
	OR		
	C.2.1	Collect gaseous grab sample.	once per 8 hours
	AND		
	C.2.2	Analyze sample for radioactivity.	Within 24 hours of sample collection

(continued)

ACTIONS (continued)

D. As required by Required Action A.1 and referenced in Table 6.2-1	D.1	Establish continuous sampling with auxiliary equipment.	8 hours
	AND		
	D.2	Collect sample(s)	Once per 7 days
	AND		
	D.3	Analyze sample(s) for radioactivity.	Within 24 hours of sample collection
E. As required by Required Action A.1 and referenced in Table 6.2-1	E.1	Estimate flow rate initially and whenever operation of a main exhaust fan combination is changed in the system.	8 Hours
F. As required by Required Action A.1 and referenced in Table 6.2-1	F.1	Collect gaseous grab sample.	once per 24 hours
	AND		
	F.2	Analyze sample for radioactivity.	Within 24 hours of sample collection
G. Minimum required instrumentation not returned to FUNCTIONAL status within 30 days.	G.1	Explain in next Annual Radioactive Material Release Report why the instrument was not made functional in a timely manner.	

SURVEILLANCE REQUIREMENTS

------------------------------NOTE------------------------------

Instrumentation shall be FUNCTIONAL and in service except that channels out of service are permitted for the purpose of required tests, checks and calibrations without declaring the channel to be non-functional.

SURVEILLANCE		FREQUENCY
OSR 7.2.1.1.1	Perform CHANNEL CHECK -------------NOTE------------- During releases via this pathway. ------------------------------	24 Hours
OSR 7.2.1.1.2	Perform CHANNEL CHECK	7 Days
OSR 7.2.1.1.3	Perform SOURCE CHECK	30 Days
OSR 7.2.1.1.4	Determine the monitor setpoints in accordance with the method described in the DODAM.	92 Days
OSR 7.2.1.1.5	Perform CHANNEL FUNCTIONAL TEST	92 Days
OSR 7.2.1.1.6	Determine the monitor setpoints in accordance with the method described in the DODAM.	18 Months
OSR 7.2.1.1.7	Perform CHANNEL CALIBRATION	18 Months

	TABLE 6.2-1		TABLE 7.2-1
NORMAL RANGE RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION			
Instrument	Minimum Channels Functional [a b]	Conditions Referenced from Required Action A.1	Surveillance Requirements
1.Offgas Stack Monitoring System			
a. Noble Gas Activity Monitor	1	C	OSR 7.2.1.1.1, OSR 7.2.1.1.3 OSR 7.2.1.1.4, OSR 7.2.1.1.5[(d)], OSR 7.2.1.1.7[(c)]
b. Iodine Sampler Cartridge	1	D	OSR 7.2.1.1.2
c. Particulate Sampler Filter	1	D	OSR 7.2.1.1.2
d. Effluent Flow Measuring Device	1	E	OSR 7.2.1.1.1, OSR 7.2.1.1.5, OSR 7.2.1.1.7
e. Sample Flow Measuring Device	1	E	OSR 7.2.1.1.1, OSR 7.2.1.1.5, OSR 7.2.1.1.7
2. Reactor Building Exhaust Vent Monitoring System			
a. Noble Gas Activity Monitor	1	B	OSR 7.2.1.1.1, OSR 7.2.1.1.3, OSR 7.2.1.1.5[(d)], OSR 7.2.1.1.6, OSR 7.2.1.1.7[(c)]
b. Iodine Sampler Cartridge	1	D	OSR 7.2.1.1.2
c. Particulate Sampler Filter	1	D	OSR 7.2.1.1.2
d. Effluent Flow Measuring Device	1	E	OSR 7.2.1.1.1, OSR 7.2.1.1.5, OSR 7.2.1.1.7
e. Sample Flow Measuring Device	1	E	OSR 7.2.1.1.1, OSR 7.2.1.1.5, OSR 7.2.1.1.7
3. Turbine Building Exhaust Vent Monitoring System			
a. Noble Gas Activity Monitor	1	B	OSR 7.2.1.1.1, OSR 7.2.1.1.3, OSR 7.2.1.1.5[(d)], OSR 7.2.1.1.6, OSR 7.2.1.1.7[(c)]
b. Iodine Sampler Cartridge	1	D	OSR 7.2.1.1.2
c. Particulate Sampler Filter	1	D	OSR 7.2.1.1.2
d. Effluent Flow Measuring Device	1	E	OSR 7.2.1.1.1, OSR 7.2.1.1.5, OSR 7.2.1.1.7
e. Sample Flow Measuring Device	1	E	OSR 7.2.1.1.1, OSR 7.2.1.1.5, OSR 7.2.1.1.7
4. Low-Level Radwaste Processing and Storage Facility Exhaust Vent Monitoring System			
a. Noble Gas Activity Monitor	1	F	OSR 7.2.1.1.1, OSR 7.2.1.1.3 OSR 7.2.1.1.5[(d)], OSR 7.2.1.1.6 OSR 7.2.1.1.7[(c)]
b. Iodine Sampler Cartridge	1	D	OSR 7.2.1.1.2
c. Particulate Sampler Filter	1	D	OSR 7.2.1.1.2
d. Effluent Flow Measuring Device	1	E	OSR 7.2.1.1.1, OSR 7.2.1.1.5, OSR 7.2.1.1.7
e. Sample Flow Measuring Device	1	E	OSR 7.2.1.1.1, OSR 7.2.1.1.5, OSR 7.2.1.1.7

continued

TABLE 6.2-1 NOTATIONS

[a]During releases via this pathway.

[b] Instrumentation shall be FUNCTIONAL and in service except that channels out of service are permitted for the purpose of required tests, checks and calibrations without declaring the channel to be non-functional.

TABLE 7.2-1 NOTATIONS

[c] The CHANNEL CALIBRATION shall include the use of a known radioactive source (traceable to the NIST radiation measurement system or other acceptable non-NIST standards) positioned in a reproducible geometry with respect to the sensor and emitting beta and/or gamma radiation in the range measured by the channel in accord with established station calibration procedures. Alternatively, after the initial calibration, noble gas activity monitors maybe calibrated by laboratory analyzed gas samples collected and analyzed per Table 7.2-2, item A.

[d] The CHANNEL FUNCTIONAL TEST shall also demonstrate that control room alarm annunciation occurs if any of the following conditions exist:

1.0 Instrument indicates measured levels above the alarm setpoint.
2.0 Circuit failure.
3.0 Instrument indicates a downscale failure.
4.0 Deleted.

O 6.2 Radioactive Gaseous Effluent Controls and Surveillance Requirements

O 6.2.2 Gaseous Effluent Dose Rate

OLCO 6.2.2

The dose rate in the UNRESTRICTED AREA due to the release of gaseous effluents shall not exceed:

a. 500 mrem/year to the total body or 3000 mrem/year to skin due to radioactive noble gas; and
b. 1500 mrem/year to any organ due to I-131, I-133, H-3, and to radioactive particulates having half-lives of 8 days or more.

APPLICABILITY: During Releases.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Dose rate not within limits.	A.1 Reduce the release rate within limit.	Immediately

SURVEILLANCE REQUIRMENTS

SURVEILLANCE		FREQUENCY
OSR 7.2.2.1	Monitor for Radioactive Noble Gas gamma activity.	Continuously
OSR 7.2.2.2	Sample the Effluent Stream for I-131. <u>AND</u> Analyze for I-131 within 48 hours of sample pull.	Continuously 7 Days
OSR 7.2.2.3	---------------------------NOTE------------------------ Retain samples for Quarterly Composite analysis. (OSR 7.2.2.8) -- Sample the Effluent Stream for particulates. <u>AND</u> Analyze for principal gamma emitters from particulates within 48 hours of sample pull.	 Continuously 7 Days
OSR 7.2.2.4	Perform gas grab sample of the effluent stream and analyze for principal gamma emitters.	30 Days
OSR 7.2.2.6	Perform grab sample of the effluent stream and analyze for Tritium.	92 Days
OSR 7.2.2.8	Analyze Particulate Composite sample for Sr-89, Sr-90, Gross Alpha, Fe-55, Ni-63	92 Days

TABLE 7.2-2				
RADIOACTIVE GASEOUS WASTE SAMPLING AND ANALYSIS PROGRAM				
Gaseous Release Type	Surveillance Requirements	Frequency	Type of Activity Analysis	Lower Limit of Detection (LLD)[a]
A. Deleted				
B. Offgas Stack, Reactor Building Vent, Turbine Building Vent, and Low-Level Radwaste Processing Storage Facility Vent	OSR 7.2.2.2 [e]	Routine	Charcoal Sample I-131	1×10^{-12}
	OSR 7.2.2.3 [e]	Routine	Particulate Sample Principal Gamma Emitters (I-131, Others)	1×10^{-11}[d]
	OSR 7.2.2.4	Routine	Gas Grab Sample Principal Gamma Emitters	1×10^{-4}[d]
	OSR 7.2.2.6	Routine	Tritium Grab Sample (H-3)	1×10^{-6}
	OSR 7.2.2.8 [e]	Routine [g]	Composite Particulate Sample Sr-89, Sr-90 Gross Alpha Fe-55, Ni-63	1×10^{-11} 1×10^{-11} 1×10^{-11}
C. Offgas Stack, Reactor Building Vent, Turbine Building Vent, and Low-Level Radwaste Processing and Storage Facility	OSR 7.2.2.1	Continuous	Monitor Radioactive Noble Gas gamma activity	1×10^{-6}

TABLE NOTATIONS

[a] Units for the listed values are µCi/cc. See Table 7.1-2 for a definition of the lower limit of detection (LLD).

[d] The principal gamma emitters for which the LLD will apply are exclusively the following radionuclides: Kr-87, Kr-88, Xe-133, Xe-133m, Xe-135, and Xe-138 for gaseous emissions and Mn-54, Fe-59, Co-58, Co-60, Zn-65, Mo-99,Cs--134, Cs-137, Ce-141 and Ce-144 for particulate emissions. This list does not mean that only these nuclides are to be detected and reported. Other peaks which are measurable and identifiable, together with the above nuclides, shall also be identified and reported. Nuclides which are below the LLD may be reported as "less than" their respective LLD and should not be reported as being present at the LLD of the nuclide. Each measured radionuclide concentration is used in a required concentration or dose calculation only if it is detected at or above the LLD. When unusual circumstances persist more than 30 days and cause LLD higher than required, the reasons shall be documented in the Annual Radioactive Material Release Report.

[e] The ratio of the sample flow rate to the sampled stream flow rate shall be known for the time period covered by each dose or dose rate calculations made in accordance with Sections 6.2.2, 6.2.3, and 6.2.4.

(f) Sample media shall be changed at least once per seven days and the analysis completed within 48 hours after changing (or after removal from the sampler). When samples collected for 24 hours or less are analyzed, the corresponding LLD may be increased by a factor of 10.

(g) A quarterly composite sample shall include an equal fraction of each weekly particulate sample collected during the quarter. Quarterly samples are sent to an offsite laboratory to perform analysis.

O 6.2 Radioactive Gaseous Effluent Controls and Surveillance Requirements

O 6.2.3 Doses Due to Noble Gases

OLCO 6.2.3 The Air Dose in the UNRESTRICTED AREA due to noble gases released in gaseous effluents shall not exceed:

5.0 mrad from gamma radiation during any calendar quarter;
10.0 mrad from beta radiation during any calendar quarter;
10.0 mrad from gamma radiation during any calendar year; or
20.0 mrad from beta radiation during any calendar year.

APPLICABILITY: During Releases

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Calculated air dose not within limits.	A.1 Submit a Special Report to the NRC identifying the cause(s) for exceeding the limit and define the corrective actions taken.	30 days

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
OSR 7.2.3.1	An assessment shall be performed in accordance with the DODAM to verify that the cumulative air dose during the quarter and year due to noble gases does not exceed the limits in DODAM section 6.2.3.	30 Days

O 6.2 Radioactive Gaseous Effluent Controls and Surveillance Requirements

O 6.2.4 Doses Due to Iodine and Particulates in Air

OLCO 6.2.4 The dose to a MEMBER OF THE PUBLIC from Iodine-131, I-133, H-3, and from radionuclides in particulate form having half-lives greater than eight days in gaseous effluents released from the site to the UNRESTRICTED AREA shall not exceed:

7.5 mrem to any organ during any calendar quarter; or
15.0 mrem to any organ during any calendar year.

APPLICABILITY: During Releases.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Calculated dose not within limits.	A.1 Submit a Special Report to the NRC identifying the cause(s) for exceeding the limit and define the corrective actions taken.	30 days

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
OSR 7.2.4.1	An assessment shall be performed in accordance with the DODAM to verify that the cumulative dose commitment due to I-131, I-133, H-3, and radioactive particulates having half-lives greater than eight days in gaseous effluents does not exceed the limits in DODAM Section 6.2.4.	30 Days

O 6.3 Offsite Dose Assessment Controls and Surveillance Requirements

O 6.3.1 Dose

OLCO 6.3.1 The annual dose or dose commitment to any MEMBER OF THE PUBLIC due to radiation and radioactive material in effluents from DAEC shall not exceed 75 mrem to the thyroid or 25 mrem to the total body or any other organ.

APPLICABILITY: At all times.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Calculated dose from radioactive material released in liquid or gaseous effluents exceeds twice the limits of Sections 6.1.3, 6.2.3 or 6.2.4	A.1 Perform an assessment of compliance with 40 CFR 190 and 10 CFR 72.104 and limit subsequent releases such that the dose or dose commitment to a MEMBER OF THE PUBLIC is ≤75 mrem to his thyroid and ≤25 mrem to his total body or any other organ over 12 consecutive months including the period of elevated release.	7 days
B. The estimated dose exceeds either limit in Section 6.3.1.	B.1 Prepare and submit a Special Report to the NRC in lieu of any other report; it shall include the cause of the release of exposure, an estimate of the dose to the likely most exposed MEMBER(s) OF THE PUBLIC, corrective actions taken or planned to prevent a recurrence, and a schedule for achieving compliance. If the condition causing the limit(s) to be exceeded has not been corrected, the Special Report may also state a request for a variance in accordance with the provisions of 40 CFR Part 190. In that event, the request is timely, and a variance is granted until NRC action on the request is complete.	30 days

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
OSR 7.3.1.1	Cumulative dose contributions from liquid and gaseous effluents to a MEMBER OF THE PUBLIC offsite shall be evaluated as described in the DODAM.	12 Months

O 6.3 Offsite Dose Assessment Controls and Surveillance Requirements

O 6.3.2 Radiological Environmental Monitoring Program (REMP) and Ground Water Protection Program (GWPP)

OLCO 6.3.2 A radiological environmental monitoring program shall be conducted as specified in Table 6.3-1.

APPLICABILITY: At all times.

ACTIONS

CONDITION		REQUIRED ACTION		COMPLETION TIME
A.	Deviation from Table 6.3-1	A.1	Deviations are permitted from Table 6.3-1 if specimens are unobtainable due to hazardous conditions, seasonal unavailability, malfunction of automatic sampling equipment and other legitimate reasons. If specimens are unobtainable due to sampling equipment malfunction, every effort shall be made to complete corrective action prior to the end of the next sampling period. All deviations from Table 6.3-1 shall be documented in the Annual Radiological Environmental Report.	Once per year
B.	In the event radioactivity in a sampled environmental medium, averaged over a calendar quarter, is attributable to DAEC and exceeds an appropriate value listed in Table 6.3-3 or, if not listed, causes a potential annual dose exceeding two times the quarterly dose limit in Section 6.1.3 or 6.2.4	B.1	Prepare and submit to the Commission within 30 days after discovery a Special Report which includes an evaluation of any release conditions, environmental factors or other conditions which caused the value(s) of Table 6.3-3 or two times the quarterly dose limit to be exceeded and which defines the corrective actions to be taken. If the radioactivity in environmental sample(s) is not attributable to releases from the Station, the Special Report is not required. Instead, the sample(s) result(s) shall be reported and explained in the Annual Radiological Environmental Report.	30 Days

(continued)

ACTIONS (continued)

CONDITION		REQUIRED ACTION		COMPLETION TIME
C.	When environmental sampling medium is not available from a sampling location or the location is no longer appropriate.	C.1	The cause and the location where replacement samples were obtained and/or will be obtained shall be reported in the Annual Radiological Environmental Report.	Once per year
D.	A location is identified at which the calculated personal dose associated with one or more exposure pathways exceeds by 20% the maximum calculated dose associated with like pathway(s) at a location where sampling is conducted as specified by Table 6.3-1	D.1	The pathway(s) having maximum exposure potential at the newly identified location will be added to the radiological monitoring program at a subsequent Operations Committee meeting, if samples are reasonably attainable at the new location. Like pathway(s) monitored (sampled) at a location, excluding the control station location(s), having a lesser associated calculated personal dose may be deleted from the program at the time the new pathway(s) and location are added.	Once per year

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
OSR 7.3.2.1	Sampling and analyses required in Table 6.3-1 shall be performed such that the detection capabilities specified in Table 6.3-2 are achieved under routine conditions. If a sample analysis does not meet the LLD specified, report the reason attributed in the next Annual Radiological Environmental Report.	12 Months
OSR 7.3.2.2	DAEC shall conduct annually a land use census within three miles of the Station to identify radiologically important changes in land use.	12 Months

TABLE 6.3-1 RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM			
Exposure Pathway and/or Sample Type	Minimum Number of Sampling Stations	Sampling and Collection Schedule [(a)]	Type and Frequency of Analysis
Airborne Particulates	Five	Continuous operation of sampler with sample collection WEEKLY or as required by dust loading	Analyze for gross beta activity ≥ 72 hours after filter change. Perform gamma isotopic analysis on each sample having gross beta activity > 10 times the yearly mean of control samples. Perform gamma isotopic analysis on composite (by sampling location) of samples collected during each quarter
Airborne Iodine	Five	Continuous operation of sampler with sample collection WEEKLY	Analyze each cartridge for I-131 within 8 days of collection.
Ambient Radiation	Forty-Two	QUARTERLY	Read gamma radiation dose quarterly.
Surface Water	Two	MONTHLY	Tritium and Gamma isotopic analysis of each sample.
Drinking Water	Two	MONTHLY	Tritium and Gamma isotopic analysis of each sample. If reactor by-product gamma emitters are identified, OR tritium concentration is >MDA, then analyze, Sr-89, Sr-90, Fe-55 and Ni-63.
Drinking Water	Five	QUARTERLY	Tritium and Gamma isotopic analysis of each sample. If reactor by-product gamma emitters are identified, OR tritium concentration is > MDA, then analyze for Sr-89, Sr-90, Fe-55 and Ni-63.
River Sediment	One	SEMIANNUALLY	Gamma isotopic analysis of each sample.
Fish	Two	SEMIANNUALLY	Gamma isotopic analysis on edible portion[TD5].
Vegetation	Four	ANNUALLY at harvest time. At least two samples of each: grain green leafy vegetation forage.	Gamma isotopic and I-131 analysis of each sample.

[(a)]The following definitions of schedule timing shall apply to Table 6.3-1 only:

WEEKLY – Not less than once per calendar week. A maximum interval of 11 days is allowed between the collection of any two consecutive samples.

BIWEEKLY – Not less than once every two calendar weeks. A maximum interval of 18 days is allowed between the collection of any two consecutive samples.

MONTHLY – Not less than once per calendar month. An interval of not less than 10 days will be provided between collection of any two consecutive samples.

QUARTERLY – Not less than once per calendar quarter. An interval of not less than 30 days will be provided between collection of any two consecutive samples.

SEMIANNUALLY – One sample each between calendar dates (January 1 – June 30) and (July 1 – December 31). An interval of not less than 60 days will be provided between collection of any two consecutive samples.

ANNUALLY – Not less than once per calendar year with an interval of not less than 120 days between collection of any two consecutive samples.

TABLE 6.3-2			
GROUNDWATER PROTECTION PROGRAM			
Exposure Pathway and/or Sample Type	Minimum Number of Sampling Stations	Sampling and Collection Schedule [(a)]	Type and Frequency of Analysis
Groundwater	Twenty-Eight [(b)]	QUARTERLY	Tritium and Gamma isotopic analysis of each sample. If reactor by-product gamma emitters are identified, OR tritium concentration is >MDA, then analyze for Sr-89, Sr-90, Fe-55 and Ni-63.[(d)]
Groundwater Conditional	One [(c)]	MONTHLY	Tritium and Gamma isotopic analysis of each sample. If reactor by-product gamma emitters are identified, OR tritium concentration is >MDA, then analyze for Sr-89, Sr-90, Fe-55 and Ni-63.[(d)]
Electrical Vaults	One	SEMIANNUALLY	Tritium and Gamma isotopic analysis of each sample.
DAEC Sewage Plant Effluent	One	BIWEEKLY	Tritium and Gamma isotopic analysis of each sample.

[(a)]The following definitions of schedule timing shall apply to Table 6.3-2 only:

WEEKLY – At least once per calendar week.

BIWEEKLY – Not less than once every two calendar weeks. A maximum interval of 18 days is allowed between the collection of any two consecutive samples.

MONTHLY – Not less than once per calendar month. An interval of not less than 10 days will be provided between collection of any two consecutive samples.

QUARTERLY – Not less than once per calendar quarter. An interval of not less than 30 days will be provided between collection of any two consecutive samples.

SEMIANNUALLY – One sample each between calendar dates (January 1 – June 30) and (July 1 – December 31). An interval of not less than 60 days will be provided between collection of any two consecutive samples.

ANNUALLY – Not less than once per calendar year with an interval of not less than 120 days between collection of any two consecutive samples.

[(b)]As of January 1, 2017, fifty-six groundwater wells are operational.

[(c)]The number of conditional wells sampled in a calendar month depends upon identified isotope, concentration of isotope, targeted aquifer, and extraction well operations.

[(d)]An offsite laboratory performs the analysis of Sr-89, Sr-90, Fe-55 and Ni-63. Gross alpha is not analyzed due to the interference of naturally occurring radiological material in groundwater, and subsequently, groundwater samples.

TABLE 6.3-3						
MAXIMUM VALUES OF THE LOWER LIMIT OF DETECTION FOR ENVIRONMENTAL SAMPLE ANALYSIS [(a)]						
Analysis	Medium					
	Water (pCi/L)	Airborne Particulate or Gas (pCi/m^3)	Fish (pCi/kg, wet)	Milk (pCi/L)	Food Products (pCi/kg, wet)	Sediment (pCi/kg, dry)
Gross beta	4	1×10^{-2}				
H-3	2000 [(b)] 3000 [(c)]					
Mn-54	15		130			
Fe-59	30		260			
Co-58, Co-60	15		130			
Zn-65	30		260			
Zr-95	30					
Nb-95	15					
I-131	1 [(d)]	7×10^{-2}		1	60	
Cs-134	15	5×10^{-2}	130	15	60	150
Cs-137	18	6×10^{-2}	150	18	80	180
Ba-140	60			60		
La-140	15			15		
Other	30 [(e)]			30 [(e)]		

<u>TABLE NOTATIONS</u> Applies to Table 6.3-1 & Table 6.3-2

[(a)] The LLD is defined, for purposes of these specifications, as the smallest concentration of radioactive material in a sample that will yield a new count, above system background, that will be detected with 95% probability with only 5% probability of falsely concluding that a blank observation represents a "real" signal.

For a particular measurement, which may include radiochemical separation

LLD = $(4.66S_b)/(E \cdot V \cdot 2.22 \cdot Y \cdot e^{-\lambda\Delta t})$ where:

- LLD is the lower limit of detection as defined above (picocuries per unit mass or volume)
- Sb is the standard deviation of the background counting rate or of the counting rate of a blank sample as appropriate (counts per minute)
- E is the counting efficiency (counts per disintegration)
- V is the sample size (units of mass or volume)
- 2.22 is the number of disintegrations per minute per picocurie,
- Y is the fractional radiochemical yield, when applicable,
- λ is the radioactive decay constant for the particular radionuclide, and D
- t for environmental samples is the elapsed time between sample collection, or end of the sample collection period, and time of counting

Analyses shall be performed in such a manner that the stated LLDs will be achieved under routine conditions. With typical values of E, V, Y, and Δt for the radionuclides named in the Table. Occasionally background fluctuations, unavoidably small sample sizes, the presence of interfering nuclides, or other uncontrollable circumstances may render these LLDs unachievable. In such cases, the contributing factors shall be identified and described in the Annual Radiological Environmental Report. When a radionuclide attributable to DAEC but not listed in this table is measured it shall be reported.

[b] For Drinking Water.

[c] For samples of water not used as a source of drinking water.

[d] If no drinking water pathway exists, a value of 15 pCi/l may be used.

[e] "Other" refers to non-specified gamma emitters resulting from operation of DAEC. Naturally occuring radioculides are not included.

TABLE 6.3-4					
REPORTING LEVELS FOR RADIOACTIVITY CONCENTRATIONS IN ENVIRONMENTAL SAMPLES					
Reporting Levels[(a)]					
Analysis	Water (pCi/liter)	Airborne Particulate or Gases (pCi/m^3)	Fish (pCi/Kg, wet)	Milk (pCi/liter)	Food Products (pCi/Kg, wet)
H-3	2×10^4 [(b)] 3×10^4 [(c)]				
Mn-54	1×10^3		3×10^4		
Fe-59	4×10^2		1×10^4		
Co-58	1×10^3		3×10^4		
Co-60	3×10^2		1×10^4		
Zn-65	3×10^2		2×10^4		
Zr-Nb95	4×10^2 [(c)]				
I-131	2 [d]	0.9		3	1×10^2
Cs-134	30	10	1×10^3	60	1×10^3
Cs-137	50	20	2×10^3	70	2×10^3
Ba-La140	2×10^2 [(e)]			3×10^2 [(e)]	
Other	30 [(f)]			30 [(f)]	

TABLE NOTATIONS Table 6.3-3

(a) The reporting level is exceeded when one or more radionuclides are detected in a sample and $\Sigma[(\text{concentration})/(\text{reporting level})] \geq 1$.

(b) For drinking water samples. This is 40 CFR Part 141 value.

(c) For samples of water not used as a source of drinking water.

(d) If no drinking water pathway exists, a value of 20 pCi/l may be used.

(e) Concentration of parent or daughter.

(f) "Other" refers to non-specified gamma emitters resulting from operation of DAEC. Naturally occurring radionuclides are not included.

BASES SECTION 6.1 / 7.1

6.1.1 and 7.1.1 Radioactive Liquid Effluent Instrumentation

The radioactive liquid effluent instrumentation is provided to monitor and control, as applicable, the release of radioactive material in liquid effluents. The FUNCTIONALITY and use of these instruments implements the requirements of 10 CFR Part 50, Appendix A, General Design Criteria 60, 63, and 64. The alarm and/or trip setpoints for these instruments are calculated in the manner described in the DODAM to assure that the alarm and/or trip will occur before ten times the values specified in 10 CFR Part 20.2001 - 20.2402, Appendix B are exceeded. Instrumentation is expected to be FUNCTIONAL and in service when required by controls. An instrument may be removed from service voluntarily for the purpose of tests, checks, or calibration, without declaring the channel non-functional.

The radwaste effluent line radiation monitor recorder RR3972 is not required in order to comply with the controls stated in the DODAM. It is provided for recording trends during a release and need not be maintained FUNCTIONAL - i.e. calibrated and functionally tested - other than during a release.

BASES SECTION 6.1 / 7.1

6.1.2 and 7.1.2 Liquid Effluent Concentration

The basic requirements concerning effluents from nuclear power are stated in 10 CFR 50.36a. These requirements indicate that compliance with Technical Specifications will keep average annual releases of radioactive material in effluents to a small percentage of the limits specified in 10 CFR 20.106. These (new 10 CFR 20.1301) requirements further indicate that operational flexibility is allowed, compatible with considerations of health and safety, which may temporarily result in releases higher than such small percentages, but still within the limits specified in the old 10 CFR 20.106. These referenced concentrations are specific values which relate to an annual average dose of 500 millirems.

As stated in the Introduction to Appendix B of the new 10 CFR 20, the liquid effluent concentrations limits, given in the appendix, are based on an annual dose of 50 millirem. Prior to the issuance of the revision to Part 20, a release concentration corresponding to a limiting dose of 500 millirem had been acceptable as a TS limit for liquid effluents. The limit was applicable at all times and has been used to provide assurance that the limits of 10 CFR 50 Appendix I were not likely to be exceeded. Since the limits of 10 CFR 50 are the ultimate value to which a license must adhere, and since the old concentration limits adequately provide such assurance, it should not be necessary to reduce this limit by a factor of 10.

Conformance with the dose limits in section 6.1.3 will necessitate average annual liquid effluent concentrations being below those specified in 10 CFR 20.1001 - 20.2402 Appendix B to §§ 20.1001-20.2402.

Demonstrating compliance with section 6.1.3 will result in a de facto demonstration of compliance with 10 CFR 20 limits.

The continuous service water sample points noted in Table 7.1-2 are adequate to ensure sampling of potential liquid radioactive effluents from the service water systems. The sample points include the General Service Water System and the RHR Service Water Systems A and B. The sample point for the RHR Service Water Systems is at a location downstream of the point where Emergency Service Water discharge joins with the RHR Service Water System, and upstream of the point where the RHRSW/ESW Rupture Disc Line branches off the RHR Service Water System. This sample point will therefore provide for sampling effluents from the RHR Service Water System, Emergency Service Water System and/or RHRSW/ESW Rupture Disc Line.

The "clean" system water sample points noted in Table 7.1-2 are adequate to ensure sampling of potential liquid radioactive effluents from systems believed to NOT contain principal gamma emitters and iodine. The sample points include the Transformer Pit, the CST Pit, FRAC tanks, and the Neut. Tank (1T022).

6.1.2 / 7.1.2 (cont'd)	The GWPP mitigation system sample points noted in Table 7.1-2 are adequate to ensure sampling of potential liquid radioactive effluents in the shallow groundwater. Further, the GWPP mitigation plan support a reduction in contaminate concentration such that releases are expected below Administrative Control Procedure (ACP) 1411.35 and Environmental Protection Agency (EPA) drinking water limit of 20,000 pCi/L.

6.1.3 and 7.1.3

1. Dose Due to Radioactive Effluents

Section 6.1.3, 6.2.3 and 6.2.4 implement the requirements of 10 CFR Part 50.36a and of 10 CFR Part 50, Appendix I, Section IV. These sections keep levels of radioactive materials in LWR effluents as low as is reasonably achievable. Compliance with these sections will also keep average releases of radioactive material to effluent at small percentages of the limits specified in 10 CFR Part 20.106. Surveillance requirements provide for the measurement of releases and calculation of doses to verify compliance with the controls. Action statements in these sections implement the requirements of 10 CFR Part 50.36(c)(2) and 10 CFR Part 50, Appendix I, Section IV.A in the event a control is not met.

2. Liquid Effluents

With the implementation of Section 6.1.3, there is reasonable assurance that Station operation will not cause a radionuclide concentration in public drinking water taken from the Cedar River that exceeds the standard for anthropogenic radioactivity in community drinking water. The equations in the DODAM for calculating doses due to measured releases of radioactive material in liquid effluent are consistent with the methodology in Regulatory Guide 1.109 and 1.113. The assessment of personal doses will examine potential exposure pathways including, as appropriate, consumption of fish and water taken from the Cedar River downstream of the discharge pipe.

6.1.4 and 7.1.4

Liquid Waste Treatment

This section implements the requirements of 10 CFR Part 50.36a (a)(1) that operating procedures be established and followed, and that equipment be maintained and used to keep releases to the environment as low as is reasonably achievable. The section intends that appropriate portions of the system which were used to establish compliance with the design objectives in 10 CFR Part 50, Appendix I, Section II be used when specified to provide reasonable assurance that releases of radioactive material in liquid effluent will be kept as low as is reasonably achievable. The components in the liquid radwaste system which are appropriate to process liquid waste in order to satisfy Section 7.1.4 are the floor drain demineralizer and the radwaste demineralizer. The activity concentration, µCi/mL, below which liquid radwaste treatment would not be cost-beneficial, and therefore not required, is demonstrated below. The quantity of radioactive material in liquid effluent released annually from the DAEC has been calculated to be:

total iodines	0.11 curie
total others (less H3)	0.25
Total	0.36 curie

The population dose commitment resulting from the radioactive material in liquid effluent released annually has been calculated to be*

thyroid		0.164 man rem
total body		0.114
	Total	0.278 man rem

Therefore, population doses are about 1.5 man rem per curie of iodine released and about 0.5 man rem per curie of other radionuclides (less H3) released in liquids. On the basis of gross activity, the population dose is about one man rem per curie released in liquids.

The volume of liquid waste processed and intended for discharge is estimated to be:

Low Purity Waste	5700 gal/day = 1.8 x 106 gal/yr
Chemical Waste	600 gal/day = 1.9 x 105 gal/yr

Since the same DAEC equipment is used to process both streams, the total volume to be processed is about 2 x 106 gal/yr. The annual cost to operate the radwaste processing equipment, based on Dirty Waste Ion Exchange operation, has been estimated* (neglecting credit for capital recovery) to be $88,000 per year. Thus the unit volume operating cost is about:

$$\frac{\$88{,}000\,/\,yr}{2\ x\ 10^{6}\ \ gal\,/\,yr} = \$0.05\,/\,gal$$

Thus, the operating cost to treat a 4000 gallon batch of chemical waste by ion exchange would be about $200. The operating cost to treat a 10000 gallon batch of floor drain waste by ion exchange would be about $500. Assuming the cost-benefit balance is $1000 expenditure per man rem reduced and assuming treatment removes all radioactivity from the liquid, then

(1) the activity concentration in a Chemical Waste batch below which treatment is not cost-beneficial is

$$C = \frac{\$200}{4000\ gal\ x\ 3785\,\frac{ml}{gal}}\,x\,\frac{1\ curie}{man\,rem}\,x\,\frac{10^{6}\ \mu Ci}{curie}\,x\,\frac{1\ man\,rem}{\$1000}$$

$$C = 0.013\ \mu Ci/ml$$

(continued)

6.1.4 and 7.1.4

(2) the activity concentration in a batch of Floor Drain Waste below which treatment is not cost-beneficial is

$$C = \frac{\$500}{10000\ gal\ x\ 3785 \frac{ml}{gal}} x \frac{1\ curie}{man\ rem} x \frac{10^6\ \mu Ci}{curie} x \frac{1\ man\ rem}{\$1000}$$

$$C = 0.013\ \mu Ci/ml$$

Liquid waste treatment with the evaporator at DAEC has been shown to be neither cost-beneficial nor necessary to comply with 10 CFR 50 Appendix I, Section II. Consequently, liquid radwaste treatment to achieve an activity concentration below 0.01 μCi/mL in liquid effluent is not justified.

6.1.5 and 7.1.5 Liquid Holdup Tanks

The tanks listed in the specification include all liquid radwaste tanks (1T-88 and 1T-269) located in the Low-Level Radwaste Processing Facility (LLRPSF). Because the LLRPSF is not seismically designed, these tanks are considered as outdoor tanks that are not surrounded by liners, dikes, or walls capable of holding the tanks' contents.

Restricting the quantity of radioactive material contained in the specified tanks provides assurance that in the event of an uncontrolled release of the tanks' contents, the resulting concentrations would be less than the limits of 10 CFR 20, Appendix B to 20.1001 - 20.2402, Table 2, Column 2, at the nearest potable water supply in an UNRESTRICTED AREA.

BASES SECTION 6.2 / 7.2

6.2.1 and 7.2.1 Radioactive Gaseous Effluent Instrumentation

The radioactive gaseous effluent instrumentation is provided to monitor the release of radioactive materials in gaseous effluents and, as appropriate, to control potential releases. The presence of instruments for monitoring radioactive effluents is depicted in DODAM Figure 3-1. The FUNCTIONALITY and use of these instruments implement the requirements of 10 CFR Part 50, Appendix A, General Design Criteria 60, 63, and 64.

DAEC is equipped with a radioactive gaseous effluent monitoring system which includes detectors at the offgas stack (R3), the reactor building vent (R4), the turbine building vent (R5), and the LLRPSF vent (R7). A remote indication and control unit located near each detector displays the detector reading and, whenever the setpoint is exceeded, an indicator light. The data are also routed to a control computer and a control room display, but do not cause a trip to isolate the ventilated area. In the event the control computer and/or control room display fail to function or are voluntarily taken out of service, each remote indication and control unit is designed to acquire data for up to 30 hours. It is intended that each affected remote indication and control unit display be observed at least once per 24 hours (in which case the affected channel remains FUNCTIONAL).

If an alarm trip setpoint is exceeded at the same time the control computer and/or control room display are neither functioning nor in service, alarm annunciation will still occur in the control room. In the event the detector reading and the indication of exceeding the monitor setpoint are not provided at either the control room or the remote indication and control unit, then the affected channel is not FUNCTIONAL and DAEC will either perform the appropriate ACTION or will provide an alternate monitoring system. This permits DAEC to retain the GE gaseous monitoring system as an alternate system for normal effluent monitoring when the Kaman system is temporarily non-functional. When used as an alternate monitoring system, the GE system is subject to the requirements stated in Sections 6.2.1 and 7.2.1 and to LLD requirements stated in Table 7.2-2, Note (d).

2. Not used

3. Gaseous Effluents

Assessments of dose required by Sections 7.2.3 and 7.2.4 to verify compliance with Appendix I, Section IV are based on measured radioactivity in gaseous effluent and on calculation methods stated in the DODAM. Pathways of exposure and location of individuals are selected such that the dose to a nearby resident is unlikely to be underestimated. Dose assessment methodology described in the DODAM for gaseous effluent will be consistent with the methodology in Regulatory Guides 1.109 and 1.111. Cumulative and projected assessments of dose made during a quarter are based on historical average meteorological conditions measured at DAEC. Assessment made for the annual radiological environmental report will be based on annual averages of atmospheric conditions during the period of release.

6.2.2 and 7.2.2 **Gaseous Effluent Concentration**

The basic requirements concerning effluents from nuclear power are stated in 10 CFR 50.36a. These requirements indicate that compliance with Technical Specifications will keep average annual releases of radioactive material in effluents to a small percentage of the limits specified in 10 CFR 20. These (new 10 CFR 20.1301) requirements further indicate that operational flexibility is allowed, compatible with considerations of health and safety, which may temporarily result in releases higher than such small percentages, but still within the limits specified in the old 10 CFR 20.106. These referenced concentrations are specific values which relate to an annual average dose of 500 millirems.

As stated in the Introduction to Appendix B of the new 10 CFR 20, the gaseous effluent concentration limits given in the appendix are based on an annual dose of 50 millirem for isotopes for which inhalation or ingestion is limiting or 100 millirem for isotopes for which submersion (noble gases) is limiting. Prior to the issuance of the revision to Part 20 a release concentration corresponding to limiting dose rates less than or equal to 500 mrem/yr to the whole body, 3000 mrem/yr to the skin from noble gases, and 1500 mrem/yr to any organ from iodine, and tritium, had been acceptable as a TS limit for airborne effluents. This limit was applicable at all times and had been used to provide assurance that the limits of 10 CFR 50 Appendix I and 40 CFR 190 were not likely to be exceeded.

Since the limits of 10 CFR 50 Appendix I and 40 CFR 90 are more restrictive than 10 CFR 20, and because the dose limits specified have been successfully used to assure compliance with these regulations, it should not be necessary to reduce the dose rate basis to 50 or 100 millirem.

Conformance with the dose limits of 7.3.1 will necessitate the average annual airborne effluent concentrations being below those specified in 10 CFR 20 Appendix B. Demonstrating compliance with section 6.3.1 will result in a de facto demonstration of compliance with 10 CFR 20 limits. Assessment of compliance is based upon an effluents measurement program defined in Table 7.2-2 and methodology stated in the DODAM. The resolving time of the measurements, i.e., the sample integration time, bounds the minimum averaging time of the effluent measurements waste streams.

6.2.3 and 7.2.3 **Doses due to Noble Gases**

These specifications implement the requirements of 10 CFR Part 50, Appendix I.

6.2.4 and 7.2.4 **Doses due to Iodine and Particulates in Air**

These specifications implement 10 CFR Part 50, Appendix I. The dose calculation methods in the DODAM depend on existing pathways of exposure to a member of the public or more conservative conditions assumed (yielding a higher calculated dose). Calculations and methods are such that an estimate of the dose to a member of the public is not likely to be underestimated substantially.

6.3.1 and 7.3.1 Dose

Section 6.3.1 is provided to comply with the dose limitation requirement of 40 CFR 190. This section requires the assessment of dose to demonstrate that a person (a nearby resident) has not received a radiation dose exceeding that specified in 40 CFR 190 including doses from direct radiation. There is no other licensed nuclear fuel cycle facility within 50 miles of DAEC, thus it is assumed that the dose from other uranium fuel cycle facilities is negligible. In the event a report is required to satisfy Action b, it shall be deemed adequate to satisfy the reporting requirement in Section 8.2.2.

By demonstrating compliance with 40 CFR 190, DAEC will be, de facto, in compliance with the dose limits specified in 10 CFR 20.1301 and 10 CFR 72.104. Such a position is in keeping with that stated by the NRC in the preamble to the revised 10 CFR 20 (56 CFR 23360).

6.3.2 and 7.3.2 Radiological Environmental Monitoring

The radiological environmental monitoring program, including the land use census, is conducted to satisfy the requirements of 10 CFR Part 50, Appendix I, Section IV.B.2 and .3. The minimum radiological monitoring program required by this specification provides measurements of radiation and of radioactive materials in those exposure pathways and for those radionuclides which lead to the highest potential radiation exposures of individuals resulting from the station operation. This monitoring program thereby supplements the radiological effluent monitoring program by verifying that the measurable concentrations of radioactive materials and levels of radiation are not higher than expected on the basis of the effluent measurements and modeling of the environmental exposure pathways.

The land use census is conducted annually to identify changes in use of the UNRESTRICTED AREA in order to recommend modifications in monitoring programs for evaluating individual doses from principal exposure pathways. It may be conducted by door-to-door survey, by aerial survey, or by consulting with local agricultural or governmental authorities.

In order that radiological environmental monitoring stations may be relocated to reflect current conditions, the locations of stations required by Table 6.3-1 are described in the Offsite Dose Assessment Manual. Revisions thereto are administered in accordance with TS 5.5.1 and DODAM 8.3.1. DAEC may conduct additional environmental monitoring exclusive of the requirements of Sections 6.3.2.

8.0 ADMINISTRATIVE CONTROLS

8.1 Programs and Manuals

8.1.1 Radioactive Effluent Controls Program

This program, conforming to 10 CFR 50.36a, provides for the control of radioactive effluents and for maintaining the doses to members of the public from radioactive effluents as low as reasonably achievable. The program shall be contained in the DODAM, shall be implemented by procedures, and shall include remedial actions to be taken whenever the program limits are exceeded. The program shall include the following elements.

a. Limitations on the functional capacity of radioactive liquid and gaseous monitoring instrumentation including surveillance tests and setpoint determination in accordance with the methodology in the DODAM;

b. Limitations on the concentrations of radioactive material released in liquid effluents from the site to UNRESTRICTED AREAs, conforming to ten times (10x) the concentrations listed in Appendix B, Table 2, Column 2 to 10 CFR 20.1001 - 20.2402;

c. Monitoring, sampling, and analysis of radioactive liquid and gaseous effluents pursuant to 10 CFR 20.1302 and with the methodology and parameters in the DODAM conforming to 10CFR50, Appendix I;

d. Limitations on the annual and quarterly doses or dose commitment to a member of the public from radioactive materials in liquid effluents released to UNRESTRICTED AREAs;

e. Determination of cumulative and projected dose contributions from radioactive effluents for the current calendar quarter and current calendar year in accordance with the methodology and parameters in the DODAM at least every 30 days. If 30-day projected quarterly dose exceeds quarterly dose limits, a utilization evaluation of liquid waste processing system is needed;

f. Limitations on the functional capability and use of the liquid and gaseous effluent treatment systems to ensure that the appropriate portions of these systems which were used to establish compliance with the design objectives in 10 CFR 50, Appendix I, Section II be used when specified to provide reasonable assurance that releases of radioactive material in liquid and gaseous effluents be kept as low as reasonably achievable;

g. Limitations on the dose rate resulting from radioactive material released in gaseous effluents to areas at or beyond the site boundary shall be limited to the following:

   1. For noble gases: less than or equal to a dose rate of 500 mrem/yr to the whole body and less than or equal to a dose rate of 3000 mrem/yr to the skin, and
   2. For iodine-131, iodine-133, tritium, and for all radionuclides in particulate form with half-lives > 8 days: less than or equal to a dose rate of 1500 mrem/yr to any organ;

h. Limitations on the annual and quarterly air doses resulting from noble gases released in gaseous effluents to areas beyond the site boundary, conforming to 10 CFR 50, Appendix I;

I. Limitations on the annual and quarterly doses to a member of the public from iodine-131, iodine-133, tritium, and all radionuclides in particulate form with half-lives > 8 days in gaseous effluents released to areas beyond the site boundary, conforming to 10 CFR 50, Appendix I; and

j. Limitations on the annual dose or dose commitment to any member of the public due to releases of radioactivity and to radiation from uranium fuel cycle sources, conforming to 40 CFR 190 and 10 CFR 72.104.

8.1.2 Radiological Environmental Monitoring Program

The Radiological Environmental Monitoring Program is described in section 5 of the DODAM. Specifications for implementation are located in section 6.3.2

8.1.3 Interlaboratory Comparison Program

Analyses shall be performed on radioactive materials supplied in an Interlaboratory Comparison Program.

In the event analyses were not performed as required in Section 8.1.3, report the corrective actions taken to prevent a recurrence in the Annual Radiological Environmental Report.

The requirement for participation in an Interlaboratory Comparison Program is provided to ensure that independent checks on the precision and accuracy of the measurements of radioactive material in environmental sample matrices are performed as part of the quality assurance program for environmental monitoring in order to demonstrate that the results are reasonably valid.

8.2 Reporting Requirements

8.2.1 Annual Radioactive Material Release Report

(1) A report of radioactive materials released from the Station shall be submitted to the NRC on or before May 1 of each year in accordance with 10 CFR 50.36a. Each report shall include the information specified in item (2) below covering the preceding twelve months.

(2) An Annual Radioactive Material Release Report shall include a summary by calendar quarter of the quantities of radioactive liquid and gaseous effluents and radioactive solid waste released from the Station. The data on radioactive liquid and gaseous effluents should be reported in the format in Tables 8.2-1 and 8.2-2. The data on radioactive solid waste should include:

1. classification of the waste (per 10 CFR Part 61)
2. total volume shipped
3. total radioactive material shipped (curies)
4. identification of principal radionuclides
5. solidification agent
6. physical description of the waste

(3) A summary description of any changes to the DODAM.

(4) A summary of meteorological data collected during the year will be submitted in the annual report following January 1. Alternatively, summary meteorological data may be retained by DAEC and made available to the NRC upon request.

(5) Include a description for all sample analyses/conditions for which communications were made to State and Local officials due to exceeding the applicable DODAM reporting levels for environmental samples for locations that have not been added to the plant's Radiological Environmental Monitoring Program.

(6) Include a description of all spills or leaks (of radioactive material) that were communicated to State/Local Stakeholders.

(continued)

TABLE 8.2-1			
ANNUAL RADIOACTIVE MATERIAL RELEASE REPORT (YEAR) LIQUID EFFLUENTS			
Nuclides Released	Unit	Quarter	Quarter
strontium-89	Ci	. E	. E
strontium-90	Ci	. E	. E
cesium-134	Ci	. E	. E
cesium-137	Ci	. E	. E
iodine-131	Ci	. E	. E
cobalt-58	Ci	. E	. E
cobalt-60	Ci	. E	. E
iron-55	Ci	. E	. E
iron-59	Ci	. E	. E
zinc-65	Ci	. E	. E
manganese-54	Ci	. E	. E
chromium-51	Ci	. E	. E
zirconium-niobium-95	Ci	. E	. E
molybdenum-99	Ci	. E	. E
technetium-99m	Ci	. E	. E
barium-lanthanum-140	Ci	. E	. E
cerium-141	Ci	. E	. E
Other (specify)	Ci	. E	. E
	Ci	. E	. E
	Ci	. E	. E
	Ci	. E	. E
	Ci	. E	. E
Total for period (above)	Ci	. E	. E
xenon-133	Ci	. E	. E
xenon-135	Ci	. E	. E

TABLE 8.2-2			
ANNUAL RADIOACTIVE MATERIAL RELEASE REPORT (YEAR) GASEOUS EFFLUENTS			
Nuclides Released	Unit	Quarter	Quarter
1. Fission gases			
krypton-85	Ci	. E	. E
krypton-85m	Ci	. E	. E
krypton-87	Ci	. E	. E
krypton-88	Ci	. E	. E
xenon-133	Ci	. E	. E
xenon-135	Ci	. E	. E
xenon-135m	Ci	. E	. E
xenon-138	Ci	. E	. E
Others (specify)	Ci	. E	. E
	Ci	. E	. E
	Ci	. E	. E
Total for period	Ci	. E	. E
2. Iodines			
iodine-131	Ci	. E	. E
iodine-133	Ci	. E	. E
iodine-135	Ci	. E	. E
Others (specify)			
Total for period	Ci	. E	. E
3. Particulates			
strontium-89	Ci	. E	. E
strontium-90	Ci	. E	. E
cesium-134	Ci	. E	. E
cesium-137	Ci	. E	. E
barium-lanthanum-140	Ci	. E	. E
Others (specify)	Ci	. E	. E
	Ci	. E	. E

8.2.2 Annual Radiological Environmental Operating Report

An annual report of radiological environmental surveillance activities required by Section 6.3.2 shall be submitted to the NRC by May 15th of each year. Each report shall be consistent with the objectives outlined in the DODAM, and with 10 CFR 50, Appendix I, Sections IV.B.2, IV.B.3, and IV.C. The report shall include the following information:

(1) A summary description of the radiological environmental monitoring program required by Section 6.3.2.

(2) A map and a table of distances and directions of locations of sampling stations required in Table 6.3-1.

(3) A summary of the land use census required in Section 7.3.2.2.

(4) Results of analyses of samples required by the radiological environmental monitoring program, Table 6.3-1. In the event some results are not available, the reasons shall be explained in the report. In the event the missing results are obtained, they shall be submitted in a supplementary report as soon as is reasonable.

(5) An assessment of radiation doses to a MEMBER OF THE PUBLIC likely to be the most exposed due to radioactive liquid and gaseous effluents released from DAEC during the year. The assessment shall be performed as described in the DODAM.

(6) Deleted.

(7) Results of participation in the Interlaboratory Comparison Program.

(8) Deviation from environmental sampling schedule.

(9) A report of all analyses in which the LLD, required by Table 6.3-2, was not achieved.

(10) A report of any changes in sample locations.

(11) Include a description for all sample analyses/conditions for which communications were made to State and Local Stakeholders due to exceeding the REMP reporting levels provided in the plant's OCDM/DODAM for locations that are described in the plant's REMP or GWPP programs.

(12) Include onsite ground water sample results.

8.2.3 Special Reports

Special reports shall be submitted to the Director of Inspection and Enforcement Regional Office within the time period specified for each report. These reports shall be submitted covering the activities identified below pursuant to the requirements of the applicable reference specification.

1. Radioactive Liquid or Gaseous Effluent - calculated dose exceeding specified limit (DODAM Sections 6.1.3, 6.2.3, and 6.2.4).

2. Deleted.

3. Measured levels of radioactivity in an environmental sampling medium determined to exceed the reporting level values of DODAM Table 6.3-3 when averaged over any calendar quarter sampling period (DODAM Section 6.3.2). Report to State and Local authorities in accordance with Administrative Control Procedure (ACP) 1402.3.

4. Annual dose to a MEMBER OF THE PUBLIC determined to exceed 40 CFR Part 190 dose limit (DODAM Section 6.3.1.1).

5. Radioactive liquid waste release without treatment when activity concentration is equal to or greater than 0.01 μCi\mL (DODAM Section 6.1.4).

8.3 Changes to the DODAM

a. Shall be documented, and records of reviews performed shall be retained for the duration of the facility operating license. This documentation shall contain:
    1) Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s) and:
    2) A determination that the change will maintain the level of radioactive effluent control required by 10 CFR 20.1302, 40 CFR Part 190, 10 CFR 72.104, 10 CFR 50.36a, and Appendix I to 10 CFR Part 50 and not adversely impact the accuracy or reliability of effluent dose or setpoint calculations.

b. Shall become effective after review and acceptance by the Onsite Review Group and approval by the DAEC Decommissioning Director.

c. Shall be submitted to the commission in the form of a complete, legible copy of the entire DODAM as a part of or concurrent with the Annual Radioactive Material Release Report for the period of the report in which any change to the DODAM was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed and shall indicate the date the change was implemented.

APPENDIX A: DOSE TRANSFER FACTORS FOR AIRBORNE PATHWAYS

Environmental pathway models have been solved on the bases of unit radionuclide release in effluent (1 Ci/yr) and unit atmospheric dispersion (1 sec/m^3) or deposition (1/m^2) to derive dose transfer factors for airborne effluent. The dose transfer factors in this appendix were computed with the GASPAR II computer program[a], using default values of parameters applicable to the most exposed members of the public as recommended in Regulatory Guide 1.109, revision 1, with the following exceptions.

- Significant revisions of data since publication of the Regulatory Guide 1.109, revision 1 and incorporated into GASPAR II were employed. Data differing from those in the Regulatory Guide 1.109 are identified in GASPAR II documentation.[b]

- After publishing Reg. Guide 1.109, the NRC recommended that soil-to-plant bioaccumulation factors, B_{iv}, of cesium and strontium be changed.[c] The revised values were used to derive dose transfer factors tabulated for Sr89, Sr90, and Cs137.

- Values of environmental transit time recommended in Reg. Guide 1.109,[d] namely 1440 hr from harvest of stored vegetables to ingestion, were retained in lieu of default values in GASPAR II.[e]

These factors affected dose transfer factors more than 10% only for H3, Sr89, Sr90, Cs137, Ce141, and Ce144.

Dose transfer factors from C14 via inhalation and from Kr90 via irradiation by an airborne cloud are the same as in the previous MIDAS library since GASPAR II does not produce them. Skin dose transfer factors are assumed to be the same as total body dose transfer factors for H3 and C14 in exposure pathways involving inhalation or ingestion also because GASPAR II does not calculate them.

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[a] Strenge, D.L., et. al., GASPAR II - Technical Reference and User Guide, NUREG/CR-4653, March 1987

[b] Ibid., 3.3.1.1, 3.3.2.3

[c] USNRC, SECY-79-653A, January 30, 1980.

[d] Regulatory Guide 1.109, rev. 1, Table E-15

[e] Strenge, et. al., p. C.3.

Dose transfer factors are included hereafter for the following parameters.

Pathway	Age Group	Organ
Inhalation Ground irradiation Grass-cow-meat Vegetables, leafy + produce Plume irradiation	Adult Teenager	Total Body GI tract Kidney Thyroid Lung Skin

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = ADULT
PATHWAY = INHALATION

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m²)/Ci				
H 3	2.28E+01	2.28E+01	0.00E-01	2.28E+01	2.28E+01	2.28E+01	2.28E+01	2.28E+01
C 14	1.08E+02	1.08E+02	5.76E+02	1.08E+02	1.08E+02	1.08E+02	1.08E+02	0.00E-01
AR 41	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 83m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 87	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE131m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	3.17E+00	1.05E+02	0.00E-01	0.00E-01	7.23E-01	1.89E+00	4.56E+02	0.00E-01
MN 54	2.00E+02	2.45E+03	0.00E-01	1.25E+03	3.12E+02	0.00E-01	4.44E+04	0.00E-01
FE 55	1.25E+02	1.91E+02	7.81E+02	5.38E+02	0.00E-01	0.00E-01	2.29E+03	0.00E-01
FE 59	3.36E+02	5.96E+03	3.74E+02	8.81E+02	0.00E-01	0.00E-01	3.23E+04	0.00E-01
CO 58	6.56E+01	3.36E+03	0.00E-01	5.01E+01	0.00E-01	0.00E-01	2.94E+04	0.00E-01
CO 60	4.69E+02	9.03E+03	0.00E-01	3.64E+02	0.00E-01	0.00E-01	1.89E+05	0.00E-01
ZN 65	1.48E+03	1.70E+03	1.03E+03	3.26E+03	2.19E+03	0.00E-01	2.74E+04	0.00E-01
SR 89	2.76E+02	1.11E+04	9.63E+03	0.00E-01	0.00E-01	0.00E-01	4.44E+04	0.00E-01
SR 90	1.83E+04	2.29E+04	9.10E+05	0.00E-01	0.00E-01	0.00E-01	3.04E+05	0.00E-01
ZR 95	7.38E+02	4.75E+03	3.39E+03	1.09E+03	1.72E+03	0.00E-01	5.61E+04	0.00E-01
SB124	3.93E+02	1.29E+04	9.89E+02	1.87E+01	0.00E-01	2.40E+00	7.86E+04	0.00E-01
CS134	2.31E+04	3.30E+02	1.18E+04	2.69E+04	9.10E+03	0.00E-01	3.09E+03	0.00E-01
CS136	3.49E+03	3.71E+02	1.24E+03	4.63E+03	2.71E+03	0.00E-01	3.80E+02	0.00E-01
CS137	1.36E+04	2.66E+02	1.52E+04	1.97E+04	7.07E+03	0.00E-01	2.38E+03	0.00E-01
BA140	8.14E+01	6.91E+03	1.24E+03	1.56E+00	5.29E-01	0.00E-01	4.02E+04	0.00E-01
CE141	4.85E+01	3.80E+03	6.31E+02	4.28E+02	1.99E+02	0.00E-01	1.15E+04	0.00E-01
CE144	5.83E+03	2.59E+04	1.09E+05	4.53E+04	2.69E+04	0.00E-01	2.47E+05	0.00E-01
I 131	6.50E+02	1.99E+02	7.99E+02	1.13E+03	1.94E+03	3.77E+05	0.00E-01	0.00E-01
I 133	1.43E+02	2.81E+02	2.74E+02	4.69E+02	8.21E+02	6.81E+04	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = ADULT
PATHWAY = GROUND PLANE

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem m^2)/Ci	KIDNEY	THYROID	LUNG	SKIN
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
AR 41	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 83m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 87	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE131m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	1.47E+05	1.47E+05	1.47E+05	1.47E+05	1.47E+05	1.47E+05	1.47E+05	1.74E+05
MN 54	4.37E+07	4.37E+07	4.37E+07	4.37E+07	4.37E+07	4.37E+07	4.37E+07	5.13E+07
FE 55	9.80E+05	2.77E+05	5.48E+06	1.61E+06	0.00E-01	0.00E-01	3.22E+06	0.00E-01
FE 59	8.65E+06	8.65E+06	8.65E+06	8.65E+06	8.65E+06	8.65E+06	8.65E+06	1.01E+07
CO 58	1.20E+07	1.20E+07	1.20E+07	1.20E+07	1.20E+07	1.20E+07	1.20E+07	1.41E+07
CO 60	6.81E+08	6.81E+08	6.81E+08	6.81E+08	6.81E+08	6.81E+08	6.81E+08	8.02E+08
ZN 65	2.36E+07	2.36E+07	2.36E+07	2.36E+07	2.36E+07	2.36E+07	2.36E+07	2.72E+07
SR 89	6.85E+02	6.85E+02	6.85E+02	6.85E+02	6.85E+02	6.85E+02	6.85E+02	7.95E+02
SR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	7.76E+06	7.76E+06	7.76E+06	7.76E+06	7.76E+06	7.76E+06	7.76E+06	9.03E+06
SB124	1.90E+07	1.90E+07	1.90E+07	1.90E+07	1.90E+07	1.90E+07	1.90E+07	2.19E+07
CS134	2.17E+08	2.17E+08	2.17E+08	2.17E+08	2.17E+08	2.17E+08	2.17E+08	2.53E+08
CS136	4.75E+06	4.75E+06	4.75E+06	4.75E+06	4.75E+06	4.75E+06	4.75E+06	5.39E+06
CS137	3.26E+08	3.26E+08	3.26E+08	3.26E+08	3.26E+08	3.26E+08	3.26E+08	3.80E+08
BA140	6.50E+05	6.50E+05	6.50E+05	6.50E+05	6.50E+05	6.50E+05	6.50E+05	7.45E+05
CE141	4.34E+05	4.34E+05	4.34E+05	4.34E+05	4.34E+05	4.34E+05	4.34E+05	4.88E+05
CE144	2.21E+06	2.21E+06	2.21E+06	2.21E+06	2.21E+06	2.21E+06	2.21E+06	2.55E+06
I 131	2.73E+05	2.73E+05	2.73E+05	2.73E+05	2.73E+05	2.73E+05	2.73E+05	3.33E+05
I 133	3.90E+04	3.90E+04	3.90E+04	3.90E+04	3.90E+04	3.90E+04	3.90E+04	4.72E+04

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = ADULT
PATHWAY = ANIMAL MEAT

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m²)/Ci				
H 3	5.93E+00	5.93E+00	0.00E-01	5.93E+00	5.93E+00	5.93E+00	5.93E+00	5.93E+00
C 14	2.13E+03	2.13E+03	1.06E+04	2.13E+03	2.13E+03	2.13E+03	2.13E+03	2.13E+03
AR 41	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 83m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 87	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE131m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	1.99E+02	5.01E+04	0.00E-01	0.00E-01	4.40E+01	1.19E+02	2.65E+02	0.00E-01
MN 54	4.44E+04	7.10E+05	0.00E-01	2.32E+05	6.91E+04	0.00E-01	0.00E-01	0.00E-01
FE 55	7.36E+05	1.81E+06	4.58E+06	3.17E+06	0.00E-01	0.00E-01	1.77E+06	0.00E-01
FE 59	6.50E+06	5.64E+07	7.23E+06	1.70E+07	0.00E-01	0.00E-01	4.75E+06	0.00E-01
CO 58	1.08E+06	9.76E+06	0.00E-01	4.82E+05	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	4.15E+06	3.55E+07	0.00E-01	1.89E+06	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 65	1.46E+07	2.03E+07	1.01E+07	3.23E+07	2.16E+07	0.00E-01	0.00E-01	0.00E-01
SR 89	2.51E+05	1.40E+06	8.75E+06	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	3.74E+07	4.66E+07	1.85E+09	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	1.08E+04	5.04E+07	4.98E+04	1.59E+04	2.50E+04	0.00E-01	0.00E-01	0.00E-01
SB124	2.09E+05	1.50E+07	5.26E+05	9.98E+03	0.00E-01	1.28E+03	4.12E+05	0.00E-01
CS134	3.23E+07	6.91E+05	1.66E+07	3.93E+07	1.28E+07	0.00E-01	4.25E+06	0.00E-01
CS136	1.03E+06	1.63E+05	3.61E+05	1.43E+06	7.95E+05	0.00E-01	1.09E+05	0.00E-01
CS137	2.08E+07	6.15E+05	2.33E+07	3.17E+07	1.08E+07	0.00E-01	3.58E+06	0.00E-01
BA140	5.70E+04	1.79E+06	8.71E+05	1.09E+03	3.71E+02	0.00E-01	6.27E+02	0.00E-01
CE141	3.01E+01	1.01E+06	3.93E+02	2.65E+02	1.23E+02	0.00E-01	0.00E-01	0.00E-01
CE144	1.96E+03	1.24E+07	3.64E+04	1.53E+04	9.06E+03	0.00E-01	0.00E-01	0.00E-01
I 131	1.37E+05	6.31E+04	1.67E+05	2.39E+05	4.09E+05	7.83E+07	0.00E-01	0.00E-01
I 133	3.11E-03	9.16E-03	5.86E-03	1.02E-02	1.78E-02	1.50E+00	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = ADULT
PATHWAY = VEGETABLES

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m^2)/Ci				
H 3	4.12E+01	4.12E+01	0.00E-01	4.12E+01	4.12E+01	4.12E+01	4.12E+01	4.12E+01
C 14	5.74E+03	5.74E+03	2.87E+04	5.74E+03	5.74E+03	5.74E+03	5.74E+03	5.74E+03
AR 41	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 83m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 87	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE131m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	1.45E+03	3.64E+05	0.00E-01	0.00E-01	3.20E+02	8.68E+02	1.93E+03	0.00E-01
MN 54	1.86E+06	2.99E+07	0.00E-01	9.76E+06	2.90E+06	0.00E-01	0.00E-01	0.00E-01
FE 55	3.99E+05	9.84E+05	2.49E+06	1.71E+06	0.00E-01	0.00E-01	9.56E+05	0.00E-01
FE 59	3.52E+06	3.07E+07	3.93E+06	9.19E+06	0.00E-01	0.00E-01	2.57E+06	0.00E-01
CO 58	2.13E+06	1.93E+07	0.00E-01	9.51E+05	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	1.16E+07	9.92E+07	0.00E-01	5.29E+06	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 65	1.83E+07	2.55E+07	1.27E+07	4.06E+07	2.70E+07	0.00E-01	0.00E-01	0.00E-01
SR 89	9.19E+06	5.13E+07	3.20E+08	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	1.26E+09	1.57E+09	6.27E+10	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	7.89E+03	3.68E+07	3.64E+04	1.16E+04	1.83E+04	0.00E-01	0.00E-01	0.00E-01
SB124	1.27E+06	9.13E+07	3.20E+06	6.08E+04	0.00E-01	7.80E+03	2.50E+06	0.00E-01
CS134	2.87E+08	6.15E+06	1.47E+08	3.52E+08	1.13E+08	0.00E-01	3.77E+07	0.00E-01
CS136	3.83E+06	6.02E+05	1.35E+06	5.32E+06	2.96E+06	0.00E-01	4.06E+05	0.00E-01
CS137	2.06E+08	6.08E+06	2.30E+08	3.14E+08	1.07E+08	0.00E-01	3.55E+07	0.00E-01
BA140	2.67E+05	8.37E+06	4.06E+06	5.10E+03	1.74E+03	0.00E-01	2.92E+03	0.00E-01
CE141	4.72E+02	1.59E+07	6.15E+03	4.15E+03	1.93E+03	0.00E-01	0.00E-01	0.00E-01
CE144	5.36E+04	3.39E+08	9.98E+05	4.18E+05	2.48E+05	0.00E-01	0.00E-01	0.00E-01
I 131	1.05E+06	4.82E+05	1.28E+06	1.83E+06	3.14E+06	5.99E+08	0.00E-01	0.00E-01
I 133	1.75E+04	5.17E+04	3.30E+04	5.74E+04	1.00E+05	8.43E+06	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = ADULT
PATHWAY = PLUME

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m³)/(Ci sec)				
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
AR 41	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	3.14E+02
KR 83m	1.68E-03	1.68E-03	1.68E-03	1.68E-03	1.68E-03	1.68E-03	9.38E-02	4.75E-01
KR 85m	2.60E+01	2.60E+01	2.60E+01	2.60E+01	2.60E+01	2.60E+01	2.66E+01	7.67E+01
KR 85	3.58E-01	3.58E-01	3.58E-01	3.58E-01	3.58E-01	3.58E-01	9.51E-01	4.28E+01
KR 87	1.32E+02	1.32E+02	1.32E+02	1.32E+02	1.32E+02	1.32E+02	1.35E+02	4.60E+02
KR 88	3.26E+02	3.26E+02	3.26E+02	3.26E+02	3.26E+02	3.26E+02	3.26E+02	4.50E+02
KR 89	3.68E+02	3.68E+02	3.68E+02	3.68E+02	3.68E+02	3.68E+02	3.71E+02	7.48E+02
KR 90	3.46E+02	3.46E+02	3.46E+02	3.46E+02	3.46E+02	3.46E+02	3.49E+02	6.33E+02
XE131m	2.03E+00	2.03E+00	2.03E+00	2.03E+00	2.03E+00	2.03E+00	2.38E+00	1.89E+01
XE133m	5.58E+00	5.58E+00	5.58E+00	5.58E+00	5.58E+00	5.58E+00	6.02E+00	3.96E+01
XE133	6.53E+00	6.53E+00	6.53E+00	6.53E+00	6.53E+00	6.53E+00	6.85E+00	1.84E+01
XE135m	6.91E+01	6.91E+01	6.91E+01	6.91E+01	6.91E+01	6.91E+01	6.94E+01	1.05E+02
XE135	4.02E+01	4.02E+01	4.02E+01	4.02E+01	4.02E+01	4.02E+01	4.09E+01	1.06E+02
XE137	3.15E+01	3.15E+01	3.15E+01	3.15E+01	3.15E+01	3.15E+01	3.55E+01	4.25E+02
XE138	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.97E+02	3.58E+02
CR 51	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MN 54	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 55	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 59	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 58	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 65	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SB124	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS134	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS136	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA140	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = TEENAGER
PATHWAY = INHALATION

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m^3)/(Ci sec)				
H 3	2.30E+01	2.30E+01	0.00E-01	2.30E+01	2.30E+01	2.30E+01	2.30E+01	2.30E+01
C 14	1.54E+02	1.54E+02	8.24E+02	1.54E+02	1.54E+02	1.54E+02	1.54E+02	0.00E-01
AR 41	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 83m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 87	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE131m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	4.28E+00	9.51E+01	0.00E-01	0.00E-01	9.73E-01	2.38E+00	6.65E+02	0.00E-01
MN 54	2.66E+02	2.12E+03	0.00E-01	1.62E+03	4.02E+02	0.00E-01	6.27E+04	0.00E-01
FE 55	1.75E+02	2.02E+02	1.06E+03	7.55E+02	0.00E-01	0.00E-01	3.94E+03	0.00E-01
FE 59	4.53E+02	5.64E+03	5.04E+02	1.17E+03	0.00E-01	0.00E-01	4.85E+04	0.00E-01
CO 58	8.81E+01	3.02E+03	0.00E-01	6.56E+01	0.00E-01	0.00E-01	4.25E+04	0.00E-01
CO 60	6.27E+02	8.21E+03	0.00E-01	4.79E+02	0.00E-01	0.00E-01	2.76E+05	0.00E-01
ZN 65	1.98E+03	1.48E+03	1.22E+03	4.25E+03	2.74E+03	0.00E-01	3.93E+04	0.00E-01
SR 89	3.96E+02	1.18E+04	1.38E+04	0.00E-01	0.00E-01	0.00E-01	7.67E+04	0.00E-01
SR 90	2.11E+04	2.42E+04	1.05E+06	0.00E-01	0.00E-01	0.00E-01	5.23E+05	0.00E-01
ZR 95	9.98E+02	4.72E+03	4.63E+03	1.45E+03	2.14E+03	0.00E-01	8.52E+04	0.00E-01
SB124	5.32E+02	1.26E+04	1.37E+03	2.52E+01	0.00E-01	3.09E+00	1.22E+05	0.00E-01
CS134	1.74E+04	3.09E+02	1.59E+04	3.58E+04	1.19E+04	0.00E-01	4.63E+03	0.00E-01
CS136	4.34E+03	3.45E+02	1.63E+03	6.15E+03	3.49E+03	0.00E-01	5.64E+02	0.00E-01
CS137	9.86E+03	2.69E+02	2.13E+04	2.69E+04	9.63E+03	0.00E-01	3.83E+03	0.00E-01
BA140	1.12E+02	7.26E+03	1.73E+03	2.13E+00	7.23E-01	0.00E-01	6.43E+04	0.00E-01
CE141	6.88E+01	3.99E+03	9.00E+02	6.02E+02	2.81E+02	0.00E-01	1.95E+04	0.00E-01
CE144	8.33E+03	2.74E+04	1.55E+05	6.40E+04	3.83E+04	0.00E-01	4.25E+05	0.00E-01
I 131	8.37E+02	2.06E+02	1.12E+03	1.56E+03	2.66E+03	4.63E+05	0.00E-01	0.00E-01
I 133	1.97E+02	3.26E+02	3.87E+02	6.50E+02	1.14E+03	9.25E+04	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = TEENAGER
PATHWAY = GROUND PLANE

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m²)/Ci				
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
AR 41	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 83m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 87	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE131m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	1.47E+05	1.47E+05	1.47E+05	1.47E+05	1.47E+05	1.47E+05	1.47E+05	1.74E+05
MN 54	4.37E+07	4.37E+07	4.37E+07	4.37E+07	4.37E+07	4.37E+07	4.37E+07	5.13E+07
FE 55	1.06E+06	3.10E+05	5.57E+06	1.69E+06	0.00E-01	0.00E-01	0.00E-01	3.22E+06
FE 59	8.65E+06	8.65E+06	8.65E+06	8.65E+06	8.65E+06	8.65E+06	8.65E+06	1.01E+07
CO 58	1.20E+07	1.20E+07	1.20E+07	1.20E+07	1.20E+07	1.20E+07	1.20E+07	1.41E+07
CO 60	6.81E+08	6.81E+08	6.81E+08	6.81E+08	6.81E+08	6.81E+08	6.81E+08	8.02E+08
ZN 65	2.36E+07	2.36E+07	2.36E+07	2.36E+07	2.36E+07	2.36E+07	2.36E+07	2.72E+07
SR 89	6.85E+02	6.85E+02	6.85E+02	6.85E+02	6.85E+02	6.85E+02	6.85E+02	7.95E+02
SR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	7.76E+06	7.76E+06	7.76E+06	7.76E+06	7.76E+06	7.76E+06	7.76E+06	9.03E+06
SB124	1.90E+07	1.90E+07	1.90E+07	1.90E+07	1.90E+07	1.90E+07	1.90E+07	2.19E+07
CS134	2.17E+08	2.17E+08	2.17E+08	2.17E+08	2.17E+08	2.17E+08	2.17E+08	2.53E+08
CS136	4.75E+06	4.75E+06	4.75E+06	4.75E+06	4.75E+06	4.75E+06	4.75E+06	5.39E+06
CS137	3.26E+08	3.26E+08	3.26E+08	3.26E+08	3.26E+08	3.26E+08	3.26E+08	3.80E+08
BA140	6.50E+05	6.50E+05	6.50E+05	6.50E+05	6.50E+05	6.50E+05	6.50E+05	7.45E+05
CE141	4.34E+05	4.34E+05	4.34E+05	4.34E+05	4.34E+05	4.34E+05	4.34E+05	4.88E+05
CE144	2.21E+06	2.21E+06	2.21E+06	2.21E+06	2.21E+06	2.21E+06	2.21E+06	2.55E+06
I 131	2.73E+05	2.73E+05	2.73E+05	2.73E+05	2.73E+05	2.73E+05	2.73E+05	3.33E+05
I 133	3.90E+04	3.90E+04	3.90E+04	3.90E+04	3.90E+04	3.90E+04	3.90E+04	4.72E+04

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = TEENAGER
PATHWAY = ANIMAL MEAT

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m²)/Ci				
H 3	3.55E+00	3.55E+00	0.00E-01	3.55E+00	3.55E+00	3.55E+00	3.55E+00	3.55E+00
C 14	1.80E+03	1.80E+03	9.00E+03	1.80E+03	1.80E+03	1.80E+03	1.80E+03	1.80E+03
AR 41	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 83m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 87	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE131m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	1.59E+02	2.68E+04	0.00E-01	0.00E-01	3.49E+01	8.84E+01	2.28E+02	0.00E-01
MN 54	3.52E+04	3.64E+05	0.00E-01	1.77E+05	5.29E+04	0.00E-01	0.00E-01	0.00E-01
FE 55	6.14E+05	1.13E+06	3.72E+06	2.64E+06	0.00E-01	0.00E-01	1.67E+06	0.00E-01
FE 59	5.20E+06	3.17E+07	5.77E+06	1.35E+07	0.00E-01	0.00E-01	4.25E+06	0.00E-01
CO 58	8.56E+05	5.10E+06	0.00E-01	3.71E+05	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	3.30E+06	1.91E+07	0.00E-01	1.46E+06	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 65	1.16E+07	1.05E+07	7.13E+06	2.47E+07	1.58E+07	0.00E-01	0.00E-01	0.00E-01
SR 89	2.12E+05	8.81E+05	7.38E+06	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	2.57E+07	2.93E+07	1.28E+09	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	8.62E+03	2.90E+07	3.99E+04	1.25E+04	1.84E+04	0.00E-01	0.00E-01	0.00E-01
SB124	1.68E+05	8.68E+06	4.31E+05	7.95E+03	0.00E-01	9.76E+02	3.77E+05	0.00E-01
CS134	1.44E+07	3.87E+05	1.32E+07	3.10E+07	9.86E+06	0.00E-01	3.77E+06	0.00E-01
CS136	7.48E+05	8.94E+04	2.83E+05	1.11E+06	6.05E+05	0.00E-01	9.54E+04	0.00E-01
CS137	8.94E+06	3.64E+05	1.93E+07	2.57E+07	8.75E+06	0.00E-01	3.39E+06	0.00E-01
BA140	4.63E+04	1.11E+06	7.19E+05	8.81E+02	2.99E+02	0.00E-01	5.93E+02	0.00E-01
CE141	2.52E+01	6.27E+05	3.30E+02	2.20E+02	1.03E+02	0.00E-01	0.00E-01	0.00E-01
CE144	1.65E+03	7.73E+06	3.08E+04	1.27E+04	7.61E+03	0.00E-01	0.00E-01	0.00E-01
I 131	1.05E+05	3.83E+04	1.39E+05	1.95E+05	3.36E+05	5.67E+07	0.00E-01	0.00E-01
I 133	2.54E-03	6.31E-03	4.91E-03	8.30E-03	1.46E-02	1.16E+00	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = TEENAGER
PATHWAY = VEGETABLES

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m^2)/Ci				
H 3	4.72E+01	4.72E+01	0.00E-01	4.72E+01	4.72E+01	4.72E+01	4.72E+01	4.72E+01
C 14	9.32E+03	9.32E+03	4.66E+04	9.32E+03	9.32E+03	9.32E+03	9.32E+03	9.32E+03
AR 41	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 83m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 87	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE131m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	1.93E+03	3.23E+05	0.00E-01	0.00E-01	4.21E+02	1.07E+03	2.75E+03	0.00E-01
MN 54	2.81E+06	2.91E+07	0.00E-01	1.42E+07	4.21E+06	0.00E-01	0.00E-01	0.00E-01
FE 55	5.92+05	1.10E+06	3.59E+06	2.54E+06	0.00E-01	0.00E-01	1.61E+06	0.00E-01
FE 59	5.01E+06	3.07E+07	5.58E+06	1.30E+07	0.00E-01	0.00E-01	4.09E+06	0.00E-01
CO 58	3.11E+06	1.86E+07	0.00E-01	1.35E+06	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	1.77E+07	1.02E+08	0.00E-01	7.86E+06	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 65	2.75E+07	2.50E+07	1.70E+07	5.89E+07	3.77E+07	0.00E-01	0.00E-01	0.00E-01
SR 89	1.39E+07	5.80E+07	4.88E+08	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	1.66E+09	1.90E+09	8.33E+10	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	1.15E+04	3.87E+07	5.32E+04	1.68E+04	2.47E+04	0.00E-01	0.00E-01	0.00E-01
SB124	1.87E+06	9.63E+07	4.79E+06	8.81E+04	0.00E-01	1.09E+04	4.18E+06	0.00E-01
CS134	2.45E+08	6.56E+06	2.24E+08	5.29E+08	1.68E+08	0.00E-01	6.40E+07	0.00E-01
CS136	3.64E+06	4.37E+05	1.38E+06	5.42E+06	2.95E+06	0.00E-01	4.66E+05	0.00E-01
CS137	1.70E+08	6.94E+06	3.68E+08	4.88E+08	1.66E+08	0.00E-01	6.43E+07	0.00E-01
BA140	2.82E+05	6.75E+06	4.37E+06	5.36E+03	1.82E+03	0.00E-01	3.61E+03	0.00E-01
CE141	6.78E+02	1.69E+07	8.84E+03	5.89E+03	2.78E+03	0.00E-01	0.00E-01	0.00E-01
CE144	8.62E+04	4.02E+08	1.60E+06	6.62E+05	3.96E+05	0.00E-01	0.00E-01	0.00E-01
I 131	9.16E+05	3.36E+05	1.22E+06	1.70E+06	2.93E+06	4.98E+08	0.00E-01	0.00E-01
I 133	1.58E+04	3.93E+04	3.06E+04	5.20E+04	9.13E+04	7.26E+06	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = TEENAGER
PATHWAY = PLUME

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m^3)/(Ci sec)				
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
AR 41	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	3.14E+02
KR 83m	1.68E-03	1.68E-03	1.68E-03	1.68E-03	1.68E-03	1.68E-03	9.38E-02	4.75E-01
KR 85m	2.60E+01	2.60E+01	2.60E+01	2.60E+01	2.60E+01	2.60E+01	2.66E+01	7.67E+01
KR 85	3.58E-01	3.58E-01	3.58E-01	3.58E-01	3.58E-01	3.58E-01	9.51E-01	4.28E+01
KR 87	1.32E+02	1.32E+02	1.32E+02	1.32E+02	1.32E+02	1.32E+02	1.35E+02	4.60E+02
KR 88	3.26E+02	3.26E+02	3.26E+02	3.26E+02	3.26E+02	3.26E+02	3.26E+02	4.50E+02
KR 89	3.68E+02	3.68E+02	3.68E+02	3.68E+02	3.68E+02	3.68E+02	3.71E+02	7.48E+02
KR 90	3.46E+02	3.46E+02	3.46E+02	3.46E+02	3.46E+02	3.46E+02	3.49E+02	6.33E+02
XE131m	2.03E+00	2.03E+00	2.03E+00	2.03E+00	2.03E+00	2.03E+00	2.38E+00	1.89E+01
XE133m	5.58E+00	5.58E+00	5.58E+00	5.58E+00	5.58E+00	5.58E+00	6.02E+00	3.96E+01
XE133	6.53E+00	6.53E+00	6.53E+00	6.53E+00	6.53E+00	6.53E+00	6.85E+00	1.84E+01
XE135m	6.91E+01	6.91E+01	6.91E+01	6.91E+01	6.91E+01	6.91E+01	6.94E+01	1.05E+02
XE135	4.02E+01	4.02E+01	4.02E+01	4.02E+01	4.02E+01	4.02E+01	4.09E+01	1.06E+02
XE137	3.15E+01	3.15E+01	3.15E+01	3.15E+01	3.15E+01	3.15E+01	3.55E+01	4.25E+02
XE138	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.97E+02	3.58E+02
CR 51	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MN 54	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 55	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 59	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 58	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 65	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SB124	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS134	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS136	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA140	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = CHILD
PATHWAY = INHALATION

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m^3)/(Ci sec)				
H 3	2.03E+01	2.03E+01	0.00E-01	2.03E+01	2.03E+01	2.03E+01	2.03E+01	2.03E+01
C 14	2.13E+02	2.13E+02	1.14E+03	2.13E+02	2.13E+02	2.13E+02	2.13E+02	0.00E-01
AR 41	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 83m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 87	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE131m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	4.88E+00	3.42E+01	0.00E-01	0.00E-01	7.70E-01	2.71E+00	5.39E+02	0.00E-01
MN 54	3.01E+02	7.26E+02	0.00E-01	1.36E+03	3.17E+02	0.00E-01	5.01E+04	0.00E-01
FE 55	2.46E+02	9.09E+02	1.50E+03	7.97E+02	0.00E-01	0.00E-01	3.52E+03	0.00E-01
FE 59	5.29E+02	2.24E+03	6.56E+02	1.06E+03	0.00E-01	0.00E-01	4.02E+04	0.00E-01
CO 58	1.00E+02	1.09E+03	0.00E-01	5.61E+01	0.00E-01	0.00E-01	3.52E+04	0.00E-01
CO 60	7.19E+02	3.05E+03	0.00E-01	4.15E+02	0.00E-01	0.00E-01	2.24E+05	0.00E-01
ZN 65	2.23E+03	5.17E+02	1.35E+03	3.58E+03	2.26E+03	0.00E-01	3.16E+04	0.00E-01
SR 89	5.45E+02	5.29E+03	1.90E+04	0.00E-01	0.00E-01	0.00E-01	6.85E+04	0.00E-01
SR 90	2.43E+04	1.09E+04	1.22E+06	0.00E-01	0.00E-01	0.00E-01	4.69E+05	0.00E-01
ZR 95	1.17E+03	1.94E+03	6.02E+03	1.32E+03	1.89E+03	0.00E-01	7.07E+04	0.00E-01
SB124	6.34E+02	5.20E+03	1.82E+03	2.35E+01	0.00E-01	3.99E+00	1.03E+05	0.00E-01
CS134	7.13E+03	1.22E+02	2.06E+04	3.20E+04	1.05E+04	0.00E-01	3.83E+03	0.00E-01
CS136	3.68E+03	1.32E+02	2.06E+03	5.42E+03	3.03E+03	0.00E-01	4.60E+02	0.00E-01
CS137	4.06E+03	1.15E+02	2.87E+04	2.61E+04	8.94E+03	0.00E-01	3.30E+03	0.00E-01
BA140	1.37E+02	3.23E+03	2.35E+03	2.05E+00	6.69E-01	0.00E-01	5.51E+04	0.00E-01
CE141	9.19E+01	1.79E+03	1.24E+03	6.18E+02	2.71E+02	0.00E-01	1.72E+04	0.00E-01
CE144	1.15E+04	1.23E+04	2.15E+05	6.72E+04	3.71E+04	0.00E-01	3.80E+05	0.00E-01
I 131	8.65E+02	9.00E+01	1.52E+03	1.52E+03	2.50E+03	5.17E+05	0.00E-01	0.00E-01
I 133	2.44E+02	1.74E+02	5.26E+02	6.43E+02	1.07E+03	1.22E+05	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = CHILD
PATHWAY = GROUND PLANE

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m²)/Ci				
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
AR 41	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 83m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 87	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE131m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	1.47E+05	1.47E+05	1.47E+05	1.47E+05	1.47E+05	1.47E+05	1.47E+05	1.74E+05
MN 54	4.37E+07	4.37E+07	4.37E+07	4.37E+07	4.37E+07	4.37E+07	4.37E+07	5.13E+07
FE 55	1.23E+06	3.52E+05	6.03E+06	1.98E+06	0.00E-01	0.00E-01	3.86E+06	0.00E-01
FE 59	8.65E+06	8.65E+06	8.65E+06	8.65E+06	8.65E+06	8.65E+06	8.65E+06	1.01E+07
CO 58	1.20E+07	1.20E+07	1.20E+07	1.20E+07	1.20E+07	1.20E+07	1.20E+07	1.41E+07
CO 60	6.81E+08	6.81E+08	6.81E+08	6.81E+08	6.81E+08	6.81E+08	6.81E+08	8.02E+08
ZN 65	2.36E+07	2.36E+07	2.36E+07	2.36E+07	2.36E+07	2.36E+07	2.36E+07	2.72E+07
SR 89	6.85E+02	6.85E+02	6.85E+02	6.85E+02	6.85E+02	6.85E+02	6.85E+02	7.95E+02
SR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	7.76E+06	7.76E+06	7.76E+06	7.76E+06	7.76E+06	7.76E+06	7.76E+06	9.03E+06
SB124	1.90E+07	1.90E+07	1.90E+07	1.90E+07	1.90E+07	1.90E+07	1.90E+07	2.19E+07
CS134	2.17E+08	2.17E+08	2.17E+08	2.17E+08	2.17E+08	2.17E+08	2.17E+08	2.53E+08
CS136	4.75E+06	4.75E+06	4.75E+06	4.75E+06	4.75E+06	4.75E+06	4.75E+06	5.39E+06
CS137	3.26E+08	3.26E+08	3.26E+08	3.26E+08	3.26E+08	3.26E+08	3.26E+08	3.80E+08
BA140	6.50E+05	6.50E+05	6.50E+05	6.50E+05	6.50E+05	6.50E+05	6.50E+05	7.45E+05
CE141	4.34E+05	4.34E+05	4.34E+05	4.34E+05	4.34E+05	4.34E+05	4.34E+05	4.88E+05
CE144	2.21E+06	2.21E+06	2.21E+06	2.21E+06	2.21E+06	2.21E+06	2.21E+06	2.55E+06
I 131	2.73E+05	2.73E+05	2.73E+05	2.73E+05	2.73E+05	2.73E+05	2.73E+05	3.33E+05
I 133	3.90E+04	3.90E+04	3.90E+04	3.90E+04	3.90E+04	3.90E+04	3.90E+04	4.72E+04

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = CHILD
PATHWAY = ANIMAL MEAT

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m^2)/Ci				
H 3	4.28E+00	4.28E+00	0.00E-01	4.28E+00	4.28E+00	4.28E+00	4.28E+00	4.28E+00
C 14	3.39E+03	3.39E+03	1.69E+04	3.39E+03	3.39E+03	3.39E+03	3.39E+03	3.39E+03
AR 41	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 83m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 87	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE131m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	2.48E+02	1.32E+04	0.00E-01	0.00E-01	3.77E+01	1.38E+02	2.52E+02	0.00E-01
MN 54	5.39E+04	1.70E+05	0.00E-01	2.02E+05	5.67E+04	0.00E-01	0.00E-01	0.00E-01
FE 55	1.17E+06	6.99E+05	7.11E+06	3.77E+06	0.00E-01	0.00E-01	2.14E+06	0.00E-01
FE 59	8.24E+06	1.72E+07	1.02E+07	1.65E+07	0.00E-01	0.00E-01	4.79E+06	0.00E-01
CO 58	1.32E+06	2.53E+06	0.00E-01	4.34E+05	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	5.13E+06	9.63E+06	0.00E-01	1.74E+06	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 65	1.77E+07	5.01E+06	1.07E+07	2.85E+07	1.80E+07	0.00E-01	0.00E-01	0.00E-01
SR 89	3.99E+05	5.42E+05	1.40E+07	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	4.09E+07	1.82E+07	2.03E+09	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	1.38E+04	1.62E+07	7.07E+04	1.55E+04	2.22E+04	0.00E-01	0.00E-01	0.00E-01
SB124	2.73E+05	4.88E+06	7.80E+05	1.01E+04	0.00E-01	1.72E+03	4.34E+05	0.00E-01
CS134	8.05E+06	2.06E+05	2.33E+07	3.80E+07	1.18E+07	0.00E-01	4.25E+06	0.00E-01
CS136	8.68E+05	4.72E+04	4.88E+05	1.34E+06	7.13E+05	0.00E-01	1.06E+05	0.00E-01
CS137	5.01E+06	2.13E+05	3.55E+07	3.39E+07	1.11E+07	0.00E-01	3.99E+06	0.00E-01
BA140	7.76E+04	6.72E+05	1.33E+06	1.16E+03	3.80E+02	0.00E-01	6.94E+02	0.00E-01
CE141	4.60E+01	3.87E+05	6.21E+02	3.09E+02	1.36E+02	0.00E-01	0.00E-01	0.00E-01
CE144	3.10E+03	4.75E+06	5.80E+04	1.82E+04	1.01E+04	0.00E-01	0.00E-01	0.00E-01
I 131	1.47E+05	2.31E+04	2.58E+05	2.59E+05	4.25E+05	8.56E+07	0.00E-01	0.00E-01
I 133	4.25E-03	4.53E-03	9.10E-03	1.12E-02	1.88E-02	2.09E+00	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = CHILD
PATHWAY = VEGETABLES

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m^2)/Ci				
H 3	7.32E+01	7.32E+01	0.00E-01	7.32E+01	7.32E+01	7.32E+01	7.32E+01	7.32E+01
C 14	2.24E+04	2.24E+04	1.12E+05	2.24E+04	2.24E+04	2.24E+04	2.24E+04	2.24E+04
AR 41	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 83m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 87	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE131m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	3.68E+03	1.94E+05	0.00E-01	0.00E-01	5.55E+02	2.03E+03	3.71E+03	0.00E-01
MN 54	5.51E+06	1.74E+07	0.00E-01	2.07E+07	5.80E+06	0.00E-01	0.00E-01	0.00E-01
FE 55	1.41E+06	8.45E+05	8.57E+06	4.57E+06	0.00E-01	0.00E-01	2.59E+06	0.00E-01
FE 59	9.95E+06	2.08E+07	1.23E+07	2.00E+07	0.00E-01	0.00E-01	5.80E+06	0.00E-01
CO 58	6.08E+06	1.16E+07	0.00E-01	1.99E+06	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	3.52E+07	6.62E+07	0.00E-01	1.19E+07	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 65	5.39E+07	1.52E+07	3.26E+07	8.68E+07	5.45E+07	0.00E-01	0.00E-01	0.00E-01
SR 89	3.30E+07	4.47E+07	1.16E+09	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	3.39E+09	1.51E+09	1.69E+11	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	2.33E+04	2.73E+07	1.19E+05	2.62E+04	3.74E+04	0.00E-01	0.00E-01	0.00E-01
SB124	3.83E+06	6.81E+07	1.09E+07	1.42E+05	0.00E-01	2.41E+04	6.05E+06	0.00E-01
CS134	1.75E+08	4.47E+06	5.07E+08	8.30E+08	2.58E+08	0.00E-01	9.25E+07	0.00E-01
CS136	4.63E+06	2.50E+05	2.59E+06	7.13E+06	3.80E+06	0.00E-01	5.67E+05	0.00E-01
CS137	1.22E+08	5.20E+06	8.65E+08	8.27E+08	2.70E+08	0.00E-01	9.70E+07	0.00E-01
BA140	5.10E+05	4.44E+06	8.78E+06	7.67E+03	2.50E+03	0.00E-01	4.56E+03	0.00E-01
CE141	1.52E+03	1.27E+07	2.05E+04	1.02E+04	4.47E+03	0.00E-01	0.00E-01	0.00E-01
CE144	2.06E+05	3.16E+08	3.87E+06	1.21E+06	6.69E+05	0.00E-01	0.00E-01	0.00E-01
I 131	1.29E+06	2.03E+05	2.27E+06	2.28E+06	3.74E+06	7.54E+08	0.00E-01	0.00E-01
I 133	2.61E+04	2.78E+04	5.58E+04	6.91E+04	1.15E+05	1.28E+07	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = CHILD
PATHWAY = PLUME

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m^3)/(Ci sec)				
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
AR 41	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	3.14E+02
KR 83m	1.68E-03	1.68E-03	1.68E-03	1.68E-03	1.68E-03	1.68E-03	9.38E-02	4.75E-01
KR 85m	2.60E+01	2.60E+01	2.60E+01	2.60E+01	2.60E+01	2.60E+01	2.66E+01	7.67E+01
KR 85	3.58E-01	3.58E-01	3.58E-01	3.58E-01	3.58E-01	3.58E-01	9.51E-01	4.28E+01
KR 87	1.32E+02	1.32E+02	1.32E+02	1.32E+02	1.32E+02	1.32E+02	1.35E+02	4.60E+02
KR 88	3.26E+02	3.26E+02	3.26E+02	3.26E+02	3.26E+02	3.26E+02	3.26E+02	4.50E+02
KR 89	3.68E+02	3.68E+02	3.68E+02	3.68E+02	3.68E+02	3.68E+02	3.71E+02	7.48E+02
KR 90	3.46E+02	3.46E+02	3.46E+02	3.46E+02	3.46E+02	3.46E+02	3.49E+02	6.33E+02
XE131m	2.03E+00	2.03E+00	2.03E+00	2.03E+00	2.03E+00	2.03E+00	2.38E+00	1.89E+01
XE133m	5.58E+00	5.58E+00	5.58E+00	5.58E+00	5.58E+00	5.58E+00	6.02E+00	3.96E+01
XE133	6.53E+00	6.53E+00	6.53E+00	6.53E+00	6.53E+00	6.53E+00	6.85E+00	1.84E+01
XE135m	6.91E+01	6.91E+01	6.91E+01	6.91E+01	6.91E+01	6.91E+01	6.94E+01	1.05E+02
XE135	4.02E+01	4.02E+01	4.02E+01	4.02E+01	4.02E+01	4.02E+01	4.09E+01	1.06E+02
XE137	3.15E+01	3.15E+01	3.15E+01	3.15E+01	3.15E+01	3.15E+01	3.55E+01	4.25E+02
XE138	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.97E+02	3.58E+02
CR 51	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MN 54	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 55	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 59	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 58	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 65	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SB124	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS134	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS136	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA140	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = INFANT
PATHWAY = INHALATION

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m^3)/(Ci sec)				
H 3	1.17E+01	1.17E+01	0.00E-01	1.17E+01	1.17E+01	1.17E+01	1.17E+01	1.17E+01
C 14	1.68E+02	1.68E+02	8.39E+02	1.68E+02	1.68E+02	1.68E+02	1.68E+02	0.00E-01
AR 41	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 83m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 87	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE131m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	2.84E+00	1.13E+01	0.00E-01	0.00E-01	4.18E-01	1.83E+00	4.06E+02	0.00E-01
MN 54	1.58E+02	2.24E+02	0.00E-01	8.05E+02	1.58E+02	0.00E-01	3.17E+04	0.00E-01
FE 55	1.05E+02	3.47E+01	6.27E+02	3.72E+02	0.00E-01	0.00E-01	2.77E+03	0.00E-01
FE 59	3.00E+02	7.86E+02	4.31E+02	7.45E+02	0.00E-01	0.00E-01	3.23E+04	0.00E-01
CO 58	5.77E+01	3.52E+02	0.00E-01	3.87E+01	0.00E-01	0.00E-01	2.46E+04	0.00E-01
CO 60	3.74E+02	1.01E+03	0.00E-01	2.54E+02	0.00E-01	0.00E-01	1.43E+05	0.00E-01
ZN 65	9.86E+02	1.63E+03	6.12E+02	1.98E+03	1.03E+03	0.00E-01	2.05E+04	0.00E-01
SR 89	3.61E+02	2.03E+03	1.26E+04	0.00E-01	0.00E-01	0.00E-01	6.43E+04	0.00E-01
SR 90	9.89E+03	4.15E+03	4.91E+05	0.00E-01	0.00E-01	0.00E-01	3.55E+05	0.00E-01
ZR 95	6.43E+02	6.88E+02	3.64E+03	8.84E+02	9.86E+02	0.00E-01	5.55E+04	0.00E-01
SB124	3.80E+02	1.87E+03	1.20E+03	1.76E+01	0.00E-01	3.20E+00	8.40E+04	0.00E-01
CS134	2.36E+03	4.21E+01	1.25E+04	2.23E+04	6.02E+03	0.00E-01	2.53E+03	0.00E-01
CS136	1.68E+03	4.53E+01	1.53E+03	4.28E+03	1.79E+03	0.00E-01	3.74E+02	0.00E-01
CS137	1.44E+03	4.21E+01	1.74E+04	1.94E+04	5.45E+03	0.00E-01	2.26E+03	0.00E-01
BA140	9.19E+01	1.22E+03	1.77E+03	1.77E+00	4.25E-01	0.00E-01	5.07E+04	0.00E-01
CE141	6.31E+01	6.85E+02	8.78E+02	5.29E+02	1.66E+02	0.00E-01	1.64E+04	0.00E-01
CE144	5.58E+03	4.69E+03	1.01E+05	3.83E+04	1.70E+04	0.00E-01	3.12E+05	0.00E-01
I 131	6.21E+02	3.36E+01	1.20E+03	1.41E+03	1.64E+03	4.69E+05	0.00E-01	0.00E-01
I 133	1.77E+02	6.85E+01	4.18E+02	6.08E+02	7.10E+02	1.13E+05	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = INFANT
PATHWAY = GROUND PLANE

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m^2)/Ci				
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
AR 41	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 83m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 87	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE131m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	1.47E+05	1.47E+05	1.47E+05	1.47E+05	1.47E+05	1.47E+05	1.47E+05	1.74E+05
MN 54	4.37E+07	4.37E+07	4.37E+07	4.37E+07	4.37E+07	4.37E+07	4.37E+07	5.13E+07
FE 55	9.79E+05	3.84E+05	3.90E+06	1.44E+06	0.00E-01	0.00E-01	2.39E+06	0.00E-01
FE 59	8.65E+06	8.65E+06	8.65E+06	8.65E+06	8.65E+06	8.65E+06	8.65E+06	1.01E+07
CO 58	1.20E+07	1.20E+07	1.20E+07	1.20E+07	1.20E+07	1.20E+07	1.20E+07	1.41E+07
CO 60	6.81E+08	6.81E+08	6.81E+08	6.81E+08	6.81E+08	6.81E+08	6.81E+08	8.02E+08
ZN 65	2.36E+07	2.36E+07	2.36E+07	2.36E+07	2.36E+07	2.36E+07	2.36E+07	2.72E+07
SR 89	6.85E+02	6.85E+02	6.85E+02	6.85E+02	6.85E+02	6.85E+02	6.85E+02	7.95E+02
SR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	7.76E+06	7.76E+06	7.76E+06	7.76E+06	7.76E+06	7.76E+06	7.76E+06	9.03E+06
SB124	1.90E+07	1.90E+07	1.90E+07	1.90E+07	1.90E+07	1.90E+07	1.90E+07	2.19E+07
CS134	2.17E+08	2.17E+08	2.17E+08	2.17E+08	2.17E+08	2.17E+08	2.17E+08	2.53E+08
CS136	4.75E+06	4.75E+06	4.75E+06	4.75E+06	4.75E+06	4.75E+06	4.75E+06	5.39E+06
CS137	3.26E+08	3.26E+08	3.26E+08	3.26E+08	3.26E+08	3.26E+08	3.26E+08	3.80E+08
BA140	6.50E+05	6.50E+05	6.50E+05	6.50E+05	6.50E+05	6.50E+05	6.50E+05	7.45E+05
CE141	4.34E+05	4.34E+05	4.34E+05	4.34E+05	4.34E+05	4.34E+05	4.34E+05	4.88E+05
CE144	2.21E+06	2.21E+06	2.21E+06	2.21E+06	2.21E+06	2.21E+06	2.21E+06	2.55E+06
I 131	2.73E+05	2.73E+05	2.73E+05	2.73E+05	2.73E+05	2.73E+05	2.73E+05	3.33E+05
I 133	3.90E+04	3.90E+04	3.90E+04	3.90E+04	3.90E+04	3.90E+04	3.90E+04	4.72E+04

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = INFANT
PATHWAY = ANIMAL MEAT

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m²)/Ci				
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
AR 41	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 83m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 87	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE131m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MN 54	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 55	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 59	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 58	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 65	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SB124	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS134	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS136	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA140	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = INFANT
PATHWAY = VEGETABLES

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m²)/Ci				
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
AR 41	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 83m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 87	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE131m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MN 54	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 55	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 59	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 58	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 65	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SB124	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS134	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS136	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA140	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = INFANT
PATHWAY = PLUME

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m^3)/(Ci sec)				
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
AR 41	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	3.14E+02
KR 83m	1.68E-03	1.68E-03	1.68E-03	1.68E-03	1.68E-03	1.68E-03	9.38E-02	4.75E-01
KR 85m	2.60E+01	2.60E+01	2.60E+01	2.60E+01	2.60E+01	2.60E+01	2.66E+01	7.67E+01
KR 85	3.58E-01	3.58E-01	3.58E-01	3.58E-01	3.58E-01	3.58E-01	9.51E-01	4.28E+01
KR 87	1.32E+02	1.32E+02	1.32E+02	1.32E+02	1.32E+02	1.32E+02	1.35E+02	4.60E+02
KR 88	3.26E+02	3.26E+02	3.26E+02	3.26E+02	3.26E+02	3.26E+02	3.26E+02	4.50E+02
KR 89	3.68E+02	3.68E+02	3.68E+02	3.68E+02	3.68E+02	3.68E+02	3.71E+02	7.48E+02
KR 90	3.46E+02	3.46E+02	3.46E+02	3.46E+02	3.46E+02	3.46E+02	3.49E+02	6.33E+02
XE131m	2.03E+00	2.03E+00	2.03E+00	2.03E+00	2.03E+00	2.03E+00	2.38E+00	1.89E+01
XE133m	5.58E+00	5.58E+00	5.58E+00	5.58E+00	5.58E+00	5.58E+00	6.02E+00	3.96E+01
XE133	6.53E+00	6.53E+00	6.53E+00	6.53E+00	6.53E+00	6.53E+00	6.85E+00	1.84E+01
XE135m	6.91E+01	6.91E+01	6.91E+01	6.91E+01	6.91E+01	6.91E+01	6.94E+01	1.05E+02
XE135	4.02E+01	4.02E+01	4.02E+01	4.02E+01	4.02E+01	4.02E+01	4.09E+01	1.06E+02
XE137	3.15E+01	3.15E+01	3.15E+01	3.15E+01	3.15E+01	3.15E+01	3.55E+01	4.25E+02
XE138	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.97E+02	3.58E+02
CR 51	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MN 54	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 55	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 59	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 58	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 65	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SB124	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS134	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS136	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA140	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

APPENDIX B: TECHNICAL BASES FOR EFFECTIVE DOSE FACTORS

Overview

The evaluation of doses due to releases of radioactive material to the atmosphere can be simplified by the use of effective dose transfer factors instead of using dose factors which are radionuclide specific. These effective factors, which are based on the total radioactivity released to approximate the dose in the environment, i.e., instead of having to sum the isotopic distribution multiplied by the isotope specific dose factor only a single multiplication times the total quantity of radioactive material released would be needed. This approach provides a reasonable estimate of the actual dose while eliminating the need for a detailed calculational technique.

Determination of Effective Dose Factors

The effective dose transfer factors are based on past operating data. The radioactive effluent distribution for the past years can be used to derive single effective factors by the following equations:

$$A\gamma_{s\,eff} = \sum_i A\gamma_i \ x \ f_i$$

where

$A\gamma_{s\,eff}$ = the effective gamma-air dose factor due to stack releases of noble gases (mrad/μCi)

$A\gamma_i$ = the gamma-air dose factor due to stack releases of each noble gas radionuclide i (mrad/μCi)

f_i = the fraction of noble gas radioactivity constituted by radionuclide i

$$A\gamma_{v\,eff} = \sum_i A\gamma_{vi} \ x \ f_i$$

where

$A\gamma_{v\,eff}$ = the effective gamma-air dose factor due to vent releases of all noble gases $\left(\frac{mrad}{\mu Ci \ \sec/m^3}\right)$

$A\gamma_{vi}$ = the gamma-air dose factor due to vent releases of each noble gas radionuclide i $\left(\frac{mrad}{\mu Ci \ \sec/m^3}\right)$

$$A\beta_{v\,eff} = \sum_i A\beta_i \ x \ f_i$$

where

$A\beta_{eff}$ = the effective beta-air dose factor due to either vent or stack releases of all noble gases $\left(\frac{mrad}{\mu Ci\ \sec/m^3}\right)$

$A\beta_i$ = the beta air dose factor due to either vent or stack releases of each noble gas radionuclide *i* $\left(\frac{mrad}{\mu Ci\ \sec/m^3}\right)$

To determine the appropriate effective factors to be used and to evaluate the degree of variability, the atmospheric radioactive effluents for the past 3 years have been evaluated.

Table B-1 presents the radionuclide distribution for stack and vent releases as measured by isotopic analysis of periodic grab samples from the respective effluent release points. Table B-2 presents the effective dose factors (gamma-air and beta-air) derived on the basis of the radionuclide distribution.

Except for the year 1981, the variability of the effective factors is minor. For 1981, Xe-138 contributes significantly to the derivation of the effective factors for stack releases. The Xe-138 contribution for the years 1979 and 1980 is not so significant. This increase in Xe-138 from 1981 results in a larger variability of the yearly values from the average than what is considered typical. Therefore, in order to assure adequate conservatism, the effective dose factors for stack releases will be based on the radionuclide distribution for the year 1981. Because this is considered an atypical distribution resulting in higher doses, use of the data will provide dose estimates which are conservative. As more data become available to further establish a typical radionuclide distribution, the effective dose factors for stack releases may be reevaluated.

To provide an additional degree of conservatism, a factor of 0.8 is introduced into the dose calculational process when the effective dose transfer factor is used. This added conservatism provides additional assurance that the evaluation of doses by the use of a single effective factor will not significantly underestimate any actual doses in the environment.

By evaluating doses using these effective dose factors, maximum allowable releases of noble gases for any calendar quarter may be determined. As discussed in Section 3.6.1, the maximum allowable releases based on the gamma-air effective dose factor have been determined to be 250,000 Ci/quarter for stack releases and 12,700 Ci/quarter for vent releases.

For the beta air effective dose factors, the releases of noble gases corresponding to the quarterly limit of 10 mrads corresponds to 307,000 Ci/quarter for stack releases and 29,600 Ci/quarter for vent releases. Comparing these values for allowable releases with the values based on the gamma-air effective dose factors, it is demonstrated that the gamma-air doses are more restrictive than the beta-air doses. In other words, the doses calculated by using the gamma-air effective dose factors represent a larger fraction of the allowable dose than does the dose calculated by using the beta-air effective dose factors. Therefore, when using the effective dose factors for evaluating compliance with the quarterly dose limits of Section 6.2.3, only the gamma-air dose need be evaluated; compliance with the gamma-air dose limit represents a de facto compliance with the beta-air dose limit.

Reevaluation

The doses due to the gaseous effluents are evaluated by the more detailed calculational methods (i.e., use of nuclide specific dose factors) on a yearly basis. At that time, a comparison can be made between the simplified method and the detailed method to assure the overall reasonableness of this limited analysis approach. If the comparison indicates that the radionuclide distribution has changed significantly, thereby causing the simplified method to underestimate the doses, the value of the effective factors will need to be reexamined to assure the overall acceptability of this approach. However, this reexamination will only be needed if the doses as calculated by the detailed analysis exceed 50% of the design bases doses (i.e., greater than 50% of the 10 mrad gamma air dose or 20 mrad beta air dose).

Table B-1
RADIONUCLIDE DISTRIBUTION OF STACK AND VENT RELEASES

Radionuclide	Fraction of Total Releases					
	Stack			Vent		
	1979	1980	1981	1979	1980	1981
Kr-85m	.11	.05	.09	.02	---	---
Kr-87	.01	---	.02	---	.01	---
Kr-88	.07	.04	.08	---	---	---
Xe-133	.76	.82	.45	.24	.24	.14
Xe-135	.01	.02	.03	.72	.50	.59
Xe-135m	---	.02	.08	.02	.22	.21
Xe-138	.02	.06	.25	---	.03	.05

Table B-2
EFFECTIVE DOSE FACTORS NOBLE GASES - AIR DOSES

Year	Stack Releases		Vent Releases	
	Gamma-Air Effective Dose Factor $A\gamma_{seff}$ $\left(\frac{mrad}{\mu Ci}\right)$	Beta-Air Effective Dose Factor $A\beta_{seff}$ $\left(\frac{mrad}{\mu Ci\ \sec/m^3}\right)$	Gamma-Air Effective Dose Factor $A\gamma_{veff}$ $\left(\frac{mrad}{\mu Ci\ \sec/m^3}\right)$	Beta-Air Effective Dose Factor $A\beta_{veff}$ $\left(\frac{mrad}{\mu Ci\ \sec/m^3}\right)$
1979	$7.0\text{x}10^{-12}$	$5.9\text{x}10^{-5}$	$5.0\text{x}10^{-5}$	$6.5\text{x}10^{-5}$
1980	$6.7\text{x}10^{-12}$	$5.3\text{x}10^{-5}$	$6.7\text{x}10^{-5}$	$6.0\text{x}10^{-5}$
1981	$1.6\text{x}10^{-11}$	$9.3\text{x}10^{-12}$	$6.4\text{x}10^{-5}$	$6.3\text{x}10^{-5}$
Average	$9.9\text{x}10^{-12}$	$6.8\text{x}10^{-5}$	$6.4\text{x}10^{-5}$	$6.3\text{x}10^{-5}$

APPENDIX C: DOSE TRANSFER FACTORS FOR WATERBORNE PATHWAYS

Dose transfer factors for waterborne effluent have been derived by solving environmental pathway models on the bases of unit radionuclide release in effluent (1 Ci/yr) discharged in 1 gallon/minute of water. The dose transfer factors in this appendix were computed with the LADTAP II computer program, using default values of parameters applicable to the most exposed members of the public as recommended in Regulatory 1.109, revision 1, with the following exceptions:

- In order to account for significant revisions of data since publication of the Regulatory Guide, data differing from those in Regulatory Guide 1.109, revision 1 are identified in LADTAP II documentation.[15]
- After publishing Reg. Guide 1.109, the NRC recommended that soil-to-plant bioaccumulation factors, B_{iv}, of cesium and strontium be changed.[16]
- The revised values were used to derive dose transfer factors tabulated for Sr89, Sr90, and Cs137 in irrigated vegetation.
- Values of environmental transit time recommended in Reg. Guide 1.109[17], namely 1440 hr from harvest of stored vegetables to ingestion, were retained.
- LADTAP II divergence from Reg. Guide 1.109 is reflected in tritium dose transfer factors that are typically 43% lower than those described in the Reg. Guide.

Dose transfer factors are included hereafter for the following parameters. Only those pathways applicable at the time of a radioactive liquid effluent release will be used for dose calculations. Likely pathways would include potable water, freshwater fish and irrigated fresh leafy vegetables (including strawberries).

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15 Strenge, D.L., et. al., LADTAP II - Technical Reference and User guide, NUREG/CR-4013, April 1986.

16 NRC, SECY-79-653A, January 30, 1980.

17 Regulatory Guide 1.109, rev. 1, Table E-15.

Pathway	Age Group	Organ
Potable water	Adult	Total Body
Freshwater fish	Teenager	GI tract
Animal drinks river water-milk		
Animal drinks river water-meat		
River shoreline deposits-irradiation		
Swimming		
Boating		

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = ADULT
PATHWAY = POTABLE WATER

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	2.19E+01	2.19E+01	0.00E-01	2.19E+01	2.19E+01	2.19E+01	2.19E+01	0.00E-01
C 14	2.08E+02	2.08E+02	1.04E+03	2.08E+02	2.08E+02	2.08E+02	2.08E+02	0.00E-01
NA 24	2.06E+02	2.06E+02	2.06E+02	2.06E+02	2.06E+02	2.06E+02	2.06E+02	0.00E-01
P 32	2.60E+03	7.57E+03	6.73E+04	4.19E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	9.50E-01	2.39E+02	0.00E-01	0.00E-01	2.09E-01	5.68E-01	1.26E+00	0.00E-01
MN 54	3.19E+02	5.12E+03	0.00E-01	1.67E+03	4.97E+02	0.00E-01	0.00E-01	0.00E-01
MN 56	1.15E-02	2.06E+00	0.00E-01	6.46E-02	8.20E-02	0.00E-01	0.00E-01	0.00E-01
FE 55	1.62E+02	3.99E+02	1.01E+03	6.95E+02	0.00E-01	0.00E-01	3.88E+02	0.00E-01
FE 59	1.41E+03	1.23E+04	1.56E+03	3.68E+03	0.00E-01	0.00E-01	1.03E+03	0.00E-01
CO 58	6.06E+02	5.48E+03	0.00E-01	2.70E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	1.73E+03	1.47E+04	0.00E-01	7.84E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 63	1.60E+03	6.89E+02	4.76E+04	3.30E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	1.56E-02	8.66E-01	2.63E-01	3.42E-02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CU 64	3.85E+00	6.99E+02	0.00E-01	8.21E+00	2.07E+01	0.00E-01	0.00E-01	0.00E-01
ZN 65	2.54E+03	3.54E+03	1.77E+03	5.62E+03	3.76E+03	0.00E-01	0.00E-01	0.00E-01
ZN 69	1.21E-08	2.62E-08	9.11E-08	1.74E-07	1.13E-07	0.00E-01	0.00E-01	0.00E-01
BR 83	1.40E-02	2.02E-02	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 84	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 86	3.47E+03	1.47E+03	0.00E-01	7.45E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	3.19E+03	1.78E+04	1.11E+05	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	6.41E+04	8.02E+04	3.19E+06	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 91	1.45E+01	1.71E+03	3.59E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 92	7.38E-02	3.38E+01	1.71E+00	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 90	7.29E-02	2.88E+04	2.72E+00	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91	1.37E+00	2.81E+04	5.10E+01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 92	8.37E-05	5.02E+01	2.86E-03	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 93	5.20E-03	5.98E+03	1.89E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	2.39E+00	1.12E+04	1.10E+01	3.53E+00	5.54E+00	0.00E-01	0.00E-01	0.00E-01
ZR 97	2.12E-02	1.44E+04	2.30E-01	4.64E-02	7.00E-02	0.00E-01	0.00E-01	0.00E-01
NB 95	6.68E-01	7.54E+03	2.23E+00	1.24E+00	1.23E+00	0.00E-01	0.00E-01	0.00E-01
MO 99	2.33E+02	2.84E+03	0.00E-01	1.23E+03	2.78E+03	0.00E-01	0.00E-01	0.00E-01
TC 99M	2.05E-01	9.53E+00	5.70E-03	1.61E-02	2.45E-01	0.00E-01	7.89E-03	0.00E-01
TC101	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = ADULT
PATHWAY = POTABLE WATER

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
RU103	2.87E+01	7.77E+03	6.66E+01	0.00E-01	2.54E+02	0.00E-01	0.00E-01	0.00E-01
RU105	5.24E-02	8.12E+01	1.33E-01	0.00E-01	1.72E+00	0.00E-01	0.00E-01	0.00E-01
RU106	1.27E+02	6.51E+04	1.00E+03	0.00E-01	1.94E+03	0.00E-01	0.00E-01	0.00E-01
AG110M	3.21E+01	2.21E+04	5.85E+01	5.41E+01	1.06E+02	0.00E-01	0.00E-01	0.00E-01
TE125M	1.30E+02	3.87E+03	9.70E+02	3.51E+02	3.94E+03	2.92E+02	0.00E-01	0.00E-01
TE127M	3.00E+02	8.26E+03	2.46E+03	8.81E+02	1.00E+04	6.30E+02	0.00E-01	0.00E-01
TE127	1.47E+00	5.36E+02	6.80E+00	2.44E+00	2.77E+01	5.03E+00	0.00E-01	0.00E-01
TE129M	6.53E+02	2.08E+04	4.13E+03	1.54E+03	1.72E+04	1.42E+03	0.00E-01	0.00E-01
TE129	1.66E-06	5.13E-06	6.79E-06	2.55E-06	2.85E-05	5.21E-06	0.00E-01	0.00E-01
TE131M	1.48E+02	1.77E+04	3.64E+02	1.78E+02	1.80E+03	2.82E+02	0.00E-01	0.00E-01
TE131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE132	4.53E+02	2.28E+04	7.46E+02	4.83E+02	4.65E+03	5.33E+02	0.00E-01	0.00E-01
I 130	8.37E+01	1.83E+02	7.19E+01	2.12E+02	3.31E+02	1.80E+04	0.00E-01	0.00E-01
I 131	1.15E+03	5.28E+02	1.40E+03	2.00E+03	3.43E+03	6.55E+05	0.00E-01	0.00E-01
I 132	5.03E-02	2.70E-02	5.38E-02	1.44E-01	2.29E-01	5.03E+00	0.00E-01	0.00E-01
I 133	1.24E+02	3.66E+02	2.34E+02	4.07E+02	7.10E+02	5.98E+04	0.00E-01	0.00E-01
I 134	2.10E-07	5.11E-10	2.16E-07	5.86E-07	9.32E-07	1.02E-05	0.00E-01	0.00E-01
I 135	1.26E+01	3.85E+01	1.30E+01	3.41E+01	5.47E+01	2.25E+03	0.00E-01	0.00E-01
CS134	4.43E+04	9.48E+02	2.28E+04	5.42E+04	1.75E+04	0.00E-01	5.82E+03	0.00E-01
CS136	6.43E+03	1.01E+03	2.26E+03	8.93E+03	4.97E+03	0.00E-01	6.81E+02	0.00E-01
CS137	2.61E+04	7.73E+02	2.92E+04	3.99E+04	1.35E+04	0.00E-01	4.50E+03	0.00E-01
CS138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA139	5.81E-06	3.52E-04	1.98E-04	1.41E-07	1.32E-07	0.00E-01	8.02E-08	0.00E-01
BA140	4.61E+02	1.45E+04	7.04E+03	8.85E+00	3.01E+00	0.00E-01	5.07E+00	0.00E-01
BA141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA142	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA140	8.07E-02	2.24E+04	6.06E-01	3.05E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA142	1.08E-07	3.18E-03	9.57E-07	4.35E-07	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	2.57E-01	8.68E+03	3.36E+00	2.27E+00	1.05E+00	0.00E-01	0.00E-01	0.00E-01
CE143	2.99E-02	1.01E+04	3.66E-01	2.70E+02	1.19E-01	0.00E-01	0.00E-01	0.00E-01
CE144	9.57E+00	6.03E+04	1.78E+02	7.46E+01	4.42E+01	0.00E-01	0.00E-01	0.00E-01
PR143	1.59E-01	1.40E+04	3.20E+00	1.28E+00	7.42E-01	0.00E-01	0.00E-01	0.00E-01
PR144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ND147	1.50E-01	1.20E+04	2.16E+00	2.50E+00	1.46E+00	0.00E-01	0.00E-01	0.00E-01
W 187	5.50E+00	5.15E+03	1.88E+01	1.57E+01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NP239	1.76E-02	6.55E+03	3.25E-01	3.20E-02	9.97E-02	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = ADULT
PATHWAY = FRESH WATER FISH

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	5.68E-01	5.68E-01	0.00E-01	5.68E-01	5.68E-01	5.68E-01	5.68E-01	0.00E-01
C 14	2.75E+04	2.75E+04	1.38E+05	2.75E+04	2.75E+04	2.75E+04	2.75E+04	0.00E-01
NA 24	5.90E+02	5.90E+02	5.90E+02	5.90E+02	5.90E+02	5.90E+02	5.90E+02	0.00E-01
P 32	7.49E+06	2.18E+07	1.94E+08	1.20E+07	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	5.47E+00	1.37E+03	0.00E-01	0.00E-01	1.20E+00	3.27E+00	7.26E+00	0.00E-01
MN 54	3.67E+03	5.89E+04	0.00E-01	1.92E+04	5.72E+03	0.00E-01	0.00E-01	0.00E-01
MN 56	1.28E-01	2.31E+01	0.00E-01	7.24E-01	9.19E-01	0.00E-01	0.00E-01	0.00E-01
FE 55	4.66E+02	1.15E+03	2.90E+03	2.00E+03	0.00E-01	0.00E-01	1.12E+03	0.00E-01
FE 59	4.06E+03	3.53E+04	4.50E+03	1.06E+04	0.00E-01	0.00E-01	2.96E+03	0.00E-01
CO 58	8.71E+02	7.88E+03	0.00E-01	3.89E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	2.49E+03	2.12E+04	0.00E-01	1.13E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 63	4.59E+03	1.98E+03	1.37E+05	9.49E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	4.36E-02	2.42E+00	7.36E-01	9.56E-02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CU 64	5.51E+00	1.00E+03	0.00E-01	1.17E+01	2.96E+01	0.00E-01	0.00E-01	0.00E-01
ZN 65	1.46E+05	2.04E+05	1.02E+05	3.24E+05	2.16E+05	0.00E-01	0.00E-01	0.00E-01
ZN 69	6.48E-07	1.40E-06	4.87E-06	9.32E-06	6.05E-06	0.00E-01	0.00E-01	0.00E-01
BR 83	1.65E-01	2.37E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 84	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 86	2.00E+05	8.45E+04	0.00E-01	4.28E+05	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	2.76E+03	1.54E+04	9.60E+04	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	5.53E+04	6.92E+04	2.75E+06	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 91	1.24E+01	1.47E+03	3.08E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 92	6.21E-02	2.85E+01	1.44E+00	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 90	5.24E-02	2.07E+04	1.95E+00	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91	9.81E-01	2.02E+04	3.67E+01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 92	5.90E-05	3.54E+01	2.02E-03	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 93	3.72E-03	4.27E+03	1.35E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	2.27E-01	1.06E+03	1.05E+00	3.35E-01	5.26E-01	0.00E-01	0.00E-01	0.00E-01
ZR 97	2.00E-03	1.36E+03	2.17E-02	4.38E-03	6.62E-03	0.00E-01	0.00E-01	0.00E-01
NB 95	5.76E+02	6.51E+06	1.93E+03	1.07E+03	1.06E+03	0.00E-01	0.00E-01	0.00E-01
MO 99	6.71E+01	8.17E+02	0.00E-01	3.52E+02	7.98E+02	0.00E-01	0.00E-01	0.00E-01
TC 99M	8.75E-02	4.07E+00	2.43E-03	6.87E-03	1.04E-01	0.00E-01	3.37E-03	0.00E-01
TC101	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = ADULT
PATHWAY = FRESH WATER FISH

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
RU103	8.25E+00	2.24E+03	1.91E+01	0.00E-01	7.31E+01	0.00E-01	0.00E-01	0.00E-01
RU105	1.48E-02	2.30E+01	3.76E-02	0.00E-01	4.86E-01	0.00E-01	0.00E-01	0.00E-01
RU106	3.66E+01	1.87E+04	2.89E+02	0.00E-01	5.58E+02	0.00E-01	0.00E-01	0.00E-01
AG110M	2.12E+00	1.46E+03	3.87E+00	3.58E+00	7.03E+00	0.00E-01	0.00E-01	0.00E-01
TE125M	1.49E+03	4.46E+04	1.12E+04	4.04E+03	4.54E+04	3.36E+03	0.00E-01	0.00E-01
TE127M	3.45E+03	9.51E+04	2.83E+04	1.01E+04	1.15E+05	7.25E+03	0.00E-01	0.00E-01
TE127	1.68E+01	6.12E+03	7.76E+01	2.79E+01	3.16E+02	5.75E+01	0.00E-01	0.00E-01
TE129M	7.51E+03	2.39E+05	4.75E+04	1.77E+04	1.98E+05	1.63E+04	0.00E-01	0.00E-01
TE129	1.79E-05	5.56E-05	7.36E-05	2.77E-05	3.09E-04	5.65E-05	0.00E-01	0.00E-01
TE131M	1.70E+03	2.03E+05	4.18E+03	2.04E+03	2.07E+04	3.24E+03	0.00E-01	0.00E-01
TE131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE132	5.21E+03	2.62E+05	8.58E+03	5.55E+03	5.35E+04	6.13E+03	0.00E-01	0.00E-01
I 130	3.59E+01	7.84E+01	3.09E+01	9.11E+01	1.42E+02	7.72E+03	0.00E-01	0.00E-01
I 131	4.94E+02	2.28E+02	6.03E+02	8.62E+02	1.48E+03	2.83E+05	0.00E-01	0.00E-01
I 132	2.11E-02	1.13E-02	2.25E-02	6.02E-02	9.60E-02	2.11E+00	0.00E-01	0.00E-01
I 133	5.33E+01	1.57E+02	1.01E+02	1.75E+02	3.05E+02	2.57E+04	0.00E-01	0.00E-01
I 134	8.36E-08	2.04E-10	8.60E-08	2.34E-07	3.72E-07	4.05E-06	0.00E-01	0.00E-01
I 135	5.37E+00	1.64E+01	5.56E+00	1.46E+01	2.33E+01	9.60E+02	0.00E-01	0.00E-01
CS134	2.55E+06	5.45E+04	1.31E+06	3.12E+06	1.01E+06	0.00E-01	3.35E+05	0.00E-01
CS136	3.70E+05	5.84E+04	1.30E+05	5.14E+05	2.86E+05	0.00E-01	3.92E+04	0.00E-01
CS137	1.50E+06	4.45E+04	1.68E+06	2.30E+06	7.80E+05	0.00E-01	2.59E+05	0.00E-01
CS138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA139	6.35E-07	3.85E-05	2.17E-05	1.55E-08	1.45E-08	0.00E-01	8.77E-09	0.00E-01
BA140	5.31E+01	1.67E+03	8.10E+02	1.02E+00	3.46E-01	0.00E-01	5.83E-01	0.00E-01
BA141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA142	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA140	5.80E-02	1.61E+04	4.35E-01	2.19E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA142	7.45E-08	2.18E-03	6.58E-07	2.99E-07	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	7.41E-03	2.50E+02	9.65E-02	6.53E-02	3.03E-02	0.00E-01	0.00E-01	0.00E-01
CE143	8.59E-04	2.90E+02	1.05E-02	7.77E+00	3.42E-03	0.00E-01	0.00E-01	0.00E-01
CE144	2.75E-01	1.73E+03	5.13E+00	2.14E+00	1.27E+00	0.00E-01	0.00E-01	0.00E-01
PR143	1.14E-01	1.01E+04	2.30E+00	9.24E-01	5.33E-01	0.00E-01	0.00E-01	0.00E-01
PR144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ND147	1.08E-01	8.63E+03	1.56E+00	1.80E+00	1.05E+00	0.00E-01	0.00E-01	0.00E-01
W 187	1.89E+02	1.77E+05	6.48E+02	5.41E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NP239	5.06E-03	1.88E+03	9.34E-02	9.18E-03	2.86E-02	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = ADULT
PATHWAY = ANIMAL DRINKING WATER—MEAT

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	1.65E+00	1.65E+00	0.00E-01	1.65E+00	1.65E+00	1.65E+00	1.65E+00	0.00E-01
C 14	4.86E+01	4.86E+01	2.43E+02	4.86E+01	4.86E+01	4.86E+01	4.86E+01	0.00E-01
NA 24	3.33E-08	3.33E-08	3.33E-08	3.33E-08	3.33E-08	3.33E-08	3.33E-08	0.00E-01
P 32	3.58E+02	1.04E+03	9.27E+03	5.77E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	1.07E-02	2.68E+00	0.00E-01	0.00E-01	2.35E-03	6.37E-03	1.42E-02	0.00E-01
MN 54	1.84E+00	2.96E+01	0.00E-01	9.65E+00	2.87E+00	0.00E-01	0.00E-01	0.00E-01
MN 56	3.01E-28	5.42E-26	0.00E-01	1.70E-27	2.16E-27	0.00E-01	0.00E-01	0.00E-01
FE 55	4.82E+01	1.19E+02	2.99E+02	2.07E+02	0.00E-01	0.00E-01	1.15E+02	0.00E-01
FE 59	3.16E+02	2.75E+03	3.51E+02	8.24E+02	0.00E-01	0.00E-01	2.30E+02	0.00E-01
CO 58	4.93E+01	4.45E+02	0.00E-01	2.20E+01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	1.68E+02	1.43E+03	0.00E-01	7.62E+01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 63	6.37E+02	2.75E+02	1.90E+04	1.32E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	3.05E-26	1.69E-24	5.14E-25	6.67E-26	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CU 64	3.20E-12	5.81E-10	0.00E-01	6.82E-12	1.72E-11	0.00E-01	0.00E-01	0.00E-01
ZN 65	5.44E+02	7.59E+02	3.78E+02	1.20E+03	8.06E+02	0.00E-01	0.00E-01	0.00E-01
ZN 69	4.78E-28	1.03E-27	3.60E-27	6.88E-27	4.47E-27	0.00E-01	0.00E-01	0.00E-01
BR 83	1.89E-26	2.72E-26	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 84	8.86E-27	6.96E-32	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 85	6.73E-34	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 86	4.00E+02	1.69E+02	0.00E-01	8.59E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 88	2.34E-27	6.10E-38	0.00E-01	4.41E-27	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 89	1.38E-27	1.14E-40	0.00E-01	1.97E-27	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	1.11E+01	6.21E+01	3.87E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	2.89E+02	3.62E+02	1.44E+04	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 91	2.06E-16	2.42E-14	5.09E-15	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 92	1.04E-27	4.78E-25	2.41E-26	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 90	1.82E-05	7.18E+00	6.77E-04	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91M	1.69E-31	1.28E-29	4.37E-30	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91	3.77E-02	7.77E+02	1.41E+00	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 92	2.26E-30	1.35E-24	7.73E-29	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 93	4.07E-18	4.67E-12	1.47E-16	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	4.99E-01	2.33E+03	2.30E+00	7.37E-01	1.16E+00	0.00E-01	0.00E-01	0.00E-01
ZR 97	3.89E-11	2.63E-05	4.21E-10	8.50E-11	1.28E-10	0.00E-01	0.00E-01	0.00E-01
NB 95	9.69E-01	1.09E+04	3.24E+00	1.80E+00	1.78E+00	0.00E-01	0.00E-01	0.00E-01
MO 99	1.15E-01	1.40E+00	0.00E-01	6.06E-01	1.37E+00	0.00E-01	0.00E-01	0.00E-01
TC 99M	8.45E-24	3.93E-22	2.35E-25	6.64E-25	1.01E-23	0.00E-01	3.25E-25	0.00E-01
TC101	1.85E-27	5.68E-40	1.31E-28	1.89E-28	3.40E-27	0.00E-01	9.65E-29	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = ADULT
PATHWAY = ANIMAL DRINKING WATER--MEAT

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
RU103	6.18E+01	1.67E+04	1.43E+02	0.00E-01	5.47E+02	0.00E-01	0.00E-01	0.00E-01
RU105	5.03E-26	7.79E-23	1.27E-25	0.00E-01	1.65E-24	0.00E-01	0.00E-01	0.00E-01
RU106	3.70E+02	1.89E+05	2.92E+03	0.00E-01	5.64E+03	0.00E-01	0.00E-01	0.00E-01
AG110M	3.90E+00	2.68E+03	7.10E+00	6.57E+00	1.29E+01	0.00E-01	0.00E-01	0.00E-01
TE125M	6.01E+01	1.79E+03	4.48E+02	1.62E+02	1.82E+03	1.35E+02	0.00E-01	0.00E-01
TE127M	1.54E+02	4.25E+03	1.27E+03	4.53E+02	5.14E+03	3.24E+02	0.00E-01	0.00E-01
TE127	1.63E-15	5.95E-13	7.54E-15	2.71E-15	3.07E-14	5.58E-15	0.00E-01	0.00E-01
TE129M	2.56E+02	8.13E+03	1.62E+03	6.03E+02	6.74E+03	5.55E+02	0.00E-01	0.00E-01
TE129	7.83E-27	2.43E-26	3.21E-26	1.21E-26	1.35E-25	2.47E-26	0.00E-01	0.00E-01
TE131M	2.23E-03	2.65E-01	5.46E-03	2.67E-03	2.71E-02	4.23E-03	0.00E-01	0.00E-01
TE131	2.20E-27	9.87E-28	6.97E-27	2.91E-27	3.06E-26	5.73E-27	0.00E-01	0.00E-01
TE132	4.59E+00	2.31E+02	7.57E+00	4.89E+00	4.71E+01	5.40E+00	0.00E-01	0.00E-01
I 130	1.31E-11	2.85E-11	1.12E-11	3.31E-11	5.17E-11	2.81E-09	0.00E-01	0.00E-01
I 131	4.85E+00	2.23E+00	5.91E+00	8.46E+00	1.45E+01	2.77E+03	0.00E-01	0.00E-01
I 132	9.85E-27	5.29E-27	1.05E-26	2.81E-26	4.48E-26	9.85E-25	0.00E-01	0.00E-01
I 133	6.67E-07	1.96E-06	1.26E-06	2.19E-06	3.82E-06	3.21E-04	0.00E-01	0.00E-01
I 134	3.27E-27	7.97E-30	3.36E-27	9.14E-27	1.45E-26	1.58E-25	0.00E-01	0.00E-01
I 135	3.76E-22	1.15E-21	3.90E-22	1.02E-21	1.64E-21	6.73E-20	0.00E-01	0.00E-01
CS134	1.31E+03	2.81E+01	6.74E+02	1.60E+03	5.19E+02	0.00E-01	1.72E+02	0.00E-01
CS136	7.08E+01	1.12E+01	2.49E+01	9.83E+01	5.47E+01	0.00E-01	7.50E+00	0.00E-01
CS137	7.87E+02	2.33E+01	8.79E+02	1.20E+03	4.08E+02	0.00E-01	1.36E+02	0.00E-01
CS138	1.44E-27	1.24E-32	1.47E-27	2.90E-27	2.13E-27	0.00E-01	2.11E-28	0.00E-01
BA139	1.33E-28	8.04E-27	4.53E-27	3.23E-30	3.02E-30	0.00E-01	1.83E-30	0.00E-01
BA140	3.97E+00	1.25E+02	6.05E+01	7.60E-02	2.58E-02	0.00E-01	4.35E-02	0.00E-01
BA141	1.26E-29	1.76E-37	3.74E-28	2.83E-31	2.63E-31	0.00E-01	1.61E-31	0.00E-01
BA142	2.12E-30	0.00E-01	3.37E-29	3.47E-32	2.93E-32	0.00E-01	1.96E-32	0.00E-01
LA140	4.67E-08	1.30E-02	3.51E-07	1.77E-07	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA142	4.47E-32	1.31E-27	3.94E-31	1.79E-31	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	1.55E-03	5.23E+01	2.02E-02	1.37E-02	6.35E-03	0.00E-01	0.00E-01	0.00E-01
CE143	1.91E-08	6.45E-03	2.34E-07	1.73E-04	7.60E-08	0.00E-01	0.00E-01	0.00E-01
CE144	8.26E-02	5.20E+02	1.54E+00	6.43E-01	3.82E-01	0.00E-01	0.00E-01	0.00E-01
PR143	2.13E-03	1.88E+02	4.30E-02	1.72E-02	9.95E-03	0.00E-01	0.00E-01	0.00E-01
PR144	1.56E-32	4.42E-38	3.08E-31	1.28E-31	7.20E-32	0.00E-01	0.00E-01	0.00E-01
ND147	1.13E-03	9.09E+01	1.64E-02	1.89E-02	1.11E-02	0.00E-01	0.00E-01	0.00E-01
W 187	9.54E-08	8.94E-05	3.26E-07	2.73E-07	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NP239	9.88E-08	3.67E-02	1.82E-06	1.79E-07	5.59E-07	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = ADULT
PATHWAY = RIVER SHORELINE DEPOSITS

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NA 24	1.88E+00	1.88E+00	1.88E+00	1.88E+00	1.88E+00	1.88E+00	1.88E+00	2.18E+00
P 32	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	7.33E-01	7.33E-01	7.33E-01	7.33E-01	7.33E-01	7.33E-01	7.33E-01	8.66E-01
MN 54	2.18E+02	2.18E+02	2.18E+02	2.18E+02	2.18E+02	2.18E+02	2.18E+02	2.56E+02
MN 56	1.38E-01	1.38E-01	1.38E-01	1.38E-01	1.38E-01	1.38E-01	1.38E-01	1.63E-01
FE 55	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 59	4.29E+01	4.29E+01	4.29E+01	4.29E+01	4.29E+01	4.29E+01	4.29E+01	5.04E+01
CO 58	5.98E+01	5.98E+01	5.98E+01	5.98E+01	5.98E+01	5.98E+01	5.98E+01	7.01E+01
CO 60	3.38E+03	3.38E+03	3.38E+03	3.38E+03	3.38E+03	3.38E+03	3.38E+03	3.98E+03
NI 63	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	4.55E-02	4.55E-02	4.55E-02	4.55E-02	4.55E-02	4.55E-02	4.55E-02	5.29E-02
CU 64	9.48E-02	9.48E-02	9.48E-02	9.48E-02	9.48E-02	9.48E-02	9.48E-02	1.07E-01
ZN 65	1.17E+02	1.17E+02	1.17E+02	1.17E+02	1.17E+02	1.17E+02	1.17E+02	1.35E+02
ZN 69	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 83	7.46E-04	7.46E-04	7.46E-04	7.46E-04	7.46E-04	7.46E-04	7.46E-04	1.08E-03
BR 84	2.80E-02	2.80E-02	2.80E-02	2.80E-02	2.80E-02	2.80E-02	2.80E-02	3.27E-02
BR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 86	1.42E+00	1.42E+00	1.42E+00	1.42E+00	1.42E+00	1.42E+00	1.42E+00	1.62E+00
RB 88	4.14E-03	4.14E-03	4.14E-03	4.14E-03	4.14E-03	4.14E-03	4.14E-03	4.73E-03
RB 89	1.46E-02	1.46E-02	1.46E-02	1.46E-02	1.46E-02	1.46E-02	1.46E-02	1.75E-02
SR 89	3.40E-03	3.40E-03	3.40E-03	3.40E-03	3.40E-03	3.40E-03	3.40E-03	3.95E-03
SR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 91	3.35E-01	3.35E-01	3.35E-01	3.35E-01	3.35E-01	3.35E-01	3.35E-01	3.92E-01
SR 92	1.19E-01	1.19E-01	1.19E-01	1.19E-01	1.19E-01	1.19E-01	1.19E-01	1.33E-01
Y 90	7.08E-04	7.08E-04	7.08E-04	7.08E-04	7.08E-04	7.08E-04	7.08E-04	8.36E-04
Y 91M	1.45E-02	1.45E-02	1.45E-02	1.45E-02	1.45E-02	1.45E-02	1.45E-02	1.68E-02
Y 91	1.69E-01	1.69E-01	1.69E-01	1.69E-01	1.69E-01	1.69E-01	1.69E-01	1.90E-01
Y 92	2.80E-02	2.80E-02	2.80E-02	2.80E-02	2.80E-02	2.80E-02	2.80E-02	3.32E-02
Y 93	2.86E-02	2.86E-02	2.86E-02	2.86E-02	2.86E-02	2.86E-02	2.86E-02	3.92E-02
ZR 95	3.86E+01	3.86E+01	3.86E+01	3.86E+01	3.86E+01	3.86E+01	3.86E+01	4.48E+01
ZR 97	4.64E-01	4.64E-01	4.64E-01	4.64E-01	4.64E-01	4.64E-01	4.64E-01	5.40E-01
NB 95	2.16E+01	2.16E+01	2.16E+01	2.16E+01	2.16E+01	2.16E+01	2.16E+01	2.54E+01
MO 99	6.28E-01	6.28E-01	6.28E-01	6.28E-01	6.28E-01	6.28E-01	6.28E-01	7.27E-01
TC 99M	2.87E-02	2.87E-02	2.87E-02	2.87E-02	2.87E-02	2.87E-02	2.87E-02	3.28E-02
TC101	2.39E-03	2.39E-03	2.39E-03	2.39E-03	2.39E-03	2.39E-03	2.39E-03	2.66E-03

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = ADULT
PATHWAY = RIVER SHORELINE DEPOSITS

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
RU103	1.70E+01	1.70E+01	1.70E+01	1.70E+01	1.70E+01	1.70E+01	1.70E+01	1.99E+01
RU105	9.86E-02	9.86E-02	9.86E-02	9.86E-02	9.86E-02	9.86E-02	9.86E-02	1.12E-01
RU106	6.65E+01	6.65E+01	6.65E+01	6.65E+01	6.65E+01	6.65E+01	6.65E+01	7.97E+01
AG110M	5.43E+02	5.43E+02	5.43E+02	5.43E+02	5.43E+02	5.43E+02	5.43E+02	6.34E+02
TE125M	2.45E-01	2.45E-01	2.45E-01	2.45E-01	2.45E-01	2.45E-01	2.45E-01	3.36E-01
TE127M	1.44E-02	1.44E-02	1.44E-02	1.44E-02	1.44E-02	1.44E-02	1.44E-02	1.71E-02
TE127	4.65E-04	4.65E-04	4.65E-04	4.65E-04	4.65E-04	4.65E-04	4.65E-04	5.12E-04
TE129M	3.11E+00	3.11E+00	3.11E+00	3.11E+00	3.11E+00	3.11E+00	3.11E+00	3.64E+00
TE129	3.89E-03	3.89E-03	3.89E-03	3.89E-03	3.89E-03	3.89E-03	3.89E-03	4.60E-03
TE131M	1.26E+00	1.26E+00	1.26E+00	1.26E+00	1.26E+00	1.26E+00	1.26E+00	1.49E+00
TE131	3.90E-03	3.90E-03	3.90E-03	3.90E-03	3.90E-03	3.90E-03	3.90E-03	4.61E+00
TE132	6.67E-01	6.67E-01	6.67E-01	6.67E-01	6.67E-01	6.67E-01	6.67E-01	7.85E-01
I 130	8.62E-01	8.62E-01	8.62E-01	8.62E-01	8.62E-01	8.62E-01	8.62E-01	1.05E+00
I 131	2.71E+00	2.71E+00	2.71E+00	2.71E+00	2.71E+00	2.71E+00	2.71E+00	3.29E+00
I 132	1.90E-01	1.90E-01	1.90E-01	1.90E-01	1.90E-01	1.90E-01	1.90E-01	2.24E-01
I 133	3.85E-01	3.85E-01	3.85E-01	3.85E-01	3.85E-01	3.85E-01	3.85E-01	4.68E-01
I 134	6.49E-02	6.49E-02	6.49E-02	6.49E-02	6.49E-02	6.49E-02	6.49E-02	7.71E-02
I 135	3.93E-01	3.93E-01	3.93E-01	3.93E-01	3.93E-01	3.93E-01	3.93E-01	4.58E-01
CS134	1.08E+03	1.08E+03	1.08E+03	1.08E+03	1.08E+03	1.08E+03	1.08E+03	1.26E+03
CS136	2.37E+01	2.37E+01	2.37E+01	2.37E+01	2.37E+01	2.37E+01	2.37E+01	2.68E+01
CS137	1.62E+03	1.62E+03	1.62E+03	1.62E+03	1.62E+03	1.62E+03	1.62E+03	1.89E+03
CS138	4.98E-02	4.98E-02	4.98E-02	4.98E-02	4.98E-02	4.98E-02	4.98E-02	5.69E-02
BA139	1.58E-02	1.58E-02	1.58E-02	1.58E-02	1.58E-02	1.58E-02	1.58E-02	1.77E-02
BA140	3.23E+00	3.23E+00	3.23E+00	3.23E+00	3.23E+00	3.23E+00	3.23E+00	3.70E+00
BA141	5.24E-03	5.24E-03	5.24E-03	5.24E-03	5.24E-03	5.24E-03	5.24E-03	5.97E-03
BA142	4.79E-03	4.79E-03	4.79E-03	4.79E-03	4.79E-03	4.79E-03	4.79E-03	5.46E-03
LA140	3.03E+00	3.03E+00	3.03E+00	3.03E+00	3.03E+00	3.03E+00	3.03E+00	3.43E+00
LA142	1.11E-01	1.11E-01	1.11E-01	1.11E-01	1.11E-01	1.11E-01	1.11E-01	1.33E-01
CE141	2.15E+00	2.15E+00	2.15E+00	2.15E+00	2.15E+00	2.15E+00	2.15E+00	2.42E+00
CE143	3.65E-01	3.65E-01	3.65E-01	3.65E-01	3.65E-01	3.65E-01	3.65E-01	4.15E-01
CE144	1.10E+01	1.10E+01	1.10E+01	1.10E+01	1.10E+01	1.10E+01	1.10E+01	1.27E+01
PR143	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
PR144	2.27E-04	2.27E-04	2.27E-04	2.27E-04	2.27E-04	2.27E-04	2.27E-04	2.61E-04
ND147	1.34E+00	1.34E+00	1.34E+00	1.34E+00	1.34E+00	1.34E+00	1.34E+00	1.60E+00
W 187	3.71E-01	3.71E-01	3.71E-01	3.71E-01	3.71E-01	3.71E-01	3.71E-01	4.31E-01
NP239	2.70E-01	2.70E-01	2.70E-01	2.70E-01	2.70E-01	2.70E-01	2.70E-01	3.12E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = ADULT
PATHWAY = SWIMMING

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NA 24	3.12E+01	3.12E+01	3.12E+01	3.12E+01	3.12E+01	3.12E+01	3.12E+01	0.00E-01
P 32	2.57E-02	2.57E-02	2.57E-02	2.57E-02	2.57E-02	2.57E-02	2.57E-02	0.00E-01
CR 51	2.09E-01	2.09E-01	2.09E-01	2.09E-01	2.09E-01	2.09E-01	2.09E-01	0.00E-01
MN 54	6.02E+00	6.02E+00	6.02E+00	6.02E+00	6.02E+00	6.02E+00	6.02E+00	0.00E-01
MN 56	1.25E+01	1.25E+01	1.25E+01	1.25E+01	1.25E+01	1.25E+01	1.25E+01	0.00E-01
FE 55	2.57E-04	2.57E-04	2.57E-04	2.57E-04	2.57E-04	2.57E-04	2.57E-04	0.00E-01
FE 59	8.83E+00	8.83E+00	8.83E+00	8.83E+00	8.83E+00	8.83E+00	8.83E+00	0.00E-01
CO 58	7.23E+00	7.23E+00	7.23E+00	7.23E+00	7.23E+00	7.23E+00	7.23E+00	0.00E-01
CO 60	1.85E+01	1.85E+01	1.85E+01	1.85E+01	1.85E+01	1.85E+01	1.85E+01	0.00E-01
NI 63	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	3.90E+00	3.90E+00	3.90E+00	3.90E+00	3.90E+00	3.90E+00	3.90E+00	0.00E-01
CU 64	1.48E+00	1.48E+00	1.48E+00	1.48E+00	1.48E+00	1.48E+00	1.48E+00	0.00E-01
ZN 65	4.42E+00	4.42E+00	4.42E+00	4.42E+00	4.42E+00	4.42E+00	4.42E+00	0.00E-01
ZN 69	5.97E-03	5.97E-03	5.97E-03	5.97E-03	5.97E-03	5.97E-03	5.97E-03	0.00E-01
BR 83	6.63E-02	6.63E-02	6.63E-02	6.63E-02	6.63E-02	6.63E-02	6.63E-02	0.00E-01
BR 84	1.23E+01	1.23E+01	1.23E+01	1.23E+01	1.23E+01	1.23E+01	1.23E+01	0.00E-01
BR 85	1.32E-02	1.32E-02	1.32E-02	1.32E-02	1.32E-02	1.32E-02	1.32E-02	0.00E-01
RB 86	6.82E-01	6.82E-01	6.82E-01	6.82E-01	6.82E-01	6.82E-01	6.82E-01	0.00E-01
RB 88	3.82E+00	3.82E+00	3.82E+00	3.82E+00	3.82E+00	3.82E+00	3.82E+00	0.00E-01
RB 89	1.38E+01	1.38E+01	1.38E+01	1.38E+01	1.38E+01	1.38E+01	1.38E+01	0.00E-01
SR 89	1.85E-02	1.85E-02	1.85E-02	1.85E-02	1.85E-02	1.85E-02	1.85E-02	0.00E-01
SR 90	2.17E-03	2.17E-03	2.17E-03	2.17E-03	2.17E-03	2.17E-03	2.17E-03	0.00E-01
SR 91	7.57E+00	7.57E+00	7.57E+00	7.57E+00	7.57E+00	7.57E+00	7.57E+00	0.00E-01
SR 92	1.02E+01	1.02E+01	1.02E+01	1.02E+01	1.02E+01	1.02E+01	1.02E+01	0.00E-01
Y 90	5.21E-02	5.21E-02	5.21E-02	5.21E-02	5.21E-02	5.21E-02	5.21E-02	0.00E-01
Y 91M	3.69E+00	3.69E+00	3.69E+00	3.69E+00	3.69E+00	3.69E+00	3.69E+00	0.00E-01
Y 91	2.69E-02	2.69E-02	2.69E-02	2.69E-02	2.69E-02	2.69E-02	2.69E-02	0.00E-01
Y 92	1.81E+00	1.81E+00	1.81E+00	1.81E+00	1.81E+00	1.81E+00	1.81E+00	0.00E-01
Y 93	7.57E-01	7.57E-01	7.57E-01	7.57E-01	7.57E-01	7.57E-01	7.57E-01	0.00E-01
ZR 95	6.02E+00	6.02E+00	6.02E+00	6.02E+00	6.02E+00	6.02E+00	6.02E+00	0.00E-01
ZR 97	6.00E+00	6.00E+00	6.00E+00	6.00E+00	6.00E+00	6.00E+00	6.00E+00	0.00E-01
NB 95	5.62E+00	5.62E+00	5.62E+00	5.62E+00	5.62E+00	5.62E+00	5.62E+00	0.00E-01
MO 99	1.88E+00	1.88E+00	1.88E+00	1.88E+00	1.88E+00	1.88E+00	1.88E+00	0.00E-01
TC 99M	9.52E-01	9.52E-01	9.52E-01	9.52E-01	9.52E-01	9.52E-01	9.52E-01	0.00E-01
TC101	2.04E+00	2.04E+00	2.04E+00	2.04E+00	2.04E+00	2.04E+00	2.04E+00	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = ADULT
PATHWAY = SWIMMING

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
RU103	3.57E+00	3.57E+00	3.57E+00	3.57E+00	3.57E+00	3.57E+00	3.57E+00	0.00E-01
RU105	4.74E+00	4.74E+00	4.74E+00	4.74E+00	4.74E+00	4.74E+00	4.74E+00	0.00E-01
RU106	1.52E+00	1.52E+00	1.52E+00	1.52E+00	1.52E+00	1.52E+00	1.52E+00	0.00E-01
AG110M	1.97E+01	1.97E+01	1.97E+01	1.97E+01	1.97E+01	1.97E+01	1.97E+01	0.00E-01
TE125M	1.48E-02	1.48E-02	1.48E-02	1.48E-02	1.48E-02	1.48E-02	1.48E-02	0.00E-01
TE127M	1.04E-03	1.04E-03	1.04E-03	1.04E-03	1.04E-03	1.04E-03	1.04E-03	0.00E-01
TE127	1.12E-02	1.12E-02	1.12E-02	1.12E-02	1.12E-02	1.12E-02	1.12E-02	0.00E-01
TE129M	8.43E-01	8.43E-01	8.43E-01	8.43E-01	8.43E-01	8.43E-01	8.43E-01	0.00E-01
TE129	7.19E-01	7.19E-01	7.19E-01	7.19E-01	7.19E-01	7.19E-01	7.19E-01	0.00E-01
TE131M	8.81E+00	8.81E+00	8.81E+00	8.81E+00	8.81E+00	8.81E+00	8.81E+00	0.00E-01
TE131	2.52E+00	2.52E+00	2.52E+00	2.52E+00	2.52E+00	2.52E+00	2.52E+00	0.00E-01
TE132	1.60E+00	1.60E+00	1.60E+00	1.60E+00	1.60E+00	1.60E+00	1.60E+00	0.00E-01
I 130	1.56E+01	1.56E+01	1.56E+01	1.56E+01	1.56E+01	1.56E+01	1.56E+01	0.00E-01
I 131	3.13E+00	3.13E+00	3.13E+00	3.13E+00	3.13E+00	3.13E+00	3.13E+00	0.00E-01
I 132	1.71E+01	1.71E+01	1.71E+01	1.71E+01	1.71E+01	1.71E+01	1.71E+01	0.00E-01
I 133	3.84E+00	3.84E+00	3.84E+00	3.84E+00	3.84E+00	3.84E+00	3.84E+00	0.00E-01
I 134	1.56E+01	1.56E+01	1.56E+01	1.56E+01	1.56E+01	1.56E+01	1.56E+01	0.00E-01
I 135	1.31E+01	1.31E+01	1.31E+01	1.31E+01	1.31E+01	1.31E+01	1.31E+01	0.00E-01
CS134	1.16E+01	1.16E+01	1.16E+01	1.16E+01	1.16E+01	1.16E+01	1.16E+01	0.00E-01
CS136	1.65E+01	1.65E+01	1.65E+01	1.65E+01	1.65E+01	1.65E+01	1.65E+01	0.00E-01
CS137	4.01E+00	4.01E+00	4.01E+00	4.01E+00	4.01E+00	4.01E+00	4.01E+00	0.00E-01
CS138	1.41E+01	1.41E+01	1.41E+01	1.41E+01	1.41E+01	1.41E+01	1.41E+01	0.00E-01
BA139	2.94E-01	2.94E-01	2.94E-01	2.94E-01	2.94E-01	2.94E-01	2.94E-01	0.00E-01
BA140	1.97E+00	1.97E+00	1.97E+00	1.97E+00	1.97E+00	1.97E+00	1.97E+00	0.00E-01
BA141	3.52E+00	3.52E+00	3.52E+00	3.52E+00	3.52E+00	3.52E+00	3.52E+00	0.00E-01
BA142	5.99E+00	5.99E+00	5.99E+00	5.99E+00	5.99E+00	5.99E+00	5.99E+00	0.00E-01
LA140	1.64E+01	1.64E+01	1.64E+01	1.64E+01	1.64E+01	1.64E+01	1.64E+01	0.00E-01
LA142	1.73E+01	1.73E+01	1.73E+01	1.73E+01	1.73E+01	1.73E+01	1.73E+01	0.00E-01
CE141	5.22E-01	5.22E-01	5.22E-01	5.22E-01	5.22E-01	5.22E-01	5.22E-01	0.00E-01
CE143	2.28E+00	2.28E+00	2.28E+00	2.28E+00	2.28E+00	2.28E+00	2.28E+00	0.00E-01
CE144	3.45E-01	3.45E-01	3.45E-01	3.45E-01	3.45E-01	3.45E-01	3.45E-01	0.00E-01
PR143	6.42E-03	6.42E-03	6.42E-03	6.42E-03	6.42E-03	6.42E-03	6.42E-03	0.00E-01
PR144	1.77E-01	1.77E-01	1.77E-01	1.77E-01	1.77E-01	1.77E-01	1.77E-01	0.00E-01
ND147	1.12E+00	1.12E+00	1.12E+00	1.12E+00	1.12E+00	1.12E+00	1.12E+00	0.00E-01
W 187	3.32E+00	3.32E+00	3.32E+00	3.32E+00	3.32E+00	3.32E+00	3.32E+00	0.00E-01
NP239	9.62E-01	9.62E-01	9.62E-01	9.62E-01	9.62E-01	9.62E-01	9.62E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = ADULT
PATHWAY = BOATING

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NA 24	5.65E+01	5.65E+01	5.65E+01	5.65E+01	5.65E+01	5.65E+01	5.65E+01	0.00E-01
P 32	4.66E-02	4.66E-02	4.66E-02	4.66E-02	4.66E-02	4.66E-02	4.66E-02	0.00E-01
CR 51	3.78E-01	3.78E-01	3.78E-01	3.78E-01	3.78E-01	3.78E-01	3.78E-01	0.00E-01
MN 54	1.09E+01	1.09E+01	1.09E+01	1.09E+01	1.09E+01	1.09E+01	1.09E+01	0.00E-01
MN 56	2.27E+01	2.27E+01	2.27E+01	2.27E+01	2.27E+01	2.27E+01	2.27E+01	0.00E-01
FE 55	4.66E-04	4.66E-04	4.66E-04	4.66E-04	4.66E-04	4.66E-04	4.66E-04	0.00E-01
FE 59	1.60E+01	1.60E+01	1.60E+01	1.60E+01	1.60E+01	1.60E+01	1.60E+01	0.00E-01
CO 58	1.31E+01	1.31E+01	1.31E+01	1.31E+01	1.31E+01	1.31E+01	1.31E+01	0.00E-01
CO 60	3.35E+01	3.35E+01	3.35E+01	3.35E+01	3.35E+01	3.35E+01	3.35E+01	0.00E-01
NI 63	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	7.08E+00	7.08E+00	7.08E+00	7.08E+00	7.08E+00	7.08E+00	7.08E+00	0.00E-01
CU 64	2.68E+00	2.68E+00	2.68E+00	2.68E+00	2.68E+00	2.68E+00	2.68E+00	0.00E-01
ZN 65	8.00E+00	8.00E+00	8.00E+00	8.00E+00	8.00E+00	8.00E+00	8.00E+00	0.00E-01
ZN 69	1.08E-02	1.08E-02	1.08E-02	1.08E-02	1.08E-02	1.08E-02	1.08E-02	0.00E-01
BR 83	1.20E-01	1.20E-01	1.20E-01	1.20E-01	1.20E-01	1.20E-01	1.20E-01	0.00E-01
BR 84	2.24E+01	2.24E+01	2.24E+01	2.24E+01	2.24E+01	2.24E+01	2.24E+01	0.00E-01
BR 85	2.39E-02	2.39E-02	2.39E-02	2.39E-02	2.39E-02	2.39E-02	2.39E-02	0.00E-01
RB 86	1.24E+00	1.24E+00	1.24E+00	1.24E+00	1.24E+00	1.24E+00	1.24E+00	0.00E-01
RB 88	6.92E+00	6.92E+00	6.92E+00	6.92E+00	6.92E+00	6.92E+00	6.92E+00	0.00E-01
RB 89	2.49E+01	2.49E+01	2.49E+01	2.49E+01	2.49E+01	2.49E+01	2.49E+01	0.00E-01
SR 89	3.35E-02	3.35E-02	3.35E-02	3.35E-02	3.35E-02	3.35E-02	3.35E-02	0.00E-01
SR 90	3.93E-03	3.93E-03	3.93E-03	3.93E-03	3.93E-03	3.93E-03	3.93E-03	0.00E-01
SR 91	1.37E+01	1.37E+01	1.37E+01	1.37E+01	1.37E+01	1.37E+01	1.37E+01	0.00E-01
SR 92	1.84E+01	1.84E+01	1.84E+01	1.84E+01	1.84E+01	1.84E+01	1.84E+01	0.00E-01
Y 90	9.45E-02	9.45E-02	9.45E-02	9.45E-02	9.45E-02	9.45E-02	9.45E-02	0.00E-01
Y 91M	6.69E+00	6.69E+00	6.69E+00	6.69E+00	6.69E+00	6.69E+00	6.69E+00	0.00E-01
Y 91	4.87E-02	4.87E-02	4.87E-02	4.87E-02	4.87E-02	4.87E-02	4.87E-02	0.00E-01
Y 92	3.28E+00	3.28E+00	3.28E+00	3.28E+00	3.28E+00	3.28E+00	3.28E+00	0.00E-01
Y 93	1.37E+00	1.37E+00	1.37E+00	1.37E+00	1.37E+00	1.37E+00	1.37E+00	0.00E-01
ZR 95	1.09E+01	1.09E+01	1.09E+01	1.09E+01	1.09E+01	1.09E+01	1.09E+01	0.00E-01
ZR 97	1.09E+01	1.09E+01	1.09E+01	1.09E+01	1.09E+01	1.09E+01	1.09E+01	0.00E-01
NB 95	1.02E+01	1.02E+01	1.02E+01	1.02E+01	1.02E+01	1.02E+01	1.02E+01	0.00E-01
MO 99	3.42E+00	3.42E+00	3.42E+00	3.42E+00	3.42E+00	3.42E+00	3.42E+00	0.00E-01
TC 99M	1.73E+00	1.73E+00	1.73E+00	1.73E+00	1.73E+00	1.73E+00	1.73E+00	0.00E-01
TC101	3.69E+00	3.69E+00	3.69E+00	3.69E+00	3.69E+00	3.69E+00	3.69E+00	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = ADULT
PATHWAY = BOATING

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem gal)/(Ci min)				
RU103	6.47E+00	6.47E+00	6.47E+00	6.47E+00	6.47E+00	6.47E+00	6.47E+00	0.00E-01
RU105	8.60E+00	8.60E+00	8.60E+00	8.60E+00	8.60E+00	8.60E+00	8.60E+00	0.00E-01
RU106	2.77E+00	2.77E+00	2.77E+00	2.77E+00	2.77E+00	2.77E+00	2.77E+00	0.00E-01
AG110M	3.57E+01	3.57E+01	3.57E+01	3.57E+01	3.57E+01	3.57E+01	3.57E+01	0.00E-01
TE125M	2.69E-02	2.69E-02	2.69E-02	2.69E-02	2.69E-02	2.69E-02	2.69E-02	0.00E-01
TE127M	1.89E-03	1.89E-03	1.89E-03	1.89E-03	1.89E-03	1.89E-03	1.89E-03	0.00E-01
TE127	2.02E-02	2.02E-02	2.02E-02	2.02E-02	2.02E-02	2.02E-02	2.02E-02	0.00E-01
TE129M	1.53E+00	1.53E+00	1.53E+00	1.53E+00	1.53E+00	1.53E+00	1.53E+00	0.00E-01
TE129	1.30E+00	1.30E+00	1.30E+00	1.30E+00	1.30E+00	1.30E+00	1.30E+00	0.00E-01
TE131M	1.60E+01	1.60E+01	1.60E+01	1.60E+01	1.60E+01	1.60E+01	1.60E+01	0.00E-01
TE131	4.56E+00	4.56E+00	4.56E+00	4.56E+00	4.56E+00	4.56E+00	4.56E+00	0.00E-01
TE132	2.91E+00	2.91E+00	2.91E+00	2.91E+00	2.91E+00	2.91E+00	2.91E+00	0.00E-01
I 130	2.82E+01	2.82E+01	2.82E+01	2.82E+01	2.82E+01	2.82E+01	2.82E+01	0.00E-01
I 131	5.67E+00	5.67E+00	5.67E+00	5.67E+00	5.67E+00	5.67E+00	5.67E+00	0.00E-01
I 132	3.11E+01	3.11E+01	3.11E+01	3.11E+01	3.11E+01	3.11E+01	3.11E+01	0.00E-01
I 133	6.96E+00	6.96E+00	6.96E+00	6.96E+00	6.96E+00	6.96E+00	6.96E+00	0.00E-01
I 134	2.82E+01	2.82E+01	2.82E+01	2.82E+01	2.82E+01	2.82E+01	2.82E+01	0.00E-01
I 135	2.38E+01	2.38E+01	2.38E+01	2.38E+01	2.38E+01	2.38E+01	2.38E+01	0.00E-01
CS134	2.11E+01	2.11E+01	2.11E+01	2.11E+01	2.11E+01	2.11E+01	2.11E+01	0.00E-01
CS136	2.98E+01	2.98E+01	2.98E+01	2.98E+01	2.98E+01	2.98E+01	2.98E+01	0.00E-01
CS137	7.28E+00	7.28E+00	7.28E+00	7.28E+00	7.28E+00	7.28E+00	7.28E+00	0.00E-01
CS138	2.56E+01	2.56E+01	2.56E+01	2.56E+01	2.56E+01	2.56E+01	2.56E+01	0.00E-01
BA139	5.33E-01	5.33E-01	5.33E-01	5.33E-01	5.33E-01	5.33E-01	5.33E-01	0.00E-01
BA140	3.56E+00	3.56E+00	3.56E+00	3.56E+00	3.56E+00	3.56E+00	3.56E+00	0.00E-01
BA141	6.37E+00	6.37E+00	6.37E+00	6.37E+00	6.37E+00	6.37E+00	6.37E+00	0.00E-01
BA142	1.09E+01	1.09E+01	1.09E+01	1.09E+01	1.09E+01	1.09E+01	1.09E+01	0.00E-01
LA140	2.98E+01	2.98E+01	2.98E+01	2.98E+01	2.98E+01	2.98E+01	2.98E+01	0.00E-01
LA142	3.13E+01	3.13E+01	3.13E+01	3.13E+01	3.13E+01	3.13E+01	3.13E+01	0.00E-01
CE141	9.46E-01	9.46E-01	9.46E-01	9.46E-01	9.46E-01	9.46E-01	9.46E-01	0.00E-01
CE143	4.14E+00	4.14E+00	4.14E+00	4.14E+00	4.14E+00	4.14E+00	4.14E+00	0.00E-01
CE144	6.26E-01	6.26E-01	6.26E-01	6.26E-01	6.26E-01	6.26E-01	6.26E-01	0.00E-01
PR143	1.16E-02	1.16E-02	1.16E-02	1.16E-02	1.16E-02	1.16E-02	1.16E-02	0.00E-01
PR144	3.20E-01	3.20E-01	3.20E-01	3.20E-01	3.20E-01	3.20E-01	3.20E-01	0.00E-01
ND147	2.04E+00	2.04E+00	2.04E+00	2.04E+00	2.04E+00	2.04E+00	2.04E+00	0.00E-01
W 187	6.02E+00	6.02E+00	6.02E+00	6.02E+00	6.02E+00	6.02E+00	6.02E+00	0.00E-01
NP239	1.74E+00	1.74E+00	1.74E+00	1.74E+00	1.74E+00	1.74E+00	1.74E+00	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = TEENAGER
PATHWAY = POTABLE WATER

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem gal)/(Ci min)				
H 3	1.54E+01	1.54E+01	0.00E-01	1.54E+01	1.54E+01	1.54E+01	1.54E+01	0.00E-01
C 14	2.08E+02	2.08E+02	1.04E+03	2.08E+02	2.08E+02	2.08E+02	2.08E+02	0.00E-01
NA 24	1.95E+02	1.95E+02	1.95E+02	1.95E+02	1.95E+02	1.95E+02	1.95E+02	0.00E-01
P 32	2.61E+03	5.66E+03	6.73E+04	4.17E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	8.99E-01	1.51E+02	0.00E-01	0.00E-01	1.97E-01	4.99E-01	1.28E+00	0.00E-01
MN 54	2.99E+02	3.09E+03	0.00E-01	1.51E+03	4.49E+02	0.00E-01	0.00E-01	0.00E-01
MN 56	1.10E-02	4.08E+00	0.00E-01	6.20E-02	7.85E-02	0.00E-01	0.00E-01	0.00E-01
FE 55	1.60E+02	2.97E+02	9.67E+02	6.85E+02	0.00E-01	0.00E-01	4.35E+02	0.00E-01
FE 59	1.33E+03	8.16E+03	1.48E+03	3.45E+03	0.00E-01	0.00E-01	1.09E+03	0.00E-01
CO 58	5.68E+02	3.40E+03	0.00E-01	2.46E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	1.62E+03	9.36E+03	0.00E-01	7.19E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 63	1.53E+03	5.09E+02	4.53E+04	3.20E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	1.52E-02	1.80E+00	2.61E-01	3.33E-02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CU 64	3.72E+00	6.14E+02	0.00E-01	7.91E+00	2.00E+01	0.00E-01	0.00E-01	0.00E-01
ZN 65	2.38E+03	2.16E+03	1.47E+03	5.10E+03	3.27E+03	0.00E-01	0.00E-01	0.00E-01
ZN 69	1.21E-08	3.19E-07	9.08E-08	1.73E-07	1.13E-07	0.00E-01	0.00E-01	0.00E-01
BR 83	1.40E-02	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 84	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 86	3.45E+03	1.09E+03	0.00E-01	7.35E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	3.18E+03	1.32E+04	1.11E+05	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	5.22E+04	5.96E+04	2.61E+06	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 91	1.42E+01	1.62E+03	3.57E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 92	7.21E-02	4.31E+01	1.69E+00	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 90	7.29E-02	2.23E+04	2.70E+00	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91	1.36E+00	2.08E+04	5.08E+01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 92	8.29E-05	7.86E+01	2.86E-03	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 93	5.16E-03	5.75E+03	1.88E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	2.26E+00	7.59E+03	1.04E+01	3.29E+00	4.84E+00	0.00E-01	0.00E-01	0.00E-01
ZR 97	2.06E-02	1.21E+04	2.27E-01	4.48E-02	6.80E-02	0.00E-01	0.00E-01	0.00E-01
NB 95	6.30E-01	4.89E+03	2.06E+00	1.14E+00	1.11E+00	0.00E-01	0.00E-01	0.00E-01
MO 99	2.29E+02	2.15E+03	0.00E-01	1.20E+03	2.74E+03	0.00E-01	0.00E-01	0.00E-01
TC 99M	1.93E-01	9.80E+00	5.35E-03	1.49E-02	2.22E-01	0.00E-01	8.29E-03	0.00E-01
TC101	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = TEENAGER
PATHWAY = POTABLE WATER

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem gal)/(Ci min)				
RU103	2.74E+01	5.36E+03	6.41E+01	0.00E-01	2.26E+02	0.00E-01	0.00E-01	0.00E-01
RU105	5.09E-02	1.06E+02	1.31E-01	0.00E-01	1.65E+00	0.00E-01	0.00E-01	0.00E-01
RU106	1.26E+02	4.80E+04	1.00E+03	0.00E-01	1.93E+03	0.00E-01	0.00E-01	0.00E-01
AG110M	3.01E+01	1.39E+04	5.23E+01	4.95E+01	9.44E+01	0.00E-01	0.00E-01	0.00E-01
TE125M	1.29E+02	2.86E+03	9.68E+02	3.49E+02	0.00E-01	2.71E+02	0.00E-01	0.00E-01
TE127M	2.92E+02	6.13E+03	2.46E+03	8.72E+02	9.97E+03	5.85E+02	0.00E-01	0.00E-01
TE127	1.47E+00	5.27E+02	6.82E+00	2.42E+00	2.76E+01	4.70E+00	0.00E-01	0.00E-01
TE129M	6.47E+02	1.53E+04	4.09E+03	1.52E+03	1.71E+04	1.32E+03	0.00E-01	0.00E-01
TE129	1.65E-06	3.70E-05	6.77E-06	2.52E-06	2.84E-05	4.83E-06	0.00E-01	0.00E-01
TE131M	1.43E+02	1.38E+04	3.59E+02	1.72E+02	1.79E+03	2.59E+02	0.00E-01	0.00E-01
TE131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE132	4.30E+02	1.45E+04	7.22E+02	4.57E+02	4.39E+03	4.82E+02	0.00E-01	0.00E-01
I 130	7.91E+01	1.52E+02	6.85E+01	1.98E+02	3.05E+02	1.62E+04	0.00E-01	0.00E-01
I 131	1.03E+03	3.80E+02	1.37E+03	1.92E+03	3.31E+03	5.61E+05	0.00E-01	0.00E-01
I 132	4.85E-02	5.89E-02	5.16E-02	1.35E-01	2.13E-01	4.55E+00	0.00E-01	0.00E-01
I 133	1.20E+02	2.97E+02	2.31E+02	3.92E+02	6.88E+02	5.48E+04	0.00E-01	0.00E-01
I 134	1.98E-07	7.25E-09	2.08E-07	5.50E-07	8.68E-07	9.17E-06	0.00E-01	0.00E-01
I 135	1.19E+01	3.57E+01	1.25E+01	3.22E+01	5.09E+01	2.07E+03	0.00E-01	0.00E-01
CS134	2.34E+04	6.26E+02	2.14E+04	5.04E+04	1.60E+04	0.00E-01	6.11E+03	0.00E-01
CS136	5.51E+03	6.60E+02	2.08E+03	8.20E+03	4.47E+03	0.00E-01	7.04E+02	0.00E-01
CS137	1.33E+04	5.42E+02	2.87E+04	3.81E+04	1.30E+04	0.00E-01	5.04E+03	0.00E-01
CS138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA139	5.79E-06	1.77E-03	1.99E-04	1.40E-07	1.32E-07	0.00E-01	9.63E-08	0.00E-01
BA140	4.44E+02	1.06E+04	6.88E+03	8.44E+00	2.86E+00	0.00E-01	5.67E+00	0.00E-01
BA141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA142	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA140	7.71E-02	1.66E+04	5.89E-01	2.90E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA142	1.03E-07	1.26E-02	9.35E-07	4.15E-07	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	2.55E-01	6.36E+03	3.33E+00	2.22E+00	1.05E+00	0.00E-01	0.00E-01	0.00E-01
CE143	2.96E-02	7.96E+03	3.64E-01	2.65E+02	1.19E-01	0.00E-01	0.00E-01	0.00E-01
CE144	9.55E+00	4.47E+04	1.78E+02	7.35E+01	4.39E+01	0.00E-01	0.00E-01	0.00E-01
PR143	1.59E-01	1.05E+04	3.19E+00	1.27E+00	7.39E-01	0.00E-01	0.00E-01	0.00E-01
PR144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ND147	1.47E-01	8.85E+03	2.26E+00	2.45E+00	1.44E+00	0.00E-01	0.00E-01	0.00E-01
W 187	5.32E+00	4.11E+03	1.86E+01	1.52E+01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NP239	1.76E-02	5.09E+03	3.36E-01	3.17E-02	9.94E-02	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = TEENAGER
PATHWAY = FRESH WATER FISH

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	4.36E-01	4.36E-01	0.00E-01	4.36E-01	4.36E-01	4.36E-01	4.36E-01	0.00E-01
C 14	3.00E+04	3.00E+04	1.50E+05	3.00E+04	3.00E+04	3.00E+04	3.00E+04	0.00E-01
NA 24	6.08E+02	6.08E+02	6.08E+02	6.08E+02	6.08E+02	6.08E+02	6.08E+02	0.00E-01
P 32	8.18E+06	1.77E+07	2.11E+08	1.31E+07	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	5.64E+00	9.47E+02	0.00E-01	0.00E-01	1.24E+00	3.13E+00	8.05E+00	0.00E-01
MN 54	3.75E+03	3.88E+04	0.00E-01	1.89E+04	5.64E+03	0.00E-01	0.00E-01	0.00E-01
MN 56	1.35E-01	4.99E+01	0.00E-01	7.57E-01	9.59E-01	0.00E-01	0.00E-01	0.00E-01
FE 55	5.01E+02	9.31E+02	3.03E+03	2.15E+03	0.00E-01	0.00E-01	1.36E+03	0.00E-01
FE 59	4.18E+03	2.56E+04	4.64E+03	1.08E+04	0.00E-01	0.00E-01	3.41E+03	0.00E-01
CO 58	8.90E+02	5.33E+03	0.00E-01	3.86E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	2.54E+03	1.47E+04	0.00E-01	1.13E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 63	4.82E+03	1.60E+03	1.42E+05	1.00E+04	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	4.63E-02	5.51E+00	7.95E-01	1.02E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CU 64	5.81E+00	9.58E+02	0.00E-01	1.24E+01	3.12E+01	0.00E-01	0.00E-01	0.00E-01
ZN 65	1.49E+05	1.36E+05	9.22E+04	3.20E+05	2.05E+05	0.00E-01	0.00E-01	0.00E-01
ZN 69	7.06E-07	1.86E-05	5.30E-06	1.01E-05	6.59E-06	0.00E-01	0.00E-01	0.00E-01
BR 83	1.79E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 84	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 86	2.17E+05	6.82E+04	0.00E-01	4.61E+05	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	2.99E+03	1.24E+04	1.04E+05	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	4.91E+04	5.61E+04	2.46E+06	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 91	1.33E+01	1.51E+03	3.34E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 92	6.61E-02	3.95E+01	1.55E+00	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 90	5.71E-02	1.75E+04	2.12E+00	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91	1.07E+00	1.63E+04	3.99E+01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 92	6.37E-05	6.05E+01	2.20E-03	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 93	4.02E-03	4.48E+03	1.47E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	2.34E-01	7.86E+02	1.08E+00	3.41E-01	5.01E-01	0.00E-01	0.00E-01	0.00E-01
ZR 97	2.13E-03	1.25E+03	2.34E-02	4.62E-03	7.01E-03	0.00E-01	0.00E-01	0.00E-01
NB 95	5.93E+02	4.60E+06	1.94E+03	1.08E+03	1.04E+03	0.00E-01	0.00E-01	0.00E-01
MO 99	7.17E+01	6.73E+02	0.00E-01	3.76E+02	8.60E+02	0.00E-01	0.00E-01	0.00E-01
TC 99M	9.00E-02	4.56E+00	2.49E-03	6.94E-03	1.04E-01	0.00E-01	3.85E-03	0.00E-01
TC101	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = TEENAGER
PATHWAY = FRESH WATER FISH

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
RU103	8.60E+00	1.68E+03	2.01E+01	0.00E-01	7.09E+01	0.00E-01	0.00E-01	0.00E-01
RU105	1.57E-02	3.27E+01	4.05E-02	0.00E-01	5.11E-01	0.00E-01	0.00E-01	0.00E-01
RU106	3.96E+01	1.51E+04	3.14E+02	0.00E-01	6.06E+02	0.00E-01	0.00E-01	0.00E-01
AG110M	2.17E+00	1.00E+03	3.78E+00	3.57E+00	6.81E+00	0.00E-01	0.00E-01	0.00E-01
TE125M	1.62E+03	3.59E+04	1.21E+04	4.38E+03	0.00E-01	3.39E+03	0.00E-01	0.00E-01
TE127M	3.67E+03	7.69E+04	3.09E+04	1.09E+04	1.25E+05	7.34E+03	0.00E-01	0.00E-01
TE127	1.83E+01	6.56E+03	8.49E+01	3.01E+01	3.44E+02	5.86E+01	0.00E-01	0.00E-01
TE129M	8.12E+03	1.92E+05	5.13E+04	1.90E+04	2.14E+05	1.65E+04	0.00E-01	0.00E-01
TE129	1.95E-05	4.38E-04	8.00E-05	2.98E-05	3.36E-04	5.72E-05	0.00E-01	0.00E-01
TE131M	1.80E+03	1.73E+05	4.49E+03	2.15E+03	2.24E+04	3.24E+03	0.00E-01	0.00E-01
TE131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE132	5.40E+03	1.82E+05	9.05E+03	5.73E+03	5.50E+04	6.04E+03	0.00E-01	0.00E-01
I 130	3.70E+01	7.12E+01	3.20E+01	9.27E+01	1.43E+02	7.56E+03	0.00E-01	0.00E-01
I 131	4.86E+02	1.79E+02	6.46E+02	9.04E+02	1.56E+03	2.64E+05	0.00E-01	0.00E-01
I 132	2.21E-02	2.69E-02	2.36E-02	6.17E-02	9.72E-02	2.08E+00	0.00E-01	0.00E-01
I 133	5.61E+01	1.39E+02	1.08E+02	1.84E+02	3.23E+02	2.57E+04	0.00E-01	0.00E-01
I 134	8.60E-08	3.15E-09	9.03E-08	2.39E-07	3.77E-07	3.99E-06	0.00E-01	0.00E-01
I 135	5.56E+00	1.66E+01	5.83E+00	1.50E+01	2.37E+01	9.66E+02	0.00E-01	0.00E-01
CS134	1.47E+06	3.93E+04	1.34E+06	3.16E+06	1.00E+06	0.00E-01	3.83E+05	0.00E-01
CS136	3.46E+05	4.14E+04	1.31E+05	5.15E+05	2.80E+05	0.00E-01	4.42E+04	0.00E-01
CS137	8.33E+05	3.40E+04	1.80E+06	2.39E+06	8.14E+05	0.00E-01	3.16E+05	0.00E-01
CS138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA139	6.90E-07	2.11E-04	2.37E-05	1.67E-08	1.57E-08	0.00E-01	1.15E-08	0.00E-01
BA140	5.57E+01	1.33E+03	8.64E+02	1.06E+00	3.59E-01	0.00E-01	7.12E-01	0.00E-01
BA141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA142	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA140	6.03E-02	1.30E+04	4.61E-01	2.27E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA142	7.75E-08	9.47E-03	7.01E-07	3.11E-07	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	8.02E-03	2.00E+02	1.05E-01	6.98E-02	3.29E-02	0.00E-01	0.00E-01	0.00E-01
CE143	9.26E-04	2.49E+02	1.14E-02	8.29E+00	3.72E-03	0.00E-01	0.00E-01	0.00E-01
CE144	2.99E-01	1.40E+03	5.57E+00	2.31E+00	1.38E+00	0.00E-01	0.00E-01	0.00E-01
PR143	1.24E-01	8.22E+03	2.50E+00	9.97E-01	5.80E-01	0.00E-01	0.00E-01	0.00E-01
PR144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ND147	1.15E-01	6.94E+03	1.77E+00	1.92E+00	1.13E+00	0.00E-01	0.00E-01	0.00E-01
W 187	2.00E+02	1.54E+05	7.00E+02	5.70E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NP239	5.51E-03	1.60E+03	1.05E-01	9.92E-03	3.11E-02	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = TEENAGER
PATHWAY = ANIMAL DRINKING WATER—MEAT

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	9.85E-01	9.85E-01	0.00E-01	9.85E-01	9.85E-01	9.85E-01	9.85E-01	0.00E-01
C 14	4.10E+01	4.10E+01	2.05E+02	4.10E+01	4.10E+01	4.10E+01	4.10E+01	0.00E-01
NA 24	2.66E-08	2.66E-08	2.66E-08	2.66E-08	2.66E-08	2.66E-08	2.66E-08	0.00E-01
P 32	3.04E+02	6.59E+02	7.84E+03	4.86E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	8.53E-03	1.43E+00	0.00E-01	0.00E-01	1.87E-03	4.74E-03	1.22E-02	0.00E-01
MN 54	1.46E+00	1.51E+01	0.00E-01	7.36E+00	2.20E+00	0.00E-01	0.00E-01	0.00E-01
MN 56	2.45E-28	9.07E-26	0.00E-01	1.38E-27	1.74E-27	0.00E-01	0.00E-01	0.00E-01
FE 55	4.02E+01	7.46E+01	2.43E+02	1.72E+02	0.00E-01	0.00E-01	1.09E+02	0.00E-01
FE 59	2.53E+02	1.55E+03	2.80E+02	6.54E+02	0.00E-01	0.00E-01	2.06E+02	0.00E-01
CO 58	3.90E+01	2.34E+02	0.00E-01	1.69E+01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	1.33E+02	7.70E+02	0.00E-01	5.91E+01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 63	5.18E+02	1.72E+02	1.53E+04	1.08E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	2.51E-26	2.98E-24	4.31E-25	5.50E-26	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CU 64	2.62E-12	4.32E-10	0.00E-01	5.56E-12	1.41E-11	0.00E-01	0.00E-01	0.00E-01
ZN 65	4.31E+02	3.91E+02	2.66E+02	9.24E+02	5.92E+02	0.00E-01	0.00E-01	0.00E-01
ZN 69	4.04E-28	1.06E-26	3.03E-27	5.78E-27	3.77E-27	0.00E-01	0.00E-01	0.00E-01
BR 83	1.60E-26	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 84	7.26E-27	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 85	5.67E-34	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 86	3.37E+02	1.06E+02	0.00E-01	7.17E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 88	1.96E-27	3.15E-34	0.00E-01	3.67E-27	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 89	1.13E-27	2.45E-36	0.00E-01	1.60E-27	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	9.36E+00	3.89E+01	3.27E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	1.99E+02	2.28E+02	9.97E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 91	1.70E-16	1.94E-14	4.28E-15	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 92	8.63E-28	5.16E-25	2.02E-26	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 90	1.54E-05	4.70E+00	5.70E-04	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91M	1.40E-31	1.73E-28	3.66E-30	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91	3.19E-02	4.88E+02	1.19E+00	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 92	1.89E-30	1.79E-24	6.54E-29	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 93	3.41E-18	3.80E-12	1.24E-16	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	3.99E-01	1.34E+03	1.84E+00	5.81E-01	8.53E-01	0.00E-01	0.00E-01	0.00E-01
ZR 97	3.20E-11	1.88E-05	3.51E-10	6.95E-11	1.05E-10	0.00E-01	0.00E-01	0.00E-01
NB 95	7.72E-01	6.00E+03	2.53E+00	1.40E+00	1.36E+00	0.00E-01	0.00E-01	0.00E-01
MO 99	9.56E-02	8.97E-01	0.00E-01	5.01E-01	1.15E+00	0.00E-01	0.00E-01	0.00E-01
TC 99M	6.74E-24	3.42E-22	1.87E-25	5.20E-25	7.75E-24	0.00E-01	2.89E-25	0.00E-01
TC101	1.53E-27	2.67E-35	1.10E-28	1.56E-28	2.82E-27	0.00E-01	9.51E-29	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = TEENAGER
PATHWAY = ANIMAL DRINKING WATER—MEAT

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
RU103	4.99E+01	9.76E+03	1.17E+02	0.00E-01	4.12E+02	0.00E-01	0.00E-01	0.00E-01
RU105	4.13E-26	8.60E-23	1.06E-25	0.00E-01	1.34E-24	0.00E-01	0.00E-01	0.00E-01
RU106	3.10E+02	1.18E+05	2.46E+03	0.00E-01	4.75E+03	0.00E-01	0.00E-01	0.00E-01
AG110M	3.10E+00	1.43E+03	5.38E+00	5.09E+00	9.70E+00	0.00E-01	0.00E-01	0.00E-01
TE125M	5.06E+01	1.12E+03	3.79E+02	1.36E+02	0.00E-01	1.06E+02	0.00E-01	0.00E-01
TE127M	1.27E+02	2.66E+03	1.07E+03	3.79E+02	4.33E+03	2.54E+02	0.00E-01	0.00E-01
TE127	1.38E-15	4.94E-13	6.40E-15	2.27E-15	2.59E-14	4.41E-15	0.00E-01	0.00E-01
TE129M	2.14E+02	5.08E+03	1.35E+03	5.02E+02	5.66E+03	4.37E+02	0.00E-01	0.00E-01
TE129	6.59E-27	1.48E-25	2.71E-26	1.01E-26	1.14E-25	1.94E-26	0.00E-01	0.00E-01
TE131M	1.82E-03	1.75E-01	4.55E-03	2.18E-03	2.28E-02	3.28E-03	0.00E-01	0.00E-01
TE131	1.82E-27	4.79E-28	5.84E-27	2.41E-27	2.55E-26	4.50E-27	0.00E-01	0.00E-01
TE132	3.69E+00	1.24E+02	6.19E+00	3.92E+00	3.76E+01	4.13E+00	0.00E-01	0.00E-01
I 130	1.04E-11	2.01E-11	9.04E-12	2.62E-11	4.03E-11	2.13E-09	0.00E-01	0.00E-01
I 131	3.70E+00	1.36E+00	4.91E+00	6.88E+00	1.18E+01	2.01E+03	0.00E-01	0.00E-01
I 132	8.03E-27	9.74E-27	8.55E-27	2.24E-26	3.52E-26	7.54E-25	0.00E-01	0.00E-01
I 133	5.44E-07	1.35E-06	1.05E-06	1.78E-06	3.13E-06	2.49E-04	0.00E-01	0.00E-01
I 134	2.61E-27	9.57E-29	2.74E-27	7.26E-27	1.14E-26	1.21E-25	0.00E-01	0.00E-01
I 135	3.02E-22	9.04E-22	3.17E-22	8.16E-22	1.29E-21	5.25E-20	0.00E-01	0.00E-01
CS134	5.85E+02	1.57E+01	5.36E+02	1.26E+03	4.01E+02	0.00E-01	1.53E+02	0.00E-01
CS136	5.13E+01	6.15E+00	1.94E+01	7.64E+01	4.16E+01	0.00E-01	6.56E+00	0.00E-01
CS137	3.38E+02	1.38E+01	7.30E+02	9.71E+02	3.30E+02	0.00E-01	1.28E+02	0.00E-01
CS138	1.17E-27	1.06E-30	1.22E-27	2.35E-27	1.73E-27	0.00E-01	2.01E-28	0.00E-01
BA139	1.12E-28	3.42E-26	3.84E-27	2.70E-30	2.55E-30	0.00E-01	1.86E-30	0.00E-01
BA140	3.22E+00	7.72E+01	5.00E+01	6.13E-02	2.08E-02	0.00E-01	4.12E-02	0.00E-01
BA141	1.05E-29	6.72E-34	3.15E-28	2.35E-31	2.18E-31	0.00E-01	1.61E-31	0.00E-01
BA142	1.72E-30	8.59E-41	2.80E-29	2.80E-32	2.37E-32	0.00E-01	1.86E-32	0.00E-01
LA140	3.77E-08	8.14E-03	2.89E-07	1.42E-07	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA142	3.61E-32	4.41E-27	3.26E-31	1.45E-31	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	1.30E-03	3.24E+01	1.70E-02	1.13E-02	5.33E-03	0.00E-01	0.00E-01	0.00E-01
CE143	1.60E-08	4.30E-03	1.96E-07	1.43E-04	6.41E-08	0.00E-01	0.00E-01	0.00E-01
CE144	6.97E-02	3.26E+02	1.30E+00	5.37E-01	3.21E-01	0.00E-01	0.00E-01	0.00E-01
PR143	1.80E-03	1.19E+02	3.62E-02	1.44E-02	8.39E-03	0.00E-01	0.00E-01	0.00E-01
PR144	1.32E-32	2.86E-34	2.60E-31	1.06E-31	6.10E-32	0.00E-01	0.00E-01	0.00E-01
ND147	9.40E-04	5.66E+01	1.44E-02	1.57E-02	9.22E-03	0.00E-01	0.00E-01	0.00E-01
W 187	7.81E-08	6.03E-05	2.74E-07	2.23E-07	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NP239	8.34E-08	2.42E-02	1.59E-06	1.50E-07	4.71E-07	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = TEENAGER
PATHWAY = RIVER SHORELINE DEPOSITS

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem gal)/(Ci min)				
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NA 24	1.05E+01	1.05E+01	1.05E+01	1.05E+01	1.05E+01	1.05E+01	1.05E+01	1.22E+01
P 32	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	4.09E+00	4.09E+00	4.09E+00	4.09E+00	4.09E+00	4.09E+00	4.09E+00	4.84E+00
MN 54	1.22E+03	1.22E+03	1.22E+03	1.22E+03	1.22E+03	1.22E+03	1.22E+03	1.43E+03
MN 56	7.70E-01	7.70E-01	7.70E-01	7.70E-01	7.70E-01	7.70E-01	7.70E-01	9.10E-01
FE 55	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 59	2.40E+02	2.40E+02	2.40E+02	2.40E+02	2.40E+02	2.40E+02	2.40E+02	2.82E+02
CO 58	3.34E+02	3.34E+02	3.34E+02	3.34E+02	3.34E+02	3.34E+02	3.34E+02	3.91E+02
CO 60	1.89E+04	1.89E+04	1.89E+04	1.89E+04	1.89E+04	1.89E+04	1.89E+04	2.22E+04
NI 63	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	2.54E-01	2.54E-01	2.54E-01	2.54E-01	2.54E-01	2.54E-01	2.54E-01	2.95E-01
CU 64	5.29E-01	5.29E-01	5.29E-01	5.29E-01	5.29E-01	5.29E-01	5.29E-01	6.00E-01
ZN 65	6.56E+02	6.56E+02	6.56E+02	6.56E+02	6.56E+02	6.56E+02	6.56E+02	7.54E+02
ZN 69	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 83	4.16E-03	4.16E-03	4.16E-03	4.16E-03	4.16E-03	4.16E-03	4.16E-03	6.05E-03
BR 84	1.56E-01	1.56E-01	1.56E-01	1.56E-01	1.56E-01	1.56E-01	1.56E-01	1.83E-01
BR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 86	7.92E+00	7.92E+00	7.92E+00	7.92E+00	7.92E+00	7.92E+00	7.92E+00	9.05E+00
RB 88	2.31E-02	2.31E-02	2.31E-02	2.31E-02	2.31E-02	2.31E-02	2.31E-02	2.64E-02
RB 89	8.14E-02	8.14E-02	8.14E-02	8.14E-02	8.14E-02	8.14E-02	8.14E-02	9.76E-02
SR 89	1.90E-02	1.90E-02	1.90E-02	1.90E-02	1.90E-02	1.90E-02	1.90E-02	2.21E-02
SR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 91	1.87E+00	1.87E+00	1.87E+00	1.87E+00	1.87E+00	1.87E+00	1.87E+00	2.19E+00
SR 92	6.66E-01	6.66E-01	6.66E-01	6.66E-01	6.66E-01	6.66E-01	6.66E-01	7.40E-01
Y 90	3.95E-03	3.95E-03	3.95E-03	3.95E-03	3.95E-03	3.95E-03	3.95E-03	4.67E-03
Y 91M	8.09E-02	8.09E-02	8.09E-02	8.09E-02	8.09E-02	8.09E-02	8.09E-02	9.36E-02
Y 91	9.45E-01	9.45E-01	9.45E-01	9.45E-01	9.45E-01	9.45E-01	9.45E-01	1.06E+00
Y 92	1.56E-01	1.56E-01	1.56E-01	1.56E-01	1.56E-01	1.56E-01	1.56E-01	1.85E-01
Y 93	1.60E-01	1.60E-01	1.60E-01	1.60E-01	1.60E-01	1.60E-01	1.60E-01	2.19E-01
ZR 95	2.16E+02	2.16E+02	2.16E+02	2.16E+02	2.16E+02	2.16E+02	2.16E+02	2.50E+02
ZR 97	2.59E+00	2.59E+00	2.59E+00	2.59E+00	2.59E+00	2.59E+00	2.59E+00	3.02E+00
NB 95	1.21E+02	1.21E+02	1.21E+02	1.21E+02	1.21E+02	1.21E+02	1.21E+02	1.42E+02
MO 99	3.51E+00	3.51E+00	3.51E+00	3.51E+00	3.51E+00	3.51E+00	3.51E+00	4.06E+00
TC 99M	1.60E-01	1.60E-01	1.60E-01	1.60E-01	1.60E-01	1.60E-01	1.60E-01	1.83E-01
TC101	1.33E-02	1.33E-02	1.33E-02	1.33E-02	1.33E-02	1.33E-02	1.33E-02	1.48E-02

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = TEENAGER
PATHWAY = RIVER SHORELINE DEOSITS

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
RU103	9.52E+01	9.52E+01	9.52E+01	9.52E+01	9.52E+01	9.52E+01	9.52E+01	1.11E+02
RU105	5.50E-01	5.50E-01	5.50E-01	5.50E-01	5.50E-01	5.50E-01	5.50E-01	6.24E-01
RU106	3.71E+02	3.71E+02	3.71E+02	3.71E+02	3.71E+02	3.71E+02	3.71E+02	4.45E+02
AG110M	3.03E+03	3.03E+03	3.03E+03	3.03E+03	3.03E+03	3.03E+03	3.03E+03	3.54E+03
TE125M	1.37E+00	1.37E+00	1.37E+00	1.37E+00	1.37E+00	1.37E+00	1.37E+00	1.88E+00
TE127M	8.06E-02	8.06E-02	8.06E-02	8.06E-02	8.06E-02	8.06E-02	8.06E-02	9.53E-02
TE127	2.60E-03	2.60E-03	2.60E-03	2.60E-03	2.60E-03	2.60E-03	2.60E-03	2.86E-03
TE129M	1.74E+01	1.74E+01	1.74E+01	1.74E+01	1.74E+01	1.74E+01	1.74E+01	2.03E+01
TE129	2.17E-02	2.17E-02	2.17E-02	2.17E-02	2.17E-02	2.17E-02	2.17E-02	2.57E-02
TE131M	7.04E+00	7.04E+00	7.04E+00	7.04E+00	7.04E+00	7.04E+00	7.04E+00	8.30E+00
TE131	2.18E-02	2.18E-02	2.18E-02	2.18E-02	2.18E-02	2.18E-02	2.18E-02	2.58E+01
TE132	3.72E+00	3.72E+00	3.72E+00	3.72E+00	3.72E+00	3.72E+00	3.72E+00	4.38E+00
I 130	4.81E+00	4.81E+00	4.81E+00	4.81E+00	4.81E+00	4.81E+00	4.81E+00	5.84E+00
I 131	1.51E+01	1.51E+01	1.51E+01	1.51E+01	1.51E+01	1.51E+01	1.51E+01	1.84E+01
I 132	1.06E+00	1.06E+00	1.06E+00	1.06E+00	1.06E+00	1.06E+00	1.06E+00	1.25E+00
I 133	2.15E+00	2.15E+00	2.15E+00	2.15E+00	2.15E+00	2.15E+00	2.15E+00	2.62E+00
I 134	3.62E-01	3.62E-01	3.62E-01	3.62E-01	3.62E-01	3.62E-01	3.62E-01	4.30E-01
I 135	2.19E+00	2.19E+00	2.19E+00	2.19E+00	2.19E+00	2.19E+00	2.19E+00	2.56E+00
CS134	6.01E+03	6.01E+03	6.01E+03	6.01E+03	6.01E+03	6.01E+03	6.01E+03	7.01E+03
CS136	1.32E+02	1.32E+02	1.32E+02	1.32E+02	1.32E+02	1.32E+02	1.32E+02	1.50E+02
CS137	9.06E+03	9.06E+03	9.06E+03	9.06E+03	9.06E+03	9.06E+03	9.06E+03	1.06E+04
CS138	2.78E-01	2.78E-01	2.78E-01	2.78E-01	2.78E-01	2.78E-01	2.78E-01	3.18E-01
BA139	8.79E-02	8.79E-02	8.79E-02	8.79E-02	8.79E-02	8.79E-02	8.79E-02	9.89E-02
BA140	1.81E+01	1.81E+01	1.81E+01	1.81E+01	1.81E+01	1.81E+01	1.81E+01	2.06E+01
BA141	2.92E-02	2.92E-02	2.92E-02	2.92E-02	2.92E-02	2.92E-02	2.92E-02	3.33E-02
BA142	2.67E-02	2.67E-02	2.67E-02	2.67E-02	2.67E-02	2.67E-02	2.67E-02	3.05E-02
LA140	1.69E+01	1.69E+01	1.69E+01	1.69E+01	1.69E+01	1.69E+01	1.69E+01	1.91E+01
LA142	6.19E-01	6.19E-01	6.19E-01	6.19E-01	6.19E-01	6.19E-01	6.19E-01	7.43E-01
CE141	1.20E+01	1.20E+01	1.20E+01	1.20E+01	1.20E+01	1.20E+01	1.20E+01	1.35E+01
CE143	2.04E+00	2.04E+00	2.04E+00	2.04E+00	2.04E+00	2.04E+00	2.04E+00	2.32E+00
CE144	6.12E+01	6.12E+01	6.12E+01	6.12E+01	6.12E+01	6.12E+01	6.12E+01	7.08E+01
PR143	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
PR144	1.27E-03	1.27E-03	1.27E-03	1.27E-03	1.27E-03	1.27E-03	1.27E-03	1.46E-03
ND147	7.46E+00	7.46E+00	7.46E+00	7.46E+00	7.46E+00	7.46E+00	7.46E+00	8.95E+00
W 187	2.07E+00	2.07E+00	2.07E+00	2.07E+00	2.07E+00	2.07E+00	2.07E+00	2.40E+00
NP239	1.50E+00	1.50E+00	1.50E+00	1.50E+00	1.50E+00	1.50E+00	1.50E+00	1.74E+00

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = TEENAGER
PATHWAY = SWIMMING

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NA 24	1.75E+02	1.75E+02	1.75E+02	1.75E+02	1.75E+02	1.75E+02	1.75E+02	0.00E-01
P 32	1.45E-01	1.45E-01	1.45E-01	1.45E-01	1.45E-01	1.45E-01	1.45E-01	0.00E-01
CR 51	1.17E+00	1.17E+00	1.17E+00	1.17E+00	1.17E+00	1.17E+00	1.17E+00	0.00E-01
MN 54	3.39E+01	3.39E+01	3.39E+01	3.39E+01	3.39E+01	3.39E+01	3.39E+01	0.00E-01
MN 56	7.03E+01	7.03E+01	7.03E+01	7.03E+01	7.03E+01	7.03E+01	7.03E+01	0.00E-01
FE 55	1.44E-03	1.44E-03	1.44E-03	1.44E-03	1.44E-03	1.44E-03	1.44E-03	0.00E-01
FE 59	4.97E+01	4.97E+01	4.97E+01	4.97E+01	4.97E+01	4.97E+01	4.97E+01	0.00E-01
CO 58	4.06E+01	4.06E+01	4.06E+01	4.06E+01	4.06E+01	4.06E+01	4.06E+01	0.00E-01
CO 60	1.04E+02	1.04E+02	1.04E+02	1.04E+02	1.04E+02	1.04E+02	1.04E+02	0.00E-01
NI 63	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	2.20E+01	2.20E+01	2.20E+01	2.20E+01	2.20E+01	2.20E+01	2.20E+01	0.00E-01
CU 64	8.31E+00	8.31E+00	8.31E+00	8.31E+00	8.31E+00	8.31E+00	8.31E+00	0.00E-01
ZN 65	2.48E+01	2.48E+01	2.48E+01	2.48E+01	2.48E+01	2.48E+01	2.48E+01	0.00E-01
ZN 69	3.36E-02	3.36E-02	3.36E-02	3.36E-02	3.36E-02	3.36E-02	3.36E-02	0.00E-01
BR 83	3.73E-01	3.73E-01	3.73E-01	3.73E-01	3.73E-01	3.73E-01	3.73E-01	0.00E-01
BR 84	6.93E+01	6.93E+01	6.93E+01	6.93E+01	6.93E+01	6.93E+01	6.93E+01	0.00E-01
BR 85	7.41E-02	7.41E-02	7.41E-02	7.41E-02	7.41E-02	7.41E-02	7.41E-02	0.00E-01
RB 86	3.84E+00	3.84E+00	3.84E+00	3.84E+00	3.84E+00	3.84E+00	3.84E+00	0.00E-01
RB 88	2.15E+01	2.15E+01	2.15E+01	2.15E+01	2.15E+01	2.15E+01	2.15E+01	0.00E-01
RB 89	7.74E+01	7.74E+01	7.74E+01	7.74E+01	7.74E+01	7.74E+01	7.74E+01	0.00E-01
SR 89	1.04E-01	1.04E-01	1.04E-01	1.04E-01	1.04E-01	1.04E-01	1.04E-01	0.00E-01
SR 90	1.22E-02	1.22E-02	1.22E-02	1.22E-02	1.22E-02	1.22E-02	1.22E-02	0.00E-01
SR 91	4.26E+01	4.26E+01	4.26E+01	4.26E+01	4.26E+01	4.26E+01	4.26E+01	0.00E-01
SR 92	5.72E+01	5.72E+01	5.72E+01	5.72E+01	5.72E+01	5.72E+01	5.72E+01	0.00E-01
Y 90	2.93E-01	2.93E-01	2.93E-01	2.93E-01	2.93E-01	2.93E-01	2.93E-01	0.00E-01
Y 91M	2.08E+01	2.08E+01	2.08E+01	2.08E+01	2.08E+01	2.08E+01	2.08E+01	0.00E-01
Y 91	1.51E-01	1.51E-01	1.51E-01	1.51E-01	1.51E-01	1.51E-01	1.51E-01	0.00E-01
Y 92	1.02E+01	1.02E+01	1.02E+01	1.02E+01	1.02E+01	1.02E+01	1.02E+01	0.00E-01
Y 93	4.26E+00	4.26E+00	4.26E+00	4.26E+00	4.26E+00	4.26E+00	4.26E+00	0.00E-01
ZR 95	3.39E+01	3.39E+01	3.39E+01	3.39E+01	3.39E+01	3.39E+01	3.39E+01	0.00E-01
ZR 97	3.37E+01	3.37E+01	3.37E+01	3.37E+01	3.37E+01	3.37E+01	3.37E+01	0.00E-01
NB 95	3.16E+01	3.16E+01	3.16E+01	3.16E+01	3.16E+01	3.16E+01	3.16E+01	0.00E-01
MO 99	1.06E+01	1.06E+01	1.06E+01	1.06E+01	1.06E+01	1.06E+01	1.06E+01	0.00E-01
TC 99M	5.36E+00	5.36E+00	5.36E+00	5.36E+00	5.36E+00	5.36E+00	5.36E+00	0.00E-01
TC101	1.14E+01	1.14E+01	1.14E+01	1.14E+01	1.14E+01	1.14E+01	1.14E+01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = TEENAGER
PATHWAY = SWIMMING

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
RU103	2.01E+01	2.01E+01	2.01E+01	2.01E+01	2.01E+01	2.01E+01	2.01E+01	0.00E-01
RU105	2.67E+01	2.67E+01	2.67E+01	2.67E+01	2.67E+01	2.67E+01	2.67E+01	0.00E-01
RU106	8.58E+00	8.58E+00	8.58E+00	8.58E+00	8.58E+00	8.58E+00	8.58E+00	0.00E-01
AG110M	1.11E+02	1.11E+02	1.11E+02	1.11E+02	1.11E+02	1.11E+02	1.11E+02	0.00E-01
TE125M	8.33E-02	8.33E-02	8.33E-02	8.33E-02	8.33E-02	8.33E-02	8.33E-02	0.00E-01
TE127M	5.87E-03	5.87E-03	5.87E-03	5.87E-03	5.87E-03	5.87E-03	5.87E-03	0.00E-01
TE127	6.28E-02	6.28E-02	6.28E-02	6.28E-02	6.28E-02	6.28E-02	6.28E-02	0.00E-01
TE129M	4.74E+00	4.74E+00	4.74E+00	4.74E+00	4.74E+00	4.74E+00	4.74E+00	0.00E-01
TE129	4.04E+00	4.04E+00	4.04E+00	4.04E+00	4.04E+00	4.04E+00	4.04E+00	0.00E-01
TE131M	4.96E+01	4.96E+01	4.96E+01	4.96E+01	4.96E+01	4.96E+01	4.96E+01	0.00E-01
TE131	1.41E+01	1.41E+01	1.41E+01	1.41E+01	1.41E+01	1.41E+01	1.41E+01	0.00E-01
TE132	9.02E+00	9.02E+00	9.02E+00	9.02E+00	9.02E+00	9.02E+00	9.02E+00	0.00E-01
I 130	8.76E+01	8.76E+01	8.76E+01	8.76E+01	8.76E+01	8.76E+01	8.76E+01	0.00E-01
I 131	1.76E+01	1.76E+01	1.76E+01	1.76E+01	1.76E+01	1.76E+01	1.76E+01	0.00E-01
I 132	9.64E+01	9.64E+01	9.64E+01	9.64E+01	9.64E+01	9.64E+01	9.64E+01	0.00E-01
I 133	2.16E+01	2.16E+01	2.16E+01	2.16E+01	2.16E+01	2.16E+01	2.16E+01	0.00E-01
I 134	8.76E+01	8.76E+01	8.76E+01	8.76E+01	8.76E+01	8.76E+01	8.76E+01	0.00E-01
I 135	7.37E+01	7.37E+01	7.37E+01	7.37E+01	7.37E+01	7.37E+01	7.37E+01	0.00E-01
CS134	6.55E+01	6.55E+01	6.55E+01	6.55E+01	6.55E+01	6.55E+01	6.55E+01	0.00E-01
CS136	9.26E+01	9.26E+01	9.26E+01	9.26E+01	9.26E+01	9.26E+01	9.26E+01	0.00E-01
CS137	2.26E+01	2.26E+01	2.26E+01	2.26E+01	2.26E+01	2.26E+01	2.26E+01	0.00E-01
CS138	7.94E+01	7.94E+01	7.94E+01	7.94E+01	7.94E+01	7.94E+01	7.94E+01	0.00E-01
BA139	1.65E+00	1.65E+00	1.65E+00	1.65E+00	1.65E+00	1.65E+00	1.65E+00	0.00E-01
BA140	1.11E+01	1.11E+01	1.11E+01	1.11E+01	1.11E+01	1.11E+01	1.11E+01	0.00E-01
BA141	1.98E+01	1.98E+01	1.98E+01	1.98E+01	1.98E+01	1.98E+01	1.98E+01	0.00E-01
BA142	3.37E+01	3.37E+01	3.37E+01	3.37E+01	3.37E+01	3.37E+01	3.37E+01	0.00E-01
LA140	9.24E+01	9.24E+01	9.24E+01	9.24E+01	9.24E+01	9.24E+01	9.24E+01	0.00E-01
LA142	9.71E+01	9.71E+01	9.71E+01	9.71E+01	9.71E+01	9.71E+01	9.71E+01	0.00E-01
CE141	2.93E+00	2.93E+00	2.93E+00	2.93E+00	2.93E+00	2.93E+00	2.93E+00	0.00E-01
CE143	1.28E+01	1.28E+01	1.28E+01	1.28E+01	1.28E+01	1.28E+01	1.28E+01	0.00E-01
CE144	1.94E+00	1.94E+00	1.94E+00	1.94E+00	1.94E+00	1.94E+00	1.94E+00	0.00E-01
PR143	3.61E-02	3.61E-02	3.61E-02	3.61E-02	3.61E-02	3.61E-02	3.61E-02	0.00E-01
PR144	9.94E-01	9.94E-01	9.94E-01	9.94E-01	9.94E-01	9.94E-01	9.94E-01	0.00E-01
ND147	6.32E+00	6.32E+00	6.32E+00	6.32E+00	6.32E+00	6.32E+00	6.32E+00	0.00E-01
W 187	1.87E+01	1.87E+01	1.87E+01	1.87E+01	1.87E+01	1.87E+01	1.87E+01	0.00E-01
NP239	5.41E+00	5.41E+00	5.41E+00	5.41E+00	5.41E+00	5.41E+00	5.41E+00	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = TEENAGER
PATHWAY = BOATING

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NA 24	1.01E+02	1.01E+02	1.01E+02	1.01E+02	1.01E+02	1.01E+02	1.01E+02	0.00E-01
P 32	8.35E-02	8.35E-02	8.35E-02	8.35E-02	8.35E-02	8.35E-02	8.35E-02	0.00E-01
CR 51	6.78E-01	6.78E-01	6.78E-01	6.78E-01	6.78E-01	6.78E-01	6.78E-01	0.00E-01
MN 54	1.96E+01	1.96E+01	1.96E+01	1.96E+01	1.96E+01	1.96E+01	1.96E+01	0.00E-01
MN 56	4.06E+01	4.06E+01	4.06E+01	4.06E+01	4.06E+01	4.06E+01	4.06E+01	0.00E-01
FE 55	8.35E-04	8.35E-04	8.35E-04	8.35E-04	8.35E-04	8.35E-04	8.35E-04	0.00E-01
FE 59	2.87E+01	2.87E+01	2.87E+01	2.87E+01	2.87E+01	2.87E+01	2.87E+01	0.00E-01
CO 58	2.35E+01	2.35E+01	2.35E+01	2.35E+01	2.35E+01	2.35E+01	2.35E+01	0.00E-01
CO 60	6.00E+01	6.00E+01	6.00E+01	6.00E+01	6.00E+01	6.00E+01	6.00E+01	0.00E-01
NI 63	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	1.27E+01	1.27E+01	1.27E+01	1.27E+01	1.27E+01	1.27E+01	1.27E+01	0.00E-01
CU 64	4.80E+00	4.80E+00	4.80E+00	4.80E+00	4.80E+00	4.80E+00	4.80E+00	0.00E-01
ZN 65	1.44E+01	1.44E+01	1.44E+01	1.44E+01	1.44E+01	1.44E+01	1.44E+01	0.00E-01
ZN 69	1.94E-02	1.94E-02	1.94E-02	1.94E-02	1.94E-02	1.94E-02	1.94E-02	0.00E-01
BR 83	2.15E-01	2.15E-01	2.15E-01	2.15E-01	2.15E-01	2.15E-01	2.15E-01	0.00E-01
BR 84	4.01E+01	4.01E+01	4.01E+01	4.01E+01	4.01E+01	4.01E+01	4.01E+01	0.00E-01
BR 85	4.28E-02	4.28E-02	4.28E-02	4.28E-02	4.28E-02	4.28E-02	4.28E-02	0.00E-01
RB 86	2.22E+00	2.22E+00	2.22E+00	2.22E+00	2.22E+00	2.22E+00	2.22E+00	0.00E-01
RB 88	1.24E+01	1.24E+01	1.24E+01	1.24E+01	1.24E+01	1.24E+01	1.24E+01	0.00E-01
RB 89	4.47E+01	4.47E+01	4.47E+01	4.47E+01	4.47E+01	4.47E+01	4.47E+01	0.00E-01
SR 89	6.00E-02	6.00E-02	6.00E-02	6.00E-02	6.00E-02	6.00E-02	6.00E-02	0.00E-01
SR 90	7.04E-03	7.04E-03	7.04E-03	7.04E-03	7.04E-03	7.04E-03	7.04E-03	0.00E-01
SR 91	2.46E+01	2.46E+01	2.46E+01	2.46E+01	2.46E+01	2.46E+01	2.46E+01	0.00E-01
SR 92	3.31E+01	3.31E+01	3.31E+01	3.31E+01	3.31E+01	3.31E+01	3.31E+01	0.00E-01
Y 90	1.69E-01	1.69E-01	1.69E-01	1.69E-01	1.69E-01	1.69E-01	1.69E-01	0.00E-01
Y 91M	1.20E+01	1.20E+01	1.20E+01	1.20E+01	1.20E+01	1.20E+01	1.20E+01	0.00E-01
Y 91	8.74E-02	8.74E-02	8.74E-02	8.74E-02	8.74E-02	8.74E-02	8.74E-02	0.00E-01
Y 92	5.89E+00	5.89E+00	5.89E+00	5.89E+00	5.89E+00	5.89E+00	5.89E+00	0.00E-01
Y 93	2.46E+00	2.46E+00	2.46E+00	2.46E+00	2.46E+00	2.46E+00	2.46E+00	0.00E-01
ZR 95	1.96E+01	1.96E+01	1.96E+01	1.96E+01	1.96E+01	1.96E+01	1.96E+01	0.00E-01
ZR 97	1.95E+01	1.95E+01	1.95E+01	1.95E+01	1.95E+01	1.95E+01	1.95E+01	0.00E-01
NB 95	1.83E+01	1.83E+01	1.83E+01	1.83E+01	1.83E+01	1.83E+01	1.83E+01	0.00E-01
MO 99	6.12E+00	6.12E+00	6.12E+00	6.12E+00	6.12E+00	6.12E+00	6.12E+00	0.00E-01
TC 99M	3.10E+00	3.10E+00	3.10E+00	3.10E+00	3.10E+00	3.10E+00	3.10E+00	0.00E-01
TC101	6.62E+00	6.62E+00	6.62E+00	6.62E+00	6.62E+00	6.62E+00	6.62E+00	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = TEENAGER
PATHWAY = BOATING

(mrem gal)/(Ci min)

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
RU103	1.16E+01	1.16E+01	1.16E+01	1.16E+01	1.16E+01	1.16E+01	1.16E+01	0.00E-01
RU105	1.54E+01	1.54E+01	1.54E+01	1.54E+01	1.54E+01	1.54E+01	1.54E+01	0.00E-01
RU106	4.96E+00	4.96E+00	4.96E+00	4.96E+00	4.96E+00	4.96E+00	4.96E+00	0.00E-01
AG110M	6.39E+01	6.39E+01	6.39E+01	6.39E+01	6.39E+01	6.39E+01	6.39E+01	0.00E-01
TE125M	4.81E-02	4.81E-02	4.81E-02	4.81E-02	4.81E-02	4.81E-02	4.81E-02	0.00E-01
TE127M	3.39E-03	3.39E-03	3.39E-03	3.39E-03	3.39E-03	3.39E-03	3.39E-03	0.00E-01
TE127	3.63E-02	3.63E-02	3.63E-02	3.63E-02	3.63E-02	3.63E-02	3.63E-02	0.00E-01
TE129M	2.74E+00	2.74E+00	2.74E+00	2.74E+00	2.74E+00	2.74E+00	2.74E+00	0.00E-01
TE129	2.34E+00	2.34E+00	2.34E+00	2.34E+00	2.34E+00	2.34E+00	2.34E+00	0.00E-01
TE131M	2.86E+01	2.86E+01	2.86E+01	2.86E+01	2.86E+01	2.86E+01	2.86E+01	0.00E-01
TE131	8.18E+00	8.18E+00	8.18E+00	8.18E+00	8.18E+00	8.18E+00	8.18E+00	0.00E-01
TE132	5.21E+00	5.21E+00	5.21E+00	5.21E+00	5.21E+00	5.21E+00	5.21E+00	0.00E-01
I 130	5.06E+01	5.06E+01	5.06E+01	5.06E+01	5.06E+01	5.06E+01	5.06E+01	0.00E-01
I 131	1.02E+01	1.02E+01	1.02E+01	1.02E+01	1.02E+01	1.02E+01	1.02E+01	0.00E-01
I 132	5.57E+01	5.57E+01	5.57E+01	5.57E+01	5.57E+01	5.57E+01	5.57E+01	0.00E-01
I 133	1.25E+01	1.25E+01	1.25E+01	1.25E+01	1.25E+01	1.25E+01	1.25E+01	0.00E-01
I 134	5.06E+01	5.06E+01	5.06E+01	5.06E+01	5.06E+01	5.06E+01	5.06E+01	0.00E-01
I 135	4.26E+01	4.26E+01	4.26E+01	4.26E+01	4.26E+01	4.26E+01	4.26E+01	0.00E-01
CS134	3.78E+01	3.78E+01	3.78E+01	3.78E+01	3.78E+01	3.78E+01	3.78E+01	0.00E-01
CS136	5.35E+01	5.35E+01	5.35E+01	5.35E+01	5.35E+01	5.35E+01	5.35E+01	0.00E-01
CS137	1.31E+01	1.31E+01	1.31E+01	1.31E+01	1.31E+01	1.31E+01	1.31E+01	0.00E-01
CS138	4.59E+01	4.59E+01	4.59E+01	4.59E+01	4.59E+01	4.59E+01	4.59E+01	0.00E-01
BA139	9.55E-01	9.55E-01	9.55E-01	9.55E-01	9.55E-01	9.55E-01	9.55E-01	0.00E-01
BA140	6.39E+00	6.39E+00	6.39E+00	6.39E+00	6.39E+00	6.39E+00	6.39E+00	0.00E-01
BA141	1.14E+01	1.14E+01	1.14E+01	1.14E+01	1.14E+01	1.14E+01	1.14E+01	0.00E-01
BA142	1.95E+01	1.95E+01	1.95E+01	1.95E+01	1.95E+01	1.95E+01	1.95E+01	0.00E-01
LA140	5.34E+01	5.34E+01	5.34E+01	5.34E+01	5.34E+01	5.34E+01	5.34E+01	0.00E-01
LA142	5.61E+01	5.61E+01	5.61E+01	5.61E+01	5.61E+01	5.61E+01	5.61E+01	0.00E-01
CE141	1.70E+00	1.70E+00	1.70E+00	1.70E+00	1.70E+00	1.70E+00	1.70E+00	0.00E-01
CE143	7.42E+00	7.42E+00	7.42E+00	7.42E+00	7.42E+00	7.42E+00	7.42E+00	0.00E-01
CE144	1.12E+00	1.12E+00	1.12E+00	1.12E+00	1.12E+00	1.12E+00	1.12E+00	0.00E-01
PR143	2.09E-02	2.09E-02	2.09E-02	2.09E-02	2.09E-02	2.09E-02	2.09E-02	0.00E-01
PR144	5.74E-01	5.74E-01	5.74E-01	5.74E-01	5.74E-01	5.74E-01	5.74E-01	0.00E-01
ND147	3.65E+00	3.65E+00	3.65E+00	3.65E+00	3.65E+00	3.65E+00	3.65E+00	0.00E-01
W 187	1.08E+01	1.08E+01	1.08E+01	1.08E+01	1.08E+01	1.08E+01	1.08E+01	0.00E-01
NP239	3.13E+00	3.13E+00	3.13E+00	3.13E+00	3.13E+00	3.13E+00	3.13E+00	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = CHILD
PATHWAY = POTABLE WATER

(mrem gal)/(Ci min)

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
H 3	2.97E+01	2.97E+01	0.00E-01	2.97E+01	2.97E+01	2.97E+01	2.97E+01	0.00E-01
C 14	6.19E+02	6.19E+02	3.10E+03	6.19E+02	6.19E+02	6.19E+02	6.19E+02	0.00E-01
NA 24	4.91E+02	4.91E+02	4.91E+02	4.91E+02	4.91E+02	4.91E+02	4.91E+02	0.00E-01
P 32	7.75E+03	5.56E+03	2.01E+05	9.41E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	2.22E+00	1.18E+02	0.00E-01	0.00E-01	3.37E-01	1.23E+00	2.25E+00	0.00E-01
MN 54	7.28E+02	2.29E+03	0.00E-01	2.73E+03	7.66E+02	0.00E-01	0.00E-01	0.00E-01
MN 56	2.96E-02	1.90E+01	0.00E-01	1.31E-01	1.59E-01	0.00E-01	0.00E-01	0.00E-01
FE 55	4.83E+02	2.89E+02	2.94E+03	1.56E+03	0.00E-01	0.00E-01	8.82E+02	0.00E-01
FE 59	3.35E+03	7.00E+03	4.16E+03	6.73E+03	0.00E-01	0.00E-01	1.95E+03	0.00E-01
CO 58	1.40E+03	2.66E+03	0.00E-01	4.56E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	3.99E+03	7.50E+03	0.00E-01	1.35E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 63	4.68E+03	4.96E+02	1.38E+05	7.37E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	4.24E-02	8.90E+00	7.72E-01	7.27E-02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CU 64	1.02E+01	7.91E+02	0.00E-01	1.69E+01	4.07E+01	0.00E-01	0.00E-01	0.00E-01
ZN 65	5.79E+03	1.64E+03	3.50E+03	9.31E+03	5.87E+03	0.00E-01	0.00E-01	0.00E-01
ZN 69	3.61E-08	2.47E-05	2.71E-07	3.91E-07	2.37E-07	0.00E-01	0.00E-01	0.00E-01
BR 83	4.17E-02	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 84	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 86	1.02E+04	1.06E+03	0.00E-01	1.65E+04	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	9.52E+03	1.29E+04	3.33E+05	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	1.32E+05	5.86E+04	6.55E+06	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 91	4.01E+01	2.35E+03	1.06E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 92	2.01E-01	9.48E+01	5.01E+00	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 90	2.17E-01	2.31E+04	8.12E+00	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91	4.07E+00	2.03E+04	1.52E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 92	2.44E-04	2.46E+02	8.52E-03	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 93	1.54E-02	8.35E+03	5.60E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	5.75E+00	6.73E+03	2.94E+01	6.46E+00	9.24E+00	0.00E-01	0.00E-01	0.00E-01
ZR 97	5.70E-02	1.46E+04	6.68E-01	9.65E-02	1.39E-01	0.00E-01	0.00E-01	0.00E-01
NB 95	1.57E+00	4.06E+03	5.65E+00	2.20E+00	2.07E+00	0.00E-01	0.00E-01	0.00E-01
MO 99	6.54E+02	2.19E+03	0.00E-01	2.64E+03	5.65E+03	0.00E-01	0.00E-01	0.00E-01
TC 99M	4.84E-01	1.66E+01	1.49E-02	2.92E-02	4.24E-01	0.00E-01	1.48E-02	0.00E-01
TC101	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = CHILD
PATHWAY = POTABLE WATER

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem gal)/(Ci min)				
RU103	7.07E+01	4.75E+03	1.84E+02	0.00E-01	4.63E+02	0.00E-01	0.00E-01	0.00E-01
RU105	1.41E-01	2.53E+02	3.88E-01	0.00E-01	3.41E+00	0.00E-01	0.00E-01	0.00E-01
RU106	3.73E+02	4.65E+04	2.99E+03	0.00E-01	4.04E+03	0.00E-01	0.00E-01	0.00E-01
AG110M	7.43E+01	1.10E+04	1.38E+02	9.29E+01	1.73E+02	0.00E-01	0.00E-01	0.00E-01
TE125M	3.84E+02	2.78E+03	2.88E+03	7.81E+02	0.00E-01	8.09E+02	0.00E-01	0.00E-01
TE127M	8.72E+02	5.95E+03	7.35E+03	1.98E+03	2.09E+04	1.76E+03	0.00E-01	0.00E-01
TE127	4.36E+00	7.94E+02	2.03E+01	5.48E+00	5.78E+01	1.41E+01	0.00E-01	0.00E-01
TE129M	1.89E+03	1.49E+04	1.22E+04	3.41E+03	3.58E+04	3.94E+03	0.00E-01	0.00E-01
TE129	4.81E-06	1.26E-03	2.02E-05	5.65E-06	5.92E-05	1.44E-05	0.00E-01	0.00E-01
TE131M	3.89E+02	1.48E+04	1.06E+03	3.66E+02	3.54E+03	7.52E+02	0.00E-01	0.00E-01
TE131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE132	1.12E+03	9.31E+03	2.09E+03	9.25E+02	8.59E+03	1.35E+03	0.00E-01	0.00E-01
I 130	2.02E+02	1.83E+02	1.94E+02	3.92E+02	5.86E+02	4.32E+04	0.00E-01	0.00E-01
I 131	2.31E+03	3.61E+02	4.04E+03	4.06E+03	6.67E+03	1.34E+06	0.00E-01	0.00E-01
I 132	1.25E-01	3.20E-01	1.48E-01	2.72E-01	4.17E-01	1.26E+01	0.00E-01	0.00E-01
I 133	3.19E+02	3.39E+02	6.81E+02	8.42E+02	1.40E+03	1.56E+05	0.00E-01	0.00E-01
I 134	5.09E-07	7.34E-07	5.96E-07	1.11E-06	1.69E-06	2.55E-05	0.00E-01	0.00E-01
I 135	3.06E+01	4.93E+01	3.59E+01	6.47E+01	9.92E+01	5.73E+03	0.00E-01	0.00E-01
CS134	2.07E+04	5.29E+02	5.98E+04	9.82E+04	3.04E+04	0.00E-01	1.09E+04	0.00E-01
CS136	1.01E+04	5.51E+02	5.70E+03	1.57E+04	8.35E+03	0.00E-01	1.24E+03	0.00E-01
CS137	1.18E+04	5.02E+02	8.37E+04	8.01E+04	2.61E+04	0.00E-01	9.39E+03	0.00E-01
CS138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA139	1.71E-05	3.41E-02	5.91E-04	3.16E-07	2.76E-07	0.00E-01	1.86E-07	0.00E-01
BA140	1.18E+03	1.02E+04	2.01E+04	1.76E+01	5.74E+00	0.00E-01	1.05E+01	0.00E-01
BA141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA142	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA140	2.01E-01	1.67E+04	1.71E+00	5.98E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA142	2.73E-07	1.73E-01	2.74E-06	8.72E-07	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	7.37E-01	6.19E+03	9.94E+00	4.96E+00	2.17E+00	0.00E-01	0.00E-01	0.00E-01
CE143	8.51E-02	8.60E+03	1.08E+00	5.87E+02	2.46E-01	0.00E-01	0.00E-01	0.00E-01
CE144	2.83E+01	4.34E+04	5.31E+02	1.66E+02	9.22E+01	0.00E-01	0.00E-01	0.00E-01
PR143	4.74E-01	1.03E+04	9.56E+00	2.87E+00	1.55E+00	0.00E-01	0.00E-01	0.00E-01
PR144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ND147	4.21E-01	8.61E+03	6.71E+00	5.43E+00	2.98E+00	0.00E-01	0.00E-01	0.00E-01
W 187	1.46E+01	4.56E+03	5.48E+01	3.24E+01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NP239	5.05E-02	5.32E+03	1.00E+00	7.19E-02	2.08E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = CHILD
PATHWAY = FRESH WATER FISH

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem gal)/(Ci min)				
H 3	3.61E-01	3.61E-01	0.00E-01	3.61E-01	3.61E-01	3.61E-01	3.61E-01	0.00E-01
C 14	3.85E+04	3.85E+04	1.93E+05	3.85E+04	3.85E+04	3.85E+04	3.85E+04	0.00E-01
NA 24	6.61E+02	6.61E+02	6.61E+02	6.61E+02	6.61E+02	6.61E+02	6.61E+02	0.00E-01
P 32	1.05E+07	7.52E+06	2.72E+08	1.27E+07	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	6.01E+00	3.19E+02	0.00E-01	0.00E-01	9.12E-01	3.34E+00	6.09E+00	0.00E-01
MN 54	3.94E+03	1.24E+04	0.00E-01	1.48E+04	4.15E+03	0.00E-01	0.00E-01	0.00E-01
MN 56	1.56E-01	1.00E+02	0.00E-01	6.91E-01	8.35E-01	0.00E-01	0.00E-01	0.00E-01
FE 55	6.54E+02	3.91E+02	3.98E+03	2.11E+03	0.00E-01	0.00E-01	1.19E+03	0.00E-01
FE 59	4.53E+03	9.48E+03	5.62E+03	9.10E+03	0.00E-01	0.00E-01	2.64E+03	0.00E-01
CO 58	9.45E+02	1.80E+03	0.00E-01	3.09E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	2.70E+03	5.07E+03	0.00E-01	9.15E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 63	6.34E+03	6.72E+02	1.86E+05	9.97E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	5.58E-02	1.17E+01	1.02E+00	9.57E-02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CU 64	6.85E+00	5.32E+02	0.00E-01	1.13E+01	2.74E+01	0.00E-01	0.00E-01	0.00E-01
ZN 65	1.57E+05	4.43E+04	9.46E+04	2.52E+05	1.59E+05	0.00E-01	0.00E-01	0.00E-01
ZN 69	9.09E-07	6.20E-04	6.81E-06	9.84E-06	5.97E-06	0.00E-01	0.00E-01	0.00E-01
BR 83	2.30E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 84	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 86	2.75E+05	2.88E+04	0.00E-01	4.47E+05	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	3.86E+03	5.23E+03	1.35E+05	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	5.35E+04	2.38E+04	2.66E+06	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 91	1.62E+01	9.46E+02	4.28E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 92	7.94E-02	3.75E+01	1.98E+00	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 90	7.34E-02	7.81E+03	2.74E+00	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91	1.38E+00	6.86E+03	5.15E+01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 92	8.09E-05	8.17E+01	2.83E-03	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 93	5.17E-03	2.81E+03	1.88E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	2.57E-01	3.01E+02	1.31E+00	2.88E-01	4.12E-01	0.00E-01	0.00E-01	0.00E-01
ZR 97	2.53E-03	6.50E+02	2.97E-02	4.29E-03	6.16E-03	0.00E-01	0.00E-01	0.00E-01
NB 95	6.37E+02	1.65E+06	2.29E+03	8.92E+02	8.38E+02	0.00E-01	0.00E-01	0.00E-01
MO 99	8.84E+01	2.96E+02	0.00E-01	3.57E+02	7.63E+02	0.00E-01	0.00E-01	0.00E-01
TC 99M	9.70E-02	3.33E+00	2.98E-03	5.85E-03	8.51E-02	0.00E-01	2.97E-03	0.00E-01
TC101	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = CHILD
PATHWAY = FRESH WATER FISH

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem gal)/(Ci min)				
RU103	9.56E+00	6.43E+02	2.49E+01	0.00E-01	6.26E+01	0.00E-01	0.00E-01	0.00E-01
RU105	1.88E-02	3.38E+01	5.17E-02	0.00E-01	4.55E-01	0.00E-01	0.00E-01	0.00E-01
RU106	5.05E+01	6.29E+03	4.04E+02	0.00E-01	5.46E+02	0.00E-01	0.00E-01	0.00E-01
AG110M	2.31E+00	3.44E+02	4.28E+00	2.89E+00	5.38E+00	0.00E-01	0.00E-01	0.00E-01
TE125M	2.08E+03	1.50E+04	1.56E+04	4.23E+03	0.00E-01	4.38E+03	0.00E-01	0.00E-01
TE127M	4.72E+03	3.22E+04	3.98E+04	1.07E+04	1.13E+05	9.51E+03	0.00E-01	0.00E-01
TE127	2.34E+01	4.27E+03	1.09E+02	2.95E+01	3.11E+02	7.56E+01	0.00E-01	0.00E-01
TE129M	1.03E+04	8.06E+04	6.61E+04	1.84E+04	1.94E+05	2.13E+04	0.00E-01	0.00E-01
TE129	2.45E-05	6.42E-03	1.03E-04	2.88E-05	3.02E-04	7.36E-05	0.00E-01	0.00E-01
TE131M	2.10E+03	8.01E+04	5.71E+03	1.98E+03	1.91E+04	4.06E+03	0.00E-01	0.00E-01
TE131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE132	6.04E+03	5.03E+04	1.13E+04	5.00E+03	4.64E+04	7.28E+03	0.00E-01	0.00E-01
I 130	4.08E+01	3.70E+01	3.92E+01	7.92E+01	1.18E+02	8.72E+03	0.00E-01	0.00E-01
I 131	4.68E+02	7.33E+01	8.19E+02	8.24E+02	1.35E+03	2.72E+05	0.00E-01	0.00E-01
I 132	2.46E-02	6.31E-02	2.92E-02	5.36E-02	8.20E-02	2.49E+00	0.00E-01	0.00E-01
I 133	6.45E+01	6.87E+01	1.38E+02	1.70E+02	2.84E+02	3.17E+04	0.00E-01	0.00E-01
I 134	9.55E-08	1.38E-07	1.12E-07	2.07E-07	3.17E-07	4.77E-06	0.00E-01	0.00E-01
I 135	6.14E+00	9.90E+00	7.22E+00	1.30E+01	1.99E+01	1.15E+03	0.00E-01	0.00E-01
CS134	5.60E+05	1.43E+04	1.62E+06	2.66E+06	8.23E+05	0.00E-01	2.95E+05	0.00E-01
CS136	2.74E+05	1.49E+04	1.54E+05	4.24E+05	2.26E+05	0.00E-01	3.37E+04	0.00E-01
CS137	3.20E+05	1.36E+04	2.26E+06	2.17E+06	7.06E+05	0.00E-01	2.54E+05	0.00E-01
CS138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA139	8.82E-07	1.76E-03	3.04E-05	1.62E-08	1.42E-08	0.00E-01	9.56E-09	0.00E-01
BA140	6.36E+01	5.52E+02	1.09E+03	9.55E-01	3.11E-01	0.00E-01	5.69E-01	0.00E-01
BA141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA142	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA140	6.80E-02	5.63E+03	5.77E-01	2.02E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA142	8.83E-08	5.59E-02	8.85E-07	2.82E-07	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	9.96E-03	8.37E+01	1.34E-01	6.71E-02	2.94E-02	0.00E-01	0.00E-01	0.00E-01
CE143	1.15E-03	1.16E+02	1.46E-02	7.93E+00	3.32E-03	0.00E-01	0.00E-01	0.00E-01
CE144	3.83E-01	5.87E+02	7.18E+00	2.25E+00	1.25E+00	0.00E-01	0.00E-01	0.00E-01
PR143	1.60E-01	3.49E+03	3.23E+00	9.70E-01	5.25E-01	0.00E-01	0.00E-01	0.00E-01
PR144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ND147	1.42E-01	2.91E+03	2.27E+00	1.84E+00	1.01E+00	0.00E-01	0.00E-01	0.00E-01
W 187	2.36E+02	7.38E+04	8.86E+02	5.25E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NP239	6.83E-03	7.19E+02	1.35E-01	9.72E-03	2.81E-02	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = CHILD
PATHWAY = ANIMAL DRINKING WATER—MEAT

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	1.19E+00	1.19E+00	0.00E-01	1.19E+00	1.19E+00	1.19E+00	1.19E+00	0.00E-01
C 14	7.72E+01	7.72E+01	3.86E+02	7.72E+01	7.72E+01	7.72E+01	7.72E+01	0.00E-01
NA 24	4.24E-08	4.24E-08	4.24E-08	4.24E-08	4.24E-08	4.24E-08	4.24E-08	0.00E-01
P 32	5.70E+02	4.08E+02	1.48E+04	6.91E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	1.33E-02	7.05E-01	0.00E-01	0.00E-01	2.02E-03	7.38E-03	1.35E-02	0.00E-01
MN 54	2.24E+00	7.07E+00	0.00E-01	8.42E+00	2.36E+00	0.00E-01	0.00E-01	0.00E-01
MN 56	4.15E-28	2.66E-25	0.00E-01	1.84E-27	2.22E-27	0.00E-01	0.00E-01	0.00E-01
FE 55	7.67E+01	4.58E+01	4.67E+02	2.47E+02	0.00E-01	0.00E-01	1.40E+02	0.00E-01
FE 59	4.01E+02	8.38E+02	4.97E+02	8.04E+02	0.00E-01	0.00E-01	2.33E+02	0.00E-01
CO 58	6.06E+01	1.15E+02	0.00E-01	1.98E+01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	2.07E+02	3.89E+02	0.00E-01	7.02E+01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 63	9.97E+02	1.06E+02	2.93E+04	1.57E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	4.42E-26	9.28E-24	8.05E-25	7.58E-26	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CU 64	4.52E-12	3.51E-10	0.00E-01	7.48E-12	1.81E-11	0.00E-01	0.00E-01	0.00E-01
ZN 65	6.62E+02	1.87E+02	3.99E+02	1.06E+03	6.70E+02	0.00E-01	0.00E-01	0.00E-01
ZN 69	7.61E-28	5.19E-25	5.70E-27	8.24E-27	5.00E-27	0.00E-01	0.00E-01	0.00E-01
BR 83	3.00E-26	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 84	1.25E-26	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 85	1.07E-33	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 86	6.25E+02	6.54E+01	0.00E-01	1.02E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 88	3.59E-27	2.53E-28	0.00E-01	5.17E-27	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 89	1.90E-27	1.87E-29	0.00E-01	2.14E-27	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	1.77E+01	2.40E+01	6.19E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	3.17E+02	1.41E+02	1.58E+04	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 91	3.03E-16	1.77E-14	8.03E-15	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 92	1.51E-27	7.16E-25	3.78E-26	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 90	2.89E-05	3.07E+00	1.08E-03	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91M	2.49E-31	1.34E-26	6.84E-30	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91	6.01E-02	2.99E+02	2.25E+00	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 92	3.51E-30	3.55E-24	1.23E-28	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 93	6.41E-18	3.48E-12	2.34E-16	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	6.39E-01	7.49E+02	3.27E+00	7.18E-01	1.03E+00	0.00E-01	0.00E-01	0.00E-01
ZR 97	5.57E-11	1.43E-05	6.53E-10	9.44E-11	1.36E-10	0.00E-01	0.00E-01	0.00E-01
NB 95	1.22E+00	3.14E+03	4.37E+00	1.70E+00	1.60E+00	0.00E-01	0.00E-01	0.00E-01
MO 99	1.72E-01	5.77E-01	0.00E-01	6.97E-01	1.49E+00	0.00E-01	0.00E-01	0.00E-01
TC 99M	1.06E-23	3.65E-22	3.27E-25	6.42E-25	9.32E-24	0.00E-01	3.26E-25	0.00E-01
TC101	2.73E-27	6.85E-28	2.06E-28	2.15E-28	3.67E-27	0.00E-01	1.14E-28	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = CHILD
PATHWAY = ANIMAL DRINKING WATER—MEAT

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem gal)/(Ci min)				
RU103	8.12E+01	5.46E+03	2.11E+02	0.00E-01	5.32E+02	0.00E-01	0.00E-01	0.00E-01
RU105	7.21E-26	1.30E-22	1.99E-25	0.00E-01	1.75E-24	0.00E-01	0.00E-01	0.00E-01
RU106	5.78E+02	7.21E+04	4.64E+03	0.00E-01	6.26E+03	0.00E-01	0.00E-01	0.00E-01
AG110M	4.82E+00	7.16E+02	8.92E+00	6.02E+00	1.12E+01	0.00E-01	0.00E-01	0.00E-01
TE125M	9.48E+01	6.86E+02	7.11E+02	1.93E+02	0.00E-01	2.00E+02	0.00E-01	0.00E-01
TE127M	2.39E+02	1.63E+03	2.01E+03	5.42E+02	5.75E+03	4.82E+02	0.00E-01	0.00E-01
TE127	2.58E-15	4.70E-13	1.20E-14	3.24E-15	3.42E-14	8.32E-15	0.00E-01	0.00E-01
TE129M	3.96E+02	3.11E+03	2.55E+03	7.12E+02	7.49E+03	8.22E+02	0.00E-01	0.00E-01
TE129	1.21E-26	3.18E-24	5.11E-26	1.43E-26	1.50E-25	3.65E-26	0.00E-01	0.00E-01
TE131M	3.12E-03	1.19E-01	8.47E-03	2.93E-03	2.84E-02	6.03E-03	0.00E-01	0.00E-01
TE131	3.26E-27	5.75E-26	1.10E-26	3.34E-27	3.31E-26	8.38E-27	0.00E-01	0.00E-01
TE132	6.04E+00	5.03E+01	1.13E+01	5.00E+00	4.64E+01	7.28E+00	0.00E-01	0.00E-01
I 130	1.68E-11	1.53E-11	1.62E-11	3.27E-11	4.88E-11	3.60E-09	0.00E-01	0.00E-01
I 131	5.21E+00	8.16E-01	9.11E+00	9.17E+00	1.51E+01	3.03E+03	0.00E-01	0.00E-01
I 132	1.31E-26	3.34E-26	1.55E-26	2.84E-26	4.35E-26	1.32E-24	0.00E-01	0.00E-01
I 133	9.14E-07	9.73E-07	1.95E-06	2.41E-06	4.03E-06	4.49E-04	0.00E-01	0.00E-01
I 134	4.24E-27	6.10E-27	4.96E-27	9.20E-27	1.41E-26	2.12E-25	0.00E-01	0.00E-01
I 135	4.88E-22	7.87E-22	5.74E-22	1.03E-21	1.58E-21	9.14E-20	0.00E-01	0.00E-01
CS134	3.27E+02	8.36E+00	9.45E+02	1.55E+03	4.81E+02	0.00E-01	1.72E+02	0.00E-01
CS136	5.96E+01	3.24E+00	3.35E+01	9.21E+01	4.90E+01	0.00E-01	7.32E+00	0.00E-01
CS137	1.90E+02	8.06E+00	1.34E+03	1.29E+03	4.19E+02	0.00E-01	1.51E+02	0.00E-01
CS138	2.00E-27	1.45E-27	2.26E-27	3.15E-27	2.21E-27	0.00E-01	2.38E-28	0.00E-01
BA139	2.09E-28	4.16E-25	7.21E-27	3.85E-30	3.36E-30	0.00E-01	2.26E-30	0.00E-01
BA140	5.39E+00	4.68E+01	9.24E+01	8.09E-02	2.63E-02	0.00E-01	4.82E-02	0.00E-01
BA141	1.93E-29	3.38E-28	5.93E-28	3.32E-31	2.87E-31	0.00E-01	1.95E-30	0.00E-01
BA142	2.88E-30	6.73E-31	5.16E-29	3.71E-32	3.01E-32	0.00E-01	2.18E-32	0.00E-01
LA140	6.23E-08	5.15E-03	5.28E-07	1.85E-07	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA142	6.01E-32	3.80E-26	6.02E-31	1.92E-31	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	2.37E-03	1.99E+01	3.20E-02	1.59E-02	6.99E-03	0.00E-01	0.00E-01	0.00E-01
CE143	2.90E-08	2.93E-03	3.69E-07	2.00E-04	8.39E-08	0.00E-01	0.00E-01	0.00E-01
CE144	1.31E-01	2.00E+02	2.44E+00	7.66E-01	4.24E-01	0.00E-01	0.00E-01	0.00E-01
PR143	3.39E-03	7.38E+01	6.84E-02	2.05E-02	1.11E-02	0.00E-01	0.00E-01	0.00E-01
PR144	2.47E-32	3.27E-28	4.91E-31	1.52E-31	8.04E-32	0.00E-01	0.00E-01	0.00E-01
ND147	1.70E-03	3.47E+01	2.71E-02	2.19E-02	1.20E-02	0.00E-01	0.00E-01	0.00E-01
W 187	1.35E-07	4.22E-05	5.07E-07	3.00E-07	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NP239	1.51E-07	1.59E-02	3.00E-06	2.15E-07	6.22E-07	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = CHILD
PATHWAY = RIVER SHORELINE DEPOSITS

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem gal)/(Ci min)				
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NA 24	2.19E+00	2.19E+00	2.19E+00	2.19E+00	2.19E+00	2.19E+00	2.19E+00	2.54E+00
P 32	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	8.55E-01	8.55E-01	8.55E-01	8.55E-01	8.55E-01	8.55E-01	8.55E-01	1.01E+00
MN 54	2.54E+02	2.54E+02	2.54E+02	2.54E+02	2.54E+02	2.54E+02	2.54E+02	2.98E+02
MN 56	1.61E-01	1.61E-01	1.61E-01	1.61E-01	1.61E-01	1.61E-01	1.61E-01	1.90E-01
FE 55	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 59	5.01E+01	5.01E+01	5.01E+01	5.01E+01	5.01E+01	5.01E+01	5.01E+01	5.88E+01
CO 58	6.98E+01	6.98E+01	6.98E+01	6.98E+01	6.98E+01	6.98E+01	6.98E+01	8.18E+01
CO 60	3.95E+03	3.95E+03	3.95E+03	3.95E+03	3.95E+03	3.95E+03	3.95E+03	4.65E+03
NI 63	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	5.31E-02	5.31E-02	5.31E-02	5.31E-02	5.31E-02	5.31E-02	5.31E-02	6.17E-02
CU 64	1.11E-01	1.11E-01	1.11E-01	1.11E-01	1.11E-01	1.11E-01	1.11E-01	1.25E-01
ZN 65	1.37E+02	1.37E+02	1.37E+02	1.37E+02	1.37E+02	1.37E+02	1.37E+02	1.58E+02
ZN 69	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 83	8.70E-04	8.70E-04	8.70E-04	8.70E-04	8.70E-04	8.70E-04	8.70E-04	1.26E-03
BR 84	3.27E-02	3.27E-02	3.27E-02	3.27E-02	3.27E-02	3.27E-02	3.27E-02	3.81E-02
BR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 86	1.65E+00	1.65E+00	1.65E+00	1.65E+00	1.65E+00	1.65E+00	1.65E+00	1.89E+00
RB 88	4.83E-03	4.83E-03	4.83E-03	4.83E-03	4.83E-03	4.83E-03	4.83E-03	5.52E-03
RB 89	1.70E-02	1.70E-02	1.70E-02	1.70E-02	1.70E-02	1.70E-02	1.70E-02	2.04E-02
SR 89	3.97E-03	3.97E-03	3.97E-03	3.97E-03	3.97E-03	3.97E-03	3.97E-03	4.61E-03
SR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 91	3.91E-01	3.91E-01	3.91E-01	3.91E-01	3.91E-01	3.91E-01	3.91E-01	4.57E-01
SR 92	1.39E-01	1.39E-01	1.39E-01	1.39E-01	1.39E-01	1.39E-01	1.39E-01	1.55E-01
Y 90	8.25E-04	8.25E-04	8.25E-04	8.25E-04	8.25E-04	8.25E-04	8.25E-04	9.76E-04
Y 91M	1.69E-02	1.69E-02	1.69E-02	1.69E-02	1.69E-02	1.69E-02	1.69E-02	1.96E-02
Y 91	1.97E-01	1.97E-01	1.97E-01	1.97E-01	1.97E-01	1.97E-01	1.97E-01	2.22E-01
Y 92	3.26E-02	3.26E-02	3.26E-02	3.26E-02	3.26E-02	3.26E-02	3.26E-02	3.87E-02
Y 93	3.34E-02	3.34E-02	3.34E-02	3.34E-02	3.34E-02	3.34E-02	3.34E-02	4.57E-02
ZR 95	4.51E+01	4.51E+01	4.51E+01	4.51E+01	4.51E+01	4.51E+01	4.51E+01	5.23E+01
ZR 97	5.41E-01	5.41E-01	5.41E-01	5.41E-01	5.41E-01	5.41E-01	5.41E-01	6.30E-01
NB 95	2.52E+01	2.52E+01	2.52E+01	2.52E+01	2.52E+01	2.52E+01	2.52E+01	2.96E+01
MO 99	7.33E-01	7.33E-01	7.33E-01	7.33E-01	7.33E-01	7.33E-01	7.33E-01	8.48E-01
TC 99M	3.34E-02	3.34E-02	3.34E-02	3.34E-02	3.34E-02	3.34E-02	3.34E-02	3.83E-02
TC101	2.79E-03	2.79E-03	2.79E-03	2.79E-03	2.79E-03	2.79E-03	2.79E-03	3.10E-03

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = CHILD
PATHWAY = RIVER SHORELINE DEPOSITS

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
RU103	1.99E+01	1.99E+01	1.99E+01	1.99E+01	1.99E+01	1.99E+01	1.99E+01	2.32E+01
RU105	1.15E-01	1.15E-01	1.15E-01	1.15E-01	1.15E-01	1.15E-01	1.15E-01	1.30E-01
RU106	7.75E+01	7.75E+01	7.75E+01	7.75E+01	7.75E+01	7.75E+01	7.75E+01	9.31E+01
AG110M	6.34E+02	6.34E+02	6.34E+02	6.34E+02	6.34E+02	6.34E+02	6.34E+02	7.39E+02
TE125M	2.86E-01	2.86E-01	2.86E-01	2.86E-01	2.86E-01	2.86E-01	2.86E-01	3.92E-01
TE127M	1.68E-02	1.68E-02	1.68E-02	1.68E-02	1.68E-02	1.68E-02	1.68E-02	1.99E-02
TE127	5.43E-04	5.43E-04	5.43E-04	5.43E-04	5.43E-04	5.43E-04	5.43E-04	5.97E-04
TE129M	3.63E+00	3.63E+00	3.63E+00	3.63E+00	3.63E+00	3.63E+00	3.63E+00	4.24E+00
TE129	4.54E-03	4.54E-03	4.54E-03	4.54E-03	4.54E-03	4.54E-03	4.54E-03	5.37E-03
TE131M	1.47E+00	1.47E+00	1.47E+00	1.47E+00	1.47E+00	1.47E+00	1.47E+00	1.73E+00
TE131	4.56E-03	4.56E-03	4.56E-03	4.56E-03	4.56E-03	4.56E-03	4.56E-03	5.38E+00
TE132	7.78E-01	7.78E-01	7.78E-01	7.78E-01	7.78E-01	7.78E-01	7.78E-01	9.15E-01
I 130	1.01E+00	1.01E+00	1.01E+00	1.01E+00	1.01E+00	1.01E+00	1.01E+00	1.22E+00
I 131	3.16E+00	3.16E+00	3.16E+00	3.16E+00	3.16E+00	3.16E+00	3.16E+00	3.84E+00
I 132	2.22E-01	2.22E-01	2.22E-01	2.22E-01	2.22E-01	2.22E-01	2.22E-01	2.61E-01
I 133	4.49E-01	4.49E-01	4.49E-01	4.49E-01	4.49E-01	4.49E-01	4.49E-01	5.46E-01
I 134	7.57E-02	7.57E-02	7.57E-02	7.57E-02	7.57E-02	7.57E-02	7.57E-02	8.99E-02
I 135	4.58E-01	4.58E-01	4.58E-01	4.58E-01	4.58E-01	4.58E-01	4.58E-01	5.35E-01
CS134	1.26E+03	1.26E+03	1.26E+03	1.26E+03	1.26E+03	1.26E+03	1.26E+03	1.46E+03
CS136	2.76E+01	2.76E+01	2.76E+01	2.76E+01	2.76E+01	2.76E+01	2.76E+01	3.13E+01
CS137	1.89E+03	1.89E+03	1.89E+03	1.89E+03	1.89E+03	1.89E+03	1.89E+03	2.21E+03
CS138	5.81E-02	5.81E-02	5.81E-02	5.81E-02	5.81E-02	5.81E-02	5.81E-02	6.64E-02
BA139	1.84E-02	1.84E-02	1.84E-02	1.84E-02	1.84E-02	1.84E-02	1.84E-02	2.07E-02
BA140	3.77E+00	3.77E+00	3.77E+00	3.77E+00	3.77E+00	3.77E+00	3.77E+00	4.31E+00
BA141	6.11E-03	6.11E-03	6.11E-03	6.11E-03	6.11E-03	6.11E-03	6.11E-03	6.96E-03
BA142	5.59E-03	5.59E-03	5.59E-03	5.59E-03	5.59E-03	5.59E-03	5.59E-03	6.37E-03
LA140	3.53E+00	3.53E+00	3.53E+00	3.53E+00	3.53E+00	3.53E+00	3.53E+00	4.00E+00
LA142	1.29E-01	1.29E-01	1.29E-01	1.29E-01	1.29E-01	1.29E-01	1.29E-01	1.55E-01
CE141	2.51E+00	2.51E+00	2.51E+00	2.51E+00	2.51E+00	2.51E+00	2.51E+00	2.83E+00
CE143	4.26E-01	4.26E-01	4.26E-01	4.26E-01	4.26E-01	4.26E-01	4.26E-01	4.84E-01
CE144	1.28E+01	1.28E+01	1.28E+01	1.28E+01	1.28E+01	1.28E+01	1.28E+01	1.48E+01
PR143	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
PR144	2.65E-04	2.65E-04	2.65E-04	2.65E-04	2.65E-04	2.65E-04	2.65E-04	3.04E-04
ND147	1.56E+00	1.56E+00	1.56E+00	1.56E+00	1.56E+00	1.56E+00	1.56E+00	1.87E+00
W 187	4.33E-01	4.33E-01	4.33E-01	4.33E-01	4.33E-01	4.33E-01	4.33E-01	5.02E-01
NP239	3.15E-01	3.15E-01	3.15E-01	3.15E-01	3.15E-01	3.15E-01	3.15E-01	3.64E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = CHILD
PATHWAY = SWIMMING

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NA 24	1.09E+02	1.09E+02	1.09E+02	1.09E+02	1.09E+02	1.09E+02	1.09E+02	0.00E-01
P 32	8.99E-02	8.99E-02	8.99E-02	8.99E-02	8.99E-02	8.99E-02	8.99E-02	0.00E-01
CR 51	7.30E-01	7.30E-01	7.30E-01	7.30E-01	7.30E-01	7.30E-01	7.30E-01	0.00E-01
MN 54	2.11E+01	2.11E+01	2.11E+01	2.11E+01	2.11E+01	2.11E+01	2.11E+01	0.00E-01
MN 56	4.38E+01	4.38E+01	4.38E+01	4.38E+01	4.38E+01	4.38E+01	4.38E+01	0.00E-01
FE 55	8.99E-04	8.99E-04	8.99E-04	8.99E-04	8.99E-04	8.99E-04	8.99E-04	0.00E-01
FE 59	3.09E+01	3.09E+01	3.09E+01	3.09E+01	3.09E+01	3.09E+01	3.09E+01	0.00E-01
CO 58	2.53E+01	2.53E+01	2.53E+01	2.53E+01	2.53E+01	2.53E+01	2.53E+01	0.00E-01
CO 60	6.46E+01	6.46E+01	6.46E+01	6.46E+01	6.46E+01	6.46E+01	6.46E+01	0.00E-01
NI 63	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	1.37E+01	1.37E+01	1.37E+01	1.37E+01	1.37E+01	1.37E+01	1.37E+01	0.00E-01
CU 64	5.17E+00	5.17E+00	5.17E+00	5.17E+00	5.17E+00	5.17E+00	5.17E+00	0.00E-01
ZN 65	1.55E+01	1.55E+01	1.55E+01	1.55E+01	1.55E+01	1.55E+01	1.55E+01	0.00E-01
ZN 69	2.09E-02	2.09E-02	2.09E-02	2.09E-02	2.09E-02	2.09E-02	2.09E-02	0.00E-01
BR 83	2.32E-01	2.32E-01	2.32E-01	2.32E-01	2.32E-01	2.32E-01	2.32E-01	0.00E-01
BR 84	4.32E+01	4.32E+01	4.32E+01	4.32E+01	4.32E+01	4.32E+01	4.32E+01	0.00E-01
BR 85	4.61E-02	4.61E-02	4.61E-02	4.61E-02	4.61E-02	4.61E-02	4.61E-02	0.00E-01
RB 86	2.39E+00	2.39E+00	2.39E+00	2.39E+00	2.39E+00	2.39E+00	2.39E+00	0.00E-01
RB 88	1.34E+01	1.34E+01	1.34E+01	1.34E+01	1.34E+01	1.34E+01	1.34E+01	0.00E-01
RB 89	4.81E+01	4.81E+01	4.81E+01	4.81E+01	4.81E+01	4.81E+01	4.81E+01	0.00E-01
SR 89	6.46E-02	6.46E-02	6.46E-02	6.46E-02	6.46E-02	6.46E-02	6.46E-02	0.00E-01
SR 90	7.59E-03	7.59E-03	7.59E-03	7.59E-03	7.59E-03	7.59E-03	7.59E-03	0.00E-01
SR 91	2.65E+01	2.65E+01	2.65E+01	2.65E+01	2.65E+01	2.65E+01	2.65E+01	0.00E-01
SR 92	3.56E+01	3.56E+01	3.56E+01	3.56E+01	3.56E+01	3.56E+01	3.56E+01	0.00E-01
Y 90	1.83E-01	1.83E-01	1.83E-01	1.83E-01	1.83E-01	1.83E-01	1.83E-01	0.00E-01
Y 91M	1.29E+01	1.29E+01	1.29E+01	1.29E+01	1.29E+01	1.29E+01	1.29E+01	0.00E-01
Y 91	9.41E-02	9.41E-02	9.41E-02	9.41E-02	9.41E-02	9.41E-02	9.41E-02	0.00E-01
Y 92	6.34E+00	6.34E+00	6.34E+00	6.34E+00	6.34E+00	6.34E+00	6.34E+00	0.00E-01
Y 93	2.65E+00	2.65E+00	2.65E+00	2.65E+00	2.65E+00	2.65E+00	2.65E+00	0.00E-01
ZR 95	2.11E+01	2.11E+01	2.11E+01	2.11E+01	2.11E+01	2.11E+01	2.11E+01	0.00E-01
ZR 97	2.10E+01	2.10E+01	2.10E+01	2.10E+01	2.10E+01	2.10E+01	2.10E+01	0.00E-01
NB 95	1.97E+01	1.97E+01	1.97E+01	1.97E+01	1.97E+01	1.97E+01	1.97E+01	0.00E-01
MO 99	6.60E+00	6.60E+00	6.60E+00	6.60E+00	6.60E+00	6.60E+00	6.60E+00	0.00E-01
TC 99M	3.33E+00	3.33E+00	3.33E+00	3.33E+00	3.33E+00	3.33E+00	3.33E+00	0.00E-01
TC101	7.13E+00	7.13E+00	7.13E+00	7.13E+00	7.13E+00	7.13E+00	7.13E+00	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = CHILD
PATHWAY = SWIMMING

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem gal)/(Ci min)				
RU103	1.25E+01	1.25E+01	1.25E+01	1.25E+01	1.25E+01	1.25E+01	1.25E+01	0.00E-01
RU105	1.66E+01	1.66E+01	1.66E+01	1.66E+01	1.66E+01	1.66E+01	1.66E+01	0.00E-01
RU106	5.34E+00	5.34E+00	5.34E+00	5.34E+00	5.34E+00	5.34E+00	5.34E+00	0.00E-01
AG110M	6.88E+01	6.88E+01	6.88E+01	6.88E+01	6.88E+01	6.88E+01	6.88E+01	0.00E-01
TE125M	5.18E-02	5.18E-02	5.18E-02	5.18E-02	5.18E-02	5.18E-02	5.18E-02	0.00E-01
TE127M	3.65E-03	3.65E-03	3.65E-03	3.65E-03	3.65E-03	3.65E-03	3.65E-03	0.00E-01
TE127	3.91E-02	3.91E-02	3.91E-02	3.91E-02	3.91E-02	3.91E-02	3.91E-02	0.00E-01
TE129M	2.95E+00	2.95E+00	2.95E+00	2.95E+00	2.95E+00	2.95E+00	2.95E+00	0.00E-01
TE129	2.52E+00	2.52E+00	2.52E+00	2.52E+00	2.52E+00	2.52E+00	2.52E+00	0.00E-01
TE131M	3.08E+01	3.08E+01	3.08E+01	3.08E+01	3.08E+01	3.08E+01	3.08E+01	0.00E-01
TE131	8.81E+00	8.81E+00	8.81E+00	8.81E+00	8.81E+00	8.81E+00	8.81E+00	0.00E-01
TE132	5.61E+00	5.61E+00	5.61E+00	5.61E+00	5.61E+00	5.61E+00	5.61E+00	0.00E-01
I 130	5.45E+01	5.45E+01	5.45E+01	5.45E+01	5.45E+01	5.45E+01	5.45E+01	0.00E-01
I 131	1.10E+01	1.10E+01	1.10E+01	1.10E+01	1.10E+01	1.10E+01	1.10E+01	0.00E-01
I 132	6.00E+01	6.00E+01	6.00E+01	6.00E+01	6.00E+01	6.00E+01	6.00E+01	0.00E-01
I 133	1.34E+01	1.34E+01	1.34E+01	1.34E+01	1.34E+01	1.34E+01	1.34E+01	0.00E-01
I 134	5.45E+01	5.45E+01	5.45E+01	5.45E+01	5.45E+01	5.45E+01	5.45E+01	0.00E-01
I 135	4.59E+01	4.59E+01	4.59E+01	4.59E+01	4.59E+01	4.59E+01	4.59E+01	0.00E-01
CS134	4.07E+01	4.07E+01	4.07E+01	4.07E+01	4.07E+01	4.07E+01	4.07E+01	0.00E-01
CS136	5.76E+01	5.76E+01	5.76E+01	5.76E+01	5.76E+01	5.76E+01	5.76E+01	0.00E-01
CS137	1.41E+01	1.41E+01	1.41E+01	1.41E+01	1.41E+01	1.41E+01	1.41E+01	0.00E-01
CS138	4.94E+01	4.94E+01	4.94E+01	4.94E+01	4.94E+01	4.94E+01	4.94E+01	0.00E-01
BA139	1.03E+00	1.03E+00	1.03E+00	1.03E+00	1.03E+00	1.03E+00	1.03E+00	0.00E-01
BA140	6.88E+00	6.88E+00	6.88E+00	6.88E+00	6.88E+00	6.88E+00	6.88E+00	0.00E-01
BA141	1.23E+01	1.23E+01	1.23E+01	1.23E+01	1.23E+01	1.23E+01	1.23E+01	0.00E-01
BA142	2.10E+01	2.10E+01	2.10E+01	2.10E+01	2.10E+01	2.10E+01	2.10E+01	0.00E-01
LA140	5.75E+01	5.75E+01	5.75E+01	5.75E+01	5.75E+01	5.75E+01	5.75E+01	0.00E-01
LA142	6.04E+01	6.04E+01	6.04E+01	6.04E+01	6.04E+01	6.04E+01	6.04E+01	0.00E-01
CE141	1.83E+00	1.83E+00	1.83E+00	1.83E+00	1.83E+00	1.83E+00	1.83E+00	0.00E-01
CE143	7.99E+00	7.99E+00	7.99E+00	7.99E+00	7.99E+00	7.99E+00	7.99E+00	0.00E-01
CE144	1.21E+00	1.21E+00	1.21E+00	1.21E+00	1.21E+00	1.21E+00	1.21E+00	0.00E-01
PR143	2.25E-02	2.25E-02	2.25E-02	2.25E-02	2.25E-02	2.25E-02	2.25E-02	0.00E-01
PR144	6.18E-01	6.18E-01	6.18E-01	6.18E-01	6.18E-01	6.18E-01	6.18E-01	0.00E-01
ND147	3.93E+00	3.93E+00	3.93E+00	3.93E+00	3.93E+00	3.93E+00	3.93E+00	0.00E-01
W 187	1.16E+01	1.16E+01	1.16E+01	1.16E+01	1.16E+01	1.16E+01	1.16E+01	0.00E-01
NP239	3.37E+00	3.37E+00	3.37E+00	3.37E+00	3.37E+00	3.37E+00	3.37E+00	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = CHILD
PATHWAY = BOATING

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NA 24	1.01E+02	1.01E+02	1.01E+02	1.01E+02	1.01E+02	1.01E+02	1.01E+02	0.00E-01
P 32	8.35E-02	8.35E-02	8.35E-02	8.35E-02	8.35E-02	8.35E-02	8.35E-02	0.00E-01
CR 51	6.78E-01	6.78E-01	6.78E-01	6.78E-01	6.78E-01	6.78E-01	6.78E-01	0.00E-01
MN 54	1.96E+01	1.96E+01	1.96E+01	1.96E+01	1.96E+01	1.96E+01	1.96E+01	0.00E-01
MN 56	4.06E+01	4.06E+01	4.06E+01	4.06E+01	4.06E+01	4.06E+01	4.06E+01	0.00E-01
FE 55	8.35E-04	8.35E-04	8.35E-04	8.35E-04	8.35E-04	8.35E-04	8.35E-04	0.00E-01
FE 59	2.87E+01	2.87E+01	2.87E+01	2.87E+01	2.87E+01	2.87E+01	2.87E+01	0.00E-01
CO 58	2.35E+01	2.35E+01	2.35E+01	2.35E+01	2.35E+01	2.35E+01	2.35E+01	0.00E-01
CO 60	6.00E+01	6.00E+01	6.00E+01	6.00E+01	6.00E+01	6.00E+01	6.00E+01	0.00E-01
NI 63	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	1.27E+01	1.27E+01	1.27E+01	1.27E+01	1.27E+01	1.27E+01	1.27E+01	0.00E-01
CU 64	4.80E+00	4.80E+00	4.80E+00	4.80E+00	4.80E+00	4.80E+00	4.80E+00	0.00E-01
ZN 65	1.44E+01	1.44E+01	1.44E+01	1.44E+01	1.44E+01	1.44E+01	1.44E+01	0.00E-01
ZN 69	1.94E-02	1.94E-02	1.94E-02	1.94E-02	1.94E-02	1.94E-02	1.94E-02	0.00E-01
BR 83	2.15E-01	2.15E-01	2.15E-01	2.15E-01	2.15E-01	2.15E-01	2.15E-01	0.00E-01
BR 84	4.01E+01	4.01E+01	4.01E+01	4.01E+01	4.01E+01	4.01E+01	4.01E+01	0.00E-01
BR 85	4.28E-02	4.28E-02	4.28E-02	4.28E-02	4.28E-02	4.28E-02	4.28E-02	0.00E-01
RB 86	2.22E+00	2.22E+00	2.22E+00	2.22E+00	2.22E+00	2.22E+00	2.22E+00	0.00E-01
RB 88	1.24E+01	1.24E+01	1.24E+01	1.24E+01	1.24E+01	1.24E+01	1.24E+01	0.00E-01
RB 89	4.47E+01	4.47E+01	4.47E+01	4.47E+01	4.47E+01	4.47E+01	4.47E+01	0.00E-01
SR 89	6.00E-02	6.00E-02	6.00E-02	6.00E-02	6.00E-02	6.00E-02	6.00E-02	0.00E-01
SR 90	7.04E-03	7.04E-03	7.04E-03	7.04E-03	7.04E-03	7.04E-03	7.04E-03	0.00E-01
SR 91	2.46E+01	2.46E+01	2.46E+01	2.46E+01	2.46E+01	2.46E+01	2.46E+01	0.00E-01
SR 92	3.31E+01	3.31E+01	3.31E+01	3.31E+01	3.31E+01	3.31E+01	3.31E+01	0.00E-01
Y 90	1.69E-01	1.69E-01	1.69E-01	1.69E-01	1.69E-01	1.69E-01	1.69E-01	0.00E-01
Y 91M	1.20E+01	1.20E+01	1.20E+01	1.20E+01	1.20E+01	1.20E+01	1.20E+01	0.00E-01
Y 91	8.74E-02	8.74E-02	8.74E-02	8.74E-02	8.74E-02	8.74E-02	8.74E-02	0.00E-01
Y 92	5.89E+00	5.89E+00	5.89E+00	5.89E+00	5.89E+00	5.89E+00	5.89E+00	0.00E-01
Y 93	2.46E+00	2.46E+00	2.46E+00	2.46E+00	2.46E+00	2.46E+00	2.46E+00	0.00E-01
ZR 95	1.96E+01	1.96E+01	1.96E+01	1.96E+01	1.96E+01	1.96E+01	1.96E+01	0.00E-01
ZR 97	1.95E+01	1.95E+01	1.95E+01	1.95E+01	1.95E+01	1.95E+01	1.95E+01	0.00E-01
NB 95	1.83E+01	1.83E+01	1.83E+01	1.83E+01	1.83E+01	1.83E+01	1.83E+01	0.00E-01
MO 99	6.12E+00	6.12E+00	6.12E+00	6.12E+00	6.12E+00	6.12E+00	6.12E+00	0.00E-01
TC 99M	3.10E+00	3.10E+00	3.10E+00	3.10E+00	3.10E+00	3.10E+00	3.10E+00	0.00E-01
TC101	6.62E+00	6.62E+00	6.62E+00	6.62E+00	6.62E+00	6.62E+00	6.62E+00	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = CHILD
PATHWAY = BOATING

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
RU103	1.16E+01	1.16E+01	1.16E+01	1.16E+01	1.16E+01	1.16E+01	1.16E+01	0.00E-01
RU105	1.54E+01	1.54E+01	1.54E+01	1.54E+01	1.54E+01	1.54E+01	1.54E+01	0.00E-01
RU106	4.96E+00	4.96E+00	4.96E+00	4.96E+00	4.96E+00	4.96E+00	4.96E+00	0.00E-01
AG110M	6.39E+01	6.39E+01	6.39E+01	6.39E+01	6.39E+01	6.39E+01	6.39E+01	0.00E-01
TE125M	4.81E-02	4.81E-02	4.81E-02	4.81E-02	4.81E-02	4.81E-02	4.81E-02	0.00E-01
TE127M	3.39E-03	3.39E-03	3.39E-03	3.39E-03	3.39E-03	3.39E-03	3.39E-03	0.00E-01
TE127	3.63E-02	3.63E-02	3.63E-02	3.63E-02	3.63E-02	3.63E-02	3.63E-02	0.00E-01
TE129M	2.74E+00	2.74E+00	2.74E+00	2.74E+00	2.74E+00	2.74E+00	2.74E+00	0.00E-01
TE129	2.34E+00	2.34E+00	2.34E+00	2.34E+00	2.34E+00	2.34E+00	2.34E+00	0.00E-01
TE131M	2.86E+01	2.86E+01	2.86E+01	2.86E+01	2.86E+01	2.86E+01	2.86E+01	0.00E-01
TE131	8.18E+00	8.18E+00	8.18E+00	8.18E+00	8.18E+00	8.18E+00	8.18E+00	0.00E-01
TE132	5.21E+00	5.21E+00	5.21E+00	5.21E+00	5.21E+00	5.21E+00	5.21E+00	0.00E-01
I 130	5.06E+01	5.06E+01	5.06E+01	5.06E+01	5.06E+01	5.06E+01	5.06E+01	0.00E-01
I 131	1.02E+01	1.02E+01	1.02E+01	1.02E+01	1.02E+01	1.02E+01	1.02E+01	0.00E-01
I 132	5.57E+01	5.57E+01	5.57E+01	5.57E+01	5.57E+01	5.57E+01	5.57E+01	0.00E-01
I 133	1.25E+01	1.25E+01	1.25E+01	1.25E+01	1.25E+01	1.25E+01	1.25E+01	0.00E-01
I 134	5.06E+01	5.06E+01	5.06E+01	5.06E+01	5.06E+01	5.06E+01	5.06E+01	0.00E-01
I 135	4.26E+01	4.26E+01	4.26E+01	4.26E+01	4.26E+01	4.26E+01	4.26E+01	0.00E-01
CS134	3.78E+01	3.78E+01	3.78E+01	3.78E+01	3.78E+01	3.78E+01	3.78E+01	0.00E-01
CS136	5.35E+01	5.35E+01	5.35E+01	5.35E+01	5.35E+01	5.35E+01	5.35E+01	0.00E-01
CS137	1.31E+01	1.31E+01	1.31E+01	1.31E+01	1.31E+01	1.31E+01	1.31E+01	0.00E-01
CS138	4.59E+01	4.59E+01	4.59E+01	4.59E+01	4.59E+01	4.59E+01	4.59E+01	0.00E-01
BA139	9.55E-01	9.55E-01	9.55E-01	9.55E-01	9.55E-01	9.55E-01	9.55E-01	0.00E-01
BA140	6.39E+00	6.39E+00	6.39E+00	6.39E+00	6.39E+00	6.39E+00	6.39E+00	0.00E-01
BA141	1.14E+01	1.14E+01	1.14E+01	1.14E+01	1.14E+01	1.14E+01	1.14E+01	0.00E-01
BA142	1.95E+01	1.95E+01	1.95E+01	1.95E+01	1.95E+01	1.95E+01	1.95E+01	0.00E-01
LA140	5.34E+01	5.34E+01	5.34E+01	5.34E+01	5.34E+01	5.34E+01	5.34E+01	0.00E-01
LA142	5.61E+01	5.61E+01	5.61E+01	5.61E+01	5.61E+01	5.61E+01	5.61E+01	0.00E-01
CE141	1.70E+00	1.70E+00	1.70E+00	1.70E+00	1.70E+00	1.70E+00	1.70E+00	0.00E-01
CE143	7.42E+00	7.42E+00	7.42E+00	7.42E+00	7.42E+00	7.42E+00	7.42E+00	0.00E-01
CE144	1.12E+00	1.12E+00	1.12E+00	1.12E+00	1.12E+00	1.12E+00	1.12E+00	0.00E-01
PR143	2.09E-02	2.09E-02	2.09E-02	2.09E-02	2.09E-02	2.09E-02	2.09E-02	0.00E-01
PR144	5.74E-01	5.74E-01	5.74E-01	5.74E-01	5.74E-01	5.74E-01	5.74E-01	0.00E-01
ND147	3.65E+00	3.65E+00	3.65E+00	3.65E+00	3.65E+00	3.65E+00	3.65E+00	0.00E-01
W 187	1.08E+01	1.08E+01	1.08E+01	1.08E+01	1.08E+01	1.08E+01	1.08E+01	0.00E-01
NP239	3.13E+00	3.13E+00	3.13E+00	3.13E+00	3.13E+00	3.13E+00	3.13E+00	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = INFANT
PATHWAY = POTABLE WATER

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	2.91E+01	2.91E+01	0.00E-01	2.91E+01	2.91E+01	2.91E+01	2.91E+01	0.00E-01
C 14	8.38E+02	8.38E+02	3.92E+03	8.38E+02	8.38E+02	8.38E+02	8.38E+02	0.00E-01
NA 24	5.53E+02	5.53E+02	5.53E+02	5.53E+02	5.53E+02	5.53E+02	5.53E+02	0.00E-01
P 32	1.04E+04	3.63E+03	2.68E+05	1.58E+04	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	2.28E+00	6.64E+01	0.00E-01	0.00E-01	3.25E-01	1.49E+00	2.89E+00	0.00E-01
MN 54	7.45E+02	1.21E+03	0.00E-01	3.29E+03	7.29E+02	0.00E-01	0.00E-01	0.00E-01
MN 56	3.58E-02	1.89E+01	0.00E-01	2.08E-01	1.79E-01	0.00E-01	0.00E-01	0.00E-01
FE 55	3.97E+02	1.89E+02	2.30E+03	1.49E+03	0.00E-01	0.00E-01	7.26E+02	0.00E-01
FE 59	3.46E+03	4.19E+03	5.02E+03	8.77E+03	0.00E-01	0.00E-01	2.59E+03	0.00E-01
CO 58	1.47E+03	1.47E+03	0.00E-01	5.90E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	4.22E+03	4.25E+03	0.00E-01	1.79E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 63	3.64E+03	3.23E+02	1.05E+05	6.49E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	5.45E-02	9.11E+00	1.06E+00	1.20E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CU 64	1.26E+01	5.57E+02	0.00E-01	2.71E+01	4.59E+01	0.00E-01	0.00E-01	0.00E-01
ZN 65	4.80E+03	8.80E+03	3.04E+03	1.04E+04	5.05E+03	0.00E-01	0.00E-01	0.00E-01
ZN 69	5.00E-08	5.48E-05	3.73E-07	6.72E-07	2.79E-07	0.00E-01	0.00E-01	0.00E-01
BR 83	5.73E-02	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 84	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 86	1.34E+04	6.94E+02	0.00E-01	2.71E+04	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	1.18E+04	8.43E+03	4.10E+05	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	9.50E+04	3.82E+04	4.69E+06	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 91	5.19E+01	1.70E+03	1.43E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 92	2.56E-01	7.43E+01	6.89E+00	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 90	2.98E-01	1.53E+04	1.11E+01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91	4.93E+00	1.33E+04	1.85E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 92	3.29E-04	2.24E+02	1.17E-02	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 93	2.11E-02	6.10E+03	7.73E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	5.83E+00	4.09E+03	3.37E+01	8.22E+00	8.86E+00	0.00E-01	0.00E-01	0.00E-01
ZR 97	7.17E-02	1.00E+04	9.15E-01	1.57E-01	1.58E-01	0.00E-01	0.00E-01	0.00E-01
NB 95	1.62E+00	2.37E+03	6.82E+00	2.81E+00	2.01E+00	0.00E-01	0.00E-01	0.00E-01
MO 99	8.53E+02	1.44E+03	0.00E-01	4.37E+03	6.54E+03	0.00E-01	0.00E-01	0.00E-01
TC 99M	5.32E-01	1.20E+01	2.00E-02	4.13E-02	4.44E-01	0.00E-01	2.16E-02	0.00E-01
TC101	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = INFANT
PATHWAY = POTABLE WATER

(mrem gal)/(Ci min)

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
RU103	8.05E+01	2.93E+03	2.41E+02	0.00E-01	5.01E+02	0.00E-01	0.00E-01	0.00E-01
RU105	1.78E-01	2.11E+02	5.30E-01	0.00E-01	3.89E+00	0.00E-01	0.00E-01	0.00E-01
RU106	4.97E+02	3.02E+04	3.98E+03	0.00E-01	4.71E+03	0.00E-01	0.00E-01	0.00E-01
AG110M	7.94E+01	6.23E+03	1.64E+02	1.20E+02	1.72E+02	0.00E-01	0.00E-01	0.00E-01
TE125M	5.15E+02	1.82E+03	3.81E+03	1.27E+03	0.00E-01	1.28E+03	0.00E-01	0.00E-01
TE127M	1.16E+03	3.88E+03	9.63E+03	3.19E+03	2.37E+04	2.78E+03	0.00E-01	0.00E-01
TE127	6.01E+00	5.86E+02	2.79E+01	9.36E+00	6.82E+01	2.27E+01	0.00E-01	0.00E-01
TE129M	2.50E+03	9.68E+03	1.62E+04	5.56E+03	4.05E+04	6.23E+03	0.00E-01	0.00E-01
TE129	6.48E-06	2.22E-03	2.78E-05	9.57E-06	6.91E-05	2.33E-05	0.00E-01	0.00E-01
TE131M	4.80E+02	9.79E+03	1.44E+03	5.82E+02	4.00E+03	1.18E+03	0.00E-01	0.00E-01
TE131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE132	1.29E+03	5.10E+03	2.78E+03	1.38E+03	8.62E+03	2.03E+03	0.00E-01	0.00E-01
I 130	2.28E+02	1.22E+02	2.58E+02	5.68E+02	6.24E+02	6.37E+04	0.00E-01	0.00E-01
I 131	2.83E+03	2.29E+02	5.45E+03	6.43E+03	7.50E+03	2.11E+06	0.00E-01	0.00E-01
I 132	1.44E-01	3.27E-01	1.99E-01	4.04E-01	4.50E-01	1.89E+01	0.00E-01	0.00E-01
I 133	3.97E+02	2.29E+02	9.31E+02	1.35E+03	1.59E+03	2.46E+05	0.00E-01	0.00E-01
I 134	5.83E-07	1.69E-06	8.00E-07	1.64E-06	1.83E-06	3.82E-05	0.00E-01	0.00E-01
I 135	3.51E+01	3.48E+01	4.84E+01	9.62E+01	1.07E+02	8.62E+03	0.00E-01	0.00E-01
CS134	1.17E+04	3.16E+02	6.24E+04	1.16E+05	2.99E+04	0.00E-01	1.23E+04	0.00E-01
CS136	7.92E+03	3.22E+02	7.21E+03	2.12E+04	8.45E+03	0.00E-01	1.73E+03	0.00E-01
CS137	7.17E+03	3.16E+02	8.64E+04	1.01E+05	2.72E+04	0.00E-01	1.10E+04	0.00E-01
CS138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA139	2.36E-05	5.16E-02	8.14E-04	5.40E-07	3.24E-07	0.00E-01	3.27E-07	0.00E-01
BA140	1.38E+03	6.59E+03	2.68E+04	2.68E+01	6.37E+00	0.00E-01	1.65E+01	0.00E-01
BA141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA142	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA140	2.35E-01	1.07E+04	2.31E+00	9.12E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA142	3.27E-07	2.32E-01	3.72E-06	1.37E-06	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	9.16E-01	4.02E+03	1.28E+01	7.78E+00	2.40E+00	0.00E-01	0.00E-01	0.00E-01
CE143	1.12E-01	5.74E+03	1.48E+00	9.84E+02	2.87E-01	0.00E-01	0.00E-01	0.00E-01
CE144	2.76E+01	2.82E+04	4.92E+02	2.01E+02	8.14E+01	0.00E-01	0.00E-01	0.00E-01
PR143	6.34E-01	6.75E+03	1.28E+01	4.78E+00	1.78E+00	0.00E-01	0.00E-01	0.00E-01
PR144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ND147	5.41E-01	5.60E+03	8.60E+00	8.84E+00	3.41E+00	0.00E-01	0.00E-01	0.00E-01
W 187	1.79E+01	3.05E+03	7.46E+01	5.19E+01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NP239	6.93E-02	3.54E+03	1.37E+00	1.23E-01	2.44E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = INFANT
PATHWAY = FRESH WATER FISH

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NA 24	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
P 32	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MN 54	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MN 56	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 55	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 59	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 58	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 63	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CU 64	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 65	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 69	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 83	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 84	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 86	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 91	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 92	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 92	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 93	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 97	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NB 95	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MO 99	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TC 99M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TC101	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = INFANT
PATHWAY = FRESH WATER FISH

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
RU103	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RU105	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RU106	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
AG110M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE125M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE127M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE127	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE129M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE129	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE131M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE132	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 130	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 132	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 134	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS134	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS136	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA139	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA140	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA142	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA140	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA142	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE143	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
PR143	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
PR144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ND147	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
W 187	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NP239	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = INFANT
PATHWAY = ANIMAL DRINKING WATER—MEAT

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NA 24	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
P 32	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MN 54	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MN 56	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 55	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 59	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 58	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 63	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CU 64	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 65	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 69	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 83	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 84	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 86	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 91	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 92	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 92	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 93	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 97	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NB 95	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MO 99	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TC 99M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TC101	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = INFANT
PATHWAY = ANIMAL DRINKING WATER—MEAT

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
RU103	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RU105	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RU106	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
AG110M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE125M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE127M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE127	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE129M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE129	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE131M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE132	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 130	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 132	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 134	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS134	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS136	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA139	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA140	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA142	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA140	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA142	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE143	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
PR143	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
PR144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ND147	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
W 187	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NP239	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = INFANT
PATHWAY = RIVER SHORELINE DEPOSITS

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NA 24	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
P 32	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MN 54	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MN 56	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 55	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 59	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 58	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 63	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CU 64	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 65	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 69	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 83	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 84	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 86	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 91	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 92	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 92	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 93	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 97	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NB 95	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MO 99	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TC 99M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TC101	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = INFANT
PATHWAY = RIVER SHORELINE DEPOSITS

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
RU103	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RU105	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RU106	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
AG110M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE125M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE127M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE127	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE129M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE129	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE131M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE132	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 130	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 132	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 134	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS134	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS136	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA139	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA140	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA142	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA140	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA142	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE143	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
PR143	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
PR144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ND147	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
W 187	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NP239	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = INFANT
PATHWAY = SWIMMING

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NA 24	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
P 32	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MN 54	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MN 56	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 55	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 59	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 58	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 63	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CU 64	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 65	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 69	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 83	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 84	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 86	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 91	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 92	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 92	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 93	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 97	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NB 95	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MO 99	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TC 99M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TC101	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = INFANT
PATHWAY = SWIMMING

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
RU103	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RU105	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RU106	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
AG110M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE125M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE127M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE127	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE129M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE129	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE131M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE132	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 130	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 132	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 134	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS134	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS136	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA139	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA140	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA142	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA140	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA142	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE143	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
PR143	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
PR144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ND147	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
W 187	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NP239	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = INFANT
PATHWAY = BOATING

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NA 24	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
P 32	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MN 54	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MN 56	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 55	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 59	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 58	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 63	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CU 64	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 65	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 69	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 83	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 84	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 86	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 91	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 92	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 92	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 93	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 97	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NB 95	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MO 99	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TC 99M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TC101	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = INFANT
PATHWAY = BOATING

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
RU103	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RU105	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RU106	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
AG110M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE125M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE127M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE127	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE129M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE129	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE131M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE132	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 130	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 132	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 134	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS134	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS136	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA139	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA140	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA142	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA140	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA142	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE143	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
PR143	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
PR144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ND147	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
W 187	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NP239	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

ATTACHMENT 3: DEFUELED OFFSITE DOSE ASSESSMENT MANUAL (DODAM)

Cover Page

DEFUELED OFFSITE DOSE ASSESSMENT MANUAL

GASEOUS AND LIQUID EFFLUENTS

Duane Arnold Energy Center

Record the following: Date/Time: ____________ / ________ Initials: ________

NOTE: *User shall perform and document a Temp Issue / Rev. Check to ensure revision is current, in accordance with procedure use and adherence requirements.*

Prepared By:	J DIERICKX / J Diericks	Date: 11/29/2021
	Print / Signature	

LICENSING MANAGER CROSS DISCIPLINE REVIEW		
Approved By	M. Davis / M Davis (Print / Signature)	Date: 11/29/21

CHEMISTRY MANAGER PROCEDURE APPROVAL		
Approved By	Mike Case / Mike Case (Print / Signature)	Date: 11/29/21

ORG REVIEW		
Approved By	M. Davis / M Davis (Print / Signature)	Date: 11/29/21

SITE DIRECTOR APPROVAL		
Approved By	P HANSEN / P Hansen (Print / Signature)	Date: 11.29.2021

DEFUELED OFFSITE DOSE ASSESSMENT MANUAL

GASEOUS AND LIQUID EFFLUENTS

Duane Arnold Energy Center

Record the following: Date/Time: ____________ / _________ Initials: _______

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Prepared By: ____________________ / ____________________________ Date: _______
Print Signature

LICENSING MANAGER CROSS DISCIPLINE REVIEW

Approved By ____________________ / ____________________________ Date: _______
Print Signature

CHEMISTRY MANAGER PROCEDURE APPROVAL

Approved By ____________________ / ____________________________ Date: _______
Print Signature

ORG REVIEW

Approved By ____________________ / ____________________________ Date: _______
Print Signature

SITE DIRECTOR APPROVAL

Approved By ____________________ / ____________________________ Date: _______
Print Signature

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DEFUELED OFFSITE DOSE ASSESSMENT MANUAL
FOR GASEOUS AND LIQUID EFFLUENTS

1.0 INTRODUCTION

This manual provides a description of the facility's Defueled Offsite Dose Assessment Program (DODAM), the Radiological Effluents Controls Program and the Radiological Environmental Monitoring Program (REMP).

CESSATION OF POWER OPERATIONS

In July of 2018 NextEra Energy Duane Arnold L.L.C announced the cessation of power operations planned for the 4th quarter of 2020, however a severe windstorm on August 10, 2020 damaged the plants cooling towers. There were no abnormal releases as all safety systems functioned as designed. The reactor was permanently defueled on October 8, 2020. The decommissioning process has started with system layup plans for long term dormancy period prior to returning the area to greenfield. The plant is being placed in SAFSTOR.

EFFLUENTS CONTROLS PROGRAM

This program, conforming to 10 CFR 50.36a, provides for the control of radioactive effluents and for maintaining the doses to members of the public from radioactive effluents as low as reasonably achievable (ALARA). The program is contained in sections six and seven of the DODAM, is implemented by procedures, and includes remedial actions to be taken whenever the program limits are exceeded.

Section eight of the DODAM includes the requirement for the Annual Radioactive Material Release Report (ARMRR). The ARMRR covers the operation of the unit during the previous calendar year shall be submitted prior to May 1 of each year in accordance with 10 CFR 50.36a. The report includes a summary of the quantities of radioactive liquid and gaseous effluents and solid waste released from the unit. The material provided in the report is consistent with the objectives outlined in the DODAM and Process Control Program and in conformance with 10 CFR 50.36a and 10 CFR Part 50, Appendix I, Section IV.B.1.

RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM

The REMP provides for representative measurements of radioactivity in the highest potential exposure pathways, verification of the accuracy of the effluent monitoring program and modeling of environmental exposure pathways. Additionally, the REMP is designed to demonstrate that radioactive effluents from DAEC are ALARA.

GROUNDWATER PROTECTION PROGRAM

The Groundwater Protection Program (GWPP) provides representative measurements of radioactivity in the highest potentially exposed aquifer, verification of the accuracy of the GWPP monitoring efforts and modeling of the subsurface. Additionally, the GWPP is designed to protect and mitigate radiological contaminates to groundwater. The GWPP is incorporated within REMP and includes standards set forth in Nuclear Energy Institute, NEI 07-07.

The REMP and GWPP programs are described in sections five, six and seven of the DODAM and conforms to the guidance of Appendix I to 10 CFR Part 50 and 10 CFR 72, Section eight of the DODAM includes the requirement for the Annual Radiological Environmental Operating Report (AREOR). The Annual Radiological Environmental Operating Report covers the operation of the unit during the previous calendar year and is submitted by May 15 of each year. The report includes summaries, interpretations, and analyses of trends of the results of the Radiological Environmental Monitoring Program for the reporting period. The material provided is consistent with the objectives outlined in the DODAM, and in 10 CFR 50, Appendix I, Sections IV.B.2, IV.B.3, and IV.C.

DEFUELED OFFSITE DOSE ASSESSMENT PROGRAM

The methodology and parameters used in the calculation of offsite doses resulting from radioactive gaseous and liquid effluents are included in sections two, three and four of the DODAM. These chapters describe acceptable methods of calculating radioactivity concentrations in the environment and the potentially resultant committed doses to a member of the public in the unrestricted area[a] that are associated with LWR liquid and gaseous effluents.

The methodology stated in this Manual is acceptable for use in demonstrating operational compliance with 10 CFR 20.1301, 10 CFR 50 Appendix I, 10 CFR 72.104, and 40 CFR 190. Only the dose attributable to the Duane Arnold Energy Center[b] is considered in demonstrating compliance with 40 CFR 190 since no other nuclear facility exists within 50 miles of the DAEC.

Calculations are made monthly to assess the potential air doses offsite and to a nearby resident in order to guide the management of station effluents. The receptor is described such that the dose to any resident near the Station is unlikely to be underestimated. Calculations made to assess the radioactive noble gas dose to air are based on the location offsite that could be occupied by a person where the maximum air dose is expected. For these monthly-accumulated dose calculations, atmospheric dispersion and deposition of gaseous effluents may be based on reference meteorological conditions.[c] More conservative conditions (i.e., location and/or exposure pathways expected to yield higher computed doses) than appropriate for the maximally exposed person may be assumed in the dose estimated.

Calculations of dose committed from radioactive releases over extended time (3 and 12 months) are also made for the purpose of verifying compliance with regulatory limits on offsite dose. For these calculations the receptor is selected on the basis of the combination of applicable exposure pathways identified in the land use census and the maximum ground level $\frac{\chi}{Q}$ at a residence, or on the basis of more conservative conditions such that the dose to any resident near the Station is unlikely to be underestimated.

[a] Unrestricted area means outside of the boundary of property owned, leased, or controlled by the Company on which DAEC is sited. The DAEC site boundary is identified by UFSAR Figure 1.2-1.

[b] The Duane Arnold Energy Center, also referred to henceforth as the Station, is defined as including BOTH the Nuclear Reactor Facility and the Independent Spent Fuel Storage Installation (ISFSI).

[c] Reference meteorological conditions are 1971, 1974 and 1975 data composited as discussed in "Duane Arnold Energy Center, Evaluation of Liquid and Gaseous Effluent Releases in Accordance With 10 CFR 50 Appendix I," submitted to the NRC June 3, 1976.

2.0 LIQUID EFFLUENT

2.1 Radioactivity In Liquid Waste

The concentration of radionuclides in liquid waste is determined by sampling and analysis in accord with the surveillance requirements of Section 7.1.2, Table 7.1-2. When a radionuclide is identified, it is reported as being present in the sample even if the concentration is below the required LLD for the analysis.

In November 2020 the plant began to discharge liquid radiation waste to the river. The aqueous concentration is managed and tracked batch by batch to maintain margin significantly below 50CFR, Appendix I regulatory limits. The system has a calibrated radiation monitor and an automatic isolation function. Filters were installed prior to the radiation monitor to ensure the process is ALARA; this focus on best practices has significantly lowered the dose to the public.

2.2 Aqueous Concentration

Radioactive material in liquid effluent is diluted successively by water flowing in the discharge pipe and in the Cedar River. The diluted concentration of radionuclide *i* in a receiving stream is estimated with the equation

$$C_{zi} = C_i \frac{F_1}{F_2}$$

where

C_i = concentration of radionuclide *i* in liquid radwaste released (μCi/mL)

C_{zi} = concentration of radionuclide *i* in the receiving stream (μCi/mL)

F_1 = release rate of liquid radwaste (mL/sec)[d]

F_2 = dilution flow of receiving stream of water (mL/sec)[d]

For the purpose of calculating the radioactivity concentration in water at the restricted area boundary (section 2.5), the flow in the discharge pipe, F_c, is assigned to F_2. The water flow in the discharge pipe may include the liquid waste effluent flow, the liquid radwaste dilution water flow, the cooling tower blowdown flow and other streams such as RHR, GWPP discharges, and emergency service water discharged via the dilution structure and discharge pipe. These streams are illustrated in Figure 2-1.

[d] F_1, F_2, and F_c may have any convenient units of flow (i.e., volume/time) provided the units of all are identical.

In the Cedar River immediately beyond the discharge pipe and the restricted area boundary, the effective dilution is

$$F_2 = F_c \, x \, M$$

Where

F_c = discharge pipe flow

M = factor of additional mixing in the River

A near field mixing ratio from the pipe into the near field of the River, M = 5, is assigned when estimating maximum potential individual doses involving exposure by eating fish. Current and historical field surveys of the Cedar River downstream of DAEC do not indicate the presence of irrigation systems withdrawing water for irrigated crops such as strawberries or other produce. In the event water is drawn from the Cedar River downstream of the Station for drinking water or another exposure pathway, F_2 represents the portion of the Cedar River flow into which the liquid effluent from the Station is effectively mixed.

2.3 Basis of Mixing Ratios

Downstream dilution of aqueous discharge from the DAEC has been estimated based on thermal plume studies conducted in 1974 at the DAEC.[e] Measurements of the discharge temperature and river temperature indicated that the 1°F excess temperature isotherm was about 350 feet downstream of the discharge. This 1°F isotherm represented a dilution of the discharge by the Cedar River of about a factor of 12.

In determining additional dilution within the receiving water for evaluating doses from a plant with cooling towers, the NRC guideline[*f] is that the factor should be limited to a number such that the product of the number and the average blowdown flow to the receiving water body is 1000 cfs or less. At the DAEC, the discharge rate can be conservatively approximated by a cooling tower blowdown rate of 4000 gpm, or about 9 cfs. Using the NRC guideline, an additional dilution of 100 in the Cedar River could be assumed for evaluating doses due to liquid effluent. Since the average flow in the Cedar River is about 3,775 cfs, the additional dilution in the Cedar River is achievable.

These results indicate both a dilution factor of 12 at a downstream distance of 350 feet and conformance to the NRC guideline.

Land Use Censuses (field surveys) have shown that the nearest use of river water (from alluvial wells) is 2.2 miles downstream for the City of Palo and more than 8 miles downstream for the City of Cedar Rapids.

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[e] IELP, Cedar River Baseline Ecological Study, DAEC, annual report, Jan. 1974-Jan.1975.

[f] Boegli, J.S., et. al., Preparation of Radiological Effluent Technical Specifications for Nuclear Power Plants, NUREG-0133, p. 16, October, 1978

For the sake of simplification and conservatism in routine liquid dose calculations, a dilution factor of 5 is assumed for fish and a dilution factor of 10 is assumed for drinking water for the evaluation of doses during DAEC operation.

2.4 Method of Establishing Alarm Setpoints

The liquid radwaste effluent line has a monitor which provides automatic isolation when 10 times the water effluent concentration listed in 10 CFR 20 Appendix B, Table 2, is being exceeded in the unrestricted area. The other liquid effluent pathways have monitors which provide alarms when 10 times the 10 CFR 20 water effluent concentrations is being exceeded in the unrestricted area. Given the nature and frequency of discharges, prompt action to reduce radioactive releases following an alarm, will assure the requirements of 10 CFR Part 20. 1301; 10 CFR Part 50 Appendix I, Section IV; and 40 CFR Part 190 are not exceeded.

The alarm setpoint for the liquid effluent radiation monitor is derived from the concentration limit provided in 10 CFR Part 20.1001-20.2402 Appendix B Table 2 Column 2 applied at the unrestricted area boundary where the discharge pipe flows into the river. The alarm setpoint does not consider dilution, dispersion, or decay of radioactive material beyond the site boundary. That is, the alarm setpoint is based on a concentration limit at the end of the discharge pipe. The radiation monitoring and isolation points are located in each line through which radioactive waste effluent is eventually discharged into the discharge pipe. ALARA is achieved by way of ensuring three times tank volume mixing, sample analysis, and filtration prior to release of individual batches.

The alarm setpoint for effluent monitors on batch releases is based on measurements, according to Table 7.1-2. For liquids released in continuous aqueous discharge which are normally radioactively clean, the setpoint is based on the effective Water Effluent Concentration (WEC) for the most likely contaminating source, i.e., the primary coolant water. A measured spectrum from the primary coolant water is used to determine the effective WEC based on WEC fractions according to 10 CFR 20 Appendix B.

2.4.1 Setpoint for a Batch Release

A sample of each batch of liquid radwaste is analyzed for I-131 and other principal gamma emitters, or for total activity concentration prior to release. The ratio, $FWEC_b$, of the activity concentration in the tank to the unrestricted area WEC (10 CFR Part 20, Appendix B, Table 2, Column 2) is calculated with the equation

$$FWEC_b = \sum_i \frac{C_{bi}}{WEC_i}$$

where

$FWEC_b$ = fraction of unrestricted area WEC in batch derived from activity measured prior to release.

C_{bi} = concentration of radionuclide *i* (including I-131 and principal gamma emitters) in batch sample taken prior to release (μCi/mL)

Whether radioiodine and primary gamma emitters are identified prior to a batch release or not, the liquid radwaste effluent line radiation monitor alarm setpoint is determined with the equation

$$S = 10\,x \left[\frac{A}{FWEC_b} \bullet \frac{Fs_2}{Fs_1} \bullet g \right] + Bkg$$

where

S = radiation monitor alarm setpoint (cpm)

10 = factor to account for fact that DAEC's instantaneous release limit is ten times the listed WECs

A = counting rate (cpm/mL) or activity concentration (μCi/mL) of sample in laboratory analysis. A equals $\sum_i C_{bi}$ if an isotopic analysis were performed or C_b if a gross activity analysis was performed.

g = ratio of effluent radiation monitor counting rate to laboratory counting rate or activity concentration in a given batch of liquid (cpm per cpm/mL or cpm per μCi/mL)

Bkg = monitoring instrument background (cpm)

Fs_1 = flow in the batch release line (gal/min).* Value not greater than the discharge line flow alarm maximum setpoint.

Fs_2 = minimum flow in the discharge pipe (gal/min).[g] Value not less than the discharge pipe flow alarm minimum setpoint.

Note that $A/FWEC_b$ represents the counting rate of a solution having the same radionuclide distribution as the sample and having the water effluent concentration of that mixture.

2.4.2 Setpoint for a Continuous Release

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[g] Any suitable but identical units of flow (volume/time)

Continuous aqueous discharges are sampled and analyzed according to the schedule in Table 7.1-2. The ratio, $FWEC_c$, of the activity concentration in each of the continuous release streams to the unrestricted area WEC is calculated with the equations

$$FWEC_c = \sum_i \frac{C_{ci}}{WEC_i}$$

where

$FWEC_c$ = fraction of unrestricted area WEC in continuous release based upon activity measured in primary coolant sample(s)

C_{ci} = concentration of radionuclide *i* in sample(s) (µCi/mL)

The alarm setpoint of the radiation monitor on a continuous radioactive discharge line is determined with the equation

$$S = 10\,x\left[\frac{A}{FWEC_c} \bullet \frac{Fs_2}{Fs_1} \bullet g\right] + Bkg$$

where

10 = factor to account for fact that DAEC's instantaneous release limit is ten times the listed WECs

A = activity concentration (µCi/mL) or counting rate (cpm/mL) in laboratory of monthly reactor primary coolant sample.

Fs_1 = Flow in the liquid discharge line (mL/sec).[h] Value not greater than off discharge line flow alarm maximum setpoint.

Fs_2 = flow in the discharge pipe (mL/sec).[h] Value not less than discharge pipe flow alarm minimum setpoint.

g = ratio of effluent radiation monitor counting rate to laboratory counting rate or activity concentration in a given batch of liquid (cpm per cpm/mL or cpm per µCi/mL)

10-2021 The radioactivity concentration in continuous aqueous effluent is usually so low that measurement of a representative radionuclide in a sample of the water is uncertain.
10-2021 Thus, the ratio, $A/FWEC_c$, which represents the WEC of a given spectrum of radionuclides, is usually derived from fuel pool or other liquid source analysis(es). Alternatively, it may be determined from analyses of the continuous effluent itself. Once calculated, a more conservative value may be selected.

In the event the concentration of radioactive material in the sample from the continuous release is below measurable levels (i.e., less than the lower limit of detection), the value

h Any suitable but identical units of flow (volume/time) of 1 x 10^{-8} µCi/mL or the equivalent counting rate (cpm/mL) may be substituted for the factor

$$\frac{A}{FWEC_c}\ (i.e., \frac{A}{FWEC_c} = 1 \bullet 10^{-8}).$$

2.5 Radioactivity Concentration in Water at the Restricted Area Boundary

Section 6.1.2 provides limits on instantaneous radioactivity concentration in the unrestricted area due to aqueous effluents from DAEC. Compliance is assessed by monitoring, sampling, analyzing and establishing setpoints according to Section 7.1.2. As long as a liquid effluent monitor named in Table 6.1-1 does not exceed an alarm or trip setpoint, determined in accordance with section 2.3, or as long as the total or gross activity concentration, measured as required in Section 7.1.2, does not exceed 1 x 10^{-7} µCi/mL after dilution in the discharge pipe, Section 6.1.2 is satisfied.

In the event of an alarm, indicating concentrations in the unrestricted area in excess of section 6.1.2 limits, the release shall be terminated and dose calculations will be performed to assure the limits specified in sections 6.1.3 and 6.3.1 are not exceeded.

Compliance with 10 CFR 20.1301 shall not be demonstrated on the basis of determining the average annual liquid effluent concentration. But rather by demonstrating compliance with 40 CFR 190 (i.e., section 6.3.1). Such a practice was deemed acceptable by the NRC in their preamble to the revised 10 CFR 20.[i]

2.6 Accumulated Personal Maximum Dose

Section 7.1.3 requires an assessment to be performed at least once every 30 days in any quarter in which radioactive effluent is discharged which determines whether the dose or dose commitment to a person offsite due to radioactive material released in liquid effluent calculated on a <u>cumulative</u> basis exceeds the limits of Section 6.1.3. The requirement is satisfied by computing the accumulated dose commitment to the most exposed organ and to the whole body of a hypothetical person exposed by eating fish and drinking water taken from the river offsite downstream of the discharge pipe.

The pathway(s) and or age group(s) selected may vary by season. For instance, fishing near the DAEC is practically non-existent during the winter; thus, a dose evaluation of the fish pathway is not required for aqueous effluent discharged during the winter months of January, February, or March.

The accumulated dose commitment is computed at least once every 30 days but may be computed as analyses become available.

Normally, DAEC employs the MIDAS computer program to calculate dose to a member of the public from aqueous effluent that uses the equations in Reg Guide 1.109[j] and standard values therein for maximally exposed people.

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[i] Federal Register Volume 56, #98, Tuesday, May 21, 1991, p. 23360
[j] USNRC, Regulatory Guide 1.109, revision 1, Position C.1, pp. 1.109-2 thru 1.109-4, Oct. 1977

Alternatively, the dose may be calculated in the following way, for instance, in the event calculations by hand were necessary.

$$\Delta D_{ank} = 3.785 \bullet x\, 10^{-3} \sum_{i} C_{ik} \bullet \Delta T_k \sum_{e} \frac{F\,1_k}{F\,2_{ek}} \bullet A_{eani}$$

$$D_{an} = \sum_{k} \Delta D_{ank}$$

where

ΔD_{ank} = the dose commitment (mrem) to organ *n* of age group *a* due to the isotopes identified in analysis *k*, where

the analyses are those required by Table 7.1-2. Thus the contribution to the dose from gamma emitters become available on a batch basis for batch releases and on a weekly basis for continuous releases. Similarly the contributions from H-3 is available on a monthly basis and the contributions from Fe-55, Sr-89, and Sr-90 become available on a quarterly basis.

D_{an} = the dose commitment during the quarter-to-date to organ *n*, including whole body, of the maximally exposed person in age group *a* (mrem)

A_{eani} = transfer factor relating a unit release of radionuclide *i* (Ci) in a unit stream flow (gal/min) to dose commitment to organ *n*, or whole body, of an exposed person in age group *a* $\left[\frac{mrem\ gal}{Ci\ \min}\right]$ via environmental pathway *e*.

C_{ik} = the concentration of radionuclide *i* in the undiluted liquid waste represented by sample *k* to be discharged (µCi/mL)

Δt_k = duration of radioactive release represented by sample *k* which occurs within time boundaries TB and TE and during which concentration C_{ik} and flows $F1_k$ and $F2_k$ exist. (min.)

$3.785 \bullet 10^{-3}$ = conversion constant (3785 mL/gal • 10^{-6} Ci/µCi)

$F\,1_k$ = flow in the radioactive waste release line (gal/min)* represented by sample *k*.

$F\,2_{ke}$ = flow into which radioactive release represented by sample *k* is mixed in the river at the point of exposure or withdrawal of water for use (same units as $F1_k$)*

$= \quad M \times F_{ck}$

where

F_{ck} = discharge pipe flow (gal/min)[k] during release represented by sample *k*

M = factor of additional mixing in the Cedar River

Pathway-to-dose transfer factors, A_{eani}, for use in calculating the dose commitment arising from radioactive material released in aqueous effluents are tabulated in Appendix C. These dose transfer factors were derived using LADTAP II and standard values from Regulatory Guide 1.109, revision 1, except where corrections have been incorporated in LADTAP II. Appropriate tables representing applicable environmental pathways of exposure and most exposed age group(s) are selected and used in calculating the dose commitment. The pathway(s) and/or age group(s) selected may vary by season.

Pathways of Maximum Exposure to a Member of the Public:

- Ingestion of Fish taken from the river near the discharge pipe
  - Age Group: Adult
  - Dilution: $F_2 = 5F_c$
  - Food: Sport Fish

- Drinking Water Pathway
  - Age Group: Infant
  - Dilution: $F_2 = 10F_c$
  - Dilution: $F_2 = 5F_c$ only when Land Use Census identifies this pathway within 3 miles of the plant

Variables F_1, F_2, and F_c are also defined in section 2.2.

2.7 Projected Maximum Dose to a Person Offsite

The dose commitment to a person offsite due to radioactive material released in liquid effluent may be projected by calculating the extrapolated whole body and most exposed organ dose commitments to a hypothetical person exposed via the same pathways evaluated in section 2.6. The potential dose commitments to organs and to the whole body are computed separately.

[k] Any suitable, identical units of flow (volume/time)

The dose commitment to a maximally exposed hypothetical person will be projected by calculating the doses accumulated during the most recent three months (according to the method described in section 2.6) and by assuming the result represents the projected doses during the current quarter. Alternatively, the quarterly dose commitment may be projected by using the equation:

$$P_{an} = \frac{92\ D_{an}}{X}$$

where

P_{an} = projected dose commitment (mrem) to organ *n* (including whole body) of age group *a* for the current quarter

92 = number of days in a quarter

X = number of days to date in current quarter

D_{an} = dose commitment to organ *n*, including whole body, of the maximally exposed person in age group *a* based on available aqueous effluent measurements during the quarter to date (mrem)

2.8 Groundwater Pathway

Low levels of radioactive contamination have been identified in sub-surface water on site. There is the possibility that this water could represent potential exposure to the public. Hydrogeology studies indicate that this water will migrate towards the Cedar River in sectors ranging from south-southeast to the southeast. The only potential exposure pathways are described in Section 2.6.

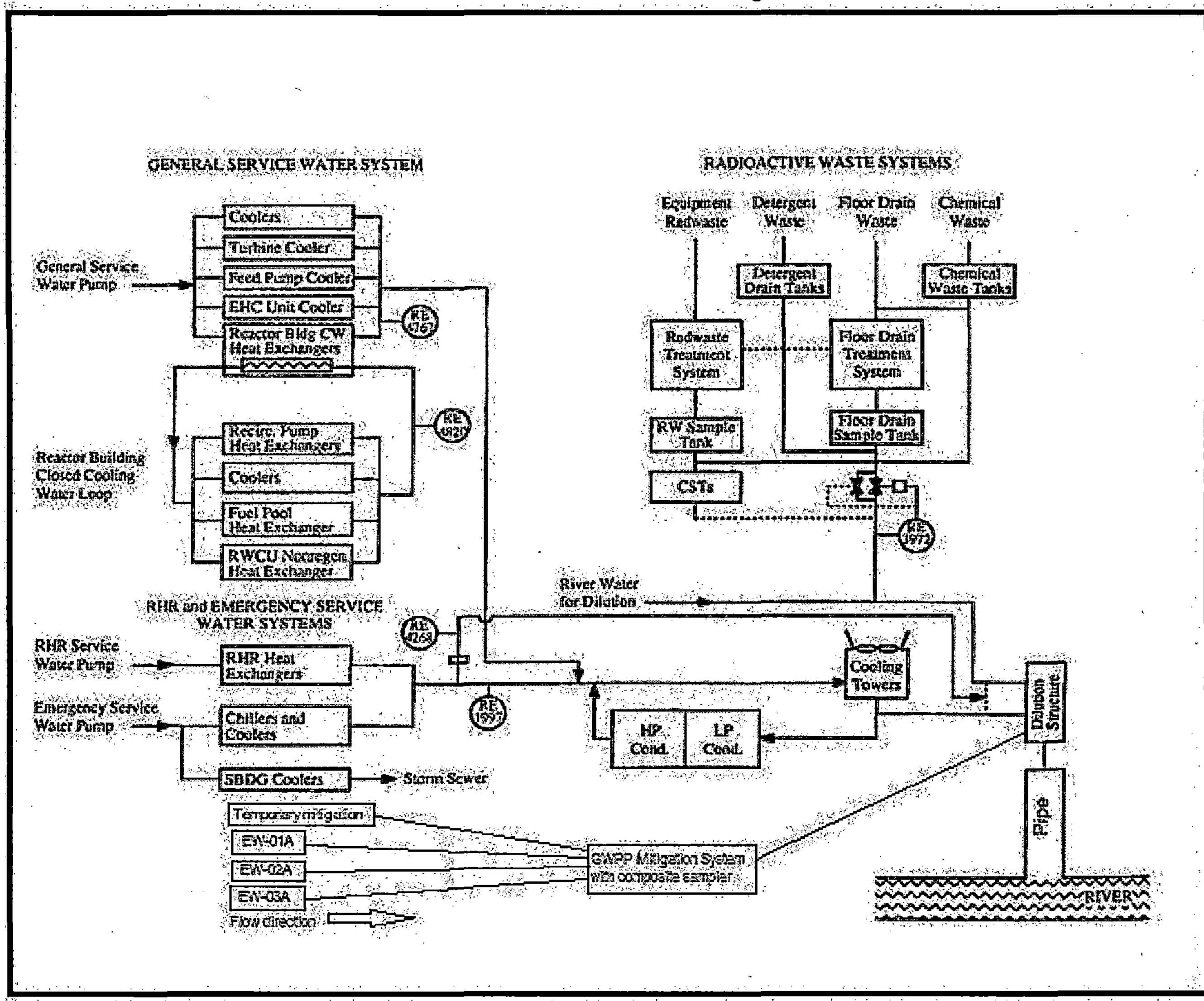
To monitor for the migration of any radioactive contamination beyond the owner controlled area, sampling of on-site ground water, Cedar River water, and down gradient drinking water is performed as a function of the REMP and GWPP. These programs are described in Section 5.0.

Release standards of contaminated groundwater are set forth in Section 6.0, Radiological Liquid Effluent Release O.6.1.2, Table 7.1-2. In accordance with GWPP Administrative Control Procedure (ACP) 1411.35 and Environmental Protection Agency (EPA) drinking water standards for tritium, 20,000 pCi/L, groundwater batch and continuous releases are expected less than (<) 20,000 pCi/L. Groundwater samples are analyzed on-site and validated by off-site secondary laboratory. Off-site laboratory results are published in Annual Radiological Environmental Operating Report.

In February 2016, routine GWPP sampling identified a contaminate plume in the shallow aquifer (less than 25 feet deep). By February 2017, three Extraction Wells were installed in the shallow aquifer to facilitate continuous groundwater withdrawal to mitigate the contaminated plume. Extraction well EW01A (Table 5-1, D-68) located inside the Protected Area, is the primary well designed to remove tritiated groundwater at less than 17.3 gallons per minute from a narrow plume less than 25 feet below the surface. Extraction wells EW02A (D-69) and EW03A (D-70) are located southeast of EW01A and were similarly designed to withdraw the same volume of contaminated or uncontaminated groundwater from the shallow aquifer. These Extraction Wells, their AC-powered pumps, and the associated piping system are considered to be tools that may be used by Chemistry for groundwater mitigation.

Based on calibration of pump speed, flow rate, dilution flow rates, and groundwater tritium concentrations, tritiated groundwater is released with anticipated concentrations less than the EPA drinking water limit of 20,000 pCi/L. Liquid effluent releases from groundwater are published in the Annual Radiological Material Release Report.

Figure 2-1
Duane Arnold Energy Center
Radioactive Water Effluent Diagram



3.0 GASEOUS EFFLUENT

3.1 Introduction

10-2021 The Station discharges gaseous effluent through discharges ventilation air from the reactor and radwaste building through the reactor building vents. Ventilation air from the Turbine Building is discharged through the Turbine Building vent and through the Reactor Building vent. Ventilation from the LLRPSF is discharged through the LLRPSF vent. These gaseous effluent streams, radioactivity monitoring points, and effluent discharge points are shown schematically in Figure 3-1. Gaseous release point
10-2021 locations and elevations at the Station are described in Table 3-1. Gaseous discharges via the building vents are assumed to be ground-level, building wake, or split wake releases.

3.2 Radioactivity in Gaseous Effluent

For the purpose of estimating offsite radionuclide concentrations and radiation doses, measured radionuclide concentrations in gaseous effluent and in ventilation air exhausted from the Station are relied upon.

The gross radioactivity of noble gases discharged is measured by the radioactive noble gas effluent monitors according to Tables 6.2-1 and 7.2-2. Radionuclides other than noble gases in gaseous effluents are measured by sampling and analyses in accordance with Table 7.2-2. Each radionuclide measured in an effluent may be assumed to be discharged uniformly during the sampling period. When radioactivity is identified at a concentration below the LLD for the analysis, that concentration is reported. When radioactivity is not identified in a sample, it is not reported as being present in that sample.

The quantity of radioactive noble gas discharged via a vent during an interval of time is
10-2021 determined by integrating the release rate measurement of each effluent noble gas monitor. An hourly interval is normally used for dose rate assessments and a daily or longer interval is used for dose assessments. If ΔQ_j represents the gross activity of noble gas discharged via a vent and g_i represents the fraction of radionuclide *i* in the distribution of radioactive gases in that effluent stream, then the quantity of radionuclide *i*
10-2021 released in the gaseous effluent stream during counting interval *j* is estimated by the relation:

$$\Delta Q_{ij} = \Delta Q_j \ x \ g_i$$

The distribution of radioactive noble gases in gaseous effluent streams is determined by gamma spectrum analysis of gaseous effluent samples in accordance with Table 7.2-2. Results of one or more previous analyses may be averaged to obtain a representative spectrum. In the event a representative distribution is not available or is unobtainable from sample(s) of an effluent stream taken during the current quarter, it will be derived from past measurements, e.g., earlier sample results or annual radioactive material release reports. Alternatively, a noble gas spectrum for a given effluent stream in Table 3-2 herein, may be assumed.

An airborne discharge of radionuclides other than noble gases may be represented by multiple samples with each sample providing a measure of the concentration of specific radionuclides, C_i, in gaseous effluent discharged at flow, F_a, during a time increment Δt. Thus, each release is quantified according to the relation:

$$Q_{ik} = \sum_{j} C_{ik} \; F_{aj} \; \Delta t_j$$

where

Q_{ik}	=	the quantity of radionuclide *i* released in a given effluent stream based on analysis *k* (Ci)
C_{ik}	=	concentration of radionuclide *i* in gaseous effluent identified by analysis *k* (µCi/ML or Ci/m^3)
F_{aj}	=	effluent stream discharge rate during the increment Dt_j (m^3/sec)
Δt_j	=	time increment during which radionuclide *i* at concentration C_{ik} is being discharged (sec)

The analysis index *k* may represent either a grab sample, integrated sample, or a composite sample required by the effluent sampling and analysis program specified in Table 7.2-2.

3.3 Not Used

3.4 Effluent Noble Gas Monitor Alarm Setpoint

Section 6.2.2 provides limits on dose equivalent rates associated with airborne radioactive materials concentrations in the unrestricted area due to airborne effluents from the Station. Instrumentation is provided to monitor gamma radiation in the airborne effluents according to Table 6.2-1. Each effluent noble gas monitor includes an alarm that can be set to activate when the dose rate off site or the noble gas concentration at ground level offsite is expected (calculated) to exceed a specified level. Compliance with the limits on dose rate from noble gases is demonstrated by setting each gaseous effluent monitor alarm setpoint so that an alarm will occur at or before the dose rate limit for noble gases is reached. If an alarm occurs with the setpoint at the limit, compliance with Section 6.2.2.a is assessed as described in section 3.5.

On the basis of effluent noble gases from the DAEC during recent years, the gamma dose rate to a person's whole body is expected to be a larger fraction of the limit, 500 mrem/yr, than is the beta plus gamma dose rate to skin is to its limit, 3000 mrem/yr. As a result, a gaseous effluent monitor setpoint may be derived on the basis of the gamma dose rate to a person's whole body alone such that an alarm is set to occur at or before the whole body dose rate offsite exceeds 500 mrem/yr.

A noble gas monitor may be set to activate an alarm at a lower setting than the derived setpoint corresponding to the dose rate limit (or corresponding concentration limit). In the event an alarm occurs at the lower setting, the monitor record is compared with the derived setpoint. If the derived setpoint is exceeded, compliance with Section 6.2.2.a is assessed as described in section 3.5.

Each radioactive noble gas effluent monitor setpoint is derived on the basis of whole body dose equivalent rate in the unrestricted area. Setpoints for gaseous effluent monitors may be set independently because excessive effluent release via the turbine building vent is unlikely. Other releases are likely to be initiated independently and released from different levels, i.e., vent and stack releases with points of maximum concentration offsite not likely to coincide.

For the purpose of deriving a setpoint, the distribution of radioactive noble gases in an effluent stream is determined as described in section 3.2.

Setpoint Determination

The alarm setpoint of a radioactive noble gas effluent monitor may be calculated on the basis of whole body dose equivalent rate offsite, 500 mrem/yr.

10-2021

The setpoint of a monitor of a ground-level or building release, e.g., from the turbine building vent or 3 reactor building vents or LLRPSF vent may be calculated with the equation:

$$S = \left[1.06 \bullet \frac{h}{f\left(\frac{\chi}{Q}\right)} \frac{\sum_i C_i}{\sum_i C_i \bullet DF_i^v} \right] + Bkg$$

where

s = the alarm setpoint (cpm) or (MR/hr)

h = monitor response to activity concentration of effluent being monitored,

$$\left(\frac{cpm}{\mu Ci / cm^3}\right) \quad or \quad \left(\frac{mR / hr}{\mu Ci / cm^3}\right)$$

C_i = relative concentration of noble gas radionuclide *i* in effluent at the point of monitoring ($\mu Ci/cm^3$)

χ / Q = atmospheric dispersion from point of ground-level or building wake release to the location of potential exposure (sec/m^3)

10-2021 DF_i^v = factor converting ground-level of split-wake release of radionuclide *i* to the whole body dose equivalent rate at the location of potential exposure

$$\left(\frac{mrem}{yr \bullet \frac{\mu Ci}{m^3}}\right)$$

f = flow of gaseous effluent stream, i.e., flow past the monitor (ft^3/min)

Bkg = monitoring instrument background (cpm) or (MR/hr)

$$1.06 = 500 \frac{mrem}{yr} \bullet 60 \frac{\sec}{\min} \bullet 35.3 \frac{ft^3}{m^3} \bullet \frac{1\, m^3}{10^6\, cm^3}$$

Kaman Industries performed primary calibrations using Xe-133 and Kr-85. The average response for Xe-133 was 4.83e+07, and for Kr-085 was 7.65e+07 cpm/μCi/cc. Because Xe-133 response factor is more conservative, *h* will be set at 4.83e+07 cpm/μCi/cc. Response factor *h* for Kr-85 may be considered if conditions are more appropriate.

The concentration of each noble gas radionuclide, C_i, in a gaseous effluent is determined as discussed earlier in this section.

Since the dose rate limits for airborne effluents apply everywhere offsite, alarm setpoints are determined and compliance is assessed at the site boundary where the minimum atmospheric dispersion (maximum χ / Q) occurs. The atmospheric dispersion factor and the dose conversion factor DF_i^s depend on local conditions. The value of χ / Q adopted in a setpoint calculation will be based either on prevailing meteorological conditions or on reference meteorological conditions at the DAEC. The minimum atmospheric dispersion offsite from a ground-level or building wake release derived from reference meteorological conditions is at the site boundary 1260 meters NNW of the Station where:

$$\left(\frac{\chi}{Q}\right)_{vent} = 4.3 \bullet 10^{-6} \sec / m^3$$

The dose conversion factors, DF_i^s, used in setpoint calculations for gaseous effluent monitors are in Table 3-4. In the event DF_i^s is derived on the basis of prevailing meteorology, it will be calculated in accordance with Regulatory Guide 1.109, Appendix B.

3.5 Dose Equivalent Rate Offsite

Section 6.2.2 provides limits on dose equivalent rates associated with airborne radioactive materials concentrations in the unrestricted area due to airborne effluents from the Station. Compliance is assessed on the basis of measurements specified in Table 7.2-2.

3.5.1 Noble Gas

10-2021 Limits on radioactive noble gas in the unrestricted area are provided in Section 6.2.2.a. Each radioactive noble gas effluent monitor is set to alarm when, or below when, the noble gas in airborne effluent from a vent is expected to cause either dose rate limit in Section 6.2.2.a to be exceeded. In the event an airborne effluent release from the Station exceeds the derived setpoint (limit) for an effluent noble gas monitor (except when caused by the performance of a Surveillance Test Procedure), an assessment of compliance is performed as described herein.

The quantity of radioactive noble gas released in a increment of time is measured by the radioactive noble gas effluent monitors and the distribution of radioactive noble gases in a gaseous effluent stream is determined as described in section 3.2 herein.

Compliance with Section 6.2.2.a may be assessed by calculating the dose equivalent rate as described hereafter and by comparing it with the limiting dose rate in the Specification.

3.5.1.1 Total Body Dose Rate

For evaluating compliance with Effluent Control, Section 6.2.2.a, the total body dose equivalent rate due to noble gas gamma radiation is calculated with the equation:

10-2021

$$\overline{D_\gamma} = \frac{1}{3600}\left[\sum_v \sum_i \left(\frac{Qg_i}{t} \bullet \left(\frac{x}{Q}\right) \bullet DF_i^v\right)_v\right]$$

where

$\bar{D}_\gamma$ = noble gas gamma dose rate to total body (mrem/yr)

Qg_i / t = quantity of noble gas radionuclide *i* discharged (µCi) during time increment *t* (hr)

10-2021 DF_i^v = factor converting time integrated, ground level concentration of noble gas nuclide *i* to total body dose from gamma dose from gamma radiation $\left(\frac{mrem}{\mu Ci\ yr / m^3}\right)$

$\frac{1}{3600}$ = conversion (hr/sec)

10-2021

The dose from noble gases released from a vent (near ground level) or to skin from an elevated release is derived from a semi-infinite cloud model. Noble gas semi-infinite cloud gamma-to-total body dose factors, DF_i^v, are listed in Table 3-4.

When the total body and organ doses from noble gases are computed as required by Section 7.3.1, the nearby resident exposed to the maximal ground-level noble gas concentration (maximum $\frac{\chi}{Q}$) is selected as the receptor.

10-2021 Alternatively, the dose from noble gases may be computed at 1260 meters NNW of the reactor, a location identified in Figure 3-2, where the nearby resident who may be exposed to maximal ground-level noble gas concentrations (maximum $\frac{\chi}{Q}$) is selected as the receptor.[c] In that case, values of the dose factors DF_i^v in Table 3-4 are employed.[m] In those computations, reference meteorology was assumed as the basis of atmospheric dispersion. For discharge from a vent, reference meteorological dispersion at the 1260 m NNW, $\left(\frac{\chi}{Q}\right)_v$, is 4.3×10^{-6} sec/m^3.

[m] Dose transfer factors DF_i^v in Table 3.4 are a units conversion of values in Regulatory Guide 1.109, Table B-1, column 5.

[c] Due to limitations of the Midas© software used to perform these calculations, a conservative higher calculated dose may be reported. When using Annualized Average Meteorological data, the Midas system uses known X/Q values from the 1 mile radius to extrapolate X/Q values at the site boundary. The resulting maximum site boundary X/Q value is greater than 4.3×10^{-6} sec/m^3 and is located on the site boundary toward the SSE.

3.5.1.2 Skin Dose Rate

The skin dose equivalent rate due to radioactive noble gas is calculated with the equation:

10-2021

$$\overline{D_\beta} = \frac{1}{3600}\left[\sum_v \sum_i S_{\beta i}\left(\frac{Q_i}{t} \bullet \left(\frac{x}{Q}\right)\right)_v\right]$$

where

$\overline{D_\beta}$ = noble gas beta dose rate to skin (mrem/yr)

$\frac{Q_i}{t}$ = quantity of noble gas radionuclide *i* (µCi) discharged during time increment *t* (hr)

$S_{\beta i}$ = factor converting time integrated ground level concentration of noble gas to skin dose from beta radiation $\left[\frac{mrem\ m3}{\mu Ci\ yr}\right]$

$\frac{1}{3600}$ = conversion (hr/sec)

Compliance with Section 6.2.2.a dose rate to skin is evaluated by calculating the noble gas beta dose equivalent rate offsite at a location 1260 meters NNW of the Station, which is also identified in Figure 3-2.[d] At that location, the reference atmospheric dispersion factors to be used in the calculations are:

10-2021

$$\left(\frac{X}{Q}\right)_v = 4.3 \bullet 10^{-6}\ sec\ /\ m^3$$

Alternatively, averaged meteorological dispersion data coincident with the period of release may be used to evaluate the dose rate. The semi-infinite noble gas cloud-to-skin dose equivalent factors are in Table 3-4. They are also derived from Regulatory Guide 1.109, Table B-1.

[d] Due to limitations of the Midas© software used to perform these calculations, a conservative higher calculated dose may be reported. When using Annualized Average Meteorological data, the Midas system uses known X/Q values from the 1 mile radius to extrapolate X/Q values at the site boundary. The resulting maximum site boundary X/Q value is greater than 4.3 x 10^{-6} sec/m^3 and is located on the site boundary toward the SSE.

3.5.2 Iodine, Tritium, and Particulates

Section 6.2.2.b provides a limit on iodine-131, iodine-133, H-3, and on radioactive particulates having 8 day or longer half-lives in air in the unrestricted area around the Station. In the event airborne effluent from the Station causes a radioactive noble gas effluent monitor to alarm (except when alarm is due to the performance of a Surveillance Test Procedure) or if the assessment required by Section 7.2.4 shows Section 6.2.4 to have been exceeded, an assessment of compliance with Section 6.2.2.b will be performed using a method described in this section.

3.5.2.1 Organ Dose Rate[e]

Compliance with Section 6.2.2.b is assessed by calculating the dose rate* to the most exposed organ of an assumed adult member of the public inhaling airborne I-131, I-133, H-3, and inhaling radioactive particulates having half-lives of 8 days or longer at the location in the unrestricted area having the maximum potential concentration of the effluents (i.e., the location at which reference meteorological data indicates minimum atmospheric dispersion from the Station (max χ / Q).

The organ dose rate is calculated with the following equations:

For a vent discharge:

$$\overline{D}_{anv} = \frac{8.766E-3}{TE-TB} \sum_{i} \sum_{k} Q_{ikv} \; TA_{ani} \left(\frac{\chi_i}{Q}\right)_v$$

10-2021 where

$\overline{D}_{anv}$ = dose equivalent rate from a vent discharge (mrem/yr)

10-2021

Q_{ikv} = quantity of radionuclide *i* released in a given effluent stream based on
10-2021 analysis *k* (µCi) during discharged time increment TB to TE (hr) of interest

10-2021 Q_{ikv} = quantity of radionuclide *i* released in a given effluent stream based on analysis *k* µCi) during discharged time increment TB to TE (hr) of interest

TA_{ani} = factor converting airborne concentration of radionuclide *i* to dose commitment to organ *n* of a person in age group *a* where exposure is directly to airborne material

$$\left(\frac{mrem}{\left(Ci \sec\right) / m3}\right)$$

[e]For inhaled or ingested radioactive material, the consequent "dose" means the committed dose equivalent. The "dose rate" is the committed dose equivalent per unit of time of exposure to the radioactive material in the environment.

10-2021 $\left(\frac{\chi_i}{Q}\right)_v$ = atmospheric dispersion from stack and vent, respectively, to ground level at location of interest (sec/m^3)

$8.766E-3$ = Conversion (1 Ci/^{1}E6 µCi)(8766 hr/yr)

Radionuclides other than noble gases airborne effluent are measured and quantified as described in section 3.2. Normally, radioactive material measured in effluent is assumed to be discharged uniformly over the period represented by the sample.

The averaging time of the measured releases used to evaluate compliance will not exceed 92 days for Sr-89 and Sr-90 and will not exceed 31 days for the other radionuclides.

The maximum offsite exposure potential is expected to occur at 1260 meters NNW of the Station where the reference atmospheric dispersion, to be used in the calculation is

10-2021 $\left(\frac{\chi}{Q}\right)_v$ = $4.3 \bullet 10^{-6}$ sec/m^3

Currently, compliance with Section 6.2.2.b is evaluated by calculating an adult inhalation dose rate at 1260 meters NNW of the Station[nf]. The dose transfer factors, TA_{ani}, used in the computation are tabulated in Appendix A.

3.5.2.2 Rainwater Recapture of Tritium

The phenomenon of rainwater recapture of the tritium present in gaseous effluents has been observed on site. A liquid pathway analysis of the activity present in the rainwater runoff has been performed. The maximum dose to a member of the public from this liquid has been conservatively estimated to be less than 0.01 mrem per year.

3.6 Noble Gas Gamma Radiation Dose Accumulated in Air

Section 6.2.3 requires that the offsite air dose during any calendar quarter not exceed 5 mrad and the annual air dose not exceed 10 mrad from noble gas gamma radiation. Section 7.2.3.1 requires a monthly calculation assessment to verify that the cumulative air dose due to gamma radiation from radioactive noble gas released in gaseous effluents during the quarter and year do not exceed Section 6.2.3.

The distribution of radioactive noble gases in gaseous releases and the quantity of radioactive noble gas discharged during an interval of time are determined as described in section 3.2 herein.

The gamma radiation dose to air offsite as a consequence of noble gas discharge from DAEC is calculated with the

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[f]Due to limitations of the Midas© software used to perform these calculations, a conservative higher calculated dose may be reported. When using Annualized Average Meteorological data, the Midas system uses known X/Q values from the 1 mile radius to extrapolate X/Q values at the site boundary. The resulting maximum site boundary X/Q value is greater than 4.3 x 10^{-6} sec/m^3 and is located on the site boundary toward the SSE.

10-2021

$$D_{\gamma} = \sum_{v}\sum_{i}\left(A_{\gamma_i^v}\sum_{j}\Delta Q_j \bullet gi \bullet \left(\frac{X}{Q}\right)_v\right)$$

where

10-2021 D_{γ} = noble gas gamma dose to air due to effluent from vent (mrad)

10-2021 ΔQ_j = total measured radioactivity release via vent measured by noble gas effluent monitor during counting interval *j* (µCi)

g_i = the fraction of radioactive gas in a given effluent stream attributable to noble gas radionuclide *i*.

10-2021 $A\gamma_i^v$ = factor converting time integrated, ground-level concentration of noble gas to air dose from gamma radiation

$$\left(\frac{mrad}{(\mu Ci \sec)/m^3}\right)$$

$\left(\frac{\chi}{Q}\right)_v$ = atmospheric dispersion factor for a vent (ground-level or building wake) discharge (sec/m^3)

Section 7.2.3.1 is satisfied by calculating the noble gas gamma radiation dose to air at the offsite location identified in Figure 3-2. At that location, 1260 meters NNW of the Station[h], the reference[*g] atmospheric dispersion factor to be used is

$$\left(\frac{\chi}{Q}\right)_v = 4.3 \bullet 10^{-6} \sec/m^3$$

Value of $A\gamma^v$ appropriate for use at that location, assuming reference meteorological

10-2021 conditions, are listed in Table 3-3. Dose transfer factors for vent discharges, $A\gamma_i^v$, are equivalent to factors in Regulatory Guide 1.109, Table B-1, g-air expressed in different units.

[g] Due to limitations of the Midas© software used to perform these calculations, a conservative higher calculated dose may be reported. When using Annualized Average Meteorological data, the Midas system uses known X/Q values from the 1 mile radius to extrapolate X/Q values at the site boundary. The resulting maximum site boundary X/Q value is greater than 4.3 x 10^{-6} sec/m^3 and is located on the site boundary toward the SSE.

[h] Reference atmospheric conditions are summarized and discussed in "Duane Arnold Energy Center, Evaluation of Liquid and Gaseous Effluent Releases in Accordance with 10 CFR 50 Appendix I," submitted to NRC June 3, 1976, Reference atmospheric dispersion factors tabulated therein, also appear in Appendix B herein.

3.6.1 Alternate Method of Evaluating Compliance with Gamma Air Dose Limits

Alternatively, the gamma radiation dose to air offsite may be calculated with the equation

10-2021

$$D_\gamma = \frac{1}{0.8} \sum_v \sum_j \left(\Delta Q_j \bullet \left(\frac{\chi}{Q}\right) A_{\gamma veff} \right)_v$$

where

10-2021

$A_{\gamma_{veff}}$ = an effective dose conversion factor based on the typical radioactive distribution in vent releases converting a time integrated, ground level concentration of noble gases to air dose from gamma radiation $\left(\frac{mrad}{(\mu Ci \ \sec/m^3)}\right)$

0.8 = a factor of conservatism which compensates for variability in radionuclide distribution

The derivation and basis of the effective gamma air dose conversion factor are provided in Appendix B. Values of the effective factors are tabulated in Table 3-3. By inserting
10-2021 the appropriate values for $D\gamma$ (5 mrad/quarter γ-air dose) and Ag_{veff} (6.4×10^{-5} mrad/(µCi sec/m^3)) into the equation above and solving for $(\Delta Q_j)v$, release quantities of noble gases from the vent corresponding to the technical specification limit of 5 mrad/quarter
10-2021 (total for all release points) may be determined. The limit of 5 mrad/quarter is described in section 6.2.3 and is for the total of all release points. At the location, 1260 meters NNW of the station, (which is the controlling location based on reference meteorology) the release limits are individually

10-2021

Release Point	Quarterly Limit (Ci)	Annual Limit (Ci)
Vent	1.25×10^4	2.5×10^4

10-2021 The following equations may be used to assess discharges for compliance with the quarterly release limits on noble gas gamma dose to air.

10-2021

$$\frac{\sum_v \sum_j (\Delta Q_j)_v}{12{,}500} \leq 1$$

or, on a monthly rate basis (although not a requirement)

10-2021

$$\frac{\sum_{v}\sum_{j}(\Delta Q_j)_v}{12{,}500} \le \frac{1}{3}$$

10-2021 The equations which may be used to assess discharges for compliance with the annual air dose limits from noble gas are:

10-2021

$$\frac{\sum_{v}\sum_{j}(\Delta Q_j)_v}{25{,}000} \le 1$$

or, on a monthly rate basis (although not a requirement)

10-2021

$$\frac{\sum_{v}\sum_{j}(\Delta Q_j)_v}{25{,}000} \le \frac{1}{12}$$

10-2021 As long as these relations are satisfied for vent releases of noble gases, no additional calculations are needed to verify compliance with the gamma-air dose limits of Section 6.2.3. Calculations of beta air doses per Section 3.6 may be omitted as discussed in Appendix B.

3.7 Noble Gas Beta Radiation Dose Accumulated in Air

Section 6.2.3 requires that the offsite air dose during any calendar quarter not exceed 10 mrad from noble gas beta radiation and not exceed 20 mrad during any calendar year. Section 7.2.3.1 requires a monthly assessment to verify that the cumulative air dose due to beta radiation from radioactive noble gas released in gaseous effluents not exceed either limit of Section 6.2.3.

The radioactive noble gas distribution and activity discharged are determined as described in paragraph 3.6 herein.

The beta radiation dose to air offsite as a consequence of noble gas released from the Station is calculated with the equation:

10-2021

$$D_\beta = \sum_v \sum_i A_{\beta_i} \sum_j \left[\Delta Q_{jgi} \frac{\chi}{Q}\right]_v$$

where

D_β = noble gas beta dose to air due to stack and vent releases (mrad)

A_{β_i} = factor converting time-integrated, ground-level concentration of noble gas radionuclide *i* to air dose from beta radiation $\left(\frac{mrad - m^3}{\mu Ci - \sec}\right)$

10-2021

$\left(\frac{\chi}{Q}\right)_v$ = atmospheric dispersion factor for a vent (ground level or building wake) discharge (sec/m^3).

Specification 7.2.3.1 is satisfied by calculating the noble gas beta radiation dose to air at the location identified on Figure 3-2. At that location, 1260 meters NNW of the reactor[i], the reference atmospheric dispersion factors to be used are

10-2021

$\left(\frac{\chi}{Q}\right)_v$ = 4.3×10^{-6}

[i]Due to limitations of the Midas© software used to perform these calculations, a conservative higher calculated dose may be reported. When using Annualized Average Meteorological data, the Midas system uses known X/Q values from the 1 mile radius to extrapolate X/Q values at the site boundary. The resulting maximum site boundary X/Q value is greater than 4.3×10^{-6} sec/m^3 and is located on the site boundary toward the SSE.

3.8 Dose Due to Iodine and Particulates in Gaseous Effluents[j]

Section 6.2.4 requires that I-131, I-133, H-3, and radioactive material in particulate form having half-lives greater than 8 days in gaseous effluents released to the area offsite cause no more than 7.5 mrem to any organ of a member of the public during a calendar quarter and no more than 15 mrem during any calendar year. Section 7.2.4 requires an assessment at least once every month to verify that the cumulative dose commitment does not exceed either limit of Section 6.2.4.

10-2021 Airborne releases are discharged via building vents and treated as a ground-level, building wake, or split wake release. Radionuclides mentioned above in airborne effluents that are measured by the sampling and analysis schedule in Table 7.2-2 are included in the release term used to calculate doses. Section 3.2 describes the quantification of these radionuclides other than noble gases.

A person may be exposed directly to an airborne concentration of radioactive material discharged in effluent and indirectly via pathways involving deposition of radioactive material onto the ground. Dose estimates account for the separate exposure pathways. The dose commitment to a person offsite associated with a gaseous release, Q_{ik}, of radioactive material other than noble gas is calculated with the following equation for a vent release:

10-2021

$$D_{anvke} = \sum_i Q_{ikv} \left[TA_{anie} \left(\frac{\chi}{Q}\right)_v + TG_{anie} \left(\frac{D}{Q}\right)_v \right]$$

where

10-2021

D_{anvke} = the dose commitment via pathway *e* from a vent release (mrem)

TA_{anie} = factor converting airborne concentration of radionuclide *i* to dose commitment to organ *n* of a person in any group *a* where exposure is directly to airborne material via exposure pathway *e*.

$$\left(\frac{mrem}{(Ci\ \sec)/m^3}\right)$$

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[j]The dose to any organ of a person arising from radioactive iodine-131, iodine-133, tritium, and radioactive material in particulate form having half-lives greater than 8 days. Noble gases not considered.

TG_{anie} = factor converting ground deposition of radionuclide *i* to dose commitment organ *n* of a person in age group *a* where exposure is directly or indirectly to radioactive material that has been deposited on the ground via exposure pathway *e*. $\left(\frac{mrem}{Ci/m^2}\right)$

Q_{ik} = quantity of radionuclide *i* released in a given effluent stream based on analysis *k* (Ci)

10-2021 $\left(\frac{D}{Q}\right)_v$ = relative deposition factor, i.e., factor converting airborne effluent discharge

10-2021 from vent respectively, to a real deposition on land (m^{-2}).

The analysis index *k* may represent either an analysis of a grab sample, a weekly composite analysis, a monthly composite analysis, or a quarterly composite analysis.

Since tritium in water vapor is absorbed directly by vegetation, the tritium concentration in growing vegetation is proportional to the airborne concentration rather than to relative deposition as in the case of particulates. Thus the dose commitment from airborne tritium via vegetation (fruit and vegetables), air-grass-cow-milk, or air-grass-cow-meat

10-2021 pathways is calculated with the equation for a vent release:

$$D_{anve} = \left(\frac{\chi}{Q}\right)_v \sum_i \sum_k Q_{ikv} \; TA_{anie}$$

The dose commitment accumulated by a person offsite is computed at least every 30 days to satisfy Section 7.2.4.1 but may be calculated as analytical results of effluent

10-2021 measurements, performed as specified in Table 7.2-2, become available.

10-2021 The dose accumulated as a result of vent discharge is computed with

$$D_{anv} = \sum_e D_{anve}$$

10-2021

When the dose to a person from iodine and particulates discharged in gaseous effluents is calculated as required by Section 7.2.4, appropriate environmental pathways (from among those for which dose transfer factors are provided in Appendix A) will be evaluated. The dose calculated is to a receptor at the location of the nearby residence experiencing the minimum atmospheric dispersion at ground-level from the station, i.e., maximum $\frac{X}{Q}$, concurrent with the effluent discharge. Alternatively, the dose may be calculated to a receptor at the location identified in Figure 3-2 where reference atmospheric dispersion and deposition factors are:

10-2021

$$\left(\frac{\chi}{Q}\right)_v = 3.9 \bullet 10^{-6} \sec / m^3 \quad \left(\frac{D}{Q}\right)_v = 1.3 \bullet 10^{-8} m^{-2}$$

Food pathways are evaluated at the location of food production based on minimum atmospheric dispersion at ground-level concurrent with the effluent discharge or, alternatively, with reference meteorology applicable at that location. Seasonal appropriateness of pathways is considered. The air-grass (fresh or stored)-cow-milk-man pathway is evaluated** where a cow is located, 2650 meters WNW of DAEC, reference atmospheric deposition factors are:

10-2021

$$\left(\frac{D}{Q}\right)_v = 4.28 \bullet 10^{-9} m^{-2}$$

**This receptor is historical in nature. See the current land use census to verify this point as the most conservative receptor location for the pathway.

10-2021 3.8.1 Not Used

3.9 Dose to a Person from Noble Gases

Section 7.3.1 requires the calculation of the dose or dose commitment to a person offsite exposed to 12 consecutive months of radioactive liquid and gaseous effluents from the Station. One component of personal dose is total body irradiation by gamma rays from noble gases. Another is irradiation of skin by beta and gamma radiation from noble gases. The methods of calculating these doses are presented in sections 3.9.1 and 3.9.2.

The amount of radioactive noble gas discharged is determined in the manner described in section 3.2.

3.9.1 Gamma Dose to Total Body

The gamma radiation dose to the whole body of a member of the public as a consequence of noble gas released from the Station is calculated with the equation:

10-2021

$$D_{\gamma_i} = \sum_v \sum_i \left(Q_i \bullet \left(\frac{\chi}{Q}\right) \bullet P_{\gamma_i^v}\right)_v$$

where:

D_γ = noble gas gamma dose to total body (mrem)

10-2021 Q_i = quantity of noble gas nuclide *i* released via vent (Ci)

10-2021

$P\gamma_i^v$ = factor converting time integrated, ground level concentration of noble gas nuclide *i* to total body dose from gamma radiation

$$\left(\frac{mrem}{Ci\ \sec/m^3}\right)$$

10-2021

The dose from noble gases released from a vent (near ground level) or to skin from an elevated release is derived from a semi-infinite cloud model. Noble gas semi-infinite cloud gamma-to-total body dose factors, $P\gamma_i^v$, are listed in Appendix A under the plume pathway.

When the total body and organ doses from noble gases are computed as required by Section 7.3.1, the nearby resident exposed to the maximal ground-level noble gas concentration (minimum χ/Q) is selected as the receptor.

Alternatively, the dose from noble gases may be computed at 1260 meters NNW of the reactor where the nearby resident who may be exposed to maximal ground-level noble gas concentrations (minimum χ/Q) is selected as the receptor. In that case, values of

10-2021 the dose factor $P\gamma_i^v$ in Table 3-5 is employed.[l] Reference meteorological dispersion for vent discharges at that location, $(\chi/Q)_v$, is $4.3 \times 10^{-6}\ \sec/m^3$.

3.9.2 Dose to Skin

The beta radiation dose to the skin of a member of the public due to beta radiation from noble gas released from the Station may be calculated with the equation

10-2021

$$D_\beta = \sum_v \sum_i S_{\beta_i} \bullet \left[Q_i \bullet \left(\frac{\chi}{Q}\right)\right]_v$$

where

D_β = noble as dose to skin (mrem)

$S_{\beta i}$ = factor converting time integrated ground level concentration of noble gas to skin dose from beta radiation

$$\left(\frac{mrem}{Ci\ \sec/m^3}\right)$$

[k]David Slade, ed., Meteorology and Atomic Energy 1968, TID-24190, pp. 350-355.

Semi-infinite cloud noble gas beta-to-skin dose factors, $S_{\beta i}$ appear in Table 3-5.

The total dose to the skin from noble gases is approximately equal to the beta radiation dose to the skin plus the gamma radiation dose to the total body.

When the skin dose due to noble gas beta radiation is computed as required by Section 7.3.1, the receptor selected is the nearby resident exposed to maximal ground-level concentrations (maximum χ / Q).

Alternatively, the skin dose to a postulated receptor (resident) at 1260 meters NNW of the reactor may be calculated.

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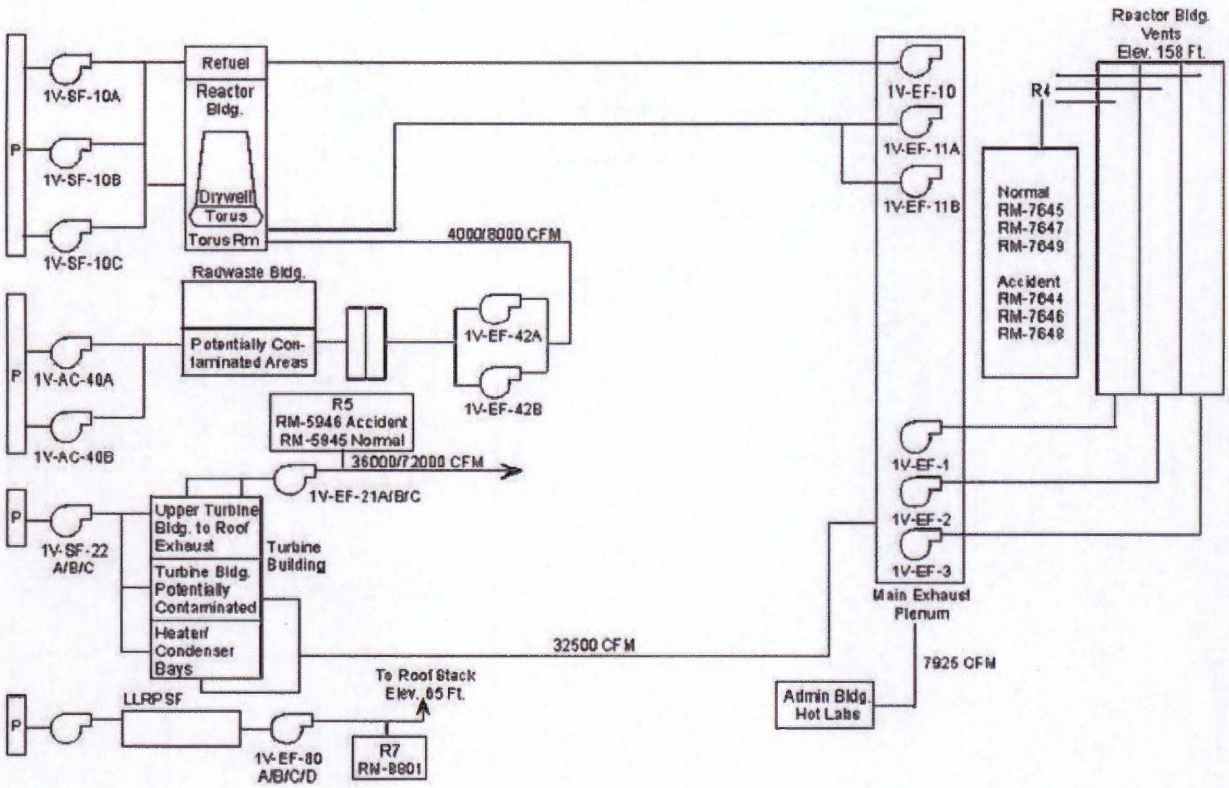
[1]Dose transfer factors $P\gamma_i^v$ in Table 3.5 and in Appendix A under the plume pathway are the same.

Figure 3.1
Gaseous Radioactive Waste Flow Diagram



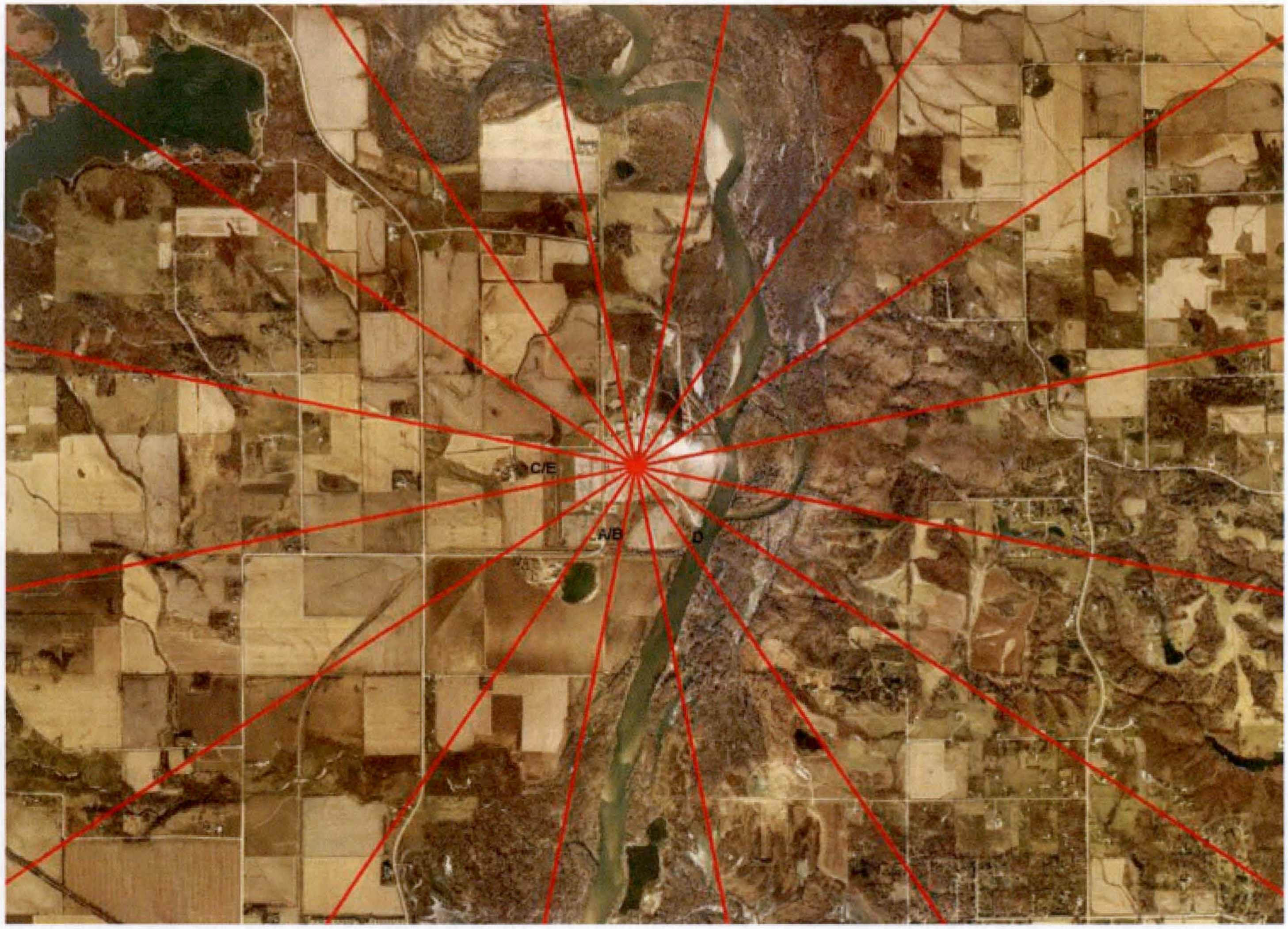
10-2021

R4 Reactor Building Exhaust Vent Monitoring System

R5 Turbine Building Exhaust Vent Monitoring System

R7 LLRPSF Exhaust Vent Monitoring System

Figure 3-2
Dose Calculation Sites



Site	Description	Distance from Plant
A	Noble gas gamma air dose:	481 meters South-Southeast
B	Noble gas beta air dose:	481 meters South-Southeast
C	Iodines and Particulates:	805 meters West
D	Aquatic pathways:	Cedar River
E	Most Exposed resident:	805 meters West

10-2021

Table 3-1

Atmospheric Gaseous Release Points
at the Duane Arnold Energy Center

Parameter	**RELEASE POINT**		
	Reactor Building Vent	Turbine Building Vent	LLRPSF Building Vent
Release Height	156 feet	90 feet	65 feet
Release Mode	Wake-split	Wake-split	Wake-split
Effluent Source	Reactor Building Radwaste Building Lower Turbine Building	Upper Turbine Building	LLRPSF Building and Storage Facility

10-2021

Table 3-2
Computed Releases of Radioactive Noble Gases in Gaseous Effluent from Duane Arnold Energy Center

	PLANT VENTS RELEASE	
Nuclide	(Ci/yr)[a]	Fraction[b]
Kr-83m	0	0
Kr-85m	7.40E+01	1.98E-02
Kr-85	0	0
Kr-87	1.36E+02	3.64E-02
Kr-88	2.36E+02	6.32E-02
Kr-89	0	0
Xe-131m	0	0
Xe-133m	0	0
Xe-133	3.92E+02	1.05E-01
Xe-135m	7.42E+02	1.99E-01
Xe-135	7.43E+02	1.99E-01
Xe-137	0	0
Xe-138	1.41E+03	3.78E-01
	3733.	1.0

TABLE NOTATION

[a] Releases computed by BWR-GALE for DAEC Base Case gaseous radwaste treatment. Computed releases are included only to show the basis of the radionuclide distribution.

[b] This is the calculated distribution of radionuclides in gaseous effluents in each release pathway. To estimate radionuclide concentrations in a sample in which only the total activity concentration has been measured, multiply the total activity concentration by the fraction of respective radionuclides listed above.

Table 3-3
Transfer Factors for Maximum Offsite Air Dose

10-2021

Radionuclide	Air Dose Transfer Factors	
	$A\gamma_i^v$ [b] $\left(\frac{mrad}{\mu Ci\ \sec/m^3}\right)$	A_{β_i} [b] $\left(\frac{mrad}{\mu Ci\ \sec/m^3}\right)$
Kr-83m	6.1E-7	9.1E-6
Kr-85m	3.9E-5	6.2E-5
Kr-85	5.4E-7	6.2E-5
Kr-87	2.0E-4	3.3E-4
Kr-88	4.8E-4	9.3E-5
Kr-89	5.5E-4	3.4E-4
Kr-90	5.2E-4	2.5E-4
Xe-131m	4.9E-6	3.5E-5
Xe-133m	1.0E-5	4.7E-5
Xe-133	1.1E-5	3.3E-5
Xe-135m	1.1E-4	2.3E-5
Xe-135	6.1E-5	7.8E-5
Xe-137	4.8E-5	4.0E-4
Xe-138	2.9E-4	1.5E-4
Ar-41	2.9E-4	1.0E-4

TABLE NOTATIONS

b Dose transfer factors for vent discharges, $A\gamma_i^v$ and $A_{\beta i}$ are equivalent to factors in Regulatory Guide 1.109, Table B-1, g-air and b-air, expressed in different units.

10-2021

Table 3-4
Transfer Factors for Dose Equivalent Rate to A Person Offsite Due to Radioactive Noble Gases

Radionuclide	Dose Transfer Factors	
	DF_i^{γ} [d] $\left[\frac{mrem}{(\mu Ci\ yr)/m^3}\right]$	$S_{\beta i}$ $\left[\frac{mrem}{(\mu Ci\ yr)/m^3}\right]$
Kr-83m	7.56E-2	0
Kr-85m	1.17E+3	1.46E+3
Kr-85	1.61E+1	9.73E+3
Kr-87	5.92E+3	9.73E+3
Kr-88	1.47E+4	2.37E+3
Kr-89	1.66E+4	1.01E+4
Kr-90	1.56E+4	7.29E+3
Xe-131m	9.15E+1	4.76E+2
Xe-133m	2.51E+2	9.94E+2
Xe-133	2.94E+2	3.06E+2
Xe-135m	3.12E+3	7.11E+2
Xe-135	1.81E+3	1.86E+3
Xe-137	1.42E+3	1.22E+4
Xe-138	8.83E+3	4.13E+3
Ar-41	8.84E+3	2.69E+3

TABLE NOTATIONS

[d] Factors DF_i^{γ} from Regulatory Guide 1.109, revision 1, Table B-1, column 5

10-2021

Table 3-5
Transfer Factors for Dose Equivalent Rate to A Person Offsite Due to Radioactive Noble Gases

Radionuclide	Dose Transfer Factors	
	$P\gamma_i^v$ [c, d] $\left[\frac{mrem}{(Ci\ sec)/m^3}\right]$	$S_{\beta i}$ [d] $\left[\frac{mrem}{(Ci\ sec)/m^3}\right]$
Kr-83m	1.68E-3	---
Kr-85m	2.60E+1	4.63E+1
Kr-85	3.58E-1	4.25E+1
Kr-87	1.31E+2	3.08E+2
Kr-88	3.26E+2	7.51E+1
Kr-89	3.68E+2	3.20E+2
Kr-90	3.46E+2	2.31E+2
Xe-131m	2.03	1.51E+1
Xe-133m	5.57	3.15E+1
Xe-133	6.52	9.70
Xe-135m	6.92E+1	2.25E+1
Xe-135	4.01E+1	5.89E+1
Xe-137	3.15E+1	3.87E+2
Xe-138	1.96E+2	1.31E+2
Ar-41	1.96E+2	8.52E+1

TABLE NOTATIONS

[c] Factors $P\gamma_i^v$ already account for a 0.7 residential shielding factor.

[d] Factors $P\gamma_i^v$ and $S_{\beta i}$ from Regulatory Guide 1.109, revision 1, Table B-1

4.0 DOSE COMMITMENT FROM RELEASE OVER EXTENDED TIME

4.1 Dose Assessment for 10 CFR Part 50, Appendix I

Sections 7.1.3, 7.2.3, and 7.2.4 require quarterly and annual assessments to demonstrate compliance with Appendix I dose limits. The assessment includes the following calculations of dose as described by equations for:

1. total body and maximally exposed organ doses due to liquid effluent via drinking water and eating fish from the River and from consuming food irrigated with river water as in paragraph 2.5.

2. total body and maximally exposed organ doses due to gaseous effluents[a] other than noble gases as in paragraph 3.8.

3. doses to air offsite due to noble gas g as in paragraph 3.6 and due to noble gas β as in paragraph 3.7.

The dose calculations are based on liquid and gaseous effluents from the Station during each calendar quarter and for a calendar year, determined in accord with Tables 7.1-2 and 7.2-2.

Environmental concentrations depend on dispersion and dilution of the effluent. For aqueous effluents over extended time, the aquatic concentration is estimated according to section 2.2. Atmospheric dispersion and deposition factors used to estimate the dose commitment due to gaseous effluents are ordinarily derived from reference meteorological data. Otherwise, quarterly averaged or annual averaged meteorological conditions concurrent with the gaseous release being evaluated will be used to estimate atmospheric dispersion and deposition.

The receptor of the dose is described such that the dose to any resident near the Station is unlikely to be underestimated. That is, the receptor is selected on the basis of the combination of applicable pathways of exposure to gaseous effluent identified in the annual land use census and maximum ground level χ / Q at the residence. Conditions (i.e., location, χ / Q, and/or pathways) more conservative (i.e., expected to yield higher calculated doses) than appropriate for the maximally exposed individual may be assumed in the dose assessment.

---

[a] Radioactive iodine-131, iodine-133, tritium, and radioactive material in particulate form having half-lives greater than 8 days.

Seasonal appropriateness of exposure pathways may be considered. Exposure by eating fresh vegetation or drinking milk from cows or goats fed fresh forage is an inappropriate assumption during the first or fourth calendar quarter; rather consumption of stored vegetation and stored forage is assumed during those quarters. Otherwise, during the second and third calendar quarters, exposure by eating fresh vegetation and/or drinking milk from cows or goats fed fresh forage is assumed where those pathways exist. Similarly, the liquid effluent-river-fish-man pathway is not assumed during the winter quarter.

Factors converting stack-released noble gas to gamma radiation dose from the overhead plume are calculated on the basis of reference meteorological data for the receptor location or alternatively, by the MIDAS program for a residential location offsite where maximum χ / Q at ground level occurs.

Other environmental pathway-to-dose transfer factors used in the dose calculations are provided in Appendix A.

4.2 Dose Assessment for 40 CFR Part 190 and 10 CFR 72.104

The regulations governing the maximum allowable dose or dose commitment to a member of the public from all uranium fuel cycle-sources of radiation and radioactive material in the environment is stated in 40 CFR Part 190 and 10 CFR 72.104. It requires that the dose or dose commitment to a member of the public from all sources not exceed 25 mrem/yr to the whole body or 25 mrem/yr to any organ or 75 mrem/yr to the thyroid. Section 7.3.1 requires calculation of the dose at least once every year to assess compliance with the regulation. More frequent calculations may be performed if higher than normal releases are experienced (twice the design objective rates in a single quarter).

Fuel cycle sources or nuclear power reactors other than the Station[b] itself do not measurably or significantly increase the radioactivity concentration in the vicinity of the Station; therefore, only radiation and radioactivity in the environment attributable to the Station itself are considered in the assessment of compliance with 40 CFR Part 190 and 10 CFR 72.104.

Contributions to the dose due to liquid and gaseous effluent are calculated as described by the equations for:

10-2021

1. total body and maximally exposed organ doses due to liquid effluent via drinking water and from eating fish from the River as in paragraph 2.5

2. total body dose due to noble gas γ as in paragraph 3.9.1

3. skin dose due to noble gas β as in paragraph 3.9.2

[b] The Station is defined as BOTH the Nuclear Reactor Facility and the Independent Spent Fuel Storage Installation (ISFSI).

4. total body and maximally exposed organ doses due to gaseous effluents[c] other than noble gases as in Paragraph 3.8.

Additionally, the contribution to total dose from direct radiation is assessed annually by using environmental TLDs.

The doses are calculated on the basis of liquid and gaseous effluents from the Station during 12 consecutive months, determined in accord with Tables 7.1-2 and 7.2-2. For the purpose of the Annual Radiological Environmental Report, doses are based upon release during a calendar year.

Aqueous radioactive material concentrations are estimated according to paragraph 2.2 on the basis of annual averaged stream flow. Annual averaged meteorological conditions concurrent with gaseous releases being evaluated are used to estimate atmospheric dispersion, deposition, and elevated plume gamma exposure.

The receptor of the dose is described such that the dose to any resident near the Station is not likely to be underestimated, although conditions more conservative than appropriate for the maximally exposed person may be assumed in the dose assessment.

Ordinarily, the receptor is selected on the basis of the applicable combination of existing pathways of exposure to gaseous effluent identified in the annual land use census and the maximum ground level χ / Q at the residence.

When assessing compliance with 40 CFR 190, Radiological Environmental Monitoring Program results may be used to indicate actual radioactivity levels in the environment attributable to the DAEC. These measured levels may be used to supplement the evaluation of doses to members of the public for assessing compliance with 40 CFR 190.

Factors converting stack-released noble gas to gamma radiation dose from the overhead plume are calculated on the basis of annual averaged meteorological data for the receptor location. Other environmental pathway-to-dose transfer factors are listed in Appendices A, B and C.

[c] Radioactive iodine-131, iodine-133, tritium, and radioactive material in particulate form having half-lives greater than 8 days.

5.0 ENVIRONMENTAL MONITORING PROGRAMS

Section 5.0 of the DODAM provides a description of the Radiological Environmental Monitoring Program (REMP) and the Ground Water Protection Program (GWPP). This section also contains descriptions of the Environmental Sampling Program Station Locations.

5.1 Radiological Environmental Monitoring Program

A program shall be provided to monitor the radiation and radionuclides in the environs of the station.

The program shall provide:

(1) representative measurements of radioactivity in the highest potential exposure pathways,

(2) verification of the accuracy of the effluent monitoring program and modeling of environmental exposure pathways.

The program shall:

(1) be contained in the DODAM,

(2) conform to the guidance of Appendix I to 10 CFR Part 50 and 10 CFR 72,

(3) include the following:

(a) Monitoring, sampling, analysis, and reporting of radiation and radionuclides in the environment in accordance with the methodology and parameters in the DODAM.

(b) A Land Use Census to ensure that changes in the use of areas at and beyond the SITE BOUNDARY are identified and that modifications to the monitoring program are made if required by the results of this census.

(c) Participation in an Interlaboratory Comparison Program to ensure that independent checks on the precision and accuracy of the measurements of radioactive materials in environmental sample matrices are performed as part of the quality assurance program for environmental monitoring.

5.2 Ground Water Protection Program (GWPP)

A program shall be provided to prevent, detect and respond to inadvertent and radiological releases with the potential to reach ground water.

The program shall provide:

(1) for the prevention of inadvertent/unplanned radiological releases from plant systems, structures and components (SSCs) or during plant evolutions or work practices that represent an elevated risk of experiencing a release of licensed radiological material into the environment.

(2) aspects for monitoring, detecting and responding to unplanned/unmonitored releases of licensed radioactive material to the environment and a communications/notification plan that addresses internal notifications to management and communications/reporting to State and local stakeholders and regulators for specified events or conditions.

The program shall:

(1) be contained in the site administrative control procedure ACP 1411.35, "The DAEC Groundwater Protection Program".

(2) implement the Ground Water Protection Initiative Final Guidance Document, NEI-07-07[Final].

5.3 Sampling Station Locations

DODAM Table 5-1 "ENVIRONMENTAL SAMPLE STATIONS" is a list of locations where samples may be collected and does not represent a list of required samples. Environmental monitoring locations are shown on Figures 5-1 and 5-2. DAEC may conduct additional environmental monitoring exclusive of the requirements of Specifications 6.3.2 and ACP 1411.35.

10-2021 Sampling and analyzing for I-131 will be discontinued as I-131 has decayed since shutdown.

Figure 5-1
Environmental Monitoring Programs
Sampling Near the Duane Arnold Energy Center

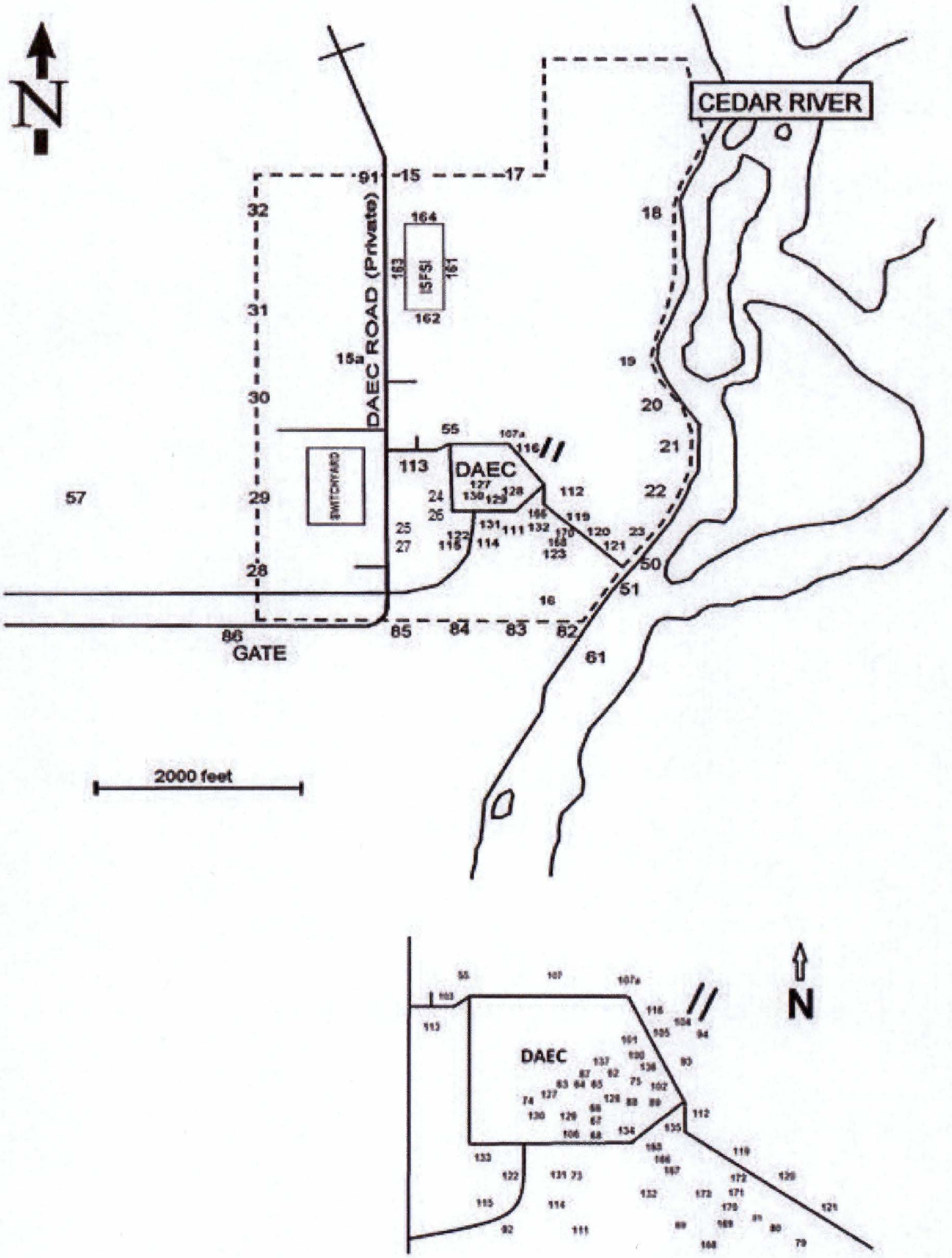


Figure 5-2
Radiological Environmental Monitoring Program
Sampling Stations Outside 0.5 Miles from DAEC

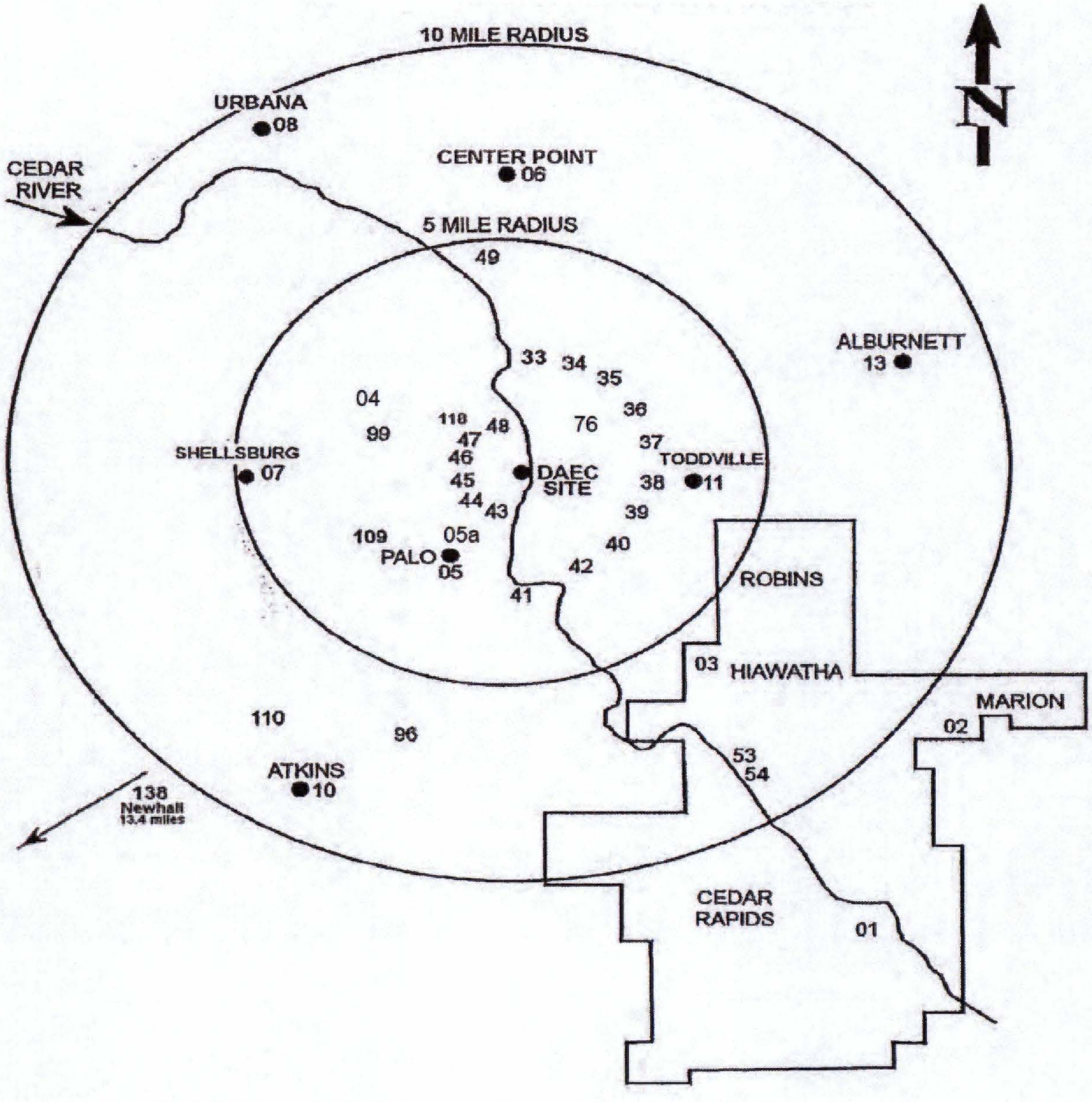


Table 5-1
ENVIRONMENTAL SAMPLE STATIONS

Station Number	Program	Station Location and Sample Type	
1	REMP	Location: Type:	Cedar Rapids, 20,800 meters SE TLD Control - discontinued Airborne Particulate and Iodine Control – Not Currently Active
2	REMP	Location: Type:	Marion, 16,900 meters ESE TLD Control - discontinued Airborne Particulate and Iodine Control – Not Currently Active
3	REMP	Location: Type:	Hiawatha, 10,800 meters SE TLD Control - discontinued Airborne Particulate and Iodine – Not Currently Active
4	REMP	Location: Type:	Pleasant Creek SRA, 4960 meters NW TLD Control - discontinued Airborne Particulate and Iodine – Not Currently Active
5	REMP	Location: Type:	Palo, 4,500 meters SSW TLD Control - discontinued
5a	REMP	Location: Type:	Palo, 3,470 meters SSW TLD Control - discontinued Airborne Particulate and Iodine – Not Currently Active
6	REMP	Location: Type:	Center Point, 9,660 meters N TLD Control - discontinued Airborne Particulate and Iodine – Not Currently Active
7	REMP	Location: Type:	Shellsburg, 7,950 meters W TLD Control - discontinued Airborne Particulate and Iodine – Not Currently Active
8	REMP	Location: Type:	Urbana, 15,000 meters NNW TLD Control - discontinued Airborne Particulate and Iodine Control – Not Currently Active
9		Not Used	
10	REMP	Location: Type:	Atkins, 13,600 meters SSW TLD Control - discontinued Airborne Particulate and Iodine Control – Not Currently Active
11	REMP	Location: Type:	Toddville, 4,980 meters E TLD Control - discontinued Airborne Particulate and Iodine – Not Currently Active
12		Not Used	
13	REMP	Location: Type:	Alburnett, 14,500 meters ENE TLD Control - discontinued Airborne Particulate and Iodine Control - discontinued
14		Not Used	
15	REMP	Location: Type:	On-site, 1,050 meters NNW TLD Airborne Particulate and Iodine
15a	GWPP	Location: Type:	On-site, 505 meters NNW Soil – discontinued
16	REMP	Location: Type:	On-site, 520 meters SSE TLD - discontinued Airborne Particulate and Iodine Vegetation - discontinued Meteorology Tower
16a	GWPP	Location: Type:	On-site, 520 meters SSE Soil – discontinued Precipitation – discontinued
17	REMP	Location: Type:	On-site, 1,050 meters N TLD

Table 5-1
ENVIRONMENTAL SAMPLE STATIONS

	Station Number	Program	Station Location and Sample Type	
	18	REMP	Location: Type:	630 meters NNE TLD
10-2021	19	REMP	Location: Type:	On-site, 590 meters NE TLD – discontinued
	20	REMP	Location: Type:	On-site, 550 meters ENE TLD
10-2021	21	REMP	Location: Type:	On-site, 515 meters ENE TLD – discontinued
	22	REMP	Location: Type:	On-site, 535 meters ESE TLD
10-2021	23	REMP	Location: Type:	On-site, 490 meters SE TLD – discontinued
10-2021	24	GWPP	Location: Type:	MW104 electrical vault W of lower level RadWaste Building inside Protected Area Ground Water – discontinued
10-2021	25	GWPP	Location: Type:	MW105 electrical vault N of contractor parking lot along DAEC Road Ground Water – discontinued
10-2021	26	GWPP	Location: Type:	MW106 electrical vault W of Lower Level RadWaste Building inside Protected Area Ground Water – discontinued
10-2021	27	GWPP	Location: Type:	MW107 electrical vault N of contractor parking lot along DAEC Road Ground Water – discontinued
10-2021	28	REMP	Location: Type:	On-site, 730 meters WSW TLD – discontinued
	29	REMP	Location: Type:	On-site, 630 meters W TLD
10-2021	30	REMP	Location: Type :	On-site, 640 meters WNW TLD – discontinued
	31	REMP	Location: Type:	On-site, 1,020 meters NW TLD
10-2021	32	REMP	Location: Type:	On-site, 1,110 meters NNW TLD – discontinued
	33	REMP	Location: Type:	4,340 meters N TLD
10-2021	34	REMP	Location: Type:	3,930 meters NNE TLD – discontinued
	35	REMP	Location: Type:	2,800 meters NE TLD
10-2021	36	REMP	Location: Type:	3,500 meters ENE TLD- discontinued
	37	REMP	Location: Type:	2,960 meters E TLD
10-2021	38	REMP	Location: Type:	3,180 meters ESE TLD – discontinued
	39	REMP	Location: Type:	2,510 meters SE TLD
10-2021	40	REMP	Location: Type:	Wickiup Hill Learning Center 2,430 meters SSE TLD – discontinued Airborne Particulate and Iodine – Not Currently Active
10-2021	41	REMP	Location: Type:	5,680 meters S TLD – discontinued

Table 5-1
ENVIRONMENTAL SAMPLE STATIONS

	Station Number	Program	Station Location and Sample Type	
	42	REMP	Location: Type:	4,380 meters SSE TLD
	43	REMP	Location: Type:	1,590 meters SSW TLD
10-2021	44	REMP	Location: Type:	1,580 meters WSW TLD – discontinued
10-2021	45	REMP	Location: Type:	1,420 meters W TLD – discontinued
	46	REMP	Location: Type:	1,580 meters WNW TLD
10-2021	47	REMP	Location: Type:	1,760 meters NW TLD – discontinued
	48	REMP	Location: Type:	1,680 meters NNW TLD
	49	REMP	Location: Type:	Lewis Access, upstream of DAEC 6,750 meters NNW Fish Control Surface Water Control Bottom Sediment Control
	50	REMP	Location: Type:	Plant Intake, 560 meters SE Bottom Sediment Control – discontinued Surface Water – discontinued
10-2021	51	REMP	Location: Type:	Plant Discharge, 600 meters SE Bottom Sediment Surface Water – discontinued
	52	REMP	Location: Type:	Plant potable water supply Drinking Water
	53	REMP	Location: Type:	Treated Municipal Water, 13,900 meters SE Drinking Water
10-2021	54	REMP	Location: Type:	Inlet to Municipal Water Treatment System, 13,900 meters SE Drinking Water – discontinued
10-2021	55	REMP	Location: Type:	On-site, Production Wells Ground Water - discontinued
10-2021	56	REMP GWPP	Location: Type:	Generic control sample for sampling quality assurance Water, soil, vegetation, milk, sediment, fish, special samples - discontinued
10-2021	57	REMP	Location: Type:	Farm, 805 meters W Drinking Water – discontinued Vegetation
10-2021	58	REMP	Location: Type:	Farm, 974 meters WSW-SW Drinking Water – discontinued Vegetation Meat sample
	59	REMP	Location: Type:	Farm, 2,615 meters SE Vegetation
	60		Not Used	
	61	REMP	Location: Type:	Cedar River, 670 meters SSE Fish Surface Water
	62	GWPP	Location: Type:	Onsite Monitoring Well MW-18A near SE corner of the turbine building Ground Water
	63	GWPP	Location: Type:	Onsite Monitoring Well MW-19A near S wall of the turbine building Ground Water – discontinued (well sealed)

Table 5-1
ENVIRONMENTAL SAMPLE STATIONS

	Station Number	Program	Station Location and Sample Type	
	64	GWPP	Location: Type:	Onsite Monitoring Well MW-20A near S wall of the turbine building Ground Water
	65	GWPP	Location: Type:	Onsite Monitoring Well MW-21A near S wall of the turbine building Ground Water
	66	GWPP	Location: Type:	Onsite Monitoring Well MW-22A S turbine building wall and W of MW-08A Ground Water
	67	GWPP	Location: Type:	Onsite Monitoring Well MW-23A S of MW-22A and NW of MW-14A Ground Water
	68	GWPP	Location: Type:	Extraction Well EW-01A SE of MW-23A and W of MW-14A Ground Water
	69	GWPP	Location: Type:	Extraction Well EW-02A SSE of MW-12A in the Owner Controlled Area Ground Water
	70	GWPP	Location: Type:	Extraction Well EW-03A SSE between MW-24A and MW-26A. Formerly D-166. Ground water
	71		Not Used	
10-2021	72	REMP	Location: Type:	Farm, 3,200 meters SSW Drinking Water – discontinued
10-2021	73	GWPP	Location: Type:	2MH216/1MH117 electrical vault, West of the Offgas Stack Surface Water – discontinued
10-2021	74	GWPP	Location: Type:	2MH215/1MH116 electrical vault, E. of RB RR, W. of Condensate Storage Tanks Surface Water – discontinued
10-2021	75	GWPP	Location: Type:	2MH207 electrical vault, East of South End of Pump House Surface Water – discontinued
	76	REMP	Location: Type:	Farm, 2,888 meters East Northeast Milk – discontinued
	77	REMP	Location: Type:	Farm on Palo Marsh Road, 2288 meters SW Vegetation – discontinued
	78		Not Used	
	79	GWPP	Location: Type:	On-site monitoring well MW-33A SSE in Owner Controlled Area Ground Water
	80	GWPP	Location: Type:	On-site monitoring well MW-34A SSE in Owner Controlled Area Ground Water
	81	GWPP	Location: Type:	On-site monitoring well MW-35A SSE in Owner Controlled Area Ground Water Precipitation – discontinued
10-2021	82	REMP	Location: Type:	On-site, 660 meters SSE TLD – discontinued
	83	REMP	Location: Type:	On-site, 620 meters SSE TLD
10-2021	84	REMP	Location: Type:	On-site, 610 meters S TLD – discontinued
	85	REMP	Location: Type:	On-site, 660 meters SSW TLD
10-2021	86	REMP	Location: Type:	On-site, 850 meters SW TLD – discontinued
10-2021	87	GWPP	Location: Type:	MH218 electrical vault, East of the Turbine Building Surface Water - discontinued
10-2021	88	GWPP	Location: Type:	MH219 electrical vault, North of Air Compressor Building Surface Water – discontinued

Table 5-1
ENVIRONMENTAL SAMPLE STATIONS

	Station Number	Program	Station Location and Sample Type	
10-2021	89	GWPP	Location: Type:	MH221 electrical vault, Between Air Compressor Building and H2 Skid Surface Water – discontinued
10-2021	90	GWPP	Location: Type:	MH222 electrical vault, North of "B" Well Surface Water – discontinued
10-2021	91	REMP	Location: Type:	On-site, 1,090 meters NNW TLD – discontinued
10-2021	92	GWPP	Location: Type:	MH202 electrical vault, West of XR1/XR2 Transformers Surface Water – discontinued
10-2021	93	GWPP	Location: Type:	MH213 electrical vault, East of the Pump House Surface Water – discontinued
10-2021	94	GWPP	Location: Type:	MH214 electrical vault, South of "B" Cooling Tower Surface Water – discontinued
10-2021	95	GWPP	Location: Type:	MH217 electrical vault, Southwest of Intake Building Surface Water – discontinued
	96	REMP	Location: Type:	Farm, 11,400 meters SSW Vegetation – discontinued
10-2021	97	GWPP	Location: Type:	MH102 electrical vault, "A" Well Road North of the Substation Surface water – discontinued
10-2021	98	GWPP	Location: Type:	MH103 electrical vault, "A" Well Road West of the Substation Surface Water – discontinued
	99	REMP	Location: Type:	Pleasant Creek Lake, 3,880 meters WNW Surface Water and deer meat sample – discontinued
10-2021	100	GWPP	Location: Type:	MH206 electrical vault, West of Pump House Surface Water – discontinued
10-2021	101	GWPP	Location: Type:	1MH109 electrical vault, East of the North End of the Pump House Surface Water – discontinued
10-2021	102	GWPP	Location: Type:	2MH208/1MH110 electrical vault, Road to Intake Building Surface Water – discontinued
10-2021	103	GWPP	Location: Type:	MH101 electrical vault, North of PAB, East of Security Fence Surface Water – discontinued
10-2021	104	GWPP	Location: Type:	MH115 electrical vault, South of "A" Cooling Tower Surface Water – discontinued
10-2021	105	GWPP	Location: Type:	MH114 electrical vault, South of "A" Cooling Tower Surface Water – discontinued
10-2021	106	GWPP	Location: Type:	MH201 electrical vault, South of Turbine Building Surface Water – discontinued
	107	GWPP	Location: Type:	DAEC Sewage Plant Effluent Surface Water
	107 a	REMP	Location: Type:	On-Site: North Drainage Ditch Bottom Sediments – discontinued
	108	REMP	Location: Type:	4,590 meters SSW Vegetation – discontinued
	109	REMP	Location: Type:	5,890 meters SW Vegetation – discontinued
	110	REMP	Location: Type:	12,700 meters SW Milk – discontinued
	111	GWPP	Location: Type:	Onsite Monitoring Wells MW01A and MW01B (SSE) Ground Water – discontinued Precipitation – discontinued

Table 5-1
ENVIRONMENTAL SAMPLE STATIONS

	Station Number	Program	Station Location and Sample Type
	112	GWPP	Location: Onsite Monitoring Wells MW02A and MW02B (ESE) Type: Ground Water – discontinued Precipitation – discontinued
	113	GWPP	Location: Onsite Monitoring Wells MW03A and MW03B (ESE) Type: Ground Water – discontinued
	114	GWPP	Location: Onsite Monitoring Wells MW04A and MW04B (South) Type: Ground Water – discontinued Precipitation - discontinued
	115	GWPP	Location: Onsite Monitoring Wells MW05A and MW05B (SSW) Type: Ground Water – discontinued Precipitation – discontinued
	116	GWPP	Location: Onsite Monitoring Wells MW06A and MW06B (NE) Type: Ground Water – discontinued Precipitation - discontinued
	117		Not Used
	118	REMP	Location: 2,230 meters NW Type: Vegetation – discontinued
10-2021	119	GWPP	Location: 2MH209/1MH111 electrical vault, Road to Intake Building Type: Surface Water – discontinued
10-2021	120	GWPP	Location: 2MH210/1MH112 electrical vault, Road to Intake Building Type: Surface Water – discontinued
10-2021	121	GWPP	Location: 2MH211/1MH113 electrical vault, Road to Intake Building Type: Surface Water – discontinued
	122	GWPP	Location: Onsite Sluice Pond, in OCA Type: Surface Water
10-2021	123	GWPP	Location: Onsite South Drainage Ditch Type: Surface Water – discontinued
10-2021	124	GWPP	Location: Onsite North Drainage Ditch Type: Surface Water – discontinued
10-2021	125	GWPP	Location: Onsite South Storm Drain Outfall Type: Surface Water – discontinued
	127	GWPP	Location: Protected Area, North of CST Pit, Monitoring Wells MW07A and MW07B Type: Ground Water Precipitation – discontinued
	128	GWPP	Location: Protected Area, CAD Shack, Monitoring Wells MW08A and MW08B Type: Ground Water Precipitation – discontinued
	129	GWPP	Location: Protected Area, SE Corner of CST Pit, Monitoring Wells MW09A and MW09B Type: Ground Water
	130	GWPP	Location: Protected Area, SW Corner of CST Pit, Monitoring Wells MW10A and MW10B Type: Ground Water – discontinued
	131	GWPP	Location: On-site Monitoring Wells MW11A and 11B SE in Owner Controlled Area Type: Ground Water
	132	GWPP	Location: On-site Monitoring Wells MW12A and MW12B SE in Owner Controlled Area Type: Ground Water
	133	GWPP	Location: On-site Monitoring Wells MW13A and MW13B SW in Owner Controlled Area Type: Ground Water – discontinued
	134	GWPP	Location: On-site Monitoring Wells MW14A and MW14B SSE in Protected Area Type: Ground Water
	135	GWPP	Location: On-site Monitoring Wells MW15A and MW15B SE in Owner Controlled Area Type: Ground Water – discontinued

Table 5-1
ENVIRONMENTAL SAMPLE STATIONS

Station Number	Program	Station Location and Sample Type	
136	GWPP	Location: Type:	On-site Monitoring Wells MW16A and MW16B ENE in Protected Area Ground Water – discontinued
137	GWPP	Location: Type:	On-site Monitoring Well MW17C NE in Protected Area Ground Water – discontinued
138	REMP	Location: Type:	21,600 meters WSW Milk Control – discontinued Vegetation Control – discontinued
161	REMP	Location: Type:	On-site, ISFSI, East Fence Line TLD
162	REMP	Location: Type:	On-site, ISFSI, South Fence Line TLD
163	REMP	Location: Type:	On-site, ISFSI, West Fence Line TLD
164	REMP	Location: Type:	On-site, ISFSI, North Fence Line TLD
165	GWPP	Location: Type:	On-site Monitoring Well MW24A ESE in Owner Controlled Area Ground Water
166	GWPP	Location: Type:	On-site Monitoring Well MW25A ESE in Owner Controlled Area Ground Water – discontinued
167	GWPP	Location: Type:	On-site Monitoring Well MW26A ESE in Owner Controlled Area Ground Water
168	GWPP	Location: Type:	On-site Monitoring Wells MW27A and MW27B SE in Owner Controlled Area Ground Water – discontinued
169	GWPP	Location: Type:	On-site Monitoring Wells MW28A and MW28B SE in Owner Controlled Area Ground Water
170	GWPP	Location: Type:	On-site Monitoring Wells MW29A and MW29B SE in Owner Controlled Area Ground Water
171	GWPP	Location: Type:	On-site Monitoring Wells MW30A and MW30B SE in Owner Controlled Area Ground Water
172	GWPP	Location: Type:	On-site Monitoring Wells MW31A and MW31B SE in Owner Controlled Area Ground Water – discontinued
173	GWPP	Location: Type:	On-site Monitoring Wells MW32A and MW32B SE in Owner Controlled Area Ground Water

6.0 RADIOLOGICAL EFFLUENT CONTROLS AND SURVEILLANCE REQUIREMENTS

O 6.0.1 Use and Application

O 6.0.1.1 Definitions

NOTE
The defined terms of this section appear in capitalized type and are applicable throughout these OLCOs and Bases

Term	Definition
ACTIONS	ACTIONS shall be that part of an DODAM Specification that prescribes Required Actions to be taken under designated Conditions within specified Completion Times.
CHANNEL CALIBRATION	A CHANNEL CALIBRATION shall be the adjustment, as necessary, of the channel output such that it responds within the necessary range and accuracy to known values of the parameter that the channel monitors. The CHANNEL CALIBRATION shall encompass the entire channel, including the required sensor, alarm, display, and trip functions, and shall include the CHANNEL FUNCTIONAL TEST. The CHANNEL CALIBRATION may be performed by means of any series of sequential, overlapping, or total channel steps so that the entire channel is calibrated.
CHANNEL	A channel is an arrangement of a sensor and associated components used to evaluate plant variables and produce discrete outputs used in logic. A channel terminates and loses its identity where individual channel outputs are combined in logic.
CHANNEL CHECK	A CHANNEL CHECK shall be the qualitative assessment, by observation, of channel behavior during operation. This determination shall include, where possible, comparison of the channel indication and status to other indications or status derived from independent instrument channels measuring the same parameter.
CHANNEL FUNCTIONAL TEST	A CHANNEL FUNCTIONAL TEST shall be the injection of a simulated or actual signal into the channel as close to the sensor as practicable to verify FUNCTIONALITY, including required alarm, interlock, display, and trip functions, and channel failure trips. The CHANNEL FUNCTIONAL TEST may be performed by means of any series of sequential, overlapping, or total channel steps.

(continued)

GASEOUS RADWASTE TREATMENT SYSTEM	A GASEOUS RADWASTE TREATMENT SYSTEM is any system designed and installed to reduce radioactive gaseous effluents by collecting primary coolant system offgases from the primary system and providing delay or holdup for the purpose of reducing radioactivity prior to release to the environment.
MEMBER(S) OF THE PUBLIC	MEMBER(S) OF THE PUBLIC are persons who are not occupationally associated with the Company and who do not normally frequent the DAEC site. The category does not include contractors, contractor employees, vendors, or persons who enter the site to make deliveries or to service equipment.
FUNCTIONAL-FUNCTIONALITY	An SSC is functional or has functionality when it is capable of performing its function(s), as set forth in the CLB.
SITE BOUNDARY	The SITE BOUNDARY is that line beyond which the land is neither owned, nor leased, nor otherwise controlled by the Company. UFSAR Figure 1.2-1 identifies the DAEC SITE BOUNDARY.
SOURCE CHECK	A SOURCE CHECK is the assessment of channel response when the channel sensor is exposed to a source of radiation.
UNRESTRICTED AREA	The UNRESTRICTED AREA is that land (offsite) beyond the SITE BOUNDARY.

O 6.0.1 Use and Application

O 6.0.1.2 Logical Connectors

PURPOSE	The purpose of this section is to explain the meaning of logical connectors. Logical connectors are used in Defueled Offsite Dose Assessment Manual (DODAM) to discriminate between, and yet connect, discrete Conditions, Required Actions, Completion Times, Surveillances, and Frequencies. The only logical connectors that appear in the DODAM are **AND;** and **OR**. The physical arrangement of these connectors constitutes logical conventions with specific meanings.
BACKGROUND	Several levels of logic may be used to state Required Actions. These levels are identified by the placement (or nesting) of the logical connectors and by the number assigned to each Required Action. The first level of logic is identified by the first digit of the number assigned to a Required Action and the placement of the logical connector in the first level of nesting (i.e., left justified with the number of the Required Action). The successive levels of logic are identified by additional digits of the Required Action number and by successive indentions of the logical connectors. When logical connectors are used to state a Condition, Completion Time, Surveillance, or Frequency, only the first level of logic is used, and the logical connector is left justified with the statement of the Condition, Completion Time, Surveillance, or Frequency.

O 6.0.1 Use and Application

O 6.0.1.3 Completion Times

PURPOSE	The purpose of this section is to establish the Completion Time convention and to provide guidance for its use.
BACKGROUND	OLCOs specify minimum requirements for ensuring safe operation of the unit. The ACTIONS associated with an OLCO state Conditions that typically describe the ways in which the requirements of the OLCO can fail to be met. Specified with each stated Condition are Required Action(s) and Completion Times(s).
DESCRIPTION	The Completion Time is the amount of time allowed for completing a Required Action. It is referenced to the time of discovery of a situation (e.g., non-functional equipment or variable not within limits) that requires entering an ACTIONS Condition unless otherwise specified, providing the unit is in a specified condition stated in the Applicability of the OLCO. Required Actions must be completed prior to the expiration of the specified Completion Time. An ACTIONS Condition remains in effect and the Required Actions apply until the Condition no longer exists or the unit is not within the OLCO Applicability. If situations are discovered that require entry into more than one Condition at a time within a single OLCO (multiple Conditions), the Required Actions for each Condition must be performed within the associated Completion Time. When in multiple Conditions, separate Completion Times are tracked for each Condition starting from the time of discovery of the situation that required entry into the Condition.
	When "Immediately" is used as a Completion Time, the Required Action should be pursued without delay and in a controlled manner. The DODAM implements Completion Times in precisely the same manner they are applied in the Technical Requirements Manual (TRM) and the Technical Specifications (TS).

O 6.0.1 Use and Application

O 6.0.1.4 Frequency

PURPOSE	The purpose of this section is to define the proper use and application of Frequency Requirements.
DESCRIPTION	Each DODAM Surveillance Requirement (OSR) has a specified Frequency in which the Surveillance must be met in order to meet the associated OLCO. An understanding of the correct application of the specified Frequency is necessary for compliance with the OSR. The "specified Frequency" is referred to throughout this section and each of the Specifications of Section 6.0.3.0, Surveillance Requirement (SR) Applicability. The "specified Frequency" consists of the requirements of the Frequency column of each OSR, as well as certain Notes in the Surveillance column that modify performance requirements. Sometimes special situations dictate when the requirements of a Surveillance are to be met. They are "otherwise stated" conditions allowed by OSR 7.0.3.0.1. They may be stated as clarifying Notes in the Surveillance, as part of the Surveillance or both. Situations where a Surveillance could be required (i.e., its Frequency could expire), but where it is not possible or not desired that it be performed until sometime after the associated OLCO is within its Applicability, represent potential OSR 7.0.3.0.4 conflicts. To avoid these conflicts, the OSR (i.e., the Surveillance or the Frequency) is stated such that it is only "required" when it can be and should be performed. With an OSR satisfied, OSR 6.0.3.0.4 imposes no restriction. The use of "met" or "performed" in these instances conveys specific meanings. A Surveillance is "met" only when the acceptance criteria are satisfied. Known failure of the requirements of a Surveillance, even without a Surveillance specifically being "performed," constitutes a Surveillance not "met." "Performance" refers only to the requirement to specifically determine the ability to meet the acceptance criteria. OSR 7.0.3.04 restrictions would not apply if both the following conditions are satisfied: a. The surveillance is not required to be performed; and b. The Surveillance is not required to be met or, even if required to be met, is not known to be failed.

6.0.3 Limiting Conditions for Operation (OLCO) Applicability

OLCO 6.0.3.0.1	OLCOs shall be met during the specified conditions in the Applicability, except as provided in OLCO 6.0.3.0.2.
OLCO 6.0.3.0.2	Upon discovery of a failure to meet an OLCO, the Required Actions of the associated Conditions shall be met, except as provided in OLCO 6.0.3.0.5. If the OLCO is met or is no longer applicable prior to expiration of the specified Completion Time(s), completion of the Required Action(s) is not required, unless otherwise stated.
OLCO 6.0.3.0.3	Not Used.
OLCO 6.0.3.0.4	When an OLCO is not met, entry into a specified condition in the Applicability shall not be made except when the associated ACTIONS to be entered permit continued operation in the specified condition in the Applicability for an unlimited period of time.
OLCO 6.0.3.0.5	Equipment removed from service or declared non-functional to comply with ACTIONS may be returned to service under administrative control solely to perform testing required to demonstrate its FUNCTIONALITY or the FUNCTIONALITY of other equipment. This is an exception to OLCO 6.0.3.0.2 for the system returned to service under administrative control to perform the testing required to demonstrate FUNCTIONALITY.

O 7.0.3 Surveillance Requirement (OSR) Applicability

OSR 7.0.3.0.1	OSRs shall be met during the specified conditions in the Applicability for individual OLCOS, unless otherwise stated in the OSR. Failure to meet a Surveillance, whether such failure is experienced during the performance of the Surveillance or between performances of the Surveillance, shall be failure to meet the OLCO. Failure to perform a Surveillance within the specified Frequency shall be failure to meet the OLCO except as provided in OSR 7.0.3.0.3. Surveillances do not have to be performed on non-functional equipment or variables outside specified limits.
OSR 7.0.3.0.2	The specified Frequency for each OSR is met if the Surveillance is performed within 1.25 times the interval specified in the Frequency, as measured from the previous performance or as measured from the time a specified condition of the Frequency is met. For Frequencies specified as "once," the above interval extension does not apply. If a Completion Time requires periodic performance on a "once per . . ." basis, the above Frequency extension applies to each performance after the initial performance. Exceptions to this OSR are stated in the individual OLCOs.
OSR 7.0.3.0.3	If it is discovered that a Surveillance was not performed within its specified Frequency, then compliance with the requirement to declare the OLCO not met may be delayed, from the time of discovery, up to 24 hours or up to the limit of the specified Frequency, whichever is more. This delay period is permitted to allow performance of the Surveillance. If the Surveillance is not performed within the delay period, the OLCO must immediately be declared not met, and the applicable Condition(s) must be entered. When the Surveillance is performed within the delay period and the Surveillance is not met, the OLCO must immediately be declared not met, and the applicable Condition(s) must be entered.
OSR 7.0.3.0.4	Entry into a MODE or other specified condition in the Applicability of an OLCO shall not be made unless the OLCO's Surveillances have been met within their specified Frequency.

O 6.1 Radioactive Liquid Effluent Controls and Surveillance Requirements

O 6.1.1 Radioactive Liquid Instrumentation

OLCO 6.1.1 The Radioactive Liquid Instrumentation for each function in Table 6.1-1 shall be FUNCTIONAL with their alarm and trip setpoints set to ensure that the limits of Section 6.1.2 are not exceeded.

APPLICABILITY: As shown in Table 6.1-1

ACTIONS

CONDITION		REQUIRED ACTION		COMPLETION TIME
A.	One or more channels non-functional	A.1	Enter the condition(s) referenced in Table 6.1-1 for the channel(s).	Immediately
		AND		
		A.2	Restore required channel(s) to FUNCTIONAL status.	30 days
B.	As required by Required Action A.1 and referenced in Table 6.1-1	B.1	Obtain at least two samples and analyze in accordance with Section 6.1.2 and Table 7.1-2	Prior to release
		AND		
		B.1	Independently verify release rate calculations and discharge valving by technically qualified Facility Staff member.	Prior to release
C.	As required by Required Action A.1 and referenced in Table 6.1-1	C.1	Obtain sample and analyze for either gross beta/gamma or gamma radioactivity.	Once per 24 Hours
D.	Less than the minimum required channels FUNCTIONAL.	D.1	Estimate flow rate at least once per batch.	Immediately
E.	Required Action and associated Completion Time of Condition A not met.	E.1	Explain in next Annual Radioactive Material Release Report why the instrument was not made functional in a timely manner.	Next submittal of the Annual Radioactive Material Release Report

)5-2021

SURVEILLANCE REQUIREMENTS

---NOTE---
Instrumentation shall be FUNCTIONAL and in service except that channels out of service are permitted for the purpose of required tests, checks, and calibrations without declaring the channel to be non-functional.

SURVEILLANCE		FREQUENCY
OSR 7.1.1.1	Perform CHANNEL CHECK	24 Hours
OSR 7.1.1.2	Perform SOURCE CHECK	Prior to release
OSR 7.1.1.3	Perform SOURCE CHECK	30 Days
OSR 7.1.1.4	Perform CHANNEL FUNCTIONAL TEST	92 Days
OSR 7.1.1.5	Perform CHANNEL CALIBRATION	18 Months
OSR 7.1.1.6	Perform Monitor Setpoint Calculation.	18 Months
OSR 7.1.1.7	Perform CHANNEL FUNCTIONAL TEST	92 Days
OSR 7.1.1.8	Perform CHANNEL CALIBRATION	18 Months
OSR 7.1.1.9	Perform Monitor Setpoint Calculation.	Prior to release

TABLE 6.1-1 / 7.1-1

RADIOACTIVE LIQUID EFFLUENT MONITORING INSTRUMENTATION

Instrument	Applicability	Minimum Channels Functional	Conditions Referenced from Required Action A.1	Surveillance Requirements
1. Gross Radioactive Monitors Providing Automatic Termination of Release				
a. Liquid Radwaste Effluent Line	During Releases	1	B	OSR 7.1.1.1 [a] OSR 7.1.1.2 [b] OSR 7.1.1.7 [d] [e] OSR 7.1.1.8 [c] OSR 7.1.1.9
2. Gross Radioactive Monitors Not Providing Automatic Termination of Release				
10-2021 a. RHR Service Water System	During Releases	1	C	OSR 7.1.1.1 [a] OSR 7.1.1.3 OSR 7.1.1.4 [e] OSR 7.1.1.5 [c] OSR 7.1.1.6
b. RHRSW/ESW Rupture Disc Effluent Line	During Releases	1		
3. Flow Rate Measurement Devices				
a. Liquid Radwaste Effluent Line	During Releases	1	D	OSR 7.1.1.1 [a] [f] OSR 7.1.1.7 OSR 7.1.1.8
b. Liquid Radwaste Dilution Line	During Releases	1		

TABLE NOTATIONS

[a] During releases via this pathway.

[b] On any day on which a release is made, a SOURCE CHECK shall be made at least once, prior to the first release.

[c] The CHANNEL CALIBRATION shall include the use of a known radioactive source (traceable to the NIST radiation measurement system or acceptable non-NIST standards) positioned in a reproducible geometry with respect to the sensor and emitting beta or gamma radiation in the range measured by the channel. CHANNEL CALIBRATION may normally be done during refueling outages.

[d] The CHANNEL FUNCTIONAL TEST shall also demonstrate that automatic isolation of this pathway if the following condition exists:

1. Instrument indicates measured levels above the alarm/trip setpoint.

[e] The CHANNEL FUNCTIONAL TEST shall also demonstrate that control room alarm annunciation occurs if any of the following conditions exist:

1. Instrument indicates measured levels above the alarm/trip setpoint.
2. Circuit failure.
3. Instrument indicates a downscale failure.
4. Instrument controls not set in operate mode.

[f] CHANNEL CHECK shall consist of verifying indication of flow during periods of release. CHANNEL CHECK shall be made at least once daily on any day on which continuous, periodic, or batch releases are made.

O 6.1 Radioactive Liquid Effluent Controls and Surveillance Requirements

O 6.1.2 Liquid Effluent Concentration

OLCO 6.1.2 The concentration of radioactive material in liquid effluent released from the site to the UNRESTRICTED AREA shall not exceed ten times the concentrations specified in 10 CFR 20, Appendix B, Table 2, Column 2 to 10 CFR 20.1001 - 20.2402.

APPLICABILITY: At all times.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Concentration of radioactive material released from site to UNRESTRICTED AREAS not within the limit.	A.1 Restore concentration to within limits.	Immediately

SURVEILLANCE REQUIREMENTS

---NOTE---
Refer to DODAM Table 7.1-2 for details concerning Sampling and Analysis
----

	SURVEILLANCE	FREQUENCY
OSR 7.1.2.1	-------------------------NOTE------------------------- Portions of each sample are “split out” and added to the Monthly and Quarterly Composite sample. -- Sample the Batch Release.	PRIOR TO EACH RELEASE
OSR 7.1.2.2	The results of pre-release analyses shall be used with the calculation methods in the DODAM to establish trip setpoints for batch releases to assure that the concentration at the UNRESTRICTED AREA boundary does not exceed the limit in OLCO 6.1.2.	PRIOR TO EACH RELEASE
OSR 7.1.2.3	---------------------------NOTE------------------------- Portions of each sample are split out and added to the Monthly and Quarterly Composite sample. --- ---------------------------NOTE------------------------- In the event of a positive identification of reactor by-product radioactivity in the service water, additional sampling and analysis shall be performed as specified in Table 7.1-2, Table Items B.2 through B.5. --- Sample the Continuous Release during releases to the environment via this pathway. <u>AND</u> Analyze the sample from the release for Gross Beta/Gamma. <u>OR</u> Analyze the sample from the release for Principal Gamma Emitters and I-131.	7 Days

(continued)

OSR 7.1.2.4	Analyze the sample from the release for Principal Gamma Emitters and I-131.	PRIOR TO EACH RELEASE
OSR 7.1.2.5	Analyze the “Monthly” Composite sample for Tritium.	30 Days
OSR 7.1.2.6	--------------------------NOTE------------------------- For Service Water, this OSR is only required following a positive identification of reactor by-product radioactivity in the sample as determined during OSR 7.1.2.3 or OSR 7.1.2.5. --- ------------------------NOTE--------------------------- For “clean” water systems, this OSR is only required following a positive identification of reactor by-product radioactivity in the sample as determined during OSR 7.1.2.4. --- ------------------------NOTE--------------------------- For GWPP Mitigation system, this OSR is only required following a positive identification of a radioactive reactor by-product in the sample as determined during OSR 7.1.2.3. --- Analyze the Quarterly Composite sample for Sr-89, Sr-90, Fe-55	92 Days
OSR 7.1.2.7	Analyze the “Monthly” Composite sample for Gross Alpha.	30 Days

TABLE 7.1-2

RADIOACTIVE LIQUID WASTE SAMPLING AND ANALYSIS PROGRAM

Liquid Release Type	Surveillance Requirements	Table item/note	Type of Activity Analysis	Lower Limit of Detection (LLD)[(a)] (μCi/mL)
A. Radwaste Tanks Batch Release	OSR 7.1.2.1 OSR 7.1.2.4	**A.1:** Each Batch	Principal Gamma Emitters	5×10^{-7}
	OSR 7.1.2.2		I-131[c]	1×10^{-6}
	OSR 7.1.2.1 OSR 7.1.2.5	**A.2:** (b), (d)	H-3	1×10^{-5}
	OSR 7.1.2.7		Gross alpha	1×10^{-7}
	OSR 7.1.2.1 OSR 7.1.2.6	**A.3:** (b), (d)	Sr-89, Sr-90	5×10^{-8}
			Fe-55	1×10^{-6}
B. Continuous Service Water Release	OSR 7.1.2.3	**B.1:** (d)	Gross beta/gamma	1×10^{-7}
Sample Points:	OSR 7.1.2.3	**B.2:** (d)	Principal Gamma Emitters	5×10^{-7}
0-2021 • RHR Service Water System A • RHR Service Water System B			I-131	1×10^{-6}
	OSR 7.1.2.5	**B.4:** (b), (d)	H-3	1×10^{-5}
	OSR 7.1.2.7		Gross alpha	1×10^{-7}
	OSR 7.1.2.6	**B.5:** (b), (d)	Sr-89, Sr-90	5×10^{-8}
			Fe-55	1×10^{-6}
C. "Clean" Systems Batch Release	OSR 7.1.2.1 OSR 7.1.2.4	**C.1:**	Principal Gamma Emitters[(f)]	5×10^{-7}
Sample Points:			I-131[(f)]	1×10^{-6}
• CST Pit	OSR 7.1.2.5	**C.2:** (b) (d) (e)	H-3[(f)]	1×10^{-5}
• Transformer Pit • Neutralizing Tank 1T022 • FRAC tanks • Temporary groundwater	OSR 7.1.2.6	**C.3:** (b) (d)	Sr-89, Sr-90	5×10^{-8}
			Fe-55	1×10^{-6}
D. GWPP Mitigation System	OSR 7.1.2.3	D.1:	Principal Gamma Emitters	5.0×10^{-7}
Continuous Release			I-131[(c)]	1.0×10^{-6}
	OSR 7.1.2.5	D.2: (b)(d)(e)	H-3[(f)]	1.0×10^{-5}
	OSR 7.1.2.6	D.3: (b)(d)	Sr-89, Sr-90	5.0×10^{-8}
			Fe-55	1.0×10^{-6}

TABLE NOTATIONS

[(a)]The LLD is defined, for purposes of these controls, as the smallest concentration of radioactive material in a sample that will yield a net count, above system background, that will be detected with 95% probability with only 5% probability of falsely concluding that a blank observation represents a "real" signal.

For a particular measurement, which may include radiochemical separation:

$$LLD = \frac{4.66\ S_b}{E\ x\ V\ x\ 2.22E6\ x\ e^{-\lambda\Delta t}}$$

where:

LLD is the "a priori" lower limit of detection as defined above (microcuries per unit mass or volume)

and where:

S_b is the standard deviation of the background counting rate or of the counting rate of a blank sample as appropriate (counts per minute)
E is the counting efficiency (counts per disintegration)
V is the sample size (units of mass or volume)
2.22E6 is the number of disintegrations per minute per microcurie,
Y is the fractional radiochemical yield, when applicable,
λ is the radioactive decay constant for the particular radionuclide (sec^{-1}), and
Δ t for effluents is the elapsed time between the midpoint of sample collection and the time of counting (sec^{-1}).

Alternatively, exp may be replaced by

$$\frac{\lambda t_1 e^{-\lambda t_2}}{1 - e^{-\lambda t_1}} \bullet \cdot\ e^{-\lambda t_2}$$

Where:

t_1 is the total sampling time or sample compositing time
t_2 is the elapsed time between the end of sample collection and the time of counting.

It should be recognized that the LLD is defined as an a priori (before the fact) limit representing the capability of a measurement system and not as an a posteriori (after the fact) limit for a particular measurement.

Analyses shall be performed in such a manner that the stated LLDs will be achieved under routine conditions with typical values of E, V, Y, and Dt for the radionuclides Mn-54, Fe-59, Co-58, Co-60, Zn-65, Mo-99, Cs-134, Cs-137, Ce-141, and Ce-144. Occasionally background fluctuations, unavoidably small sample sizes, interfering radionuclides, or other uncontrollable circumstances may render these LLDs unachievable.

When calculating the LLD for a radionuclide determined by gamma ray spectrometry, the background may include the typical contributions of other radionuclides normally present in the samples. The background count rate of a Ge(Li) detector is determined from background counts that are determined to be within the full width of the specific energy band used for the quantitative analysis for that radionuclide.

The principal gamma emitters for which the LLD specification will apply are exclusively the following radionuclides: Mn-54, Fe-59, Co-58, Co-60, Zn-65, Mo-99, Cs-134, Cs-137 and Ce-141. Ce-144 shall be measured, but with an LLD of 5E-6. This list does not mean that only these nuclides are to be detected and reported. Other peaks which are measurable and identifiable, together with the above nuclides, shall also be identified and reported. Nuclides which are below the LLD for the analyses should not be reported as being present at the LLD level. When unusual circumstances result in LLDs higher than required, the reasons shall be documented in the Annual Radioactive Material Release Report.

(b) A composite sample is one in which the quantity of liquid sampled is proportional to the quantity of liquid waste discharged and in which the method of sampling employed results in a specimen which is representative of the liquids released.

(c) In the event a gross β or γ analysis is performed in lieu of an isotopic analysis before a batch is discharged, a sample shall be analyzed for principal gamma emitters afterward.

(d) Analysis may be performed after release.

(e) Analysis at a frequency of less than 30 days is allowed.

(f) If liquids from these systems are released from the site via a pathway that is NOT directly to the Cedar River, the required LLDs of Table 6.3.2 are applicable.

O 6.1 Radioactive Liquid Effluent Controls and Surveillance Requirements

O 6.1.3 Dose Due to Liquid Radioactive Effluents

OLCO 6.1.3 The dose or dose commitment to a MEMBER OF THE PUBLIC from radioactive materials in liquid effluents released to the UNRESTRICTED AREA shall not exceed:

1.5 mrem to the total body during any calendar quarter,
5.0 mrem to any organ during any calendar quarter,
3.0 mrem to the total body during any calendar year, or
10.0 mrem to any organ during any calendar year.

APPLICABILITY: At All Times

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Calculated dose from the release of radioactive materials in liquid effluents exceeds limits.	A.1 Prepare and submit a Special Report to the Commission which identifies the cause(s) for exceeding the limit and defines the action to be taken.	30 days

SURVEILLANCE REQUIREMENTS

	SURVEILLANCE	FREQUENCY
OSR 7.1.3.1	In any quarter in which radioactive liquid effluent is discharged, an assessment shall be performed in accordance with the DODAM in order to verify that the cumulative dose commitment does not exceed the limits in OLCO 6.1.3.	30 Days

O 6.1 Radioactive Liquid Effluent Controls and Surveillance Requirements

O 6.1.4 Liquid Waste Treatment

OLCO 6.1.4 Appropriate liquid radwaste equipment shall be used to treat any untreated batch of liquid waste prior to discharge when a pre-released analysis indicates a radioactivity concentration (exclusive of tritium and dissolved gases) of 0.01 μCi/ml or higher.

APPLICABILITY: At all times.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Radioactive liquid waste being discharge without treatment and in excess of limits.	A.1 Prepare and submit a Special Report to the Commission which includes identification of non-functional equipment or subsystems and the reason, actions taken to restore the non-functional equipment, and description of action(s) taken to prevent recurrence.	30 days

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
OSR 7.1.4.1	Each radioactive liquid waste batch shall be sampled and analyzed in accordance with Section 7.1.2 and Table 7.1-2.	PRIOR TO EACH RELEASE

O 6.1 Radioactive Liquid Effluent Controls and Surveillance Requirements

O 6.1.5 Liquid Holdup Tanks

OLCO 6.1.5 The quantity of radioactive material contained in the unprotected outdoor tanks shall be limited to less than or equal to 50 Curies, excluding tritium and dissolved or entrained noble gases.

APPLICABILITY: At all times.

ACTIONS

--NOTE--

Tanks included in this specification are those outdoor tanks that are not surrounded by liners, dikes, or walls capable of holding the tanks' contents and that do not have tank overflows and surrounding area drains connected to the liquid radwaste treatment system. (The liquid radwaste storage tanks (1T-88 and 1T-269) located in the Low-Level Radwaste Processing and Storage Facility are considered unprotected outdoor tanks.)

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Quantity of radioactive material in the tanks exceeding the limit.	A.1 Suspend all additions of the radioactive material to the tanks.	Immediately
	<u>AND</u>	
	A.2 Reduce tank contents to within the limit.	48 hours
	<u>AND</u>	
	A.3 Describe in next Annual Radioactive Effluent Release Report the events leading to this condition.	1 year
	<u>AND</u>	
	A.4 Prepare and submit a Special Report to the Commission which identifies the cause(s) for exceeding the limit and defines the action to be taken. (10CFR20.2203)	30 Days

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
OSR 7.1.5.1	When radioactive materials are being added to a tank, the quantity of radioactive material contained in all tanks shall be determined to be within the 50Curie limit by analyzing a representative sample of the tanks' contents.	7 Days

O 6.2 Radioactive Gaseous Effluent Controls and Surveillance Requirements

O 6.2.1.1 Radioactive Gaseous Effluent Instrumentation

OLCO 6.2.1.1 The Radioactive Gaseous Effluent Instrumentation for each function in Table 6.2-1 shall be FUNCTIONAL

APPLICABILITY: During releases via this pathway.

ACTIONS

CONDITION		REQUIRED ACTION		COMPLETION TIME
A.	One or more channels non-functional.	A.1	Enter the condition referenced in Table 6.2-1 for the channel.	Immediately
		<u>AND</u>		
		A.2	Restore required channel(s) to FUNCTIONAL status.	30 days
D.	As required by Required Action A.1 and referenced in Table 6.2-1	D.1	Establish continuous sampling with auxiliary equipment.	8 hours
		<u>AND</u>		
		D.2	Collect sample(s)	Once per 7 days
		<u>AND</u>		
		D.3	Analyze sample(s) for radioactivity.	Within 24 hours of sample collection
E.	As required by Required Action A.1 and referenced in Table 6.2-1	E.1	Estimate flow rate initially and whenever operation of a main exhaust fan combination is changed in the system.	8 Hours
F.	As required by Required Action A.1 and referenced in Table 6.2-1	F.1	Collect gaseous grab sample.	once per 24 hours
		<u>AND</u>		
		F.2	Analyze sample for radioactivity.	Within 24 hours of sample collection

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(continued)

ACTIONS (continued)

G. Minimum required instrumentation not returned to FUNCTIONAL status within 30 days.	G.1	Explain in next Annual Radioactive Material Release Report why the instrument was not made functional in a timely manner.	

SURVEILLANCE REQUIREMENTS

--NOTE--

Instrumentation shall be FUNCTIONAL and in service except that channels out of service are permitted for the purpose of required tests, checks and calibrations without declaring the channel to be non-functional.

SURVEILLANCE		FREQUENCY
OSR 7.2.1.1.1	Perform CHANNEL CHECK -------------------------NOTE---------------------------- During releases via this pathway. --	24 Hours
OSR 7.2.1.1.2	Perform CHANNEL CHECK	7 Days
OSR 7.2.1.1.3	Perform SOURCE CHECK	30 Days
OSR 7.2.1.1.4	Determine the monitor setpoints in accordance with the method described in the DODAM.	92 Days
OSR 7.2.1.1.5	Perform CHANNEL FUNCTIONAL TEST	92 Days
OSR 7.2.1.1.6	Determine the monitor setpoints in accordance with the method described in the DODAM.	18 Months
OSR 7.2.1.1.7	Perform CHANNEL CALIBRATION	18 Months

	TABLE 6.2-1		TABLE 7.2-1
NORMAL RANGE RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION			
Instrument	Minimum Channels Functional [a b]	Conditions Referenced from Required Action A.1	Surveillance Requirements
10-2021 **2. Reactor Building Exhaust Vent Monitoring System**			
10-2021 a. Noble Gas Activity Monitor	1	F	OSR 7.2.1.1.1, OSR 7.2.1.1.3, OSR 7.2.1.1.5 [(d)], OSR 7.2.1.1.6, OSR 7.2.1.1.7 [(c)]
b. Iodine Sampler Cartridge	1	D	OSR 7.2.1.1.2
c. Particulate Sampler Filter	1	D	OSR 7.2.1.1.2
d. Effluent Flow Measuring Device	1	E	OSR 7.2.1.1.1, OSR 7.2.1.1.5, OSR 7.2.1.1.7
e. Sample Flow Measuring Device	1	E	OSR 7.2.1.1.1, OSR 7.2.1.1.5, OSR 7.2.1.1.7
3. Turbine Building Exhaust Vent Monitoring System			
10-2021 c. Particulate Sampler Filter	1	D	OSR 7.2.1.1.2
d. Effluent Flow Measuring Device	1	E	OSR 7.2.1.1.1, OSR 7.2.1.1.5, OSR 7.2.1.1.7
e. Sample Flow Measuring Device	1	E	OSR 7.2.1.1.1, OSR 7.2.1.1.5, OSR 7.2.1.1.7
4. Low-Level Radwaste Processing and Storage Facility Exhaust Vent Monitoring System			
10-2021 c. Particulate Sampler Filter	1	D	OSR 7.2.1.1.2
d. Effluent Flow Measuring Device	1	E	OSR 7.2.1.1.1, OSR 7.2.1.1.5, OSR 7.2.1.1.7
e. Sample Flow Measuring Device	1	E	OSR 7.2.1.1.1, OSR 7.2.1.1.5, OSR 7.2.1.1.7

continued

TABLE 6.2-1 NOTATIONS

[(a)]During releases via this pathway.

[(b)] Instrumentation shall be FUNCTIONAL and in service except that channels out of service are permitted for the purpose of required tests, checks and calibrations without declaring the channel to be non-functional.

TABLE 7.2-1 NOTATIONS

[(c)] The CHANNEL CALIBRATION shall include the use of a known radioactive source (traceable to the NIST radiation measurement system or other acceptable non-NIST standards) positioned in a reproducible geometry with respect to the sensor and emitting beta and/or gamma radiation in the range measured by the channel in accord with established station calibration procedures. Alternatively, after the initial calibration, noble gas activity monitors maybe calibrated by laboratory analyzed gas samples collected and analyzed per Table 7.2-2, item A.

[(d)] The CHANNEL FUNCTIONAL TEST shall also demonstrate that control room alarm annunciation occurs if any of the following conditions exist:

1.0 Instrument indicates measured levels above the alarm setpoint.
2.0 Circuit failure.
3.0 Instrument indicates a downscale failure.
4.0 Deleted.

O 6.2 Radioactive Gaseous Effluent Controls and Surveillance Requirements

O 6.2.2 Gaseous Effluent Dose Rate

OLCO 6.2.2

The dose rate in the UNRESTRICTED AREA due to the release of gaseous effluents shall not exceed:

a. 500 mrem/year to the total body or 3000 mrem/year to skin due to radioactive noble gas; and
b. 1500 mrem/year to any organ due to I-131, I-133, H-3, and to radioactive particulates having half-lives of 8 days or more.

APPLICABILITY: During Releases.

ACTIONS

CONDITION		REQUIRED ACTION		COMPLETION TIME
A.	Dose rate not within limits.	A.1	Reduce the release rate within limit.	Immediately

SURVEILLANCE REQUIRMENTS

SURVEILLANCE		FREQUENCY
OSR 7.2.2.1	Monitor for Radioactive Noble Gas gamma activity.	Continuously
OSR 7.2.2.2	Sample the Effluent Stream for I-131. <u>AND</u> Analyze for I-131 within 48 hours of sample pull.	Continuously 7 Days
OSR 7.2.2.3	-------------NOTE------------- Retain samples for Quarterly Composite analysis. (OSR 7.2.2.8) ---------------------------- Sample the Effluent Stream for particulates. <u>AND</u> Analyze for principal gamma emitters from particulates within 48 hours of sample pull.	 Continuously 7 Days
OSR 7.2.2.4	Perform gas grab sample of the effluent stream and analyze for principal gamma emitters.	30 Days
OSR 7.2.2.6	Perform grab sample of the effluent stream and analyze for Tritium.	92 Days
OSR 7.2.2.8	Analyze Particulate Composite sample for Sr-89, Sr-90, Gross Alpha, Fe-55, Ni-63	92 Days

TABLE 7.2-2

RADIOACTIVE GASEOUS WASTE SAMPLING AND ANALYSIS PROGRAM

Gaseous Release Type	Surveillance Requirements	Frequency	Type of Activity Analysis	Lower Limit of Detection (LLD)[(a)]
A. Deleted				
B. Reactor Building Vent	OSR 7.2.2.3 [(e)]	Routine	Particulate Sample Principal Gamma Emitters (I-131, Others)	$1 \times 10^{-11(d)}$
	OSR 7.2.2.4	Routine	Gas Grab Sample Principal Gamma Emitters	$1 \times 10^{-4(d)}$
	OSR 7.2.2.6	Routine	Tritium Grab Sample (H-3)	1×10^{-6}
	OSR 7.2.2.8 [(e)]	Routine [(g)]	Composite Particulate Sample Sr-89, Sr-90 Gross Alpha Fe-55, Ni-63	1×10^{-11} 1×10^{-11} 1×10^{-11}
Turbine Building Vent	OSR 7.2.2.3 [(e)]	Routine	Particulate Sample Principal Gamma Emitters	$1 \times 10^{-11(d)}$
	OSR 7.2.2.8 [(e)]	Routine [(g)]	Composite Particulate Sample Sr-89, Sr-90 Gross Alpha Fe-55, Ni-63	1×10^{-11} 1×10^{-11} 1×10^{-11}
Low-Level Radwaste Processing Storage Facility Vent	OSR 7.2.2.3 [(e)]	Routine	Particulate Sample Principal Gamma Emitters	$1 \times 10^{-11(d)}$
	OSR 7.2.2.6	Routine	Tritium Grab Sample (H-3)	1×10^{-6}
	OSR 7.2.2.8 [(e)]	Routine [(g)]	Composite Particulate Sample Sr-89, Sr-90 Gross Alpha Fe-55, Ni-63	1×10^{-11} 1×10^{-11} 1×10^{-11}
C. Reactor Building Vent	OSR 7.2.2.1	Continuous	Monitor Radioactive Noble Gas gamma activity	1×10^{-6}

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TABLE NOTATIONS

[a] Units for the listed values are µCi/cc. See Table 7.1-2 for a definition of the lower limit of detection (LLD).

[d] The principal gamma emitters for which the LLD will apply are exclusively the following radionuclides: Kr-87, Kr-88, Xe-133, Xe-133m, Xe-135, and Xe-138 for gaseous emissions and Mn-54, Fe-59, Co-58, Co-60, Zn-65, Mo-99,Cs--134, Cs-137, Ce-141 and Ce-144 for particulate emissions. This list does not mean that only these nuclides are to be detected and reported. Other peaks which are measurable and identifiable, together with the above nuclides, shall also be identified and reported. Nuclides which are below the LLD may be reported as "less than" their respective LLD and should not be reported as being present at the LLD of the nuclide. Each measured radionuclide concentration is used in a required concentration or dose calculation only if it is detected at or above the LLD. When unusual circumstances persist more than 30 days and cause LLD higher than required, the reasons shall be documented in the Annual Radioactive Material Release Report.

[e] The ratio of the sample flow rate to the sampled stream flow rate shall be known for the time period covered by each dose or dose rate calculations made in accordance with Sections 6.2.2, 6.2.3, and 6.2.4.

[f] Sample media shall be changed at least once per seven days and the analysis completed within 48 hours after changing (or after removal from the sampler). When samples collected for 24 hours or less are analyzed, the corresponding LLD may be increased by a factor of 10.

[g] A quarterly composite sample shall include an equal fraction of each weekly particulate sample collected during the quarter. Quarterly samples are sent to an offsite laboratory to perform analysis.

O 6.2 Radioactive Gaseous Effluent Controls and Surveillance Requirements

O 6.2.3 Doses Due to Noble Gases

OLCO 6.2.3 The Air Dose in the UNRESTRICTED AREA due to noble gases released in gaseous effluents shall not exceed:

5.0 mrad from gamma radiation during any calendar quarter;
10.0 mrad from beta radiation during any calendar quarter;
10.0 mrad from gamma radiation during any calendar year; or
20.0 mrad from beta radiation during any calendar year.

APPLICABILITY: During Releases

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Calculated air dose not within limits.	A.1 Submit a Special Report to the NRC identifying the cause(s) for exceeding the limit and define the corrective actions taken.	30 days

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
OSR 7.2.3.1	An assessment shall be performed in accordance with the DODAM to verify that the cumulative air dose during the quarter and year due to noble gases does not exceed the limits in DODAM section 6.2.3.	30 Days

O 6.2 Radioactive Gaseous Effluent Controls and Surveillance Requirements

O 6.2.4 Doses Due to Iodine and Particulates in Air

OLCO 6.2.4 The dose to a MEMBER OF THE PUBLIC from Iodine-131, I-133, H-3, and from radionuclides in particulate form having half-lives greater than eight days in gaseous effluents released from the site to the UNRESTRICTED AREA shall not exceed:

7.5 mrem to any organ during any calendar quarter; or
15.0 mrem to any organ during any calendar year.

APPLICABILITY: During Releases.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Calculated dose not within limits.	A.1 Submit a Special Report to the NRC identifying the cause(s) for exceeding the limit and define the corrective actions taken.	30 days

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
OSR 7.2.4.1	An assessment shall be performed in accordance with the DODAM to verify that the cumulative dose commitment due to I-131, I-133, H-3, and radioactive particulates having half-lives greater than eight days in gaseous effluents does not exceed the limits in DODAM Section 6.2.4.	30 Days

O 6.3 Offsite Dose Assessment Controls and Surveillance Requirements

O 6.3.1 Dose

OLCO 6.3.1 The annual dose or dose commitment to any MEMBER OF THE PUBLIC due to radiation and radioactive material in effluents from DAEC shall not exceed 75 mrem to the thyroid or 25 mrem to the total body or any other organ.

APPLICABILITY: At all times.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Calculated dose from radioactive material released in liquid or gaseous effluents exceeds twice the limits of Sections 6.1.3, 6.2.3 or 6.2.4	A.1 Perform an assessment of compliance with 40 CFR 190 and 10 CFR 72.104 and limit subsequent releases such that the dose or dose commitment to a MEMBER OF THE PUBLIC is ≤75 mrem to his thyroid and ≤25 mrem to his total body or any other organ over 12 consecutive months including the period of elevated release.	7 days
B. The estimated dose exceeds either limit in Section 6.3.1.	B.1 Prepare and submit a Special Report to the NRC in lieu of any other report; it shall include the cause of the release of exposure, an estimate of the dose to the likely most exposed MEMBER(s) OF THE PUBLIC, corrective actions taken or planned to prevent a recurrence, and a schedule for achieving compliance. If the condition causing the limit(s) to be exceeded has not been corrected, the Special Report may also state a request for a variance in accordance with the provisions of 40 CFR Part 190. In that event, the request is timely, and a variance is granted until NRC action on the request is complete.	30 days

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
OSR 7.3.1.1	Cumulative dose contributions from liquid and gaseous effluents to a MEMBER OF THE PUBLIC offsite shall be evaluated as described in the DODAM.	12 Months

O 6.3 Offsite Dose Assessment Controls and Surveillance Requirements

O 6.3.2 Radiological Environmental Monitoring Program (REMP) and Ground Water Protection Program (GWPP)

OLCO 6.3.2 A radiological environmental monitoring program shall be conducted as specified in Table 6.3-1.

APPLICABILITY: At all times.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Deviation from Table 6.3-1	A.1 Deviations are permitted from Table 6.3-1 if specimens are unobtainable due to hazardous conditions, seasonal unavailability, malfunction of automatic sampling equipment and other legitimate reasons. If specimens are unobtainable due to sampling equipment malfunction, every effort shall be made to complete corrective action prior to the end of the next sampling period. All deviations from Table 6.3-1 shall be documented in the Annual Radiological Environmental Report.	Once per year
B. In the event radioactivity in a sampled environmental medium, averaged over a calendar quarter, is attributable to DAEC and exceeds an appropriate value listed in Table 6.3-3 or, if not listed, causes a potential annual dose exceeding two times the quarterly dose limit in Section 6.1.3 or 6.2.4	B.1 Prepare and submit to the Commission within 30 days after discovery a Special Report which includes an evaluation of any release conditions, environmental factors or other conditions which caused the value(s) of Table 6.3-3 or two times the quarterly dose limit to be exceeded and which defines the corrective actions to be taken. If the radioactivity in environmental sample(s) is not attributable to releases from the Station, the Special Report is not required. Instead, the sample(s) result(s) shall be reported and explained in the Annual Radiological Environmental Report.	30 Days

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
C. When environmental sampling medium is not available from a sampling location or the location is no longer appropriate.	C.1 The cause and the location where replacement samples were obtained and/or will be obtained shall be reported in the Annual Radiological Environmental Report.	Once per year
D. A location is identified at which the calculated personal dose associated with one or more exposure pathways exceeds by 20% the maximum calculated dose associated with like pathway(s) at a location where sampling is conducted as specified by Table 6.3-1	D.1 The pathway(s) having maximum exposure potential at the newly identified location will be added to the radiological monitoring program at a subsequent Operations Committee meeting, if samples are reasonably attainable at the new location. Like pathway(s) monitored (sampled) at a location, excluding the control station location(s), having a lesser associated calculated personal dose may be deleted from the program at the time the new pathway(s) and location are added.	Once per year

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
OSR 7.3.2.1	Sampling and analyses required in Table 6.3-1 shall be performed such that the detection capabilities specified in Table 6.3-2 are achieved under routine conditions. If a sample analysis does not meet the LLD specified, report the reason attributed in the next Annual Radiological Environmental Report.	12 Months
10-2021 OSR 7.3.2.2	DAEC shall conduct biennially a land use census within three miles of the Station to identify radiologically important changes in land use.	24 Months

TABLE 6.3-1

RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM

	Exposure Pathway and/or Sample Type	Minimum Number of Sampling Stations	Sampling and Collection Schedule [(a)]	Type and Frequency of Analysis
10-2021	Airborne Particulates	Two	Continuous operation of sampler with sample collection WEEKLY or as required by dust loading	Analyze for gross beta activity ≥ 72 hours after filter change. Perform gamma isotopic analysis on each sample having gross beta activity > 10 times the yearly mean of control samples. Perform gamma isotopic analysis on composite (by sampling location) of samples collected during each quarter
10-2021	Ambient Radiation	Twenty	QUARTERLY	Read gamma radiation dose quarterly.
	Surface Water	Two	MONTHLY	Tritium and Gamma isotopic analysis of each sample.
10-2021	Drinking Water	Two	MONTHLY	Tritium and Gamma isotopic analysis of each sample.
	River Sediment	One	SEMIANNUALLY	Gamma isotopic analysis of each sample.
	Fish	Two	SEMIANNUALLY	Gamma isotopic analysis on edible portion.
10-2021	Vegetation	One	ANNUALLY at harvest time. At least two samples of each: grain green leafy vegetation forage.	Gamma isotopic and I-131 analysis of each sample.

[(a)]The following definitions of schedule timing shall apply to Table 6.3-1 only:

WEEKLY – Not less than once per calendar week. A maximum interval of 11 days is allowed between the collection of any two consecutive samples.

BIWEEKLY – Not less than once every two calendar weeks. A maximum interval of 18 days is allowed between the collection of any two consecutive samples.

MONTHLY – Not less than once per calendar month. An interval of not less than 10 days will be provided between collection of any two consecutive samples.

QUARTERLY – Not less than once per calendar quarter. An interval of not less than 30 days will be provided between collection of any two consecutive samples.

SEMIANNUALLY – One sample each between calendar dates (January 1 – June 30) and (July 1 – December 31). An interval of not less than 60 days will be provided between collection of any two consecutive samples.

ANNUALLY – Not less than once per calendar year with an interval of not less than 120 days between collection of any two consecutive samples.

TABLE 6.3-2

GROUNDWATER PROTECTION PROGRAM

	Exposure Pathway and/or Sample Type	Minimum Number of Sampling Stations	Sampling and Collection Schedule [a]	Type and Frequency of Analysis
0-2021	Groundwater	Twenty-Eight	QUARTERLY	Tritium and Gamma isotopic analysis of each sample.
0-2021	Groundwater Conditional	One [c]	MONTHLY	Tritium and Gamma isotopic analysis of each sample.
	DAEC Sewage Plant Effluent	One	BIWEEKLY	Tritium and Gamma isotopic analysis of each sample.

[a]The following definitions of schedule timing shall apply to Table 6.3-2 only:

WEEKLY – At least once per calendar week.

BIWEEKLY – Not less than once every two calendar weeks. A maximum interval of 18 days is allowed between the collection of any two consecutive samples.

MONTHLY – Not less than once per calendar month. An interval of not less than 10 days will be provided between collection of any two consecutive samples.

QUARTERLY – Not less than once per calendar quarter. An interval of not less than 30 days will be provided between collection of any two consecutive samples.

SEMIANNUALLY – One sample each between calendar dates (January 1 – June 30) and (July 1 – December 31). An interval of not less than 60 days will be provided between collection of any two consecutive samples.

ANNUALLY – Not less than once per calendar year with an interval of not less than 120 days between collection of any two consecutive samples.

0-2021 [c]The number of conditional wells sampled in a calendar month depends upon identified isotope, concentration of isotope, targeted aquifer, and extraction well operations.

TABLE 6.3-3						
MAXIMUM VALUES OF THE LOWER LIMIT OF DETECTION FOR ENVIRONMENTAL SAMPLE ANALYSIS [(a)]						
Analysis	Medium					
	Water (pCi/L)	Airborne Particulate or Gas (pCi/m^3)	Fish (pCi/kg, wet)	Milk (pCi/L)	Food Products (pCi/kg, wet)	Sediment (pCi/kg, dry)
Gross beta	4	1 x 10^{-2}				
H-3	2000 [(b)] 3000 [(c)]					
Mn-54	15		130			
Fe-59	30		260			
Co-58, Co-60	15		130			
Zn-65	30		260			
Zr-95	30					
Nb-95	15					
I-131	1 [(d)]	7 x 10^{-2}		1	60	
Cs-134	15	5 x 10^{-2}	130	15	60	150
Cs-137	18	6 x 10^{-2}	150	18	80	180
Ba-140	60			60		
La-140	15			15		
Other	30 [(e)]			30 [(e)]		

TABLE NOTATIONS Applies to Table 6.3-1 & Table 6.3-2

[(a)] The LLD is defined, for purposes of these specifications, as the smallest concentration of radioactive material in a sample that will yield a new count, above system background, that will be detected with 95% probability with only 5% probability of falsely concluding that a blank observation represents a "real" signal.

For a particular measurement, which may include radiochemical separation

$LLD = (4.66S_b)/(E \cdot V \cdot 2.22 \cdot Y \cdot e^{-\lambda\Delta t})$ where:

- LLD is the lower limit of detection as defined above (picocuries per unit mass or volume)
- Sb is the standard deviation of the background counting rate or of the counting rate of a blank sample as appropriate (counts per minute)
- E is the counting efficiency (counts per disintegration)
- V is the sample size (units of mass or volume)
- 2.22 is the number of disintegrations per minute per picocurie,
- Y is the fractional radiochemical yield, when applicable,
- λ is the radioactive decay constant for the particular radionuclide, and D
- t for environmental samples is the elapsed time between sample collection, or end of the sample collection period, and time of counting

Analyses shall be performed in such a manner that the stated LLDs will be achieved under routine conditions. With typical values of E, V, Y, and Δt for the radionuclides named in the Table. Occasionally background fluctuations, unavoidably small sample sizes, the presence of interfering nuclides, or other uncontrollable circumstances may render these LLDs unachievable. In such cases, the contributing factors shall be identified and described in the Annual Radiological Environmental Report. When a radionuclide attributable to DAEC but not listed in this table is measured it shall be reported.

[b] For Drinking Water.

[c] For samples of water not used as a source of drinking water.

[d] If no drinking water pathway exists, a value of 15 pCi/l may be used.

[e] "Other" refers to non-specified gamma emitters resulting from operation of DAEC. Naturally occuring radioculides are not included.

TABLE 6.3-4					
REPORTING LEVELS FOR RADIOACTIVITY CONCENTRATIONS IN ENVIRONMENTAL SAMPLES					
Reporting Levels[(a)]					
Analysis	Water (pCi/liter)	Airborne Particulate or Gases (pCi/m^3)	Fish (pCi/Kg, wet)	Milk (pCi/liter)	Food Products (pCi/Kg, wet)
H-3	2×10^4 [(b)] 3×10^4 [(c)]				
Mn-54	1×10^3		3×10^4		
Fe-59	4×10^2		1×10^4		
Co-58	1×10^3		3×10^4		
Co-60	3×10^2		1×10^4		
Zn-65	3×10^2		2×10^4		
Zr-Nb95	4×10^2 [(c)]				
I-131	2 [d]	0.9		3	1×10^2
Cs-134	30	10	1×10^3	60	1×10^3
Cs-137	50	20	2×10^3	70	2×10^3
Ba-La140	2×10^2 [(e)]			3×10^2 [(e)]	
Other	30 [(f)]			30 [(f)]	

<u>TABLE NOTATIONS</u> Table 6.3-3

(a) The reporting level is exceeded when one or more radionuclides are detected in a sample and $\Sigma[(\text{concentration})/(\text{reporting level})] \geq 1$.

(b) For drinking water samples. This is 40 CFR Part 141 value.

(c) For samples of water not used as a source of drinking water.

(d) If no drinking water pathway exists, a value of 20 pCi/l may be used.

(e) Concentration of parent or daughter.

(f) "Other" refers to non-specified gamma emitters resulting from operation of DAEC. Naturally occurring radionuclides are not included.

BASES SECTION 6.1 / 7.1

6.1.1 and 7.1.1 Radioactive Liquid Effluent Instrumentation

The radioactive liquid effluent instrumentation is provided to monitor and control, as applicable, the release of radioactive material in liquid effluents. The FUNCTIONALITY and use of these instruments implements the requirements of 10 CFR Part 50, Appendix A, General Design Criteria 60, 63, and 64. The alarm and/or trip setpoints for these instruments are calculated in the manner described in the DODAM to assure that the alarm and/or trip will occur before ten times the values specified in 10 CFR Part 20.2001 - 20.2402, Appendix B are exceeded. Instrumentation is expected to be FUNCTIONAL and in service when required by controls. An instrument may be removed from service voluntarily for the purpose of tests, checks, or calibration, without declaring the channel non-functional.

The radwaste effluent line radiation monitor recorder RR3972 is not required in order to comply with the controls stated in the DODAM. It is provided for recording trends during a release and need not be maintained FUNCTIONAL - i.e. calibrated and functionally tested - other than during a release.

BASES SECTION 6.1 / 7.1

6.1.2 and 7.1.2 Liquid Effluent Concentration

The basic requirements concerning effluents from nuclear power are stated in 10 CFR 50.36a. These requirements indicate that compliance with Technical Specifications will keep average annual releases of radioactive material in effluents to a small percentage of the limits specified in 10 CFR 20.106. These (new 10 CFR 20.1301) requirements further indicate that operational flexibility is allowed, compatible with considerations of health and safety, which may temporarily result in releases higher than such small percentages, but still within the limits specified in the old 10 CFR 20.106. These referenced concentrations are specific values which relate to an annual average dose of 500 millirems.

As stated in the Introduction to Appendix B of the new 10 CFR 20, the liquid effluent concentrations limits, given in the appendix, are based on an annual dose of 50 millirem. Prior to the issuance of the revision to Part 20, a release concentration corresponding to a limiting dose of 500 millirem had been acceptable as a TS limit for liquid effluents. The limit was applicable at all times and has been used to provide assurance that the limits of 10 CFR 50 Appendix I were not likely to be exceeded. Since the limits of 10 CFR 50 are the ultimate value to which a license must adhere, and since the old concentration limits adequately provide such assurance, it should not be necessary to reduce this limit by a factor of 10.

Conformance with the dose limits in section 6.1.3 will necessitate average annual liquid effluent concentrations being below those specified in 10 CFR 20.1001 - 20.2402 Appendix B to §§ 20.1001-20.2402.

Demonstrating compliance with section 6.1.3 will result in a de facto demonstration of compliance with 10 CFR 20 limits.

The continuous service water sample points noted in Table 7.1-2 are adequate to ensure sampling of potential liquid radioactive effluents from the service water systems. The sample points include the General Service Water System and the RHR Service Water Systems A and B. The sample point for the RHR Service Water Systems is at a location downstream of the point where Emergency Service Water discharge joins with the RHR Service Water System, and upstream of the point where the RHRSW/ESW Rupture Disc Line branches off the RHR Service Water System. This sample point will therefore provide for sampling effluents from the RHR Service Water System, Emergency Service Water System and/or RHRSW/ESW Rupture Disc Line.

The "clean" system water sample points noted in Table 7.1-2 are adequate to ensure sampling of potential liquid radioactive effluents from systems believed to NOT contain principal gamma emitters and iodine. The sample points include the Transformer Pit, the CST Pit, FRAC tanks, and the Neut. Tank (1T022).

6.1.2 / 7.1.2 (cont'd)	The GWPP mitigation system sample points noted in Table 7.1-2 are adequate to ensure sampling of potential liquid radioactive effluents in the shallow groundwater. Further, the GWPP mitigation plan support a reduction in contaminate concentration such that releases are expected below Administrative Control Procedure (ACP) 1411.35 and Environmental Protection Agency (EPA) drinking water limit of 20,000 pCi/L.

6.1.3 and 7.1.3

1. Dose Due to Radioactive Effluents

Section 6.1.3, 6.2.3 and 6.2.4 implement the requirements of 10 CFR Part 50.36a and of 10 CFR Part 50, Appendix I, Section IV. These sections keep levels of radioactive materials in LWR effluents as low as is reasonably achievable. Compliance with these sections will also keep average releases of radioactive material to effluent at small percentages of the limits specified in 10 CFR Part 20.106. Surveillance requirements provide for the measurement of releases and calculation of doses to verify compliance with the controls. Action statements in these sections implement the requirements of 10 CFR Part 50.36(c)(2) and 10 CFR Part 50, Appendix I, Section IV.A in the event a control is not met.

2. Liquid Effluents

With the implementation of Section 6.1.3, there is reasonable assurance that Station operation will not cause a radionuclide concentration in public drinking water taken from the Cedar River that exceeds the standard for anthropogenic radioactivity in community drinking water. The equations in the DODAM for calculating doses due to measured releases of radioactive material in liquid effluent are consistent with the methodology in Regulatory Guide 1.109 and 1.113. The assessment of personal doses will examine potential exposure pathways including, as appropriate, consumption of fish and water taken from the Cedar River downstream of the discharge pipe.

6.1.4 and 7.1.4

Liquid Waste Treatment

This section implements the requirements of 10 CFR Part 50.36a (a)(1) that operating procedures be established and followed, and that equipment be maintained and used to keep releases to the environment as low as is reasonably achievable. The section intends that appropriate portions of the system which were used to establish compliance with the design objectives in 10 CFR Part 50, Appendix I, Section II be used when specified to provide reasonable assurance that releases of radioactive material in liquid effluent will be kept as low as is reasonably achievable. The components in the liquid radwaste system which are appropriate to process liquid waste in order to satisfy Section 7.1.4 are the floor drain demineralizer and the radwaste demineralizer. The activity concentration, µCi/mL, below which liquid radwaste treatment would not be cost-beneficial, and therefore not required, is demonstrated below. The quantity of radioactive material in liquid effluent released annually from the DAEC has been calculated to be:

total iodines	0.11 curie
total others (less H3)	0.25
Total	0.36 curie

The population dose commitment resulting from the radioactive material in liquid effluent released annually has been calculated to be*

thyroid		0.164 man rem
total body		0.114
	Total	0.278 man rem

Therefore, population doses are about 1.5 man rem per curie of iodine released and about 0.5 man rem per curie of other radionuclides (less H3) released in liquids. On the basis of gross activity, the population dose is about one man rem per curie released in liquids.

The volume of liquid waste processed and intended for discharge is estimated to be:

Low Purity Waste	5700 gal/day = 1.8 x 106 gal/yr
Chemical Waste	600 gal/day = 1.9 x 105 gal/yr

Since the same DAEC equipment is used to process both streams, the total volume to be processed is about 2 x 106 gal/yr. The annual cost to operate the radwaste processing equipment, based on Dirty Waste Ion Exchange operation, has been estimated* (neglecting credit for capital recovery) to be $88,000 per year. Thus the unit volume operating cost is about:

$$\frac{\frac{\$88{,}000}{yr}}{\frac{2x10^6 gal}{yr}} = \frac{\$0.05}{gal}$$

Thus, the operating cost to treat a 4000 gallon batch of chemical waste by ion exchange would be about $200. The operating cost to treat a 10000 gallon batch of floor drain waste by ion exchange would be about $500. Assuming the cost-benefit balance is $1000 expenditure per man rem reduced and assuming treatment removes all radioactivity from the liquid, then

(1) the activity concentration in a Chemical Waste batch below which treatment is not cost-beneficial is

$$C = \frac{\$200}{4000\ gal\ x\ 3785 \frac{ml}{gal}}\ x\ \frac{1\ curie}{man\ rem}\ x\ \frac{10^6\ \mu Ci}{curie}\ x\ \frac{1\ man\ rem}{\$1000}$$

$$C = 0.013 \frac{\mu Ci}{ml}$$

(continued)

6.1.4 and 7.1.4

(2) the activity concentration in a batch of Floor Drain Waste below which treatment is not cost-beneficial is

$$C = \frac{\$500}{10000\ gal \times 3785 \frac{ml}{gal}} \times \frac{1\ curie}{man\ rem} \times \frac{10^6\ \mu Ci}{curie} \times \frac{1\ man\ rem}{\$1000}$$

$$C = 0.013 \frac{\mu Ci}{ml}$$

Liquid waste treatment with the evaporator at DAEC has been shown to be neither cost-beneficial nor necessary to comply with 10 CFR 50 Appendix I, Section II. Consequently, liquid radwaste treatment to achieve an activity concentration below 0.01 µCi/mL in liquid effluent is not justified.

6.1.5 and 7.1.5 Liquid Holdup Tanks

The tanks listed in the specification include all liquid radwaste tanks (1T-88 and 1T-269) located in the Low-Level Radwaste Processing Facility (LLRPSF). Because the LLRPSF is not seismically designed, these tanks are considered as outdoor tanks that are not surrounded by liners, dikes, or walls capable of holding the tanks' contents.

Restricting the quantity of radioactive material contained in the specified tanks provides assurance that in the event of an uncontrolled release of the tanks' contents, the resulting concentrations would be less than the limits of 10 CFR 20, Appendix B to 20.1001 - 20.2402, Table 2, Column 2, at the nearest potable water supply in an UNRESTRICTED AREA.

BASES SECTION 6.2 / 7.2

6.2.1 and 7.2.1 Radioactive Gaseous Effluent Instrumentation

The radioactive gaseous effluent instrumentation is provided to monitor the release of radioactive materials in gaseous effluents and, as appropriate, to control potential releases. The presence of instruments for monitoring radioactive effluents is depicted in DODAM Figure 3-1. The FUNCTIONALITY and use of these instruments implement the requirements of 10 CFR Part 50, Appendix A, General Design Criteria 60, 63, and 64.

10-2021 DAEC is equipped with a radioactive gaseous effluent monitoring system which includes detectors at the reactor building vent (R4), the turbine building vent (R5), and the LLRPSF vent (R7). A remote indication and control unit located near each detector displays the detector reading and, whenever the setpoint is exceeded, an indicator light. The data are also routed to a control computer and a control room display, but do not cause a trip to isolate the ventilated area. In the event the control computer and/or control room display fail to function or are voluntarily taken out of service, each remote indication and control unit is designed to acquire data for up to 30 hours. It is intended that each affected remote indication and control unit display be observed at least once per 24 hours (in which case the affected channel remains FUNCTIONAL).

If an alarm trip setpoint is exceeded at the same time the control computer and/or control room display are neither functioning nor in service, alarm annunciation will still occur in the control room. In the event the detector reading and the indication of exceeding the monitor setpoint are not provided at either the control room or the remote indication and control unit, then the affected channel is not FUNCTIONAL and DAEC will either perform the appropriate ACTION or will provide an alternate monitoring system. This permits DAEC to retain the GE gaseous monitoring system as an alternate system for normal effluent monitoring when the Kaman system is temporarily non-functional. When used as an alternate monitoring system, the GE system is subject to the requirements stated in Sections 6.2.1 and 7.2.1 and to LLD requirements stated in Table 7.2-2, Note (d).

2. Not used

3. Gaseous Effluents

Assessments of dose required by Sections 7.2.3 and 7.2.4 to verify compliance with Appendix I, Section IV are based on measured radioactivity in gaseous effluent and on calculation methods stated in the DODAM. Pathways of exposure and location of individuals are selected such that the dose to a nearby resident is unlikely to be underestimated. Dose assessment methodology described in the DODAM for gaseous effluent will be consistent with the methodology in Regulatory Guides 1.109 and 1.111. Cumulative and projected assessments of dose made during a quarter are based on historical average meteorological conditions measured at DAEC. Assessment made for the annual radiological environmental report will be based on annual averages of atmospheric conditions during the period of release.

6.2.2 and 7.2.2 <u>Gaseous Effluent Concentration</u>

The basic requirements concerning effluents from nuclear power are stated in 10 CFR 50.36a. These requirements indicate that compliance with Technical Specifications will keep average annual releases of radioactive material in effluents to a small percentage of the limits specified in 10 CFR 20. These (new 10 CFR 20.1301) requirements further indicate that operational flexibility is allowed, compatible with considerations of health and safety, which may temporarily result in releases higher than such small percentages, but still within the limits specified in the old 10 CFR 20.106. These referenced concentrations are specific values which relate to an annual average dose of 500 millirems.

As stated in the Introduction to Appendix B of the new 10 CFR 20, the gaseous effluent concentration limits given in the appendix are based on an annual dose of 50 millirem for isotopes for which inhalation or ingestion is limiting or 100 millirem for isotopes for which submersion (noble gases) is limiting. Prior to the issuance of the revision to Part 20 a release concentration corresponding to limiting dose rates less than or equal to 500 mrem/yr to the whole body, 3000 mrem/yr to the skin from noble gases, and 1500 mrem/yr to any organ from iodine, and tritium, had been acceptable as a TS limit for airborne effluents. This limit was applicable at all times and had been used to provide assurance that the limits of 10 CFR 50 Appendix I and 40 CFR 190 were not likely to be exceeded.

Since the limits of 10 CFR 50 Appendix I and 40 CFR 90 are more restrictive than 10 CFR 20, and because the dose limits specified have been successfully used to assure compliance with these regulations, it should not be necessary to reduce the dose rate basis to 50 or 100 millirem.

Conformance with the dose limits of 7.3.1 will necessitate the average annual airborne effluent concentrations being below those specified in 10 CFR 20 Appendix B. Demonstrating compliance with section 6.3.1 will result in a de facto demonstration of compliance with 10 CFR 20 limits. Assessment of compliance is based upon an effluents measurement program defined in Table 7.2-2 and methodology stated in the DODAM. The resolving time of the measurements, i.e., the sample integration time, bounds the minimum averaging time of the effluent measurements waste streams.

6.2.3 and 7.2.3 <u>Doses due to Noble Gases</u>

These specifications implement the requirements of 10 CFR Part 50, Appendix I.

6.2.4 and 7.2.4 <u>Doses due to Iodine and Particulates in Air</u>

These specifications implement 10 CFR Part 50, Appendix I. The dose calculation methods in the DODAM depend on existing pathways of exposure to a member of the public or more conservative conditions assumed (yielding a higher calculated dose). Calculations and methods are such that an estimate of the dose to a member of the public is not likely to be underestimated substantially.

6.3.1 and 7.3.1 Dose

Section 6.3.1 is provided to comply with the dose limitation requirement of 40 CFR 190. This section requires the assessment of dose to demonstrate that a person (a nearby resident) has not received a radiation dose exceeding that specified in 40 CFR 190 including doses from direct radiation. There is no other licensed nuclear fuel cycle facility within 50 miles of DAEC, thus it is assumed that the dose from other uranium fuel cycle facilities is negligible. In the event a report is required to satisfy Action b, it shall be deemed adequate to satisfy the reporting requirement in Section 8.2.2.

By demonstrating compliance with 40 CFR 190, DAEC will be, de facto, in compliance with the dose limits specified in 10 CFR 20.1301 and 10 CFR 72.104. Such a position is in keeping with that stated by the NRC in the preamble to the revised 10 CFR 20 (56 CFR 23360).

6.3.2 and 7.3.2 Radiological Environmental Monitoring

The radiological environmental monitoring program, including the land use census, is conducted to satisfy the requirements of 10 CFR Part 50, Appendix I, Section IV.B.2 and .3. The minimum radiological monitoring program required by this specification provides measurements of radiation and of radioactive materials in those exposure pathways and for those radionuclides which lead to the highest potential radiation exposures of individuals resulting from the station operation. This monitoring program thereby supplements the radiological effluent monitoring program by verifying that the measurable concentrations of radioactive materials and levels of radiation are not higher than expected on the basis of the effluent measurements and modeling of the environmental exposure pathways.

10-2021 The land use census is conducted biennially to identify changes in use of the UNRESTRICTED AREA in order to recommend modifications in monitoring programs for evaluating individual doses from principal exposure pathways. It may be conducted by door-to-door survey, by aerial survey, or by consulting with local agricultural or governmental authorities.

In order that radiological environmental monitoring stations may be relocated to reflect current conditions, the locations of stations required by Table 6.3-1 are described in the Offsite Dose Assessment Manual. Revisions thereto are administered in accordance with TS 5.5.1 and DODAM 8.3.1. DAEC may conduct additional environmental monitoring exclusive of the requirements of Sections 6.3.2.

8.0 ADMINISTRATIVE CONTROLS

8.1 Programs and Manuals

8.1.1 Radioactive Effluent Controls Program

This program, conforming to 10 CFR 50.36a, provides for the control of radioactive effluents and for maintaining the doses to members of the public from radioactive effluents as low as reasonably achievable. The program shall be contained in the DODAM, shall be implemented by procedures, and shall include remedial actions to be taken whenever the program limits are exceeded. The program shall include the following elements.

a. Limitations on the functional capacity of radioactive liquid and gaseous monitoring instrumentation including surveillance tests and setpoint determination in accordance with the methodology in the DODAM;

b. Limitations on the concentrations of radioactive material released in liquid effluents from the site to UNRESTRICTED AREAs, conforming to ten times (10x) the concentrations listed in Appendix B, Table 2, Column 2 to 10 CFR 20.1001 - 20.2402;

c. Monitoring, sampling, and analysis of radioactive liquid and gaseous effluents pursuant to 10 CFR 20.1302 and with the methodology and parameters in the DODAM conforming to 10CFR50, Appendix I;

d. Limitations on the annual and quarterly doses or dose commitment to a member of the public from radioactive materials in liquid effluents released to UNRESTRICTED AREAs;

e. Determination of cumulative and projected dose contributions from radioactive effluents for the current calendar quarter and current calendar year in accordance with the methodology and parameters in the DODAM at least every 30 days. If 30-day projected quarterly dose exceeds quarterly dose limits, a utilization evaluation of liquid waste processing system is needed;

f. Limitations on the functional capability and use of the liquid and gaseous effluent treatment systems to ensure that the appropriate portions of these systems which were used to establish compliance with the design objectives in 10 CFR 50, Appendix I, Section II be used when specified to provide reasonable assurance that releases of radioactive material in liquid and gaseous effluents be kept as low as reasonably achievable;

g. Limitations on the dose rate resulting from radioactive material released in gaseous effluents to areas at or beyond the site boundary shall be limited to the following:

   1. For noble gases: less than or equal to a dose rate of 500 mrem/yr to the whole body and less than or equal to a dose rate of 3000 mrem/yr to the skin, and
   2. For iodine-131, iodine-133, tritium, and for all radionuclides in particulate form with half-lives > 8 days: less than or equal to a dose rate of 1500 mrem/yr to any organ;

h. Limitations on the annual and quarterly air doses resulting from noble gases released in gaseous effluents to areas beyond the site boundary, conforming to 10 CFR 50, Appendix I;

I. Limitations on the annual and quarterly doses to a member of the public from iodine-131, iodine-133, tritium, and all radionuclides in particulate form with half-lives > 8 days in gaseous effluents released to areas beyond the site boundary, conforming to 10 CFR 50, Appendix I; and

j. Limitations on the annual dose or dose commitment to any member of the public due to releases of radioactivity and to radiation from uranium fuel cycle sources, conforming to 40 CFR 190 and 10 CFR 72.104.

8.1.2 Radiological Environmental Monitoring Program

The Radiological Environmental Monitoring Program is described in section 5 of the DODAM. Specifications for implementation are located in section 6.3.2

8.1.3 Interlaboratory Comparison Program

Analyses shall be performed on radioactive materials supplied in an Interlaboratory Comparison Program.

In the event analyses were not performed as required in Section 8.1.3, report the corrective actions taken to prevent a recurrence in the Annual Radiological Environmental Report.

The requirement for participation in an Interlaboratory Comparison Program is provided to ensure that independent checks on the precision and accuracy of the measurements of radioactive material in environmental sample matrices are performed as part of the quality assurance program for environmental monitoring in order to demonstrate that the results are reasonably valid.

8.2 Reporting Requirements

8.2.1 Annual Radioactive Material Release Report

(1) A report of radioactive materials released from the Station shall be submitted to the NRC on or before May 1 of each year in accordance with 10 CFR 50.36a. Each report shall include the information specified in item (2) below covering the preceding twelve months.

(2) An Annual Radioactive Material Release Report shall include a summary by calendar quarter of the quantities of radioactive liquid and gaseous effluents and radioactive solid waste released from the Station. The data on radioactive liquid and gaseous effluents should be reported in the format in Tables 8.2-1 and 8.2-2. The data on radioactive solid waste should include:

1. classification of the waste (per 10 CFR Part 61)
2. total volume shipped
3. total radioactive material shipped (curies)
4. identification of principal radionuclides
5. solidification agent
6. physical description of the waste

(3) A summary description of any changes to the DODAM.

(4) A summary of meteorological data collected during the year will be submitted in the annual report following January 1. Alternatively, summary meteorological data may be retained by DAEC and made available to the NRC upon request.

(5) Include a description for all sample analyses/conditions for which communications were made to State and Local officials due to exceeding the applicable DODAM reporting levels for environmental samples for locations that have not been added to the plant's Radiological Environmental Monitoring Program.

(6) Include a description of all spills or leaks (of radioactive material) that were communicated to State/Local Stakeholders.

(continued)

TABLE 8.2-1

ANNUAL RADIOACTIVE MATERIAL RELEASE REPORT (YEAR)
LIQUID EFFLUENTS

Nuclides Released	Unit	Quarter	Quarter
strontium-89	Ci	. E	. E
strontium-90	Ci	. E	. E
cesium-134	Ci	. E	. E
cesium-137	Ci	. E	. E
iodine-131	Ci	. E	. E
cobalt-58	Ci	. E	. E
cobalt-60	Ci	. E	. E
iron-55	Ci	. E	. E
iron-59	Ci	. E	. E
zinc-65	Ci	. E	. E
manganese-54	Ci	. E	. E
chromium-51	Ci	. E	. E
zirconium-niobium-95	Ci	. E	. E
molybdenum-99	Ci	. E	. E
technetium-99m	Ci	. E	. E
barium-lanthanum-140	Ci	. E	. E
cerium-141	Ci	. E	. E
Other (specify)	Ci	. E	. E
	Ci	. E	. E
	Ci	. E	. E
	Ci	. E	. E
	Ci	. E	. E
Total for period (above)	Ci	. E	. E
xenon-133	Ci	. E	. E
xenon-135	Ci	. E	. E

TABLE 8.2-2			
ANNUAL RADIOACTIVE MATERIAL RELEASE REPORT (YEAR) GASEOUS EFFLUENTS			
Nuclides Released	Unit	Quarter	Quarter
1. Fission gases			
krypton-85	Ci	. E	. E
krypton-85m	Ci	. E	. E
krypton-87	Ci	. E	. E
krypton-88	Ci	. E	. E
xenon-133	Ci	. E	. E
xenon-135	Ci	. E	. E
xenon-135m	Ci	. E	. E
xenon-138	Ci	. E	. E
Others (specify)	Ci	. E	. E
	Ci	. E	. E
	Ci	. E	. E
Total for period	Ci	. E	. E
2. Iodines			
iodine-131	Ci	. E	. E
iodine-133	Ci	. E	. E
iodine-135	Ci	. E	. E
Others (specify)			
Total for period	Ci	. E	. E
3. Particulates			
strontium-89	Ci	. E	. E
strontium-90	Ci	. E	. E
cesium-134	Ci	. E	. E
cesium-137	Ci	. E	. E
barium-lanthanum-140	Ci	. E	. E
Others (specify)	Ci	. E	. E
	Ci	. E	. E

8.2.2 Annual Radiological Environmental Operating Report

An annual report of radiological environmental surveillance activities required by Section 6.3.2 shall be submitted to the NRC by May 15th of each year. Each report shall be consistent with the objectives outlined in the DODAM, and with 10 CFR 50, Appendix I, Sections IV.B.2, IV.B.3, and IV.C. The report shall include the following information:

(1) A summary description of the radiological environmental monitoring program required by Section 6.3.2.

(2) A map and a table of distances and directions of locations of sampling stations required in Table 6.3-1.

(3) A summary of the land use census required in Section 7.3.2.2.

(4) Results of analyses of samples required by the radiological environmental monitoring program, Table 6.3-1. In the event some results are not available, the reasons shall be explained in the report. In the event the missing results are obtained, they shall be submitted in a supplementary report as soon as is reasonable.

(5) An assessment of radiation doses to a MEMBER OF THE PUBLIC likely to be the most exposed due to radioactive liquid and gaseous effluents released from DAEC during the year. The assessment shall be performed as described in the DODAM.

(6) Deleted.

(7) Results of participation in the Interlaboratory Comparison Program.

(8) Deviation from environmental sampling schedule.

(9) A report of all analyses in which the LLD, required by Table 6.3-2, was not achieved.

(10) A report of any changes in sample locations.

(11) Include a description for all sample analyses/conditions for which communications were made to State and Local Stakeholders due to exceeding the REMP reporting levels provided in the plant's OCDM/DODAM for locations that are described in the plant's REMP or GWPP programs.

(12) Include onsite ground water sample results.

8.2.3 Special Reports

Special reports shall be submitted to the Director of Inspection and Enforcement Regional Office within the time period specified for each report. These reports shall be submitted covering the activities identified below pursuant to the requirements of the applicable reference specification.

1. Radioactive Liquid or Gaseous Effluent - calculated dose exceeding specified limit (DODAM Sections 6.1.3, 6.2.3, and 6.2.4).

2. Deleted.

3. Measured levels of radioactivity in an environmental sampling medium determined to exceed the reporting level values of DODAM Table 6.3-3 when averaged over any calendar quarter sampling period (DODAM Section 6.3.2). Report to State and Local authorities in accordance with Administrative Control Procedure (ACP) 1402.3.

4. Annual dose to a MEMBER OF THE PUBLIC determined to exceed 40 CFR Part 190 dose limit (DODAM Section 6.3.1.1).

5. Radioactive liquid waste release without treatment when activity concentration is equal to or greater than 0.01 μCi\mL (DODAM Section 6.1.4).

8.3 Changes to the DODAM

a. Shall be documented, and records of reviews performed shall be retained for the duration of the facility operating license. This documentation shall contain:
   1) Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s) and:
   2) A determination that the change will maintain the level of radioactive effluent control required by 10 CFR 20.1302, 40 CFR Part 190, 10 CFR 72.104, 10 CFR 50.36a, and Appendix I to 10 CFR Part 50 and not adversely impact the accuracy or reliability of effluent dose or setpoint calculations.

b. Shall become effective after review and acceptance by the Onsite Review Group and approval by the DAEC Decommissioning Director.

c. Shall be submitted to the commission in the form of a complete, legible copy of the entire DODAM as a part of or concurrent with the Annual Radioactive Material Release Report for the period of the report in which any change to the DODAM was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed and shall indicate the date the change was implemented.

APPENDIX A: DOSE TRANSFER FACTORS FOR AIRBORNE PATHWAYS

Environmental pathway models have been solved on the bases of unit radionuclide release in effluent (1 Ci/yr) and unit atmospheric dispersion (1 sec/m^3) or deposition (1/m^2) to derive dose transfer factors for airborne effluent. The dose transfer factors in this appendix were computed with the GASPAR II computer program[a], using default values of parameters applicable to the most exposed members of the public as recommended in Regulatory Guide 1.109, revision 1, with the following exceptions.

- Significant revisions of data since publication of the Regulatory Guide 1.109, revision 1 and incorporated into GASPAR II were employed. Data differing from those in the Regulatory Guide 1.109 are identified in GASPAR II documentation.[b]

- After publishing Reg. Guide 1.109, the NRC recommended that soil-to-plant bioaccumulation factors, B_{iv}, of cesium and strontium be changed.[c] The revised values were used to derive dose transfer factors tabulated for Sr89, Sr90, and Cs137.

- Values of environmental transit time recommended in Reg. Guide 1.109,[d] namely 1440 hr from harvest of stored vegetables to ingestion, were retained in lieu of default values in GASPAR II.[e]

These factors affected dose transfer factors more than 10% only for H3, Sr89, Sr90, Cs137, Ce141, and Ce144.

Dose transfer factors from C14 via inhalation and from Kr90 via irradiation by an airborne cloud are the same as in the previous MIDAS library since GASPAR II does not produce them. Skin dose transfer factors are assumed to be the same as total body dose transfer factors for H3 and C14 in exposure pathways involving inhalation or ingestion also because GASPAR II does not calculate them.

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[a] Strenge, D.L., et. al., GASPAR II - Technical Reference and User Guide, NUREG/CR-4653, March 1987

[b] Ibid., 3.3.1.1, 3.3.2.3

[c] USNRC, SECY-79-653A, January 30, 1980.

[d] Regulatory Guide 1.109, rev. 1, Table E-15

[e] Strenge, et. al., p. C.3.

Dose transfer factors are included hereafter for the following parameters.

Pathway	Age Group	Organ
Inhalation Ground irradiation Grass-cow-meat Vegetables, leafy + produce Plume irradiation	Adult Teenager	Total Body GI tract Kidney Thyroid Lung Skin

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = ADULT
PATHWAY = INHALATION

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m²)/Ci				
H 3	2.28E+01	2.28E+01	0.00E-01	2.28E+01	2.28E+01	2.28E+01	2.28E+01	2.28E+01
C 14	1.08E+02	1.08E+02	5.76E+02	1.08E+02	1.08E+02	1.08E+02	1.08E+02	0.00E-01
AR 41	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 83m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 87	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE131m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	3.17E+00	1.05E+02	0.00E-01	0.00E-01	7.23E-01	1.89E+00	4.56E+02	0.00E-01
MN 54	2.00E+02	2.45E+03	0.00E-01	1.25E+03	3.12E+02	0.00E-01	4.44E+04	0.00E-01
FE 55	1.25E+02	1.91E+02	7.81E+02	5.38E+02	0.00E-01	0.00E-01	2.29E+03	0.00E-01
FE 59	3.36E+02	5.96E+03	3.74E+02	8.81E+02	0.00E-01	0.00E-01	3.23E+04	0.00E-01
CO 58	6.56E+01	3.36E+03	0.00E-01	5.01E+01	0.00E-01	0.00E-01	2.94E+04	0.00E-01
CO 60	4.69E+02	9.03E+03	0.00E-01	3.64E+02	0.00E-01	0.00E-01	1.89E+05	0.00E-01
ZN 65	1.48E+03	1.70E+03	1.03E+03	3.26E+03	2.19E+03	0.00E-01	2.74E+04	0.00E-01
SR 89	2.76E+02	1.11E+04	9.63E+03	0.00E-01	0.00E-01	0.00E-01	4.44E+04	0.00E-01
SR 90	1.83E+04	2.29E+04	9.10E+05	0.00E-01	0.00E-01	0.00E-01	3.04E+05	0.00E-01
ZR 95	7.38E+02	4.75E+03	3.39E+03	1.09E+03	1.72E+03	0.00E-01	5.61E+04	0.00E-01
SB124	3.93E+02	1.29E+04	9.89E+02	1.87E+01	0.00E-01	2.40E+00	7.86E+04	0.00E-01
CS134	2.31E+04	3.30E+02	1.18E+04	2.69E+04	9.10E+03	0.00E-01	3.09E+03	0.00E-01
CS136	3.49E+03	3.71E+02	1.24E+03	4.63E+03	2.71E+03	0.00E-01	3.80E+02	0.00E-01
CS137	1.36E+04	2.66E+02	1.52E+04	1.97E+04	7.07E+03	0.00E-01	2.38E+03	0.00E-01
BA140	8.14E+01	6.91E+03	1.24E+03	1.56E+00	5.29E-01	0.00E-01	4.02E+04	0.00E-01
CE141	4.85E+01	3.80E+03	6.31E+02	4.28E+02	1.99E+02	0.00E-01	1.15E+04	0.00E-01
CE144	5.83E+03	2.59E+04	1.09E+05	4.53E+04	2.69E+04	0.00E-01	2.47E+05	0.00E-01
I 131	6.50E+02	1.99E+02	7.99E+02	1.13E+03	1.94E+03	3.77E+05	0.00E-01	0.00E-01
I 133	1.43E+02	2.81E+02	2.74E+02	4.69E+02	8.21E+02	6.81E+04	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = ADULT
PATHWAY = GROUND PLANE

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m²)/Ci				
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
AR 41	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 83m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 87	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE131m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	1.47E+05	1.47E+05	1.47E+05	1.47E+05	1.47E+05	1.47E+05	1.47E+05	1.74E+05
MN 54	4.37E+07	4.37E+07	4.37E+07	4.37E+07	4.37E+07	4.37E+07	4.37E+07	5.13E+07
FE 55	9.80E+05	2.77E+05	5.48E+06	1.61E+06	0.00E-01	0.00E-01	3.22E+06	0.00E-01
FE 59	8.65E+06	8.65E+06	8.65E+06	8.65E+06	8.65E+06	8.65E+06	8.65E+06	1.01E+07
CO 58	1.20E+07	1.20E+07	1.20E+07	1.20E+07	1.20E+07	1.20E+07	1.20E+07	1.41E+07
CO 60	6.81E+08	6.81E+08	6.81E+08	6.81E+08	6.81E+08	6.81E+08	6.81E+08	8.02E+08
ZN 65	2.36E+07	2.36E+07	2.36E+07	2.36E+07	2.36E+07	2.36E+07	2.36E+07	2.72E+07
SR 89	6.85E+02	6.85E+02	6.85E+02	6.85E+02	6.85E+02	6.85E+02	6.85E+02	7.95E+02
SR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	7.76E+06	7.76E+06	7.76E+06	7.76E+06	7.76E+06	7.76E+06	7.76E+06	9.03E+06
SB124	1.90E+07	1.90E+07	1.90E+07	1.90E+07	1.90E+07	1.90E+07	1.90E+07	2.19E+07
CS134	2.17E+08	2.17E+08	2.17E+08	2.17E+08	2.17E+08	2.17E+08	2.17E+08	2.53E+08
CS136	4.75E+06	4.75E+06	4.75E+06	4.75E+06	4.75E+06	4.75E+06	4.75E+06	5.39E+06
CS137	3.26E+08	3.26E+08	3.26E+08	3.26E+08	3.26E+08	3.26E+08	3.26E+08	3.80E+08
BA140	6.50E+05	6.50E+05	6.50E+05	6.50E+05	6.50E+05	6.50E+05	6.50E+05	7.45E+05
CE141	4.34E+05	4.34E+05	4.34E+05	4.34E+05	4.34E+05	4.34E+05	4.34E+05	4.88E+05
CE144	2.21E+06	2.21E+06	2.21E+06	2.21E+06	2.21E+06	2.21E+06	2.21E+06	2.55E+06
I 131	2.73E+05	2.73E+05	2.73E+05	2.73E+05	2.73E+05	2.73E+05	2.73E+05	3.33E+05
I 133	3.90E+04	3.90E+04	3.90E+04	3.90E+04	3.90E+04	3.90E+04	3.90E+04	4.72E+04

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = ADULT
PATHWAY = ANIMAL MEAT

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m^2)/Ci				
H 3	5.93E+00	5.93E+00	0.00E-01	5.93E+00	5.93E+00	5.93E+00	5.93E+00	5.93E+00
C 14	2.13E+03	2.13E+03	1.06E+04	2.13E+03	2.13E+03	2.13E+03	2.13E+03	2.13E+03
AR 41	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 83m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 87	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE131m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	1.99E+02	5.01E+04	0.00E-01	0.00E-01	4.40E+01	1.19E+02	2.65E+02	0.00E-01
MN 54	4.44E+04	7.10E+05	0.00E-01	2.32E+05	6.91E+04	0.00E-01	0.00E-01	0.00E-01
FE 55	7.36E+05	1.81E+06	4.58E+06	3.17E+06	0.00E-01	0.00E-01	1.77E+06	0.00E-01
FE 59	6.50E+06	5.64E+07	7.23E+06	1.70E+07	0.00E-01	0.00E-01	4.75E+06	0.00E-01
CO 58	1.08E+06	9.76E+06	0.00E-01	4.82E+05	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	4.15E+06	3.55E+07	0.00E-01	1.89E+06	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 65	1.46E+07	2.03E+07	1.01E+07	3.23E+07	2.16E+07	0.00E-01	0.00E-01	0.00E-01
SR 89	2.51E+05	1.40E+06	8.75E+06	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	3.74E+07	4.66E+07	1.85E+09	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	1.08E+04	5.04E+07	4.98E+04	1.59E+04	2.50E+04	0.00E-01	0.00E-01	0.00E-01
SB124	2.09E+05	1.50E+07	5.26E+05	9.98E+03	0.00E-01	1.28E+03	4.12E+05	0.00E-01
CS134	3.23E+07	6.91E+05	1.66E+07	3.93E+07	1.28E+07	0.00E-01	4.25E+06	0.00E-01
CS136	1.03E+06	1.63E+05	3.61E+05	1.43E+06	7.95E+05	0.00E-01	1.09E+05	0.00E-01
CS137	2.08E+07	6.15E+05	2.33E+07	3.17E+07	1.08E+07	0.00E-01	3.58E+06	0.00E-01
BA140	5.70E+04	1.79E+06	8.71E+05	1.09E+03	3.71E+02	0.00E-01	6.27E+02	0.00E-01
CE141	3.01E+01	1.01E+06	3.93E+02	2.65E+02	1.23E+02	0.00E-01	0.00E-01	0.00E-01
CE144	1.96E+03	1.24E+07	3.64E+04	1.53E+04	9.06E+03	0.00E-01	0.00E-01	0.00E-01
I 131	1.37E+05	6.31E+04	1.67E+05	2.39E+05	4.09E+05	7.83E+07	0.00E-01	0.00E-01
I 133	3.11E-03	9.16E-03	5.86E-03	1.02E-02	1.78E-02	1.50E+00	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = ADULT
PATHWAY = VEGETABLES

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m^2)/Ci				
H 3	4.12E+01	4.12E+01	0.00E-01	4.12E+01	4.12E+01	4.12E+01	4.12E+01	4.12E+01
C 14	5.74E+03	5.74E+03	2.87E+04	5.74E+03	5.74E+03	5.74E+03	5.74E+03	5.74E+03
AR 41	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 83m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 87	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE131m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	1.45E+03	3.64E+05	0.00E-01	0.00E-01	3.20E+02	8.68E+02	1.93E+03	0.00E-01
MN 54	1.86E+06	2.99E+07	0.00E-01	9.76E+06	2.90E+06	0.00E-01	0.00E-01	0.00E-01
FE 55	3.99E+05	9.84E+05	2.49E+06	1.71E+06	0.00E-01	0.00E-01	9.56E+05	0.00E-01
FE 59	3.52E+06	3.07E+07	3.93E+06	9.19E+06	0.00E-01	0.00E-01	2.57E+06	0.00E-01
CO 58	2.13E+06	1.93E+07	0.00E-01	9.51E+05	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	1.16E+07	9.92E+07	0.00E-01	5.29E+06	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 65	1.83E+07	2.55E+07	1.27E+07	4.06E+07	2.70E+07	0.00E-01	0.00E-01	0.00E-01
SR 89	9.19E+06	5.13E+07	3.20E+08	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	1.26E+09	1.57E+09	6.27E+10	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	7.89E+03	3.68E+07	3.64E+04	1.16E+04	1.83E+04	0.00E-01	0.00E-01	0.00E-01
SB124	1.27E+06	9.13E+07	3.20E+06	6.08E+04	0.00E-01	7.80E+03	2.50E+06	0.00E-01
CS134	2.87E+08	6.15E+06	1.47E+08	3.52E+08	1.13E+08	0.00E-01	3.77E+07	0.00E-01
CS136	3.83E+06	6.02E+05	1.35E+06	5.32E+06	2.96E+06	0.00E-01	4.06E+05	0.00E-01
CS137	2.06E+08	6.08E+06	2.30E+08	3.14E+08	1.07E+08	0.00E-01	3.55E+07	0.00E-01
BA140	2.67E+05	8.37E+06	4.06E+06	5.10E+03	1.74E+03	0.00E-01	2.92E+03	0.00E-01
CE141	4.72E+02	1.59E+07	6.15E+03	4.15E+03	1.93E+03	0.00E-01	0.00E-01	0.00E-01
CE144	5.36E+04	3.39E+08	9.98E+05	4.18E+05	2.48E+05	0.00E-01	0.00E-01	0.00E-01
I 131	1.05E+06	4.82E+05	1.28E+06	1.83E+06	3.14E+06	5.99E+08	0.00E-01	0.00E-01
I 133	1.75E+04	5.17E+04	3.30E+04	5.74E+04	1.00E+05	8.43E+06	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = ADULT
PATHWAY = PLUME

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m³)/(Ci sec)				
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
AR 41	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	3.14E+02
KR 83m	1.68E-03	1.68E-03	1.68E-03	1.68E-03	1.68E-03	1.68E-03	9.38E-02	4.75E-01
KR 85m	2.60E+01	2.60E+01	2.60E+01	2.60E+01	2.60E+01	2.60E+01	2.66E+01	7.67E+01
KR 85	3.58E-01	3.58E-01	3.58E-01	3.58E-01	3.58E-01	3.58E-01	9.51E-01	4.28E+01
KR 87	1.32E+02	1.32E+02	1.32E+02	1.32E+02	1.32E+02	1.32E+02	1.35E+02	4.60E+02
KR 88	3.26E+02	3.26E+02	3.26E+02	3.26E+02	3.26E+02	3.26E+02	3.26E+02	4.50E+02
KR 89	3.68E+02	3.68E+02	3.68E+02	3.68E+02	3.68E+02	3.68E+02	3.71E+02	7.48E+02
KR 90	3.46E+02	3.46E+02	3.46E+02	3.46E+02	3.46E+02	3.46E+02	3.49E+02	6.33E+02
XE131m	2.03E+00	2.03E+00	2.03E+00	2.03E+00	2.03E+00	2.03E+00	2.38E+00	1.89E+01
XE133m	5.58E+00	5.58E+00	5.58E+00	5.58E+00	5.58E+00	5.58E+00	6.02E+00	3.96E+01
XE133	6.53E+00	6.53E+00	6.53E+00	6.53E+00	6.53E+00	6.53E+00	6.85E+00	1.84E+01
XE135m	6.91E+01	6.91E+01	6.91E+01	6.91E+01	6.91E+01	6.91E+01	6.94E+01	1.05E+02
XE135	4.02E+01	4.02E+01	4.02E+01	4.02E+01	4.02E+01	4.02E+01	4.09E+01	1.06E+02
XE137	3.15E+01	3.15E+01	3.15E+01	3.15E+01	3.15E+01	3.15E+01	3.55E+01	4.25E+02
XE138	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.97E+02	3.58E+02
CR 51	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MN 54	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 55	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 59	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 58	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 65	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SB124	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS134	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS136	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA140	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = TEENAGER
PATHWAY = INHALATION

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m^3)/(Ci sec)				
H 3	2.30E+01	2.30E+01	0.00E-01	2.30E+01	2.30E+01	2.30E+01	2.30E+01	2.30E+01
C 14	1.54E+02	1.54E+02	8.24E+02	1.54E+02	1.54E+02	1.54E+02	1.54E+02	0.00E-01
AR 41	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 83m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 87	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE131m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	4.28E+00	9.51E+01	0.00E-01	0.00E-01	9.73E-01	2.38E+00	6.65E+02	0.00E-01
MN 54	2.66E+02	2.12E+03	0.00E-01	1.62E+03	4.02E+02	0.00E-01	6.27E+04	0.00E-01
FE 55	1.75E+02	2.02E+02	1.06E+03	7.55E+02	0.00E-01	0.00E-01	3.94E+03	0.00E-01
FE 59	4.53E+02	5.64E+03	5.04E+02	1.17E+03	0.00E-01	0.00E-01	4.85E+04	0.00E-01
CO 58	8.81E+01	3.02E+03	0.00E-01	6.56E+01	0.00E-01	0.00E-01	4.25E+04	0.00E-01
CO 60	6.27E+02	8.21E+03	0.00E-01	4.79E+02	0.00E-01	0.00E-01	2.76E+05	0.00E-01
ZN 65	1.98E+03	1.48E+03	1.22E+03	4.25E+03	2.74E+03	0.00E-01	3.93E+04	0.00E-01
SR 89	3.96E+02	1.18E+04	1.38E+04	0.00E-01	0.00E-01	0.00E-01	7.67E+04	0.00E-01
SR 90	2.11E+04	2.42E+04	1.05E+06	0.00E-01	0.00E-01	0.00E-01	5.23E+05	0.00E-01
ZR 95	9.98E+02	4.72E+03	4.63E+03	1.45E+03	2.14E+03	0.00E-01	8.52E+04	0.00E-01
SB124	5.32E+02	1.26E+04	1.37E+03	2.52E+01	0.00E-01	3.09E+00	1.22E+05	0.00E-01
CS134	1.74E+04	3.09E+02	1.59E+04	3.58E+04	1.19E+04	0.00E-01	4.63E+03	0.00E-01
CS136	4.34E+03	3.45E+02	1.63E+03	6.15E+03	3.49E+03	0.00E-01	5.64E+02	0.00E-01
CS137	9.86E+03	2.69E+02	2.13E+04	2.69E+04	9.63E+03	0.00E-01	3.83E+03	0.00E-01
BA140	1.12E+02	7.26E+03	1.73E+03	2.13E+00	7.23E-01	0.00E-01	6.43E+04	0.00E-01
CE141	6.88E+01	3.99E+03	9.00E+02	6.02E+02	2.81E+02	0.00E-01	1.95E+04	0.00E-01
CE144	8.33E+03	2.74E+04	1.55E+05	6.40E+04	3.83E+04	0.00E-01	4.25E+05	0.00E-01
I 131	8.37E+02	2.06E+02	1.12E+03	1.56E+03	2.66E+03	4.63E+05	0.00E-01	0.00E-01
I 133	1.97E+02	3.26E+02	3.87E+02	6.50E+02	1.14E+03	9.25E+04	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = TEENAGER
PATHWAY = GROUND PLANE

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m^2)/Ci				
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
AR 41	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 83m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 87	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE131m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	1.47E+05	1.47E+05	1.47E+05	1.47E+05	1.47E+05	1.47E+05	1.47E+05	1.74E+05
MN 54	4.37E+07	4.37E+07	4.37E+07	4.37E+07	4.37E+07	4.37E+07	4.37E+07	5.13E+07
FE 55	1.06E+06	3.10E+05	5.57E+06	1.69E+06	0.00E-01	0.00E-01	0.00E-01	3.22E+06
FE 59	8.65E+06	8.65E+06	8.65E+06	8.65E+06	8.65E+06	8.65E+06	8.65E+06	1.01E+07
CO 58	1.20E+07	1.20E+07	1.20E+07	1.20E+07	1.20E+07	1.20E+07	1.20E+07	1.41E+07
CO 60	6.81E+08	6.81E+08	6.81E+08	6.81E+08	6.81E+08	6.81E+08	6.81E+08	8.02E+08
ZN 65	2.36E+07	2.36E+07	2.36E+07	2.36E+07	2.36E+07	2.36E+07	2.36E+07	2.72E+07
SR 89	6.85E+02	6.85E+02	6.85E+02	6.85E+02	6.85E+02	6.85E+02	6.85E+02	7.95E+02
SR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	7.76E+06	7.76E+06	7.76E+06	7.76E+06	7.76E+06	7.76E+06	7.76E+06	9.03E+06
SB124	1.90E+07	1.90E+07	1.90E+07	1.90E+07	1.90E+07	1.90E+07	1.90E+07	2.19E+07
CS134	2.17E+08	2.17E+08	2.17E+08	2.17E+08	2.17E+08	2.17E+08	2.17E+08	2.53E+08
CS136	4.75E+06	4.75E+06	4.75E+06	4.75E+06	4.75E+06	4.75E+06	4.75E+06	5.39E+06
CS137	3.26E+08	3.26E+08	3.26E+08	3.26E+08	3.26E+08	3.26E+08	3.26E+08	3.80E+08
BA140	6.50E+05	6.50E+05	6.50E+05	6.50E+05	6.50E+05	6.50E+05	6.50E+05	7.45E+05
CE141	4.34E+05	4.34E+05	4.34E+05	4.34E+05	4.34E+05	4.34E+05	4.34E+05	4.88E+05
CE144	2.21E+06	2.21E+06	2.21E+06	2.21E+06	2.21E+06	2.21E+06	2.21E+06	2.55E+06
I 131	2.73E+05	2.73E+05	2.73E+05	2.73E+05	2.73E+05	2.73E+05	2.73E+05	3.33E+05
I 133	3.90E+04	3.90E+04	3.90E+04	3.90E+04	3.90E+04	3.90E+04	3.90E+04	4.72E+04

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = TEENAGER
PATHWAY = ANIMAL MEAT

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m²)/Ci				
H 3	3.55E+00	3.55E+00	0.00E-01	3.55E+00	3.55E+00	3.55E+00	3.55E+00	3.55E+00
C 14	1.80E+03	1.80E+03	9.00E+03	1.80E+03	1.80E+03	1.80E+03	1.80E+03	1.80E+03
AR 41	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 83m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 87	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE131m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	1.59E+02	2.68E+04	0.00E-01	0.00E-01	3.49E+01	8.84E+01	2.28E+02	0.00E-01
MN 54	3.52E+04	3.64E+05	0.00E-01	1.77E+05	5.29E+04	0.00E-01	0.00E-01	0.00E-01
FE 55	6.14E+05	1.13E+06	3.72E+06	2.64E+06	0.00E-01	0.00E-01	1.67E+06	0.00E-01
FE 59	5.20E+06	3.17E+07	5.77E+06	1.35E+07	0.00E-01	0.00E-01	4.25E+06	0.00E-01
CO 58	8.56E+05	5.10E+06	0.00E-01	3.71E+05	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	3.30E+06	1.91E+07	0.00E-01	1.46E+06	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 65	1.16E+07	1.05E+07	7.13E+06	2.47E+07	1.58E+07	0.00E-01	0.00E-01	0.00E-01
SR 89	2.12E+05	8.81E+05	7.38E+06	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	2.57E+07	2.93E+07	1.28E+09	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	8.62E+03	2.90E+07	3.99E+04	1.25E+04	1.84E+04	0.00E-01	0.00E-01	0.00E-01
SB124	1.68E+05	8.68E+06	4.31E+05	7.95E+03	0.00E-01	9.76E+02	3.77E+05	0.00E-01
CS134	1.44E+07	3.87E+05	1.32E+07	3.10E+07	9.86E+06	0.00E-01	3.77E+06	0.00E-01
CS136	7.48E+05	8.94E+04	2.83E+05	1.11E+06	6.05E+05	0.00E-01	9.54E+04	0.00E-01
CS137	8.94E+06	3.64E+05	1.93E+07	2.57E+07	8.75E+06	0.00E-01	3.39E+06	0.00E-01
BA140	4.63E+04	1.11E+06	7.19E+05	8.81E+02	2.99E+02	0.00E-01	5.93E+02	0.00E-01
CE141	2.52E+01	6.27E+05	3.30E+02	2.20E+02	1.03E+02	0.00E-01	0.00E-01	0.00E-01
CE144	1.65E+03	7.73E+06	3.08E+04	1.27E+04	7.61E+03	0.00E-01	0.00E-01	0.00E-01
I 131	1.05E+05	3.83E+04	1.39E+05	1.95E+05	3.36E+05	5.67E+07	0.00E-01	0.00E-01
I 133	2.54E-03	6.31E-03	4.91E-03	8.30E-03	1.46E-02	1.16E+00	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = TEENAGER
PATHWAY = VEGETABLES

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m²)/Ci				
H 3	4.72E+01	4.72E+01	0.00E-01	4.72E+01	4.72E+01	4.72E+01	4.72E+01	4.72E+01
C 14	9.32E+03	9.32E+03	4.66E+04	9.32E+03	9.32E+03	9.32E+03	9.32E+03	9.32E+03
AR 41	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 83m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 87	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE131m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	1.93E+03	3.23E+05	0.00E-01	0.00E-01	4.21E+02	1.07E+03	2.75E+03	0.00E-01
MN 54	2.81E+06	2.91E+07	0.00E-01	1.42E+07	4.21E+06	0.00E-01	0.00E-01	0.00E-01
FE 55	5.92+05	1.10E+06	3.59E+06	2.54E+06	0.00E-01	0.00E-01	1.61E+06	0.00E-01
FE 59	5.01E+06	3.07E+07	5.58E+06	1.30E+07	0.00E-01	0.00E-01	4.09E+06	0.00E-01
CO 58	3.11E+06	1.86E+07	0.00E-01	1.35E+06	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	1.77E+07	1.02E+08	0.00E-01	7.86E+06	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 65	2.75E+07	2.50E+07	1.70E+07	5.89E+07	3.77E+07	0.00E-01	0.00E-01	0.00E-01
SR 89	1.39E+07	5.80E+07	4.88E+08	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	1.66E+09	1.90E+09	8.33E+10	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	1.15E+04	3.87E+07	5.32E+04	1.68E+04	2.47E+04	0.00E-01	0.00E-01	0.00E-01
SB124	1.87E+06	9.63E+07	4.79E+06	8.81E+04	0.00E-01	1.09E+04	4.18E+06	0.00E-01
CS134	2.45E+08	6.56E+06	2.24E+08	5.29E+08	1.68E+08	0.00E-01	6.40E+07	0.00E-01
CS136	3.64E+06	4.37E+05	1.38E+06	5.42E+06	2.95E+06	0.00E-01	4.66E+05	0.00E-01
CS137	1.70E+08	6.94E+06	3.68E+08	4.88E+08	1.66E+08	0.00E-01	6.43E+07	0.00E-01
BA140	2.82E+05	6.75E+06	4.37E+06	5.36E+03	1.82E+03	0.00E-01	3.61E+03	0.00E-01
CE141	6.78E+02	1.69E+07	8.84E+03	5.89E+03	2.78E+03	0.00E-01	0.00E-01	0.00E-01
CE144	8.62E+04	4.02E+08	1.60E+06	6.62E+05	3.96E+05	0.00E-01	0.00E-01	0.00E-01
I 131	9.16E+05	3.36E+05	1.22E+06	1.70E+06	2.93E+06	4.98E+08	0.00E-01	0.00E-01
I 133	1.58E+04	3.93E+04	3.06E+04	5.20E+04	9.13E+04	7.26E+06	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = TEENAGER
PATHWAY = PLUME

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m^3)/(Ci sec)				
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
AR 41	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	3.14E+02
KR 83m	1.68E-03	1.68E-03	1.68E-03	1.68E-03	1.68E-03	1.68E-03	9.38E-02	4.75E-01
KR 85m	2.60E+01	2.60E+01	2.60E+01	2.60E+01	2.60E+01	2.60E+01	2.66E+01	7.67E+01
KR 85	3.58E-01	3.58E-01	3.58E-01	3.58E-01	3.58E-01	3.58E-01	9.51E-01	4.28E+01
KR 87	1.32E+02	1.32E+02	1.32E+02	1.32E+02	1.32E+02	1.32E+02	1.35E+02	4.60E+02
KR 88	3.26E+02	3.26E+02	3.26E+02	3.26E+02	3.26E+02	3.26E+02	3.26E+02	4.50E+02
KR 89	3.68E+02	3.68E+02	3.68E+02	3.68E+02	3.68E+02	3.68E+02	3.71E+02	7.48E+02
KR 90	3.46E+02	3.46E+02	3.46E+02	3.46E+02	3.46E+02	3.46E+02	3.49E+02	6.33E+02
XE131m	2.03E+00	2.03E+00	2.03E+00	2.03E+00	2.03E+00	2.03E+00	2.38E+00	1.89E+01
XE133m	5.58E+00	5.58E+00	5.58E+00	5.58E+00	5.58E+00	5.58E+00	6.02E+00	3.96E+01
XE133	6.53E+00	6.53E+00	6.53E+00	6.53E+00	6.53E+00	6.53E+00	6.85E+00	1.84E+01
XE135m	6.91E+01	6.91E+01	6.91E+01	6.91E+01	6.91E+01	6.91E+01	6.94E+01	1.05E+02
XE135	4.02E+01	4.02E+01	4.02E+01	4.02E+01	4.02E+01	4.02E+01	4.09E+01	1.06E+02
XE137	3.15E+01	3.15E+01	3.15E+01	3.15E+01	3.15E+01	3.15E+01	3.55E+01	4.25E+02
XE138	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.97E+02	3.58E+02
CR 51	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MN 54	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 55	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 59	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 58	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 65	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SB124	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS134	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS136	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA140	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = CHILD
PATHWAY = INHALATION

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m^3)/(Ci sec)				
H 3	2.03E+01	2.03E+01	0.00E-01	2.03E+01	2.03E+01	2.03E+01	2.03E+01	2.03E+01
C 14	2.13E+02	2.13E+02	1.14E+03	2.13E+02	2.13E+02	2.13E+02	2.13E+02	0.00E-01
AR 41	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 83m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 87	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE131m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	4.88E+00	3.42E+01	0.00E-01	0.00E-01	7.70E-01	2.71E+00	5.39E+02	0.00E-01
MN 54	3.01E+02	7.26E+02	0.00E-01	1.36E+03	3.17E+02	0.00E-01	5.01E+04	0.00E-01
FE 55	2.46E+02	9.09E+02	1.50E+03	7.97E+02	0.00E-01	0.00E-01	3.52E+03	0.00E-01
FE 59	5.29E+02	2.24E+03	6.56E+02	1.06E+03	0.00E-01	0.00E-01	4.02E+04	0.00E-01
CO 58	1.00E+02	1.09E+03	0.00E-01	5.61E+01	0.00E-01	0.00E-01	3.52E+04	0.00E-01
CO 60	7.19E+02	3.05E+03	0.00E-01	4.15E+02	0.00E-01	0.00E-01	2.24E+05	0.00E-01
ZN 65	2.23E+03	5.17E+02	1.35E+03	3.58E+03	2.26E+03	0.00E-01	3.16E+04	0.00E-01
SR 89	5.45E+02	5.29E+03	1.90E+04	0.00E-01	0.00E-01	0.00E-01	6.85E+04	0.00E-01
SR 90	2.43E+04	1.09E+04	1.22E+06	0.00E-01	0.00E-01	0.00E-01	4.69E+05	0.00E-01
ZR 95	1.17E+03	1.94E+03	6.02E+03	1.32E+03	1.89E+03	0.00E-01	7.07E+04	0.00E-01
SB124	6.34E+02	5.20E+03	1.82E+03	2.35E+01	0.00E-01	3.99E+00	1.03E+05	0.00E-01
CS134	7.13E+03	1.22E+02	2.06E+04	3.20E+04	1.05E+04	0.00E-01	3.83E+03	0.00E-01
CS136	3.68E+03	1.32E+02	2.06E+03	5.42E+03	3.03E+03	0.00E-01	4.60E+02	0.00E-01
CS137	4.06E+03	1.15E+02	2.87E+04	2.61E+04	8.94E+03	0.00E-01	3.30E+03	0.00E-01
BA140	1.37E+02	3.23E+03	2.35E+03	2.05E+00	6.69E-01	0.00E-01	5.51E+04	0.00E-01
CE141	9.19E+01	1.79E+03	1.24E+03	6.18E+02	2.71E+02	0.00E-01	1.72E+04	0.00E-01
CE144	1.15E+04	1.23E+04	2.15E+05	6.72E+04	3.71E+04	0.00E-01	3.80E+05	0.00E-01
I 131	8.65E+02	9.00E+01	1.52E+03	1.52E+03	2.50E+03	5.17E+05	0.00E-01	0.00E-01
I 133	2.44E+02	1.74E+02	5.26E+02	6.43E+02	1.07E+03	1.22E+05	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = CHILD
PATHWAY = GROUND PLANE

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m^2)/Ci				
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
AR 41	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 83m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 87	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE131m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	1.47E+05	1.47E+05	1.47E+05	1.47E+05	1.47E+05	1.47E+05	1.47E+05	1.74E+05
MN 54	4.37E+07	4.37E+07	4.37E+07	4.37E+07	4.37E+07	4.37E+07	4.37E+07	5.13E+07
FE 55	1.23E+06	3.52E+05	6.03E+06	1.98E+06	0.00E-01	0.00E-01	3.86E+06	0.00E-01
FE 59	8.65E+06	8.65E+06	8.65E+06	8.65E+06	8.65E+06	8.65E+06	8.65E+06	1.01E+07
CO 58	1.20E+07	1.20E+07	1.20E+07	1.20E+07	1.20E+07	1.20E+07	1.20E+07	1.41E+07
CO 60	6.81E+08	6.81E+08	6.81E+08	6.81E+08	6.81E+08	6.81E+08	6.81E+08	8.02E+08
ZN 65	2.36E+07	2.36E+07	2.36E+07	2.36E+07	2.36E+07	2.36E+07	2.36E+07	2.72E+07
SR 89	6.85E+02	6.85E+02	6.85E+02	6.85E+02	6.85E+02	6.85E+02	6.85E+02	7.95E+02
SR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	7.76E+06	7.76E+06	7.76E+06	7.76E+06	7.76E+06	7.76E+06	7.76E+06	9.03E+06
SB124	1.90E+07	1.90E+07	1.90E+07	1.90E+07	1.90E+07	1.90E+07	1.90E+07	2.19E+07
CS134	2.17E+08	2.17E+08	2.17E+08	2.17E+08	2.17E+08	2.17E+08	2.17E+08	2.53E+08
CS136	4.75E+06	4.75E+06	4.75E+06	4.75E+06	4.75E+06	4.75E+06	4.75E+06	5.39E+06
CS137	3.26E+08	3.26E+08	3.26E+08	3.26E+08	3.26E+08	3.26E+08	3.26E+08	3.80E+08
BA140	6.50E+05	6.50E+05	6.50E+05	6.50E+05	6.50E+05	6.50E+05	6.50E+05	7.45E+05
CE141	4.34E+05	4.34E+05	4.34E+05	4.34E+05	4.34E+05	4.34E+05	4.34E+05	4.88E+05
CE144	2.21E+06	2.21E+06	2.21E+06	2.21E+06	2.21E+06	2.21E+06	2.21E+06	2.55E+06
I 131	2.73E+05	2.73E+05	2.73E+05	2.73E+05	2.73E+05	2.73E+05	2.73E+05	3.33E+05
I 133	3.90E+04	3.90E+04	3.90E+04	3.90E+04	3.90E+04	3.90E+04	3.90E+04	4.72E+04

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = CHILD
PATHWAY = ANIMAL MEAT

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m²)/Ci				
H 3	4.28E+00	4.28E+00	0.00E-01	4.28E+00	4.28E+00	4.28E+00	4.28E+00	4.28E+00
C 14	3.39E+03	3.39E+03	1.69E+04	3.39E+03	3.39E+03	3.39E+03	3.39E+03	3.39E+03
AR 41	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 83m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 87	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE131m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	2.48E+02	1.32E+04	0.00E-01	0.00E-01	3.77E+01	1.38E+02	2.52E+02	0.00E-01
MN 54	5.39E+04	1.70E+05	0.00E-01	2.02E+05	5.67E+04	0.00E-01	0.00E-01	0.00E-01
FE 55	1.17E+06	6.99E+05	7.11E+06	3.77E+06	0.00E-01	0.00E-01	2.14E+06	0.00E-01
FE 59	8.24E+06	1.72E+07	1.02E+07	1.65E+07	0.00E-01	0.00E-01	4.79E+06	0.00E-01
CO 58	1.32E+06	2.53E+06	0.00E-01	4.34E+05	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	5.13E+06	9.63E+06	0.00E-01	1.74E+06	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 65	1.77E+07	5.01E+06	1.07E+07	2.85E+07	1.80E+07	0.00E-01	0.00E-01	0.00E-01
SR 89	3.99E+05	5.42E+05	1.40E+07	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	4.09E+07	1.82E+07	2.03E+09	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	1.38E+04	1.62E+07	7.07E+04	1.55E+04	2.22E+04	0.00E-01	0.00E-01	0.00E-01
SB124	2.73E+05	4.88E+06	7.80E+05	1.01E+04	0.00E-01	1.72E+03	4.34E+05	0.00E-01
CS134	8.05E+06	2.06E+05	2.33E+07	3.80E+07	1.18E+07	0.00E-01	4.25E+06	0.00E-01
CS136	8.68E+05	4.72E+04	4.88E+05	1.34E+06	7.13E+05	0.00E-01	1.06E+05	0.00E-01
CS137	5.01E+06	2.13E+05	3.55E+07	3.39E+07	1.11E+07	0.00E-01	3.99E+06	0.00E-01
BA140	7.76E+04	6.72E+05	1.33E+06	1.16E+03	3.80E+02	0.00E-01	6.94E+02	0.00E-01
CE141	4.60E+01	3.87E+05	6.21E+02	3.09E+02	1.36E+02	0.00E-01	0.00E-01	0.00E-01
CE144	3.10E+03	4.75E+06	5.80E+04	1.82E+04	1.01E+04	0.00E-01	0.00E-01	0.00E-01
I 131	1.47E+05	2.31E+04	2.58E+05	2.59E+05	4.25E+05	8.56E+07	0.00E-01	0.00E-01
I 133	4.25E-03	4.53E-03	9.10E-03	1.12E-02	1.88E-02	2.09E+00	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = CHILD
PATHWAY = VEGETABLES

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m^2)/Ci				
H 3	7.32E+01	7.32E+01	0.00E-01	7.32E+01	7.32E+01	7.32E+01	7.32E+01	7.32E+01
C 14	2.24E+04	2.24E+04	1.12E+05	2.24E+04	2.24E+04	2.24E+04	2.24E+04	2.24E+04
AR 41	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 83m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 87	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE131m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	3.68E+03	1.94E+05	0.00E-01	0.00E-01	5.55E+02	2.03E+03	3.71E+03	0.00E-01
MN 54	5.51E+06	1.74E+07	0.00E-01	2.07E+07	5.80E+06	0.00E-01	0.00E-01	0.00E-01
FE 55	1.41E+06	8.45E+05	8.57E+06	4.57E+06	0.00E-01	0.00E-01	2.59E+06	0.00E-01
FE 59	9.95E+06	2.08E+07	1.23E+07	2.00E+07	0.00E-01	0.00E-01	5.80E+06	0.00E-01
CO 58	6.08E+06	1.16E+07	0.00E-01	1.99E+06	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	3.52E+07	6.62E+07	0.00E-01	1.19E+07	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 65	5.39E+07	1.52E+07	3.26E+07	8.68E+07	5.45E+07	0.00E-01	0.00E-01	0.00E-01
SR 89	3.30E+07	4.47E+07	1.16E+09	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	3.39E+09	1.51E+09	1.69E+11	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	2.33E+04	2.73E+07	1.19E+05	2.62E+04	3.74E+04	0.00E-01	0.00E-01	0.00E-01
SB124	3.83E+06	6.81E+07	1.09E+07	1.42E+05	0.00E-01	2.41E+04	6.05E+06	0.00E-01
CS134	1.75E+08	4.47E+06	5.07E+08	8.30E+08	2.58E+08	0.00E-01	9.25E+07	0.00E-01
CS136	4.63E+06	2.50E+05	2.59E+06	7.13E+06	3.80E+06	0.00E-01	5.67E+05	0.00E-01
CS137	1.22E+08	5.20E+06	8.65E+08	8.27E+08	2.70E+08	0.00E-01	9.70E+07	0.00E-01
BA140	5.10E+05	4.44E+06	8.78E+06	7.67E+03	2.50E+03	0.00E-01	4.56E+03	0.00E-01
CE141	1.52E+03	1.27E+07	2.05E+04	1.02E+04	4.47E+03	0.00E-01	0.00E-01	0.00E-01
CE144	2.06E+05	3.16E+08	3.87E+06	1.21E+06	6.69E+05	0.00E-01	0.00E-01	0.00E-01
I 131	1.29E+06	2.03E+05	2.27E+06	2.28E+06	3.74E+06	7.54E+08	0.00E-01	0.00E-01
I 133	2.61E+04	2.78E+04	5.58E+04	6.91E+04	1.15E+05	1.28E+07	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = CHILD
PATHWAY = PLUME

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m^3)/(Ci sec)				
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
AR 41	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	3.14E+02
KR 83m	1.68E-03	1.68E-03	1.68E-03	1.68E-03	1.68E-03	1.68E-03	9.38E-02	4.75E-01
KR 85m	2.60E+01	2.60E+01	2.60E+01	2.60E+01	2.60E+01	2.60E+01	2.66E+01	7.67E+01
KR 85	3.58E-01	3.58E-01	3.58E-01	3.58E-01	3.58E-01	3.58E-01	9.51E-01	4.28E+01
KR 87	1.32E+02	1.32E+02	1.32E+02	1.32E+02	1.32E+02	1.32E+02	1.35E+02	4.60E+02
KR 88	3.26E+02	3.26E+02	3.26E+02	3.26E+02	3.26E+02	3.26E+02	3.26E+02	4.50E+02
KR 89	3.68E+02	3.68E+02	3.68E+02	3.68E+02	3.68E+02	3.68E+02	3.71E+02	7.48E+02
KR 90	3.46E+02	3.46E+02	3.46E+02	3.46E+02	3.46E+02	3.46E+02	3.49E+02	6.33E+02
XE131m	2.03E+00	2.03E+00	2.03E+00	2.03E+00	2.03E+00	2.03E+00	2.38E+00	1.89E+01
XE133m	5.58E+00	5.58E+00	5.58E+00	5.58E+00	5.58E+00	5.58E+00	6.02E+00	3.96E+01
XE133	6.53E+00	6.53E+00	6.53E+00	6.53E+00	6.53E+00	6.53E+00	6.85E+00	1.84E+01
XE135m	6.91E+01	6.91E+01	6.91E+01	6.91E+01	6.91E+01	6.91E+01	6.94E+01	1.05E+02
XE135	4.02E+01	4.02E+01	4.02E+01	4.02E+01	4.02E+01	4.02E+01	4.09E+01	1.06E+02
XE137	3.15E+01	3.15E+01	3.15E+01	3.15E+01	3.15E+01	3.15E+01	3.55E+01	4.25E+02
XE138	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.97E+02	3.58E+02
CR 51	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MN 54	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 55	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 59	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 58	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 65	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SB124	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS134	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS136	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA140	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = INFANT
PATHWAY = INHALATION

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m^3)/(Ci sec)				
H 3	1.17E+01	1.17E+01	0.00E-01	1.17E+01	1.17E+01	1.17E+01	1.17E+01	1.17E+01
C 14	1.68E+02	1.68E+02	8.39E+02	1.68E+02	1.68E+02	1.68E+02	1.68E+02	0.00E-01
AR 41	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 83m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 87	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE131m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	2.84E+00	1.13E+01	0.00E-01	0.00E-01	4.18E-01	1.83E+00	4.06E+02	0.00E-01
MN 54	1.58E+02	2.24E+02	0.00E-01	8.05E+02	1.58E+02	0.00E-01	3.17E+04	0.00E-01
FE 55	1.05E+02	3.47E+01	6.27E+02	3.72E+02	0.00E-01	0.00E-01	2.77E+03	0.00E-01
FE 59	3.00E+02	7.86E+02	4.31E+02	7.45E+02	0.00E-01	0.00E-01	3.23E+04	0.00E-01
CO 58	5.77E+01	3.52E+02	0.00E-01	3.87E+01	0.00E-01	0.00E-01	2.46E+04	0.00E-01
CO 60	3.74E+02	1.01E+03	0.00E-01	2.54E+02	0.00E-01	0.00E-01	1.43E+05	0.00E-01
ZN 65	9.86E+02	1.63E+03	6.12E+02	1.98E+03	1.03E+03	0.00E-01	2.05E+04	0.00E-01
SR 89	3.61E+02	2.03E+03	1.26E+04	0.00E-01	0.00E-01	0.00E-01	6.43E+04	0.00E-01
SR 90	9.89E+03	4.15E+03	4.91E+05	0.00E-01	0.00E-01	0.00E-01	3.55E+05	0.00E-01
ZR 95	6.43E+02	6.88E+02	3.64E+03	8.84E+02	9.86E+02	0.00E-01	5.55E+04	0.00E-01
SB124	3.80E+02	1.87E+03	1.20E+03	1.76E+01	0.00E-01	3.20E+00	8.40E+04	0.00E-01
CS134	2.36E+03	4.21E+01	1.25E+04	2.23E+04	6.02E+03	0.00E-01	2.53E+03	0.00E-01
CS136	1.68E+03	4.53E+01	1.53E+03	4.28E+03	1.79E+03	0.00E-01	3.74E+02	0.00E-01
CS137	1.44E+03	4.21E+01	1.74E+04	1.94E+04	5.45E+03	0.00E-01	2.26E+03	0.00E-01
BA140	9.19E+01	1.22E+03	1.77E+03	1.77E+00	4.25E-01	0.00E-01	5.07E+04	0.00E-01
CE141	6.31E+01	6.85E+02	8.78E+02	5.29E+02	1.66E+02	0.00E-01	1.64E+04	0.00E-01
CE144	5.58E+03	4.69E+03	1.01E+05	3.83E+04	1.70E+04	0.00E-01	3.12E+05	0.00E-01
I 131	6.21E+02	3.36E+01	1.20E+03	1.41E+03	1.64E+03	4.69E+05	0.00E-01	0.00E-01
I 133	1.77E+02	6.85E+01	4.18E+02	6.08E+02	7.10E+02	1.13E+05	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = INFANT
PATHWAY = GROUND PLANE

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m^2)/Ci				
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
AR 41	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 83m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 87	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE131m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	1.47E+05	1.47E+05	1.47E+05	1.47E+05	1.47E+05	1.47E+05	1.47E+05	1.74E+05
MN 54	4.37E+07	4.37E+07	4.37E+07	4.37E+07	4.37E+07	4.37E+07	4.37E+07	5.13E+07
FE 55	9.79E+05	3.84E+05	3.90E+06	1.44E+06	0.00E-01	0.00E-01	2.39E+06	0.00E-01
FE 59	8.65E+06	8.65E+06	8.65E+06	8.65E+06	8.65E+06	8.65E+06	8.65E+06	1.01E+07
CO 58	1.20E+07	1.20E+07	1.20E+07	1.20E+07	1.20E+07	1.20E+07	1.20E+07	1.41E+07
CO 60	6.81E+08	6.81E+08	6.81E+08	6.81E+08	6.81E+08	6.81E+08	6.81E+08	8.02E+08
ZN 65	2.36E+07	2.36E+07	2.36E+07	2.36E+07	2.36E+07	2.36E+07	2.36E+07	2.72E+07
SR 89	6.85E+02	6.85E+02	6.85E+02	6.85E+02	6.85E+02	6.85E+02	6.85E+02	7.95E+02
SR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	7.76E+06	7.76E+06	7.76E+06	7.76E+06	7.76E+06	7.76E+06	7.76E+06	9.03E+06
SB124	1.90E+07	1.90E+07	1.90E+07	1.90E+07	1.90E+07	1.90E+07	1.90E+07	2.19E+07
CS134	2.17E+08	2.17E+08	2.17E+08	2.17E+08	2.17E+08	2.17E+08	2.17E+08	2.53E+08
CS136	4.75E+06	4.75E+06	4.75E+06	4.75E+06	4.75E+06	4.75E+06	4.75E+06	5.39E+06
CS137	3.26E+08	3.26E+08	3.26E+08	3.26E+08	3.26E+08	3.26E+08	3.26E+08	3.80E+08
BA140	6.50E+05	6.50E+05	6.50E+05	6.50E+05	6.50E+05	6.50E+05	6.50E+05	7.45E+05
CE141	4.34E+05	4.34E+05	4.34E+05	4.34E+05	4.34E+05	4.34E+05	4.34E+05	4.88E+05
CE144	2.21E+06	2.21E+06	2.21E+06	2.21E+06	2.21E+06	2.21E+06	2.21E+06	2.55E+06
I 131	2.73E+05	2.73E+05	2.73E+05	2.73E+05	2.73E+05	2.73E+05	2.73E+05	3.33E+05
I 133	3.90E+04	3.90E+04	3.90E+04	3.90E+04	3.90E+04	3.90E+04	3.90E+04	4.72E+04

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = INFANT
PATHWAY = ANIMAL MEAT

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m²)/Ci				
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
AR 41	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 83m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 87	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE131m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MN 54	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 55	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 59	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 58	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 65	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SB124	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS134	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS136	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA140	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = INFANT
PATHWAY = VEGETABLES

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m^2)/Ci				
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
AR 41	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 83m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 87	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
KR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE131m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135m	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
XE138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MN 54	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 55	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 59	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 58	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 65	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SB124	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS134	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS136	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA140	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT AIR
Computed by GASPAR II

AGE = INFANT
PATHWAY = PLUME

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem m^3)/(Ci sec)				
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
AR 41	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	3.14E+02
KR 83m	1.68E-03	1.68E-03	1.68E-03	1.68E-03	1.68E-03	1.68E-03	9.38E-02	4.75E-01
KR 85m	2.60E+01	2.60E+01	2.60E+01	2.60E+01	2.60E+01	2.60E+01	2.66E+01	7.67E+01
KR 85	3.58E-01	3.58E-01	3.58E-01	3.58E-01	3.58E-01	3.58E-01	9.51E-01	4.28E+01
KR 87	1.32E+02	1.32E+02	1.32E+02	1.32E+02	1.32E+02	1.32E+02	1.35E+02	4.60E+02
KR 88	3.26E+02	3.26E+02	3.26E+02	3.26E+02	3.26E+02	3.26E+02	3.26E+02	4.50E+02
KR 89	3.68E+02	3.68E+02	3.68E+02	3.68E+02	3.68E+02	3.68E+02	3.71E+02	7.48E+02
KR 90	3.46E+02	3.46E+02	3.46E+02	3.46E+02	3.46E+02	3.46E+02	3.49E+02	6.33E+02
XE131m	2.03E+00	2.03E+00	2.03E+00	2.03E+00	2.03E+00	2.03E+00	2.38E+00	1.89E+01
XE133m	5.58E+00	5.58E+00	5.58E+00	5.58E+00	5.58E+00	5.58E+00	6.02E+00	3.96E+01
XE133	6.53E+00	6.53E+00	6.53E+00	6.53E+00	6.53E+00	6.53E+00	6.85E+00	1.84E+01
XE135m	6.91E+01	6.91E+01	6.91E+01	6.91E+01	6.91E+01	6.91E+01	6.94E+01	1.05E+02
XE135	4.02E+01	4.02E+01	4.02E+01	4.02E+01	4.02E+01	4.02E+01	4.09E+01	1.06E+02
XE137	3.15E+01	3.15E+01	3.15E+01	3.15E+01	3.15E+01	3.15E+01	3.55E+01	4.25E+02
XE138	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.97E+02	3.58E+02
CR 51	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MN 54	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 55	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 59	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 58	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 65	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SB124	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS134	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS136	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA140	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

APPENDIX B: TECHNICAL BASES FOR EFFECTIVE DOSE FACTORS

Overview

The evaluation of doses due to releases of radioactive material to the atmosphere can be simplified by the use of effective dose transfer factors instead of using dose factors which are radionuclide specific. These effective factors, which are based on the total radioactivity released to approximate the dose in the environment, i.e., instead of having to sum the isotopic distribution multiplied by the isotope specific dose factor only a single multiplication times the total quantity of radioactive material released would be needed. This approach provides a reasonable estimate of the actual dose while eliminating the need for a detailed calculational technique.

Determination of Effective Dose Factors

The effective dose transfer factors are based on past operating data. The radioactive effluent distribution for the past years can be used to derive single effective factors by the following equations:

$$A\gamma_{s\,eff} = \sum_i A\gamma_i \ x \ f_i$$

where

$A\gamma_{s\,eff}$ = the effective gamma-air dose factor due to stack releases of noble gases (mrad/μCi)

$A\gamma_i$ = the gamma-air dose factor due to stack releases of each noble gas radionuclide i (mrad/μCi)

f_i = the fraction of noble gas radioactivity constituted by radionuclide i

$$A\gamma_{v\,eff} = \sum_i A\gamma_{vi} \ x \ f_i$$

where

$A\gamma_{v\,eff}$ = the effective gamma-air dose factor due to vent releases of all noble gases $\left(\frac{mrad}{\mu Ci \ \sec/m^3}\right)$

$A\gamma_{vi}$ = the gamma-air dose factor due to vent releases of each noble gas radionuclide i $\left(\frac{mrad}{\mu Ci \ \sec/m^3}\right)$

$$A\beta_{v\,eff} = \sum_i A\beta_i \ x \ f_i$$

where

$A\beta_{eff}$ = the effective beta-air dose factor due to either vent or stack releases of all noble gases $\left(\frac{mrad}{\mu Ci\ \sec/m^3}\right)$

$A\beta_i$ = the beta air dose factor due to either vent or stack releases of each noble gas radionuclide i $\left(\frac{mrad}{\mu Ci\ \sec/m^3}\right)$

To determine the appropriate effective factors to be used and to evaluate the degree of variability, the atmospheric radioactive effluents for the past 3 years have been evaluated.

Table B-1 presents the radionuclide distribution for stack and vent releases as measured by isotopic analysis of periodic grab samples from the respective effluent release points. Table B-2 presents the effective dose factors (gamma-air and beta-air) derived on the basis of the radionuclide distribution.

Except for the year 1981, the variability of the effective factors is minor. For 1981, Xe-138 contributes significantly to the derivation of the effective factors for stack releases. The Xe-138 contribution for the years 1979 and 1980 is not so significant. This increase in Xe-138 from 1981 results in a larger variability of the yearly values from the average than what is considered typical. Therefore, in order to assure adequate conservatism, the effective dose factors for stack releases will be based on the radionuclide distribution for the year 1981. Because this is considered an atypical distribution resulting in higher doses, use of the data will provide dose estimates which are conservative. As more data become available to further establish a typical radionuclide distribution, the effective dose factors for stack releases may be reevaluated.

To provide an additional degree of conservatism, a factor of 0.8 is introduced into the dose calculational process when the effective dose transfer factor is used. This added conservatism provides additional assurance that the evaluation of doses by the use of a single effective factor will not significantly underestimate any actual doses in the environment.

By evaluating doses using these effective dose factors, maximum allowable releases of noble gases for any calendar quarter may be determined. As discussed in Section 3.6.1, the maximum allowable releases based on the gamma-air effective dose factor have been determined to be 250,000 Ci/quarter for stack releases and 12,700 Ci/quarter for vent releases.

For the beta air effective dose factors, the releases of noble gases corresponding to the quarterly limit of 10 mrads corresponds to 307,000 Ci/quarter for stack releases and 29,600 Ci/quarter for vent releases. Comparing these values for allowable releases with the values based on the gamma-air effective dose factors, it is demonstrated that the gamma-air doses are more restrictive than the beta-air doses. In other words, the doses calculated by using the gamma-air effective dose factors represent a larger fraction of the allowable dose than does the dose calculated by using the beta-air effective dose factors. Therefore, when using the effective dose factors for evaluating compliance with the quarterly dose limits of Section 6.2.3, only the gamma-air dose need be evaluated; compliance with the gamma-air dose limit represents a de facto compliance with the beta-air dose limit.

Reevaluation

The doses due to the gaseous effluents are evaluated by the more detailed calculational methods (i.e., use of nuclide specific dose factors) on a yearly basis. At that time, a comparison can be made between the simplified method and the detailed method to assure the overall reasonableness of this limited analysis approach. If the comparison indicates that the radionuclide distribution has changed significantly, thereby causing the simplified method to underestimate the doses, the value of the effective factors will need to be reexamined to assure the overall acceptability of this approach. However, this reexamination will only be needed if the doses as calculated by the detailed analysis exceed 50% of the design bases doses (i.e., greater than 50% of the 10 mrad gamma air dose or 20 mrad beta air dose).

Table B-1
RADIONUCLIDE DISTRIBUTION OF STACK AND VENT RELEASES

Radionuclide	Fraction of Total Releases					
	Stack			Vent		
	1979	1980	1981	1979	1980	1981
Kr-85m	.11	.05	:09	.02	---	---
Kr-87	.01	---	.02	---	.01	---
Kr-88	.07	.04	.08	---	---	---
Xe-133	.76	.82	.45	.24	.24	.14
Xe-135	.01	.02	.03	.72	.50	.59
Xe-135m	---	.02	.08	.02	.22	.21
Xe-138	.02	.06	.25	---	.03	.05

Table B-2
EFFECTIVE DOSE FACTORS NOBLE GASES - AIR DOSES

Year	Stack Releases		Vent Releases	
	Gamma-Air Effective Dose Factor $A\gamma_{seff}$ $\left(\frac{mrad}{\mu Ci}\right)$	Beta-Air Effective Dose Factor $A\beta_{seff}$ $\left(\frac{mrad}{\mu Ci\ \sec/m^3}\right)$	Gamma-Air Effective Dose Factor $A\gamma_{veff}$ $\left(\frac{mrad}{\mu Ci\ \sec/m^3}\right)$	Beta-Air Effective Dose Factor $A\beta_{veff}$ $\left(\frac{mrad}{\mu Ci\ \sec/m^3}\right)$
1979	7.0×10^{-12}	5.9×10^{-5}	5.0×10^{-5}	6.5×10^{-5}
1980	6.7×10^{-12}	5.3×10^{-5}	6.7×10^{-5}	6.0×10^{-5}
1981	1.6×10^{-11}	9.3×10^{-12}	6.4×10^{-5}	6.3×10^{-5}
Average	9.9×10^{-12}	6.8×10^{-5}	6.4×10^{-5}	6.3×10^{-5}

APPENDIX C: DOSE TRANSFER FACTORS FOR WATERBORNE PATHWAYS

Dose transfer factors for waterborne effluent have been derived by solving environmental pathway models on the bases of unit radionuclide release in effluent (1 Ci/yr) discharged in 1 gallon/minute of water. The dose transfer factors in this appendix were computed with the LADTAP II computer program, using default values of parameters applicable to the most exposed members of the public as recommended in Regulatory 1.109, revision 1, with the following exceptions:

- In order to account for significant revisions of data since publication of the Regulatory Guide, data differing from those in Regulatory Guide 1.109, revision 1 are identified in LADTAP II documentation.[15]
- After publishing Reg. Guide 1.109, the NRC recommended that soil-to-plant bioaccumulation factors, B_{iv}, of cesium and strontium be changed.[16]
- The revised values were used to derive dose transfer factors tabulated for Sr89, Sr90, and Cs137 in irrigated vegetation.
- Values of environmental transit time recommended in Reg. Guide 1.109[17], namely 1440 hr from harvest of stored vegetables to ingestion, were retained.
- LADTAP II divergence from Reg. Guide 1.109 is reflected in tritium dose transfer factors that are typically 43% lower than those described in the Reg. Guide.

Dose transfer factors are included hereafter for the following parameters. Only those pathways applicable at the time of a radioactive liquid effluent release will be used for dose calculations. Likely pathways would include potable water, freshwater fish and irrigated fresh leafy vegetables (including strawberries).

15 Strenge, D.L., et. al., LADTAP II - Technical Reference and User guide, NUREG/CR-4013, April 1986.
16 NRC, SECY-79-653A, January 30, 1980.
17 Regulatory Guide 1.109, rev. 1, Table E-15.

10-2021

Pathway	Age Group	Organ
Potable water Freshwater fish Animal drinks river water-milk Animal drinks river water-meat River shoreline deposits-irradiation Swimming Boating	Adult Infant	Total Body GI tract

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = ADULT
PATHWAY = POTABLE WATER

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	2.19E+01	2.19E+01	0.00E-01	2.19E+01	2.19E+01	2.19E+01	2.19E+01	0.00E-01
C 14	2.08E+02	2.08E+02	1.04E+03	2.08E+02	2.08E+02	2.08E+02	2.08E+02	0.00E-01
NA 24	2.06E+02	2.06E+02	2.06E+02	2.06E+02	2.06E+02	2.06E+02	2.06E+02	0.00E-01
P 32	2.60E+03	7.57E+03	6.73E+04	4.19E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	9.50E-01	2.39E+02	0.00E-01	0.00E-01	2.09E-01	5.68E-01	1.26E+00	0.00E-01
MN 54	3.19E+02	5.12E+03	0.00E-01	1.67E+03	4.97E+02	0.00E-01	0.00E-01	0.00E-01
MN 56	1.15E-02	2.06E+00	0.00E-01	6.46E-02	8.20E-02	0.00E-01	0.00E-01	0.00E-01
FE 55	1.62E+02	3.99E+02	1.01E+03	6.95E+02	0.00E-01	0.00E-01	3.88E+02	0.00E-01
FE 59	1.41E+03	1.23E+04	1.56E+03	3.68E+03	0.00E-01	0.00E-01	1.03E+03	0.00E-01
CO 58	6.06E+02	5.48E+03	0.00E-01	2.70E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	1.73E+03	1.47E+04	0.00E-01	7.84E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 63	1.60E+03	6.89E+02	4.76E+04	3.30E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	1.56E-02	8.66E-01	2.63E-01	3.42E-02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CU 64	3.85E+00	6.99E+02	0.00E-01	8.21E+00	2.07E+01	0.00E-01	0.00E-01	0.00E-01
ZN 65	2.54E+03	3.54E+03	1.77E+03	5.62E+03	3.76E+03	0.00E-01	0.00E-01	0.00E-01
ZN 69	1.21E-08	2.62E-08	9.11E-08	1.74E-07	1.13E-07	0.00E-01	0.00E-01	0.00E-01
BR 83	1.40E-02	2.02E-02	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 84	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 86	3.47E+03	1.47E+03	0.00E-01	7.45E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	3.19E+03	1.78E+04	1.11E+05	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	6.41E+04	8.02E+04	3.19E+06	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 91	1.45E+01	1.71E+03	3.59E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 92	7.38E-02	3.38E+01	1.71E+00	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 90	7.29E-02	2.88E+04	2.72E+00	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91	1.37E+00	2.81E+04	5.10E+01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 92	8.37E-05	5.02E+01	2.86E-03	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 93	5.20E-03	5.98E+03	1.89E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	2.39E+00	1.12E+04	1.10E+01	3.53E+00	5.54E+00	0.00E-01	0.00E-01	0.00E-01
ZR 97	2.12E-02	1.44E+04	2.30E-01	4.64E-02	7.00E-02	0.00E-01	0.00E-01	0.00E-01
NB 95	6.68E-01	7.54E+03	2.23E+00	1.24E+00	1.23E+00	0.00E-01	0.00E-01	0.00E-01
MO 99	2.33E+02	2.84E+03	0.00E-01	1.23E+03	2.78E+03	0.00E-01	0.00E-01	0.00E-01
TC 99M	2.05E-01	9.53E+00	5.70E-03	1.61E-02	2.45E-01	0.00E-01	7.89E-03	0.00E-01
TC101	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = ADULT
PATHWAY = POTABLE WATER

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
RU103	2.87E+01	7.77E+03	6.66E+01	0.00E-01	2.54E+02	0.00E-01	0.00E-01	0.00E-01
RU105	5.24E-02	8.12E+01	1.33E-01	0.00E-01	1.72E+00	0.00E-01	0.00E-01	0.00E-01
RU106	1.27E+02	6.51E+04	1.00E+03	0.00E-01	1.94E+03	0.00E-01	0.00E-01	0.00E-01
AG110M	3.21E+01	2.21E+04	5.85E+01	5.41E+01	1.06E+02	0.00E-01	0.00E-01	0.00E-01
TE125M	1.30E+02	3.87E+03	9.70E+02	3.51E+02	3.94E+03	2.92E+02	0.00E-01	0.00E-01
TE127M	3.00E+02	8.26E+03	2.46E+03	8.81E+02	1.00E+04	6.30E+02	0.00E-01	0.00E-01
TE127	1.47E+00	5.36E+02	6.80E+00	2.44E+00	2.77E+01	5.03E+00	0.00E-01	0.00E-01
TE129M	6.53E+02	2.08E+04	4.13E+03	1.54E+03	1.72E+04	1.42E+03	0.00E-01	0.00E-01
TE129	1.66E-06	5.13E-06	6.79E-06	2.55E-06	2.85E-05	5.21E-06	0.00E-01	0.00E-01
TE131M	1.48E+02	1.77E+04	3.64E+02	1.78E+02	1.80E+03	2.82E+02	0.00E-01	0.00E-01
TE131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE132	4.53E+02	2.28E+04	7.46E+02	4.83E+02	4.65E+03	5.33E+02	0.00E-01	0.00E-01
I 130	8.37E+01	1.83E+02	7.19E+01	2.12E+02	3.31E+02	1.80E+04	0.00E-01	0.00E-01
I 131	1.15E+03	5.28E+02	1.40E+03	2.00E+03	3.43E+03	6.55E+05	0.00E-01	0.00E-01
I 132	5.03E-02	2.70E-02	5.38E-02	1.44E-01	2.29E-01	5.03E+00	0.00E-01	0.00E-01
I 133	1.24E+02	3.66E+02	2.34E+02	4.07E+02	7.10E+02	5.98E+04	0.00E-01	0.00E-01
I 134	2.10E-07	5.11E-10	2.16E-07	5.86E-07	9.32E-07	1.02E-05	0.00E-01	0.00E-01
I 135	1.26E+01	3.85E+01	1.30E+01	3.41E+01	5.47E+01	2.25E+03	0.00E-01	0.00E-01
CS134	4.43E+04	9.48E+02	2.28E+04	5.42E+04	1.75E+04	0.00E-01	5.82E+03	0.00E-01
CS136	6.43E+03	1.01E+03	2.26E+03	8.93E+03	4.97E+03	0.00E-01	6.81E+02	0.00E-01
CS137	2.61E+04	7.73E+02	2.92E+04	3.99E+04	1.35E+04	0.00E-01	4.50E+03	0.00E-01
CS138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA139	5.81E-06	3.52E-04	1.98E-04	1.41E-07	1.32E-07	0.00E-01	8.02E-08	0.00E-01
BA140	4.61E+02	1.45E+04	7.04E+03	8.85E+00	3.01E+00	0.00E-01	5.07E+00	0.00E-01
BA141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA142	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA140	8.07E-02	2.24E+04	6.06E-01	3.05E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA142	1.08E-07	3.18E-03	9.57E-07	4.35E-07	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	2.57E-01	8.68E+03	3.36E+00	2.27E+00	1.05E+00	0.00E-01	0.00E-01	0.00E-01
CE143	2.99E-02	1.01E+04	3.66E-01	2.70E+02	1.19E-01	0.00E-01	0.00E-01	0.00E-01
CE144	9.57E+00	6.03E+04	1.78E+02	7.46E+01	4.42E+01	0.00E-01	0.00E-01	0.00E-01
PR143	1.59E-01	1.40E+04	3.20E+00	1.28E+00	7.42E-01	0.00E-01	0.00E-01	0.00E-01
PR144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ND147	1.50E-01	1.20E+04	2.16E+00	2.50E+00	1.46E+00	0.00E-01	0.00E-01	0.00E-01
W 187	5.50E+00	5.15E+03	1.88E+01	1.57E+01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NP239	1.76E-02	6.55E+03	3.25E-01	3.20E-02	9.97E-02	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = ADULT
PATHWAY = FRESH WATER FISH

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	5.68E-01	5.68E-01	0.00E-01	5.68E-01	5.68E-01	5.68E-01	5.68E-01	0.00E-01
C 14	2.75E+04	2.75E+04	1.38E+05	2.75E+04	2.75E+04	2.75E+04	2.75E+04	0.00E-01
NA 24	5.90E+02	5.90E+02	5.90E+02	5.90E+02	5.90E+02	5.90E+02	5.90E+02	0.00E-01
P 32	7.49E+06	2.18E+07	1.94E+08	1.20E+07	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	5.47E+00	1.37E+03	0.00E-01	0.00E-01	1.20E+00	3.27E+00	7.26E+00	0.00E-01
MN 54	3.67E+03	5.89E+04	0.00E-01	1.92E+04	5.72E+03	0.00E-01	0.00E-01	0.00E-01
MN 56	1.28E-01	2.31E+01	0.00E-01	7.24E-01	9.19E-01	0.00E-01	0.00E-01	0.00E-01
FE 55	4.66E+02	1.15E+03	2.90E+03	2.00E+03	0.00E-01	0.00E-01	1.12E+03	0.00E-01
FE 59	4.06E+03	3.53E+04	4.50E+03	1.06E+04	0.00E-01	0.00E-01	2.96E+03	0.00E-01
CO 58	8.71E+02	7.88E+03	0.00E-01	3.89E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	2.49E+03	2.12E+04	0.00E-01	1.13E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 63	4.59E+03	1.98E+03	1.37E+05	9.49E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	4.36E-02	2.42E+00	7.36E-01	9.56E-02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CU 64	5.51E+00	1.00E+03	0.00E-01	1.17E+01	2.96E+01	0.00E-01	0.00E-01	0.00E-01
ZN 65	1.46E+05	2.04E+05	1.02E+05	3.24E+05	2.16E+05	0.00E-01	0.00E-01	0.00E-01
ZN 69	6.48E-07	1.40E-06	4.87E-06	9.32E-06	6.05E-06	0.00E-01	0.00E-01	0.00E-01
BR 83	1.65E-01	2.37E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 84	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 86	2.00E+05	8.45E+04	0.00E-01	4.28E+05	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	2.76E+03	1.54E+04	9.60E+04	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	5.53E+04	6.92E+04	2.75E+06	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 91	1.24E+01	1.47E+03	3.08E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 92	6.21E-02	2.85E+01	1.44E+00	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 90	5.24E-02	2.07E+04	1.95E+00	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91	9.81E-01	2.02E+04	3.67E+01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 92	5.90E-05	3.54E+01	2.02E-03	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 93	3.72E-03	4.27E+03	1.35E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	2.27E-01	1.06E+03	1.05E+00	3.35E-01	5.26E-01	0.00E-01	0.00E-01	0.00E-01
ZR 97	2.00E-03	1.36E+03	2.17E-02	4.38E-03	6.62E-03	0.00E-01	0.00E-01	0.00E-01
NB 95	5.76E+02	6.51E+06	1.93E+03	1.07E+03	1.06E+03	0.00E-01	0.00E-01	0.00E-01
MO 99	6.71E+01	8.17E+02	0.00E-01	3.52E+02	7.98E+02	0.00E-01	0.00E-01	0.00E-01
TC 99M	8.75E-02	4.07E+00	2.43E-03	6.87E-03	1.04E-01	0.00E-01	3.37E-03	0.00E-01
TC101	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = ADULT
PATHWAY = FRESH WATER FISH

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
RU103	8.25E+00	2.24E+03	1.91E+01	0.00E-01	7.31E+01	0.00E-01	0.00E-01	0.00E-01
RU105	1.48E-02	2.30E+01	3.76E-02	0.00E-01	4.86E-01	0.00E-01	0.00E-01	0.00E-01
RU106	3.66E+01	1.87E+04	2.89E+02	0.00E-01	5.58E+02	0.00E-01	0.00E-01	0.00E-01
AG110M	2.12E+00	1.46E+03	3.87E+00	3.58E+00	7.03E+00	0.00E-01	0.00E-01	0.00E-01
TE125M	1.49E+03	4.46E+04	1.12E+04	4.04E+03	4.54E+04	3.36E+03	0.00E-01	0.00E-01
TE127M	3.45E+03	9.51E+04	2.83E+04	1.01E+04	1.15E+05	7.25E+03	0.00E-01	0.00E-01
TE127	1.68E+01	6.12E+03	7.76E+01	2.79E+01	3.16E+02	5.75E+01	0.00E-01	0.00E-01
TE129M	7.51E+03	2.39E+05	4.75E+04	1.77E+04	1.98E+05	1.63E+04	0.00E-01	0.00E-01
TE129	1.79E-05	5.56E-05	7.36E-05	2.77E-05	3.09E-04	5.65E-05	0.00E-01	0.00E-01
TE131M	1.70E+03	2.03E+05	4.18E+03	2.04E+03	2.07E+04	3.24E+03	0.00E-01	0.00E-01
TE131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE132	5.21E+03	2.62E+05	8.58E+03	5.55E+03	5.35E+04	6.13E+03	0.00E-01	0.00E-01
I 130	3.59E+01	7.84E+01	3.09E+01	9.11E+01	1.42E+02	7.72E+03	0.00E-01	0.00E-01
I 131	4.94E+02	2.28E+02	6.03E+02	8.62E+02	1.48E+03	2.83E+05	0.00E-01	0.00E-01
I 132	2.11E-02	1.13E-02	2.25E-02	6.02E-02	9.60E-02	2.11E+00	0.00E-01	0.00E-01
I 133	5.33E+01	1.57E+02	1.01E+02	1.75E+02	3.05E+02	2.57E+04	0.00E-01	0.00E-01
I 134	8.36E-08	2.04E-10	8.60E-08	2.34E-07	3.72E-07	4.05E-06	0.00E-01	0.00E-01
I 135	5.37E+00	1.64E+01	5.56E+00	1.46E+01	2.33E+01	9.60E+02	0.00E-01	0.00E-01
CS134	2.55E+06	5.45E+04	1.31E+06	3.12E+06	1.01E+06	0.00E-01	3.35E+05	0.00E-01
CS136	3.70E+05	5.84E+04	1.30E+05	5.14E+05	2.86E+05	0.00E-01	3.92E+04	0.00E-01
CS137	1.50E+06	4.45E+04	1.68E+06	2.30E+06	7.80E+05	0.00E-01	2.59E+05	0.00E-01
CS138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA139	6.35E-07	3.85E-05	2.17E-05	1.55E-08	1.45E-08	0.00E-01	8.77E-09	0.00E-01
BA140	5.31E+01	1.67E+03	8.10E+02	1.02E+00	3.46E-01	0.00E-01	5.83E-01	0.00E-01
BA141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA142	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA140	5.80E-02	1.61E+04	4.35E-01	2.19E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA142	7.45E-08	2.18E-03	6.58E-07	2.99E-07	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	7.41E-03	2.50E+02	9.65E-02	6.53E-02	3.03E-02	0.00E-01	0.00E-01	0.00E-01
CE143	8.59E-04	2.90E+02	1.05E-02	7.77E+00	3.42E-03	0.00E-01	0.00E-01	0.00E-01
CE144	2.75E-01	1.73E+03	5.13E+00	2.14E+00	1.27E+00	0.00E-01	0.00E-01	0.00E-01
PR143	1.14E-01	1.01E+04	2.30E+00	9.24E-01	5.33E-01	0.00E-01	0.00E-01	0.00E-01
PR144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ND147	1.08E-01	8.63E+03	1.56E+00	1.80E+00	1.05E+00	0.00E-01	0.00E-01	0.00E-01
W 187	1.89E+02	1.77E+05	6.48E+02	5.41E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NP239	5.06E-03	1.88E+03	9.34E-02	9.18E-03	2.86E-02	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = ADULT
PATHWAY = ANIMAL DRINKING WATER—MEAT

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	1.65E+00	1.65E+00	0.00E-01	1.65E+00	1.65E+00	1.65E+00	1.65E+00	0.00E-01
C 14	4.86E+01	4.86E+01	2.43E+02	4.86E+01	4.86E+01	4.86E+01	4.86E+01	0.00E-01
NA 24	3.33E-08	3.33E-08	3.33E-08	3.33E-08	3.33E-08	3.33E-08	3.33E-08	0.00E-01
P 32	3.58E+02	1.04E+03	9.27E+03	5.77E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	1.07E-02	2.68E+00	0.00E-01	0.00E-01	2.35E-03	6.37E-03	1.42E-02	0.00E-01
MN 54	1.84E+00	2.96E+01	0.00E-01	9.65E+00	2.87E+00	0.00E-01	0.00E-01	0.00E-01
MN 56	3.01E-28	5.42E-26	0.00E-01	1.70E-27	2.16E-27	0.00E-01	0.00E-01	0.00E-01
FE 55	4.82E+01	1.19E+02	2.99E+02	2.07E+02	0.00E-01	0.00E-01	1.15E+02	0.00E-01
FE 59	3.16E+02	2.75E+03	3.51E+02	8.24E+02	0.00E-01	0.00E-01	2.30E+02	0.00E-01
CO 58	4.93E+01	4.45E+02	0.00E-01	2.20E+01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	1.68E+02	1.43E+03	0.00E-01	7.62E+01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 63	6.37E+02	2.75E+02	1.90E+04	1.32E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	3.05E-26	1.69E-24	5.14E-25	6.67E-26	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CU 64	3.20E-12	5.81E-10	0.00E-01	6.82E-12	1.72E-11	0.00E-01	0.00E-01	0.00E-01
ZN 65	5.44E+02	7.59E+02	3.78E+02	1.20E+03	8.06E+02	0.00E-01	0.00E-01	0.00E-01
ZN 69	4.78E-28	1.03E-27	3.60E-27	6.88E-27	4.47E-27	0.00E-01	0.00E-01	0.00E-01
BR 83	1.89E-26	2.72E-26	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 84	8.86E-27	6.96E-32	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 85	6.73E-34	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 86	4.00E+02	1.69E+02	0.00E-01	8.59E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 88	2.34E-27	6.10E-38	0.00E-01	4.41E-27	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 89	1.38E-27	1.14E-40	0.00E-01	1.97E-27	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	1.11E+01	6.21E+01	3.87E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	2.89E+02	3.62E+02	1.44E+04	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 91	2.06E-16	2.42E-14	5.09E-15	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 92	1.04E-27	4.78E-25	2.41E-26	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 90	1.82E-05	7.18E+00	6.77E-04	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91M	1.69E-31	1.28E-29	4.37E-30	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91	3.77E-02	7.77E+02	1.41E+00	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 92	2.26E-30	1.35E-24	7.73E-29	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 93	4.07E-18	4.67E-12	1.47E-16	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	4.99E-01	2.33E+03	2.30E+00	7.37E-01	1.16E+00	0.00E-01	0.00E-01	0.00E-01
ZR 97	3.89E-11	2.63E-05	4.21E-10	8.50E-11	1.28E-10	0.00E-01	0.00E-01	0.00E-01
NB 95	9.69E-01	1.09E+04	3.24E+00	1.80E+00	1.78E+00	0.00E-01	0.00E-01	0.00E-01
MO 99	1.15E-01	1.40E+00	0.00E-01	6.06E-01	1.37E+00	0.00E-01	0.00E-01	0.00E-01
TC 99M	8.45E-24	3.93E-22	2.35E-25	6.64E-25	1.01E-23	0.00E-01	3.25E-25	0.00E-01
TC101	1.85E-27	5.68E-40	1.31E-28	1.89E-28	3.40E-27	0.00E-01	9.65E-29	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = ADULT
PATHWAY = ANIMAL DRINKING WATER--MEAT

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
RU103	6.18E+01	1.67E+04	1.43E+02	0.00E-01	5.47E+02	0.00E-01	0.00E-01	0.00E-01
RU105	5.03E-26	7.79E-23	1.27E-25	0.00E-01	1.65E-24	0.00E-01	0.00E-01	0.00E-01
RU106	3.70E+02	1.89E+05	2.92E+03	0.00E-01	5.64E+03	0.00E-01	0.00E-01	0.00E-01
AG110M	3.90E+00	2.68E+03	7.10E+00	6.57E+00	1.29E+01	0.00E-01	0.00E-01	0.00E-01
TE125M	6.01E+01	1.79E+03	4.48E+02	1.62E+02	1.82E+03	1.35E+02	0.00E-01	0.00E-01
TE127M	1.54E+02	4.25E+03	1.27E+03	4.53E+02	5.14E+03	3.24E+02	0.00E-01	0.00E-01
TE127	1.63E-15	5.95E-13	7.54E-15	2.71E-15	3.07E-14	5.58E-15	0.00E-01	0.00E-01
TE129M	2.56E+02	8.13E+03	1.62E+03	6.03E+02	6.74E+03	5.55E+02	0.00E-01	0.00E-01
TE129	7.83E-27	2.43E-26	3.21E-26	1.21E-26	1.35E-25	2.47E-26	0.00E-01	0.00E-01
TE131M	2.23E-03	2.65E-01	5.46E-03	2.67E-03	2.71E-02	4.23E-03	0.00E-01	0.00E-01
TE131	2.20E-27	9.87E-28	6.97E-27	2.91E-27	3.06E-26	5.73E-27	0.00E-01	0.00E-01
TE132	4.59E+00	2.31E+02	7.57E+00	4.89E+00	4.71E+01	5.40E+00	0.00E-01	0.00E-01
I 130	1.31E-11	2.85E-11	1.12E-11	3.31E-11	5.17E-11	2.81E-09	0.00E-01	0.00E-01
I 131	4.85E+00	2.23E+00	5.91E+00	8.46E+00	1.45E+01	2.77E+03	0.00E-01	0.00E-01
I 132	9.85E-27	5.29E-27	1.05E-26	2.81E-26	4.48E-26	9.85E-25	0.00E-01	0.00E-01
I 133	6.67E-07	1.96E-06	1.26E-06	2.19E-06	3.82E-06	3.21E-04	0.00E-01	0.00E-01
I 134	3.27E-27	7.97E-30	3.36E-27	9.14E-27	1.45E-26	1.58E-25	0.00E-01	0.00E-01
I 135	3.76E-22	1.15E-21	3.90E-22	1.02E-21	1.64E-21	6.73E-20	0.00E-01	0.00E-01
CS134	1.31E+03	2.81E+01	6.74E+02	1.60E+03	5.19E+02	0.00E-01	1.72E+02	0.00E-01
CS136	7.08E+01	1.12E+01	2.49E+01	9.83E+01	5.47E+01	0.00E-01	7.50E+00	0.00E-01
CS137	7.87E+02	2.33E+01	8.79E+02	1.20E+03	4.08E+02	0.00E-01	1.36E+02	0.00E-01
CS138	1.44E-27	1.24E-32	1.47E-27	2.90E-27	2.13E-27	0.00E-01	2.11E-28	0.00E-01
BA139	1.33E-28	8.04E-27	4.53E-27	3.23E-30	3.02E-30	0.00E-01	1.83E-30	0.00E-01
BA140	3.97E+00	1.25E+02	6.05E+01	7.60E-02	2.58E-02	0.00E-01	4.35E-02	0.00E-01
BA141	1.26E-29	1.76E-37	3.74E-28	2.83E-31	2.63E-31	0.00E-01	1.61E-31	0.00E-01
BA142	2.12E-30	0.00E-01	3.37E-29	3.47E-32	2.93E-32	0.00E-01	1.96E-32	0.00E-01
LA140	4.67E-08	1.30E-02	3.51E-07	1.77E-07	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA142	4.47E-32	1.31E-27	3.94E-31	1.79E-31	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	1.55E-03	5.23E+01	2.02E-02	1.37E-02	6.35E-03	0.00E-01	0.00E-01	0.00E-01
CE143	1.91E-08	6.45E-03	2.34E-07	1.73E-04	7.60E-08	0.00E-01	0.00E-01	0.00E-01
CE144	8.26E-02	5.20E+02	1.54E+00	6.43E-01	3.82E-01	0.00E-01	0.00E-01	0.00E-01
PR143	2.13E-03	1.88E+02	4.30E-02	1.72E-02	9.95E-03	0.00E-01	0.00E-01	0.00E-01
PR144	1.56E-32	4.42E-38	3.08E-31	1.28E-31	7.20E-32	0.00E-01	0.00E-01	0.00E-01
ND147	1.13E-03	9.09E+01	1.64E-02	1.89E-02	1.11E-02	0.00E-01	0.00E-01	0.00E-01
W 187	9.54E-08	8.94E-05	3.26E-07	2.73E-07	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NP239	9.88E-08	3.67E-02	1.82E-06	1.79E-07	5.59E-07	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = ADULT
PATHWAY = RIVER SHORELINE DEPOSITS

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NA 24	1.88E+00	1.88E+00	1.88E+00	1.88E+00	1.88E+00	1.88E+00	1.88E+00	2.18E+00
P 32	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	7.33E-01	7.33E-01	7.33E-01	7.33E-01	7.33E-01	7.33E-01	7.33E-01	8.66E-01
MN 54	2.18E+02	2.18E+02	2.18E+02	2.18E+02	2.18E+02	2.18E+02	2.18E+02	2.56E+02
MN 56	1.38E-01	1.38E-01	1.38E-01	1.38E-01	1.38E-01	1.38E-01	1.38E-01	1.63E-01
FE 55	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 59	4.29E+01	4.29E+01	4.29E+01	4.29E+01	4.29E+01	4.29E+01	4.29E+01	5.04E+01
CO 58	5.98E+01	5.98E+01	5.98E+01	5.98E+01	5.98E+01	5.98E+01	5.98E+01	7.01E+01
CO 60	3.38E+03	3.38E+03	3.38E+03	3.38E+03	3.38E+03	3.38E+03	3.38E+03	3.98E+03
NI 63	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	4.55E-02	4.55E-02	4.55E-02	4.55E-02	4.55E-02	4.55E-02	4.55E-02	5.29E-02
CU 64	9.48E-02	9.48E-02	9.48E-02	9.48E-02	9.48E-02	9.48E-02	9.48E-02	1.07E-01
ZN 65	1.17E+02	1.17E+02	1.17E+02	1.17E+02	1.17E+02	1.17E+02	1.17E+02	1.35E+02
ZN 69	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 83	7.46E-04	7.46E-04	7.46E-04	7.46E-04	7.46E-04	7.46E-04	7.46E-04	1.08E-03
BR 84	2.80E-02	2.80E-02	2.80E-02	2.80E-02	2.80E-02	2.80E-02	2.80E-02	3.27E-02
BR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 86	1.42E+00	1.42E+00	1.42E+00	1.42E+00	1.42E+00	1.42E+00	1.42E+00	1.62E+00
RB 88	4.14E-03	4.14E-03	4.14E-03	4.14E-03	4.14E-03	4.14E-03	4.14E-03	4.73E-03
RB 89	1.46E-02	1.46E-02	1.46E-02	1.46E-02	1.46E-02	1.46E-02	1.46E-02	1.75E-02
SR 89	3.40E-03	3.40E-03	3.40E-03	3.40E-03	3.40E-03	3.40E-03	3.40E-03	3.95E-03
SR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 91	3.35E-01	3.35E-01	3.35E-01	3.35E-01	3.35E-01	3.35E-01	3.35E-01	3.92E-01
SR 92	1.19E-01	1.19E-01	1.19E-01	1.19E-01	1.19E-01	1.19E-01	1.19E-01	1.33E-01
Y 90	7.08E-04	7.08E-04	7.08E-04	7.08E-04	7.08E-04	7.08E-04	7.08E-04	8.36E-04
Y 91M	1.45E-02	1.45E-02	1.45E-02	1.45E-02	1.45E-02	1.45E-02	1.45E-02	1.68E-02
Y 91	1.69E-01	1.69E-01	1.69E-01	1.69E-01	1.69E-01	1.69E-01	1.69E-01	1.90E-01
Y 92	2.80E-02	2.80E-02	2.80E-02	2.80E-02	2.80E-02	2.80E-02	2.80E-02	3.32E-02
Y 93	2.86E-02	2.86E-02	2.86E-02	2.86E-02	2.86E-02	2.86E-02	2.86E-02	3.92E-02
ZR 95	3.86E+01	3.86E+01	3.86E+01	3.86E+01	3.86E+01	3.86E+01	3.86E+01	4.48E+01
ZR 97	4.64E-01	4.64E-01	4.64E-01	4.64E-01	4.64E-01	4.64E-01	4.64E-01	5.40E-01
NB 95	2.16E+01	2.16E+01	2.16E+01	2.16E+01	2.16E+01	2.16E+01	2.16E+01	2.54E+01
MO 99	6.28E-01	6.28E-01	6.28E-01	6.28E-01	6.28E-01	6.28E-01	6.28E-01	7.27E-01
TC 99M	2.87E-02	2.87E-02	2.87E-02	2.87E-02	2.87E-02	2.87E-02	2.87E-02	3.28E-02
TC101	2.39E-03	2.39E-03	2.39E-03	2.39E-03	2.39E-03	2.39E-03	2.39E-03	2.66E-03

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = ADULT
PATHWAY = RIVER SHORELINE DEPOSITS

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
RU103	1.70E+01	1.70E+01	1.70E+01	1.70E+01	1.70E+01	1.70E+01	1.70E+01	1.99E+01
RU105	9.86E-02	9.86E-02	9.86E-02	9.86E-02	9.86E-02	9.86E-02	9.86E-02	1.12E-01
RU106	6.65E+01	6.65E+01	6.65E+01	6.65E+01	6.65E+01	6.65E+01	6.65E+01	7.97E+01
AG110M	5.43E+02	5.43E+02	5.43E+02	5.43E+02	5.43E+02	5.43E+02	5.43E+02	6.34E+02
TE125M	2.45E-01	2.45E-01	2.45E-01	2.45E-01	2.45E-01	2.45E-01	2.45E-01	3.36E-01
TE127M	1.44E-02	1.44E-02	1.44E-02	1.44E-02	1.44E-02	1.44E-02	1.44E-02	1.71E-02
TE127	4.65E-04	4.65E-04	4.65E-04	4.65E-04	4.65E-04	4.65E-04	4.65E-04	5.12E-04
TE129M	3.11E+00	3.11E+00	3.11E+00	3.11E+00	3.11E+00	3.11E+00	3.11E+00	3.64E+00
TE129	3.89E-03	3.89E-03	3.89E-03	3.89E-03	3.89E-03	3.89E-03	3.89E-03	4.60E-03
TE131M	1.26E+00	1.26E+00	1.26E+00	1.26E+00	1.26E+00	1.26E+00	1.26E+00	1.49E+00
TE131	3.90E-03	3.90E-03	3.90E-03	3.90E-03	3.90E-03	3.90E-03	3.90E-03	4.61E+00
TE132	6.67E-01	6.67E-01	6.67E-01	6.67E-01	6.67E-01	6.67E-01	6.67E-01	7.85E-01
I 130	8.62E-01	8.62E-01	8.62E-01	8.62E-01	8.62E-01	8.62E-01	8.62E-01	1.05E+00
I 131	2.71E+00	2.71E+00	2.71E+00	2.71E+00	2.71E+00	2.71E+00	2.71E+00	3.29E+00
I 132	1.90E-01	1.90E-01	1.90E-01	1.90E-01	1.90E-01	1.90E-01	1.90E-01	2.24E-01
I 133	3.85E-01	3.85E-01	3.85E-01	3.85E-01	3.85E-01	3.85E-01	3.85E-01	4.68E-01
I 134	6.49E-02	6.49E-02	6.49E-02	6.49E-02	6.49E-02	6.49E-02	6.49E-02	7.71E-02
I 135	3.93E-01	3.93E-01	3.93E-01	3.93E-01	3.93E-01	3.93E-01	3.93E-01	4.58E-01
CS134	1.08E+03	1.08E+03	1.08E+03	1.08E+03	1.08E+03	1.08E+03	1.08E+03	1.26E+03
CS136	2.37E+01	2.37E+01	2.37E+01	2.37E+01	2.37E+01	2.37E+01	2.37E+01	2.68E+01
CS137	1.62E+03	1.62E+03	1.62E+03	1.62E+03	1.62E+03	1.62E+03	1.62E+03	1.89E+03
CS138	4.98E-02	4.98E-02	4.98E-02	4.98E-02	4.98E-02	4.98E-02	4.98E-02	5.69E-02
BA139	1.58E-02	1.58E-02	1.58E-02	1.58E-02	1.58E-02	1.58E-02	1.58E-02	1.77E-02
BA140	3.23E+00	3.23E+00	3.23E+00	3.23E+00	3.23E+00	3.23E+00	3.23E+00	3.70E+00
BA141	5.24E-03	5.24E-03	5.24E-03	5.24E-03	5.24E-03	5.24E-03	5.24E-03	5.97E-03
BA142	4.79E-03	4.79E-03	4.79E-03	4.79E-03	4.79E-03	4.79E-03	4.79E-03	5.46E-03
LA140	3.03E+00	3.03E+00	3.03E+00	3.03E+00	3.03E+00	3.03E+00	3.03E+00	3.43E+00
LA142	1.11E-01	1.11E-01	1.11E-01	1.11E-01	1.11E-01	1.11E-01	1.11E-01	1.33E-01
CE141	2.15E+00	2.15E+00	2.15E+00	2.15E+00	2.15E+00	2.15E+00	2.15E+00	2.42E+00
CE143	3.65E-01	3.65E-01	3.65E-01	3.65E-01	3.65E-01	3.65E-01	3.65E-01	4.15E-01
CE144	1.10E+01	1.10E+01	1.10E+01	1.10E+01	1.10E+01	1.10E+01	1.10E+01	1.27E+01
PR143	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
PR144	2.27E-04	2.27E-04	2.27E-04	2.27E-04	2.27E-04	2.27E-04	2.27E-04	2.61E-04
ND147	1.34E+00	1.34E+00	1.34E+00	1.34E+00	1.34E+00	1.34E+00	1.34E+00	1.60E+00
W 187	3.71E-01	3.71E-01	3.71E-01	3.71E-01	3.71E-01	3.71E-01	3.71E-01	4.31E-01
NP239	2.70E-01	2.70E-01	2.70E-01	2.70E-01	2.70E-01	2.70E-01	2.70E-01	3.12E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = ADULT
PATHWAY = SWIMMING

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem gal)/(Ci min)				
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NA 24	3.12E+01	3.12E+01	3.12E+01	3.12E+01	3.12E+01	3.12E+01	3.12E+01	0.00E-01
P 32	2.57E-02	2.57E-02	2.57E-02	2.57E-02	2.57E-02	2.57E-02	2.57E-02	0.00E-01
CR 51	2.09E-01	2.09E-01	2.09E-01	2.09E-01	2.09E-01	2.09E-01	2.09E-01	0.00E-01
MN 54	6.02E+00	6.02E+00	6.02E+00	6.02E+00	6.02E+00	6.02E+00	6.02E+00	0.00E-01
MN 56	1.25E+01	1.25E+01	1.25E+01	1.25E+01	1.25E+01	1.25E+01	1.25E+01	0.00E-01
FE 55	2.57E-04	2.57E-04	2.57E-04	2.57E-04	2.57E-04	2.57E-04	2.57E-04	0.00E-01
FE 59	8.83E+00	8.83E+00	8.83E+00	8.83E+00	8.83E+00	8.83E+00	8.83E+00	0.00E-01
CO 58	7.23E+00	7.23E+00	7.23E+00	7.23E+00	7.23E+00	7.23E+00	7.23E+00	0.00E-01
CO 60	1.85E+01	1.85E+01	1.85E+01	1.85E+01	1.85E+01	1.85E+01	1.85E+01	0.00E-01
NI 63	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	3.90E+00	3.90E+00	3.90E+00	3.90E+00	3.90E+00	3.90E+00	3.90E+00	0.00E-01
CU 64	1.48E+00	1.48E+00	1.48E+00	1.48E+00	1.48E+00	1.48E+00	1.48E+00	0.00E-01
ZN 65	4.42E+00	4.42E+00	4.42E+00	4.42E+00	4.42E+00	4.42E+00	4.42E+00	0.00E-01
ZN 69	5.97E-03	5.97E-03	5.97E-03	5.97E-03	5.97E-03	5.97E-03	5.97E-03	0.00E-01
BR 83	6.63E-02	6.63E-02	6.63E-02	6.63E-02	6.63E-02	6.63E-02	6.63E-02	0.00E-01
BR 84	1.23E+01	1.23E+01	1.23E+01	1.23E+01	1.23E+01	1.23E+01	1.23E+01	0.00E-01
BR 85	1.32E-02	1.32E-02	1.32E-02	1.32E-02	1.32E-02	1.32E-02	1.32E-02	0.00E-01
RB 86	6.82E-01	6.82E-01	6.82E-01	6.82E-01	6.82E-01	6.82E-01	6.82E-01	0.00E-01
RB 88	3.82E+00	3.82E+00	3.82E+00	3.82E+00	3.82E+00	3.82E+00	3.82E+00	0.00E-01
RB 89	1.38E+01	1.38E+01	1.38E+01	1.38E+01	1.38E+01	1.38E+01	1.38E+01	0.00E-01
SR 89	1.85E-02	1.85E-02	1.85E-02	1.85E-02	1.85E-02	1.85E-02	1.85E-02	0.00E-01
SR 90	2.17E-03	2.17E-03	2.17E-03	2.17E-03	2.17E-03	2.17E-03	2.17E-03	0.00E-01
SR 91	7.57E+00	7.57E+00	7.57E+00	7.57E+00	7.57E+00	7.57E+00	7.57E+00	0.00E-01
SR 92	1.02E+01	1.02E+01	1.02E+01	1.02E+01	1.02E+01	1.02E+01	1.02E+01	0.00E-01
Y 90	5.21E-02	5.21E-02	5.21E-02	5.21E-02	5.21E-02	5.21E-02	5.21E-02	0.00E-01
Y 91M	3.69E+00	3.69E+00	3.69E+00	3.69E+00	3.69E+00	3.69E+00	3.69E+00	0.00E-01
Y 91	2.69E-02	2.69E-02	2.69E-02	2.69E-02	2.69E-02	2.69E-02	2.69E-02	0.00E-01
Y 92	1.81E+00	1.81E+00	1.81E+00	1.81E+00	1.81E+00	1.81E+00	1.81E+00	0.00E-01
Y 93	7.57E-01	7.57E-01	7.57E-01	7.57E-01	7.57E-01	7.57E-01	7.57E-01	0.00E-01
ZR 95	6.02E+00	6.02E+00	6.02E+00	6.02E+00	6.02E+00	6.02E+00	6.02E+00	0.00E-01
ZR 97	6.00E+00	6.00E+00	6.00E+00	6.00E+00	6.00E+00	6.00E+00	6.00E+00	0.00E-01
NB 95	5.62E+00	5.62E+00	5.62E+00	5.62E+00	5.62E+00	5.62E+00	5.62E+00	0.00E-01
MO 99	1.88E+00	1.88E+00	1.88E+00	1.88E+00	1.88E+00	1.88E+00	1.88E+00	0.00E-01
TC 99M	9.52E-01	9.52E-01	9.52E-01	9.52E-01	9.52E-01	9.52E-01	9.52E-01	0.00E-01
TC101	2.04E+00	2.04E+00	2.04E+00	2.04E+00	2.04E+00	2.04E+00	2.04E+00	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = ADULT
PATHWAY = SWIMMING

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
RU103	3.57E+00	3.57E+00	3.57E+00	3.57E+00	3.57E+00	3.57E+00	3.57E+00	0.00E-01
RU105	4.74E+00	4.74E+00	4.74E+00	4.74E+00	4.74E+00	4.74E+00	4.74E+00	0.00E-01
RU106	1.52E+00	1.52E+00	1.52E+00	1.52E+00	1.52E+00	1.52E+00	1.52E+00	0.00E-01
AG110M	1.97E+01	1.97E+01	1.97E+01	1.97E+01	1.97E+01	1.97E+01	1.97E+01	0.00E-01
TE125M	1.48E-02	1.48E-02	1.48E-02	1.48E-02	1.48E-02	1.48E-02	1.48E-02	0.00E-01
TE127M	1.04E-03	1.04E-03	1.04E-03	1.04E-03	1.04E-03	1.04E-03	1.04E-03	0.00E-01
TE127	1.12E-02	1.12E-02	1.12E-02	1.12E-02	1.12E-02	1.12E-02	1.12E-02	0.00E-01
TE129M	8.43E-01	8.43E-01	8.43E-01	8.43E-01	8.43E-01	8.43E-01	8.43E-01	0.00E-01
TE129	7.19E-01	7.19E-01	7.19E-01	7.19E-01	7.19E-01	7.19E-01	7.19E-01	0.00E-01
TE131M	8.81E+00	8.81E+00	8.81E+00	8.81E+00	8.81E+00	8.81E+00	8.81E+00	0.00E-01
TE131	2.52E+00	2.52E+00	2.52E+00	2.52E+00	2.52E+00	2.52E+00	2.52E+00	0.00E-01
TE132	1.60E+00	1.60E+00	1.60E+00	1.60E+00	1.60E+00	1.60E+00	1.60E+00	0.00E-01
I 130	1.56E+01	1.56E+01	1.56E+01	1.56E+01	1.56E+01	1.56E+01	1.56E+01	0.00E-01
I 131	3.13E+00	3.13E+00	3.13E+00	3.13E+00	3.13E+00	3.13E+00	3.13E+00	0.00E-01
I 132	1.71E+01	1.71E+01	1.71E+01	1.71E+01	1.71E+01	1.71E+01	1.71E+01	0.00E-01
I 133	3.84E+00	3.84E+00	3.84E+00	3.84E+00	3.84E+00	3.84E+00	3.84E+00	0.00E-01
I 134	1.56E+01	1.56E+01	1.56E+01	1.56E+01	1.56E+01	1.56E+01	1.56E+01	0.00E-01
I 135	1.31E+01	1.31E+01	1.31E+01	1.31E+01	1.31E+01	1.31E+01	1.31E+01	0.00E-01
CS134	1.16E+01	1.16E+01	1.16E+01	1.16E+01	1.16E+01	1.16E+01	1.16E+01	0.00E-01
CS136	1.65E+01	1.65E+01	1.65E+01	1.65E+01	1.65E+01	1.65E+01	1.65E+01	0.00E-01
CS137	4.01E+00	4.01E+00	4.01E+00	4.01E+00	4.01E+00	4.01E+00	4.01E+00	0.00E-01
CS138	1.41E+01	1.41E+01	1.41E+01	1.41E+01	1.41E+01	1.41E+01	1.41E+01	0.00E-01
BA139	2.94E-01	2.94E-01	2.94E-01	2.94E-01	2.94E-01	2.94E-01	2.94E-01	0.00E-01
BA140	1.97E+00	1.97E+00	1.97E+00	1.97E+00	1.97E+00	1.97E+00	1.97E+00	0.00E-01
BA141	3.52E+00	3.52E+00	3.52E+00	3.52E+00	3.52E+00	3.52E+00	3.52E+00	0.00E-01
BA142	5.99E+00	5.99E+00	5.99E+00	5.99E+00	5.99E+00	5.99E+00	5.99E+00	0.00E-01
LA140	1.64E+01	1.64E+01	1.64E+01	1.64E+01	1.64E+01	1.64E+01	1.64E+01	0.00E-01
LA142	1.73E+01	1.73E+01	1.73E+01	1.73E+01	1.73E+01	1.73E+01	1.73E+01	0.00E-01
CE141	5.22E-01	5.22E-01	5.22E-01	5.22E-01	5.22E-01	5.22E-01	5.22E-01	0.00E-01
CE143	2.28E+00	2.28E+00	2.28E+00	2.28E+00	2.28E+00	2.28E+00	2.28E+00	0.00E-01
CE144	3.45E-01	3.45E-01	3.45E-01	3.45E-01	3.45E-01	3.45E-01	3.45E-01	0.00E-01
PR143	6.42E-03	6.42E-03	6.42E-03	6.42E-03	6.42E-03	6.42E-03	6.42E-03	0.00E-01
PR144	1.77E-01	1.77E-01	1.77E-01	1.77E-01	1.77E-01	1.77E-01	1.77E-01	0.00E-01
ND147	1.12E+00	1.12E+00	1.12E+00	1.12E+00	1.12E+00	1.12E+00	1.12E+00	0.00E-01
W 187	3.32E+00	3.32E+00	3.32E+00	3.32E+00	3.32E+00	3.32E+00	3.32E+00	0.00E-01
NP239	9.62E-01	9.62E-01	9.62E-01	9.62E-01	9.62E-01	9.62E-01	9.62E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = ADULT
PATHWAY = BOATING

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NA 24	5.65E+01	5.65E+01	5.65E+01	5.65E+01	5.65E+01	5.65E+01	5.65E+01	0.00E-01
P 32	4.66E-02	4.66E-02	4.66E-02	4.66E-02	4.66E-02	4.66E-02	4.66E-02	0.00E-01
CR 51	3.78E-01	3.78E-01	3.78E-01	3.78E-01	3.78E-01	3.78E-01	3.78E-01	0.00E-01
MN 54	1.09E+01	1.09E+01	1.09E+01	1.09E+01	1.09E+01	1.09E+01	1.09E+01	0.00E-01
MN 56	2.27E+01	2.27E+01	2.27E+01	2.27E+01	2.27E+01	2.27E+01	2.27E+01	0.00E-01
FE 55	4.66E-04	4.66E-04	4.66E-04	4.66E-04	4.66E-04	4.66E-04	4.66E-04	0.00E-01
FE 59	1.60E+01	1.60E+01	1.60E+01	1.60E+01	1.60E+01	1.60E+01	1.60E+01	0.00E-01
CO 58	1.31E+01	1.31E+01	1.31E+01	1.31E+01	1.31E+01	1.31E+01	1.31E+01	0.00E-01
CO 60	3.35E+01	3.35E+01	3.35E+01	3.35E+01	3.35E+01	3.35E+01	3.35E+01	0.00E-01
NI 63	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	7.08E+00	7.08E+00	7.08E+00	7.08E+00	7.08E+00	7.08E+00	7.08E+00	0.00E-01
CU 64	2.68E+00	2.68E+00	2.68E+00	2.68E+00	2.68E+00	2.68E+00	2.68E+00	0.00E-01
ZN 65	8.00E+00	8.00E+00	8.00E+00	8.00E+00	8.00E+00	8.00E+00	8.00E+00	0.00E-01
ZN 69	1.08E-02	1.08E-02	1.08E-02	1.08E-02	1.08E-02	1.08E-02	1.08E-02	0.00E-01
BR 83	1.20E-01	1.20E-01	1.20E-01	1.20E-01	1.20E-01	1.20E-01	1.20E-01	0.00E-01
BR 84	2.24E+01	2.24E+01	2.24E+01	2.24E+01	2.24E+01	2.24E+01	2.24E+01	0.00E-01
BR 85	2.39E-02	2.39E-02	2.39E-02	2.39E-02	2.39E-02	2.39E-02	2.39E-02	0.00E-01
RB 86	1.24E+00	1.24E+00	1.24E+00	1.24E+00	1.24E+00	1.24E+00	1.24E+00	0.00E-01
RB 88	6.92E+00	6.92E+00	6.92E+00	6.92E+00	6.92E+00	6.92E+00	6.92E+00	0.00E-01
RB 89	2.49E+01	2.49E+01	2.49E+01	2.49E+01	2.49E+01	2.49E+01	2.49E+01	0.00E-01
SR 89	3.35E-02	3.35E-02	3.35E-02	3.35E-02	3.35E-02	3.35E-02	3.35E-02	0.00E-01
SR 90	3.93E-03	3.93E-03	3.93E-03	3.93E-03	3.93E-03	3.93E-03	3.93E-03	0.00E-01
SR 91	1.37E+01	1.37E+01	1.37E+01	1.37E+01	1.37E+01	1.37E+01	1.37E+01	0.00E-01
SR 92	1.84E+01	1.84E+01	1.84E+01	1.84E+01	1.84E+01	1.84E+01	1.84E+01	0.00E-01
Y 90	9.45E-02	9.45E-02	9.45E-02	9.45E-02	9.45E-02	9.45E-02	9.45E-02	0.00E-01
Y 91M	6.69E+00	6.69E+00	6.69E+00	6.69E+00	6.69E+00	6.69E+00	6.69E+00	0.00E-01
Y 91	4.87E-02	4.87E-02	4.87E-02	4.87E-02	4.87E-02	4.87E-02	4.87E-02	0.00E-01
Y 92	3.28E+00	3.28E+00	3.28E+00	3.28E+00	3.28E+00	3.28E+00	3.28E+00	0.00E-01
Y 93	1.37E+00	1.37E+00	1.37E+00	1.37E+00	1.37E+00	1.37E+00	1.37E+00	0.00E-01
ZR 95	1.09E+01	1.09E+01	1.09E+01	1.09E+01	1.09E+01	1.09E+01	1.09E+01	0.00E-01
ZR 97	1.09E+01	1.09E+01	1.09E+01	1.09E+01	1.09E+01	1.09E+01	1.09E+01	0.00E-01
NB 95	1.02E+01	1.02E+01	1.02E+01	1.02E+01	1.02E+01	1.02E+01	1.02E+01	0.00E-01
MO 99	3.42E+00	3.42E+00	3.42E+00	3.42E+00	3.42E+00	3.42E+00	3.42E+00	0.00E-01
TC 99M	1.73E+00	1.73E+00	1.73E+00	1.73E+00	1.73E+00	1.73E+00	1.73E+00	0.00E-01
TC101	3.69E+00	3.69E+00	3.69E+00	3.69E+00	3.69E+00	3.69E+00	3.69E+00	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = ADULT
PATHWAY = BOATING

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
RU103	6.47E+00	6.47E+00	6.47E+00	6.47E+00	6.47E+00	6.47E+00	6.47E+00	0.00E-01
RU105	8.60E+00	8.60E+00	8.60E+00	8.60E+00	8.60E+00	8.60E+00	8.60E+00	0.00E-01
RU106	2.77E+00	2.77E+00	2.77E+00	2.77E+00	2.77E+00	2.77E+00	2.77E+00	0.00E-01
AG110M	3.57E+01	3.57E+01	3.57E+01	3.57E+01	3.57E+01	3.57E+01	3.57E+01	0.00E-01
TE125M	2.69E-02	2.69E-02	2.69E-02	2.69E-02	2.69E-02	2.69E-02	2.69E-02	0.00E-01
TE127M	1.89E-03	1.89E-03	1.89E-03	1.89E-03	1.89E-03	1.89E-03	1.89E-03	0.00E-01
TE127	2.02E-02	2.02E-02	2.02E-02	2.02E-02	2.02E-02	2.02E-02	2.02E-02	0.00E-01
TE129M	1.53E+00	1.53E+00	1.53E+00	1.53E+00	1.53E+00	1.53E+00	1.53E+00	0.00E-01
TE129	1.30E+00	1.30E+00	1.30E+00	1.30E+00	1.30E+00	1.30E+00	1.30E+00	0.00E-01
TE131M	1.60E+01	1.60E+01	1.60E+01	1.60E+01	1.60E+01	1.60E+01	1.60E+01	0.00E-01
TE131	4.56E+00	4.56E+00	4.56E+00	4.56E+00	4.56E+00	4.56E+00	4.56E+00	0.00E-01
TE132	2.91E+00	2.91E+00	2.91E+00	2.91E+00	2.91E+00	2.91E+00	2.91E+00	0.00E-01
I 130	2.82E+01	2.82E+01	2.82E+01	2.82E+01	2.82E+01	2.82E+01	2.82E+01	0.00E-01
I 131	5.67E+00	5.67E+00	5.67E+00	5.67E+00	5.67E+00	5.67E+00	5.67E+00	0.00E-01
I 132	3.11E+01	3.11E+01	3.11E+01	3.11E+01	3.11E+01	3.11E+01	3.11E+01	0.00E-01
I 133	6.96E+00	6.96E+00	6.96E+00	6.96E+00	6.96E+00	6.96E+00	6.96E+00	0.00E-01
I 134	2.82E+01	2.82E+01	2.82E+01	2.82E+01	2.82E+01	2.82E+01	2.82E+01	0.00E-01
I 135	2.38E+01	2.38E+01	2.38E+01	2.38E+01	2.38E+01	2.38E+01	2.38E+01	0.00E-01
CS134	2.11E+01	2.11E+01	2.11E+01	2.11E+01	2.11E+01	2.11E+01	2.11E+01	0.00E-01
CS136	2.98E+01	2.98E+01	2.98E+01	2.98E+01	2.98E+01	2.98E+01	2.98E+01	0.00E-01
CS137	7.28E+00	7.28E+00	7.28E+00	7.28E+00	7.28E+00	7.28E+00	7.28E+00	0.00E-01
CS138	2.56E+01	2.56E+01	2.56E+01	2.56E+01	2.56E+01	2.56E+01	2.56E+01	0.00E-01
BA139	5.33E-01	5.33E-01	5.33E-01	5.33E-01	5.33E-01	5.33E-01	5.33E-01	0.00E-01
BA140	3.56E+00	3.56E+00	3.56E+00	3.56E+00	3.56E+00	3.56E+00	3.56E+00	0.00E-01
BA141	6.37E+00	6.37E+00	6.37E+00	6.37E+00	6.37E+00	6.37E+00	6.37E+00	0.00E-01
BA142	1.09E+01	1.09E+01	1.09E+01	1.09E+01	1.09E+01	1.09E+01	1.09E+01	0.00E-01
LA140	2.98E+01	2.98E+01	2.98E+01	2.98E+01	2.98E+01	2.98E+01	2.98E+01	0.00E-01
LA142	3.13E+01	3.13E+01	3.13E+01	3.13E+01	3.13E+01	3.13E+01	3.13E+01	0.00E-01
CE141	9.46E-01	9.46E-01	9.46E-01	9.46E-01	9.46E-01	9.46E-01	9.46E-01	0.00E-01
CE143	4.14E+00	4.14E+00	4.14E+00	4.14E+00	4.14E+00	4.14E+00	4.14E+00	0.00E-01
CE144	6.26E-01	6.26E-01	6.26E-01	6.26E-01	6.26E-01	6.26E-01	6.26E-01	0.00E-01
PR143	1.16E-02	1.16E-02	1.16E-02	1.16E-02	1.16E-02	1.16E-02	1.16E-02	0.00E-01
PR144	3.20E-01	3.20E-01	3.20E-01	3.20E-01	3.20E-01	3.20E-01	3.20E-01	0.00E-01
ND147	2.04E+00	2.04E+00	2.04E+00	2.04E+00	2.04E+00	2.04E+00	2.04E+00	0.00E-01
W 187	6.02E+00	6.02E+00	6.02E+00	6.02E+00	6.02E+00	6.02E+00	6.02E+00	0.00E-01
NP239	1.74E+00	1.74E+00	1.74E+00	1.74E+00	1.74E+00	1.74E+00	1.74E+00	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = TEENAGER
PATHWAY = POTABLE WATER

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	1.54E+01	1.54E+01	0.00E-01	1.54E+01	1.54E+01	1.54E+01	1.54E+01	0.00E-01
C 14	2.08E+02	2.08E+02	1.04E+03	2.08E+02	2.08E+02	2.08E+02	2.08E+02	0.00E-01
NA 24	1.95E+02	1.95E+02	1.95E+02	1.95E+02	1.95E+02	1.95E+02	1.95E+02	0.00E-01
P 32	2.61E+03	5.66E+03	6.73E+04	4.17E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	8.99E-01	1.51E+02	0.00E-01	0.00E-01	1.97E-01	4.99E-01	1.28E+00	0.00E-01
MN 54	2.99E+02	3.09E+03	0.00E-01	1.51E+03	4.49E+02	0.00E-01	0.00E-01	0.00E-01
MN 56	1.10E-02	4.08E+00	0.00E-01	6.20E-02	7.85E-02	0.00E-01	0.00E-01	0.00E-01
FE 55	1.60E+02	2.97E+02	9.67E+02	6.85E+02	0.00E-01	0.00E-01	4.35E+02	0.00E-01
FE 59	1.33E+03	8.16E+03	1.48E+03	3.45E+03	0.00E-01	0.00E-01	1.09E+03	0.00E-01
CO 58	5.68E+02	3.40E+03	0.00E-01	2.46E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	1.62E+03	9.36E+03	0.00E-01	7.19E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 63	1.53E+03	5.09E+02	4.53E+04	3.20E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	1.52E-02	1.80E+00	2.61E-01	3.33E-02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CU 64	3.72E+00	6.14E+02	0.00E-01	7.91E+00	2.00E+01	0.00E-01	0.00E-01	0.00E-01
ZN 65	2.38E+03	2.16E+03	1.47E+03	5.10E+03	3.27E+03	0.00E-01	0.00E-01	0.00E-01
ZN 69	1.21E-08	3.19E-07	9.08E-08	1.73E-07	1.13E-07	0.00E-01	0.00E-01	0.00E-01
BR 83	1.40E-02	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 84	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 86	3.45E+03	1.09E+03	0.00E-01	7.35E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	3.18E+03	1.32E+04	1.11E+05	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	5.22E+04	5.96E+04	2.61E+06	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 91	1.42E+01	1.62E+03	3.57E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 92	7.21E-02	4.31E+01	1.69E+00	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 90	7.29E-02	2.23E+04	2.70E+00	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91	1.36E+00	2.08E+04	5.08E+01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 92	8.29E-05	7.86E+01	2.86E-03	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 93	5.16E-03	5.75E+03	1.88E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	2.26E+00	7.59E+03	1.04E+01	3.29E+00	4.84E+00	0.00E-01	0.00E-01	0.00E-01
ZR 97	2.06E-02	1.21E+04	2.27E-01	4.48E-02	6.80E-02	0.00E-01	0.00E-01	0.00E-01
NB 95	6.30E-01	4.89E+03	2.06E+00	1.14E+00	1.11E+00	0.00E-01	0.00E-01	0.00E-01
MO 99	2.29E+02	2.15E+03	0.00E-01	1.20E+03	2.74E+03	0.00E-01	0.00E-01	0.00E-01
TC 99M	1.93E-01	9.80E+00	5.35E-03	1.49E-02	2.22E-01	0.00E-01	8.29E-03	0.00E-01
TC101	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = TEENAGER
PATHWAY = POTABLE WATER

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
RU103	2.74E+01	5.36E+03	6.41E+01	0.00E-01	2.26E+02	0.00E-01	0.00E-01	0.00E-01
RU105	5.09E-02	1.06E+02	1.31E-01	0.00E-01	1.65E+00	0.00E-01	0.00E-01	0.00E-01
RU106	1.26E+02	4.80E+04	1.00E+03	0.00E-01	1.93E+03	0.00E-01	0.00E-01	0.00E-01
AG110M	3.01E+01	1.39E+04	5.23E+01	4.95E+01	9.44E+01	0.00E-01	0.00E-01	0.00E-01
TE125M	1.29E+02	2.86E+03	9.68E+02	3.49E+02	0.00E-01	2.71E+02	0.00E-01	0.00E-01
TE127M	2.92E+02	6.13E+03	2.46E+03	8.72E+02	9.97E+03	5.85E+02	0.00E-01	0.00E-01
TE127	1.47E+00	5.27E+02	6.82E+00	2.42E+00	2.76E+01	4.70E+00	0.00E-01	0.00E-01
TE129M	6.47E+02	1.53E+04	4.09E+03	1.52E+03	1.71E+04	1.32E+03	0.00E-01	0.00E-01
TE129	1.65E-06	3.70E-05	6.77E-06	2.52E-06	2.84E-05	4.83E-06	0.00E-01	0.00E-01
TE131M	1.43E+02	1.38E+04	3.59E+02	1.72E+02	1.79E+03	2.59E+02	0.00E-01	0.00E-01
TE131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE132	4.30E+02	1.45E+04	7.22E+02	4.57E+02	4.39E+03	4.82E+02	0.00E-01	0.00E-01
I 130	7.91E+01	1.52E+02	6.85E+01	1.98E+02	3.05E+02	1.62E+04	0.00E-01	0.00E-01
I 131	1.03E+03	3.80E+02	1.37E+03	1.92E+03	3.31E+03	5.61E+05	0.00E-01	0.00E-01
I 132	4.85E-02	5.89E-02	5.16E-02	1.35E-01	2.13E-01	4.55E+00	0.00E-01	0.00E-01
I 133	1.20E+02	2.97E+02	2.31E+02	3.92E+02	6.88E+02	5.48E+04	0.00E-01	0.00E-01
I 134	1.98E-07	7.25E-09	2.08E-07	5.50E-07	8.68E-07	9.17E-06	0.00E-01	0.00E-01
I 135	1.19E+01	3.57E+01	1.25E+01	3.22E+01	5.09E+01	2.07E+03	0.00E-01	0.00E-01
CS134	2.34E+04	6.26E+02	2.14E+04	5.04E+04	1.60E+04	0.00E-01	6.11E+03	0.00E-01
CS136	5.51E+03	6.60E+02	2.08E+03	8.20E+03	4.47E+03	0.00E-01	7.04E+02	0.00E-01
CS137	1.33E+04	5.42E+02	2.87E+04	3.81E+04	1.30E+04	0.00E-01	5.04E+03	0.00E-01
CS138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA139	5.79E-06	1.77E-03	1.99E-04	1.40E-07	1.32E-07	0.00E-01	9.63E-08	0.00E-01
BA140	4.44E+02	1.06E+04	6.88E+03	8.44E+00	2.86E+00	0.00E-01	5.67E+00	0.00E-01
BA141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA142	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA140	7.71E-02	1.66E+04	5.89E-01	2.90E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA142	1.03E-07	1.26E-02	9.35E-07	4.15E-07	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	2.55E-01	6.36E+03	3.33E+00	2.22E+00	1.05E+00	0.00E-01	0.00E-01	0.00E-01
CE143	2.96E-02	7.96E+03	3.64E-01	2.65E+02	1.19E-01	0.00E-01	0.00E-01	0.00E-01
CE144	9.55E+00	4.47E+04	1.78E+02	7.35E+01	4.39E+01	0.00E-01	0.00E-01	0.00E-01
PR143	1.59E-01	1.05E+04	3.19E+00	1.27E+00	7.39E-01	0.00E-01	0.00E-01	0.00E-01
PR144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ND147	1.47E-01	8.85E+03	2.26E+00	2.45E+00	1.44E+00	0.00E-01	0.00E-01	0.00E-01
W 187	5.32E+00	4.11E+03	1.86E+01	1.52E+01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NP239	1.76E-02	5.09E+03	3.36E-01	3.17E-02	9.94E-02	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = TEENAGER
PATHWAY = FRESH WATER FISH

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	4.36E-01	4.36E-01	0.00E-01	4.36E-01	4.36E-01	4.36E-01	4.36E-01	0.00E-01
C 14	3.00E+04	3.00E+04	1.50E+05	3.00E+04	3.00E+04	3.00E+04	3.00E+04	0.00E-01
NA 24	6.08E+02	6.08E+02	6.08E+02	6.08E+02	6.08E+02	6.08E+02	6.08E+02	0.00E-01
P 32	8.18E+06	1.77E+07	2.11E+08	1.31E+07	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	5.64E+00	9.47E+02	0.00E-01	0.00E-01	1.24E+00	3.13E+00	8.05E+00	0.00E-01
MN 54	3.75E+03	3.88E+04	0.00E-01	1.89E+04	5.64E+03	0.00E-01	0.00E-01	0.00E-01
MN 56	1.35E-01	4.99E+01	0.00E-01	7.57E-01	9.59E-01	0.00E-01	0.00E-01	0.00E-01
FE 55	5.01E+02	9.31E+02	3.03E+03	2.15E+03	0.00E-01	0.00E-01	1.36E+03	0.00E-01
FE 59	4.18E+03	2.56E+04	4.64E+03	1.08E+04	0.00E-01	0.00E-01	3.41E+03	0.00E-01
CO 58	8.90E+02	5.33E+03	0.00E-01	3.86E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	2.54E+03	1.47E+04	0.00E-01	1.13E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 63	4.82E+03	1.60E+03	1.42E+05	1.00E+04	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	4.63E-02	5.51E+00	7.95E-01	1.02E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CU 64	5.81E+00	9.58E+02	0.00E-01	1.24E+01	3.12E+01	0.00E-01	0.00E-01	0.00E-01
ZN 65	1.49E+05	1.36E+05	9.22E+04	3.20E+05	2.05E+05	0.00E-01	0.00E-01	0.00E-01
ZN 69	7.06E-07	1.86E-05	5.30E-06	1.01E-05	6.59E-06	0.00E-01	0.00E-01	0.00E-01
BR 83	1.79E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 84	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 86	2.17E+05	6.82E+04	0.00E-01	4.61E+05	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	2.99E+03	1.24E+04	1.04E+05	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	4.91E+04	5.61E+04	2.46E+06	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 91	1.33E+01	1.51E+03	3.34E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 92	6.61E-02	3.95E+01	1.55E+00	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 90	5.71E-02	1.75E+04	2.12E+00	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91	1.07E+00	1.63E+04	3.99E+01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 92	6.37E-05	6.05E+01	2.20E-03	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 93	4.02E-03	4.48E+03	1.47E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	2.34E-01	7.86E+02	1.08E+00	3.41E-01	5.01E-01	0.00E-01	0.00E-01	0.00E-01
ZR 97	2.13E-03	1.25E+03	2.34E-02	4.62E-03	7.01E-03	0.00E-01	0.00E-01	0.00E-01
NB 95	5.93E+02	4.60E+06	1.94E+03	1.08E+03	1.04E+03	0.00E-01	0.00E-01	0.00E-01
MO 99	7.17E+01	6.73E+02	0.00E-01	3.76E+02	8.60E+02	0.00E-01	0.00E-01	0.00E-01
TC 99M	9.00E-02	4.56E+00	2.49E-03	6.94E-03	1.04E-01	0.00E-01	3.85E-03	0.00E-01
TC101	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = TEENAGER
PATHWAY = FRESH WATER FISH

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
RU103	8.60E+00	1.68E+03	2.01E+01	0.00E-01	7.09E+01	0.00E-01	0.00E-01	0.00E-01
RU105	1.57E-02	3.27E+01	4.05E-02	0.00E-01	5.11E-01	0.00E-01	0.00E-01	0.00E-01
RU106	3.96E+01	1.51E+04	3.14E+02	0.00E-01	6.06E+02	0.00E-01	0.00E-01	0.00E-01
AG110M	2.17E+00	1.00E+03	3.78E+00	3.57E+00	6.81E+00	0.00E-01	0.00E-01	0.00E-01
TE125M	1.62E+03	3.59E+04	1.21E+04	4.38E+03	0.00E-01	3.39E+03	0.00E-01	0.00E-01
TE127M	3.67E+03	7.69E+04	3.09E+04	1.09E+04	1.25E+05	7.34E+03	0.00E-01	0.00E-01
TE127	1.83E+01	6.56E+03	8.49E+01	3.01E+01	3.44E+02	5.86E+01	0.00E-01	0.00E-01
TE129M	8.12E+03	1.92E+05	5.13E+04	1.90E+04	2.14E+05	1.65E+04	0.00E-01	0.00E-01
TE129	1.95E-05	4.38E-04	8.00E-05	2.98E-05	3.36E-04	5.72E-05	0.00E-01	0.00E-01
TE131M	1.80E+03	1.73E+05	4.49E+03	2.15E+03	2.24E+04	3.24E+03	0.00E-01	0.00E-01
TE131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE132	5.40E+03	1.82E+05	9.05E+03	5.73E+03	5.50E+04	6.04E+03	0.00E-01	0.00E-01
I 130	3.70E+01	7.12E+01	3.20E+01	9.27E+01	1.43E+02	7.56E+03	0.00E-01	0.00E-01
I 131	4.86E+02	1.79E+02	6.46E+02	9.04E+02	1.56E+03	2.64E+05	0.00E-01	0.00E-01
I 132	2.21E-02	2.69E-02	2.36E-02	6.17E-02	9.72E-02	2.08E+00	0.00E-01	0.00E-01
I 133	5.61E+01	1.39E+02	1.08E+02	1.84E+02	3.23E+02	2.57E+04	0.00E-01	0.00E-01
I 134	8.60E-08	3.15E-09	9.03E-08	2.39E-07	3.77E-07	3.99E-06	0.00E-01	0.00E-01
I 135	5.56E+00	1.66E+01	5.83E+00	1.50E+01	2.37E+01	9.66E+02	0.00E-01	0.00E-01
CS134	1.47E+06	3.93E+04	1.34E+06	3.16E+06	1.00E+06	0.00E-01	3.83E+05	0.00E-01
CS136	3.46E+05	4.14E+04	1.31E+05	5.15E+05	2.80E+05	0.00E-01	4.42E+04	0.00E-01
CS137	8.33E+05	3.40E+04	1.80E+06	2.39E+06	8.14E+05	0.00E-01	3.16E+05	0.00E-01
CS138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA139	6.90E-07	2.11E-04	2.37E-05	1.67E-08	1.57E-08	0.00E-01	1.15E-08	0.00E-01
BA140	5.57E+01	1.33E+03	8.64E+02	1.06E+00	3.59E-01	0.00E-01	7.12E-01	0.00E-01
BA141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA142	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA140	6.03E-02	1.30E+04	4.61E-01	2.27E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA142	7.75E-08	9.47E-03	7.01E-07	3.11E-07	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	8.02E-03	2.00E+02	1.05E-01	6.98E-02	3.29E-02	0.00E-01	0.00E-01	0.00E-01
CE143	9.26E-04	2.49E+02	1.14E-02	8.29E+00	3.72E-03	0.00E-01	0.00E-01	0.00E-01
CE144	2.99E-01	1.40E+03	5.57E+00	2.31E+00	1.38E+00	0.00E-01	0.00E-01	0.00E-01
PR143	1.24E-01	8.22E+03	2.50E+00	9.97E-01	5.80E-01	0.00E-01	0.00E-01	0.00E-01
PR144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ND147	1.15E-01	6.94E+03	1.77E+00	1.92E+00	1.13E+00	0.00E-01	0.00E-01	0.00E-01
W 187	2.00E+02	1.54E+05	7.00E+02	5.70E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NP239	5.51E-03	1.60E+03	1.05E-01	9.92E-03	3.11E-02	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = TEENAGER
PATHWAY = ANIMAL DRINKING WATER—MEAT

(mrem gal)/(Ci min)

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
H 3	9.85E-01	9.85E-01	0.00E-01	9.85E-01	9.85E-01	9.85E-01	9.85E-01	0.00E-01
C 14	4.10E+01	4.10E+01	2.05E+02	4.10E+01	4.10E+01	4.10E+01	4.10E+01	0.00E-01
NA 24	2.66E-08	2.66E-08	2.66E-08	2.66E-08	2.66E-08	2.66E-08	2.66E-08	0.00E-01
P 32	3.04E+02	6.59E+02	7.84E+03	4.86E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	8.53E-03	1.43E+00	0.00E-01	0.00E-01	1.87E-03	4.74E-03	1.22E-02	0.00E-01
MN 54	1.46E+00	1.51E+01	0.00E-01	7.36E+00	2.20E+00	0.00E-01	0.00E-01	0.00E-01
MN 56	2.45E-28	9.07E-26	0.00E-01	1.38E-27	1.74E-27	0.00E-01	0.00E-01	0.00E-01
FE 55	4.02E+01	7.46E+01	2.43E+02	1.72E+02	0.00E-01	0.00E-01	1.09E+02	0.00E-01
FE 59	2.53E+02	1.55E+03	2.80E+02	6.54E+02	0.00E-01	0.00E-01	2.06E+02	0.00E-01
CO 58	3.90E+01	2.34E+02	0.00E-01	1.69E+01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	1.33E+02	7.70E+02	0.00E-01	5.91E+01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 63	5.18E+02	1.72E+02	1.53E+04	1.08E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	2.51E-26	2.98E-24	4.31E-25	5.50E-26	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CU 64	2.62E-12	4.32E-10	0.00E-01	5.56E-12	1.41E-11	0.00E-01	0.00E-01	0.00E-01
ZN 65	4.31E+02	3.91E+02	2.66E+02	9.24E+02	5.92E+02	0.00E-01	0.00E-01	0.00E-01
ZN 69	4.04E-28	1.06E-26	3.03E-27	5.78E-27	3.77E-27	0.00E-01	0.00E-01	0.00E-01
BR 83	1.60E-26	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 84	7.26E-27	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 85	5.67E-34	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 86	3.37E+02	1.06E+02	0.00E-01	7.17E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 88	1.96E-27	3.15E-34	0.00E-01	3.67E-27	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 89	1.13E-27	2.45E-36	0.00E-01	1.60E-27	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	9.36E+00	3.89E+01	3.27E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	1.99E+02	2.28E+02	9.97E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 91	1.70E-16	1.94E-14	4.28E-15	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 92	8.63E-28	5.16E-25	2.02E-26	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 90	1.54E-05	4.70E+00	5.70E-04	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91M	1.40E-31	1.73E-28	3.66E-30	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91	3.19E-02	4.88E+02	1.19E+00	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 92	1.89E-30	1.79E-24	6.54E-29	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 93	3.41E-18	3.80E-12	1.24E-16	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	3.99E-01	1.34E+03	1.84E+00	5.81E-01	8.53E-01	0.00E-01	0.00E-01	0.00E-01
ZR 97	3.20E-11	1.88E-05	3.51E-10	6.95E-11	1.05E-10	0.00E-01	0.00E-01	0.00E-01
NB 95	7.72E-01	6.00E+03	2.53E+00	1.40E+00	1.36E+00	0.00E-01	0.00E-01	0.00E-01
MO 99	9.56E-02	8.97E-01	0.00E-01	5.01E-01	1.15E+00	0.00E-01	0.00E-01	0.00E-01
TC 99M	6.74E-24	3.42E-22	1.87E-25	5.20E-25	7.75E-24	0.00E-01	2.89E-25	0.00E-01
TC101	1.53E-27	2.67E-35	1.10E-28	1.56E-28	2.82E-27	0.00E-01	9.51E-29	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = TEENAGER
PATHWAY = ANIMAL DRINKING WATER—MEAT

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
RU103	4.99E+01	9.76E+03	1.17E+02	0.00E-01	4.12E+02	0.00E-01	0.00E-01	0.00E-01
RU105	4.13E-26	8.60E-23	1.06E-25	0.00E-01	1.34E-24	0.00E-01	0.00E-01	0.00E-01
RU106	3.10E+02	1.18E+05	2.46E+03	0.00E-01	4.75E+03	0.00E-01	0.00E-01	0.00E-01
AG110M	3.10E+00	1.43E+03	5.38E+00	5.09E+00	9.70E+00	0.00E-01	0.00E-01	0.00E-01
TE125M	5.06E+01	1.12E+03	3.79E+02	1.36E+02	0.00E-01	1.06E+02	0.00E-01	0.00E-01
TE127M	1.27E+02	2.66E+03	1.07E+03	3.79E+02	4.33E+03	2.54E+02	0.00E-01	0.00E-01
TE127	1.38E-15	4.94E-13	6.40E-15	2.27E-15	2.59E-14	4.41E-15	0.00E-01	0.00E-01
TE129M	2.14E+02	5.08E+03	1.35E+03	5.02E+02	5.66E+03	4.37E+02	0.00E-01	0.00E-01
TE129	6.59E-27	1.48E-25	2.71E-26	1.01E-26	1.14E-25	1.94E-26	0.00E-01	0.00E-01
TE131M	1.82E-03	1.75E-01	4.55E-03	2.18E-03	2.28E-02	3.28E-03	0.00E-01	0.00E-01
TE131	1.82E-27	4.79E-28	5.84E-27	2.41E-27	2.55E-26	4.50E-27	0.00E-01	0.00E-01
TE132	3.69E+00	1.24E+02	6.19E+00	3.92E+00	3.76E+01	4.13E+00	0.00E-01	0.00E-01
I 130	1.04E-11	2.01E-11	9.04E-12	2.62E-11	4.03E-11	2.13E-09	0.00E-01	0.00E-01
I 131	3.70E+00	1.36E+00	4.91E+00	6.88E+00	1.18E+01	2.01E+03	0.00E-01	0.00E-01
I 132	8.03E-27	9.74E-27	8.55E-27	2.24E-26	3.52E-26	7.54E-25	0.00E-01	0.00E-01
I 133	5.44E-07	1.35E-06	1.05E-06	1.78E-06	3.13E-06	2.49E-04	0.00E-01	0.00E-01
I 134	2.61E-27	9.57E-29	2.74E-27	7.26E-27	1.14E-26	1.21E-25	0.00E-01	0.00E-01
I 135	3.02E-22	9.04E-22	3.17E-22	8.16E-22	1.29E-21	5.25E-20	0.00E-01	0.00E-01
CS134	5.85E+02	1.57E+01	5.36E+02	1.26E+03	4.01E+02	0.00E-01	1.53E+02	0.00E-01
CS136	5.13E+01	6.15E+00	1.94E+01	7.64E+01	4.16E+01	0.00E-01	6.56E+00	0.00E-01
CS137	3.38E+02	1.38E+01	7.30E+02	9.71E+02	3.30E+02	0.00E-01	1.28E+02	0.00E-01
CS138	1.17E-27	1.06E-30	1.22E-27	2.35E-27	1.73E-27	0.00E-01	2.01E-28	0.00E-01
BA139	1.12E-28	3.42E-26	3.84E-27	2.70E-30	2.55E-30	0.00E-01	1.86E-30	0.00E-01
BA140	3.22E+00	7.72E+01	5.00E+01	6.13E-02	2.08E-02	0.00E-01	4.12E-02	0.00E-01
BA141	1.05E-29	6.72E-34	3.15E-28	2.35E-31	2.18E-31	0.00E-01	1.61E-31	0.00E-01
BA142	1.72E-30	8.59E-41	2.80E-29	2.80E-32	2.37E-32	0.00E-01	1.86E-32	0.00E-01
LA140	3.77E-08	8.14E-03	2.89E-07	1.42E-07	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA142	3.61E-32	4.41E-27	3.26E-31	1.45E-31	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	1.30E-03	3.24E+01	1.70E-02	1.13E-02	5.33E-03	0.00E-01	0.00E-01	0.00E-01
CE143	1.60E-08	4.30E-03	1.96E-07	1.43E-04	6.41E-08	0.00E-01	0.00E-01	0.00E-01
CE144	6.97E-02	3.26E+02	1.30E+00	5.37E-01	3.21E-01	0.00E-01	0.00E-01	0.00E-01
PR143	1.80E-03	1.19E+02	3.62E-02	1.44E-02	8.39E-03	0.00E-01	0.00E-01	0.00E-01
PR144	1.32E-32	2.86E-34	2.60E-31	1.06E-31	6.10E-32	0.00E-01	0.00E-01	0.00E-01
ND147	9.40E-04	5.66E+01	1.44E-02	1.57E-02	9.22E-03	0.00E-01	0.00E-01	0.00E-01
W 187	7.81E-08	6.03E-05	2.74E-07	2.23E-07	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NP239	8.34E-08	2.42E-02	1.59E-06	1.50E-07	4.71E-07	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = TEENAGER
PATHWAY = RIVER SHORELINE DEPOSITS

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NA 24	1.05E+01	1.05E+01	1.05E+01	1.05E+01	1.05E+01	1.05E+01	1.05E+01	1.22E+01
P 32	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	4.09E+00	4.09E+00	4.09E+00	4.09E+00	4.09E+00	4.09E+00	4.09E+00	4.84E+00
MN 54	1.22E+03	1.22E+03	1.22E+03	1.22E+03	1.22E+03	1.22E+03	1.22E+03	1.43E+03
MN 56	7.70E-01	7.70E-01	7.70E-01	7.70E-01	7.70E-01	7.70E-01	7.70E-01	9.10E-01
FE 55	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 59	2.40E+02	2.40E+02	2.40E+02	2.40E+02	2.40E+02	2.40E+02	2.40E+02	2.82E+02
CO 58	3.34E+02	3.34E+02	3.34E+02	3.34E+02	3.34E+02	3.34E+02	3.34E+02	3.91E+02
CO 60	1.89E+04	1.89E+04	1.89E+04	1.89E+04	1.89E+04	1.89E+04	1.89E+04	2.22E+04
NI 63	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	2.54E-01	2.54E-01	2.54E-01	2.54E-01	2.54E-01	2.54E-01	2.54E-01	2.95E-01
CU 64	5.29E-01	5.29E-01	5.29E-01	5.29E-01	5.29E-01	5.29E-01	5.29E-01	6.00E-01
ZN 65	6.56E+02	6.56E+02	6.56E+02	6.56E+02	6.56E+02	6.56E+02	6.56E+02	7.54E+02
ZN 69	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 83	4.16E-03	4.16E-03	4.16E-03	4.16E-03	4.16E-03	4.16E-03	4.16E-03	6.05E-03
BR 84	1.56E-01	1.56E-01	1.56E-01	1.56E-01	1.56E-01	1.56E-01	1.56E-01	1.83E-01
BR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 86	7.92E+00	7.92E+00	7.92E+00	7.92E+00	7.92E+00	7.92E+00	7.92E+00	9.05E+00
RB 88	2.31E-02	2.31E-02	2.31E-02	2.31E-02	2.31E-02	2.31E-02	2.31E-02	2.64E-02
RB 89	8.14E-02	8.14E-02	8.14E-02	8.14E-02	8.14E-02	8.14E-02	8.14E-02	9.76E-02
SR 89	1.90E-02	1.90E-02	1.90E-02	1.90E-02	1.90E-02	1.90E-02	1.90E-02	2.21E-02
SR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 91	1.87E+00	1.87E+00	1.87E+00	1.87E+00	1.87E+00	1.87E+00	1.87E+00	2.19E+00
SR 92	6.66E-01	6.66E-01	6.66E-01	6.66E-01	6.66E-01	6.66E-01	6.66E-01	7.40E-01
Y 90	3.95E-03	3.95E-03	3.95E-03	3.95E-03	3.95E-03	3.95E-03	3.95E-03	4.67E-03
Y 91M	8.09E-02	8.09E-02	8.09E-02	8.09E-02	8.09E-02	8.09E-02	8.09E-02	9.36E-02
Y 91	9.45E-01	9.45E-01	9.45E-01	9.45E-01	9.45E-01	9.45E-01	9.45E-01	1.06E+00
Y 92	1.56E-01	1.56E-01	1.56E-01	1.56E-01	1.56E-01	1.56E-01	1.56E-01	1.85E-01
Y 93	1.60E-01	1.60E-01	1.60E-01	1.60E-01	1.60E-01	1.60E-01	1.60E-01	2.19E-01
ZR 95	2.16E+02	2.16E+02	2.16E+02	2.16E+02	2.16E+02	2.16E+02	2.16E+02	2.50E+02
ZR 97	2.59E+00	2.59E+00	2.59E+00	2.59E+00	2.59E+00	2.59E+00	2.59E+00	3.02E+00
NB 95	1.21E+02	1.21E+02	1.21E+02	1.21E+02	1.21E+02	1.21E+02	1.21E+02	1.42E+02
MO 99	3.51E+00	3.51E+00	3.51E+00	3.51E+00	3.51E+00	3.51E+00	3.51E+00	4.06E+00
TC 99M	1.60E-01	1.60E-01	1.60E-01	1.60E-01	1.60E-01	1.60E-01	1.60E-01	1.83E-01
TC101	1.33E-02	1.33E-02	1.33E-02	1.33E-02	1.33E-02	1.33E-02	1.33E-02	1.48E-02

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = TEENAGER
PATHWAY = RIVER SHORELINE DEOSITS

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
RU103	9.52E+01	9.52E+01	9.52E+01	9.52E+01	9.52E+01	9.52E+01	9.52E+01	1.11E+02
RU105	5.50E-01	5.50E-01	5.50E-01	5.50E-01	5.50E-01	5.50E-01	5.50E-01	6.24E-01
RU106	3.71E+02	3.71E+02	3.71E+02	3.71E+02	3.71E+02	3.71E+02	3.71E+02	4.45E+02
AG110M	3.03E+03	3.03E+03	3.03E+03	3.03E+03	3.03E+03	3.03E+03	3.03E+03	3.54E+03
TE125M	1.37E+00	1.37E+00	1.37E+00	1.37E+00	1.37E+00	1.37E+00	1.37E+00	1.88E+00
TE127M	8.06E-02	8.06E-02	8.06E-02	8.06E-02	8.06E-02	8.06E-02	8.06E-02	9.53E-02
TE127	2.60E-03	2.60E-03	2.60E-03	2.60E-03	2.60E-03	2.60E-03	2.60E-03	2.86E-03
TE129M	1.74E+01	1.74E+01	1.74E+01	1.74E+01	1.74E+01	1.74E+01	1.74E+01	2.03E+01
TE129	2.17E-02	2.17E-02	2.17E-02	2.17E-02	2.17E-02	2.17E-02	2.17E-02	2.57E-02
TE131M	7.04E+00	7.04E+00	7.04E+00	7.04E+00	7.04E+00	7.04E+00	7.04E+00	8.30E+00
TE131	2.18E-02	2.18E-02	2.18E-02	2.18E-02	2.18E-02	2.18E-02	2.18E-02	2.58E+01
TE132	3.72E+00	3.72E+00	3.72E+00	3.72E+00	3.72E+00	3.72E+00	3.72E+00	4.38E+00
I 130	4.81E+00	4.81E+00	4.81E+00	4.81E+00	4.81E+00	4.81E+00	4.81E+00	5.84E+00
I 131	1.51E+01	1.51E+01	1.51E+01	1.51E+01	1.51E+01	1.51E+01	1.51E+01	1.84E+01
I 132	1.06E+00	1.06E+00	1.06E+00	1.06E+00	1.06E+00	1.06E+00	1.06E+00	1.25E+00
I 133	2.15E+00	2.15E+00	2.15E+00	2.15E+00	2.15E+00	2.15E+00	2.15E+00	2.62E+00
I 134	3.62E-01	3.62E-01	3.62E-01	3.62E-01	3.62E-01	3.62E-01	3.62E-01	4.30E-01
I 135	2.19E+00	2.19E+00	2.19E+00	2.19E+00	2.19E+00	2.19E+00	2.19E+00	2.56E+00
CS134	6.01E+03	6.01E+03	6.01E+03	6.01E+03	6.01E+03	6.01E+03	6.01E+03	7.01E+03
CS136	1.32E+02	1.32E+02	1.32E+02	1.32E+02	1.32E+02	1.32E+02	1.32E+02	1.50E+02
CS137	9.06E+03	9.06E+03	9.06E+03	9.06E+03	9.06E+03	9.06E+03	9.06E+03	1.06E+04
CS138	2.78E-01	2.78E-01	2.78E-01	2.78E-01	2.78E-01	2.78E-01	2.78E-01	3.18E-01
BA139	8.79E-02	8.79E-02	8.79E-02	8.79E-02	8.79E-02	8.79E-02	8.79E-02	9.89E-02
BA140	1.81E+01	1.81E+01	1.81E+01	1.81E+01	1.81E+01	1.81E+01	1.81E+01	2.06E+01
BA141	2.92E-02	2.92E-02	2.92E-02	2.92E-02	2.92E-02	2.92E-02	2.92E-02	3.33E-02
BA142	2.67E-02	2.67E-02	2.67E-02	2.67E-02	2.67E-02	2.67E-02	2.67E-02	3.05E-02
LA140	1.69E+01	1.69E+01	1.69E+01	1.69E+01	1.69E+01	1.69E+01	1.69E+01	1.91E+01
LA142	6.19E-01	6.19E-01	6.19E-01	6.19E-01	6.19E-01	6.19E-01	6.19E-01	7.43E-01
CE141	1.20E+01	1.20E+01	1.20E+01	1.20E+01	1.20E+01	1.20E+01	1.20E+01	1.35E+01
CE143	2.04E+00	2.04E+00	2.04E+00	2.04E+00	2.04E+00	2.04E+00	2.04E+00	2.32E+00
CE144	6.12E+01	6.12E+01	6.12E+01	6.12E+01	6.12E+01	6.12E+01	6.12E+01	7.08E+01
PR143	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
PR144	1.27E-03	1.27E-03	1.27E-03	1.27E-03	1.27E-03	1.27E-03	1.27E-03	1.46E-03
ND147	7.46E+00	7.46E+00	7.46E+00	7.46E+00	7.46E+00	7.46E+00	7.46E+00	8.95E+00
W 187	2.07E+00	2.07E+00	2.07E+00	2.07E+00	2.07E+00	2.07E+00	2.07E+00	2.40E+00
NP239	1.50E+00	1.50E+00	1.50E+00	1.50E+00	1.50E+00	1.50E+00	1.50E+00	1.74E+00

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = TEENAGER
PATHWAY = SWIMMING

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NA 24	1.75E+02	1.75E+02	1.75E+02	1.75E+02	1.75E+02	1.75E+02	1.75E+02	0.00E-01
P 32	1.45E-01	1.45E-01	1.45E-01	1.45E-01	1.45E-01	1.45E-01	1.45E-01	0.00E-01
CR 51	1.17E+00	1.17E+00	1.17E+00	1.17E+00	1.17E+00	1.17E+00	1.17E+00	0.00E-01
MN 54	3.39E+01	3.39E+01	3.39E+01	3.39E+01	3.39E+01	3.39E+01	3.39E+01	0.00E-01
MN 56	7.03E+01	7.03E+01	7.03E+01	7.03E+01	7.03E+01	7.03E+01	7.03E+01	0.00E-01
FE 55	1.44E-03	1.44E-03	1.44E-03	1.44E-03	1.44E-03	1.44E-03	1.44E-03	0.00E-01
FE 59	4.97E+01	4.97E+01	4.97E+01	4.97E+01	4.97E+01	4.97E+01	4.97E+01	0.00E-01
CO 58	4.06E+01	4.06E+01	4.06E+01	4.06E+01	4.06E+01	4.06E+01	4.06E+01	0.00E-01
CO 60	1.04E+02	1.04E+02	1.04E+02	1.04E+02	1.04E+02	1.04E+02	1.04E+02	0.00E-01
NI 63	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	2.20E+01	2.20E+01	2.20E+01	2.20E+01	2.20E+01	2.20E+01	2.20E+01	0.00E-01
CU 64	8.31E+00	8.31E+00	8.31E+00	8.31E+00	8.31E+00	8.31E+00	8.31E+00	0.00E-01
ZN 65	2.48E+01	2.48E+01	2.48E+01	2.48E+01	2.48E+01	2.48E+01	2.48E+01	0.00E-01
ZN 69	3.36E-02	3.36E-02	3.36E-02	3.36E-02	3.36E-02	3.36E-02	3.36E-02	0.00E-01
BR 83	3.73E-01	3.73E-01	3.73E-01	3.73E-01	3.73E-01	3.73E-01	3.73E-01	0.00E-01
BR 84	6.93E+01	6.93E+01	6.93E+01	6.93E+01	6.93E+01	6.93E+01	6.93E+01	0.00E-01
BR 85	7.41E-02	7.41E-02	7.41E-02	7.41E-02	7.41E-02	7.41E-02	7.41E-02	0.00E-01
RB 86	3.84E+00	3.84E+00	3.84E+00	3.84E+00	3.84E+00	3.84E+00	3.84E+00	0.00E-01
RB 88	2.15E+01	2.15E+01	2.15E+01	2.15E+01	2.15E+01	2.15E+01	2.15E+01	0.00E-01
RB 89	7.74E+01	7.74E+01	7.74E+01	7.74E+01	7.74E+01	7.74E+01	7.74E+01	0.00E-01
SR 89	1.04E-01	1.04E-01	1.04E-01	1.04E-01	1.04E-01	1.04E-01	1.04E-01	0.00E-01
SR 90	1.22E-02	1.22E-02	1.22E-02	1.22E-02	1.22E-02	1.22E-02	1.22E-02	0.00E-01
SR 91	4.26E+01	4.26E+01	4.26E+01	4.26E+01	4.26E+01	4.26E+01	4.26E+01	0.00E-01
SR 92	5.72E+01	5.72E+01	5.72E+01	5.72E+01	5.72E+01	5.72E+01	5.72E+01	0.00E-01
Y 90	2.93E-01	2.93E-01	2.93E-01	2.93E-01	2.93E-01	2.93E-01	2.93E-01	0.00E-01
Y 91M	2.08E+01	2.08E+01	2.08E+01	2.08E+01	2.08E+01	2.08E+01	2.08E+01	0.00E-01
Y 91	1.51E-01	1.51E-01	1.51E-01	1.51E-01	1.51E-01	1.51E-01	1.51E-01	0.00E-01
Y 92	1.02E+01	1.02E+01	1.02E+01	1.02E+01	1.02E+01	1.02E+01	1.02E+01	0.00E-01
Y 93	4.26E+00	4.26E+00	4.26E+00	4.26E+00	4.26E+00	4.26E+00	4.26E+00	0.00E-01
ZR 95	3.39E+01	3.39E+01	3.39E+01	3.39E+01	3.39E+01	3.39E+01	3.39E+01	0.00E-01
ZR 97	3.37E+01	3.37E+01	3.37E+01	3.37E+01	3.37E+01	3.37E+01	3.37E+01	0.00E-01
NB 95	3.16E+01	3.16E+01	3.16E+01	3.16E+01	3.16E+01	3.16E+01	3.16E+01	0.00E-01
MO 99	1.06E+01	1.06E+01	1.06E+01	1.06E+01	1.06E+01	1.06E+01	1.06E+01	0.00E-01
TC 99M	5.36E+00	5.36E+00	5.36E+00	5.36E+00	5.36E+00	5.36E+00	5.36E+00	0.00E-01
TC101	1.14E+01	1.14E+01	1.14E+01	1.14E+01	1.14E+01	1.14E+01	1.14E+01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = TEENAGER
PATHWAY = SWIMMING

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
RU103	2.01E+01	2.01E+01	2.01E+01	2.01E+01	2.01E+01	2.01E+01	2.01E+01	0.00E-01
RU105	2.67E+01	2.67E+01	2.67E+01	2.67E+01	2.67E+01	2.67E+01	2.67E+01	0.00E-01
RU106	8.58E+00	8.58E+00	8.58E+00	8.58E+00	8.58E+00	8.58E+00	8.58E+00	0.00E-01
AG110M	1.11E+02	1.11E+02	1.11E+02	1.11E+02	1.11E+02	1.11E+02	1.11E+02	0.00E-01
TE125M	8.33E-02	8.33E-02	8.33E-02	8.33E-02	8.33E-02	8.33E-02	8.33E-02	0.00E-01
TE127M	5.87E-03	5.87E-03	5.87E-03	5.87E-03	5.87E-03	5.87E-03	5.87E-03	0.00E-01
TE127	6.28E-02	6.28E-02	6.28E-02	6.28E-02	6.28E-02	6.28E-02	6.28E-02	0.00E-01
TE129M	4.74E+00	4.74E+00	4.74E+00	4.74E+00	4.74E+00	4.74E+00	4.74E+00	0.00E-01
TE129	4.04E+00	4.04E+00	4.04E+00	4.04E+00	4.04E+00	4.04E+00	4.04E+00	0.00E-01
TE131M	4.96E+01	4.96E+01	4.96E+01	4.96E+01	4.96E+01	4.96E+01	4.96E+01	0.00E-01
TE131	1.41E+01	1.41E+01	1.41E+01	1.41E+01	1.41E+01	1.41E+01	1.41E+01	0.00E-01
TE132	9.02E+00	9.02E+00	9.02E+00	9.02E+00	9.02E+00	9.02E+00	9.02E+00	0.00E-01
I 130	8.76E+01	8.76E+01	8.76E+01	8.76E+01	8.76E+01	8.76E+01	8.76E+01	0.00E-01
I 131	1.76E+01	1.76E+01	1.76E+01	1.76E+01	1.76E+01	1.76E+01	1.76E+01	0.00E-01
I 132	9.64E+01	9.64E+01	9.64E+01	9.64E+01	9.64E+01	9.64E+01	9.64E+01	0.00E-01
I 133	2.16E+01	2.16E+01	2.16E+01	2.16E+01	2.16E+01	2.16E+01	2.16E+01	0.00E-01
I 134	8.76E+01	8.76E+01	8.76E+01	8.76E+01	8.76E+01	8.76E+01	8.76E+01	0.00E-01
I 135	7.37E+01	7.37E+01	7.37E+01	7.37E+01	7.37E+01	7.37E+01	7.37E+01	0.00E-01
CS134	6.55E+01	6.55E+01	6.55E+01	6.55E+01	6.55E+01	6.55E+01	6.55E+01	0.00E-01
CS136	9.26E+01	9.26E+01	9.26E+01	9.26E+01	9.26E+01	9.26E+01	9.26E+01	0.00E-01
CS137	2.26E+01	2.26E+01	2.26E+01	2.26E+01	2.26E+01	2.26E+01	2.26E+01	0.00E-01
CS138	7.94E+01	7.94E+01	7.94E+01	7.94E+01	7.94E+01	7.94E+01	7.94E+01	0.00E-01
BA139	1.65E+00	1.65E+00	1.65E+00	1.65E+00	1.65E+00	1.65E+00	1.65E+00	0.00E-01
BA140	1.11E+01	1.11E+01	1.11E+01	1.11E+01	1.11E+01	1.11E+01	1.11E+01	0.00E-01
BA141	1.98E+01	1.98E+01	1.98E+01	1.98E+01	1.98E+01	1.98E+01	1.98E+01	0.00E-01
BA142	3.37E+01	3.37E+01	3.37E+01	3.37E+01	3.37E+01	3.37E+01	3.37E+01	0.00E-01
LA140	9.24E+01	9.24E+01	9.24E+01	9.24E+01	9.24E+01	9.24E+01	9.24E+01	0.00E-01
LA142	9.71E+01	9.71E+01	9.71E+01	9.71E+01	9.71E+01	9.71E+01	9.71E+01	0.00E-01
CE141	2.93E+00	2.93E+00	2.93E+00	2.93E+00	2.93E+00	2.93E+00	2.93E+00	0.00E-01
CE143	1.28E+01	1.28E+01	1.28E+01	1.28E+01	1.28E+01	1.28E+01	1.28E+01	0.00E-01
CE144	1.94E+00	1.94E+00	1.94E+00	1.94E+00	1.94E+00	1.94E+00	1.94E+00	0.00E-01
PR143	3.61E-02	3.61E-02	3.61E-02	3.61E-02	3.61E-02	3.61E-02	3.61E-02	0.00E-01
PR144	9.94E-01	9.94E-01	9.94E-01	9.94E-01	9.94E-01	9.94E-01	9.94E-01	0.00E-01
ND147	6.32E+00	6.32E+00	6.32E+00	6.32E+00	6.32E+00	6.32E+00	6.32E+00	0.00E-01
W 187	1.87E+01	1.87E+01	1.87E+01	1.87E+01	1.87E+01	1.87E+01	1.87E+01	0.00E-01
NP239	5.41E+00	5.41E+00	5.41E+00	5.41E+00	5.41E+00	5.41E+00	5.41E+00	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = TEENAGER
PATHWAY = BOATING

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NA 24	1.01E+02	1.01E+02	1.01E+02	1.01E+02	1.01E+02	1.01E+02	1.01E+02	0.00E-01
P 32	8.35E-02	8.35E-02	8.35E-02	8.35E-02	8.35E-02	8.35E-02	8.35E-02	0.00E-01
CR 51	6.78E-01	6.78E-01	6.78E-01	6.78E-01	6.78E-01	6.78E-01	6.78E-01	0.00E-01
MN 54	1.96E+01	1.96E+01	1.96E+01	1.96E+01	1.96E+01	1.96E+01	1.96E+01	0.00E-01
MN 56	4.06E+01	4.06E+01	4.06E+01	4.06E+01	4.06E+01	4.06E+01	4.06E+01	0.00E-01
FE 55	8.35E-04	8.35E-04	8.35E-04	8.35E-04	8.35E-04	8.35E-04	8.35E-04	0.00E-01
FE 59	2.87E+01	2.87E+01	2.87E+01	2.87E+01	2.87E+01	2.87E+01	2.87E+01	0.00E-01
CO 58	2.35E+01	2.35E+01	2.35E+01	2.35E+01	2.35E+01	2.35E+01	2.35E+01	0.00E-01
CO 60	6.00E+01	6.00E+01	6.00E+01	6.00E+01	6.00E+01	6.00E+01	6.00E+01	0.00E-01
NI 63	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	1.27E+01	1.27E+01	1.27E+01	1.27E+01	1.27E+01	1.27E+01	1.27E+01	0.00E-01
CU 64	4.80E+00	4.80E+00	4.80E+00	4.80E+00	4.80E+00	4.80E+00	4.80E+00	0.00E-01
ZN 65	1.44E+01	1.44E+01	1.44E+01	1.44E+01	1.44E+01	1.44E+01	1.44E+01	0.00E-01
ZN 69	1.94E-02	1.94E-02	1.94E-02	1.94E-02	1.94E-02	1.94E-02	1.94E-02	0.00E-01
BR 83	2.15E-01	2.15E-01	2.15E-01	2.15E-01	2.15E-01	2.15E-01	2.15E-01	0.00E-01
BR 84	4.01E+01	4.01E+01	4.01E+01	4.01E+01	4.01E+01	4.01E+01	4.01E+01	0.00E-01
BR 85	4.28E-02	4.28E-02	4.28E-02	4.28E-02	4.28E-02	4.28E-02	4.28E-02	0.00E-01
RB 86	2.22E+00	2.22E+00	2.22E+00	2.22E+00	2.22E+00	2.22E+00	2.22E+00	0.00E-01
RB 88	1.24E+01	1.24E+01	1.24E+01	1.24E+01	1.24E+01	1.24E+01	1.24E+01	0.00E-01
RB 89	4.47E+01	4.47E+01	4.47E+01	4.47E+01	4.47E+01	4.47E+01	4.47E+01	0.00E-01
SR 89	6.00E-02	6.00E-02	6.00E-02	6.00E-02	6.00E-02	6.00E-02	6.00E-02	0.00E-01
SR 90	7.04E-03	7.04E-03	7.04E-03	7.04E-03	7.04E-03	7.04E-03	7.04E-03	0.00E-01
SR 91	2.46E+01	2.46E+01	2.46E+01	2.46E+01	2.46E+01	2.46E+01	2.46E+01	0.00E-01
SR 92	3.31E+01	3.31E+01	3.31E+01	3.31E+01	3.31E+01	3.31E+01	3.31E+01	0.00E-01
Y 90	1.69E-01	1.69E-01	1.69E-01	1.69E-01	1.69E-01	1.69E-01	1.69E-01	0.00E-01
Y 91M	1.20E+01	1.20E+01	1.20E+01	1.20E+01	1.20E+01	1.20E+01	1.20E+01	0.00E-01
Y 91	8.74E-02	8.74E-02	8.74E-02	8.74E-02	8.74E-02	8.74E-02	8.74E-02	0.00E-01
Y 92	5.89E+00	5.89E+00	5.89E+00	5.89E+00	5.89E+00	5.89E+00	5.89E+00	0.00E-01
Y 93	2.46E+00	2.46E+00	2.46E+00	2.46E+00	2.46E+00	2.46E+00	2.46E+00	0.00E-01
ZR 95	1.96E+01	1.96E+01	1.96E+01	1.96E+01	1.96E+01	1.96E+01	1.96E+01	0.00E-01
ZR 97	1.95E+01	1.95E+01	1.95E+01	1.95E+01	1.95E+01	1.95E+01	1.95E+01	0.00E-01
NB 95	1.83E+01	1.83E+01	1.83E+01	1.83E+01	1.83E+01	1.83E+01	1.83E+01	0.00E-01
MO 99	6.12E+00	6.12E+00	6.12E+00	6.12E+00	6.12E+00	6.12E+00	6.12E+00	0.00E-01
TC 99M	3.10E+00	3.10E+00	3.10E+00	3.10E+00	3.10E+00	3.10E+00	3.10E+00	0.00E-01
TC101	6.62E+00	6.62E+00	6.62E+00	6.62E+00	6.62E+00	6.62E+00	6.62E+00	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = TEENAGER
PATHWAY = BOATING

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
RU103	1.16E+01	1.16E+01	1.16E+01	1.16E+01	1.16E+01	1.16E+01	1.16E+01	0.00E-01
RU105	1.54E+01	1.54E+01	1.54E+01	1.54E+01	1.54E+01	1.54E+01	1.54E+01	0.00E-01
RU106	4.96E+00	4.96E+00	4.96E+00	4.96E+00	4.96E+00	4.96E+00	4.96E+00	0.00E-01
AG110M	6.39E+01	6.39E+01	6.39E+01	6.39E+01	6.39E+01	6.39E+01	6.39E+01	0.00E-01
TE125M	4.81E-02	4.81E-02	4.81E-02	4.81E-02	4.81E-02	4.81E-02	4.81E-02	0.00E-01
TE127M	3.39E-03	3.39E-03	3.39E-03	3.39E-03	3.39E-03	3.39E-03	3.39E-03	0.00E-01
TE127	3.63E-02	3.63E-02	3.63E-02	3.63E-02	3.63E-02	3.63E-02	3.63E-02	0.00E-01
TE129M	2.74E+00	2.74E+00	2.74E+00	2.74E+00	2.74E+00	2.74E+00	2.74E+00	0.00E-01
TE129	2.34E+00	2.34E+00	2.34E+00	2.34E+00	2.34E+00	2.34E+00	2.34E+00	0.00E-01
TE131M	2.86E+01	2.86E+01	2.86E+01	2.86E+01	2.86E+01	2.86E+01	2.86E+01	0.00E-01
TE131	8.18E+00	8.18E+00	8.18E+00	8.18E+00	8.18E+00	8.18E+00	8.18E+00	0.00E-01
TE132	5.21E+00	5.21E+00	5.21E+00	5.21E+00	5.21E+00	5.21E+00	5.21E+00	0.00E-01
I 130	5.06E+01	5.06E+01	5.06E+01	5.06E+01	5.06E+01	5.06E+01	5.06E+01	0.00E-01
I 131	1.02E+01	1.02E+01	1.02E+01	1.02E+01	1.02E+01	1.02E+01	1.02E+01	0.00E-01
I 132	5.57E+01	5.57E+01	5.57E+01	5.57E+01	5.57E+01	5.57E+01	5.57E+01	0.00E-01
I 133	1.25E+01	1.25E+01	1.25E+01	1.25E+01	1.25E+01	1.25E+01	1.25E+01	0.00E-01
I 134	5.06E+01	5.06E+01	5.06E+01	5.06E+01	5.06E+01	5.06E+01	5.06E+01	0.00E-01
I 135	4.26E+01	4.26E+01	4.26E+01	4.26E+01	4.26E+01	4.26E+01	4.26E+01	0.00E-01
CS134	3.78E+01	3.78E+01	3.78E+01	3.78E+01	3.78E+01	3.78E+01	3.78E+01	0.00E-01
CS136	5.35E+01	5.35E+01	5.35E+01	5.35E+01	5.35E+01	5.35E+01	5.35E+01	0.00E-01
CS137	1.31E+01	1.31E+01	1.31E+01	1.31E+01	1.31E+01	1.31E+01	1.31E+01	0.00E-01
CS138	4.59E+01	4.59E+01	4.59E+01	4.59E+01	4.59E+01	4.59E+01	4.59E+01	0.00E-01
BA139	9.55E-01	9.55E-01	9.55E-01	9.55E-01	9.55E-01	9.55E-01	9.55E-01	0.00E-01
BA140	6.39E+00	6.39E+00	6.39E+00	6.39E+00	6.39E+00	6.39E+00	6.39E+00	0.00E-01
BA141	1.14E+01	1.14E+01	1.14E+01	1.14E+01	1.14E+01	1.14E+01	1.14E+01	0.00E-01
BA142	1.95E+01	1.95E+01	1.95E+01	1.95E+01	1.95E+01	1.95E+01	1.95E+01	0.00E-01
LA140	5.34E+01	5.34E+01	5.34E+01	5.34E+01	5.34E+01	5.34E+01	5.34E+01	0.00E-01
LA142	5.61E+01	5.61E+01	5.61E+01	5.61E+01	5.61E+01	5.61E+01	5.61E+01	0.00E-01
CE141	1.70E+00	1.70E+00	1.70E+00	1.70E+00	1.70E+00	1.70E+00	1.70E+00	0.00E-01
CE143	7.42E+00	7.42E+00	7.42E+00	7.42E+00	7.42E+00	7.42E+00	7.42E+00	0.00E-01
CE144	1.12E+00	1.12E+00	1.12E+00	1.12E+00	1.12E+00	1.12E+00	1.12E+00	0.00E-01
PR143	2.09E-02	2.09E-02	2.09E-02	2.09E-02	2.09E-02	2.09E-02	2.09E-02	0.00E-01
PR144	5.74E-01	5.74E-01	5.74E-01	5.74E-01	5.74E-01	5.74E-01	5.74E-01	0.00E-01
ND147	3.65E+00	3.65E+00	3.65E+00	3.65E+00	3.65E+00	3.65E+00	3.65E+00	0.00E-01
W 187	1.08E+01	1.08E+01	1.08E+01	1.08E+01	1.08E+01	1.08E+01	1.08E+01	0.00E-01
NP239	3.13E+00	3.13E+00	3.13E+00	3.13E+00	3.13E+00	3.13E+00	3.13E+00	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = CHILD
PATHWAY = POTABLE WATER

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	2.97E+01	2.97E+01	0.00E-01	2.97E+01	2.97E+01	2.97E+01	2.97E+01	0.00E-01
C 14	6.19E+02	6.19E+02	3.10E+03	6.19E+02	6.19E+02	6.19E+02	6.19E+02	0.00E-01
NA 24	4.91E+02	4.91E+02	4.91E+02	4.91E+02	4.91E+02	4.91E+02	4.91E+02	0.00E-01
P 32	7.75E+03	5.56E+03	2.01E+05	9.41E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	2.22E+00	1.18E+02	0.00E-01	0.00E-01	3.37E-01	1.23E+00	2.25E+00	0.00E-01
MN 54	7.28E+02	2.29E+03	0.00E-01	2.73E+03	7.66E+02	0.00E-01	0.00E-01	0.00E-01
MN 56	2.96E-02	1.90E+01	0.00E-01	1.31E-01	1.59E-01	0.00E-01	0.00E-01	0.00E-01
FE 55	4.83E+02	2.89E+02	2.94E+03	1.56E+03	0.00E-01	0.00E-01	8.82E+02	0.00E-01
FE 59	3.35E+03	7.00E+03	4.16E+03	6.73E+03	0.00E-01	0.00E-01	1.95E+03	0.00E-01
CO 58	1.40E+03	2.66E+03	0.00E-01	4.56E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	3.99E+03	7.50E+03	0.00E-01	1.35E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 63	4.68E+03	4.96E+02	1.38E+05	7.37E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	4.24E-02	8.90E+00	7.72E-01	7.27E-02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CU 64	1.02E+01	7.91E+02	0.00E-01	1.69E+01	4.07E+01	0.00E-01	0.00E-01	0.00E-01
ZN 65	5.79E+03	1.64E+03	3.50E+03	9.31E+03	5.87E+03	0.00E-01	0.00E-01	0.00E-01
ZN 69	3.61E-08	2.47E-05	2.71E-07	3.91E-07	2.37E-07	0.00E-01	0.00E-01	0.00E-01
BR 83	4.17E-02	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 84	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 86	1.02E+04	1.06E+03	0.00E-01	1.65E+04	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	9.52E+03	1.29E+04	3.33E+05	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	1.32E+05	5.86E+04	6.55E+06	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 91	4.01E+01	2.35E+03	1.06E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 92	2.01E-01	9.48E+01	5.01E+00	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 90	2.17E-01	2.31E+04	8.12E+00	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91	4.07E+00	2.03E+04	1.52E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 92	2.44E-04	2.46E+02	8.52E-03	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 93	1.54E-02	8.35E+03	5.60E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	5.75E+00	6.73E+03	2.94E+01	6.46E+00	9.24E+00	0.00E-01	0.00E-01	0.00E-01
ZR 97	5.70E-02	1.46E+04	6.68E-01	9.65E-02	1.39E-01	0.00E-01	0.00E-01	0.00E-01
NB 95	1.57E+00	4.06E+03	5.65E+00	2.20E+00	2.07E+00	0.00E-01	0.00E-01	0.00E-01
MO 99	6.54E+02	2.19E+03	0.00E-01	2.64E+03	5.65E+03	0.00E-01	0.00E-01	0.00E-01
TC 99M	4.84E-01	1.66E+01	1.49E-02	2.92E-02	4.24E-01	0.00E-01	1.48E-02	0.00E-01
TC101	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = CHILD
PATHWAY = POTABLE WATER

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
RU103	7.07E+01	4.75E+03	1.84E+02	0.00E-01	4.63E+02	0.00E-01	0.00E-01	0.00E-01
RU105	1.41E-01	2.53E+02	3.88E-01	0.00E-01	3.41E+00	0.00E-01	0.00E-01	0.00E-01
RU106	3.73E+02	4.65E+04	2.99E+03	0.00E-01	4.04E+03	0.00E-01	0.00E-01	0.00E-01
AG110M	7.43E+01	1.10E+04	1.38E+02	9.29E+01	1.73E+02	0.00E-01	0.00E-01	0.00E-01
TE125M	3.84E+02	2.78E+03	2.88E+03	7.81E+02	0.00E-01	8.09E+02	0.00E-01	0.00E-01
TE127M	8.72E+02	5.95E+03	7.35E+03	1.98E+03	2.09E+04	1.76E+03	0.00E-01	0.00E-01
TE127	4.36E+00	7.94E+02	2.03E+01	5.48E+00	5.78E+01	1.41E+01	0.00E-01	0.00E-01
TE129M	1.89E+03	1.49E+04	1.22E+04	3.41E+03	3.58E+04	3.94E+03	0.00E-01	0.00E-01
TE129	4.81E-06	1.26E-03	2.02E-05	5.65E-06	5.92E-05	1.44E-05	0.00E-01	0.00E-01
TE131M	3.89E+02	1.48E+04	1.06E+03	3.66E+02	3.54E+03	7.52E+02	0.00E-01	0.00E-01
TE131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE132	1.12E+03	9.31E+03	2.09E+03	9.25E+02	8.59E+03	1.35E+03	0.00E-01	0.00E-01
I 130	2.02E+02	1.83E+02	1.94E+02	3.92E+02	5.86E+02	4.32E+04	0.00E-01	0.00E-01
I 131	2.31E+03	3.61E+02	4.04E+03	4.06E+03	6.67E+03	1.34E+06	0.00E-01	0.00E-01
I 132	1.25E-01	3.20E-01	1.48E-01	2.72E-01	4.17E-01	1.26E+01	0.00E-01	0.00E-01
I 133	3.19E+02	3.39E+02	6.81E+02	8.42E+02	1.40E+03	1.56E+05	0.00E-01	0.00E-01
I 134	5.09E-07	7.34E-07	5.96E-07	1.11E-06	1.69E-06	2.55E-05	0.00E-01	0.00E-01
I 135	3.06E+01	4.93E+01	3.59E+01	6.47E+01	9.92E+01	5.73E+03	0.00E-01	0.00E-01
CS134	2.07E+04	5.29E+02	5.98E+04	9.82E+04	3.04E+04	0.00E-01	1.09E+04	0.00E-01
CS136	1.01E+04	5.51E+02	5.70E+03	1.57E+04	8.35E+03	0.00E-01	1.24E+03	0.00E-01
CS137	1.18E+04	5.02E+02	8.37E+04	8.01E+04	2.61E+04	0.00E-01	9.39E+03	0.00E-01
CS138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA139	1.71E-05	3.41E-02	5.91E-04	3.16E-07	2.76E-07	0.00E-01	1.86E-07	0.00E-01
BA140	1.18E+03	1.02E+04	2.01E+04	1.76E+01	5.74E+00	0.00E-01	1.05E+01	0.00E-01
BA141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA142	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA140	2.01E-01	1.67E+04	1.71E+00	5.98E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA142	2.73E-07	1.73E-01	2.74E-06	8.72E-07	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	7.37E-01	6.19E+03	9.94E+00	4.96E+00	2.17E+00	0.00E-01	0.00E-01	0.00E-01
CE143	8.51E-02	8.60E+03	1.08E+00	5.87E+02	2.46E-01	0.00E-01	0.00E-01	0.00E-01
CE144	2.83E+01	4.34E+04	5.31E+02	1.66E+02	9.22E+01	0.00E-01	0.00E-01	0.00E-01
PR143	4.74E-01	1.03E+04	9.56E+00	2.87E+00	1.55E+00	0.00E-01	0.00E-01	0.00E-01
PR144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ND147	4.21E-01	8.61E+03	6.71E+00	5.43E+00	2.98E+00	0.00E-01	0.00E-01	0.00E-01
W 187	1.46E+01	4.56E+03	5.48E+01	3.24E+01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NP239	5.05E-02	5.32E+03	1.00E+00	7.19E-02	2.08E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = CHILD
PATHWAY = FRESH WATER FISH

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	3.61E-01	3.61E-01	0.00E-01	3.61E-01	3.61E-01	3.61E-01	3.61E-01	0.00E-01
C 14	3.85E+04	3.85E+04	1.93E+05	3.85E+04	3.85E+04	3.85E+04	3.85E+04	0.00E-01
NA 24	6.61E+02	6.61E+02	6.61E+02	6.61E+02	6.61E+02	6.61E+02	6.61E+02	0.00E-01
P 32	1.05E+07	7.52E+06	2.72E+08	1.27E+07	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	6.01E+00	3.19E+02	0.00E-01	0.00E-01	9.12E-01	3.34E+00	6.09E+00	0.00E-01
MN 54	3.94E+03	1.24E+04	0.00E-01	1.48E+04	4.15E+03	0.00E-01	0.00E-01	0.00E-01
MN 56	1.56E-01	1.00E+02	0.00E-01	6.91E-01	8.35E-01	0.00E-01	0.00E-01	0.00E-01
FE 55	6.54E+02	3.91E+02	3.98E+03	2.11E+03	0.00E-01	0.00E-01	1.19E+03	0.00E-01
FE 59	4.53E+03	9.48E+03	5.62E+03	9.10E+03	0.00E-01	0.00E-01	2.64E+03	0.00E-01
CO 58	9.45E+02	1.80E+03	0.00E-01	3.09E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	2.70E+03	5.07E+03	0.00E-01	9.15E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 63	6.34E+03	6.72E+02	1.86E+05	9.97E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	5.58E-02	1.17E+01	1.02E+00	9.57E-02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CU 64	6.85E+00	5.32E+02	0.00E-01	1.13E+01	2.74E+01	0.00E-01	0.00E-01	0.00E-01
ZN 65	1.57E+05	4.43E+04	9.46E+04	2.52E+05	1.59E+05	0.00E-01	0.00E-01	0.00E-01
ZN 69	9.09E-07	6.20E-04	6.81E-06	9.84E-06	5.97E-06	0.00E-01	0.00E-01	0.00E-01
BR 83	2.30E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 84	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 86	2.75E+05	2.88E+04	0.00E-01	4.47E+05	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	3.86E+03	5.23E+03	1.35E+05	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	5.35E+04	2.38E+04	2.66E+06	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 91	1.62E+01	9.46E+02	4.28E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 92	7.94E-02	3.75E+01	1.98E+00	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 90	7.34E-02	7.81E+03	2.74E+00	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91	1.38E+00	6.86E+03	5.15E+01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 92	8.09E-05	8.17E+01	2.83E-03	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 93	5.17E-03	2.81E+03	1.88E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	2.57E-01	3.01E+02	1.31E+00	2.88E-01	4.12E-01	0.00E-01	0.00E-01	0.00E-01
ZR 97	2.53E-03	6.50E+02	2.97E-02	4.29E-03	6.16E-03	0.00E-01	0.00E-01	0.00E-01
NB 95	6.37E+02	1.65E+06	2.29E+03	8.92E+02	8.38E+02	0.00E-01	0.00E-01	0.00E-01
MO 99	8.84E+01	2.96E+02	0.00E-01	3.57E+02	7.63E+02	0.00E-01	0.00E-01	0.00E-01
TC 99M	9.70E-02	3.33E+00	2.98E-03	5.85E-03	8.51E-02	0.00E-01	2.97E-03	0.00E-01
TC101	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = CHILD
PATHWAY = FRESH WATER FISH

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
RU103	9.56E+00	6.43E+02	2.49E+01	0.00E-01	6.26E+01	0.00E-01	0.00E-01	0.00E-01
RU105	1.88E-02	3.38E+01	5.17E-02	0.00E-01	4.55E-01	0.00E-01	0.00E-01	0.00E-01
RU106	5.05E+01	6.29E+03	4.04E+02	0.00E-01	5.46E+02	0.00E-01	0.00E-01	0.00E-01
AG110M	2.31E+00	3.44E+02	4.28E+00	2.89E+00	5.38E+00	0.00E-01	0.00E-01	0.00E-01
TE125M	2.08E+03	1.50E+04	1.56E+04	4.23E+03	0.00E-01	4.38E+03	0.00E-01	0.00E-01
TE127M	4.72E+03	3.22E+04	3.98E+04	1.07E+04	1.13E+05	9.51E+03	0.00E-01	0.00E-01
TE127	2.34E+01	4.27E+03	1.09E+02	2.95E+01	3.11E+02	7.56E+01	0.00E-01	0.00E-01
TE129M	1.03E+04	8.06E+04	6.61E+04	1.84E+04	1.94E+05	2.13E+04	0.00E-01	0.00E-01
TE129	2.45E-05	6.42E-03	1.03E-04	2.88E-05	3.02E-04	7.36E-05	0.00E-01	0.00E-01
TE131M	2.10E+03	8.01E+04	5.71E+03	1.98E+03	1.91E+04	4.06E+03	0.00E-01	0.00E-01
TE131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE132	6.04E+03	5.03E+04	1.13E+04	5.00E+03	4.64E+04	7.28E+03	0.00E-01	0.00E-01
I 130	4.08E+01	3.70E+01	3.92E+01	7.92E+01	1.18E+02	8.72E+03	0.00E-01	0.00E-01
I 131	4.68E+02	7.33E+01	8.19E+02	8.24E+02	1.35E+03	2.72E+05	0.00E-01	0.00E-01
I 132	2.46E-02	6.31E-02	2.92E-02	5.36E-02	8.20E-02	2.49E+00	0.00E-01	0.00E-01
I 133	6.45E+01	6.87E+01	1.38E+02	1.70E+02	2.84E+02	3.17E+04	0.00E-01	0.00E-01
I 134	9.55E-08	1.38E-07	1.12E-07	2.07E-07	3.17E-07	4.77E-06	0.00E-01	0.00E-01
I 135	6.14E+00	9.90E+00	7.22E+00	1.30E+01	1.99E+01	1.15E+03	0.00E-01	0.00E-01
CS134	5.60E+05	1.43E+04	1.62E+06	2.66E+06	8.23E+05	0.00E-01	2.95E+05	0.00E-01
CS136	2.74E+05	1.49E+04	1.54E+05	4.24E+05	2.26E+05	0.00E-01	3.37E+04	0.00E-01
CS137	3.20E+05	1.36E+04	2.26E+06	2.17E+06	7.06E+05	0.00E-01	2.54E+05	0.00E-01
CS138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA139	8.82E-07	1.76E-03	3.04E-05	1.62E-08	1.42E-08	0.00E-01	9.56E-09	0.00E-01
BA140	6.36E+01	5.52E+02	1.09E+03	9.55E-01	3.11E-01	0.00E-01	5.69E-01	0.00E-01
BA141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA142	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA140	6.80E-02	5.63E+03	5.77E-01	2.02E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA142	8.83E-08	5.59E-02	8.85E-07	2.82E-07	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	9.96E-03	8.37E+01	1.34E-01	6.71E-02	2.94E-02	0.00E-01	0.00E-01	0.00E-01
CE143	1.15E-03	1.16E+02	1.46E-02	7.93E+00	3.32E-03	0.00E-01	0.00E-01	0.00E-01
CE144	3.83E-01	5.87E+02	7.18E+00	2.25E+00	1.25E+00	0.00E-01	0.00E-01	0.00E-01
PR143	1.60E-01	3.49E+03	3.23E+00	9.70E-01	5.25E-01	0.00E-01	0.00E-01	0.00E-01
PR144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ND147	1.42E-01	2.91E+03	2.27E+00	1.84E+00	1.01E+00	0.00E-01	0.00E-01	0.00E-01
W 187	2.36E+02	7.38E+04	8.86E+02	5.25E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NP239	6.83E-03	7.19E+02	1.35E-01	9.72E-03	2.81E-02	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = CHILD
PATHWAY = ANIMAL DRINKING WATER—MEAT

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem gal)/(Ci min)				
H 3	1.19E+00	1.19E+00	0.00E-01	1.19E+00	1.19E+00	1.19E+00	1.19E+00	0.00E-01
C 14	7.72E+01	7.72E+01	3.86E+02	7.72E+01	7.72E+01	7.72E+01	7.72E+01	0.00E-01
NA 24	4.24E-08	4.24E-08	4.24E-08	4.24E-08	4.24E-08	4.24E-08	4.24E-08	0.00E-01
P 32	5.70E+02	4.08E+02	1.48E+04	6.91E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	1.33E-02	7.05E-01	0.00E-01	0.00E-01	2.02E-03	7.38E-03	1.35E-02	0.00E-01
MN 54	2.24E+00	7.07E+00	0.00E-01	8.42E+00	2.36E+00	0.00E-01	0.00E-01	0.00E-01
MN 56	4.15E-28	2.66E-25	0.00E-01	1.84E-27	2.22E-27	0.00E-01	0.00E-01	0.00E-01
FE 55	7.67E+01	4.58E+01	4.67E+02	2.47E+02	0.00E-01	0.00E-01	1.40E+02	0.00E-01
FE 59	4.01E+02	8.38E+02	4.97E+02	8.04E+02	0.00E-01	0.00E-01	2.33E+02	0.00E-01
CO 58	6.06E+01	1.15E+02	0.00E-01	1.98E+01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	2.07E+02	3.89E+02	0.00E-01	7.02E+01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 63	9.97E+02	1.06E+02	2.93E+04	1.57E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	4.42E-26	9.28E-24	8.05E-25	7.58E-26	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CU 64	4.52E-12	3.51E-10	0.00E-01	7.48E-12	1.81E-11	0.00E-01	0.00E-01	0.00E-01
ZN 65	6.62E+02	1.87E+02	3.99E+02	1.06E+03	6.70E+02	0.00E-01	0.00E-01	0.00E-01
ZN 69	7.61E-28	5.19E-25	5.70E-27	8.24E-27	5.00E-27	0.00E-01	0.00E-01	0.00E-01
BR 83	3.00E-26	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 84	1.25E-26	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 85	1.07E-33	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 86	6.25E+02	6.54E+01	0.00E-01	1.02E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 88	3.59E-27	2.53E-28	0.00E-01	5.17E-27	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 89	1.90E-27	1.87E-29	0.00E-01	2.14E-27	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	1.77E+01	2.40E+01	6.19E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	3.17E+02	1.41E+02	1.58E+04	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 91	3.03E-16	1.77E-14	8.03E-15	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 92	1.51E-27	7.16E-25	3.78E-26	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 90	2.89E-05	3.07E+00	1.08E-03	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91M	2.49E-31	1.34E-26	6.84E-30	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91	6.01E-02	2.99E+02	2.25E+00	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 92	3.51E-30	3.55E-24	1.23E-28	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 93	6.41E-18	3.48E-12	2.34E-16	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	6.39E-01	7.49E+02	3.27E+00	7.18E-01	1.03E+00	0.00E-01	0.00E-01	0.00E-01
ZR 97	5.57E-11	1.43E-05	6.53E-10	9.44E-11	1.36E-10	0.00E-01	0.00E-01	0.00E-01
NB 95	1.22E+00	3.14E+03	4.37E+00	1.70E+00	1.60E+00	0.00E-01	0.00E-01	0.00E-01
MO 99	1.72E-01	5.77E-01	0.00E-01	6.97E-01	1.49E+00	0.00E-01	0.00E-01	0.00E-01
TC 99M	1.06E-23	3.65E-22	3.27E-25	6.42E-25	9.32E-24	0.00E-01	3.26E-25	0.00E-01
TC101	2.73E-27	6.85E-28	2.06E-28	2.15E-28	3.67E-27	0.00E-01	1.14E-28	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = CHILD
PATHWAY = ANIMAL DRINKING WATER—MEAT

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
RU103	8.12E+01	5.46E+03	2.11E+02	0.00E-01	5.32E+02	0.00E-01	0.00E-01	0.00E-01
RU105	7.21E-26	1.30E-22	1.99E-25	0.00E-01	1.75E-24	0.00E-01	0.00E-01	0.00E-01
RU106	5.78E+02	7.21E+04	4.64E+03	0.00E-01	6.26E+03	0.00E-01	0.00E-01	0.00E-01
AG110M	4.82E+00	7.16E+02	8.92E+00	6.02E+00	1.12E+01	0.00E-01	0.00E-01	0.00E-01
TE125M	9.48E+01	6.86E+02	7.11E+02	1.93E+02	0.00E-01	2.00E+02	0.00E-01	0.00E-01
TE127M	2.39E+02	1.63E+03	2.01E+03	5.42E+02	5.75E+03	4.82E+02	0.00E-01	0.00E-01
TE127	2.58E-15	4.70E-13	1.20E-14	3.24E-15	3.42E-14	8.32E-15	0.00E-01	0.00E-01
TE129M	3.96E+02	3.11E+03	2.55E+03	7.12E+02	7.49E+03	8.22E+02	0.00E-01	0.00E-01
TE129	1.21E-26	3.18E-24	5.11E-26	1.43E-26	1.50E-25	3.65E-26	0.00E-01	0.00E-01
TE131M	3.12E-03	1.19E-01	8.47E-03	2.93E-03	2.84E-02	6.03E-03	0.00E-01	0.00E-01
TE131	3.26E-27	5.75E-26	1.10E-26	3.34E-27	3.31E-26	8.38E-27	0.00E-01	0.00E-01
TE132	6.04E+00	5.03E+01	1.13E+01	5.00E+00	4.64E+01	7.28E+00	0.00E-01	0.00E-01
I 130	1.68E-11	1.53E-11	1.62E-11	3.27E-11	4.88E-11	3.60E-09	0.00E-01	0.00E-01
I 131	5.21E+00	8.16E-01	9.11E+00	9.17E+00	1.51E+01	3.03E+03	0.00E-01	0.00E-01
I 132	1.31E-26	3.34E-26	1.55E-26	2.84E-26	4.35E-26	1.32E-24	0.00E-01	0.00E-01
I 133	9.14E-07	9.73E-07	1.95E-06	2.41E-06	4.03E-06	4.49E-04	0.00E-01	0.00E-01
I 134	4.24E-27	6.10E-27	4.96E-27	9.20E-27	1.41E-26	2.12E-25	0.00E-01	0.00E-01
I 135	4.88E-22	7.87E-22	5.74E-22	1.03E-21	1.58E-21	9.14E-20	0.00E-01	0.00E-01
CS134	3.27E+02	8.36E+00	9.45E+02	1.55E+03	4.81E+02	0.00E-01	1.72E+02	0.00E-01
CS136	5.96E+01	3.24E+00	3.35E+01	9.21E+01	4.90E+01	0.00E-01	7.32E+00	0.00E-01
CS137	1.90E+02	8.06E+00	1.34E+03	1.29E+03	4.19E+02	0.00E-01	1.51E+02	0.00E-01
CS138	2.00E-27	1.45E-27	2.26E-27	3.15E-27	2.21E-27	0.00E-01	2.38E-28	0.00E-01
BA139	2.09E-28	4.16E-25	7.21E-27	3.85E-30	3.36E-30	0.00E-01	2.26E-30	0.00E-01
BA140	5.39E+00	4.68E+01	9.24E+01	8.09E-02	2.63E-02	0.00E-01	4.82E-02	0.00E-01
BA141	1.93E-29	3.38E-28	5.93E-28	3.32E-31	2.87E-31	0.00E-01	1.95E-30	0.00E-01
BA142	2.88E-30	6.73E-31	5.16E-29	3.71E-32	3.01E-32	0.00E-01	2.18E-32	0.00E-01
LA140	6.23E-08	5.15E-03	5.28E-07	1.85E-07	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA142	6.01E-32	3.80E-26	6.02E-31	1.92E-31	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	2.37E-03	1.99E+01	3.20E-02	1.59E-02	6.99E-03	0.00E-01	0.00E-01	0.00E-01
CE143	2.90E-08	2.93E-03	3.69E-07	2.00E-04	8.39E-08	0.00E-01	0.00E-01	0.00E-01
CE144	1.31E-01	2.00E+02	2.44E+00	7.66E-01	4.24E-01	0.00E-01	0.00E-01	0.00E-01
PR143	3.39E-03	7.38E+01	6.84E-02	2.05E-02	1.11E-02	0.00E-01	0.00E-01	0.00E-01
PR144	2.47E-32	3.27E-28	4.91E-31	1.52E-31	8.04E-32	0.00E-01	0.00E-01	0.00E-01
ND147	1.70E-03	3.47E+01	2.71E-02	2.19E-02	1.20E-02	0.00E-01	0.00E-01	0.00E-01
W 187	1.35E-07	4.22E-05	5.07E-07	3.00E-07	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NP239	1.51E-07	1.59E-02	3.00E-06	2.15E-07	6.22E-07	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = CHILD
PATHWAY = RIVER SHORELINE DEPOSITS

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NA 24	2.19E+00	2.19E+00	2.19E+00	2.19E+00	2.19E+00	2.19E+00	2.19E+00	2.54E+00
P 32	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	8.55E-01	8.55E-01	8.55E-01	8.55E-01	8.55E-01	8.55E-01	8.55E-01	1.01E+00
MN 54	2.54E+02	2.54E+02	2.54E+02	2.54E+02	2.54E+02	2.54E+02	2.54E+02	2.98E+02
MN 56	1.61E-01	1.61E-01	1.61E-01	1.61E-01	1.61E-01	1.61E-01	1.61E-01	1.90E-01
FE 55	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 59	5.01E+01	5.01E+01	5.01E+01	5.01E+01	5.01E+01	5.01E+01	5.01E+01	5.88E+01
CO 58	6.98E+01	6.98E+01	6.98E+01	6.98E+01	6.98E+01	6.98E+01	6.98E+01	8.18E+01
CO 60	3.95E+03	3.95E+03	3.95E+03	3.95E+03	3.95E+03	3.95E+03	3.95E+03	4.65E+03
NI 63	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	5.31E-02	5.31E-02	5.31E-02	5.31E-02	5.31E-02	5.31E-02	5.31E-02	6.17E-02
CU 64	1.11E-01	1.11E-01	1.11E-01	1.11E-01	1.11E-01	1.11E-01	1.11E-01	1.25E-01
ZN 65	1.37E+02	1.37E+02	1.37E+02	1.37E+02	1.37E+02	1.37E+02	1.37E+02	1.58E+02
ZN 69	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 83	8.70E-04	8.70E-04	8.70E-04	8.70E-04	8.70E-04	8.70E-04	8.70E-04	1.26E-03
BR 84	3.27E-02	3.27E-02	3.27E-02	3.27E-02	3.27E-02	3.27E-02	3.27E-02	3.81E-02
BR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 86	1.65E+00	1.65E+00	1.65E+00	1.65E+00	1.65E+00	1.65E+00	1.65E+00	1.89E+00
RB 88	4.83E-03	4.83E-03	4.83E-03	4.83E-03	4.83E-03	4.83E-03	4.83E-03	5.52E-03
RB 89	1.70E-02	1.70E-02	1.70E-02	1.70E-02	1.70E-02	1.70E-02	1.70E-02	2.04E-02
SR 89	3.97E-03	3.97E-03	3.97E-03	3.97E-03	3.97E-03	3.97E-03	3.97E-03	4.61E-03
SR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 91	3.91E-01	3.91E-01	3.91E-01	3.91E-01	3.91E-01	3.91E-01	3.91E-01	4.57E-01
SR 92	1.39E-01	1.39E-01	1.39E-01	1.39E-01	1.39E-01	1.39E-01	1.39E-01	1.55E-01
Y 90	8.25E-04	8.25E-04	8.25E-04	8.25E-04	8.25E-04	8.25E-04	8.25E-04	9.76E-04
Y 91M	1.69E-02	1.69E-02	1.69E-02	1.69E-02	1.69E-02	1.69E-02	1.69E-02	1.96E-02
Y 91	1.97E-01	1.97E-01	1.97E-01	1.97E-01	1.97E-01	1.97E-01	1.97E-01	2.22E-01
Y 92	3.26E-02	3.26E-02	3.26E-02	3.26E-02	3.26E-02	3.26E-02	3.26E-02	3.87E-02
Y 93	3.34E-02	3.34E-02	3.34E-02	3.34E-02	3.34E-02	3.34E-02	3.34E-02	4.57E-02
ZR 95	4.51E+01	4.51E+01	4.51E+01	4.51E+01	4.51E+01	4.51E+01	4.51E+01	5.23E+01
ZR 97	5.41E-01	5.41E-01	5.41E-01	5.41E-01	5.41E-01	5.41E-01	5.41E-01	6.30E-01
NB 95	2.52E+01	2.52E+01	2.52E+01	2.52E+01	2.52E+01	2.52E+01	2.52E+01	2.96E+01
MO 99	7.33E-01	7.33E-01	7.33E-01	7.33E-01	7.33E-01	7.33E-01	7.33E-01	8.48E-01
TC 99M	3.34E-02	3.34E-02	3.34E-02	3.34E-02	3.34E-02	3.34E-02	3.34E-02	3.83E-02
TC101	2.79E-03	2.79E-03	2.79E-03	2.79E-03	2.79E-03	2.79E-03	2.79E-03	3.10E-03

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = CHILD
PATHWAY = RIVER SHORELINE DEPOSITS

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
RU103	1.99E+01	1.99E+01	1.99E+01	1.99E+01	1.99E+01	1.99E+01	1.99E+01	2.32E+01
RU105	1.15E-01	1.15E-01	1.15E-01	1.15E-01	1.15E-01	1.15E-01	1.15E-01	1.30E-01
RU106	7.75E+01	7.75E+01	7.75E+01	7.75E+01	7.75E+01	7.75E+01	7.75E+01	9.31E+01
AG110M	6.34E+02	6.34E+02	6.34E+02	6.34E+02	6.34E+02	6.34E+02	6.34E+02	7.39E+02
TE125M	2.86E-01	2.86E-01	2.86E-01	2.86E-01	2.86E-01	2.86E-01	2.86E-01	3.92E-01
TE127M	1.68E-02	1.68E-02	1.68E-02	1.68E-02	1.68E-02	1.68E-02	1.68E-02	1.99E-02
TE127	5.43E-04	5.43E-04	5.43E-04	5.43E-04	5.43E-04	5.43E-04	5.43E-04	5.97E-04
TE129M	3.63E+00	3.63E+00	3.63E+00	3.63E+00	3.63E+00	3.63E+00	3.63E+00	4.24E+00
TE129	4.54E-03	4.54E-03	4.54E-03	4.54E-03	4.54E-03	4.54E-03	4.54E-03	5.37E-03
TE131M	1.47E+00	1.47E+00	1.47E+00	1.47E+00	1.47E+00	1.47E+00	1.47E+00	1.73E+00
TE131	4.56E-03	4.56E-03	4.56E-03	4.56E-03	4.56E-03	4.56E-03	4.56E-03	5.38E+00
TE132	7.78E-01	7.78E-01	7.78E-01	7.78E-01	7.78E-01	7.78E-01	7.78E-01	9.15E-01
I 130	1.01E+00	1.01E+00	1.01E+00	1.01E+00	1.01E+00	1.01E+00	1.01E+00	1.22E+00
I 131	3.16E+00	3.16E+00	3.16E+00	3.16E+00	3.16E+00	3.16E+00	3.16E+00	3.84E+00
I 132	2.22E-01	2.22E-01	2.22E-01	2.22E-01	2.22E-01	2.22E-01	2.22E-01	2.61E-01
I 133	4.49E-01	4.49E-01	4.49E-01	4.49E-01	4.49E-01	4.49E-01	4.49E-01	5.46E-01
I 134	7.57E-02	7.57E-02	7.57E-02	7.57E-02	7.57E-02	7.57E-02	7.57E-02	8.99E-02
I 135	4.58E-01	4.58E-01	4.58E-01	4.58E-01	4.58E-01	4.58E-01	4.58E-01	5.35E-01
CS134	1.26E+03	1.26E+03	1.26E+03	1.26E+03	1.26E+03	1.26E+03	1.26E+03	1.46E+03
CS136	2.76E+01	2.76E+01	2.76E+01	2.76E+01	2.76E+01	2.76E+01	2.76E+01	3.13E+01
CS137	1.89E+03	1.89E+03	1.89E+03	1.89E+03	1.89E+03	1.89E+03	1.89E+03	2.21E+03
CS138	5.81E-02	5.81E-02	5.81E-02	5.81E-02	5.81E-02	5.81E-02	5.81E-02	6.64E-02
BA139	1.84E-02	1.84E-02	1.84E-02	1.84E-02	1.84E-02	1.84E-02	1.84E-02	2.07E-02
BA140	3.77E+00	3.77E+00	3.77E+00	3.77E+00	3.77E+00	3.77E+00	3.77E+00	4.31E+00
BA141	6.11E-03	6.11E-03	6.11E-03	6.11E-03	6.11E-03	6.11E-03	6.11E-03	6.96E-03
BA142	5.59E-03	5.59E-03	5.59E-03	5.59E-03	5.59E-03	5.59E-03	5.59E-03	6.37E-03
LA140	3.53E+00	3.53E+00	3.53E+00	3.53E+00	3.53E+00	3.53E+00	3.53E+00	4.00E+00
LA142	1.29E-01	1.29E-01	1.29E-01	1.29E-01	1.29E-01	1.29E-01	1.29E-01	1.55E-01
CE141	2.51E+00	2.51E+00	2.51E+00	2.51E+00	2.51E+00	2.51E+00	2.51E+00	2.83E+00
CE143	4.26E-01	4.26E-01	4.26E-01	4.26E-01	4.26E-01	4.26E-01	4.26E-01	4.84E-01
CE144	1.28E+01	1.28E+01	1.28E+01	1.28E+01	1.28E+01	1.28E+01	1.28E+01	1.48E+01
PR143	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
PR144	2.65E-04	2.65E-04	2.65E-04	2.65E-04	2.65E-04	2.65E-04	2.65E-04	3.04E-04
ND147	1.56E+00	1.56E+00	1.56E+00	1.56E+00	1.56E+00	1.56E+00	1.56E+00	1.87E+00
W 187	4.33E-01	4.33E-01	4.33E-01	4.33E-01	4.33E-01	4.33E-01	4.33E-01	5.02E-01
NP239	3.15E-01	3.15E-01	3.15E-01	3.15E-01	3.15E-01	3.15E-01	3.15E-01	3.64E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = CHILD
PATHWAY = SWIMMING

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NA 24	1.09E+02	1.09E+02	1.09E+02	1.09E+02	1.09E+02	1.09E+02	1.09E+02	0.00E-01
P 32	8.99E-02	8.99E-02	8.99E-02	8.99E-02	8.99E-02	8.99E-02	8.99E-02	0.00E-01
CR 51	7.30E-01	7.30E-01	7.30E-01	7.30E-01	7.30E-01	7.30E-01	7.30E-01	0.00E-01
MN 54	2.11E+01	2.11E+01	2.11E+01	2.11E+01	2.11E+01	2.11E+01	2.11E+01	0.00E-01
MN 56	4.38E+01	4.38E+01	4.38E+01	4.38E+01	4.38E+01	4.38E+01	4.38E+01	0.00E-01
FE 55	8.99E-04	8.99E-04	8.99E-04	8.99E-04	8.99E-04	8.99E-04	8.99E-04	0.00E-01
FE 59	3.09E+01	3.09E+01	3.09E+01	3.09E+01	3.09E+01	3.09E+01	3.09E+01	0.00E-01
CO 58	2.53E+01	2.53E+01	2.53E+01	2.53E+01	2.53E+01	2.53E+01	2.53E+01	0.00E-01
CO 60	6.46E+01	6.46E+01	6.46E+01	6.46E+01	6.46E+01	6.46E+01	6.46E+01	0.00E-01
NI 63	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	1.37E+01	1.37E+01	1.37E+01	1.37E+01	1.37E+01	1.37E+01	1.37E+01	0.00E-01
CU 64	5.17E+00	5.17E+00	5.17E+00	5.17E+00	5.17E+00	5.17E+00	5.17E+00	0.00E-01
ZN 65	1.55E+01	1.55E+01	1.55E+01	1.55E+01	1.55E+01	1.55E+01	1.55E+01	0.00E-01
ZN 69	2.09E-02	2.09E-02	2.09E-02	2.09E-02	2.09E-02	2.09E-02	2.09E-02	0.00E-01
BR 83	2.32E-01	2.32E-01	2.32E-01	2.32E-01	2.32E-01	2.32E-01	2.32E-01	0.00E-01
BR 84	4.32E+01	4.32E+01	4.32E+01	4.32E+01	4.32E+01	4.32E+01	4.32E+01	0.00E-01
BR 85	4.61E-02	4.61E-02	4.61E-02	4.61E-02	4.61E-02	4.61E-02	4.61E-02	0.00E-01
RB 86	2.39E+00	2.39E+00	2.39E+00	2.39E+00	2.39E+00	2.39E+00	2.39E+00	0.00E-01
RB 88	1.34E+01	1.34E+01	1.34E+01	1.34E+01	1.34E+01	1.34E+01	1.34E+01	0.00E-01
RB 89	4.81E+01	4.81E+01	4.81E+01	4.81E+01	4.81E+01	4.81E+01	4.81E+01	0.00E-01
SR 89	6.46E-02	6.46E-02	6.46E-02	6.46E-02	6.46E-02	6.46E-02	6.46E-02	0.00E-01
SR 90	7.59E-03	7.59E-03	7.59E-03	7.59E-03	7.59E-03	7.59E-03	7.59E-03	0.00E-01
SR 91	2.65E+01	2.65E+01	2.65E+01	2.65E+01	2.65E+01	2.65E+01	2.65E+01	0.00E-01
SR 92	3.56E+01	3.56E+01	3.56E+01	3.56E+01	3.56E+01	3.56E+01	3.56E+01	0.00E-01
Y 90	1.83E-01	1.83E-01	1.83E-01	1.83E-01	1.83E-01	1.83E-01	1.83E-01	0.00E-01
Y 91M	1.29E+01	1.29E+01	1.29E+01	1.29E+01	1.29E+01	1.29E+01	1.29E+01	0.00E-01
Y 91	9.41E-02	9.41E-02	9.41E-02	9.41E-02	9.41E-02	9.41E-02	9.41E-02	0.00E-01
Y 92	6.34E+00	6.34E+00	6.34E+00	6.34E+00	6.34E+00	6.34E+00	6.34E+00	0.00E-01
Y 93	2.65E+00	2.65E+00	2.65E+00	2.65E+00	2.65E+00	2.65E+00	2.65E+00	0.00E-01
ZR 95	2.11E+01	2.11E+01	2.11E+01	2.11E+01	2.11E+01	2.11E+01	2.11E+01	0.00E-01
ZR 97	2.10E+01	2.10E+01	2.10E+01	2.10E+01	2.10E+01	2.10E+01	2.10E+01	0.00E-01
NB 95	1.97E+01	1.97E+01	1.97E+01	1.97E+01	1.97E+01	1.97E+01	1.97E+01	0.00E-01
MO 99	6.60E+00	6.60E+00	6.60E+00	6.60E+00	6.60E+00	6.60E+00	6.60E+00	0.00E-01
TC 99M	3.33E+00	3.33E+00	3.33E+00	3.33E+00	3.33E+00	3.33E+00	3.33E+00	0.00E-01
TC101	7.13E+00	7.13E+00	7.13E+00	7.13E+00	7.13E+00	7.13E+00	7.13E+00	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = CHILD
PATHWAY = SWIMMING

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem gal)/(Ci min)				
RU103	1.25E+01	1.25E+01	1.25E+01	1.25E+01	1.25E+01	1.25E+01	1.25E+01	0.00E-01
RU105	1.66E+01	1.66E+01	1.66E+01	1.66E+01	1.66E+01	1.66E+01	1.66E+01	0.00E-01
RU106	5.34E+00	5.34E+00	5.34E+00	5.34E+00	5.34E+00	5.34E+00	5.34E+00	0.00E-01
AG110M	6.88E+01	6.88E+01	6.88E+01	6.88E+01	6.88E+01	6.88E+01	6.88E+01	0.00E-01
TE125M	5.18E-02	5.18E-02	5.18E-02	5.18E-02	5.18E-02	5.18E-02	5.18E-02	0.00E-01
TE127M	3.65E-03	3.65E-03	3.65E-03	3.65E-03	3.65E-03	3.65E-03	3.65E-03	0.00E-01
TE127	3.91E-02	3.91E-02	3.91E-02	3.91E-02	3.91E-02	3.91E-02	3.91E-02	0.00E-01
TE129M	2.95E+00	2.95E+00	2.95E+00	2.95E+00	2.95E+00	2.95E+00	2.95E+00	0.00E-01
TE129	2.52E+00	2.52E+00	2.52E+00	2.52E+00	2.52E+00	2.52E+00	2.52E+00	0.00E-01
TE131M	3.08E+01	3.08E+01	3.08E+01	3.08E+01	3.08E+01	3.08E+01	3.08E+01	0.00E-01
TE131	8.81E+00	8.81E+00	8.81E+00	8.81E+00	8.81E+00	8.81E+00	8.81E+00	0.00E-01
TE132	5.61E+00	5.61E+00	5.61E+00	5.61E+00	5.61E+00	5.61E+00	5.61E+00	0.00E-01
I 130	5.45E+01	5.45E+01	5.45E+01	5.45E+01	5.45E+01	5.45E+01	5.45E+01	0.00E-01
I 131	1.10E+01	1.10E+01	1.10E+01	1.10E+01	1.10E+01	1.10E+01	1.10E+01	0.00E-01
I 132	6.00E+01	6.00E+01	6.00E+01	6.00E+01	6.00E+01	6.00E+01	6.00E+01	0.00E-01
I 133	1.34E+01	1.34E+01	1.34E+01	1.34E+01	1.34E+01	1.34E+01	1.34E+01	0.00E-01
I 134	5.45E+01	5.45E+01	5.45E+01	5.45E+01	5.45E+01	5.45E+01	5.45E+01	0.00E-01
I 135	4.59E+01	4.59E+01	4.59E+01	4.59E+01	4.59E+01	4.59E+01	4.59E+01	0.00E-01
CS134	4.07E+01	4.07E+01	4.07E+01	4.07E+01	4.07E+01	4.07E+01	4.07E+01	0.00E-01
CS136	5.76E+01	5.76E+01	5.76E+01	5.76E+01	5.76E+01	5.76E+01	5.76E+01	0.00E-01
CS137	1.41E+01	1.41E+01	1.41E+01	1.41E+01	1.41E+01	1.41E+01	1.41E+01	0.00E-01
CS138	4.94E+01	4.94E+01	4.94E+01	4.94E+01	4.94E+01	4.94E+01	4.94E+01	0.00E-01
BA139	1.03E+00	1.03E+00	1.03E+00	1.03E+00	1.03E+00	1.03E+00	1.03E+00	0.00E-01
BA140	6.88E+00	6.88E+00	6.88E+00	6.88E+00	6.88E+00	6.88E+00	6.88E+00	0.00E-01
BA141	1.23E+01	1.23E+01	1.23E+01	1.23E+01	1.23E+01	1.23E+01	1.23E+01	0.00E-01
BA142	2.10E+01	2.10E+01	2.10E+01	2.10E+01	2.10E+01	2.10E+01	2.10E+01	0.00E-01
LA140	5.75E+01	5.75E+01	5.75E+01	5.75E+01	5.75E+01	5.75E+01	5.75E+01	0.00E-01
LA142	6.04E+01	6.04E+01	6.04E+01	6.04E+01	6.04E+01	6.04E+01	6.04E+01	0.00E-01
CE141	1.83E+00	1.83E+00	1.83E+00	1.83E+00	1.83E+00	1.83E+00	1.83E+00	0.00E-01
CE143	7.99E+00	7.99E+00	7.99E+00	7.99E+00	7.99E+00	7.99E+00	7.99E+00	0.00E-01
CE144	1.21E+00	1.21E+00	1.21E+00	1.21E+00	1.21E+00	1.21E+00	1.21E+00	0.00E-01
PR143	2.25E-02	2.25E-02	2.25E-02	2.25E-02	2.25E-02	2.25E-02	2.25E-02	0.00E-01
PR144	6.18E-01	6.18E-01	6.18E-01	6.18E-01	6.18E-01	6.18E-01	6.18E-01	0.00E-01
ND147	3.93E+00	3.93E+00	3.93E+00	3.93E+00	3.93E+00	3.93E+00	3.93E+00	0.00E-01
W 187	1.16E+01	1.16E+01	1.16E+01	1.16E+01	1.16E+01	1.16E+01	1.16E+01	0.00E-01
NP239	3.37E+00	3.37E+00	3.37E+00	3.37E+00	3.37E+00	3.37E+00	3.37E+00	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = CHILD
PATHWAY = BOATING

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NA 24	1.01E+02	1.01E+02	1.01E+02	1.01E+02	1.01E+02	1.01E+02	1.01E+02	0.00E-01
P 32	8.35E-02	8.35E-02	8.35E-02	8.35E-02	8.35E-02	8.35E-02	8.35E-02	0.00E-01
CR 51	6.78E-01	6.78E-01	6.78E-01	6.78E-01	6.78E-01	6.78E-01	6.78E-01	0.00E-01
MN 54	1.96E+01	1.96E+01	1.96E+01	1.96E+01	1.96E+01	1.96E+01	1.96E+01	0.00E-01
MN 56	4.06E+01	4.06E+01	4.06E+01	4.06E+01	4.06E+01	4.06E+01	4.06E+01	0.00E-01
FE 55	8.35E-04	8.35E-04	8.35E-04	8.35E-04	8.35E-04	8.35E-04	8.35E-04	0.00E-01
FE 59	2.87E+01	2.87E+01	2.87E+01	2.87E+01	2.87E+01	2.87E+01	2.87E+01	0.00E-01
CO 58	2.35E+01	2.35E+01	2.35E+01	2.35E+01	2.35E+01	2.35E+01	2.35E+01	0.00E-01
CO 60	6.00E+01	6.00E+01	6.00E+01	6.00E+01	6.00E+01	6.00E+01	6.00E+01	0.00E-01
NI 63	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	1.27E+01	1.27E+01	1.27E+01	1.27E+01	1.27E+01	1.27E+01	1.27E+01	0.00E-01
CU 64	4.80E+00	4.80E+00	4.80E+00	4.80E+00	4.80E+00	4.80E+00	4.80E+00	0.00E-01
ZN 65	1.44E+01	1.44E+01	1.44E+01	1.44E+01	1.44E+01	1.44E+01	1.44E+01	0.00E-01
ZN 69	1.94E-02	1.94E-02	1.94E-02	1.94E-02	1.94E-02	1.94E-02	1.94E-02	0.00E-01
BR 83	2.15E-01	2.15E-01	2.15E-01	2.15E-01	2.15E-01	2.15E-01	2.15E-01	0.00E-01
BR 84	4.01E+01	4.01E+01	4.01E+01	4.01E+01	4.01E+01	4.01E+01	4.01E+01	0.00E-01
BR 85	4.28E-02	4.28E-02	4.28E-02	4.28E-02	4.28E-02	4.28E-02	4.28E-02	0.00E-01
RB 86	2.22E+00	2.22E+00	2.22E+00	2.22E+00	2.22E+00	2.22E+00	2.22E+00	0.00E-01
RB 88	1.24E+01	1.24E+01	1.24E+01	1.24E+01	1.24E+01	1.24E+01	1.24E+01	0.00E-01
RB 89	4.47E+01	4.47E+01	4.47E+01	4.47E+01	4.47E+01	4.47E+01	4.47E+01	0.00E-01
SR 89	6.00E-02	6.00E-02	6.00E-02	6.00E-02	6.00E-02	6.00E-02	6.00E-02	0.00E-01
SR 90	7.04E-03	7.04E-03	7.04E-03	7.04E-03	7.04E-03	7.04E-03	7.04E-03	0.00E-01
SR 91	2.46E+01	2.46E+01	2.46E+01	2.46E+01	2.46E+01	2.46E+01	2.46E+01	0.00E-01
SR 92	3.31E+01	3.31E+01	3.31E+01	3.31E+01	3.31E+01	3.31E+01	3.31E+01	0.00E-01
Y 90	1.69E-01	1.69E-01	1.69E-01	1.69E-01	1.69E-01	1.69E-01	1.69E-01	0.00E-01
Y 91M	1.20E+01	1.20E+01	1.20E+01	1.20E+01	1.20E+01	1.20E+01	1.20E+01	0.00E-01
Y 91	8.74E-02	8.74E-02	8.74E-02	8.74E-02	8.74E-02	8.74E-02	8.74E-02	0.00E-01
Y 92	5.89E+00	5.89E+00	5.89E+00	5.89E+00	5.89E+00	5.89E+00	5.89E+00	0.00E-01
Y 93	2.46E+00	2.46E+00	2.46E+00	2.46E+00	2.46E+00	2.46E+00	2.46E+00	0.00E-01
ZR 95	1.96E+01	1.96E+01	1.96E+01	1.96E+01	1.96E+01	1.96E+01	1.96E+01	0.00E-01
ZR 97	1.95E+01	1.95E+01	1.95E+01	1.95E+01	1.95E+01	1.95E+01	1.95E+01	0.00E-01
NB 95	1.83E+01	1.83E+01	1.83E+01	1.83E+01	1.83E+01	1.83E+01	1.83E+01	0.00E-01
MO 99	6.12E+00	6.12E+00	6.12E+00	6.12E+00	6.12E+00	6.12E+00	6.12E+00	0.00E-01
TC 99M	3.10E+00	3.10E+00	3.10E+00	3.10E+00	3.10E+00	3.10E+00	3.10E+00	0.00E-01
TC101	6.62E+00	6.62E+00	6.62E+00	6.62E+00	6.62E+00	6.62E+00	6.62E+00	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = CHILD
PATHWAY = BOATING

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
RU103	1.16E+01	1.16E+01	1.16E+01	1.16E+01	1.16E+01	1.16E+01	1.16E+01	0.00E-01
RU105	1.54E+01	1.54E+01	1.54E+01	1.54E+01	1.54E+01	1.54E+01	1.54E+01	0.00E-01
RU106	4.96E+00	4.96E+00	4.96E+00	4.96E+00	4.96E+00	4.96E+00	4.96E+00	0.00E-01
AG110M	6.39E+01	6.39E+01	6.39E+01	6.39E+01	6.39E+01	6.39E+01	6.39E+01	0.00E-01
TE125M	4.81E-02	4.81E-02	4.81E-02	4.81E-02	4.81E-02	4.81E-02	4.81E-02	0.00E-01
TE127M	3.39E-03	3.39E-03	3.39E-03	3.39E-03	3.39E-03	3.39E-03	3.39E-03	0.00E-01
TE127	3.63E-02	3.63E-02	3.63E-02	3.63E-02	3.63E-02	3.63E-02	3.63E-02	0.00E-01
TE129M	2.74E+00	2.74E+00	2.74E+00	2.74E+00	2.74E+00	2.74E+00	2.74E+00	0.00E-01
TE129	2.34E+00	2.34E+00	2.34E+00	2.34E+00	2.34E+00	2.34E+00	2.34E+00	0.00E-01
TE131M	2.86E+01	2.86E+01	2.86E+01	2.86E+01	2.86E+01	2.86E+01	2.86E+01	0.00E-01
TE131	8.18E+00	8.18E+00	8.18E+00	8.18E+00	8.18E+00	8.18E+00	8.18E+00	0.00E-01
TE132	5.21E+00	5.21E+00	5.21E+00	5.21E+00	5.21E+00	5.21E+00	5.21E+00	0.00E-01
I 130	5.06E+01	5.06E+01	5.06E+01	5.06E+01	5.06E+01	5.06E+01	5.06E+01	0.00E-01
I 131	1.02E+01	1.02E+01	1.02E+01	1.02E+01	1.02E+01	1.02E+01	1.02E+01	0.00E-01
I 132	5.57E+01	5.57E+01	5.57E+01	5.57E+01	5.57E+01	5.57E+01	5.57E+01	0.00E-01
I 133	1.25E+01	1.25E+01	1.25E+01	1.25E+01	1.25E+01	1.25E+01	1.25E+01	0.00E-01
I 134	5.06E+01	5.06E+01	5.06E+01	5.06E+01	5.06E+01	5.06E+01	5.06E+01	0.00E-01
I 135	4.26E+01	4.26E+01	4.26E+01	4.26E+01	4.26E+01	4.26E+01	4.26E+01	0.00E-01
CS134	3.78E+01	3.78E+01	3.78E+01	3.78E+01	3.78E+01	3.78E+01	3.78E+01	0.00E-01
CS136	5.35E+01	5.35E+01	5.35E+01	5.35E+01	5.35E+01	5.35E+01	5.35E+01	0.00E-01
CS137	1.31E+01	1.31E+01	1.31E+01	1.31E+01	1.31E+01	1.31E+01	1.31E+01	0.00E-01
CS138	4.59E+01	4.59E+01	4.59E+01	4.59E+01	4.59E+01	4.59E+01	4.59E+01	0.00E-01
BA139	9.55E-01	9.55E-01	9.55E-01	9.55E-01	9.55E-01	9.55E-01	9.55E-01	0.00E-01
BA140	6.39E+00	6.39E+00	6.39E+00	6.39E+00	6.39E+00	6.39E+00	6.39E+00	0.00E-01
BA141	1.14E+01	1.14E+01	1.14E+01	1.14E+01	1.14E+01	1.14E+01	1.14E+01	0.00E-01
BA142	1.95E+01	1.95E+01	1.95E+01	1.95E+01	1.95E+01	1.95E+01	1.95E+01	0.00E-01
LA140	5.34E+01	5.34E+01	5.34E+01	5.34E+01	5.34E+01	5.34E+01	5.34E+01	0.00E-01
LA142	5.61E+01	5.61E+01	5.61E+01	5.61E+01	5.61E+01	5.61E+01	5.61E+01	0.00E-01
CE141	1.70E+00	1.70E+00	1.70E+00	1.70E+00	1.70E+00	1.70E+00	1.70E+00	0.00E-01
CE143	7.42E+00	7.42E+00	7.42E+00	7.42E+00	7.42E+00	7.42E+00	7.42E+00	0.00E-01
CE144	1.12E+00	1.12E+00	1.12E+00	1.12E+00	1.12E+00	1.12E+00	1.12E+00	0.00E-01
PR143	2.09E-02	2.09E-02	2.09E-02	2.09E-02	2.09E-02	2.09E-02	2.09E-02	0.00E-01
PR144	5.74E-01	5.74E-01	5.74E-01	5.74E-01	5.74E-01	5.74E-01	5.74E-01	0.00E-01
ND147	3.65E+00	3.65E+00	3.65E+00	3.65E+00	3.65E+00	3.65E+00	3.65E+00	0.00E-01
W 187	1.08E+01	1.08E+01	1.08E+01	1.08E+01	1.08E+01	1.08E+01	1.08E+01	0.00E-01
NP239	3.13E+00	3.13E+00	3.13E+00	3.13E+00	3.13E+00	3.13E+00	3.13E+00	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = INFANT
PATHWAY = POTABLE WATER

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	2.91E+01	2.91E+01	0.00E-01	2.91E+01	2.91E+01	2.91E+01	2.91E+01	0.00E-01
C 14	8.38E+02	8.38E+02	3.92E+03	8.38E+02	8.38E+02	8.38E+02	8.38E+02	0.00E-01
NA 24	5.53E+02	5.53E+02	5.53E+02	5.53E+02	5.53E+02	5.53E+02	5.53E+02	0.00E-01
P 32	1.04E+04	3.63E+03	2.68E+05	1.58E+04	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	2.28E+00	6.64E+01	0.00E-01	0.00E-01	3.25E-01	1.49E+00	2.89E+00	0.00E-01
MN 54	7.45E+02	1.21E+03	0.00E-01	3.29E+03	7.29E+02	0.00E-01	0.00E-01	0.00E-01
MN 56	3.58E-02	1.89E+01	0.00E-01	2.08E-01	1.79E-01	0.00E-01	0.00E-01	0.00E-01
FE 55	3.97E+02	1.89E+02	2.30E+03	1.49E+03	0.00E-01	0.00E-01	7.26E+02	0.00E-01
FE 59	3.46E+03	4.19E+03	5.02E+03	8.77E+03	0.00E-01	0.00E-01	2.59E+03	0.00E-01
CO 58	1.47E+03	1.47E+03	0.00E-01	5.90E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	4.22E+03	4.25E+03	0.00E-01	1.79E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 63	3.64E+03	3.23E+02	1.05E+05	6.49E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	5.45E-02	9.11E+00	1.06E+00	1.20E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CU 64	1.26E+01	5.57E+02	0.00E-01	2.71E+01	4.59E+01	0.00E-01	0.00E-01	0.00E-01
ZN 65	4.80E+03	8.80E+03	3.04E+03	1.04E+04	5.05E+03	0.00E-01	0.00E-01	0.00E-01
ZN 69	5.00E-08	5.48E-05	3.73E-07	6.72E-07	2.79E-07	0.00E-01	0.00E-01	0.00E-01
BR 83	5.73E-02	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 84	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 86	1.34E+04	6.94E+02	0.00E-01	2.71E+04	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	1.18E+04	8.43E+03	4.10E+05	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	9.50E+04	3.82E+04	4.69E+06	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 91	5.19E+01	1.70E+03	1.43E+03	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 92	2.56E-01	7.43E+01	6.89E+00	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 90	2.98E-01	1.53E+04	1.11E+01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91	4.93E+00	1.33E+04	1.85E+02	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 92	3.29E-04	2.24E+02	1.17E-02	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 93	2.11E-02	6.10E+03	7.73E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	5.83E+00	4.09E+03	3.37E+01	8.22E+00	8.86E+00	0.00E-01	0.00E-01	0.00E-01
ZR 97	7.17E-02	1.00E+04	9.15E-01	1.57E-01	1.58E-01	0.00E-01	0.00E-01	0.00E-01
NB 95	1.62E+00	2.37E+03	6.82E+00	2.81E+00	2.01E+00	0.00E-01	0.00E-01	0.00E-01
MO 99	8.53E+02	1.44E+03	0.00E-01	4.37E+03	6.54E+03	0.00E-01	0.00E-01	0.00E-01
TC 99M	5.32E-01	1.20E+01	2.00E-02	4.13E-02	4.44E-01	0.00E-01	2.16E-02	0.00E-01
TC101	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = INFANT
PATHWAY = POTABLE WATER

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
RU103	8.05E+01	2.93E+03	2.41E+02	0.00E-01	5.01E+02	0.00E-01	0.00E-01	0.00E-01
RU105	1.78E-01	2.11E+02	5.30E-01	0.00E-01	3.89E+00	0.00E-01	0.00E-01	0.00E-01
RU106	4.97E+02	3.02E+04	3.98E+03	0.00E-01	4.71E+03	0.00E-01	0.00E-01	0.00E-01
AG110M	7.94E+01	6.23E+03	1.64E+02	1.20E+02	1.72E+02	0.00E-01	0.00E-01	0.00E-01
TE125M	5.15E+02	1.82E+03	3.81E+03	1.27E+03	0.00E-01	1.28E+03	0.00E-01	0.00E-01
TE127M	1.16E+03	3.88E+03	9.63E+03	3.19E+03	2.37E+04	2.78E+03	0.00E-01	0.00E-01
TE127	6.01E+00	5.86E+02	2.79E+01	9.36E+00	6.82E+01	2.27E+01	0.00E-01	0.00E-01
TE129M	2.50E+03	9.68E+03	1.62E+04	5.56E+03	4.05E+04	6.23E+03	0.00E-01	0.00E-01
TE129	6.48E-06	2.22E-03	2.78E-05	9.57E-06	6.91E-05	2.33E-05	0.00E-01	0.00E-01
TE131M	4.80E+02	9.79E+03	1.44E+03	5.82E+02	4.00E+03	1.18E+03	0.00E-01	0.00E-01
TE131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE132	1.29E+03	5.10E+03	2.78E+03	1.38E+03	8.62E+03	2.03E+03	0.00E-01	0.00E-01
I 130	2.28E+02	1.22E+02	2.58E+02	5.68E+02	6.24E+02	6.37E+04	0.00E-01	0.00E-01
I 131	2.83E+03	2.29E+02	5.45E+03	6.43E+03	7.50E+03	2.11E+06	0.00E-01	0.00E-01
I 132	1.44E-01	3.27E-01	1.99E-01	4.04E-01	4.50E-01	1.89E+01	0.00E-01	0.00E-01
I 133	3.97E+02	2.29E+02	9.31E+02	1.35E+03	1.59E+03	2.46E+05	0.00E-01	0.00E-01
I 134	5.83E-07	1.69E-06	8.00E-07	1.64E-06	1.83E-06	3.82E-05	0.00E-01	0.00E-01
I 135	3.51E+01	3.48E+01	4.84E+01	9.62E+01	1.07E+02	8.62E+03	0.00E-01	0.00E-01
CS134	1.17E+04	3.16E+02	6.24E+04	1.16E+05	2.99E+04	0.00E-01	1.23E+04	0.00E-01
CS136	7.92E+03	3.22E+02	7.21E+03	2.12E+04	8.45E+03	0.00E-01	1.73E+03	0.00E-01
CS137	7.17E+03	3.16E+02	8.64E+04	1.01E+05	2.72E+04	0.00E-01	1.10E+04	0.00E-01
CS138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA139	2.36E-05	5.16E-02	8.14E-04	5.40E-07	3.24E-07	0.00E-01	3.27E-07	0.00E-01
BA140	1.38E+03	6.59E+03	2.68E+04	2.68E+01	6.37E+00	0.00E-01	1.65E+01	0.00E-01
BA141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA142	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA140	2.35E-01	1.07E+04	2.31E+00	9.12E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA142	3.27E-07	2.32E-01	3.72E-06	1.37E-06	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	9.16E-01	4.02E+03	1.28E+01	7.78E+00	2.40E+00	0.00E-01	0.00E-01	0.00E-01
CE143	1.12E-01	5.74E+03	1.48E+00	9.84E+02	2.87E-01	0.00E-01	0.00E-01	0.00E-01
CE144	2.76E+01	2.82E+04	4.92E+02	2.01E+02	8.14E+01	0.00E-01	0.00E-01	0.00E-01
PR143	6.34E-01	6.75E+03	1.28E+01	4.78E+00	1.78E+00	0.00E-01	0.00E-01	0.00E-01
PR144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ND147	5.41E-01	5.60E+03	8.60E+00	8.84E+00	3.41E+00	0.00E-01	0.00E-01	0.00E-01
W 187	1.79E+01	3.05E+03	7.46E+01	5.19E+01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NP239	6.93E-02	3.54E+03	1.37E+00	1.23E-01	2.44E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = INFANT
PATHWAY = FRESH WATER FISH

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NA 24	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
P 32	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MN 54	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MN 56	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 55	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 59	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 58	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 63	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CU 64	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 65	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 69	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 83	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 84	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 86	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 91	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 92	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 92	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 93	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 97	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NB 95	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MO 99	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TC 99M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TC101	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = INFANT
PATHWAY = FRESH WATER FISH

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
RU103	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RU105	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RU106	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
AG110M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE125M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE127M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE127	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE129M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE129	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE131M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE132	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 130	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 132	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 134	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS134	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS136	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA139	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA140	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA142	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA140	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA142	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE143	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
PR143	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
PR144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ND147	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
W 187	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NP239	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = INFANT
PATHWAY = ANIMAL DRINKING WATER—MEAT

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NA 24	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
P 32	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MN 54	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MN 56	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 55	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 59	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 58	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 63	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CU 64	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 65	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 69	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 83	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 84	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 86	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 91	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 92	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 92	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 93	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 97	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NB 95	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MO 99	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TC 99M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TC101	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = INFANT
PATHWAY = ANIMAL DRINKING WATER—MEAT

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem gal)/(Ci min)				
RU103	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RU105	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RU106	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
AG110M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE125M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE127M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE127	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE129M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE129	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE131M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE132	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 130	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 132	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 134	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS134	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS136	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA139	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA140	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA142	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA140	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA142	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE143	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
PR143	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
PR144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ND147	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
W 187	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NP239	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = INFANT
PATHWAY = RIVER SHORELINE DEPOSITS

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NA 24	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
P 32	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MN 54	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MN 56	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 55	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 59	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 58	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 63	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CU 64	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 65	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 69	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 83	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 84	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 86	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 91	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 92	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 92	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 93	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 97	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NB 95	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MO 99	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TC 99M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TC101	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = INFANT
PATHWAY = RIVER SHORELINE DEPOSITS

	TOTAL BODY	GI-LLI	BONE	LIVER	KIDNEY	THYROID	LUNG	SKIN
				(mrem gal)/(Ci min)				
RU103	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RU105	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RU106	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
AG110M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE125M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE127M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE127	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE129M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE129	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE131M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE132	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 130	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 132	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 134	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS134	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS136	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA139	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA140	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA142	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA140	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA142	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE143	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
PR143	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
PR144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ND147	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
W 187	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NP239	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = INFANT
PATHWAY = SWIMMING

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NA 24	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
P 32	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MN 54	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MN 56	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 55	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 59	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 58	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 63	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CU 64	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 65	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 69	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 83	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 84	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 86	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 91	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 92	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 92	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 93	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 97	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NB 95	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MO 99	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TC 99M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TC101	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = INFANT
PATHWAY = SWIMMING

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
RU103	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RU105	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RU106	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
AG110M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE125M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE127M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE127	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE129M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE129	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE131M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE132	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 130	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 132	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 134	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS134	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS136	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA139	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA140	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA142	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA140	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA142	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE143	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
PR143	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
PR144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ND147	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
W 187	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NP239	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = INFANT
PATHWAY = BOATING

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
H 3	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
C 14	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NA 24	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
P 32	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CR 51	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MN 54	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MN 56	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 55	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
FE 59	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 58	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CO 60	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 63	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NI 65	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CU 64	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 65	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZN 69	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 83	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 84	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BR 85	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 86	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 88	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RB 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 89	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 91	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
SR 92	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 90	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 91	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 92	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
Y 93	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 95	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ZR 97	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NB 95	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
MO 99	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TC 99M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TC101	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01

DOSE TRANSFER FACTORS FOR RADIONUCLIDES IN EFFLUENT WATER

AGE = INFANT
PATHWAY = BOATING

	TOTAL BODY	GI-LLI	BONE	LIVER (mrem gal)/(Ci min)	KIDNEY	THYROID	LUNG	SKIN
RU103	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RU105	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
RU106	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
AG110M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE125M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE127M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE127	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE129M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE129	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE131M	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
TE132	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 130	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 131	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 132	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 133	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 134	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
I 135	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS134	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS136	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS137	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CS138	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA139	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA140	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
BA142	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA140	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
LA142	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE141	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE143	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
CE144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
PR143	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
PR144	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
ND147	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
W 187	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01
NP239	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01	0.00E-01