

Entergy Operations, Inc. 17265 River Road Killona, LA 70057-3093 Tel 504 739 6685 Fax 504 739 6698 ijarrel@entergy.com

John P. Jarrell III Manager – Regulatory Assurance Waterford 3

W3F1-2017-0038

April 24, 2017

U.S. Nuclear Regulatory Commission Attn: Document Control Desk Washington, DC 20555-0001

Subject:

Annual Radiological Effluent Release Report -2016 Waterford Steam Electric Station, Unit 3 (Waterford 3) Docket No. 50-382 License No. NPF-38

Dear Sir or Madam:

Attached is the Annual Radiological Effluent Release Report for the period of January 1 through December 31, 2016. This report is submitted pursuant to the requirements of Waterford 3 Technical Specification Sections 6.9.1.8 and 6.14.2.c.

This report contains no new commitments. Please contact John P. Jarrell, Regulatory Assurance Manager, at (504) 739-6685 if you have questions regarding this information.

Sincerely, JPJ/

Enclosure: Annual Radiological Effluent Release Report – 2016 (Including DVD)

IE48 NRR

W3F1-2017-0038 Page 2

cc: Mr. Kriss Kennedy Regional Administrator U. S. Nuclear Regulatory Commission Region IV RidsRgn4MailCenter@nrc.gov

> NRC Senior Resident Inspector Waterford Steam Electric Station Unit 3 Frances.Ramirez@nrc.gov Chris.Speer@nrc.gov

NRC/NRR Project Manager for Waterford 3 April.Pulvirenti@nrc.gov

Allyson Howie Entergy Legal, General Sr Councel AHowie@entergy.com

Joe Sears Manager Nuclear Asset Protection SearsJT@inpo.org Enclosure to

W3F1-2017-0038

(

Annual Radiological Effluent Release Report - 2016

(38 pages)



January 1, 2016 - December 31, 2016



Waterford 3 SES Entergy Operations, Inc.

Docket Number 50-382

License Number NPF-38

Originator:

h Kreand KMIOL

Danielle Breaud Chemistry Specialist Waterford 3 SES

Date

4/4/17

Reviewed By:

Clay Benton

Chemistry Supervisor Waterford 3 SES

U

17

4/6/17

Date

Approved By:

Nicole Lawless

)

Chemistry Manager Waterford 3 SES Date

Table of Contents

1.0 Introduction	1
2.0 Supplemental Information	2
2.1 Regulatory Limits	2
2.1.1 Fission and Activation Gases (Noble Gases)	2
2.1.2 Iodines, Particulates with Half Lives > Eight (8) Days, and Tritium	3
2.1.3 Liquid Effluents	3
2.1.4 Uranium Fuel Cycle Sources	4
2.2 Maximum Permissible Concentrations	4
2.2.1 Fission and Activation Gases, Iodines, and Particulates, With Half Lives > Eight (8) Days	4
2.2.2 Liquid Effluents	4
2.3 Average Energy (E-Bar)	4
2.4 Measurements and Approximations of Total Radioactivity	5
2.4.1 Fission and Activation Gases (Noble Gases)	5
2.4.2 lodines, Particulates, and Tritium	5
	6
2.5 Batch Releases	b
2.6 1 Lipplanned/Abnormal Releases	0
2.6.2 Unplanned/Abnormal Liquid Releases	0
	0
3.0 Gaseous Effluents	7
4.0 Liquid Effluents	7
5.0 Solid Wastes	7
6.0 Meteorological Data	8
6.0 Meteorological Data 7.0 Assessment of Doses	8 9
6.0 Meteorological Data 7.0 Assessment of Doses 7.1 Dose Due to Gaseous Effluents	8 9 9
6.0 Meteorological Data 7.0 Assessment of Doses 7.1 Dose Due to Gaseous Effluents 7.1.1 Air Doses at the Site Boundary	8 9 9
 6.0 Meteorological Data	9 9 9 9 .10
6.0 Meteorological Data 7.0 Assessment of Doses 7.1 Dose Due to Gaseous Effluents 7.1.1 Air Doses at the Site Boundary 7.1.2 Maximum Organ Dose to the Critical Receptor 7.2 Doses Due to Liquid Effluents	9 9 9 .10 . 11
6.0 Meteorological Data	9 9 9 .10 .11 .11
6.0 Meteorological Data	9 9 .10 .11 .11 .12
 6.0 Meteorological Data	9 9 9 .10 .11 .11 .12
6.0 Meteorological Data	9 9 9 .10 .11 .11 .12 .13
6.0 Meteorological Data 7.0 Assessment of Doses 7.1 Dose Due to Gaseous Effluents 7.1.1 Air Doses at the Site Boundary 7.1.2 Maximum Organ Dose to the Critical Receptor 7.2 Doses Due to Liquid Effluents 7.3 40 CFR Part 190 Dose Evaluation 7.4 Doses to Public Inside the Site Boundary 8.0 Related Information 8.1 Changes to the Process Control Program	9 9 9 .10 .11 .11 .12 .13 .13
6.0 Meteorological Data	9 9 9 .10 .11 .11 .12 .13 13
6.0 Meteorological Data	9 9 9 .10 .11 .12 .13 .13 .14 .14
6.0 Meteorological Data 7.0 Assessment of Doses 7.1 Dose Due to Gaseous Effluents 7.1.1 Air Doses at the Site Boundary 7.1.2 Maximum Organ Dose to the Critical Receptor 7.2 Doses Due to Liquid Effluents 7.3 40 CFR Part 190 Dose Evaluation 7.4 Doses to Public Inside the Site Boundary 8.0 Related Information 8.1 Changes to the Process Control Program 8.2 Changes to the Offsite Dose Calculation Manual 8.3 Unavailability of REMP Milk Samples 8.4 Report of Required Effluent Instrument Inoperability	9 9 .10 .11 .12 .13 .14 .14
6.0 Meteorological Data. 7.0 Assessment of Doses 7.1 Dose Due to Gaseous Effluents 7.1.1 Air Doses at the Site Boundary. 7.1.2 Maximum Organ Dose to the Critical Receptor 7.2 Doses Due to Liquid Effluents 7.3 40 CFR Part 190 Dose Evaluation 7.4 Doses to Public Inside the Site Boundary. 8.0 Related Information 8.1 Changes to the Process Control Program. 8.2 Changes to the Offsite Dose Calculation Manual 8.3 Unavailability of REMP Milk Samples. 8.4 Report of Required Effluent Instrument Inoperability 8.5 Activity Released Via Secondary Pathways	9 9 .10 .11 .12 .13 .14 .14 .14 .15
6.0 Meteorological Data 7.0 Assessment of Doses 7.1 Dose Due to Gaseous Effluents 7.1.1 Air Doses at the Site Boundary 7.1.2 Maximum Organ Dose to the Critical Receptor 7.2 Doses Due to Liquid Effluents 7.3 40 CFR Part 190 Dose Evaluation 7.4 Doses to Public Inside the Site Boundary 8.0 Related Information 8.1 Changes to the Process Control Program 8.2 Changes to the Offsite Dose Calculation Manual 8.3 Unavailability of REMP Milk Samples 8.4 Report of Required Effluent Instrument Inoperability 8.5 Activity Released Via Secondary Pathways 8.6 Missed Effluent Samples	9 9 .10 .11 .12 .13 .14 .14 .15 .15
6.0 Meteorological Data	9 9 .10 .11 .12 .13 .14 .14 .15 .15 .15
6.0 Meteorological Data 7.0 Assessment of Doses 7.1 Dose Due to Gaseous Effluents 7.1.1 Air Doses at the Site Boundary. 7.1.2 Maximum Organ Dose to the Critical Receptor 7.2 Doses Due to Liquid Effluents 7.3 40 CFR Part 190 Dose Evaluation 7.4 Doses to Public Inside the Site Boundary 8.0 Related Information 8.1 Changes to the Process Control Program 8.2 Changes to the Offsite Dose Calculation Manual 8.3 Unavailability of REMP Milk Samples 8.4 Report of Required Effluent Instrument Inoperability 8.5 Activity Released Via Secondary Pathways 8.6 Missed Effluent Samples 8.7 Major Changes to Radioactive Waste Systems 8.8 Biennial Land Use Census 8 9 Genesure 8 9 Genesure	9 9 .10 .11 .12 .13 .14 .14 .15 .15 .15
6.0 Meteorological Data	9 9 .10 .11 .12 .13 .14 .15 .15 .15 .16
6.0 Meteorological Data 7.0 Assessment of Doses 7.1 Dose Due to Gaseous Effluents 7.1.1 Air Doses at the Site Boundary 7.1.2 Maximum Organ Dose to the Critical Receptor 7.1 Ooses Due to Liquid Effluents 7.3 40 CFR Part 190 Dose Evaluation 7.4 Doses to Public Inside the Site Boundary 8.0 Related Information 8.1 Changes to the Process Control Program 8.2 Changes to the Offsite Dose Calculation Manual 8.3 Unavailability of REMP Milk Samples 8.4 Report of Required Effluent Instrument Inoperability 8.5 Activity Released Via Secondary Pathways 8.6 Missed Effluent Samples 8.7 Major Changes to Radioactive Waste Systems 8.8 Biennial Land Use Census 8.9 Gaseous Storage Tank Total Radioactivity Limit	9 9 .10 .11 .12 .13 .14 .14 .15 .15 .15 .16 .16
6.0 Meteorological Data	9 9 .10 .11 .12 .13 .14 .15 .15 .15 .16 .16
6.0 Meteorological Data	9 9 .10 .11 .12 .13 .14 .14 .15 .15 .16 .16

i [.]

.

Table of Contents

10.0 Tables	17
11.0 Attachments	

1.0 Introduction

This Annual Radioactive Effluent Release Report is submitted as required by Waterford 3 Technical Specification 6.9.1.8. It covers the period from January 1, 2016 through December 31, 2016. Information in this report is presented in the format outlined in Appendix B of Regulatory Guide 1.21 and in Section 5.8.1 of the Offsite Dose Calculation Manual (UNT-005-014).

The information contained in this report includes:

- A summary of the quantities of radioactive liquid and gaseous effluents and solid wastes released from the plant during the reporting period.
- A summary of the meteorological data collected during 2016.
- Assessment of radiation doses due to liquid and gaseous radioactive effluents released during 2016.
- A discussion of Unplanned/Abnormal releases that occurred during the reporting period.
- A submittal of changes to the Offsite Dose Calculation Manual and Process Control Program during this reporting period.
- A discussion of why required radioactive effluent monitoring instrumentation was not returned to service within the time specified.
- A discussion of any instances in which effluent samples were not collected within the required frequency.

2.0 Supplemental Information

2.1 Regulatory Limits

The limits applicable to the release of radioactive material in liquid and gaseous effluents are described in the following sections. These limits are addressed by reference in UNT-005-014, Offsite Dose Calculation Manual, and directly in the Technical Requirements Manual (TRM).

2.1.1 Fission and Activation Gases (Noble Gases)

The dose rate due to radioactive noble gases released in gaseous effluents from the site to areas at and beyond the site boundary shall be limited to less than or equal to:

- 500 mrem/yr to the total body; and,
- 3000 mrem/yr to the skin.

The air dose due to noble gases released in gaseous effluents from the site to areas at or beyond the site boundary shall be limited to the following:

- During any calendar quarter, Less than or equal to:
 - 5 mrad for gamma radiation; and,
 - 10 mrad for beta radiation.
- During any calendar year, Less than or equal to:
 - 10 mrad for gamma radiation; and,
 - 20 mrad for beta radiation.

2.1.2 Iodines, Particulates with Half Lives > Eight (8) Days, and Tritium

The dose rate due to lodine-131 and 133, tritium, and all radionuclides in particulate form with half lives greater than eight (8) days, released in gaseous effluents from the site to areas at and beyond the site boundary, shall be limited to less than or equal to:

• 1500 mrem/yr to any organ.

The dose to a member of the public from lodine-131 and 133, tritium, and all radionuclides in particulate form with half lives greater than eight (8) days in gaseous effluents released to areas at and beyond the site boundary shall be limited to the following:

- During any calendar quarter, less than or equal to:
 - 7.5 mrem to any organ.
- During any calendar year, less than or equal to:
 - 15 mrem to any organ.

2.1.3 Liquid Effluents

The concentration of radioactive material released in liquid effluents to unrestricted areas shall be limited to ten times the concentrations specified in 10 CFR Part 20, Appendix B, Table 2, Column 2 for radionuclides other than dissolved or entrained noble gases. For dissolved or entrained noble gases, the concentration shall be limited to 2.0E-4 µCi/ml.

The dose or dose commitment to a member of the public from radioactive materials in liquid effluents released to unrestricted areas shall be limited to the following:

During any calendar quarter, less than or equal to:

- 1.5 mrem to the total body; and,
- 5 mrem to any organ, and

During any calendar year, less than or equal to

- 3 mrem to the total body; and,
- 10 mrem to any organ.

2.1.4 Uranium Fuel Cycle Sources

The dose or dose commitment to any member of the public due to releases of radioactivity and radiation from uranium fuel cycle sources over 12 consecutive months shall be limited to less than or equal to:

- 25 mrem to the Total Body or any organ (except thyroid); and,
- 75 mrem to the thyroid

2.2 Maximum Permissible Concentrations

2.2.1 Fission and Activation Gases, Iodines, and Particulates, With Half Lives > Eight (8) Days

For gaseous effluents, maximum permissible concentrations are not directly used in release rate calculations since the applicable limits are expressed in terms of dose rate at the site boundary.

2.2.2 Liquid Effluents

Ten times the effluent concentration (EC) values specified in 10 CFR Part 20, Appendix B, Table 2, Column 2 are used as the permissible concentrations of liquid radioactive effluents at the unrestricted area boundary. A value of 2.0E-4 μ Ci/ml is used as the concentration limit for dissolved and entrained noble gases in liquid effluents.

2.3 Average Energy (E-Bar)

This is not applicable to Waterford 3 effluent specifications. E-Bar is not required to be calculated from effluent release data. The average energy (E-Bar) for the Reactor Coolant System (RCS) is supplied as additional information in the report further below.

2.4 Measurements and Approximations of Total Radioactivity

The quantification of radioactivity in liquid and gaseous effluents was accomplished by performing the sampling and radiological analysis of effluents in accordance with the requirements of Tables 4.11-1 and 4.11-2 of the Technical Requirements Manual (TRM).

2.4.1 Fission and Activation Gases (Noble Gases)

For continuous releases, a gas grab sample was analyzed at least monthly for noble gases. Each week a Gas Ratio (GR) was calculated according to the following equation:

 $GR = \frac{Average Weekly Noble Gas Monitor Reading}{Monitor Reading During Noble Gas Sampling}$

The monthly sample analysis and weekly Gas Ratio were then used to determine noble gases discharged continuously for the previous week. For gas decay tank and containment purge batch releases, a gas grab sample was analyzed prior to release to determine noble gas concentrations in the batch. In all cases, the total radioactivity in gaseous effluents was determined from measured concentrations of each radionuclide present and the total volume discharged.

2.4.2 Iodines, Particulates, and Tritium

lodines and particulates discharged were sampled using a continuous sampler which contained a charcoal cartridge and a particulate filter. Each week the charcoal cartridge and particulate filter were analyzed for gamma emitters using gamma spectroscopy. The determined radionuclide concentrations and effluent volumes discharged were used to calculate the previous week's activity released. The particulate samples were composited and analyzed quarterly for Sr-89 and Sr-90 by a contract laboratory (Teledyne Brown Engineering). Particulate gross alpha activity was measured weekly using alpha scintillation or gas-flow proportional counting techniques. The determined activities were used to estimate effluent concentrations in subsequent releases until the next scheduled analysis was performed. Annual Carbon-14 release estimate was obtained from the Waterford 3 Final Safety Analysis Report. Release of Carbon-14 was assumed to be continuous.

Grab samples of continuous releases were analyzed at least monthly for tritium. Containment purge batch releases are analyzed prior to release. The determined concentrations were used to estimate tritium activity in subsequent releases until the next scheduled analysis was performed.

2.4.3 Liquid Effluents

For continuous releases, samples were collected weekly and analyzed using gamma spectroscopy. The measured concentrations were used to determine radionuclide concentrations in the following week's releases. For batch releases, gamma analysis was performed on the sample prior to release.

For both continuous and batch releases, composite samples were analyzed quarterly by a contract laboratory (Teledyne Brown Engineering) for Sr-89, Sr-90, and Fe-55. Samples were composited and analyzed monthly for tritium and gross alpha using liquid scintillation and gas flow proportional counting techniques, respectively. For radionuclides measured in the composite samples, the measured concentrations in the composite samples from the previous month or quarter were used to estimate released quantities of these isotopes in liquid effluents during the current month or quarter when the analysis results became available.

The total radioactivity in liquid effluent releases was determined from the measured and estimated concentrations of each radionuclide present and the total volume of the effluent discharged.

2.5 Batch Releases

A summary of information for gaseous and liquid batch releases is included in Table 1.

2.6 Unplanned/Abnormal Releases

2.6.1 Unplanned/Abnormal Gaseous Releases

There were no unplanned/abnormal gaseous releases during the reporting period.

2.6.2 Unplanned/Abnormal Liquid Releases

There were no unplanned/abnormal liquid releases during this reporting period.

3.0 Gaseous Effluents

The quantities of radioactive material released in gaseous effluents are summarized in Tables 1A, 1B, and 1C. Note that there were no elevated releases, since all Waterford 3 releases are considered to be at ground level. The estimated total error in % is based upon several statistical uncertainties due to sample counting, efficiency, volume, etc.

4.0 Liquid Effluents

The quantities of radioactive material released in liquid effluents are summarized in Tables 2A and 2B. The estimated total error in % is based upon several statistical uncertainties due to sample counting, efficiency, volume, etc.

5.0 Solid Wastes

The summary of radioactive solid wastes shipped offsite for disposal is listed in Table 3. For certain waste forms, Waterford 3 uses volume reduction services provided by a contractor. These waste forms are included in Table 3 and the volumes reported reflect the volume of waste shipped offsite, not final disposal volumes. Final disposal volumes for wastes compacted offsite are available upon request. The estimated total error in % is based upon several statistical uncertainties due to sample counting, efficiency, volume, etc.

6.0 Meteorological Data

In Table 4, the hourly meteorological data from January 1, 2016 through December 31, 2016, is presented in the form of a joint frequency distribution of wind speed, wind direction, and atmospheric stability (hourly data is also available upon request). The standard Pasquill classification scheme, as presented in Regulatory Guide 1.23, is used to determine stability class from differential temperature measurements. The Waterford-3 data recovery results by parameter are as follows:

Differiential Temp.	99.95%
Wind Speed	99.95%
Wind Direction	99.95%
Overali*	99.95%

A. * Simultaneous occurrence of valid data for all three parameters.

Dispersion and deposition values were determined from the 2016 data and used in the assessment of doses due to gaseous effluents released from site during the 2016 period.

7.0 Assessment of Doses

7.1 Dose Due to Gaseous Effluents

7.1.1 Air Doses at the Site Boundary

Air doses from gaseous effluents were evaluated at the closest offsite location that could be occupied continuously during the term of plant operation and that would result in the highest dose. This location was determined by examining the atmospheric dispersion parameters (χ/Q) at the closest offsite locations that could be continuously occupied during plant operation in each of the meteorological sectors surrounding the plant. The location that would have the highest dose would be that location having the most restrictive (largest) χ/Q value.

Based on actual meteorological data collected during 2016, this location was determined to be in the NNE sector ($\chi/Q = 2.6E-05 \text{ sec/m}^3$) at a distance of 869 meters (0.54 miles) from the reactor building. Doses were assessed at this location in accordance with the methodology described in the Waterford 3 Offsite Dose Calculation Manual considering only beta and gamma exposures in air due to noble gas. The results of these assessments for the year 2016 are summarized as follows:

Beta air dose:	0 mrad
Gamma air dose:	0 mrad

The above beta and gamma air doses represent the following percentage of the annual dose limits:

0 % of the Beta air dose limit (20 mrad)0% of the Gamma air dose limit (10 mrad)

Dose calculation results are summarized by quarters in Table 5A. The doses were calculated in accordance with the methodology described in the Waterford 3 Offsite Dose Calculation Manual.

7.1.2 Maximum Organ Dose to the Critical Receptor

The maximum organ dose to a MEMBER OF THE PUBLIC from I-131, I-133, tritium, and all radionuclides in particulate form with half-lives greater than eight (8) days in gaseous effluents released to areas at and beyond the site boundary was determined for 2016.

An assessment of the maximum organ dose was performed for the critical receptor. The critical receptor was assumed to be located at the nearest residence to the plant having the most restrictive atmospheric dispersion (χ/Q) and deposition (D/Q) parameters. Furthermore, it was assumed that the receptor living at this residence consumed food products that were either raised or produced at this residence.

Using land use census and meteorological data for 2016 the residence with the highest χ /Q value (6.8-06 sec/m³) was determined to be in the ENE sector at a distance of 1513 meters (0.94 miles) from the reactor building while the highest D/Q value (1.5E-08 m⁻²) was determined to be in the NE sector at a distance of 1432 meters (0.89 miles) from the reactor building. The dose calculation was performed in accordance with the methodology described in the Waterford 3 Offsite Dose Calculation Manual considering the inhalation, ground plane exposure, and ingestion pathways. The maximum organ dose to the critical receptor excluding C-14 was determined for historical trending to be:

0.0260 mrem to the child liver.

This represents 0.17% of the Annual Organ Dose limit (15 mrem).

The maximum organ dose to the critical receptor including C-14 was determined to be:

3.45 mrem to the child bone.

This represents 23.0% of the Annual Organ Dose limit (15 mrem).

Dose calculation results are summarized by quarters in Table 5A. The doses were calculated in accordance with the methodology described in the Waterford 3 Offsite Dose Calculation Manual.

7.2 Doses Due to Liquid Effluents

The annual doses to the maximum exposed individual, an adult, resulting from exposure to liquid effluents released during 2016 from Waterford 3 were:

2.16E-4 mrem to the Total Body.2.65E-4 mrem to the maximum exposed organ (Gi-LLi)

The above doses represent the following percentage of the Annual Dose limits:

0.01% of the Total Body Dose Limit (3 mrem), and 0.003% of the Organ Dose Limit (10 mrem).

Dose calculation results are summarized by quarter in Table 5B. The doses were calculated in accordance with the methodology described in the Waterford 3 Offsite Dose Calculation Manual.

7.3 40 CFR Part 190 Dose Evaluation

In accordance with Technical Requirements Manual (TRM), Specification 3/4.11.4, Total Dose, dose evaluations to demonstrate compliance with Surveillance Requirements 4.11.4.1 and 4.11.4.2 of the Technical Requirements Manual (TRM), dealing with dose from the uranium fuel cycle, need to be performed only if quarterly doses exceed 3 mrem to the total body (liquid releases), 10 mrem to any organ (liquid releases), 10 mrad gamma air dose, 20 mrad beta air dose, or 15 mrem to any organ from radioiodines and particulates.

At no time during 2016 were any of these limits exceeded.

Direct shine dose was considered using the area monitoring DLR results in the 2015 Waterford 3 Area Monitoring Program Report. The highest perimeter DLR badge dose located nearest a critical receptor did not result in any additional dose to the critical receptor.

7.4 Doses to Public Inside the Site Boundary

The Member of the Public inside the site boundary expected to have the maximum exposure due to gaseous effluents would be an employee at the Waterford 1 and 2 fossil fuel plants, located in the NW sector at a distance of approximately 670 meters (0.42 miles) from the reactor building.

The doses for such an individual were determined by scaling the full-time occupancy doses due to airborne effluents by the occupancy time due to a normal working year. Based on an assumed occupancy of 25% (40 hour work week) and the fact that all employees are adults, the calculated doses were determined to be less than:

4.97E-03 mrem to the maximum exposed organ (Liver)

All doses for receptors inside the site boundary were calculated according to the methodology described in the Waterford 3 Offsite Dose Calculation Manual considering only the inhalation and ground plane exposure pathways.

8.0 Related Information

8.1 Changes to the Process Control Program

There were no changes to EN-RW-105, "Process Control Program", in 2016. The following list summarizes changes made in 2016 to procedures related to the Process Control Program (PCP):

EN-RW-102, "Radioactive Shipping Procedure", Rev.14:

The primary purpose of this revision is to update the procedure to ensure compliance with current and upcoming changes to 49 CFR, especially those changes stemming from HM-215M and HM-250 (changes are as recommended by review of EN-RW-102 by WMG). Additionally, the issue identified in NIOS audit QA-14/15-2015-IP-01 regarding retention of DOT Spec 7A test and engineering evaluation is addressed.

- Added a bullet item to step 7.0[2] specifying that shipment records are to include documentation of tests and engineering evaluations for DOT Spec 7A packages
- Step 2.0[10]: Corrected reference to WMG software packages
- Step 5.1[12]: replaced specific shipping names with generic statement regarding Marking and Labeling
- Step 5.2[11](c), Attachment 9.10: Updated terminology for Industrial Package
- Step 5.2[11](c): Added requirement to cover Specification Marking
- Step 5.2[16]: Added reference for requirements for multiple hazard radioactive material
- Step 5.2[19]: Revised step to indicate that the survey only releases the vehicle back to exclusive use service
- Added step 5.2[20] for unconditional release of formerly exclusive use vehicle.
- Step 7.0[1]: Added an additional reference for record retention
- Step 7.0[2]: Added bullet item requiring that documentation of tests and engineering evaluations be kept for records
- Attachment 9.1: replaced RAMSHP with RADMAN and replaced obsolete reference to Studsvik with Erwin Resin Solutions
- Attachment 9.2, step 21: replaced Studsvik with Erwin Resin Solutions
- Attachment 9.3, step 12: combined sub-step "a" with step 12 as there is no step "b"
- Attachment 9.3, step 13: replaced Studsvik with Erwin Resin Solutions
- Attachment 9.3, step 24: added requirement for DOT 7A test and engineering documentation
- Attachment 9.4, step 14: replaced Studsvik with Erwin Resin Solutions
- Attachment 9.4: Added a new step #5 for verifying the licensee is a registered user of Type B package
- Attachment 9.5, step #9: added reference to requirements of 49CFR173.422
- Attachment 9.6, step #8: added reference to requirements of 49CFR173.422
- Attachment 9.7, step #10: added reference to requirements of 49CFR173.422
- Attachment 9.8, step #7: added reference to requirements of 49CFR173.422
- Attachment 9.9, step #6: added reference to requirements of 49CFR173.422
- Attachment 9.10: updated references for Industrial Package (I, II, and III changed to Type 1,2,3)
- Attachment 9.11, sheet 2: updated contamination limits to reflect latest regulations
- Attachment 9.14: updated definition of Exclusive Use according to WMG recommendation
- Updated section 8.0 and related procedure steps per GGNS Commitment Review

EN-RW-106, "Integrated Transportation Security Plan", Rev. 4:

The purpose of this revision is to address:

- A QA-identified issue regarding documentation of preplanning activities
- Correct the NRC email address
- · Add a requirement for contacting ANI
- Added step 2.0[20] for reference to NUREG-2155 (which is the basis for preplanning documentation requirements)
- Step 3.0[1] added: ANI (American Nuclear Insurers), MCC Movement Control Center, SAS Secondary Alarm Station
- Step 5.3[4]: corrected step reference
- Step 5.7.4.1[3]: corrected the NRC email address
- Added steps 5.7.5.2[8] and 5.7.5.3[7] for ANI notification in the event of an accident.
- Step 5.7.6.1[2]: expanded requirements for documentation of preplanning activities
- Step 5.8[2] is reworded for clarity

Page 13 of 38

1.

EN-RW-108, "Radioactive Shipment Accident Response", Rev.2:

The primary purpose of this revision is to add instruction for contacting ANI in the event of an accident covered by this procedure. Specifically:

- ANI is added to the definitions section, section 3.0
- Step 5.4[1] is expanded to address contacting ANI

This is a complete rewrite of the procedure in order to align the procedure format with EN-AD-101-01. Content changes are made only to correct grammatical errors or to change step levels to correct formatting and are editorial. No change bars are used to reflect the changes made due to the rewrite. Changes worth noting are:

- Sections 5.1 and 5.2 are reversed in order for alignment with EN-AD-101-01
- Section 5.3.2: "CAUTION" is changed to "NOTE" in the box associated with ion chambers versus GM instruments as the associated statement does not qualify as a caution statement.
- The Note statement in section 5.4[3] (of Rev. 1) regarding evaluation for fatigue is changed to an instructional step as the statement is not appropriate for a Note statement.
- The statement in section 5.4[3] (of Rev. 1) regarding the reference to EN-OM-123 is changed to an instructional step as the statement is not appropriate for a Note statement or a Caution statement.
- The last Note statement in section 5.0 regarding disposition of documents is changed to a step as the contents are not appropriate for a Note statement
- Attachment 9.1: changed note on page one to a step because the content is inappropriate for a Note.

8.2 Changes to the Offsite Dose Calculation Manual

There were no changes to the ODCM in 2016.

8.3 Unavailability of REMP Milk Samples

Due to the unavailability of three milk sampling locations within five kilometers of the plant, Broad Leaf sampling is performed in accordance with Technical Requirements Manual (TRM) Table 3.12-1. Milk is collected, when available, from the control location and one identified sampling location as indicated in UNT-005-014, Offsite Dose Calculation Manual, Attachment 7.13.

8.4 Report of Required Effluent Instrument Inoperability

Technical Requirements Manual (TRM) Specifications 3.3.3.10 and 3.3.3.11 require reporting in the Annual Radioactive Effluent Release Report of why designated inoperable effluent monitoring instrumentation was not restored to operability within the time specified in the Action Statement.

During the reporting period, all instrumentation was restored to operability within the time specified.

8.5 Activity Released Via Secondary Pathways

The following secondary release paths were continuously monitored for radioactivity:

- The Hot Machine Shop Exhaust (AH-35),
- Decontamination Shop Exhaust (AH-34),
- The RAB H&V Equipment Room Ventilation System Exhaust (E-41A and E-41B); and,
- The Switchgear/Cable Vault Area Ventilation System (AH-25).

Continuous sampling for these areas is maintained in order to demonstrate the operability of installed treatment systems and to verify integrity of barriers separating primary and secondary ventilation systems. Sampling for these areas was limited to continuous particulate and iodine sampling and monthly noble gas grab sampling. The activity released via these secondary pathways resulted from routine operations and remained below significant levels.

8.6 Missed Effluent Samples

During the reporting period, no incident occurred for which effluent samples were not sampled and/or analyzed as required by the ODCM/TRM.

8.7 Major Changes to Radioactive Waste Systems

During the reporting period, no major changes were made to any Radioactive Waste Systems. All major changes to Radioactive Waste Systems are included in Waterford 3's FSAR updates.

8.8 Biennial Land Use Census

A land use census was last performed in 2016. The land use census performed in 2016 did not identify the need for any changes to locations being used for effluent dose calculations or radiological environmental sampling.

8.9 Gaseous Storage Tank Total Radioactivity Limit

Technical Specification 3/4.11.2.6 specifies that the quantity of radioactivity contained in each gas storage tank be maintained less than or equal to 8.5E+04 Curies noble gas (considered as Xe-133 equivalent). At no time during the reporting period was this value exceeded.

8.10 Unprotected Outside Tank Total Radioactivity Limit

Technical Specification 3/4.11.1.4 specifies that the quantity of radioactive material contained in each unprotected outdoor tank be maintained less than or equal to 7.85E-04 Curies (excluding tritium and dissolved and entrained noble gases). During this reporting period, there were no instances in which this limit was exceeded.

9.0 Additional Information

9.1 Reactor Coolant System Average Energy (E-Bar)

Reactor Coolant System E-Bar calculations were done on 3/14/16 and 8/15/16 with values of 0.0677 and 0.0452 Mev/disintegration, respectively. Reactor Coolant System E-Bar is supplied for information only and is not used for effluent dose calculations.

9.2 Groundwater Protection Initiative (NEI 07-07) Annual Reporting

Groundwater wells were monitored at Waterford 3 during 2016 as part of the NEI Groundwater Initiative. Sampling of the ten installed wells was conducted on a quarterly basis. All results were less than minimum detectable activity for gamma emitters and tritium during 2016. A summary of all groundwater monitoring well sample results for 2016 is presented in Table 6.

Waterford 3 made one voluntary communication (per NEI 07-07 Objective 2.2) on October 31, 2016 due to an onsite spill exceeding 100 gallons from a source that contained licensed material that had the potential to reach groundwater. Approximately 150 gallons of contaminated (H-3 8.2E-6 uCi/mL) water entered the storm drains as a result of a hose becoming dislodged from the inside of a temporary water storage tank located outside during a plant outage. Subsequent sampling of the storm water, via the surface water REMP location at the point of offsite release, did not result in any detectable levels of activity.

10.0 Tables

Table 1,	Batch Release Summary19
Table 1A,	Annual Summation of All Releases by Quarter All Airborne Effluents20
Table 1B,	Annual Airborne Continuous Elevated and Ground Level Releases Totals for Each Nuclide Released21
Table 1C,	Annual Airborne Batch Elevated and Ground Level Releases Totals for Each Nuclide Released22
Table 2A,	Annual Summation of All Releases by Quarter All Liquid Effluents
Table 2B,	Annual Liquid Continuous and Batch Releases Totals for Each Nuclide Released24
Table 3,	Solid Waste Shipped Offsite for Burial or Disposal25
Table 4,	Joint Frequency Distribution of Meteorological Data
Table 5A,	Doses Due to Gaseous Radioactive Effluents35
Table 5B,	Doses Due to Liquid Radioactive Effluents
Table 6,	2016 Groundwater Monitoring Well Results

11.0 Attachments

- 11.1 Copy of UNT-005-014, "Offsite Dose Calculation Manual" Revision 306.
- 11.2 Copy of EN-RW-102, "Radioactive Shipping Procedure" Revision 12.
- 11.3 Copy of EN-RW-104, "Scaling Factors" Revision 12.
- 11.4 Copy of EN-RW-105, "Process Control Program" Revision 5.
- 11.5 Copy of EN-RW-106, "Integrated Transportation Security Plan" Revision 4.

9

Table 1Batch Release Summary

Batch Release Summary information for 2016 Report Period.

Report Category Release Point Type of Release Period Start Time Period End Time	: Batch Release Summary : All : Batch Liquid and Gaseous : 01-jan-2016 00:00:00 : 31-dec-2016 23:59:59
Liqui	d Releases
Number of Releases	: 74 eases : 20123.0 Minutes

IULAL I.	rwê ro	L LT	-	rereases	-	20123.0	Hindce3
Maximum	Time	for	a	Release	:	314.0	Minutes
Average	Time	for	а	Release	4	271.9	Minutes
Minimum	Time	for	а	Release	:	187.0	Minutes
Average	Strea	am Fl	OW	,	:	607298	ft³/s
		_	-				

Gaseous Releases

Number of Releases		:	0
Total Time for All	Releases	:	0.0 Minutes
Maximum Time for a	Release	:	0.0 Minutes
Average Time for a	Release	:	0.0 Minutes
Minimum Time for a	Release	:	0.0 Minutes

Batch Release Summary information for 2016 by Quarter.

Report Category Release Point	:	Batch Release Summary All
Type of Release	:	Batch Liquid and Gaseous
Period Start Time	:	01-jan-2016 00:00:00
Period End Time	:	31-dec-2016 23:59:59

		Liquid Re	eleases			
		Qtr 1	Qtr 2	Qtr 3	Qtr 4	
Number of Releases Total Time for All Releases	:	18 4751.0	13 3570.0	22 5947.0	21 5855.0	Minutes
Maximum Time for a Release Average Time for a Release Minimum Time for a Release	:	297.0 263.9 187.0	303.0 274.6 248.0	290.0 270.3 232.0	314.0 278.8 214.0	Minutes Minutes Minutes
Average Stream Flow	:	951137 	699891 eleases	448826	329336	ft'/s
		Qtr 1	Qtr 2	Qtr 3	Qtr	4
Number of Releases	:	0	0	0	0	
Total Time for All Releases	:	0.0	0.0	0.0	0.0	Minutes
Maximum Time for a Release	:	0.0	0.0	0.0	0.0	Minutes
Average Time for a Release Minimum Time for a Release	: :	0.0 0.0	0.0 0.0	0.0	0.0	Minutes Minutes

Table 1A Annual Summation of All Releases by Quarter All Airborne Effluents

Report Category	:	Summation of All Releases
Type of Activity	:	All Airborne Effluents
Period Start Time	:	01-jan-2016 00:00:00
Period End Time	:	31-dec-2016 23:59:59

Type of Effluent	Units	Qtr 1	Qtr 2	Qtr 3	Qtr 4	Est.Total Error %
A. Fission and Activation Gases						
1. Total Release	Curies	0.00E+00	0.00E+00	0.00E+00	0.00E+00	2.50E+01
2. Average Release Rate for Period	µCi/sec	0.00E+00	0.00E+00	0.00E+00	0.00E+00	
3. Percent of Applicable Limit	80	0.00E+00	0.00E+00	0.00E+00	0.00E+00	
B. Radioiodines						
1. Total Iodine-131	Curies	0.00E+00	0.00E+00	0.00E+00	0.00E+00	2.50E+01
2. Average Release Rate for Period	µCi/sec	0.00E+00	0.00E+00	0.00E+00	0.00E+00	
3. Percent of Applicable Limit	olo	0.00E+00	0.00E+00	0.00E+00	0.00E+00	
C. Particulates						
1. Total Particulates (Half-lives>8 days)	Curies	0.00E+00	0.00E+00	0.00E+00	1.99E-6	2.50E+01
2. Average Release Rate for Period	µCi/sec	0.00E+00	0.00E+00	0.00E+00	2.51E-7	
3. Percent of Applicable Limit	90 ·	0.00E+00	0.00E+00	0.00E+00	8.96E-4	
4. Gross Alpha Activity	Curies	3.11E-7	3.63E-7	1.95E-7	8.39E-8	2.50E+01
5. Carbon-14	Curies	2.53E+00	2.53E+00	2.53E+00	2.53E+00	
D. Tritium						
1. Total Release	Curies	8.66E+00	8.80E+00	9.89E+00	4.05E+00	2.50E+01
2. Average Release Rate for Period	µCi/sec	1.10E+00	1.12E+00	1.24E+00	5.09E-01	
3. Percent of Applicable Limit	olo	9.53E-02	9.68E-02	1.09E-01	4.45E-02	

Table 1B Annual Airborne Continuous Elevated and Ground Level Releases Totals for Each Nuclide Released

Report Category Type of Activity Period Start Time Period End Time	<pre>tegory : Airborne Continuous Elevated and Ground Level Releases. : Totals for Each Nuclide Released. ctivity : Fission Gases, Iodines, and Particulates art Time : 01-jan-2016 00:00:00 d Time : 31-dec-2016 23:59:59</pre>								
······································			Elevated R	eleases			Ground Re	leases	
Nuclide	Units	Qtr 1	Qtr 2	Qtr 3	Qtr 4	Qtr 1	Qtr 2	Qtr 3	Qtr 4
Fission and Activa	tion _, Gases								
Total for Period	Curies	0.00E+00							
Radioiodines									
Total for Period	Curies	0.00E+00							
Particulates									
H-3 Cs-137 Gralpha C-14	Curies Curies Curies Curies	0.00E+00 0.00E+00 0.00E+00 0.00E+00	0.00E+00 0.00E+00 0.00E+00 0.00E+00	0.00E+00 0.00E+00 0.00E+00 0.00E+00	0.00E+00 0.00E+00 0.00E+00 0.00E+00	8.66E+00 0.00E+00 3.11E-07 2.53E+00	8.80E+00 0.00E+00 3.63E-07 2.53E+00	9.89E+00 0.00E+00 1.95E-07 2.53E+00	4.05E+00 1.99E-06 8.39E-08 2.53E+00
Total for Period	Curies	0.00E+00	0.00E+00	0.00E+00	0.00E+00	1.12E+01	1.13E+01	1.24E+01	6.58E+00

Table 1CAnnual Airborne Batch Elevated and Ground Level ReleasesTotals for Each Nuclide Released

Report Category : Airborne Batch Elevated and Ground Level Releases.									
Type of Activity Period Start Time Period End Time	: Totals f : Fission : 01-jan-2 : 31-dec-2	or Each Nuc Gases, Iodi 016 00:00:0 016 23:59:5	nes, and F 0 9	articulate	S				
			Elevated R	eleases		Gro	und Releas	es	
Nuclide	Units	Qtr 1	Qtr 2	Qtr 3	Qtr 4	Qtr 1	Qtr 2	Qtr ³	Qtr 4
Fission and Activa	tion Gases								
Total for Period	Curies	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Radioiodines									
Total for Period	Curies	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Particulates									
Total for Period	Curies	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00

Table 2AAnnual Summation of All Releases by QuarterAll Liquid Effluents

Rej Tyj Pe: Pe:	port Category : Summation of All Releases pe of Activity : All Liquid Effluents riod Start Time : 01-jan-2016 00:00:00 riod End Time : 31-dec-2016 23:59:59						
Туј	pe of Effluent	Units	Qtr 1	Qtr 2	Qtr 3	Qtr 4	Est.Total Error %
Α.	Fission and Activation Products			-			
	1. Total Release (not including Tritium,	Curies	3.72E-03	1.37E-03	3.08E-03	3.56E-03	2.50E+01
	2. Average Diluted Concentration During	µCi/ml	2.25E-10	1.17E-10	1.76E-10	1.35E-10	
	3. Percent of Applicable Limit	° .	7.44E-02	2.73E-02	6.16E-02	7.13E-02	·· <u></u>
в.	Tritium						
	1. Total Release	Curies	9.71E+00	5.91E+00	1.75E+01	1.90E+02	2.50E+01
	2. Average Diluted Concentration During	µCi/ml	5.87E-07	5.05E-07	9.96E-07	7.66E-06	
	3. Percent of Applicable Limit	%	5.87E-02	5.05E-02	9.96E-02	7.22E-01	
c.	Dissolved and Entrained Gases	_					
	1. Total Release	Curies	4.10E-05	0.00E+00	7.78E-05	1.81E-04	2.50E+01
	2. Average Diluted Concentration During	µCi/ses	2.47E-12	0.00E+00	4.43E-12	6.88E-12	
	3. Percent of Applicable Limit	%	1.24E-06	0.00E+00	2.22E-06	3.44E-06	
D.	Gross Alpha Radioactivity						
_	1. Total Release	Curies	0.00E+00	0.00E+00	0.00E+00	0.00E+00	2.50E+01
E. F.	Waste Volume Released (Pre-Dilution) Volume of Dilution Water Used	Liters Liters	5.47E+06 1.65E+10	4.72E+06 1.17E+10	4.19E+06 1.75E+10	4.53E+06 2.63E+10	2.50E+01 2.50E+01

Page 23 of 38

٠,

Table 2BAnnual Liquid Continuous and Batch ReleasesTotals for Each Nuclide Released

	Secon noted
: Totals for Each Nuclid	e Released.
Type of Activity : All Radionuclides	
Period Start Time : 01-jan-2016 00:00:00	
Period End Time : 31-dec-2016 23:59:59	

	Continuous Releases				s	Batch Releases			
Nuclide	Units	Qtr 1	Qtr 2	Qtr 3	Qtr 4	Qtr 1	Qtr 2	Qtr 3	Qtr 4
All Nuclides		_		_				-	_
Ag-110m	Curies	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	8.23E-05	0.00E+00
Br-82	Curies	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	1.35E-05	0.00E+00
Co-57	Curies	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	6.51E-06	0.00E+00
Co-58	Curies	1.89E-04	2.28E-05	0.00E+00	0.00E+00	4.63E-04	8.22E-05	2.32E-04	1.05E-04
Co-60	Curies	4.80E-05	3.58E-06	0.00E+00	0.00E+00	1.15E-04	5.96E-05	4.79E-04	1.86E-04
Cs-137	Curies	1.93E-06	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	4.55E-06	2.16E-06
Fe-55	Curies	0.00E+00	0.00E+00	0.00E+00	0.00E+00	2.87E-03	1.11E-03	1.59E-03	3.19E-03
Fe-59	Curies	0.00E+00	0.00E+00	0.00E+00	0.00E+00	1.29E-05	0.00E+00	0.00E+00	0.00E+00
H-3	Curies	1.66E-02	2.16E-02	2.29E-02	3.39E-02	9.69E+00	5.89E+00	1.74E+01	1.90E+02
Mn-54	Curies	0.00E+00	0.00E+00	0.00E+00	0.00E+00	5.96E-06	0.00E+00	6.48E-05	2.69E-05
Nb-95	Curies	5.52E-06	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	4.29E-05	1.73E-05
Sb-125	Curies	0.00E+00	0.00E+00	0.00E+00	0.00E+00	5.69E-06	8.56E-05	5.51E-04	1.57E-05
Xe-133	Curies	0.00E+00	0.00E+00	0.00E+00	0.00E+00	4.10E-05	0.00E+00	7.78E-05	1.57E-04
Xe-135	Curies	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	2.40E-05
Zn-65	Curies	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	8.68E-06
Zr-95	Curies	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	1.67E-05	1.35E-05
Total for Period	Curies	1.68E-02	2.16E-02	2.29E-02	3.39E-02	9.70E+00	5.89E+00	1.74E+01	1.90E+02

Table 3

Solid Waste Shipped Offsite for Burial or Disposal

SUMMARY BY MAJOR WASTE TYPES

Waste	Volume		Curies	%Error
Class		M^3	Shipped	(Ci)
A	1.94E+03	5.49E+01	9.31E+00	+/-25%
В	1.05E+02	2.97E+00	2.06E+02	+/-25%
С	0.00E+00	0.00E+00	0.00E+00	+/-25%
ALL	2.05E+03	5.79E+01	2.15E+02	+/-25%

Waste Stream : Resins, Filters, and Evaporator Bottoms +

Waste Stream : Dry Active Waste +

Waste	Volume	Volume	Curies	%Error
Class	Ft^3	M^3	Shipped	(Ci)
A	1.53E+04	4.33E+02	1.47E+00	+/-25%
В	0.00E+00	0.00E+00	0.00E+00	+/-25%
С	0.00E+00	0.00E+00	0.00E+00	+/-25%
ALL	1.53E+04	4.33E+02	1.47E+00	+/-25%

Waste Stream : Irradiated Components

Waste	Volume	Volume	Curies	%Error
Class	Ft ⁻ 3	M^3	Shipped	(Ci)
A	0.00E+00	0.00E+00	0.00E+00	+/-25%
В	0.00E+00	0.00E+00	0.00E+00	+/-25%
C	0.00E+00	0.00E+00	0.00E+00	+/-25%
ALL	0.00E+00	0.00E+00	0.00E+00	+/-25%

Waste Stream : Other Waste

Waste	Volume	Volume	Curies	%Error
Class	Ft^3	M^3	Shipped	(Ci)
A	4.42E+02	1.25E+01	3.36E-02	+/-25%
В	0.00E+00	0.00E+00	0.00E+00	+/-25%
C	0.00E+00	0.00E+00	0.00E+00	+/-25%
ALL	4.42E+02	1.25E+01	3.36E-02	+/-25%

Waste Stream : Sum of All 4 Categories

Waste	Volume	Volume	Curies	%Error
Class	Ft^3	M^3	Shipped	(Ci)
A	1.77E+04	5.00E+02	1.08E+01	+/-25%
В	1.05E+02	2.97E+00	2.06E+02	+/-25%
С	0.00E+00	0.00E+00	0.00E+00	+/-25%
ALL.	1.78E+04	5.03E+02	2.17E+02	+/-25%

* Activity determined by estimations

Activity determined by measurements

Table 3 (cont)

Estimate of major nuclide composition (by waste type)

Waste Stream : Resins, Filters, and Evap Bottoms

Nuclide	Percent	Curies
Name	Abundance	
H-3	0.17%	3.71E-01
C-14	0.05%	1.08E-01
Mn-54	1.63%	3.51E+00
Fe-55	4.87%	1.05E+01
Co-57	0.94%	2.03E+00
Co-58	2.46%	5.30E+00
Co-60	11.61%	2.50E+01
Ni-59	0.38%	8.10E-01
Ni-63	62.59%	1.35E+02
Zn-65	0.08%	1.71E-01
Sr-90	0.01%	2.84E-02
Nb-94	0%	7.69E-06
Tc-99	0%	5.13E-03
Sb-125	0.24%	5.27E-01
I-129	0%	5.57E-04
Cs-134	4.7%	1.01E+01
Cs-137	10.22%	2.20E+01
Ce-144	0.04%	8.41E-02
Pu-238	0%	4.41E-06
Pu-241	0%	1.65E-03
Am-241	0%	4.63E-05
Cm-243	0%	8.49E-05
Cm-244	0%	1.57E-06

Table 3 (cont)

Estimate of major nuclide composition (by waste type)

Nuclide	Percent	Curies
Name	Abundance	4.45.00
H-3	0.21%	4.41E-03
Be-7	6.1%	1.31E-01
<u>C-14</u>	0.1%	2.09E-03
<u>Cr-51</u>	0.2%	4.24E-03
Mn-54	6.21%	1.33E-01
Fe-55	24.88%	5.33E-01
Fe-59	0.07%	1.46E-03
Co-57	0.61%	1.30E-02
Co-58	26%	5.57E-01
Co-60	8.68%	1.86E-01
Ni-59	0.13%	2.71E-03
Ni-63	21.7%	4.65E-01
Zn-65	0.57%	1.22E-02
Sr-89	0.04%	9.05E-04
Sr-90	0.01%	2.26E-04
Zr-95	0.01%	1.68E-04
Nb-94	0%	1.00E-05
Nb-95	0.02%	4.28E-04
Tc-99	0%	5.93E-05
Ag-110m	0%	3.34E-06
Sn-113	0.02%	3.71E-04
Sb-125	0.16%	3.42E-03
1-129	0%	1.10E-05
Cs-134	1.38%	2.96E-02
Cs-137	2.87%	6.15E-02
Ce-144	0.04%	9.13E-04
Np-239	0%	1.28E-06
Pu-238	0%	5.79E-06
Pu-239	0%	4.96E-09
Pu-240	0%	3.77E-10
Pu-241	0%	1.74E-05
Pu-242	0%	5.62E-10
Am-241	0%	2.71E-06
Cm-242	0%	5.80E-08
Cm-243	0%	2.36E-06
Cm-244	0%	2.26E-06

Estimate of major nuclide composition (by waste type)

Waste Stream : Irradiated Components

N/A - None Shipped in 2016.

.....

Table 3 (cont)

Estimate of major nuclide composition (by waste type)

Waste Stream : Other Waste (Combined Packages)

Nuclide	Percent	Curies
Name	Abundance	
H-3	3.29%	1.10E-03
C-14	0.31%	1.03E-04
Mn-54	1.96%	6.57E-04
Fe-55	27.88%	9.36E-03
Co-57	0.41%	1.37E-04
Co-58	4.25%	1.43E-03
Co-60	16.04%	5.39E-03
Ni-63	33.99%	1.14E-02
Sr-90	0.03%	8.42E-06
Tc-99	0.01%	4.91E-06
Sb-125	0.44%	1.48E-04
I-129	0%	1.39E-06
Cs-134	1.93%	6.50E-04
Cs-137	9.36%	3.14E-03
Ce-144	0.11%	3.84E-05

Table 3 (cont)

Estimate of major nuclide composition (by waste type)

Waste Stream : Sum of All 4 Categories

Nuclide Name	Percent Abundance	Curies
H-3	0.17%	3.76E-01
Be-7	0.06%	1.31E-01
C-14	0.05%	1.10E-01
Cr-51	0%	4.24E-03
Mn-54	1.68%	3.65E+00
Fe-55	5.07%	1.10E+01
Fe-59	0%	1.46E-03
Co-57	0.94%	2.04E+00
Co-58	2.69%	5.86E+00
Co-60	11.58%	2.52E+01
Ni-59	0.37%	8.12E-01
Ni-63	62.18%	1.35E+02
Zn-65	0.08%	1.83E-01
Sr-89	0%	9.05E-04
Sr-90	0.01%	2.87E-02
Zr-95	0%	1.68E-04
Nb-94	0%	1.77E-05
Nb-95	0%	4.28E-04
Tc-99	0%	5.20E-03
Ag-110m	0%	3.34E-06
Sn-113	0%	3.71E-04
Sb-125	0.24%	5.31E-01
I-129	0%	5.70E-04
Cs-134	4.67%	1.02E+01
Cs-137	10.15%	2.21E+01
Ce-144	0.04%	8.51E-02
Np-239	0%	1.28E-06
Pu-238	0%	1.02E-05
Pu-239	0%	4.96E-09
Pu-240	0%	3.77E-10
Pu-241	0%	1.67E-03
Pu-242	0%	5.62E-10
Am-241	0%	4.90E-05
Cm-242	0%	5.80E-08
Cm-243	0%	8.72E-05
Cm-244	0%	3.83E-06

Table 3 (cont)

Solid Waste Disposition

Number of Shipments	Mode of Transportation	Destination
1	ACME	UniTech Services ORSC
22	Hittman Transport Services	Energy Solutions Bear Creek
3	Hittman Transport Services	Energy Solutions, Gallagher Rd
4	Hittman Transport Services	Energy Solutions-Memphis
5	Hittman Transport Services	Energy Solutions, Gallagher Rd

Irradiated Fuel Shipments (Disposition)

Number of Shipments	Mode of Transportation	Destination
None	N/A	N/A

-....

1

Total

0

Ō

0

463

0 -

0

Table 4 Joint Frequency Distribution of Meteorological Data

JOINT FREQU	JENCY DISTR	IBUTION OF	WIND SPEE	D AND DIRE	CTION IN H	OURS 01/0	1/2016 00:	00:00 TO 1	2/31/2016	23:59:59	PASQU	ILL CLASS	A `
					Wind Spee	d (M/S) at	10-m Leve	1					
Direction	.2250	.5175	.76-1.0	1.1-1.5	1.6-2.0	2.1-3.0	3.1-5.0	5.1-7.0	7.1-10.	10.1-13	13.1-18.0	>18.0	Total
N		0	0	0	1	6	32	14	1	0	0	0	54
NNE	0	0	0	1	0	3	11	4	0	0	0	0	19
NE	0	0	0	0	0	8	49	3	0	0	0	0	60
ENE	0	0	0	0	0	2	4	0	0	0	0	0	6
Е	0	0	0	0	0	1	1	0	0	0	0	0	2
ESE	0	0	0	0	0	2	2	0	0	0	0	0	4
SE	0	0	0	0	0	3	7	6	0	0	0	0	16
SSE	0	0	U	U	0	2	33	16	3	0	0	0	54
S .	U	- 0	0	0	U	3	20	8	1	0	0	U	32
55W	0	0	0	0	0	1	10	5	0	0	U	0	14
5W WSW	0	0	0	0	0	1	10	5	0	0	0	0	10
W	0	0	0	0	0	0	2	1	0	0	0	0	2
WNW	ů 0	n	õ	0	0	3	1	7	3	0	0	0	14
NW	õ	ŏ	ō	õ	ŏ	õ	õ	2	õ	õ	õ	õ	2
NNW	Ō	Û	0	0	0	0	9	5	Ō	0	Ō	õ	14
Total	0	0	0	1	1	35	190	76	8	0		<u> </u>	311
JOINT FREQU	ENCY DISTR	IBUTION OF	WIND SPEE	D AND DIRE	CTION IN HO	OURS 01/0	1/2016 00:	00:00 то 1	2/31/2016	23:59:59	PASQU	ILL CLASS 1	в
Wind					Wind Speed	d (M/S) at	10-m Leve	1					
Direction	.2250	.5175	.76-1.0	1.1-1.5	1.6-2.0	2.1-3.0	3.1-5.0	5.1-7.0	7.1-10.	10.1-13	13.1-18.0	>18.0	Total
N	0 .	0	0	Ō	2	14	11	4	ō	0	<u>0</u> _		31
NNE	0	0	0	0	1	10	18	2	0	0	0	0	31
NE	0	0	0	0	2	14	101	9	0	0	0	0	126
ENE	0	0	0	0	0	1	9	0	0	0	0	0	10
E	0	0	0	0	1	3	2	0	0	0	0	0	6
ESE	0	0	0	0	1	2	3	1	0	0	0	0	7
SE	0	0	0	0	0	3	18	9	0	0	0	0	30
SSE	0	0	0	0	0	6	37	23	1	0	0	0	67
S	0	U	U	0	1	9	15	9	4	1	0	0	39
55W 6W	0	0	0	0	1	4	16	1 7	0	0	0	0	13
WSW	0	0	0	0	2	4	10	3	0	U	0	0	3U 17
W	ñ	ñ	ñ	ů	3	2	2	- 1	0	0	0	ő	- /
MININ	v	ě		0	5	-			0	0	0	~	×
147414	0	0	0	0	0	3	2	ĩ	0	0	0	0	8 6
NW	0	0	0	0 0	0 2	3 2	2 1	ī 1	0	0	0 0	0	8 6 6

275

77

5

1

18

0

87

2016 Waterford 3 Steam Electric Station

PASQUILL CLASS D

Number of calms for B Stability: 0

JOINT FREQU	ENCY DISTRI	IBUTION OF	WIND SPEE	D'AND DIRE	CTION IN H	OURS 01/0	1/2016 00:	00:00 TO 1	2/31/2016	23:59:59	PASQU	ILL CLASS	с
					Wind Spee	d (M/S) at	10-m Leve	l					
Wind Direction	.2250	.5175	.76-1.0	1.1-1.5	1.6-2.0	2.1-3.0	3.1-5.0	5.1-7.0	7.1-10.	10.1-13 13	.1-18.0	>18.0	Total
<u>N</u>	0	0	- 0	0		18	15	2	ö			0	35
NNE	0	0	0	1	4	30	17	2	0	0	0	0	54
NE	0	0	0	0	1	29	98	6	0	0	0	0	134
ENE	0	0	0	0	4	12	9	4	0	0	0	0	29
Е	0	0	0	0	3	3	6	0	0	0	0	0	12
ESE	0	0	0	1	2	8	3	0	1	0	0	0	15
SE	0	0	0	0	2	6	15	5	1	0	0	0	29
SSE	0	0	0	1	3	17	37	11	0	0	0	0	69
S	0	0	0	0	4	10	16	5	4	1	0	0	40
SSW	0	0	0	0	2	6	17	2	0	0	0	0	27
SW	0	0	0	0	4	10	12	1	0	0	0	0	27
WSW	0	1	0	0	4	9	17	4	0	0	0	0	35
W	0	0	0	0	2	7	1	3	0	0	0	0	13
WNW	0	0	0	1	3	13	2	2	0	0	0	0	21
NW	0	0	0	0	3	5	3	1	1	0	0	0	13
NNW	0	0	0	1	6	11	15	2	0	0	0	0	35
Total	0	1	0	5	47	194	283	50	7	1	0	0	588

Number of calms for C Stability: 0

Wind					Wind Spee	d (M/S) at	10-m Leve	1					
Direction	.2250	.5175	.76-1.0	1.1-1.5	1.6-2.0	2.1-3.0	3.1-5.0	5.1-7.0	7.1-10.	10.1-13	13.1-18.0	>18.0	Total
N ·	0	0	2		22	89	115	74	9	0	0 -	<u> </u>	320
NNE	0	0	1	11	42	96	109	35	2	0	0	0	296
NE	0	0	4	12	23	85	222	31	1	0	0	0	378
ENE	0	1	0	7	8	27	53	40	3	0	0	0	139
E	0	0	2	3	3	17	22	16	7	0	0	0	70
ESE	0	0	0	3	2	11	41	36	3	0	0	0	96
SE	0	0	0	1	3	15	85	55	18	0	0	0	177
SSE	0	0	2	3	9	56	189	51	31	0	0	0	341
S	0	0	2	5	23	55	88	58	20	1	0	0	252
SSW	0	1	1	7	19	40	62	21	11	1	0	0	163
SW	0	2	5	7	23	43	43	4	2	0	0	0	129
WSW	0	3	1	13	24	36	40	13	2	0	0	0	132
W	0	0	1	9	16	29	18	8	0	0	0	0	81
WNW	0	2	2	7	17	11	14	10	0	0	0	0	63
NW	0	1	0	11	8	20	24	10	2	0	0	0	76
NNW	0	1	1	4	16	48	71	17	2	0	0	0	160
Total	0	11	24	112	258	678	1196	479	113	2	0	0	2873

JOINT FREQUENCY DISTRIBUTION OF WIND SPEED AND DIRECTION IN HOURS 01/01/2016 00:00:00 TO 12/31/2016 23:59:59

Number of calms for D Stability: 0

Page 32 of 38

-

1

JOINT FREQ	UENCY DISTR	IBUTION OF	WIND SPEE	D AND DIRE	CTION IN H	OURS 01/0	1/2016 00:	00:00 TO 1	2/31/2016	23:59:59	PASQU	ILL CLASS	Е
					Wind Spee	d (M/S) at	10-m Leve	1					
Wind								_					
Direction	.2250	.5175	.76-1.0	1.1-1.5	1.6-2.0	2.1-3.0	3.1-5.0	5.1-7.0	7.1-10.	10.1-13	13.1-18.0	>18.0	Total
<u>N</u>		2	- 3	16	41	117	79	10	<u> </u>	0			269
NNE	0	4	4	17	19	63	74	5	0	0	0	0	186
NE	0	0	3	14	17	76	87	5	0	0	0	0	. 202
ENE	0	0	2	16	11	36	64	15	\ 0	0	0	0	144
Е	0	0	2	8	17	27	26	9	0	0	0	0	89
ESE	0	2	2	4	7	28	65	19	0	0	0	0	127
SE	0	1	1	9	4	64	85	9	1	0	0	0	174
SSE	0	1	4	6	40	184	96	10	0	0	0	0	341
S	1	2	10	35	64	96	47	2	0	0	0	0	257
SSW	1	6	14	45	39	60	27	4	0	0	0	0	196
SW	1	3	10	39	28	42	6	0	0	0	0	. 0	129
WSW	0	5	12	46	22	21	6	0	0	0	0	0	112
W	0	5	8	36	15	22	3	0	0	0	0	0	89
WNW	0	5	13	18	19	11	1	1	0	0	0	0	68
NW	0	5	2	18	21	15	11	1	0	0	0	0	73
NNW	0	1	2	14	19	56	27	2	0	0	0	0	121
Total	3	42	92	341	383	918	704	92	2	0		0	2577
Number of a	calms for E	Stability	: 0										
JOINT FREO	UENCY DISTRI	BUTTON OF	WIND SPEEL	אקדם מאגר ב	CTION IN H	1178 01/0	1/2016 00.1	00-00 TO 1	2/31/2016	23.59.59	- PASOU	TLL CLASS	F
SOINI INBO	onder biblid	DOITON OF	WIND OTHE	5 7110 5110	orion in in	01/01	272010 00.	00.00 10 1	., 51, 2010	20.09.09	111520		
1					Wind Speed	d (M/S) at	10-m Leve	1					
Wind	22 50	E1 7E	76 1 0	1 1 1 5	1620	2120	3150	5 17 0	7 1 .10	10 1 12	12 1 10 0	N10 0	matrix 1
Direction	.2250	.5175	./0-1.0	1.1-1.5	1.0-2.0	2.1-3.0	3.1-5.0	5.147.0	/.1-10.	10.1-13	13.1-18.0	>18.0	IOTAL
<u>n</u>	0 -	4		18	16	21	0	0	0	0	0	0	66

	v		•	10	10		0	0	0	0	v	0	00
NNE ·	3	3	3	10	15	6	0	0	0	0	0	0	40
NE	0	1	2	5	11	17	5	0	0	0	0	0	41
ENE	0	0	3	4	4	1	1	0	0	0	0	0	13
E	0	1	2	3	0	3	1	0	0	0	0	0	10
ESE	0	0	1	2	0	3	3	0	0	0	0	0	9
SE	0	0	3	4	8	11	3	0	0	0	0	0	29
SSE	1	3	3	16	34	62	13	0	0	0	0	0	132
S	1	9	10	54	69	21	·1	0	0	0	0	0	165
SSW	5	16	43	104	37	14	1	1	0	0	0	0	221
SW	3	15	28	38	15	3	1	0	0	0	0	0	103
WSW	2	13	39	50	21	3	0	0	0	0	0	- O	128
W	3	23	34	52	11	1	0	0	0	0	0	0	124
WNW	2	18	15	29	10	0	0	0	0	0	0	0	74
NW	1	11	10	19	9	6	0	0	0	0	0	0	56
NNW	2	4	8	26	13	9	0	0	0	0	0	0	62
Total	23	121	211	434	273	181	29	1	0	0	0	0	1273

Number of calms for F Stability: 0

Page 33 of 38

 \mathbf{N}

JOINT FREQU	ENCY DISTR	IBUTION OF	WIND SPEE	D AND DIRE	CTION IN H	OURS 01/0	1/2016 00:	00:00 TO 1	2/31/2016	23:59:59	PASQ	UILL CLASS	G
111	Wind Speed (M/S) at 10-m Level												
Direction	.2250	.5175	.76-1.0	1.1-1.5	1.6-2.0	2.1-3.0	3.1-5.0	5.1-7.0	7.1-10.	10.1-13	13.1-18.0	>18.0	Total
N	2 -		4	6	2	ō	0		0	0	0		17
NNE	0	2	3	3	1	0	0	0	0	0	0	Ō	9
NE	0	1	1	1	2	1	·0	0	0	Ő	Ō	õ	6
ENE	1	0	2	2	0	0	0	0	o	0	Ō	õ	5
E	1	2	1	1	0	0	0	0	0	0	Ō	õ	5
ESE	0	0	0	1	0	0	0	0	0	Ó	Ō	õ	1
SE	0	0	1	1	1	0	0	0	0	0	Ō	ō	3
SSE	1	3	2	4	8	17	2	0	0	0	Ō	õ	37
S	2	1	5	31	37	2	1	0	0	0	0	ō	79
SSW	4	7	32	63	1.4	1	0	0	0	0	0	Ō	121
SW	6	27	22	22	5	0	0	0	0	0	0	0	82
WSW	8	47	24	13	2	0	0	0	0	0	0	0	94
W	11	42	36	25	0	0	0	0	0	0	0	0	114
WNW	6	27	13	12	2	0	0	0	0	0	0	0	60
NW	2	7	18	12	0	0	0	0	0	0	0	0	39
NNW	1	7	6	8	1	0	0	0	0	0	0	0	23
Total	45	176	170	205	75	21	3 -	0	0	<u>0</u>		0	695

Number of calms for G Stability: 0

.

Total valid hours for all stablities = 8780 Total invalid hours for all stablities = 3

Page 34 of 38

1

Table 5A Doses Due to Gaseous Radioactive Effluents

Doses due to Noble Gases (mRad or mrem)

Age Group : All											
Organ	Qtr 1	Qtr 2	Qtr 3	Qtr 4	Year Total						
Total-body Skin Air Beta Air Gamma	0.00E+00 0.00E+00 0.00E+00 0.00E+00	0.00E+00 0.00E+00 0.00E+00 0.00E+00	0.00E+00 0.00E+00 0.00E+00 0.00E+00	0.00E+00 0.00E+00 0.00E+00 0.00E+00 0.00E+00	0.00E+00 0.00E+00 0.00E+00 0.00E+00						

Doses due to Radioiodines/Particulates/Tritium Excluding C-14 (mrem)

	Age Group : Adult												
	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI	Skin					
Period	mrem	mrem	mrem	mrem	mrem	m mrem mrem		mrem					
Qtr 1	0.00E+00	4.56E-03	4.56E-03	4.56E-03	4.56E-03	4.56E-03	4.56E-03	0.00E+00					
Qtr 2	0.00E+00	4.63E-03	4.63E-03	4.63E-03	4.63E-03	4.63E-03	4.63E-03	0.00E+00					
Qtr 3	0.00E+00	5.21E-03	5.21E-03	5.21E-03	5.21E-03	5.21E-03	5.21E-03	0.00E+00					
Qtr 4	2.30E-05	2.16E-03	2.15E-03	2.14E-03	2.15E-03	2.14E-03	2.14E-03	1.14E-05					
Year	2.30E-05	1.66E-02	1.66E-02	1.65E-02	1.66E-02	1.65E-02	1.65E-02	1.14E-05					

		•	A	ge Group : T	een	·		
	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI	Skin
Period	mrem	mrem	mrem	mrem	mrem	mrem	mrem	mrem
Qtr 1	0.00E+00	5.16E-03	5.16E-03	5.16E-03	5.16E-03	5.16E-03	5.16E-03	0.00E+00
Qtr 2	0.00E+00	5.25E-03	5.25E-03	5.25E-03	5.25E-03	5.25E-03	5.25E-03	0.00E+00
Qtr 3	0.00E+00	5.90E-03	5.90E-03	5.90E-03	5.90E-03	5.90E-03	5.90E-03	0.00E+00
Qtr 4	3.24E-05	2.45E-03	2.43E-03	2.42E-03	2.43E-03	2.43E-03	2.42E-03	1.14E-05
Year	3.24E-05	1.88E-02	1.87E-02	1.87E-02	1.87E-02	1.87E-02	1.87E-02	1.14E-05

_	Age Group : Child												
	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI	Skin					
Period	mrem	mrem	mrem	mrem	mrem	mrem mrem		mrem					
Qtr 1	0.00E+00	7.14E-03	7.14E-03	7.14E-03	7.14E-03	7.14E-03	7.14E-03	0.00E+00					
Qtr 2	0.00E+00	7.26E-03	7.26E-03	7.26E-03	7.26E-03	7.26E-03	7.26E-03	0.00E+00					
Qtr 3	0.00E+00	8.16E-03	8.16E-03	8.16E-03	8.16E-03	8.16E-03	8.16E-03	0.00E+00					
Qtr 4	6.34E-05	3.40E-03	3.36E-03	3.35E-03	3.37E-03	3.36E-03	3.35E-03	1.14E-05					
Year	6.34E-05	2.60E-02	2.59E-02	2.59E-02	2.59E-02	2.59E-02	2.59E-02	1.14E-05					

	Age Group : Infant												
	Bone Liver		T. Body	Thyroid	Kidney	GI-LLI	Skin						
Period	mrem	mrem	mrem	mrem	mrem	mrem	mrem	mrem					
Qtr 1	.0.00E+00	3.23E-03	3.23E-03	3.23E-03	3.23E-03	3.23E-03	3.23E-03	0.00E+00					
Qtr 2	0.00E+00	3.28E-03	3.28E-03	3.28E-03	3.28E-03	3.28E-03	3.28E-03	0.00E+00					
Qtr 3	0.00E+00	3.69E-03	3.69E-03	3.69E-03	3.69E-03	3.69E-03	3.69E-03	0.00E+00					
Qtr 4	5.88E-05	1.58E-03	1.52E-03	1.52E-03	1.53E-03	1.53E-03	1.52E-03	1.14E-05					
Year	5.88E-05	1.18E-02	1.17E-02	1.17E-02	1.17E-02	1.17E-02	1.17E-02	1.14E-05					

.

 Table 5A (continued)

 Critical Dose due to Radioiodines/Particulates/Tritium Including C-14 (mrem)

	Age Group : Child											
Organ	Qtr 1	Qtr 2	Qtr 3	Qtr 4	Year Total							
Bone Total-body	8.63E-01 1.73E-01	8.63E-01 1.73E-01	8.63E-01 1.73E-01	8.63E-01 1.73E-01	3.45E+00 6.90E-01							

Table 5B **Doses Due to Liquid Radioactive Effluents**

Cumulative Dose Information for 2016 (mrem)

	Parameter	Max Receptor	Dose	Dose Limit	% of Limit
Qtr 1	Max Organ Dose (mrem)	Adult - Gi-LLi	3.28E-05	5.00E+00	0.00
	Total Body Dose (mrem)	Adult - Total Body	2.07E-05	1.50E+00	0.00
Qtr 2	Max Organ Dose (mrem)	Adult - Gi-LLi	1.45E-05	5.00E+00	0.00
	Total Body Dose (mrem)	Adult - Total Body	9.19E-06	1.50E+00	0.00
Qtr 3	Max Organ Dose (mrem)	Adult - Gi-LLi	6.08E-05	5.00E+00	0.00
	Total Body Dose (mrem)	Adult - Total Body	3.66E-05	1.50E+00	0.00
Qtr 4	Max Organ Dose (mrem)	Adult - Liver	1.59E-04	5.00E+00	0.00
	Total Body Dose (mrem)	Adult - Total Body	1.49E-04	1.50E+00	0.01
Year	Max Organ Dose (mrem)	Adult - Gi-LLi	2.65E-04	1.00E+01	0.00
<u> </u>	Total Body Dose (mrem)	Adult - Total Body	2.16E-04	<u>3.00E+00</u>	0.01

2

2016 Waterford 3 Steam Electric Station

	Table 6			
2016	Groundwater Monitoring	Well	Results	(pCi/L)

Sample Date											Cs-			
Time	Well	Tritium	Mn-54	Co-58	Fe-59	Co-60	Zn-65	Nb-95	Zr-95	I-131	134	Cs-137	Ba-140	La-140
3/8/2016 11:55	MW-3	< 466.00	< 7.01	< 6.78	< 15.90	< 8.57	< 15.90	< 6.78	< 14.90	< 9.78	< 5.98	< 7.92	< 31.00	< 7.45
6/23/2016 11:00	MW-3	< 679.00	< 7.35	< 4.40	< 12.80	< 7.06	< 14.20	< 7.76	< 14.50	< 6.39	< 5.86	< 6.67	< 19.30	< 9.24
9/21/2016 13:33	MW-3	< 675.00	< 5.03	< 2.92	< 12.30	< 3.41	< 9.60	< 5.63	< 9.18	< 9.79	< 5.16	< 5.80	< 27.20	< 8.58
12/20/2016 15:25	MW-3	< 616.00	< 7.19	< 7.86	< 16.10	< 8,00	< 9.64	< 8.84	< 14.40	< 12.40	< 7.13	< 7.82	< 35.10	< 12.20
12/20/2016 16:10	MW-3 DUP	< 620.00	< 5.82	< 6.22	< 13.00	< 6.21	< 11.00	< 7.07	< 13.10	< 11.60	< 5.85	< 5.99	< 32.60	<10.8
				-		_		-	<u> </u>					
3/8/2016 10:35	MW-4	< 470.00	< 7.69	< 5.76	< 12.40	< 7.45	< 13.00	< 7.29	< 12.20	< 11.80	< 6.78	< 6.87	< 31.00	< 14.30
6/23/2016 12:40	MW-4	< 678.00	< 6.67	< 11.00	< 13.10	< 14.40	< 16.70	< 9.43	< 13.90	< 12.00	< 7.57	< 6.60	< 28.50	<10.9
9/21/2016 10:43	MW-4	< 663.00	< 7.64	< 8.06	< 14.50	< 7.68	< 15.90	< 7.69	< 13.30	< 15.00	< 8.36	< 8.72	<40.6	< 11.40
12/20/2016 14:00	MW-4	<609	< 5.44	< 5.29	< 11.20	< 5.84	< 11.50	< 5.21	<10.4	<10.4	< 5.07	< 6.18	< 28.10	< 9.66
														_
3/8/2016 9:28	MW-5	< 470.00	< 7.76	< 6.32	< 13.40	< 9.50	< 18.90	< 9.36	< 14.10	< 9.38	< 6.95	< 8.25	< 27.40	< 12.50
6/23/2016 15:00	MW-5	< 670.00	< 7.82	< 6.56	< 15.60	< 8.45	< 11.50	< 7.12	< 9.87	< 13.20	< 5.82	< 6.48	< 31.30	< 11.90
9/21/2016 7:52	MW-5	< 664.00	< 6.95	< 5.57	< 16.30	< 5.36	< 13.50	< 6.45	< 9.17	<10.6	< 5.87	< 6.70	< 23.00	<10.4
12/20/2016 10:05	MW-5	<605	< 6.23	< 6.03	< 8.55	< 8.01	< 12.60	< 7.50	<10.3	<10.5	< 4.73	< 5.43	< 27.90	< 9.03
											_			
3/8/2016 7:45	MW-6	< 465.00	< 6.72	< 8.07	< 15.10	< 6.64	< 18.20	< 7.96	< 11.00	< 8.47	< 7.48	< 8.57	< 28.20	< 6.05
3/8/2016 8:25	MW-6 DUP	< 458.00	< 5.79	< 5.71	< 15.40	< 7.14	< 11.80	< 7.12	< 13.50	<10.5	< 7.43	< 7.80	< 32.50	< 9.10
6/27/2016 11:25	MW-6	< 684.00	< 1.37	< 1.52	< 3.56	< 1.34	< 2.73	< 1.61	< 2.84	< 8.37	< 1.28	< 1.39	< 14.10	< 4.81
9/21/2016 12:02	MW-6	< 653.00	< 6.49	< 7.40	< 14.70	< 7.08	< 16.90	< 9.02	< 13.50	< 11.70	< 6.64	< 7.64	< 36.70	< 13.30
12/20/2016 8:40	MW-6	< 621.00	< 5.95	< 5.35	< 11.20	< 8.27	< 14.80	< 7.22	<10.7	< 11.90	< 6.12	< 9.30	< 26.90	< 11.30
3/7/2016 17:15	MW-7	< 462.00	< 7.96	< 7.22	< 15.00	< 7.20	< 17.50	< 7.32	< 13.30	< 12.80	< 7.95	< 7.09	<30.3	< 12.50
6/27/2016 9:37	MW-7	< 685.00	< 6.61	< 5.77	< 21.70	< 4.85	< 1.88	< 4.54	< 13.90	< 6.63	< 3.68	<10.1	< 21.50	< 13.60
6/27/2016 10:20	MW-7	< 678.00	< 8.01	< 8.26	< 16.60	< 9.20	< 17.10	< 8.01	< 14.40	< 11.70	<10.4	< 9.56	< 37.60	< 9.79
9/21/2016 15:53	MW-7	< 450.00	< 4.90	< 4.94	<10.1	< 5.46	<10.5	< 5.20	< 8.35	< 9.09	< 5.25	< 5.99	< 29.90	<10.2
12/19/2016 13:40	MW-7	< 613.00	< 6.58	< 5.81	< 16.40	< 5.73	< 14.50	< 7.80	< 13.90	< 13.60	< 5.82	< 8.16	< 38.40	< 7.64

7

.

-

.

									~					
3/7/2016 16:06	MW-8	< 458.00	< 5.64	< 7.80	< 12.30	< 7.36	< 14.20	< 7.76	< 12.90	< 11.20	< 6.53	< 8.73	< 28,20	< 8.81
6/27/2016 12:30	MW-8	< 683.00	< 1.93	< 2.08	< 4.90	< 1.80	< 3.94	< 2.27	< 3.82	< 11.30	< 1.72	< 1.91	<20.3	< 6.69
9/21/2016 14:30	MW-8	< 651.00	< 7.35	< 6.48	< 11.20	< 5.97	< 13.80	< 8.97	<10.8	<10.7	< 6.46	< 6.38	< 33.20	< 8.92
12/19/2016 15:53	MW-8	< 623.00	< 6.94	< 7.89	< 13.90	< 5.84	< 13.00	< 7.64	< 12.30	< 11.10	< 6.90	< 6.39	< 33.70	< 8.07
3/7/2016 15:00	MW-9	< 468.00	< 7.78	< 4.53	< 13.10	< 5.76	< 18.40	< 8.75	< 12.50	<10.1	< 7.17	< 8.40	< 36.40	< 7.62
6/27/2016 13:35	MW-9	< 672.00	< 6.95	< 5.64	< 12.20	< 8.47	< 16.20	< 6.39	< 13.40	<10.1	< 7.34	< 7.26	< 33.40	< 9.07
9/20/2016 17:50	MW-9	< 659.00	< 6.87	< 4.35	< 13.20	< 7.01	< 13.50	< 6.27	< 11.50	<10.1	< 4.12	< 6.82	< 28.80	<10.4
12/19/2016 15:06	MW-9	< 619.00	< 7.33	< 9.17	< 12.50	< 8.60	< 14.30	< 8.43	< 14.90	< 14.40	< 9.45	< 7.70	< 47.90	<10.1
			·						-				•	
3/8/2016 13:00	MW-10	< 469.00	< 5.68	< 6.20	< 13.10	< 5.59	<10.7	< 5.91	<10.1	< 8.80	< 5.50	< 6.08	< 26.00	< 7.84
6/23/2016 16:20	MW-10	< 677.00	< 5.73	< 4.83	<10.4	< 3.70	< 12.30	< 7.66	<10.9	< 11.70	< 7.59	< 6.57	< 36.50	<10
9/21/2016 8:53	MW-10	< 652.00	< 7.66	< 8.29	<20.7	<10.5	< 16.40	< 9.58	< 14.50	< 13.50	< 8.62	< 8.84	< 42.00	< 12.80
9/21/2016 9:33	<u>M</u> W-10	< 658.00	< 5.60	< 6.33	< 16.00	< 7.85	< 14.80	< 8.51	< 11.40	< 13.20	< 5.77	< 8.47	< 31.10	< 7.38
12/20/2016 11:15	MW-10	< 618.00	< 7.02	< 7.15	< 15.30	< 6.06	< 13.10	< 7.17	< 13.00	< 13.70	< 6.42	< 7.06	< 29.50	< 11.30
3/8/2016 14:07	MW-11	< 468	< 7.43	< 7.57	< 14.40	< 8.06	< 14.50	< 8.83	< 12.60	< 11.60	< 6.30	. < 8.43	< 37.00	< 11.40
6/22/2016 15:07	MW-11	< 535	< 6.26	< 7.66	< 8.35	< 6.72	< 15.20	< 9.21	< 11.80	< 11.80	< 8.44	< 8.48	<40.8	< 9.04
9/20/2016 15:30	MW-11	< 665	< 5.71	< 6.06	< 9.90	< 3.65	< 14.00	< 5.77	< 7.99	<10.6	< 4.96	< 4.97	< 24.00	< 7.66
12/20/2016 12:35	MW-11	<605	< 6.68	< 7.06	< 8.66	< 6.04	< 14.00	< 7.85	< 11.10	< 11.90	< 6.91	< 7.24	< 34.00	< 14.10
														_
3/7/2016 13:32	MW-12	< 459.00	< 6.80	< 4.89	< 11.40	< 7.77	< 9.75	< 7.34	<10.3	< 9.43	< 6.17	< 6.64	< 21.60	< 5.29
6/22/2016 13:25	MW-12	< 540.00	< 6.85	< 7.40	< 12.40	< 5.83	< 14.30	< 7.37	< 14.70	<10.7	< 7.94	< 7.78	<30.3	<10.6
9/20/2016 13:27	MW-12	< 662.00	< 5.98	< 6.01	< 13.70	< 5.81	<10.2	< 7.59	< 11.70	< 12.60	< 6.65	< 6.20	< 39.30	< 12.70
12/19/2016 12:06	MW-12	< 613.00	< 6.42	< 6.86	< 14.70	< 8.09	< 15.20	< 6.61	< 12.90	< 14.60	< 6.60	< 7.02	< 35.00	< 12.80