




Information Digest

2015-2016



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U.S. Nuclear Regulatory Commission
Office of Public Affairs
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www.nrc.gov

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Section Photo Captions:

Section 1: NRC: An Independent Regulatory Agency

1. *The Prairie Island nuclear power plant is located in Minnesota.*
2. *The NRC Headquarters complex is located in Rockville, MD.*
3. *An NRC inspector conducts routine inspections of plant equipment to ensure the plant is meeting NRC regulations.*

Section 2: Nuclear Energy in the United States and Worldwide

1. *A satellite photograph captures the sunrise over the earth.*
2. *The United Nations General Assembly meets in New York to discuss, among other topics, world nuclear matters.*
3. *NRC Chairman Stephen Burns (center of the table on the right) meets with Japan Chief Cabinet Secretary Yoshihide Suga (center of the table on the left) to discuss nuclear regulatory issues.*

Section 3: Nuclear Reactors

1. *The St. Lucie nuclear power plant is located in Florida.*
2. *An NRC inspector conducts routine inspections of plant equipment to ensure the plant is meeting NRC regulations.*
3. *Transmission lines distribute electricity generated by nuclear power plants to the power grid.*

Section 4: Nuclear Materials

1. *Yttrium-90 (Y-90) are microspheres used to treat liver cancers.*
2. *A moisture density gauge indicates whether a foundation is suitable for constructing a building or roadway.*
3. *A worker displays a small ceramic fuel pellet.*

Section 5: Radioactive Waste

1. *Dry casks are transported to a storage site.*
2. *As part of the decommissioning process, the cooling tower of a nuclear power plant is imploded.*
3. *Dry cask storage of spent nuclear fuel is sometimes located aboveground.*

Section 6: Security and Emergency Preparedness

1. *Biometric access control locks within a nuclear facility provide another layer of protection.*
2. *Barbed wire provides an added layer of security while protecting a nuclear facility from intruders.*
3. *Security officers protect nuclear facilities from intruders.*

Sections 7, 8, and 9:

1. *NRC Historian Thomas Wellock examines past historical events and the NRC actions.*
2. *Social media enables the NRC to reach a broader audience and to better engage, inform and educate the public—in real-time, through multiple channels.*
3. *NRC regulations are contained in Title 10 of the Code of Federal Regulations, Volume I, Parts 1 to 50.*

Abstract

The U.S. Nuclear Regulatory Commission (NRC) 2015–2016 Information Digest provides information about the agency and the industries it regulates. It describes the agency’s responsibilities and activities and provides general information on nuclear-related topics. The Information Digest includes NRC and industry data in an easy-to-read format. Information graphics help explain the information. The non-NRC data is updated every 2 years. The NRC reviews the information from industry and international sources but does not independently verify it. The Web Link Index provides sources for more information on major topics. This edition includes any adjustments to preliminary figures from the previous edition. The NRC is the source of all photographs, graphics, and tables unless otherwise noted.

All information is final unless otherwise noted. Corrections and updates will appear in the digital version of the publication on the NRC Web site. Comments or suggestions on the Information Digest are welcomed. To submit comments, write to the Office of Public Affairs at U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or at OPA.Resource@nrc.gov.

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NRC Resident Inspectors perform routine inspection activities to ensure the plant is meeting NRC regulations.

Mission

The U.S. Nuclear Regulatory Commission (NRC) is an independent agency created by Congress. Its mission is to license and regulate the civilian use of radioactive materials in the United States to protect public health and safety, promote the common defense and security, and protect the environment.

The NRC regulates commercial nuclear power plants; research, test, and training reactors; nuclear fuel cycle facilities; and radioactive materials used in medicine, academia, and industry. The agency also regulates the transport, storage, and disposal of radioactive materials and waste, most Federal agencies' use and possession of radioactive materials, and licenses the export and import of radioactive materials.

Commission

Chairman Stephen G. Burns	Term ends June 30, 2019
Commissioner Kristine L. Svinicki	Term ends June 30, 2017
Commissioner William C. Ostendorff	Term ends June 30, 2016
Commissioner Jeff Baran	Term ends June 30, 2018

Locations

Headquarters:

U.S. Nuclear Regulatory Commission
Rockville, MD, 301-415-7000, 1-800-368-5642

Regional Offices:

<i>Region I</i>	<i>Region II</i>	<i>Region III</i>	<i>Region IV</i>
King of Prussia, PA 610-337-5000 1-800-432-1156	Atlanta, GA 404-997-4000 1-800-577-8510	Lisle, IL 630-829-9500 1-800-522-3025	Arlington, TX 817-860-8100 1-800-952-9677

Headquarters Operations Center:

Rockville, MD 301-816-5100
The NRC maintains a staffed, 24-hour Operations Center that coordinates incident response with State, local, and Federal agencies.

Training and Professional Development:

Technical Training Center, Chattanooga, TN 423-855-6500
Professional Development Center, Rockville, MD 301-492-2000

Resident Sites:

At least two NRC resident inspectors, who report to the appropriate regional office, are located at each nuclear power plant site.

NRC Fiscal Year 2015 Budget

- Total authority: \$1,015.3 million
- Total authorized staff: 3,779
- Budget amount expected to be recovered by annual fees to licensees: \$895.5 million
- NRC research funding: \$55.2 million

What Does the NRC Do?

- Regulation and guidance—rulemaking
- Policymaking
- Licensing, decommissioning, and certification
- Research
- Oversight and enforcement
- Emergency preparedness and response
- Incident response

NRC Governing Legislation

The NRC was established by the Energy Reorganization Act of 1974. The most significant laws that govern the regulatory process of the agency are in Appendix Z. NRC's regulations are found in Title 10 of the *Code of Federal Regulations*. The text of many laws may be found in NUREG-0980, "Nuclear Regulatory Legislation."

NRC by the Numbers

Nuclear Reactors

- 99 commercial nuclear power plants operating in 30 States at 61 sites
 - 65 pressurized-water reactors and 34 boiling-water reactors
- Four reactor fuel vendors
- 23 parent operating companies
- About 80 different designs
- About 6,300 total inspection hours at each operating reactor site in 2014

Reactor License Renewal

Commercial power reactor operating licenses are valid for 40 years and may be renewed for an additional 20 years.

- 25 reactors operate under their original license
- 43 sites comprised of 74 reactors were issued renewal licenses
- 12 sites have license renewal applications in review
- Six sites have submitted letters of intent to request renewal

Early Site Permit for New Reactors

- Four early site permits (ESPs) issued and one application in review

Combined License—Construction and Operating for New Reactors

18 combined license (COL) applications received and docketed for 28 reactors; of these, three applications approved for a total of five reactors, six applications under review for a total of 10 reactors, seven applications suspended for a total of 10 reactors, and two applications withdrawn for a total of three reactors.

Reactor Design Certification

- Four reactor design certifications (DC) issued, one in review, and one in rulemaking

Nuclear Research and Test Reactors

- 31 licensed research and test reactors operating in 21 States

Nuclear Materials

- The NRC and the Agreement States have approximately 20,800 licensees for medical, academic, industrial, and general users of nuclear materials
- The NRC oversees approximately 2,800 licenses
- 37 Agreement States oversee approximately 18,000 licenses

21 Uranium Recovery Sites Licensed by the NRC

- Nine in situ recovery sites
- 12 conventional mills (11 of these are undergoing decommissioning)

13 Fuel Cycle Facilities

- One uranium hexafluoride production facility
- Five uranium fuel fabrication facilities
- Four gas centrifuge uranium enrichment facilities (one operating, one test and development, and two construction pending)
- One mixed-oxide fuel fabrication facility (under construction and review)
- One laser separation enrichment facility (construction decision pending)
- One uranium hexafluoride deconversion facility (construction decision pending)

Radioactive Waste

Low-Level Radioactive Waste

- 10 regional compacts
- Four licensed disposal facilities

High-Level Radioactive Waste Management

Spent Nuclear Fuel Storage

- 74 licensed and/or operating independent spent fuel storage installations in 34 States
- 15 site-specific licenses
- 59 general licenses

Transportation—Principal Licensing and Inspection Activities

- 1,000 safety inspections of fuel, reactor, and materials licensees conducted annually
- 65 new, renewal, or amended container-design applications for the transport of nuclear materials reviewed annually
- 150 license applications for the import and export of nuclear materials from the United States reviewed annually

Decommissioning

Approximately 150 materials licenses are terminated each year. The NRC's decommissioning program focuses on the termination of licenses that are not routine and that require complex activities.

- 19 nuclear power reactors in various stages of decommissioning (DECON or SAFSTOR)
- Five research and test reactors permanently shut down and in various stages of decommissioning
- 15 complex material sites
- Two fuel cycle facilities (partial decommissioning)
- 11 uranium recovery facilities under NRC jurisdiction

Security and Emergency Preparedness

- Once every 2 years, each nuclear power plant performs a full-scale emergency preparedness exercise
- Plants conduct additional emergency drills between full-scale exercises. These emergency exercises and drills are inspected by the NRC and evaluated by the Federal Emergency Management Agency
- Once every 3 years, each nuclear plant undergoes a force-on-force security inspection. These inspections include mock combat drills

2014 Accomplishment Highlights

Nuclear Reactors

- made significant progress implementing lessons learned from the Fukushima accident in Japan, with a focus on the highest priority (Tier 1) activities
- completed over 670 licensing actions and other licensing tasks, including approving additional transitions to the National Fire Protection Association, NFPA 805, performance-based standard for fire protection, while also actively reviewing a number of nuclear power plant license renewal applications
- issued the design certification for the Economic Simplified Boiling-Water Reactor
- continued active review of seven combined license applications for a total of 11 new reactor units and oversight of construction at new reactor construction sites
- completed all required inspection and assessment activities of the Reactor Oversight Process, including initiating seven inspections in response to safety-significant events
- collaborated with States, Federal agencies, and licensees in responding to natural events and extreme weather, such as tornadoes, floods, hurricanes, and earthquakes, affecting licensed facilities
- continued implementing the rulemaking initiative for emergency preparedness
- participated in 18 new international agreements, adding to the existing 100 active international agreements for cooperative research
- published extensive research results on a variety of topics that confirmed the safety of operating facilities

Materials and Waste

- published the final rule and generic environmental impact statement on continued storage of spent nuclear fuel (previously referred to as “waste confidence rule”)
- issued two new uranium recovery facility operating licenses
- worked with other Federal agencies (DOE, DOS, EPA) to jointly completed the fifth report updating the U.S. National Report prepared under the Joint Convention on the Safety of Spent Fuel Management and on the Safety of Radioactive Waste Management
- delivered the third quadrennial report of the Radiation Source Protection and Security Task Force to President Obama and to Congress, outlining the Federal Government’s efforts over the past 4 years to enhance the security of radioactive sources
- between October 2014 and January 2015, released the final four volumes of the Yucca Mountain safety evaluation report, completing the technical safety review of the U.S. Department of Energy’s application
- reorganized NRC materials and waste programs by merging the Office of Federal and State Materials and Environmental Management Programs (FSME) and the Office of Nuclear Material Safety and Safeguards (NMSS), with the new office retaining the name of NMSS, an office established by Congress when it created the NRC in 1974
- enhanced coordination with the States through the Integrated Materials Performance Evaluation Program and the State liaison officers

Agencywide

- made substantial progress on rulemakings ranging from the standards for protection against radiation to the control and accounting of special nuclear material (see Appendix Z for specific NRC regulations)
- published a draft policy statement that establishes principles to guide the NRC’s cooperation with Native American tribes

Administration

- responded to 54 Freedom of Information Act requests regarding the Fukushima Dai-ichi accident in Japan and released the last of 258,796 pages to the public
- closed over 400 other Freedom of Information Act requests
- issued a new strategic plan covering Fiscal Year (FY) 2014 through FY 2018, which provides a blueprint for the agency to plan, implement, and monitor the work needed to achieve the NRC's mission for the next 4 years
- issued 83 escalated enforcement actions, 12 actions involving civil penalties, and 58 escalated notices of violation without a proposed civil penalty
- continued to conduct agency outreach to audiences interested in NRC activities, including through the use of social media
- awarded 49 grants to 37 higher education institutions, including 25 minority-serving institutions located in 23 states and Puerto Rico
- held the 27th annual Regulatory Information Conference with nearly 3,000 participants representing 34 countries
- combined staff into One White Flint North, Two White Flint North, and part of Three White Flint North to make significant progress toward the goal of a consolidated Headquarters staff to reduce space utilization and the Federal footprint

Public Meetings and Involvement

- the NRC hosted both the Regulatory Information Conference and the Fuel Cycle Information Exchange, where thousands of participants from around the world discussed the latest technical issues
- conducted approximately 1,000 public meetings in the Washington, DC, area and around the country addressing a full range of NRC issues
- the Advisory Committee on Reactor Safeguards held 10 full committee meetings and 61 subcommittee meetings in calendar year (CY) 2014
- the Advisory Committee on Medical Uses of Isotopes held four public meetings in CY 2014

News and Information

- the NRC news releases are available through a free listserv subscription at www.nrc.gov/public-involve/listserver.html
- the NRC uses social media as a communication tool to allow the public to stay connected through the NRC Blog, Twitter, Flickr, YouTube, and Facebook

For more information on the agency's accomplishments, find the reports to Congress on the NRC Web site at: <http://www.nrc.gov/reading-rm/doc-collections/congress-docs/monthly-reports>

Contact Us

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e-mail: opa.resource@nrc.gov

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TDD: 1-800-635-4512

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General Counsel Intern Program, Honor Law Graduate Programs,
or 2-Year Judicial Clerkship Program 301-415-1515

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Small Business 1-800-903-7227

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Washington, DC 20555-0001

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11555 Rockville Pike, Rockville, MD 20852

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Twitter



<https://twitter.com/#/nrcgov>

GovDelivery



<http://www.nrc.gov/public-involve/listserver.html#gov>

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<http://www.flickr.com/photos/nrcgov>

Facebook



<http://www.facebook.com>

RSS



<http://www.nrc.gov/public-involve/listserver.html#rss>

Report a Concern

Emergency

Involving a nuclear facility or radioactive materials, including:

- any accident involving a nuclear reactor, nuclear fuel facility, or radioactive materials
- lost or damaged radioactive materials
- any threat, theft, smuggling, vandalism, or terrorist activity involving a nuclear facility or radioactive materials

**Call the NRC's 24-Hour
Headquarters Operations Center:
301-816-5100**

We accept collect calls. We record all calls to this number.

Non-Emergency

This includes any concern involving a nuclear reactor, nuclear fuel facility, or radioactive materials.

You may send an e-mail to allegations@nrc.gov. However, because e-mail transmission may not be completely secure, if you are concerned about protecting your identity, it is preferable that you contact us by telephone or in person. You may contact any NRC employee (including a resident inspector) or call:

**NRC's Toll-Free Safety Hotline:
800-695-7403**

Calls to this number are not recorded between the hours of 7:00 a.m. and 5:00 p.m. EST. However, calls received outside these hours are answered by the Incident Response Operations Center on a recorded line.

Some materials and activities are regulated by Agreement States, and concerns should be directed to the appropriate State Radiation Control Program at <http://nrc-stp.ornl.gov/asdirectory.html>

The NRC's Office of the Inspector General

The Office of the Inspector General (OIG) at the NRC established the OIG Hotline to provide NRC employees, other government employees, licensee and utility employees, contractor employees, and the public with a confidential means of reporting incidences of suspicious activity to OIG concerning fraud, waste, abuse, and employee or management misconduct. Mismanagement of agency programs or danger to public health and safety may also be reported through the Hotline.

It is not OIG policy to attempt to identify people contacting the OIG Hotline. People may contact OIG by telephone, through an online form, or by mail. There is no caller identification feature associated with the Hotline or any other telephone line in the Inspector General's office. No identifying information is captured when you submit an online form. You may provide your name, address, or telephone number, if you wish.

**Call the OIG Hotline:
1-800-233-3497, TDD: 1-800-270-2787
7:00 a.m.–4:00 p.m. (EST)
After hours, please leave a message.**





An
Independent
Regulatory
Agency

Mission

The U.S. Nuclear Regulatory Commission (NRC) is an independent agency created by Congress. Its mission is to license and regulate the civilian use of radioactive materials in the United States to protect public health and safety, promote the common defense and security, and protect the environment.

The NRC regulates commercial nuclear power plants; research, test, and training reactors; nuclear fuel cycle facilities; and radioactive materials used in medicine, academia, and industry. The agency also regulates the transport, storage, and disposal of radioactive materials and waste; most Federal agencies' use and possession of radioactive materials; and licenses the export and import of radioactive materials. The NRC regulates industries within the United States and works with agencies around the world to enhance global nuclear safety and security. To fulfill its responsibilities, the NRC performs five principal regulatory functions, as seen in Figure 1: How We Regulate.

Vision and Values

A trusted, independent, transparent, and effective nuclear regulator

To be successful, the NRC must not only excel in carrying out its mission but must do so in a manner that engenders the trust of the public and stakeholders. This vision is an outgrowth of the NRC operating in a manner consistent with its longstanding Principles of Good Regulation—*independence, openness, efficiency, clarity, and reliability*—and its organizational values.

These principles guide the agency—from how the NRC reaches decisions on safety, security, and environmental issues; to how the NRC performs administrative tasks; to how its employees interact with fellow employees and other stakeholders. By adhering to these principles and values, the NRC maintains its regulatory competence, conveys that competence to the stakeholders, and promotes trust in the agency. The agency puts these principles into practice with effective, realistic, and timely actions.

NRC Organizational Values

Integrity in our working relationships, practices, and decisions

Service to the public and others who are affected by our work

Openness in communications and decisionmaking

Commitment to public health and safety, security, and the environment

Cooperation in the planning, management, and performance of agency work

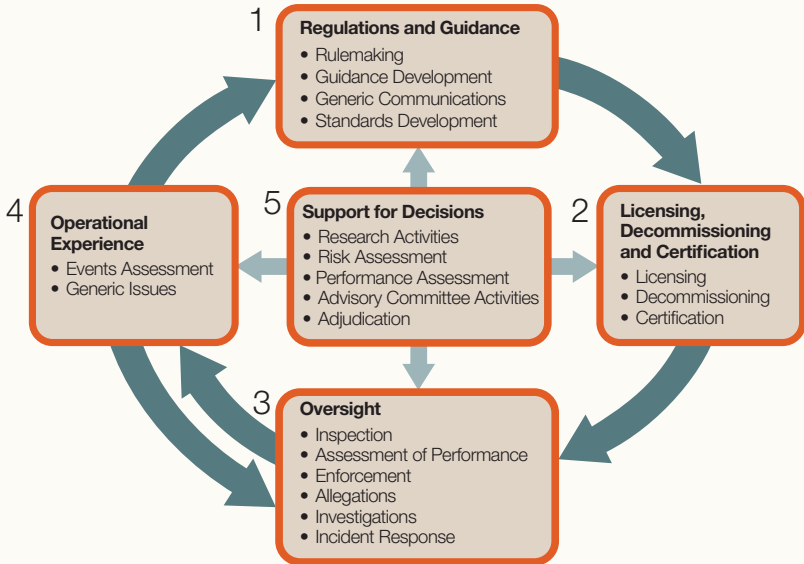
Excellence in our individual and collective actions

Respect for individuals' diversity, roles, beliefs, viewpoints, and work/life balance



NRC staff members meet with stakeholders to discuss the agency's regulatory issues.

Figure 1. How We Regulate



1. Developing regulations and guidance for applicants and licensees.
2. Licensing or certifying applicants to use nuclear materials, operate nuclear facilities, and decommission facilities.
3. Inspecting and assessing licensee operations and facilities to ensure licensees comply with NRC requirements, responding to incidents, investigating allegations of wrongdoing, and taking appropriate followup or enforcement actions when necessary.
4. Evaluating operational experience of licensed facilities and activities.
5. Conducting research, holding hearings, and obtaining independent reviews to support regulatory decisions.

Strategic Goals

Safety: Ensure the safe use of radioactive materials.

Security: Ensure the secure use of radioactive materials.

Statutory Authority

The Energy Reorganization Act of 1974 created the NRC from a portion of the former Atomic Energy Commission. The new agency was to independently oversee—but not promote—the commercial nuclear industry so the United States could benefit from the use of radioactive materials while also protecting people and the environment. The agency began operations on January 18, 1975. The NRC's regulations can be found in Title 10, "Energy," of the *Code of Federal Regulations* (10 CFR). The principal statutory authorities that govern the NRC's work can be found on the NRC's Web site (see the Web Link Index for more information).

See the complete list of NRC's authorizing legislation in Appendix Z.

The NRC, its licensees (those licensed by the NRC to use radioactive materials), and the Agreement States (States that assume regulatory authority over certain nuclear materials) share responsibility for protecting public health and safety and the environment. Federal regulations and the NRC's regulatory program play a key role. Ultimately, however, the licensees bear the primary responsibility for safely handling and using radioactive materials.

Major Activities

The NRC fulfills its responsibilities by:

- licensing the design, construction, operation, and decommissioning of commercial nuclear power plants and other nuclear facilities
- licensing the possession, use, processing, handling, exporting, and importing of nuclear materials
- licensing the siting, design, construction, operation, and closure of low-level radioactive waste (LLW) disposal sites in States under NRC jurisdiction
- certifying the design, construction, and operation of commercial transportation casks
- licensing the design, construction, and operation of spent fuel storage casks and interim storage facilities for spent fuel and high-level waste

- licensing nuclear reactor operators
- licensing uranium enrichment facilities
- conducting research to develop regulations and anticipate potential reactor and other nuclear facility safety issues
- collecting, analyzing, and disseminating information about the safe operation of commercial nuclear power reactors and certain nonreactor activities
- issuing safety and security regulations, policies, goals, and orders that govern nuclear activities
- interacting with other Federal agencies, foreign governments, and international organizations on safety and security issues
- investigating nuclear incidents and allegations concerning any matter regulated by the NRC
- inspecting NRC licensees to ensure adequate performance of safety and security programs
- enforcing NRC regulations and the conditions of NRC licenses and imposing, when necessary, civil sanctions and penalties
- conducting public hearings on nuclear and radiological safety and security and on environmental concerns
- implementing international legal commitments made by the U.S. Government in treaties and conventions
- developing effective working relationships with State and Tribal governments
- maintaining an effective incident response program and overseeing required emergency response activities at NRC-licensed facilities
- implementing lessons learned from the March 2011 nuclear accident in Japan to enhance safety at U.S. commercial nuclear facilities
- involving the public in the regulatory process through meetings, conferences, and workshops; providing opportunities for commenting on proposed new regulations, petitions, guidance documents, and technical reports; providing ways to report safety concerns; and providing documents under the Freedom of Information Act and through the NRC's Web site
- engaging and informing the public through social media platforms and by providing interactive, high-value data sets (data in a form that allows members of the public to search, filter, or repackage information)

Organizations and Functions

The NRC's Commission has five members nominated by the President and confirmed by the U.S. Senate for 5-year terms. The members' terms are staggered so one Commissioner's term expires on June 30 of each year. The President designates one member to serve as Chairman. The Chairman is the principal executive officer and spokesperson of the agency. No more than three Commissioners can belong to the same political party. The Commission as a whole formulates policies and regulations governing the safety and security of nuclear reactors and materials, issues orders to licensees, and adjudicates legal matters brought before it. The Executive Director for Operations carries out the policies and decisions of the Commission and directs the activities of the program and regional offices (see Figure 2: NRC Organizational Chart and Figure 3: NRC Regions).



During a public meeting, NRC staff members brief the Commissioners on updates to the cumulative effects of regulation and risk prioritization initiatives.

Commissioner Term Expiration*

Commissioner	Expiration of Term
<i>Stephen G. Burns, Chairman</i>	<i>June 30, 2019</i>
<i>Kristine L. Svinicki</i>	<i>June 30, 2017</i>
<i>William C. Ostendorff</i>	<i>June 30, 2016</i>
<i>Jeff Baran</i>	<i>June 30, 2018</i>
<i>Vacant</i>	

* Commissioners listed by seniority

The NRC is headquartered in Rockville, MD, and has four regional offices. They are located in King of Prussia, PA; Atlanta, GA; Lisle, IL; and Arlington, TX. The major program offices within the NRC include:

The **Office of Nuclear Reactor Regulation** handles all licensing and inspection activities for existing nuclear power reactors and research and test reactors.

The **Office of New Reactors** oversees the design, siting, licensing, and construction of new commercial nuclear power reactors.

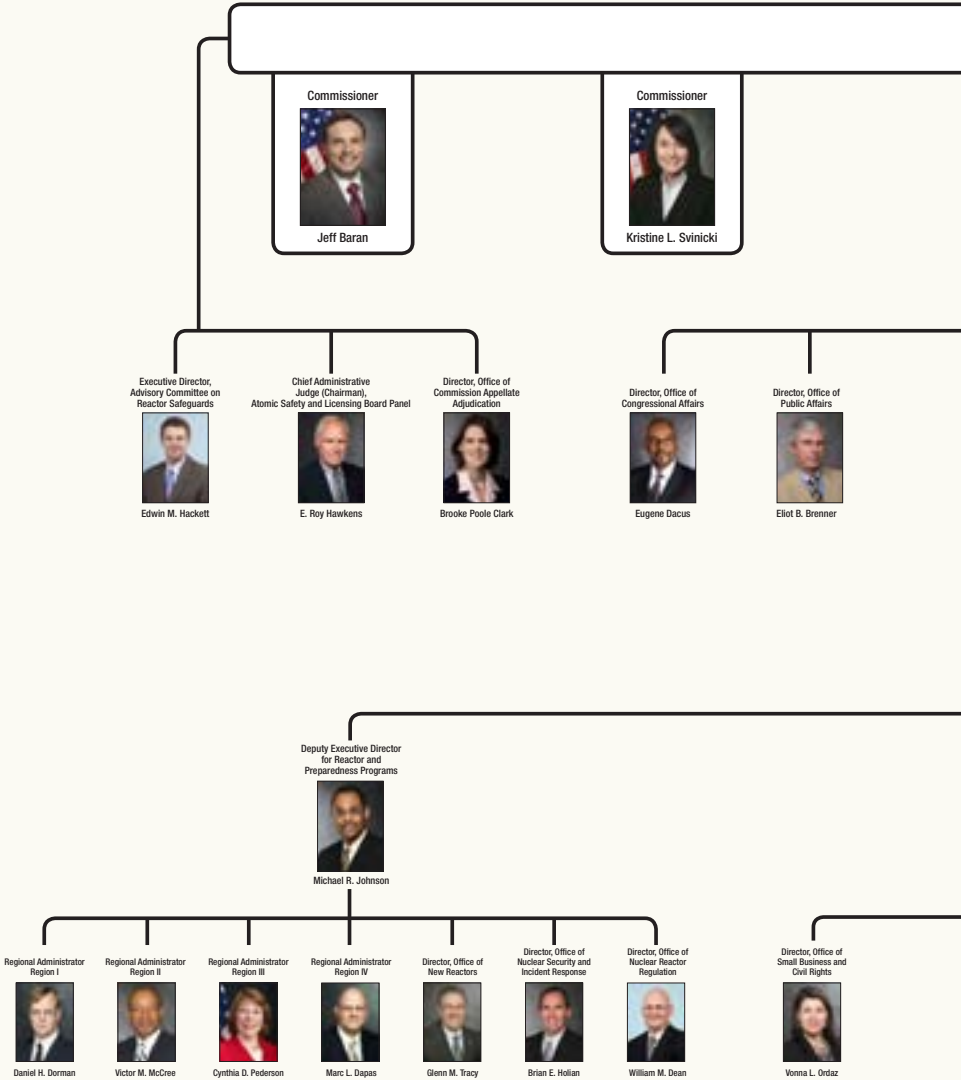
The **Office of Nuclear Regulatory Research** provides independent expertise and information for making timely regulatory judgments, anticipating problems of potential safety significance, and resolving safety issues. It helps develop technical regulations and standards and collects, analyzes, and disseminates information about the safety of commercial nuclear power plants and certain nuclear materials activities.

The **Office of Nuclear Material Safety and Safeguards** regulates activities and oversees the regulatory framework for the safe and secure production of commercial nuclear fuel and the use of nuclear material in medical, industrial, academic, and commercial applications; uranium-recovery activities; and the decommissioning of previously operating nuclear facilities. It regulates safe storage, transportation, and disposal of high- and low-level radioactive waste and spent nuclear fuel. The office also works with other Federal agencies, States, and Tribal and local governments on regulatory matters.

The **Office of Nuclear Security and Incident Response** initiates and oversees implementation of agency security policy for nuclear facilities and users of radioactive material and coordinates with other Federal agencies and international organizations on security issues. This office also maintains the NRC's emergency preparedness and incident response programs.

The **Regional Offices** conduct inspections and investigations, take enforcement actions (in coordination with the Office of Enforcement), and maintain emergency response programs for nuclear reactors, fuel facilities, and materials licensees. In addition, the regions carry out licensing for certain materials licensees.

Figure 2. NRC Organizational Chart



As of June 1, 2015

The Commission

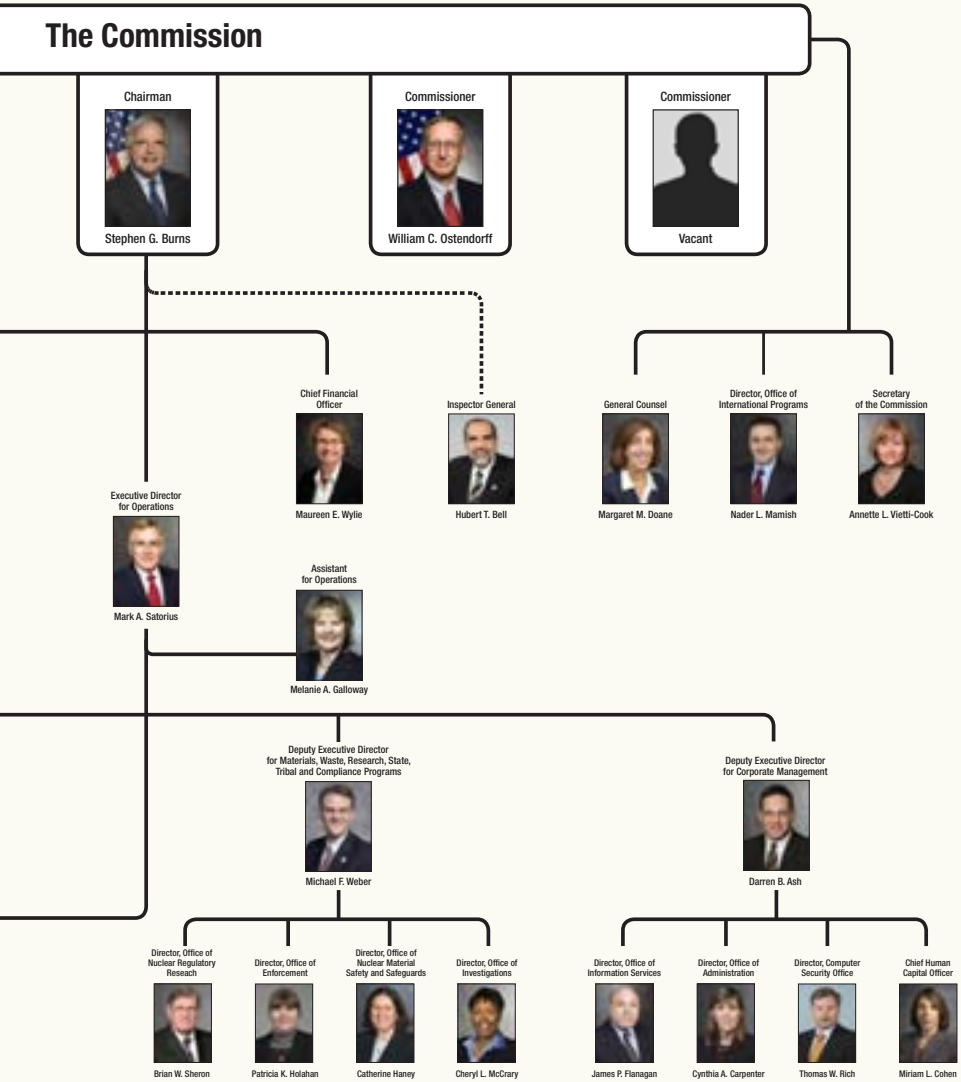
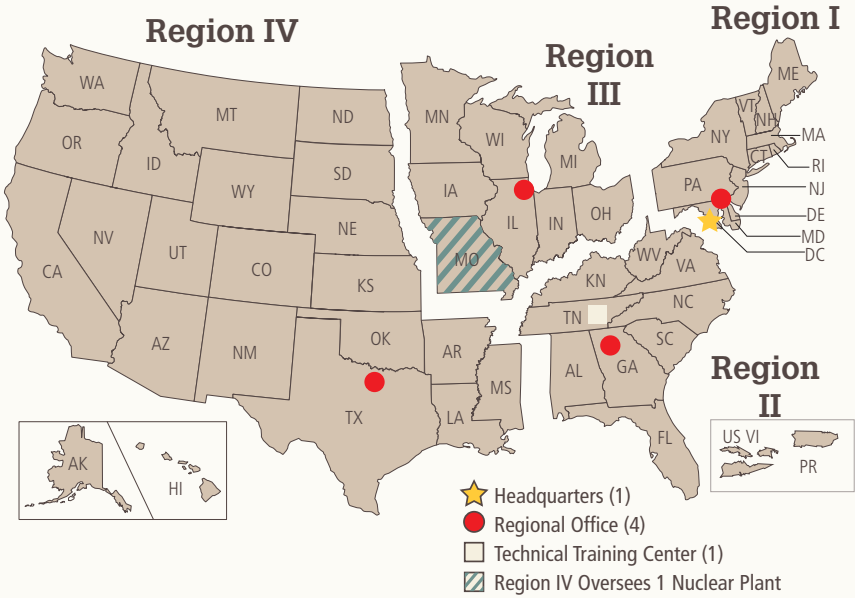


Figure 3. NRC Regions



Nuclear Power Plants

- Each regional office oversees the plants in its region—except for the Callaway plant in Missouri, which Region IV oversees.

Materials Licensees

- Region I oversees licensees and Federal facilities located geographically in Region I and Region II.
- Region III oversees licensees and Federal facilities located geographically in Region III.
- Region IV oversees licensees and Federal facilities located geographically in Region IV.

Nuclear Fuel Processing Facilities

- Region II oversees all the fuel processing facilities in all regions.
- Region II also handles all construction inspection activities for new nuclear power plants and fuel cycle facilities in all regions

Fiscal Year 2015 Budget

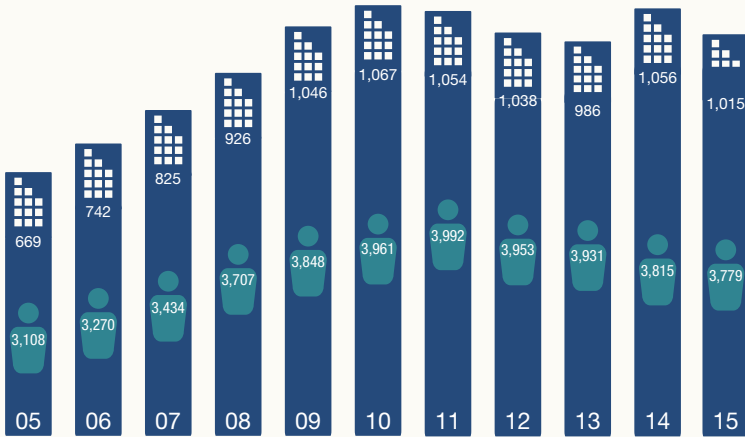
For fiscal year (FY) 2015 (October 1, 2014, through September 30, 2015), the NRC's budget is \$1,015.3 million. The NRC's FY 2015 personnel ceiling is 3,779 full-time equivalent (FTE) staff; this includes the Office of the Inspector General (see Figure 4: NRC Budget Authority and Personnel Ceiling, FYs 2005–2015). The Office of the Inspector General received its own appropriation of \$12 million. This amount is included in the total NRC budget. The breakdown of the budget is shown in Figure 5: NRC FY 2015 Distribution of Budget Authority and Staff; Recovery of NRC Budget.

By law, the NRC must recover, through fees billed to licensees, approximately 90 percent of its budget authority, less the amounts appropriated from general funds for waste-incident-to-reprocessing activities and generic homeland security activities. The NRC collects fees each year by September 30 and transfers them to the U.S. Treasury. The total budget amount the NRC will recover in FY 2015 is approximately \$895.5 million (see Figure 6: NRC Public Participation and Interaction).



The four members of the Nuclear Regulatory Commission listen to members of the Senate Energy and Water Appropriations Subcommittee during a hearing on the NRC budget. (left to right) Commissioners Kristine Svinicki and William Ostendorff, Chairman Stephen Burns, and Commissioner Jeff Baran.

Figure 4. NRC Budget Authority and Personnel Ceiling, FYs 2005–2015

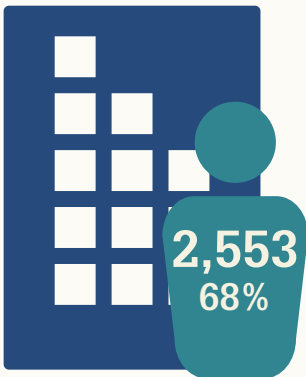


 Budget Authority
Dollars in Millions

 Full-Time Equivalent (FTE) Staff

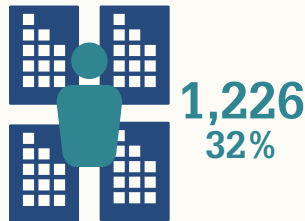
Note: Dollars are rounded to the nearest million.

Headquarters



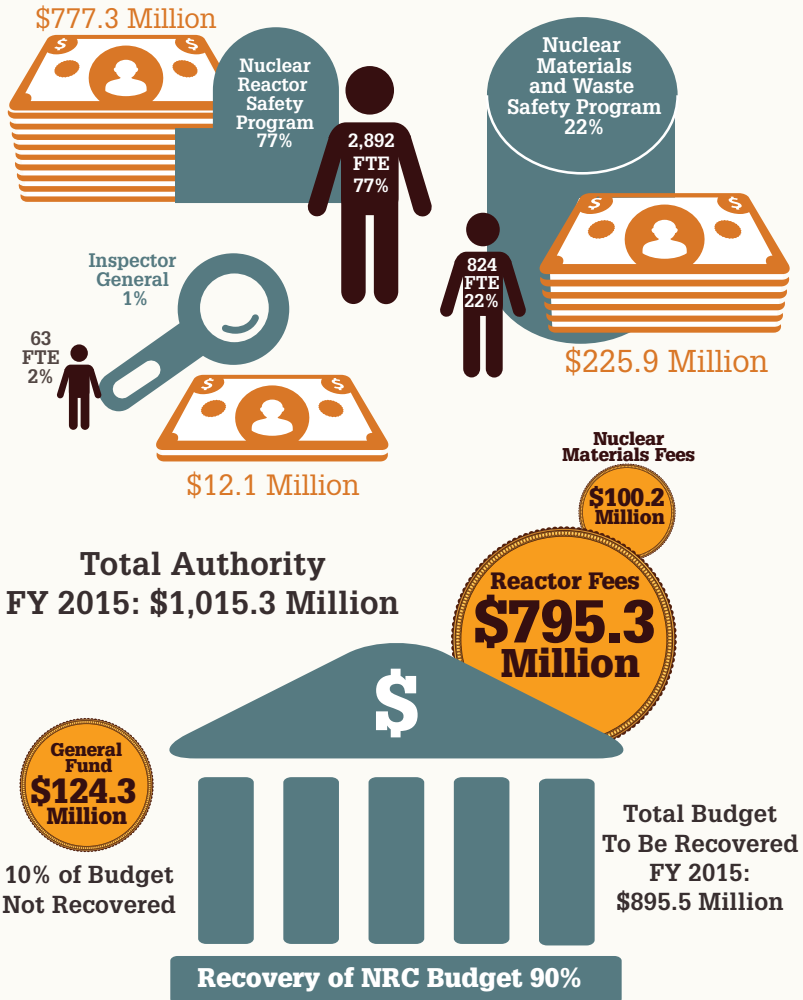
FY 2015 Staff by Location Total FTE: 3,779

Regions



Note: Numbers are rounded. Budget for FY 2015.

Figure 5. NRC FY 2015 Distribution of Budget Authority and Staff; Recovery of NRC Budget

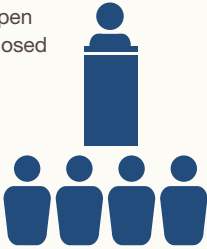


Note: The NRC incorporates corporate and administrative costs proportionately within programs. Numbers are rounded. Budget for FY 2015.

Figure 6. NRC Public Participation and Interaction

Public Meetings

Open
Closed



General Inquiries

Phone
Mail
E-mail
In Person



Information Meetings

Scoping
Preliminary
Counterpart
Information Exchanges



Resident Inspectors in the Community



10 CFR 2.206 Petition

Electronic or Hard Copy



Advisory Committee Meetings



Public Document Room

Phone
E-mail
In Person



Conferences

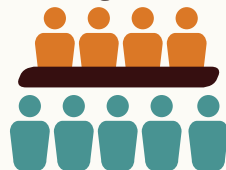
International
Trade
Industry



Open Houses



Congressional Hearings



Education and Business Outreach

Minority Groups
Small Business
Vendors/Contracts
Recruitment



Media Outreach

Press Conferences
Press Releases
Editorial Boards
Interviews



Public Comments

Regulations.gov
Mail
E-mail
Fax
Verbally at
Public Meetings



U.S. NRC

Regulatory Commission
and the Environment

Web Site

www.nrc.gov



Adjudicatory Hearings



Emergency Preparedness

Federal
State
Local



Social Media

Blog

Twitter

YouTube

Flickr

Facebook

flickr



YouTube

twitter



Visitors to the Agency



Allegations



Petitions for Rulemaking



Federal Register Notices







Nuclear Energy in the U.S. and Worldwide

2 Nuclear Energy in the U.S. and Worldwide

Nuclear science and technology are used worldwide for a variety of peaceful purposes:

- using power reactors to generate electricity
- using radioactive isotopes to diagnose and treat medical conditions
- irradiating food to make it safer and last longer
- using radiation to assist in breeding new disease- and pest-resistant seed varieties with higher yields
- using nuclear gauges to maintain quality control in industry
- measuring isotopes to date objects and identify elements

The NRC supports U.S. interests abroad in the safe and secure civilian use of nuclear materials and in guarding against the misuse of these materials and technologies for nonpeaceful purposes.

International Activities

The NRC aims its international efforts to meet needs identified by the Commission. The Office of International Programs oversees the regulatory framework for the export and import of nuclear materials. It also facilitates cooperation with other countries. Some NRC international activities are voluntary, but many are required by U.S. law or international treaties and conventions.

The NRC works with worldwide bodies, such as the International Atomic Energy Agency (IAEA), and directly with regulators in other countries through research and cooperation agreements. These activities allow the NRC to share and learn the best regulatory safety and security practices. Joint research also gives the NRC access to research facilities not available in the United States.



Photo courtesy of IAEA

The NRC participates in the annual General International Conference for the International Atomic Energy Agency in Vienna, Austria.

Conventions and Treaties

Conventions and treaties legally commit the countries that sign them to maintaining a high level of nuclear safety and security. They do this by imposing international requirements on governments, regulatory bodies, and the civilian nuclear community. These agreements help ensure safety is given proper attention. One treaty important to the NRC's work is the Treaty on the Non-Proliferation of Nuclear Weapons. This treaty promotes cooperation in peaceful uses of nuclear energy.

See Appendix AA for a list of international activities.

The NRC works with many other U.S. agencies to implement the country's international nuclear policies and obligations. The NRC develops and enforces rules, regulations, and policies that address the following issues:

- nuclear nonproliferation
- export and import licensing
- safety
- international safeguards
- physical protection
- emergency notification and assistance
- spent fuel and waste management
- liability



NRC Chairman, Stephen Burns, representing U.S. Secretary of State John Kerry, attends the 2015 Nuclear Non-Proliferation Treaty Review Conference in the Council Chamber of the United Nations in New York.

In 2015, the NRC participated in international meetings related to three conventions:

- the Convention on Nuclear Safety Diplomatic Conference
- the Fifth Review Meeting of the Contracting Parties to the Joint Convention on the Safety of Spent Fuel Management and on the Safety of Radioactive Waste Management
- the 2015 Nuclear Non-Proliferation Treaty Review Conference

At each of these meetings, the U.S. Government presents national reports detailing how the United States met its obligations. Each report was peer reviewed by participating nations, with the goal of encouraging all countries to enhance their regulatory programs.

The agency also shares information through international assistance and cooperative programs. These programs help to fulfill the NRC's obligation to share peaceful uses of nuclear technology.

Export and Import Licensing

Export and import controls help the NRC to limit proliferation and ensure the safe and secure use of nuclear materials and technology. The NRC's export and import regulations are found in 10 CFR Part 110, "Export and Import of Nuclear Equipment and Material." The agency works with other countries to implement the Code of Conduct on the Safety and Security of Radioactive Sources (see Web Link Index for Code of Conduct). Meetings with foreign regulators help to ensure consistency in how regulations are implemented around the world.

See Appendix AA for a list of export and import licenses and for a list of countries with bilateral information exchange and cooperation agreements with the NRC.

Bilateral Cooperation and Assistance

The NRC has 45 information-sharing agreements with other countries, Taiwan, and the European Atomic Energy Community (see Appendix AA for the list of countries that have bilateral information exchange and cooperation agreements with the NRC).

Cooperation

There are a wide range of programs that enhance the safety and security of peaceful nuclear activities worldwide. With countries that have mature nuclear power or radioactive materials programs, the NRC focuses on sharing information and best practices. With countries that have new programs, the NRC's focus is on helping develop and improve their regulatory activities.

Some of the benefits of consulting with other countries include:

- awareness of reactor construction activities that could apply to new reactors being built in the United States
- prompt notification to foreign partners of U.S. safety issues
- sharing information about safety issues
- sharing security information

Assistance

The NRC offers training, workshops, and peer review of regulatory documents to other countries. The agency also participates in working group meetings and exchanges of technical information and specialists. If asked, the NRC will respond directly to countries looking for help to improve their controls of radioactive material. In the past year, the NRC has assisted countries in the Caucasus region of central Asia as well as in Latin America, Africa, and the Middle East.

The NRC's Advisory Committee on Reactor Safeguards (ACRS) also works with advisory committees in other countries. The ACRS exchanges information with these committees through annual working group meetings. They hold plenary meetings every 4 years.

Foreign Assignee Program

The NRC provides on-the-job training to foreign nationals at NRC Headquarters, and the regional offices. The NRC's Foreign Assignee Program allows the NRC staff to exchange information with regulators from around the world. This helps both agencies better understand the other's regulatory programs, capabilities, and commitments. It also helps enhance the expertise of both foreign assignees and the NRC staff. And the program fosters relationships between the NRC and key officials in other countries.

Multilateral Organizations

Bilateral activities can help individual countries, but the NRC also works with many countries at the same time. These multilateral interactions allow the sharing of information with countries facing similar challenges or at similar stages of developing their national regulatory programs.

The NRC plays an active role in the different programs and committee work of global organizations. The agency works with the IAEA, the Organisation for Economic Co-operation and Development's Nuclear Energy Agency, and other multilateral organizations on issues related to:

- safety research
- radiation protection
- risk assessment
- emergency preparedness
- waste management
- transportation
- safeguards
- physical protection
- security
- standards development
- training
- technical assistance
- communications

See Appendix AA for a list of the NRC's participation with multilateral organizations.

International Cooperative Research

The NRC participates in international cooperative research programs to share U.S. operating experience and to learn from the experiences of other countries. Recent exchanges have focused on managing aging nuclear plants, fire risk in nuclear plants, and pressurized thermal shock in reactor vessels. The NRC also participates in international efforts to improve the security of radioactive materials and the management of radioactive wastes.

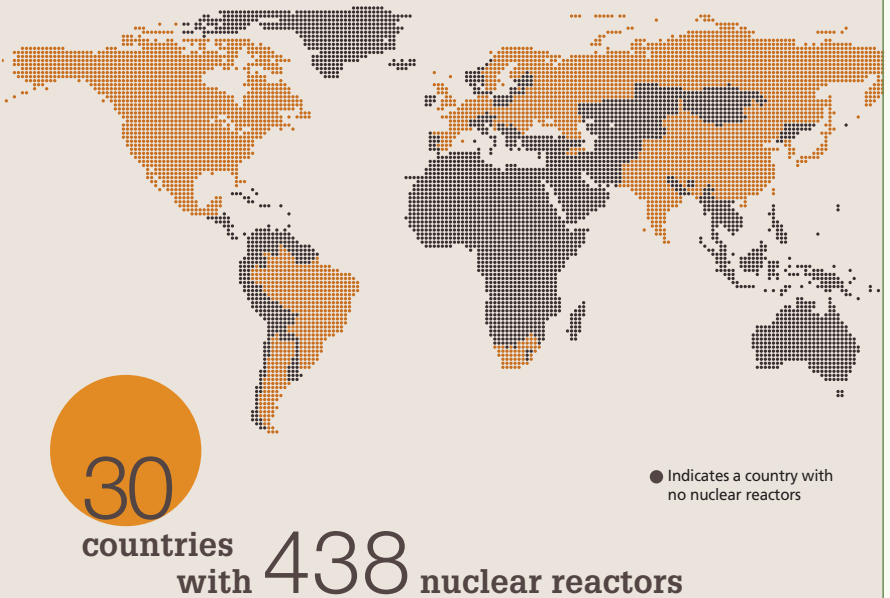
The NRC participates in cooperative research programs with 30 countries and Taiwan through approximately 100 multilateral agreements. This helps leverage access to foreign test facilities that are otherwise unavailable to the United States. Access to foreign test facilities expands the NRC's knowledge base and contributes to the best use of NRC resources.

Worldwide Electricity Generated by Commercial Nuclear Power

As of June 2015, there were 438 operating reactors in 30 countries with a total installed capacity of 379,261 megawatts electric (MWe) (see Figure 7: Operating Nuclear Power Plants Worldwide). In addition, two nuclear power plants were in long-term shutdown and 65 were under construction. Based on preliminary data from 2014, France had the highest portion (78 percent) of total domestic energy generated by nuclear power (see Figure 8: Nuclear Share of Electricity Generated by Country).

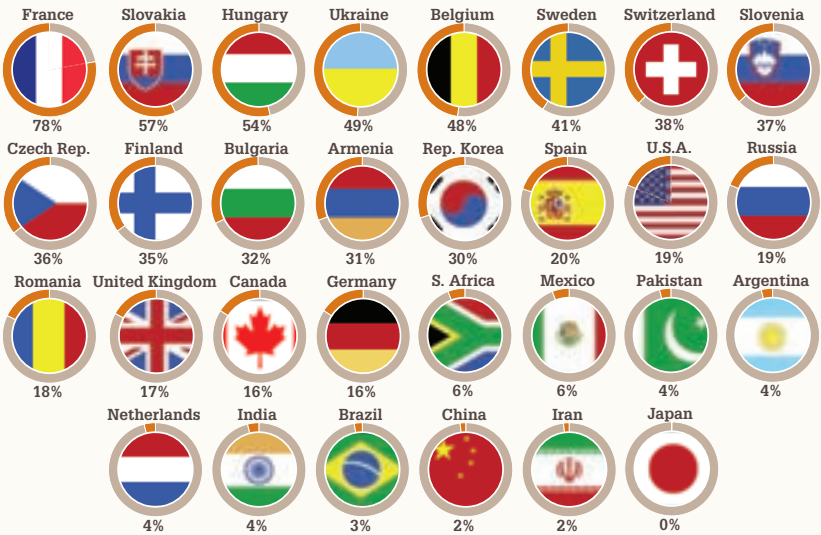
See Appendix S for the number of nuclear power reactor units by nation and Appendix T for nuclear power reactor units by reactor type, worldwide.

Figure 7. Operating Nuclear Power Plants Worldwide



Source: IAEA, Power Reactor Information System database, as of June 2015

Figure 8. Nuclear Share of Electricity Generated by Country



Note: The country's short-form name is used.

Source: IAEA, Power Reactor Information System database, as of May 2015



NRC delegation inside Unit 4 of the Fukushima Dai-ichi nuclear power site in Japan.



The Indian Point nuclear power plant located in Buchanan, NY.





Nuclear Reactors

U.S. Electricity Generated by Commercial Nuclear Power

In 2014, NRC-licensed nuclear reactors generated about 19 percent of U.S. net electricity, or about 771 billion kilowatt-hours (see Figure 9: U.S. Net Electric Generation by Energy Source, 2014, and Figure 10: U.S. Net Electric Generation by Energy Source, 2004–2014).

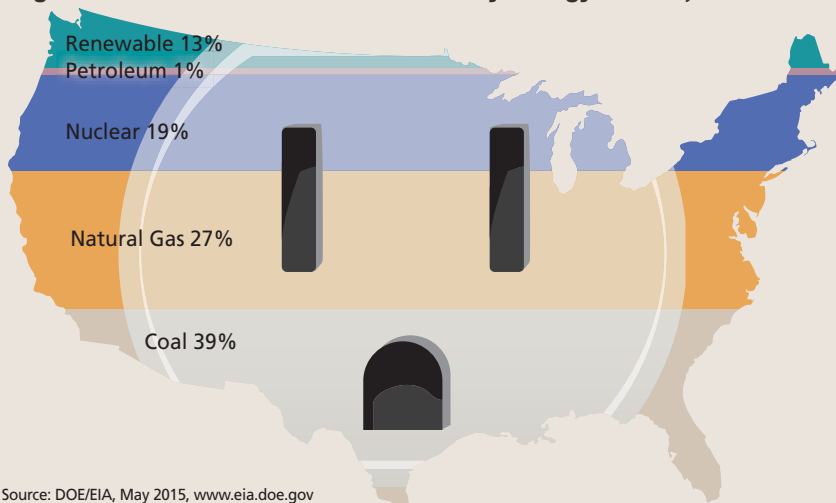
Since the 1970s, the Nation’s utilities have asked permission to generate more electricity from existing nuclear plants. By August 2014, the NRC had approved 156 of these “power uprates,” resulting in an additional 7,095 MWe from existing plants. Collectively, these uprates have added the equivalent of seven new reactors’ worth of electrical generation to the power grid. Licensees responding to a December 2014 NRC survey indicated that, by 2019, they planned to submit seven more power uprate applications, beyond those in review at that time. If approved, the resulting uprates would add another 580 MWe to the Nation’s generating capacity (see Figure 11: Power Uprates: Past, Current, and Future).



See Glossary for information on electric power grid.

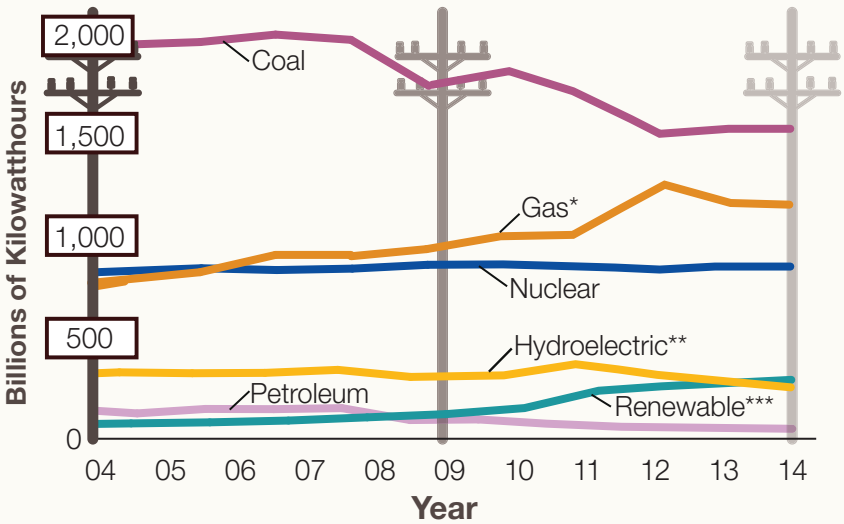
Thirty of the 50 States generate electricity from nuclear power plants. Of these States, three (New Hampshire, New Jersey, and South Carolina) generated more than 50 percent of their electricity from nuclear power last year. In addition, 11 States generated 25 to 50 percent of their electricity from nuclear power in 2013. The data cited reflect the percentages of the total net generation from nuclear sources in these States (see Figure 12: Net Electricity Generated in Each State by Nuclear Power).

Figure 9. U.S. Net Electric Generation by Energy Source, 2014



Source: DOE/EIA, May 2015, www.eia.doe.gov
 Note: Figures are rounded to the nearest whole digit.

Figure 10. U.S. Net Electric Generation by Energy Source, 2004–2014



* Gas includes natural gas, blast furnace gas, propane gas, and other manufactured and waste gases derived from fossil fuel.

** Hydroelectric includes conventional hydroelectric and hydroelectric pumped storage.

*** Renewable energy includes geothermal, wood and nonwood waste, wind, and solar energy.

Source: DOE/EIA, May 2015, www.eia.doe.gov

Figure 11. Power Upgrades: Past, Current, and Future

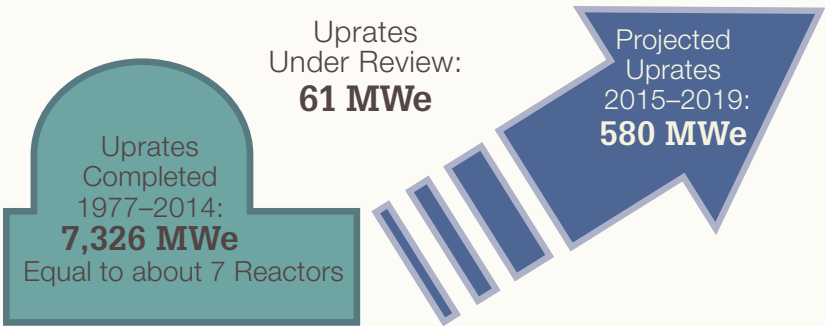
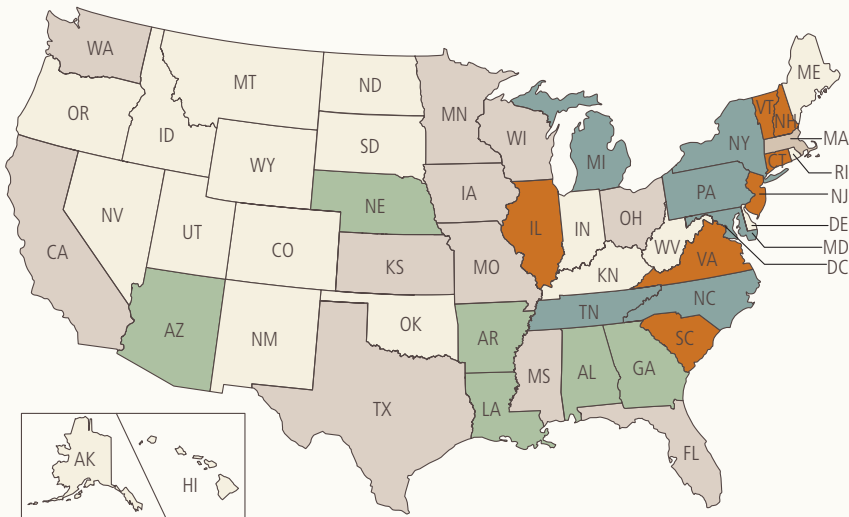


Figure 12. Net Electricity Generated in Each State by Nuclear Power

Percent of Total Nuclear Power Generated



Total Nuclear Power Generated (in Megawatts)

Illinois	97,858 MWe	Ohio	16,284 MWe
Pennsylvania	78,715 MWe	Connecticut	15,841 MWe
S. Carolina	52,419 MWe	Arkansas	14,481 MWe
New York	43,039 MWe	Maryland	14,343 MWe
Alabama	41,244 MWe	Minnesota	12,707 MWe
N. Carolina	40,967 MWe	New Hampshire	10,168 MWe
Texas	39,287 MWe	Mississippi	10,151 MWe
Georgia	32,570 MWe	Nebraska	10,102 MWe
Arizona	32,321 MWe	Washington	9,497 MWe
New Jersey	31,507 MWe	Wisconsin	9,447 MWe
Virginia	30,221 MWe	Missouri	9,276 MWe
Michigan	31,246 MWe	Kansas	8,558 MWe
Florida	27,868 MWe	Massachusetts	5,769 MWe
Tennessee	27,670 MWe	Vermont	5,061 MWe
Louisiana	17,311 MWe	Iowa	4,152 MWe
California	16,986 MWe		

Source: DOE/EIA, "State Electricity Profiles," Data from June 2015, www.eia.doe.gov

U.S. Commercial Nuclear Power Reactors

As of August 2015, the NRC oversees 99 licensed commercial nuclear power reactors operating at 61 sites in 30 States. The Vermont Yankee nuclear power reactor permanently shut down at the end of 2014 and entered the decommissioning process.

The Nation's 99 commercial power reactors fall into several categories:

- four different reactor vendors
- 23 operating companies
- about 80 different designs
- 65 pressurized-water reactors (PWRs) and 34 boiling-water reactors (BWRs)

Although commercial U.S. reactors have many similarities, each one can be considered unique.



See Glossary for typical PWR and BWR design illustrations.

See Appendix F for a listing of parent companies of U.S. commercial operating nuclear power reactors, Appendix A for a listing of reactors and their general licensing information, Appendix U for Native American Reservations and Trust lands near nuclear power plants, and Appendix M for radiation doses and regulatory limits.

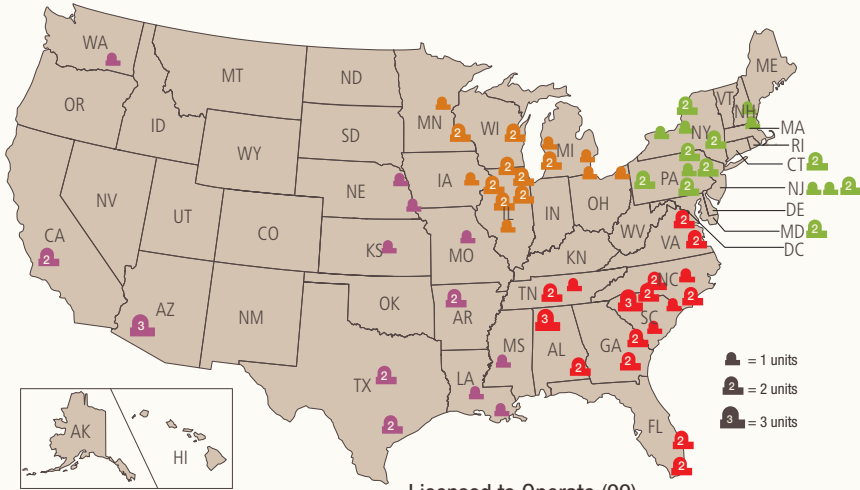
Resident Inspectors

At least two NRC inspectors work full time at each nuclear power plant site to ensure the reactors are meeting NRC regulations (see Figure 13: U.S. Operating Commercial Nuclear Power Reactors and Figure 14: Day in the Life of an NRC Resident Inspector).



An NRC inspector conducts routine inspections of plant equipment to ensure the plant is meeting NRC regulations.

Figure 13. U.S. Operating Commercial Nuclear Power Reactors



Licensed to Operate (99)

REGION I

- CONNECTICUT**
 ■ Millstone 2 and 3
- MARYLAND**
 ■ Calvert Cliffs 1 and 2
- MASSACHUSETTS**
 ■ Pilgrim
- NEW HAMPSHIRE**
 ■ Seabrook
- NEW JERSEY**
 ■ Hope Creek
 ■ Oyster Creek
 ■ Salem 1 and 2
- NEW YORK**
 ■ FitzPatrick
 ■ Ginna
 ■ Indian Point 2 and 3
 ■ Nine Mile Point 1 and 2
- PENNSYLVANIA**
 ■ Beaver Valley 1 and 2
 ■ Limerick 1 and 2
 ■ Peach Bottom 2 and 3
 ■ Susquehanna 1 and 2
 ■ Three Mile Island 1

REGION II

- ALABAMA**
 ■ Browns Ferry 1, 2, and 3
 ■ Farley 1 and 2
- FLORIDA**
 ■ St. Lucie 1 and 2
 ■ Turkey Point 3 and 4
- GEORGIA**
 ■ Edwin I. Hatch 1 and 2
 ■ Vogtle 1 and 2
- NORTH CAROLINA**
 ■ Brunswick 1 and 2
 ■ McGuire 1 and 2
 ■ Harris 1
- SOUTH CAROLINA**
 ■ Catawba 1 and 2
 ■ Oconee 1, 2, and 3
 ■ Robinson 2
 ■ Summer
- TENNESSEE**
 ■ Sequoyah 1 and 2
 ■ Watts Bar 1
- VIRGINIA**
 ■ North Anna 1 and 2
 ■ Surry 1 and 2

REGION III

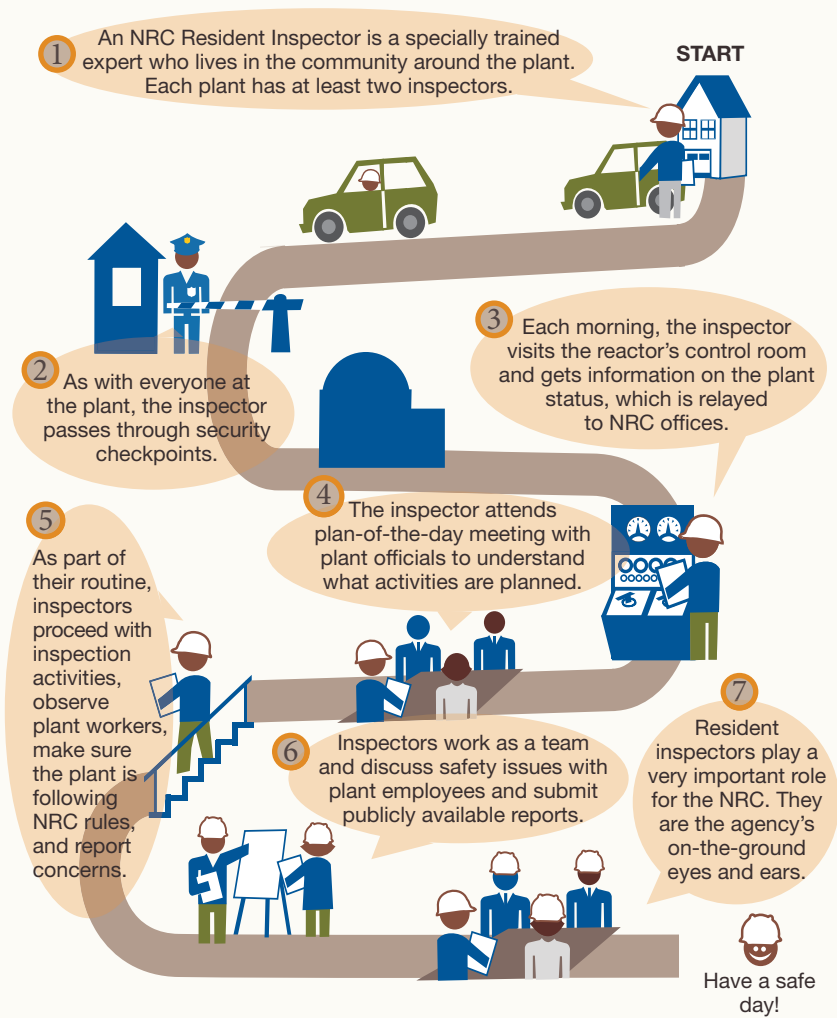
- ILLINOIS**
 ■ Braidwood 1 and 2
 ■ Byron 1 and 2
 ■ Clinton
 ■ Dresden 2 and 3
 ■ LaSalle 1 and 2
 ■ Quad Cities 1 and 2
- IOWA**
 ■ Duane Arnold
- MICHIGAN**
 ■ Cook 1 and 2
 ■ Fermi 2
 ■ Palisades
- MINNESOTA**
 ■ Monticello
 ■ Prairie Island 1 and 2
- OHIO**
 ■ Davis-Besse
 ■ Perry
- WISCONSIN**
 ■ Point Beach 1 and 2

REGION IV

- ARKANSAS**
 ■ Arkansas Nuclear 1 and 2
- ARIZONA**
 ■ Palo Verde 1, 2, and 3
- CALIFORNIA**
 ■ Diablo Canyon 1 and 2
- KANSAS**
 ■ Wolf Creek 1
- LOUISIANA**
 ■ River Bend 1
 ■ Waterford 3
- MISSISSIPPI**
 ■ Grand Gulf
- MISSOURI**
 ■ Callaway
- NEBRASKA**
 ■ Cooper
 ■ Fort Calhoun
- TEXAS**
 ■ Comanche Peak 1 and 2
 ■ South Texas Project 1 and 2
- WASHINGTON**
 ■ Columbia

NRC-abbreviated reactor names listed

Figure 14. Day in the Life of an NRC Resident Inspector



Learn more about resident inspectors. Watch the videos on the NRC YouTube Channel at: www.youtube.com/user/NRCgov

Japan Lessons Learned

A few months after Japan's Fukushima Dai-ichi nuclear accident in March 2011, an NRC task force issued several recommendations for enhancing safety at U.S. nuclear plants. Approximately 55 full-time NRC employees work with experts from across the agency to act on what the NRC learned from the events at Fukushima. A steering committee of senior NRC managers directs these activities.

The NRC developed and issued three orders in March 2012, requiring U.S. reactors to:

- obtain and protect additional emergency equipment, such as pumps and generators, to support all reactors at a given site simultaneously following a natural disaster
- install enhanced equipment for monitoring water levels in each plant's spent fuel pool
- improve and install emergency venting systems that can relieve accident pressure at reactors with designs similar to the Fukushima plant (additional NRC venting requirements added in June 2013)

The Japan Lessons-Learned efforts included detailed seismic and flooding inspections (called walkdowns) to reaffirm plants' existing abilities to withstand earthquakes and flooding. The NRC has been receiving and analyzing information from all U.S. reactors related to earthquake and flooding hazards. The agency is also working on new or revised post-Fukushima rules related to filtering radioactive material after an accident, maintaining key safety functions in an extended power blackout situation, and improving several aspects of emergency preparedness (see Figure 15: NRC Post-Fukushima Safety Enhancements).

Principal Licensing, Inspection, and Enforcement Activities

The NRC's commercial reactor licensing and inspection activities in 2014 included:

- reviewing the Tennessee Valley Authority's operating license application for the Watts Bar Unit 2 reactor, now under construction near Spring City, TN
- reviewing 670 separate license change requests from power reactor licensees in FY 2014, which averages about seven reviews per licensee
- spending an average of 6,326 hours performing inspection-related activities at each operating reactor site (see Figure 16: NRC Inspection Effort at Operating Reactors, 2014)

Figure 15. NRC Post-Fukushima Safety Enhancements

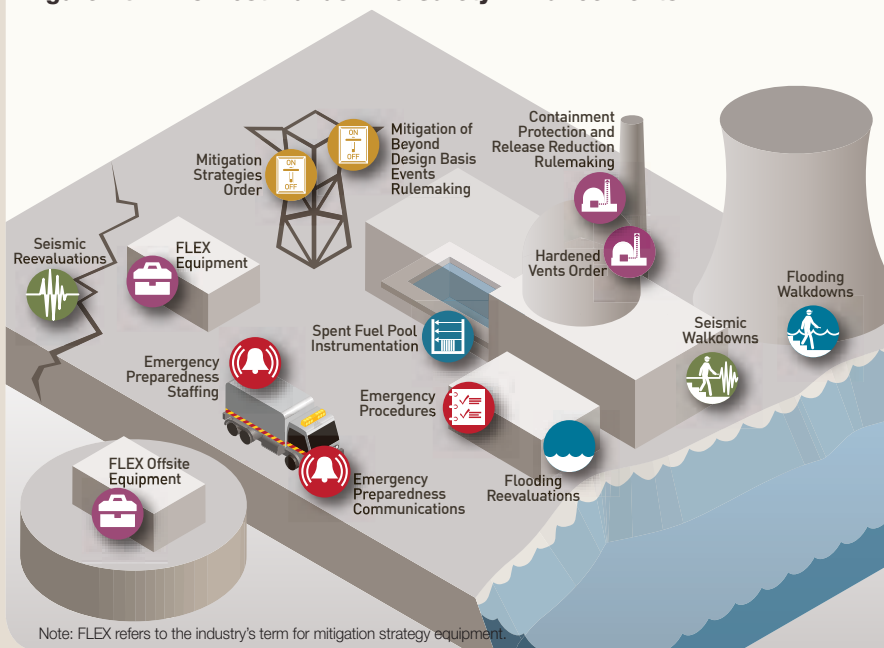
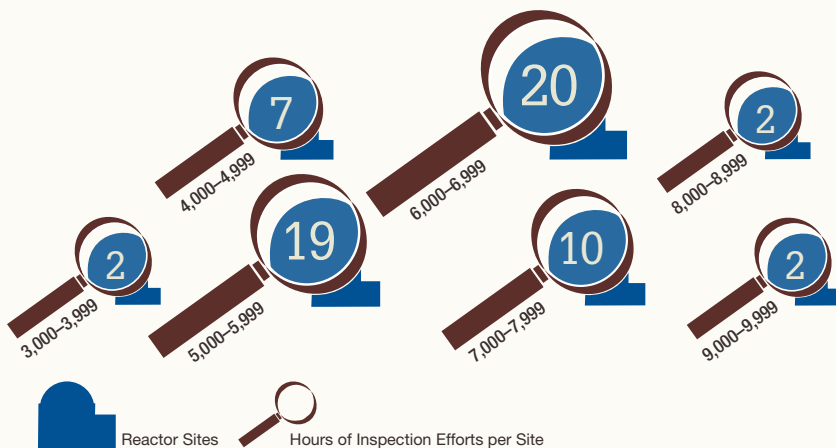


Figure 16. NRC Inspection Effort at Operating Reactors, 2014



Note: Data include calendar year (CY) 2014 hours for all activities related to baseline, plant-specific, generic safety issue, and allegation inspections.

3 Nuclear Reactors

- ensuring the qualifications of approximately 4,600 NRC-licensed reactor operators who must requalify every 2 years and ask the NRC to renew their license every 6 years
- reviewing 7 applications for proposed 11 new reactors
- inspecting construction activities at three U.S. sites
- reviewing approximately 3,000 operating experience items each year and distributing lessons learned that could help licensed facilities operate more effectively
- issuing about 36 notices of violation, civil penalties, or orders to operating reactors for significant violations of NRC regulations regarding public health and safety
- investigating approximately 500 allegations of inadequacy or impropriety associated with NRC-regulated activities
- incorporating independent advice from the ACRS, which held 10 full Committee meetings and approximately 47 subcommittee meetings during FY 2014 to examine potential safety issues for existing or proposed reactors

See Appendix C for a list of reactors undergoing decommissioning and permanently shut down and Appendix W for a list of significant enforcement actions.



An NRC inspector conducts routine inspections of plant equipment to ensure the plant is meeting NRC regulations.

Oversight of U.S. Commercial Nuclear Power Reactors

The NRC establishes requirements for the design, construction, operation, and security of U.S. commercial nuclear power plants. The agency ensures the plants operate safely and securely within these requirements by licensing the plants to operate, licensing control room personnel, establishing technical specifications for operating each plant, and inspecting plants daily.

Reactor Oversight Process

The NRC's Reactor Oversight Process (ROP) verifies that U.S. reactors are operating in accordance with NRC rules, regulations, and license requirements. If reactor performance declines, the NRC increases its oversight to protect public health and the environment. This can range from conducting additional inspections to shutting a reactor down.

The NRC staff uses the ROP to evaluate NRC inspection findings and performance records

for each reactor and uses this information to assess the reactor's safety performance and security measures. Every 3 months, through the ROP, the NRC places each reactor in one of five categories. The top category is "fully meeting all safety cornerstone objectives," while the bottom is "unacceptable performance" (see Figure 17: Reactor Oversight Action Matrix Performance Indicators). NRC inspections start with detailed baseline-level activities for every reactor. As the number of issues at a reactor increases, the NRC's inspections increase. The agency's supplemental inspections and other actions (if needed) ensure licensees promptly address significant performance issues. The latest reactor-specific inspection findings and historical performance information can be found on the NRC's Web site (see the Web Link Index).

The ROP is informed by 30 years of improvements in nuclear industry performance. The process continues to improve approaches to inspecting and evaluating the safety and security performance of NRC-licensed nuclear plants. More ROP information is available on the NRC's Web site and in NUREG-1649, Revision 5, "Reactor Oversight Process," issued February 2014 (see Figure 18: Reactor Oversight Framework). In addition to evaluating individual plant performance, the NRC compiles data on overall nuclear industry performance using industry-level statistics.

See Appendix I for a list of Industry Performance Indicators: Averages for FYs 2005–2014.)

Figure 17. Reactor Oversight Action Matrix Performance Indicators

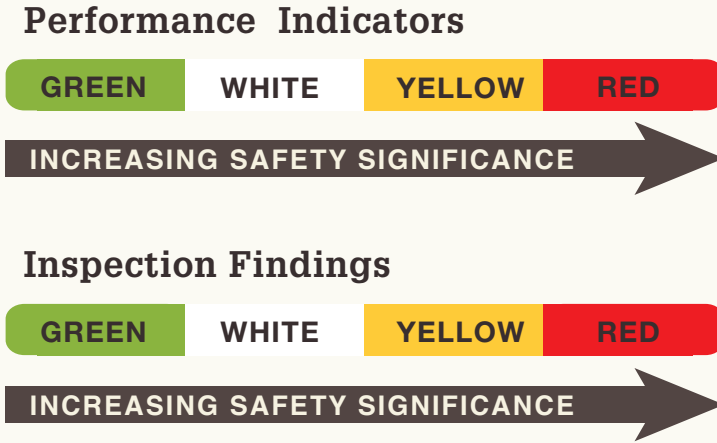
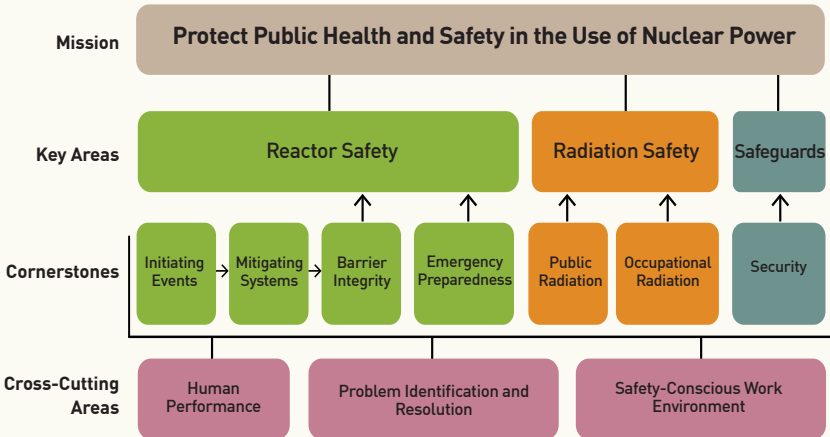


Figure 18. Reactor Oversight Framework



Reactor License Renewal

The Atomic Energy Act of 1954 (as amended) authorizes the NRC to issue 40-year initial licenses for commercial power reactors. The Act also allows the NRC to renew licenses. Under the NRC's current regulations, the agency can renew reactor licenses for 20 years at a time. Congress set the original 40-year term after considering economic and antitrust issues, as opposed to nuclear technology issues. Some parts of a reactor, however, may have been engineered based on an expected 40-year service life.

These parts must be maintained and monitored during the additional period of operation and licensees may choose to replace some components.

See Appendices G and H for power reactor operating licenses issued and expired by year.

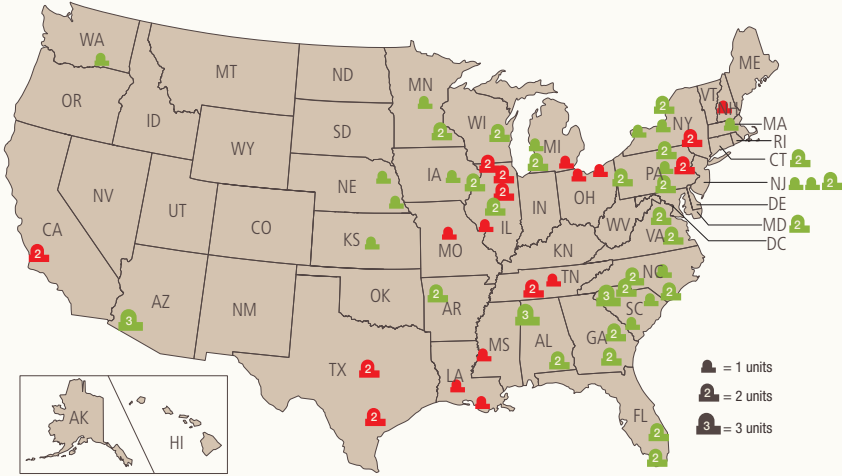
As of June 2015, 74 operating reactors (at 43 sites) have received renewed licenses (see Figure 19: License Renewals Granted for Operating Nuclear Power Reactors). To see current reactors grouped by how long they have operated, see Figure 20: U.S. Commercial Nuclear Power Reactors—Years of Operation by the End of 2015. To see reactors listed by license expiration dates, see Figure 21: U.S. Commercial Nuclear Power Reactor Operating Licenses—Expiration by Year. Nuclear power plant owners typically seek license renewal based on a plant's economic situation and on whether it can continue to meet NRC requirements in the future.

The NRC's license renewal reviews determine whether a plant can continue to operate safely beyond 40 years. The NRC reviews renewal applications for both safety issues and environmental impacts (see Figure 22: License Renewal Process). A plant owner must evaluate how the plant will address aging of plant systems not already covered by NRC inspection and maintenance rules. The plant owner must also report on the potential environmental impacts if the plant operates for up to an additional 20 years. The regulations are contained in 10 CFR Part 54, "Requirements for Renewal of Operating Licenses for Nuclear Power Plants," while the NRC's environmental review requirements for license renewal are found in 10 CFR Part 51, "Environmental Protection Regulations for Domestic Licensing and Related Regulatory Functions." If the NRC concludes extended operation will be safe, the staff verifies that safety evaluation through onsite inspections as the plant continues to operate.

The NRC considered the environmental impacts of the continued storage of spent nuclear fuel during rulemaking activities in 2013–2014. The Commission published its final Continued Storage Rule and supporting generic environmental impact statement in September 2014. The rule addresses the environmental impacts of the continued storage of spent nuclear fuel beyond a reactor's licensed operating life before ultimate disposal (previously referred to as "waste confidence"). The environmental impacts of continued storage of spent nuclear fuel are incorporated into each environmental review for license renewal. The Continued Storage Rule is discussed in more detail in Section 5.

3 Nuclear Reactors

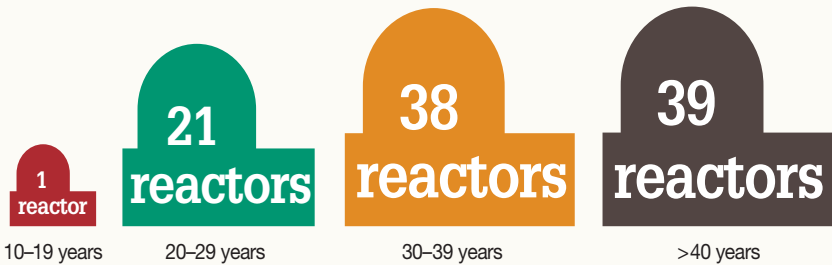
Figure 19. License Renewals Granted for Operating Nuclear Power Reactors



Licensed to Operate (99)

▲ Original License (25) ▲ License Renewal Granted (74)

Figure 20. U.S. Commercial Nuclear Power Reactors—Years of Operation by the End of 2015



Note: Ages have been rounded up to the end of the year.

Figure 21. U.S. Commercial Nuclear Power Reactor Operating Licenses—Expiration by Year

License Expiration

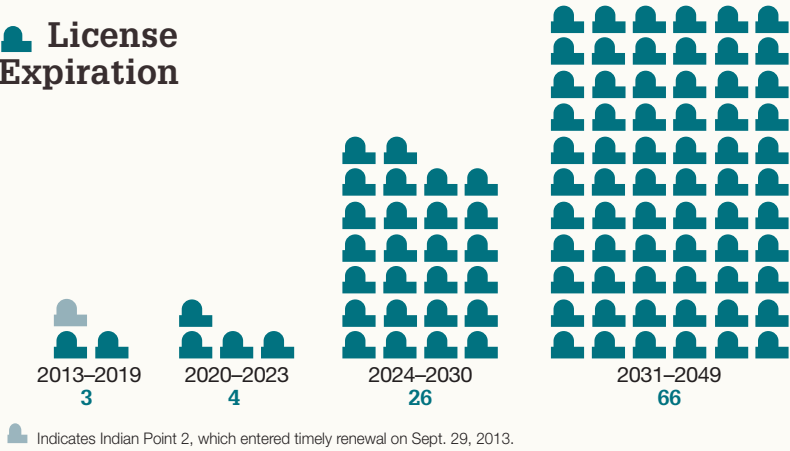
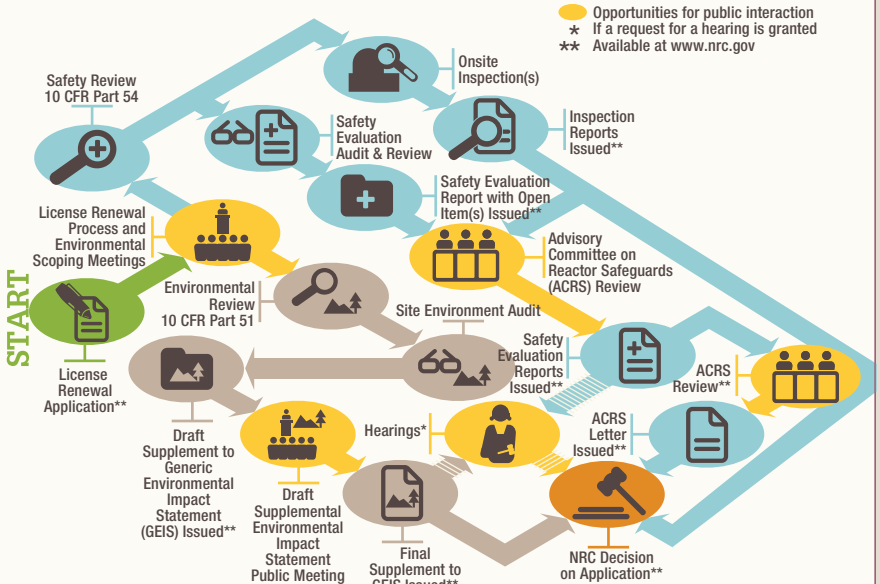


Figure 22. License Renewal Process



Public Involvement

The public plays an important role in the license renewal process. Members of the public have several opportunities to contribute to the environmental review. They can also discuss how aging will be managed if a plant operates beyond 40 years. The NRC shares information provided by the applicant and holds public meetings. The agency fully and publicly documents the results of its technical and environmental reviews. In addition, ACRS public meetings often discuss technical or safety issues related to reactor designs or a particular plant or site. Individuals or groups can raise legal arguments against a license renewal application in an Atomic Safety and Licensing Board (ASLB) hearing if they would be affected by the renewal and meet basic requirements for requesting a hearing. (For more information, see the Web Link Index.)

Research and Test Reactors

Nuclear research and test reactors (RTRs) support science and education in nuclear engineering, physics, chemistry, biology, anthropology, medicine, materials sciences, and related fields. These reactors do not produce electricity. The largest U.S. RTR (which produces 20 megawatts thermal [MWt]) is 75 times smaller than the smallest U.S. commercial power nuclear reactor (which produces 1,500 MWt). There are 36 licensed RTRs:

- 31 RTRs operating in 21 States (see Figure 23: U.S. Nuclear Research and Test Reactors)
- five RTRs shut down and are in various stages of decommissioning

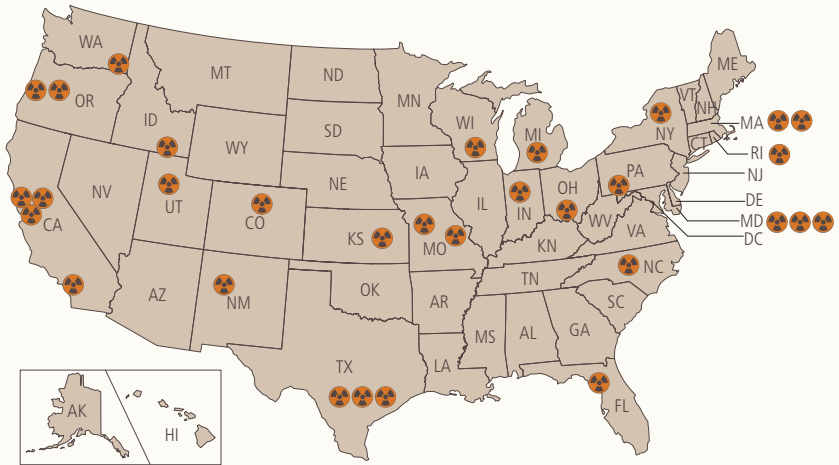
NRC inspectors visit each RTR facility at least once a year to conduct varying levels of oversight. RTRs licensed to produce 2 MWt or more receive a full NRC inspection every year. RTRs licensed to produce less than 2 MWt receive a full inspection every 2 years. Since 1958, 86 licensed RTRs have been decommissioned.



Photo courtesy Oregon State University

University students conduct experiments at the Advanced Plant Experimental Facility.

Figure 23. U.S. Nuclear Research and Test Reactors



☢ RTRs Licensed/Currently Operating (31)

1,500 MWT

**SMALLEST
COMMERCIAL
POWER REACTOR**



**1,500 Megawatts
thermal**

20 MWT

**LARGEST
RESEARCH &
TEST REACTOR**

**75x
Smaller**



**20 Megawatts
thermal**

Principal Licensing and Inspection Activities

The NRC's RTR licensing and inspection activities include:

- licensing the 31 operating sites, including license renewals and license amendments
- overseeing decommissioning
- licensing approximately 100 operators
- overseeing operator relicensing programs
- overseeing security programs
- conducting approximately 50 inspections each year, based on inspection frequency and procedures for operating RTRs

See Appendices J and K for a list of research and test reactors regulated by the NRC that are operating or are in the process of decommissioning.

New Commercial Nonpower Production and Utilization Facility Licensing

Doctors worldwide rely on a steady supply of molybdenum-99 (Mo-99), which is used in approximately 50,000 medical diagnostic procedures daily in the United States. The NRC supports the national policy objective of establishing a reliable, domestically available supply of this medical radioisotope by reviewing license applications for Mo-99 production facilities submitted in accordance with the provisions of Title 10 of the *Code of Federal Regulations*. Since 2013, the NRC staff has received two construction permit applications for nonpower production and utilization facilities from SHINE Medical Technologies, Inc. (SHINE) and Northwest Medical Isotopes, LLC. The proposed facilities would irradiate low-enriched uranium targets in utilization facilities, such as SHINE's proposed accelerator-driven subcritical operating assemblies, then separate Mo-99 from other fission products in hot cells contained within a production facility.



A supply of molybdenum-99 (Mo-99) which is used in medical diagnostic procedures.

The NRC staff will conduct safety and environmental reviews on these construction permit applications, which will also be the subject of both a mandatory hearing and an independent review by the Advisory Committee on Reactor Safeguards. If the NRC issues these construction permits, each facility must also submit an application for, and be granted, an operating license.

The NRC anticipates receiving additional construction permit applications, operating license applications, materials license applications, and license amendment requests in the coming years from other potential Mo-99 producers. Ahead of the issuance of any permit or license, the NRC continues to develop necessary infrastructure programs for these facilities, including inspection procedures for construction and operation. The agency provides updates on the status of these licensing reviews through NRC-hosted public meetings, Commission meetings, and interactions with the White House Office of Science and Technology Policy Mo-99 Interagency Working Group.

New Commercial Nuclear Power Reactor Licensing

The NRC's current review of new power reactor license applications improves on the process used through the 1990s (see Figure 24: New Reactor Licensing Process). In early 2012, the NRC issued the first combined construction permit and operating license (called a combined license or COL) under the new licensing process. In April 2015, the NRC issued a COL for the Fermi site in Michigan.

The NRC will continue reviewing six additional COL applications (which would authorize 10 new reactors) over the next several years (see Figure 25: Locations of New Nuclear Power Reactor Applications, and the Web Link Index). The NRC's ongoing design certification, COL, and early site permit (ESP) reviews are incorporating lessons learned from the Fukushima accident.

The NRC considered the environmental impacts of the continued storage of spent nuclear fuel during rulemaking activities in 2013–2014. The Commission published its final Continued Storage Rule and supporting generic environmental impact statement in September 2014. The rule addresses the environmental impacts of the continued storage of spent nuclear fuel beyond a reactor's licensed operating life before ultimate disposal (previously referred to as "waste confidence"). The environmental impacts of continued storage of spent nuclear fuel are incorporated into each environmental review for new reactor licensing. The Continued Storage Rule is discussed in more detail in Section 5.

Combined License Applications—Construction and Operating

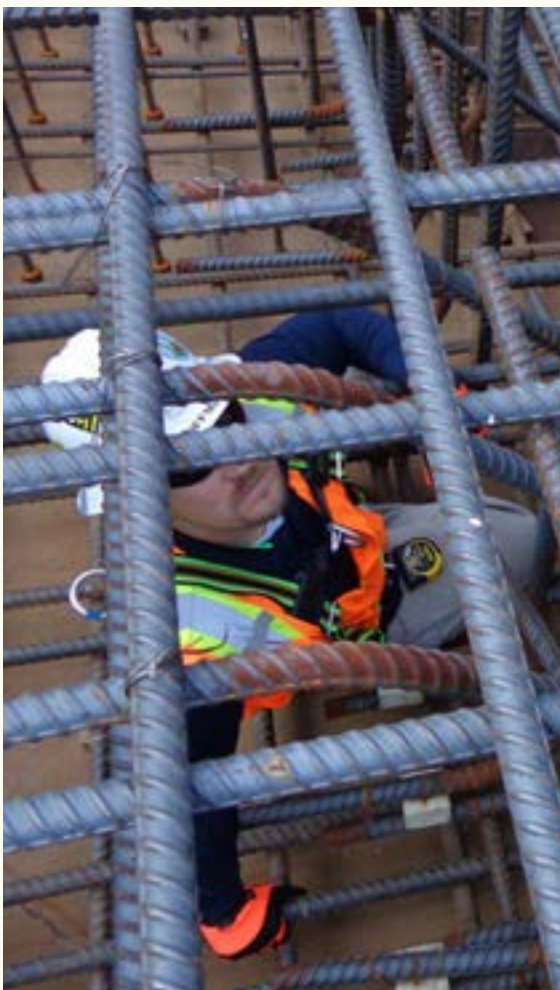
The NRC has issued COLs for five reactors at Vogtle, V.C. Summer and Fermi and as of June 2015 is currently reviewing six applications for the total of ten reactors.

- South Texas Project (TX)
- North Anna (VA)
- William States Lee III (SC)
- Levy County (FL)
- Bell Bend (PA)
- Turkey Point (FL)

Since June 2007, the NRC has received 18 COL applications for 28 new reactor units. The NRC suspended or cancelled nine COL application reviews at the request of the applicants, due to changes to their business plans (Calvert Cliffs, Grand Gulf, Callaway, Nine Mile Point, River Bend, Shearon Harris, Victoria County Station, Bellefonte, and Comanche Peak).

For the current review schedule for the active licensing applications, consult the NRC's public Web site (see the Web Link Index).

See Appendix B: List of the U.S. New Nuclear Power Plant Licensing Applications.



A construction resident inspector inspects inside the containment vessel of Vogtle Unit 3 in Georgia.

Figure 24. New Reactor Licensing Process

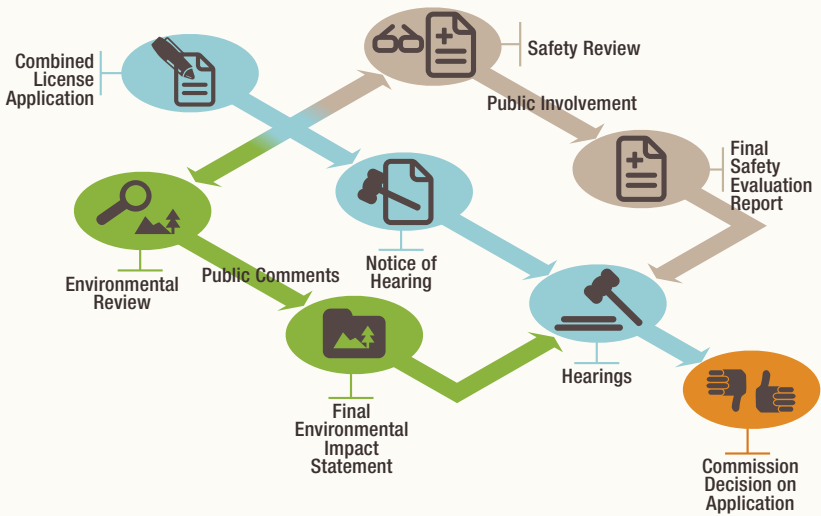
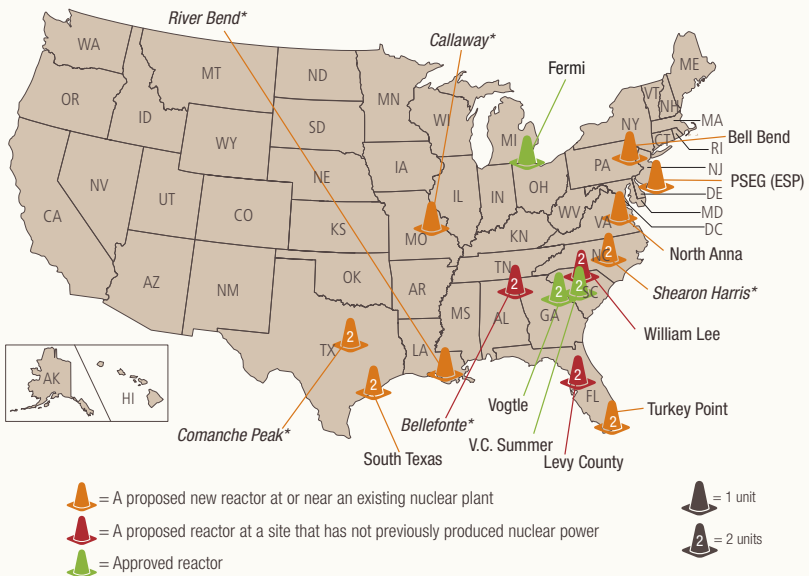


Figure 25. Locations of New Nuclear Power Reactor Applications



* Review suspended

Note: Data is as of June 2015. Withdraw was requested for Calvert Cliffs, Grand Gulf, Nine Mile Point, and Victoria County (COL and ESP). NRC-abbreviated reactor names listed

Public Involvement

Even before the NRC receives an application, the agency holds a public meeting to talk to the community near the proposed reactor location. The agency explains the review process and outlines how the public may participate in the process. After the application is submitted, the NRC asks the public to comment on which factors should be considered in the agency's environmental review conducted under the National Environmental Policy Act. The NRC later posts a draft environmental evaluation on the agency's Web site and asks for public input. There is no formal opportunity for public comment on the staff's safety evaluation, but members of the public are welcome to attend public meetings and make comments. Individuals or groups can raise legal arguments against a new reactor application in an ASLB hearing if they would be affected by the new reactor and meet basic requirements for requesting a hearing. The NRC announces opportunities to request these hearings in press releases, in the *Federal Register*, and on the NRC's Web site.



NRC staff members provide information about the agency's role and mission, and the performance of area nuclear power plants at public meetings.

Early Site Permits

An ESP review examines whether a piece of land is suitable for a nuclear power plant. The review covers site safety, environmental protection, and emergency preparedness. The ACRS reviews safety-related portions of an ESP application. As with COL reviews, the public participates in the environmental portion of the NRC's ESP review, and the public can challenge an application in a hearing.

The NRC has issued ESPs to:

- System Energy Resources, Inc. (Entergy), for the Grand Gulf site in Mississippi
- Exelon Generation Company, LLC, for the Clinton site in Illinois
- Dominion Nuclear North Anna, LLC, for the North Anna site in Virginia
- Southern Nuclear Operating Company, for the Vogtle site in Georgia

The NRC is currently reviewing an ESP application submitted in May 2010 for a site located near the Hope Creek/Salem operating reactors in New Jersey. The NRC expects to receive one additional ESP application in 2016.

Design Certifications

The NRC issues certifications for reactor designs that meet basic requirements for ensuring safe operation. Utilities can cite a certified design when applying for a nuclear power plant COL. The certification is valid for 15 years from the date issued and can be renewed for an additional 15 years. The new reactor designs under review incorporate new elements such as passive safety systems and simplified system designs. The five certified designs are:

- General Electric-Hitachi (GEH) Nuclear Energy's Advanced Boiling-Water Reactor (ABWR)
- Westinghouse's System 80+
- Westinghouse's AP600
- Westinghouse's amended AP1000
- GEH's Economic Simplified Boiling-Water Reactor

The NRC is reviewing Korea Hydro Nuclear Power's APR1400 design.

Design Certification Renewals

The NRC is reviewing applications from GEH and Toshiba to renew the ABWR design certification. The companies submitted the applications in 2010.

Advanced Reactor Designs

Several companies are considering advanced reactor designs and technologies and are conducting preapplication activities with the NRC. These technologies include light-water reactors a fraction of the size of today's designs. Other potential reactor designs are cooled by liquid metals or high-temperature gas. The NRC's advanced reactor efforts will ensure the agency has the resources and expertise to address these new technologies. While developing the regulatory framework for advanced reactor licensing, the NRC is also examining policy issues in areas such as security and emergency preparedness.

New Reactor Construction Inspections

NRC inspectors based in the agency's Region II office in Atlanta, GA, monitor reactor-construction activity. These expert staff members ensure licensees carry out construction according to NRC license specifications and related regulations. The NRC staff examines the licensee's operational programs in areas such as security, radiation protection, and operator training and qualification. Inspections at a construction site verify a licensee has completed required inspections, tests and analyses, and met associated acceptance criteria.

In February 2012, the NRC issued COLs to Southern Nuclear Operating Company for Vogtle, Units 3 and 4. In March 2012, the NRC issued COLs to South Carolina Electric and Gas for V.C. Summer, Units 2 and 3. The NRC's onsite resident construction inspectors at both Vogtle and V.C. Summer oversee day-to-day licensee and contractor activities. In addition, specialists at the NRC Region II's Center for Construction Inspection periodically visit the sites to ensure the facilities are being constructed using the approved design.

The NRC's Construction Reactor Oversight Process assesses all of these activities. Before the agency will allow a new reactor to start up, NRC inspectors must confirm the licensee has met all of the acceptance criteria in its COL.

The agency also inspects domestic and overseas factories and other vendor facilities. This ensures new U.S. reactors receive high-quality products and services that meet the NRC's regulatory requirements. The NRC's Web site has more information on new reactor licensing activities (see the Web Link Index).



Photo courtesy of Southern Nuclear Operating Company

NRC Chairman Stephen Burns (center) and NRC Regional Administrator Victor McCree (left) listen as NRC construction inspector Justin Fuller points out a feature of the new reactor construction at the Vogtle plant near Augusta, GA.

Nuclear Regulatory Research

The NRC's research supports the agency's mission by providing technical advice, tools, methods, data, and information. This research can identify, explore, and resolve safety issues, as well as provide information supporting licensing decisions and new regulations and guidance. The NRC's research includes:

- independently confirming other parties' work through experiments and analyses
- developing technical support for agency safety decisions
- preparing for the future by evaluating the safety implications of new technologies and designs for nuclear reactors, materials, waste, and security

The research program focuses on the challenges of an evolving industry, as well as on retaining technical skills when experienced staff members retire. The NRC's research covers the light-water reactor technology developed in the 1960s and 1970s, today's advanced light-water reactor designs, and fuel cycle facilities. The agency has longer-term research plans for more exotic reactor concepts, cooled by high-temperature gases or liquid metals. The NRC's research programs examine a broad range of subjects, such as:

- material performance (such as environmentally assisted degradation and cracking of metallic alloys, aging management of reactor components and materials, boric-acid corrosion, radiation effects on concrete, alkali-silica reaction in concretes, the use of high-density polyethylene material for buried piping, and embrittlement of reactor pressure vessel steels)
- events disrupting heat transfer from a reactor core, criticality safety, severe reactor accidents, how radioactive material moves through the environment, and how that material could affect human health—some of this research involves NRC-developed and approved computer codes (the codes use modern hardware, software, and plant data for the most realistic simulations. Additionally, computer codes are used to analyze fire conditions in nuclear facilities, examine how reactor fuel performs, and assess nuclear power plant risk)
- new and evolving technologies (such as the performance of mixed uranium-plutonium, computerized instrumentation and control, and safety-critical software)
- experience gained from operating reactors
- digital instrumentation and controls (such as analyzing digital system components, security aspects of digital systems, and probabilistic assessment of digital system performance)
- enhanced risk-assessment methods, tools, and models to support the increased use of probabilistic risk assessment in regulatory applications

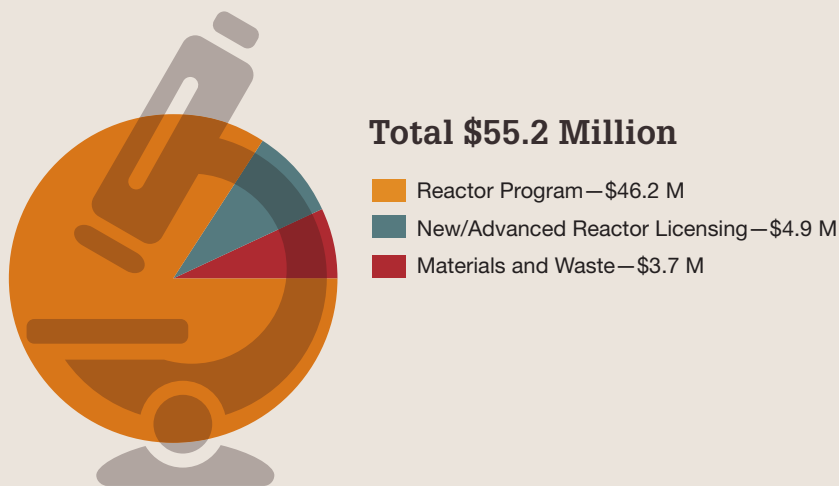
3 Nuclear Reactors

- earthquake and flooding hazards
- equipment performance under extreme conditions (e.g., heat, radiation, or humidity)
- ultrasonic testing (UT) and other nondestructive means of inspecting reactor components and dry cask storage systems; developing and accessing UT simulation tools to optimize examination procedure variables
- the human side of reactor operations, including safety culture, and computerization and automation of control rooms

The Office of Nuclear Regulatory Research also plans, develops, and manages research on fire safety and risk, including modeling, and evaluates potential security vulnerabilities and possible solutions.

The NRC's research program involves about 6 percent of the agency's personnel and uses about 14 percent of its contracting funds. The NRC's \$55.2 million research budget for FY 2015 includes contracts with national laboratories, universities, research organizations, and other Federal agencies (e.g., the National Institute of Standards and Technology, the U.S. Army Corps of Engineers, and the U.S. Geological Survey). NRC research funds support access to a broader group of experts and international research facilities. Figure 26: NRC Research Funding, FY 2015, illustrates the primary areas of research. Approximately 84 percent of the NRC's research program supports maintaining operating reactor safety and security. The remaining research budget supports regulatory activities for new and advanced reactors, industrial and medical use of nuclear materials, and nuclear

Figure 26. NRC Research Funding, FY 2015



Note: Totals may not equal sum of components because of independent rounding.

fuel cycle and radioactive waste programs. The NRC cooperates with universities and nonprofit organizations on research for the agency's specific interests.

See Appendix V for a list of the NRC's cooperative research agreements.

NRC's collaboration with the U.S. Department of Energy and the Electric Power Research Institute created a new seismic source characterization model for the Central and Eastern United States (CEUS), as well as development of a ground motion attenuation model for the CEUS. Other ongoing activities include enhancing software to perform seismic hazard calculations using the model and the latest ground motion attenuation relationships. The collaboration is also developing software to improve modeling of local site amplification of ground motion, as well as further guidance on the treatment of uncertainty and the use of experts in hazard assessment. All this work estimates the likelihood of various levels of earthquake-caused ground motions. It supports both new nuclear facility licensing and reanalysis of existing reactors east of the Rockies, as part of the NRC's lessons learned from the 2011 Fukushima nuclear accident in Japan.

NRC research activities also support regulatory activities. For example, the NRC completed the Consequence Study of a Beyond-Design-Basis Earthquake Affecting the Spent Fuel Pool for a U.S. Mark I Boiling-Water Reactor, also referred to as the Spent Fuel Pool Study. This study examined whether faster transfer of older, colder spent fuel from the pool to dry cask storage significantly reduces risks to public health and safety. The study concluded that existing

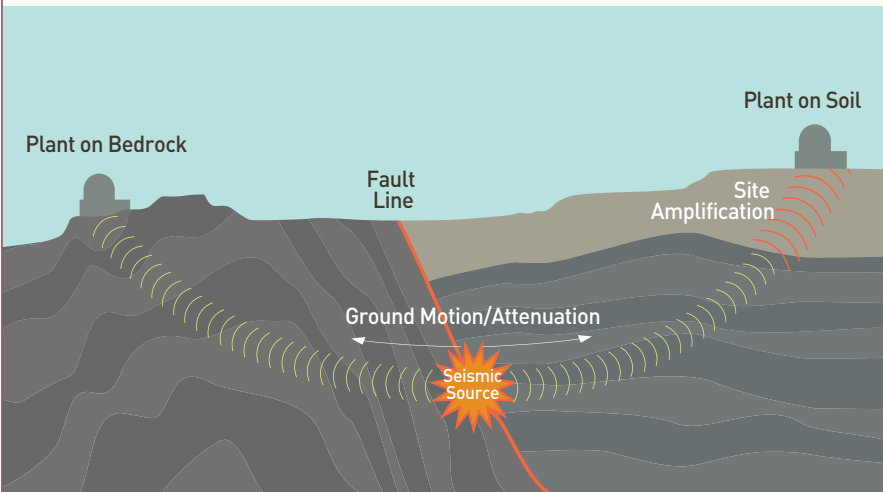


The NRC delegation listens to Naohiro Masuda, former Fukushima Dai-ni Superintendent, explain the events of March 11, 2011.

3 Nuclear Reactors

protective actions would limit any public radiation doses and keep long-term cancer fatality risk low for the scenarios studied. The NRC continues to conclude that the available evidence shows spent fuel pools adequately protect public health and safety. The insights from this analysis informed a broader regulatory analysis of the spent fuel pools at U.S. nuclear reactors as part of the Japan lessons-learned Tier 3 plan. The final report incorporated comments from the public and was published as NUREG-2161 in September 2014.

NRC research also informs the public. NUREG-1935, The State-of-the-Art Reactor Consequence Analyses (SOARCA) project, examined potential severe accidents at two pilot plants and concluded that populations near the plants would see only a very small increase in fatal cancer risk if the analyzed, but unlikely, accidents occurred. The SOARCA project, NUREG/CR-7155, “Uncertainty Analysis of the Unmitigated Long-Term Station Blackout of the Peach Bottom Atomic Power Station”, corroborated that plant’s results in NUREG-1935. A study to quantify the uncertainty of parameters used in the Surry SOARCA analysis is underway. Another SOARCA analysis is reviewing for the Sequoyah Nuclear Plant’s “ice condenser” type of containment. This study will build on the knowledge gained from the previous work to determine the effects of severe accidents on this type of containment and quantify the effectiveness of new accident mitigation techniques that have been put in place since the previous study was performed.



The NRC requires all of its licensees to take seismic activity into account when designing and maintaining their nuclear power plants. When new seismic hazard information becomes available, the NRC evaluates the new data and models and determines if any changes are needed at plants.

The NRC's international cooperation in research areas leverages agency resources, facilitates work on advancing existing technologies, and determines any safety implications of new technologies. The NRC's leadership role in international organizations such as the IAEA and the Nuclear Energy Agency helps guide the agency's collaborations.

The NRC maintains more than 100 international cooperative research agreements with more than two dozen foreign governments. This work covers technical areas from severe accident research and computer code development to materials degradation, nondestructive examination, fire risk, and human-factors research. Cooperation under these agreements is more efficient than conducting research independently. Examples of agreements include:

- the NRC's Program on Steam Generator Tube Integrity involving regulators and researchers from five foreign nations
- more than 25 agreements with foreign regulators and research organizations for participation in the NRC's Code Applications and Maintenance Program

See the Web Link Index for more information on specific NRC research projects and activities.



Universities and other academic institutions use nuclear materials in laboratory experiments while conducting research.





Nuclear Materials

The NRC regulates each phase of the nuclear fuel cycle—the steps needed to turn uranium ore into fuel for nuclear power plants, as well as storing and disposing of the fuel after it is used in a reactor. In some States, the NRC also regulates nuclear materials used for medical, industrial, and academic purposes.

Materials Licenses

See Appendix L for a list of the number of materials licenses by State.

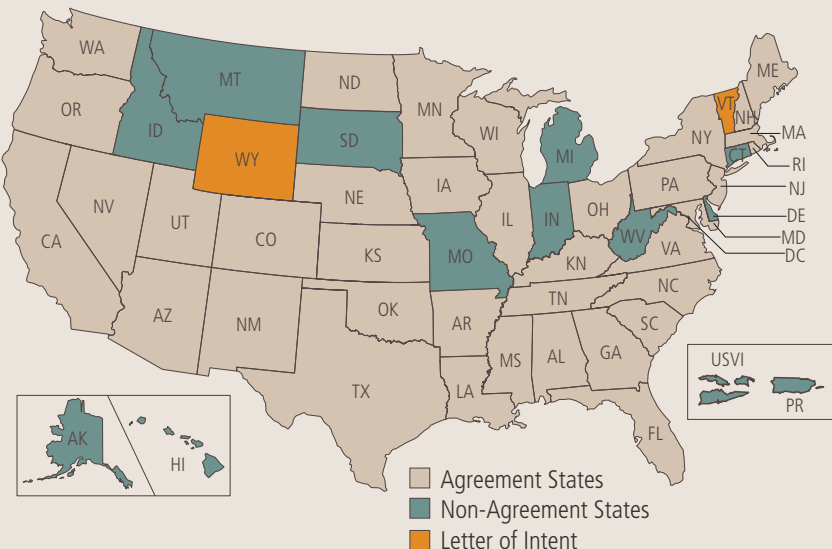
States have the option to regulate certain radioactive materials themselves under agreements with the NRC. Thirty-seven States, called Agreement States, have taken on this responsibility (see Figure 27: Agreement States). These States developed regulations and appointed officials to ensure nuclear materials are used safely and securely. Agreement States must adopt rules consistent with the NRC's. Only the NRC regulates nuclear reactors, fuel fabrication facilities, consumer product distribution, and certain amounts of what is called “special nuclear material”—that is, radioactive material that can fission or split apart.

The NRC and Agreement States administered about 20,800 licenses to use nuclear materials:

- the NRC administers approximately 2,800 licenses.
- 37 Agreement States administer approximately 18,000 licenses.

Radioactive materials, or radionuclides, are used for many purposes. They can be produced in a reactor or an accelerator—a machine that propels charged particles.

Figure 27. Agreement States



The NRC does not regulate accelerators but does license the use of radioactive materials produced in accelerators. Radionuclides are used in civilian and military industrial applications; basic and applied research; manufacture of consumer products; academic studies; and medical diagnosis, treatment, and research.

Medical and Academic

The NRC and Agreement States review the facilities, personnel, program controls, and equipment involved in using radioactive materials in medical and academic settings. These reviews ensure the safety of the public, patients, and workers who might be exposed to radiation from those materials. The NRC regulates only the use of radioactive material, which is why the NRC does not regulate x-ray machines or other devices that produce radiation without using radioactive materials.

Medical

The NRC and Agreement States license hospitals and physicians to use radioactive materials in medical treatments. The NRC also develops guidance and regulations for licensees. These regulations require licensees to have experience and special training, focusing on operating equipment safely, controlling the radioactive material, and keeping accurate records.

To help the NRC stay current, the agency sponsors an Advisory Committee on the Medical Uses of Isotopes. This expert committee includes scientists, physicians, and other health care professionals.

Nuclear Medicine

Doctors use radioactive materials to diagnose or treat about one-third of all patients admitted to hospitals. This branch of medicine is known as nuclear medicine, and the radioactive materials are called radiopharmaceuticals.

Two types of radiopharmaceutical tests can diagnose medical problems. In vivo tests (“within the living”) administer radiopharmaceuticals directly to patients. In vitro tests (“within the glass”) add radioactive materials to lab samples taken from patients.

Radiation Therapy

Doctors also use nuclear materials and radiation-producing devices to treat medical conditions. They can treat hyperthyroidism and some cancers, for example, and can also ease the pain caused by bone cancer.

Radiation therapy aims to deliver an accurate radiation dose to a target site while protecting surrounding healthy tissue. To be most effective, treatments often require several exposures over a period of time. When used to treat malignant cancers, radiation therapy is often combined with surgery or chemotherapy.

4 Nuclear Materials

Photo courtesy: Sirtex



Samples from two manufacturers of Y-90 SIR-Spheres® (left) TheraSphere® (below)

Vial containing millions of yttrium-90 (Y-90) microspheres used to treat liver cancers

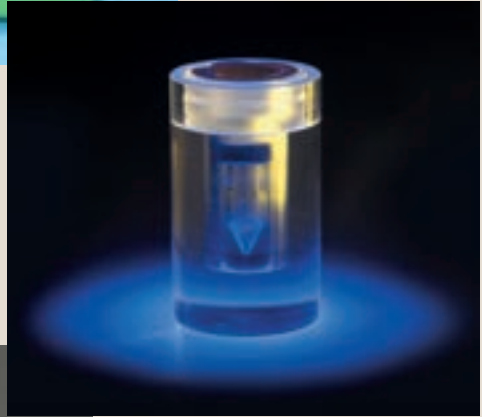


Photo courtesy: Nordion

Photo courtesy: Biodex



A shielding and syringe holder (left) is used to protect medical workers as they load a syringe from a bulk vial of a radiopharmaceutical containing fluorine-18 (F-18). The F-18 produces gamma rays, which show up in a PET Scan, allowing the radiologist to see internal organs and the blood flowing to and from them.

There are three main categories of radiation therapy:

1. External beam therapy (also called teletherapy) is a beam of radiation directed to the target tissue. Several different types of machines are used in external beam therapy. Treatment machines regulated by the NRC contain high-activity radioactive sources (usually cobalt-60) that emit photons to treat the target site.
2. Brachytherapy treatments use sealed radioactive sources placed near or even directly in cancerous tissue. The radiation dose is delivered at a distance of up to an inch (up to 2.54 centimeters) from the target area.
3. Therapeutic radiopharmaceuticals deliver a large radiation dose inside the body. Different radioactive materials can be given to patients and will concentrate in different regions or organ systems.

Academic

The NRC issues licenses to academic institutions for education and research. For example, qualified instructors may use radioactive materials in classroom demonstrations. Scientists in many disciplines use radioactive materials for laboratory research.

Industrial

The NRC and Agreement States issue licenses that specify the type, quantity, and location of radioactive materials to be used. Radionuclides can be used in industrial radiography, gauges, well logging, and manufacturing. Radiography uses radiation sources to find structural defects in metal and welds. Gas chromatography uses low-energy radiation sources to identify the chemical elements in an unknown substance. This process can determine the components of complex mixtures, such as petroleum products, smog, and cigarette smoke. (It can also be used in biological and medical research to identify the parts that make up complex proteins and enzymes.) Well-logging devices use radioactive sources and detection equipment to make a record of geological formations from inside a well. This process is used extensively for oil, gas, coal, and mineral exploration.

Nuclear Gauges

Nuclear gauges are used to measure the physical properties of products and industrial processes nondestructively as a part of quality control. Gauges use radiation sources to determine the thickness of paper products, fluid levels in oil and chemical tanks, and the moisture and density of soils and material at construction sites. Gauges may be fixed or portable.



See Glossary for illustrations of fixed and portable gauges.



A moisture density gauge indicates whether a foundation is suitable for constructing a building or roadway.

product being processed or controlled. A detector mounted opposite the source measures the radiation passing through the product. The gauge readout or computer monitor shows the measurement. The material and process being monitored dictate the type, energy, and strength of radiation used.

Fixed fluid gauges are used by the beverage, food, plastics, and chemical industries. Installed on a pipe or the side of a tank, these gauges measure the densities, flow rates, levels, thicknesses, and weights of a variety of materials and surfaces.

A portable gauge uses both a shielded radioactive source and a detector. The gauge is placed on the object to be measured. Some gauges rely on radiation from the source to reflect back to the bottom of the gauge. Other gauges insert the source into the object. The detector in the gauge measures the radiation either directly from the inserted source or from the reflected radiation.

The moisture density gauge, shown above, is a portable gauge that places a gamma source under the surface of the ground through a tube. Radiation is transmitted directly to the detector on the bottom of the gauge, allowing accurate measurements of compaction. Industry uses such gauges to monitor the structural integrity of roads, buildings, and bridges. Airport security uses nuclear gauges to detect explosives in luggage.

The radiation measurement indicates the thickness, density, moisture content, or some other property that is displayed on a gauge readout or on a computer monitor. The top of the gauge has shielding to protect the operator while the radioactive source is exposed. When the measuring process is completed, the source is retracted or a shutter closes, minimizing exposure from the source.

A fixed gauge has a radioactive source shielded in a container. When the user opens the container's shutter, a beam of radiation hits the material or

Commercial Irradiators

The U.S. Food and Drug Administration and other agencies have approved the irradiation of food. Commercial irradiators expose food and spices, as well as products such as medical supplies and wood flooring, to gamma radiation. This process can be used to eliminate harmful bacteria, germs, and insects or for hardening or other purposes. The gamma radiation does not leave radioactive residue or make the treated products radioactive. The radiation can come from radioactive materials (e.g., cobalt-60), an x-ray tube, or an electron beam.



See Glossary for information and illustration of commercial irradiators.

The NRC and Agreement States license about 50 commercial irradiators. Up to 10 million curies of radioactive material can be used in these types of irradiators. NRC regulations protect workers and the public from this radiation. Two main types of commercial irradiators are used in the United States: underwater and wet-source-storage panoramic models.

Underwater irradiators use sealed sources (radioactive material encased inside a capsule) that remain in the water at all times, providing shielding for workers and the public. The product to be irradiated is placed in a watertight container, lowered into the pool, irradiated, and then removed.

Wet-source-storage panoramic irradiators also store radioactive sealed sources in water. But the sources are raised into the air to irradiate products that are automatically moved in and out of the room on a conveyor system. Sources are then lowered back into the pool. For this type of irradiator, thick concrete walls and ceilings or steel barriers protect workers and the public when the sources are lifted from the pool.

Transportation

More than 3 million packages of radioactive materials are shipped each year in the United States by road, rail, air, or water. This represents less than 1 percent of the Nation's yearly hazardous material shipments. The NRC and the U.S. Department of Transportation (DOT) share responsibility for regulating the safety of radioactive material shipments. The vast majority of these shipments consist of small amounts of radioactive materials used in industry, research, and medicine. The NRC requires such materials to be shipped in accordance with DOT's safety regulations.

Material Security

To monitor the manufacture, distribution, and ownership of the most high-risk sources, the NRC set up a National Source Tracking System (NSTS) in January 2009. Licensees use this secure Web-based system to enter information on the receipt or transfer of tracked radioactive sources (see Figure 28: Life-Cycle Approach to Source Security). The NRC and the Agreement States use the system to monitor where high-risk sources are made, shipped, and used.

The NSTS tracks more than 80,000 sources held by about 1,400 NRC and Agreement State licensees. Of those sources, about 46 percent are Category 1 sources and 54 percent are Category 2. The majority are cobalt-60, the most widely used isotope in large sources.



See Glossary for definitions of the categories of radioactive sources.

The NRC and the Agreement States have increased controls on the most sensitive radioactive materials. Stronger physical-security requirements and stricter limits on who can access the materials give the NRC and the Agreement States added confidence in their security. The NRC has also joined with other Federal agencies, such as the U.S. Department of Homeland Security and DOE's National Nuclear Security Administration, to set up an additional layer of voluntary protection. Together, these activities help make potentially dangerous radioactive sources even more secure and less vulnerable to malevolent uses.

Major Licensing and Inspection Activities

Each year, the NRC issues about 1,800 new licenses, renewals, or amendments for existing materials licenses. The NRC conducts around 900 health, safety, and security inspections of materials licensees each year.

Nuclear Fuel Cycle

Figure 29: The Nuclear Fuel Cycle, illustrates the nuclear fuel cycle, including uranium recovery, conversion, enrichment, and fabrication to produce fuel for nuclear power plants. Uranium is recovered or extracted from ore, converted, and enriched. Then the enriched uranium is manufactured into pellets. These pellets are placed into fuel assemblies to power nuclear reactors.

Figure 28. Life-Cycle Approach to Source Security

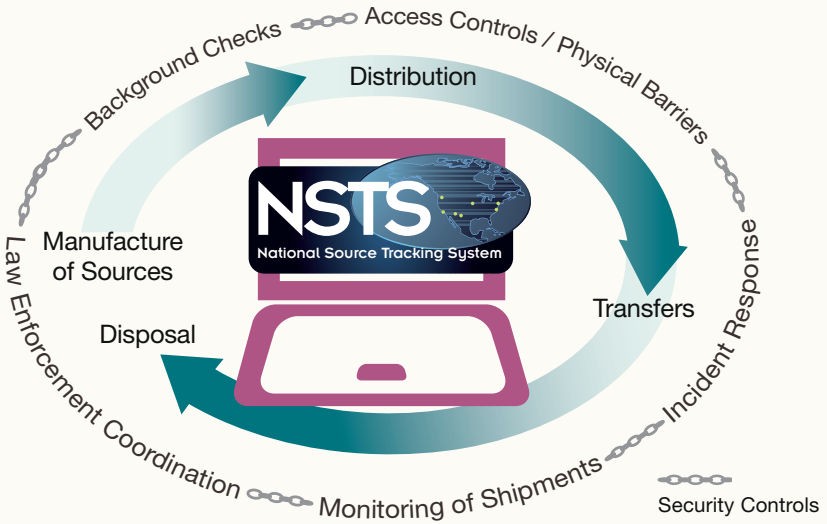
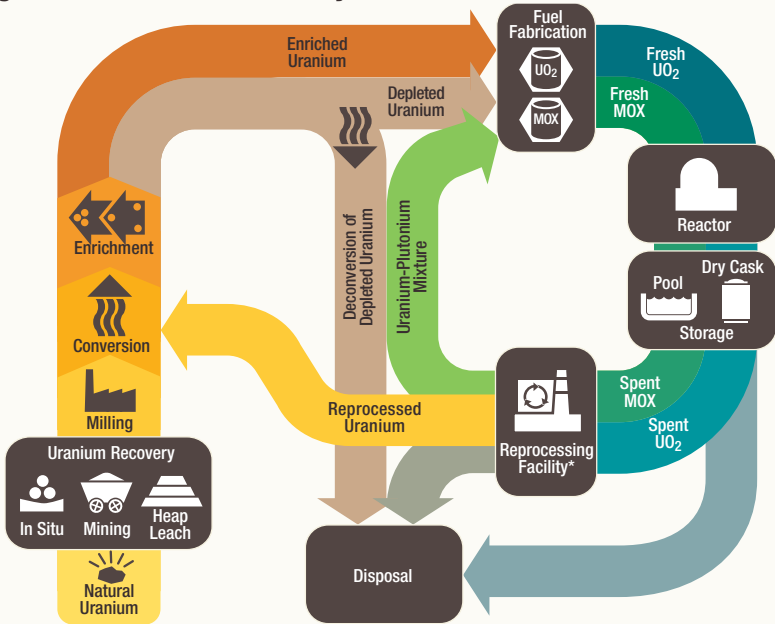


Figure 29. The Nuclear Fuel Cycle



* Reprocessing of spent nuclear fuel, including mixed-oxide (MOX) fuel is not practiced in the United States. Note: The NRC has no regulatory role in mining uranium.

Uranium Recovery

The NRC does not regulate conventional mining but does regulate the processing of uranium ore, known as milling. This processing can be done at three types of uranium recovery facilities: conventional mills, in situ recovery (ISR) facilities, and heap leach facilities. The NRC has a well-established regulatory framework for uranium recovery facilities. This framework ensures they are licensed, operated, decommissioned, and monitored to protect the public and the environment.

Conventional Uranium Mill

A conventional uranium mill is a chemical plant that extracts uranium from ore. Most conventional mills are located away from population centers and within about 30 miles (50 kilometers) of a uranium mine.

In a conventional mill, the process of uranium extraction from ore begins when ore is hauled to the mill and crushed. Sulfuric acid dissolves and removes 90 to 95 percent of the uranium from the ore. The uranium is then separated from the solution, concentrated, and dried to form yellowcake.

In Situ Recovery

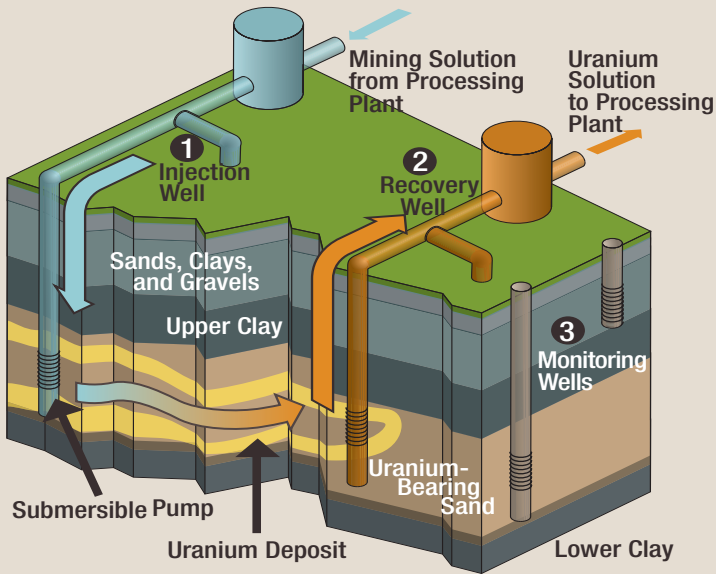
There are 21 uranium recovery sites licensed by the NRC—12 are conventional mills and nine are ISR facilities. Of these 21 facilities, 11 are in various stages of decommissioning and one is in standby status with the potential to restart in the future.

ISR is another way to extract uranium—in this case, directly from underground ore. In situ facilities recover uranium from ores that cannot be processed economically using other methods. In this process, a solution of native ground water, typically mixed with oxygen or hydrogen peroxide and sodium bicarbonate or carbon dioxide, is injected into the ore to dissolve the uranium. The solution is then pumped out of the rock and the uranium separated to form yellowcake (see Figure 30: The In Situ Uranium Recovery Process). The United States has 17 ISR facilities. Of these facilities, the NRC licenses nine and Agreement States license the rest.

Heap Leach Facility

Heap leach facilities also extract uranium from ore. At these facilities, the ore is placed in piles or heaps on top of liners. The liners prevent uranium and other chemicals from moving into the ground. Sulfuric acid is dripped onto the heap and dissolves uranium as it moves through the ore.

Figure 30. The In Situ Uranium Recovery Process



Injection wells (1) pump a chemical solution—typically ground water mixed with sodium bicarbonate, hydrogen peroxide, and oxygen—into the layer of earth containing uranium ore. The solution dissolves the uranium from the deposit in the ground and is then pumped back to the surface through recovery wells (2) and sent to the processing plant to be processed into uranium yellowcake. Monitoring wells (3) are checked regularly to ensure that uranium and chemicals are not escaping from the drilling area.



In situ recovery mining is an extraction method of obtaining uranium ore, in Crow Butte, NB.

Uranium solution drains into collection basins, where it is piped to a processing plant. At the plant, uranium is extracted, concentrated, and dried to form yellowcake. The NRC has no licensed heap leach facilities.



See Glossary for definition and illustration of heap leach recovery process.

Uranium Recovery Facilities

The NRC expects as many as 12 applications to build new uranium recovery facilities and to expand or restart existing facilities in the next few years. As of April 2015, the agency had received seven applications for new facilities and eight applications to expand or restart an existing facility.

The current status of applications can be found on the NRC's Web site (see the Web Link Index). Existing facilities and new potential sites are located in Wyoming, New Mexico, Nebraska, South Dakota, and Oregon, and in the Agreement States of Texas, Colorado, and Utah (see Figure 31: Locations of NRC-Licensed Uranium Recovery Facility Sites, and Table 1: Locations of NRC-Licensed Uranium Recovery Facilities).

The NRC works closely with stakeholders, including Native American Tribal governments, to address their concerns with licensing new uranium recovery facilities. The NRC is also responsible for the following actions:

- inspecting and overseeing both active and inactive uranium recovery facilities
- ensuring the safe management of mill tailings (waste) at facilities required by the NRC to be located and designed to minimize radon release and disturbance by weather or seismic activity
- enforcing requirements to ensure cleanup of active and closed uranium recovery facilities
- applying stringent financial requirements to ensure funds are available for decommissioning
- making sure licensees follow requirements for underground disposal of mill tailings and provide liners for tailings impoundments
- monitoring to prevent ground water contamination
- monitoring and overseeing decommissioned facilities



See Glossary for more information on mill tailings.

Figure 31. Locations of NRC-Licensed Uranium Recovery Facility Sites

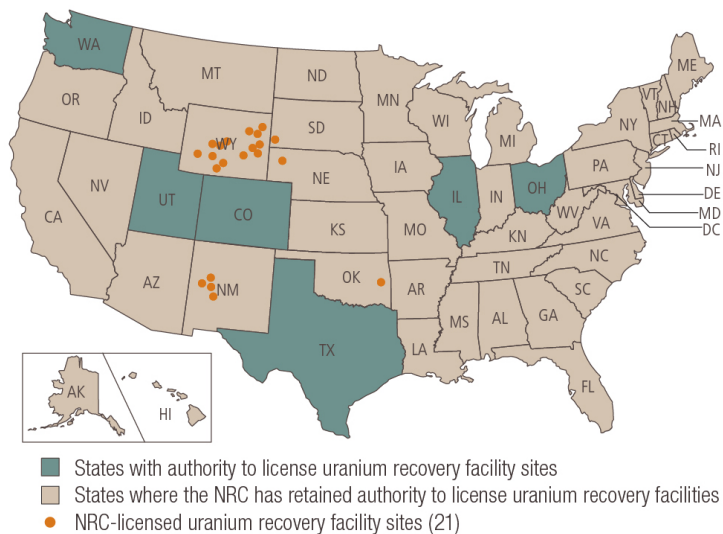


Table 1. Locations of NRC-Licensed Uranium Recovery Facilities

Licensee	Site Name, Location
In Situ Recovery Facilities	
Cameco Resources, Inc.	Crow Butte, NE*
Cameco Resources, Inc.	Smith Ranch and Highland, WY*
Hydro Resources, Inc. ^o	Crownpoint, NM
Lost Creek ISR, Inc.	Lost Creek, WY
Powertech USA	Dewey Burdock, SD
Strata Energy	Ross, WY
Uranerz Energy Corp.	Nichols Ranch, WY
Uranium One	Moore Ranch, WY
Uranium One	Willow Creek, WY
Conventional Uranium Mill Recovery Facilities	
American Nuclear Corp. [†]	Gas Hills, WY
Bear Creek Uranium Co. [†]	Bear Creek, WY
Exxon Mobil Corp. [†]	Highlands, WY
Homestake Mining Co. [†]	Homestake, NM
Kennecott Uranium Co. ^o	Sweetwater, WY
Pathfinder Mines Corp. [†]	Lucky Mc, WY
Pathfinder Mines Corp. [†]	Shirley Basin, WY
Rio Algom Mining, LLC [†]	Ambrosia Lake, NM
Sequoyah Fuels Corp. [†]	Gore, OK
Umetco Minerals Corp. [†]	Gas Hills, WY
United Nuclear Corp. [†]	Church Rock, NM
Western Nuclear, Inc. [†]	Split Rock, WY

Note: For further details on NRC-related uranium recovery facility applications in review and applications, restarts, and expansions, see the Web Link Index. This table does not include uranium recovery facilities licensed by Agreement States.

* Satellite facilities are located within the State.

† These sites are undergoing decommissioning.

^o Hydro Resources Inc. has an operating license, but the facility has not yet been constructed. Kennecott Uranium Company has an operating license but is in "standby" mode.

Fuel Cycle Facilities

The NRC licenses all commercial fuel cycle facilities involved in conversion, enrichment, and fuel fabrication (see Figure 32: Locations of Fuel Cycle Facilities, and Figure 33: Enrichment Process, and Figure 34: Simplified Fuel Fabrication Process).



See Glossary for more information on enrichment processes

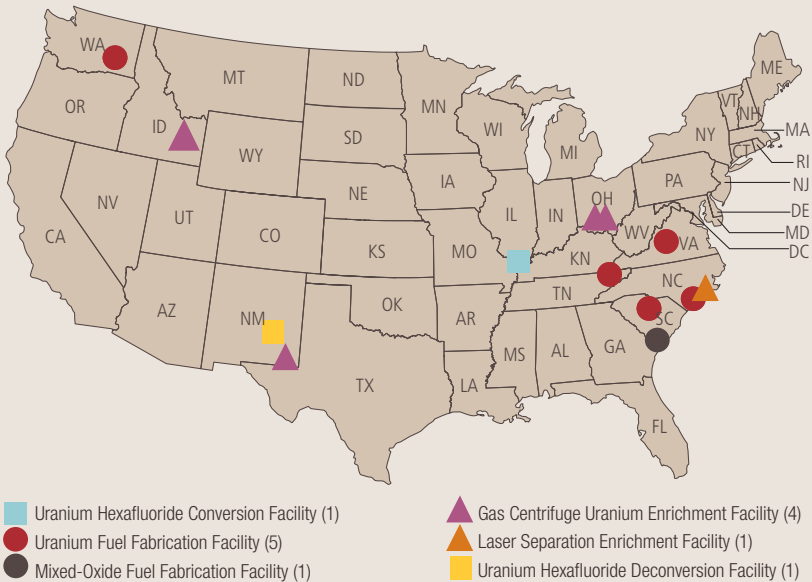
The NRC also routinely inspects their safety, safeguards, security, and environmental protection programs. On average, the NRC issues about 100 fuel cycle facility licensing actions per year, including amendments, renewals, new licenses, and safety and safeguards reviews.

These facilities turn the uranium that has been removed from ore (as yellowcake) into fuel for nuclear reactors. In this process, the conversion facility converts yellowcake into uranium hexafluoride (UF_6).

Next, an enrichment facility heats the solid UF_6 enough to turn it into a gas, which is “enriched,” or processed, to increase the concentration of the isotope uranium-235.

See Appendix N for Major U.S. Fuel Cycle Facility Sites.

Figure 32. Locations of Fuel Cycle Facilities



Note: There are no fuel cycle facilities in Alaska or Hawaii.

Figure 33. Enrichment Process

Gas Centrifuge Process

The gas centrifuge process uses many rotating cylinders that are connected in long lines. Gas is placed in the cylinder, which spins at a high speed, creating a strong centrifugal force. Heavier gas molecules move to the cylinder wall, while lighter molecules collect near the center. The stream, now slightly enriched, is fed into the next cylinder. The depleted stream is recycled back into the previous cylinder.

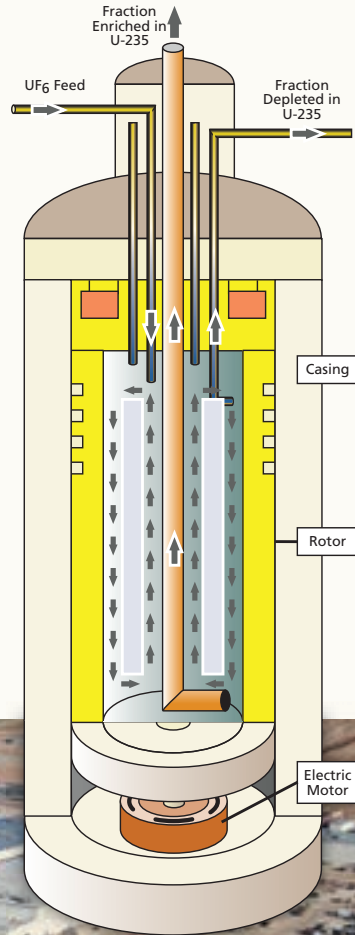
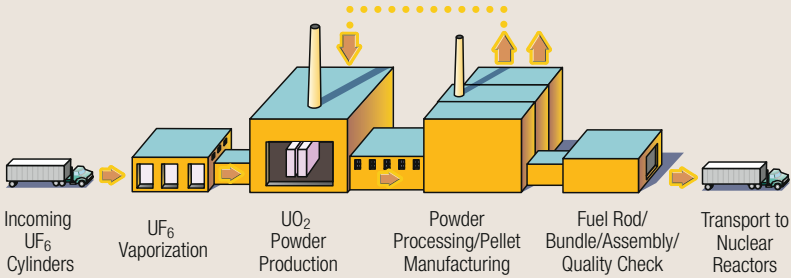


Photo courtesy: Louisiana Energy Services



URENCO USA gas centrifuge uranium enrichment facility in Eunice, NM.

Figure 34. Simplified Fuel Fabrication Process



Fabrication of commercial light-water reactor fuel consists of the following three basic steps:

- (1) the chemical conversion of UF_6 to UO_2 powder
- (2) a ceramic process that converts UO_2 powder to small ceramic pellets
- (3) a mechanical process that loads the fuel pellets into rods and constructs finished fuel assemblies



A worker displays a small ceramic fuel pellet.

The enriched uranium gas is mechanically and chemically processed back into a solid uranium dioxide (UO_2) powder. The powder is blended, milled, pressed, and fused into ceramic fuel pellets about the size of a fingertip. The pellets are stacked into tubes or rods that are about 14 feet (4.3 meters) long and made of material such as zirconium alloys; this material is referred to as cladding. These fuel rods are made to maintain both their chemical and physical properties under the extreme conditions of heat and radiation present inside an operating reactor.

After careful inspection, the fuel rods are bundled into fuel assemblies for use in reactors. The assemblies are washed, inspected, and stored in a special rack until ready for shipment to a nuclear power plant. The NRC inspects this operation to ensure it is conducted safely.

Domestic Safeguards Program

The NRC's domestic safeguards program for fuel cycle facilities and transportation is aimed at ensuring that special nuclear material (such as plutonium or enriched uranium) is not stolen and does not pose a risk to the public from sabotage or terrorism. Through licensing and inspections, the NRC verifies that licensees apply safeguards to protect special nuclear material.

The NRC and DOE developed the Nuclear Materials Management and Safeguards System (NMMSS) to track transfers and inventories of special nuclear material, source material from abroad, and other material. The NRC has issued licenses authorizing some 180 facilities to possess special nuclear material in quantities ranging from a single kilogram to multiple tons. These licensees verify and document their inventories in the NMMSS database.

The NRC or Agreement States license several hundred additional sites that possess special nuclear material in smaller quantities. Licensees possessing small amounts of special nuclear material must confirm their inventory annually in the NMMSS database.



The Willow Creek Project site for in situ recovery is located in Wyoming.





Radioactive Waste

Low-Level Radioactive Waste Disposal

Low-level radioactive waste (LLW) includes items contaminated with radioactive material or exposed to neutron radiation. This waste typically consists of contaminated protective shoe covers and clothing, wiping rags, mops, filters, reactor water treatment residues, equipment and tools, medical waste, and laboratory animal carcasses and tissue.

Some LLW is quite low in radioactivity—even as low as just above background levels found in nature. Some licensees, notably hospitals, store such waste on site until it has decayed and lost most of its radioactivity. Then it can be disposed of as ordinary trash. Other LLW, such as parts of a reactor vessel from a nuclear power plant, is more radioactive and requires special handling. Waste that does not decay fairly quickly is stored until amounts are large enough for shipment to an LLW disposal site in containers approved by DOT and the NRC.

Commercial LLW can be disposed of in facilities licensed by either the NRC or Agreement States. The facilities are designed, constructed, and operated to meet NRC safety standards. The facility operator analyzes how the facility will perform in the future based on the environmental characteristics of the site. Current LLW disposal uses shallow land disposal sites with or without concrete vaults (see Figure 35: Low-Level Waste Disposal).

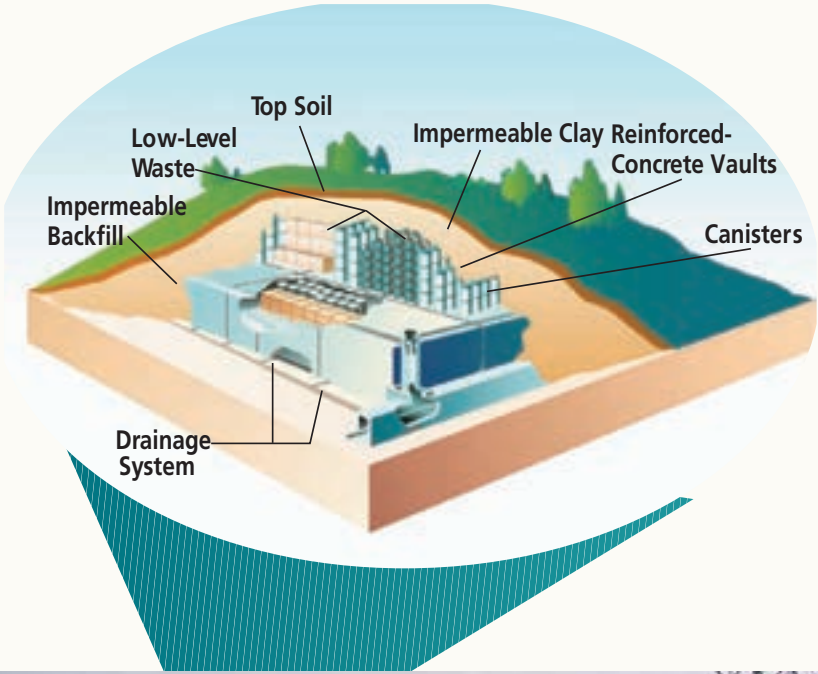
The NRC classifies LLW based on its potential hazards. It has specified disposal and waste requirements for three classes of waste—Classes A, B, and C—with progressively higher concentrations of radioactive material. Class A waste, the least radioactive, accounts for approximately 96 percent of the total volume of LLW. Determination of the classification of waste is a complex process. A fourth class of LLW, called “greater-than-Class-C,” is not generally acceptable for near-surface, shallow-depth disposal. By law, DOE is responsible for disposal of greater-than-Class-C waste generated under an NRC license. Although it falls under DOE jurisdiction, it is not high-level waste.

The volume and radioactivity of waste vary from year to year. Waste volumes currently include several million cubic feet each year from reactor facilities undergoing decommissioning and from cleanup of contaminated sites.

The Low-Level Radioactive Waste Policy Amendments Act of 1985 gave the States responsibility for LLW disposal. The Act authorized States to:

- form regional compacts, with each compact to provide for LLW disposal site access
- manage LLW import to, and export from, a compact
- exclude waste generated outside a compact

Figure 35. Low-Level Waste Disposal



This LLW disposal site accepts waste from States participating in a regional disposal agreement.

5 Radioactive Waste

The States have licensed four active LLW disposal facilities:

- EnergySolutions' Barnwell facility, located in Barnwell, SC—Previously, Barnwell accepted LLW waste from all U.S. generators of LLW. As of July 2008, Barnwell accepts waste only from the Atlantic Compact States of Connecticut, New Jersey, and South Carolina. The State of South Carolina licenses Barnwell to receive Classes A, B, and C waste.
- EnergySolutions' Clive facility, located in Clive, UT—Clive accepts waste from all regions of the United States. The State of Utah licenses Clive for Class A waste only.
- US Ecology's Richland facility, located in Richland, WA, on the Hanford Nuclear Reservation—Richland accepts waste from the Northwest Compact States (Alaska, Hawaii, Idaho, Montana, Oregon, Utah, Washington, and Wyoming) and the Rocky Mountain Compact States (Colorado, Nevada, and New Mexico). The State of Washington licenses Richland to receive Classes A, B, and C waste.
- Waste Control Specialists' Andrews facility, located in Andrews, TX—Andrews accepts waste from the Texas Compact, which consists of Texas and Vermont. It also accepts waste from out-of-the-compact generators on a case-by-case basis. Andrews began receiving waste in 2011. The State of Texas licenses Andrews to receive Classes A, B, and C waste.

See Appendix Q for Regional Compacts and Closed LLW Sites.



Licenses post locations of radioactive areas around the plants to ensure proper radiation protection measures at the facilities.

High-Level Radioactive Waste Management

Spent Nuclear Fuel Storage

Commercial spent nuclear fuel, although highly radioactive, is stored safely and securely in 34 States. This includes 30 States with operating nuclear power reactors, where spent fuel is safely stored on site in spent fuel pools and in dry casks. The remaining four States—Colorado, Idaho, Maine, and Oregon—do not have operating power reactors but are safely storing spent fuel at storage facilities. Waste can be stored safely in pools or casks for 100 years or more. The NRC licenses and regulates the storage of spent fuel, both at commercial nuclear power plants and at separate storage facilities.

Most reactor facilities were not designed to store the full amount of spent fuel that the reactors would generate during their operational lives. Facilities originally planned to store spent fuel temporarily in deep pools of continuously circulating water, which cools the spent fuel assemblies. After a few years, the facilities were expected to send the spent fuel to a reprocessing plant. However, in 1977, the Government declared a moratorium on reprocessing spent fuel in the United States.

Although the Government later lifted the restriction, reprocessing has not resumed in the United States.

See Appendices O and P for information about dry spent fuel storage and licensees.



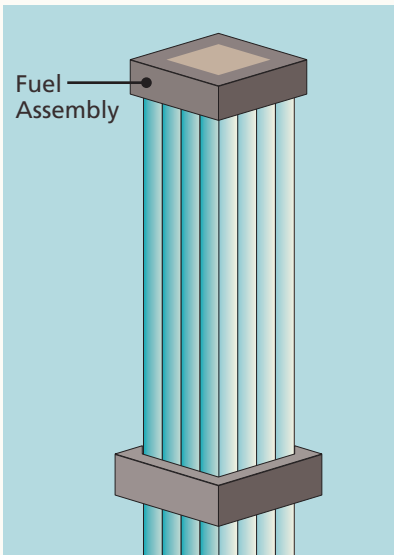
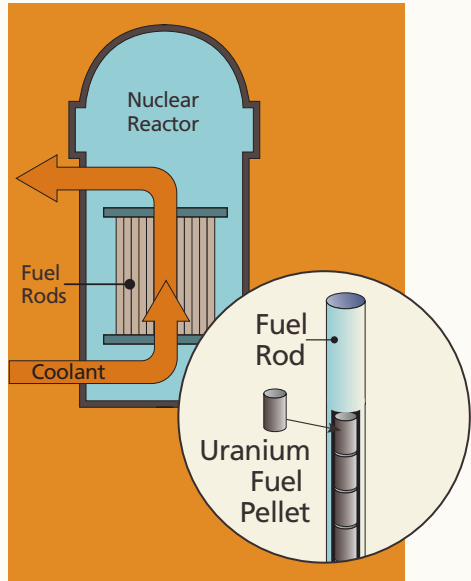
See Glossary for fuel reprocessing (recycling).

As a result, facilities expanded their storage capacity by using high-density storage racks in their spent fuel pools (see Figure 36: Spent Fuel Generation and Storage after Use). To provide supplemental storage, some fuel assemblies are stored in dry casks on site. These facilities are called independent spent fuel storage installations (ISFSIs) and are licensed by the NRC. These large casks are typically made of leak-tight, welded, and bolted steel and concrete surrounded by another layer of steel or concrete. The spent fuel sits in the center of the cask in an inert gas. Dry cask storage shields people and the environment from radiation and keeps the spent fuel inside dry and nonreactive (see Figure 37: Dry Storage of Spent Nuclear Fuel).

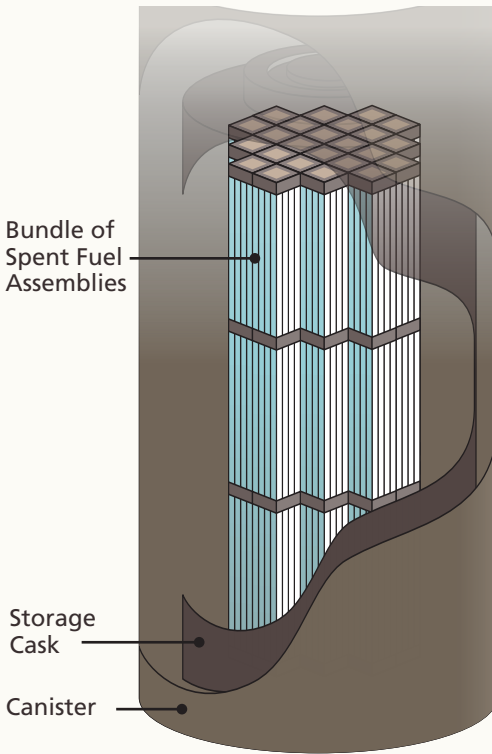
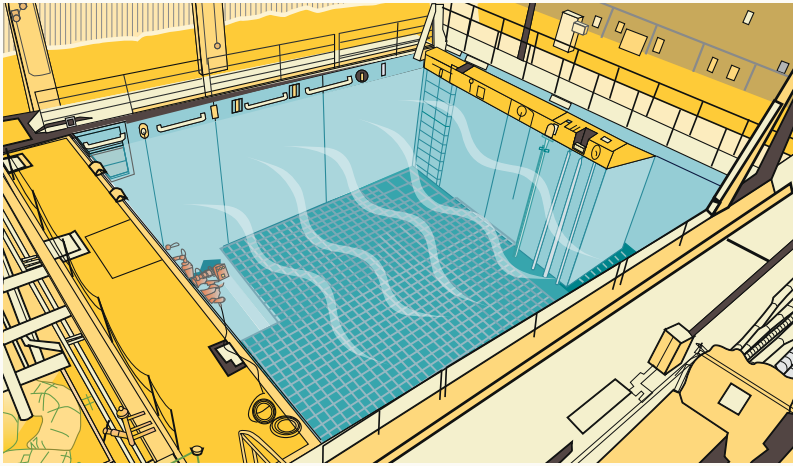
As of June 2015, there were 74 licensed ISFSIs in the United States (see Figure 38: Licensed and Operating Independent Spent Fuel Storage Installations by State). The NRC authorizes storage of spent fuel at an ISFSI by one of two licensing options: site-specific licensing or general licensing.

Figure 36. Spent Fuel Generation and Storage After Use

1 A nuclear reactor is powered by enriched uranium-235 fuel. Fission (splitting of atoms) generates heat, which produces steam that turns turbines to produce electricity. A reactor rated at several hundred megawatts may contain 100 or more tons of fuel in the form of bullet-sized pellets loaded into long metal rods that are bundled together into fuel assemblies. Pressurized-water reactors (PWRs) contain between 150 and 200 fuel assemblies. Boiling-water reactors (BWRs) contain between 370 and 800 fuel assemblies.



2 After 5–6 years, spent fuel assemblies—typically 14 feet (4.3 meters) long and containing nearly 200 fuel rods for PWRs and 80–100 fuel rods for BWRs—are removed from the reactor and allowed to cool in storage pools. At this point, the 900-pound (409-kilogram) assemblies contain only about one-fifth the original amount of uranium-235.



3 Commercial light-water nuclear reactors store spent radioactive fuel in a steel-lined, seismically designed concrete pool under about 40 feet (12.2 meters) of water that provides shielding from radiation. Water pumps supply continuously flowing water to cool the spent fuel. Extra water for the pool is provided by other pumps that can be powered from an onsite emergency diesel generator. Support features, such as water-level monitors and radiation detectors, are also in the pool. Spent fuel is stored in the pool until it is transferred to dry casks on site (as shown in Figure 37) or transported off site for interim storage or disposal.

Figure 37. Dry Storage of Spent Nuclear Fuel

At nuclear reactors across the country, spent fuel is kept on site, typically above ground, in systems basically similar to the ones shown here. The NRC reviews and approves the designs of these spent fuel storage systems before they can be used.

1 Once the spent fuel has sufficiently cooled, it is loaded into special canisters that are designed to hold nuclear fuel assemblies. Water and air are removed. The canister is filled with inert gas, welded shut, and rigorously tested for leaks. It is then placed in a cask for storage or transportation. The dry casks are then loaded onto concrete pads.

2 The canisters can also be stored in aboveground concrete bunkers, each of which is about the size of a one-car garage.

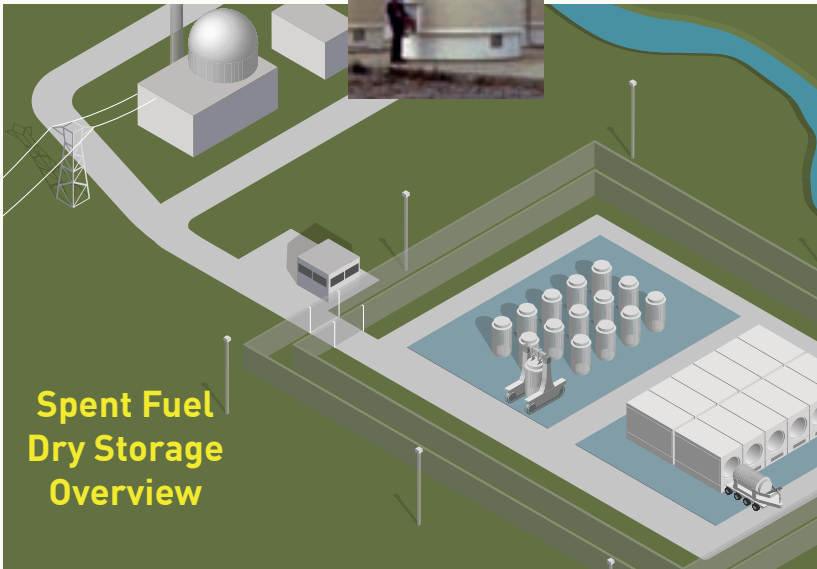
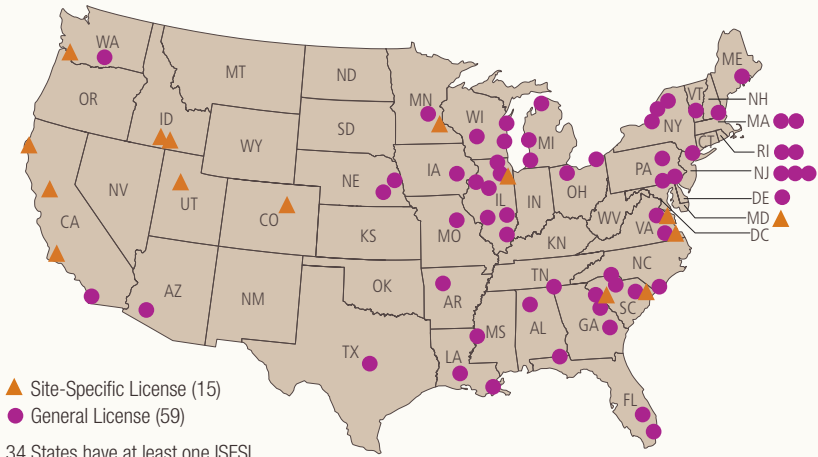


Figure 38. Licensed and Operating Independent Spent Fuel Storage Installations by State



ALABAMA	ILLINOIS	MISSISSIPPI	PENNSYLVANIA
● Browns Ferry	● Braidwood	● Grand Gulf	● Limerick
● Farley	● Byron	MISSOURI	● Susquehanna
ARIZONA	▲ GE Hitachi Morris (Wet)	● Callaway	● Peach Bottom
● Palo Verde	● Dresden	NEBRASKA	● Beaver Valley
ARKANSAS	● La Salle	● Cooper	SOUTH CAROLINA
● Arkansas Nuclear	● Quad Cities	● Ft. Calhoun	● Oconee
CALIFORNIA	● Zion	NEW HAMPSHIRE	● Robinson
▲ Diablo Canyon	IOWA	● Seabrook	● Catawba
▲ Rancho Seco	● Duane Arnold	NEW JERSEY	TENNESSEE
● San Onofre	LOUISIANA	● Hope Creek	● Sequoyah
▲ Humboldt Bay	● River Bend	● Salem	TEXAS
COLORADO	● Waterford	● Oyster Creek	● Comanche Peak
▲ Fort St. Vrain	MAINE	NEW YORK	UTAH
CONNECTICUT	● Maine Yankee	● Indian Point	▲ Private Fuel Storage
● Haddam Neck	MARYLAND	● FitzPatrick	VERMONT
● Millstone	▲ Calvert Cliffs	● Ginna	● Vermont Yankee
FLORIDA	MASSACHUSETTS	● Nine Mile Point	VIRGINIA
● St. Lucie	● Yankee Rowe	NORTH CAROLINA	● ▲ Surry
● Turkey Point	● Pilgrim	● Brunswick	● ▲ North Anna
GEORGIA	MICHIGAN	● McGuire	WASHINGTON
● Hatch	● Big Rock Point	OHIO	● Columbia
● Vogtle	● Palisades	● Davis-Besse	WISCONSIN
IDAHO	● Cook	● Perry	● Point Beach
▲ DOE: TMI-2 (Fuel Debris)	● Fermi	OREGON	● Kewaunee
▲ DOE: Idaho Spent Fuel Facility	MINNESOTA	▲ Trojan	● LaCrosse
	● Monticello		
	▲ Prairie Island		

Data as of July 2015
NRC-abbreviated site names listed

The NRC grants site-specific licenses after a safety review of the technical requirements and operating conditions for the ISFSI. The license term can be up to 40 years and can be renewed for up to another 40 years. A general license from the NRC authorizes a nuclear power reactor licensee to store spent fuel on site in dry storage casks. Under the general license, the authority to use a storage cask is tied to the cask's certificate of compliance issued through a rulemaking. The NRC has issued certificates to several dry storage cask designs. The agency issues initial and renewed certificates for terms not to exceed 40 years.

Public Involvement

The public can participate in decisions about spent fuel storage, as it can in many licensing and rulemaking decisions. The Atomic Energy Act of 1954, as amended, and the NRC's own regulations call for public hearings about site-specific licensing actions and allow the public to comment on certificate of compliance rulemakings. Members of the public may also file petitions for rulemaking. Additional information on ISFSIs is available on the NRC's Web site (see the Web Link Index).

Spent Nuclear Fuel Disposal

The current U.S. policy governing permanent disposal of high-level radioactive waste is defined by the Nuclear Waste Policy Act of 1982, as amended, and the Energy Policy Act of 1992. These acts specify that high-level radioactive waste will be disposed of underground in a deep geologic repository.

In the autumn of 2014, the NRC published NUREG-2157, "Generic Environmental Impact Statement for Continued Storage of Spent Nuclear Fuel". This report examined the environmental impacts of storing spent fuel for several decades until a repository is available. Because the timing of repository availability is uncertain, the report looked at potential impacts over three possible timeframes: a short-term timeframe, which includes 60 years of continued storage after the end of a reactor's licensed life for operation; an additional 100-year timeframe (60 years plus 100 years) to address the potential for delay in repository availability; and a third, indefinite timeframe in case a repository never becomes available.

The NRC also implemented a final rule on continued storage that adopts the findings of the generic environmental impact statement into regulation. This rule is important for issuing new or renewed licenses for nuclear power plants and spent fuel storage facilities.

Transportation

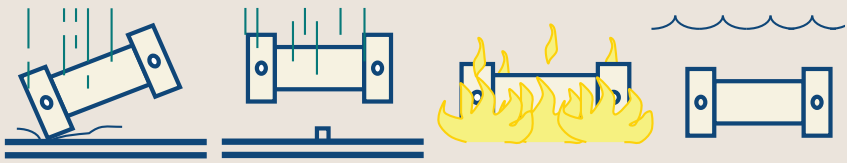
The NRC is also involved in the transportation of spent nuclear fuel. The NRC establishes safety and security requirements in collaboration with DOT, certifies transportation cask designs, and conducts inspections to ensure that requirements are being met.

Spent fuel transportation casks are designed to meet the following safety criteria under both normal and accident conditions:

- prevent the loss or dispersion of radioactive contents
- shield everything outside the cask from the radioactivity of the contents
- dissipate the heat from the contents
- prevent nuclear criticality (a self-sustaining nuclear chain reaction) from occurring inside the cask

Transportation casks must be designed to survive a sequence of tests, including a 30-foot (9-meter) drop onto an unyielding surface, a puncture test, a fully engulfing fire at 1,475 degrees Fahrenheit (802 degrees Celsius) for 30 minutes, and immersion under water. This very severe test sequence, akin to the cask striking a concrete pillar along a highway at high speed and being engulfed in a severe and long-lasting fire, and then falling into a river, simulates conditions more severe than 99 percent of vehicle accidents (see Figure 39: Ensuring Safe Spent Fuel Shipping Containers).

Figure 39. Ensuring Safe Spent Fuel Shipping Containers



The impact (free drop and puncture), fire, and water immersion tests are considered in sequence to determine their cumulative effects on a given package.

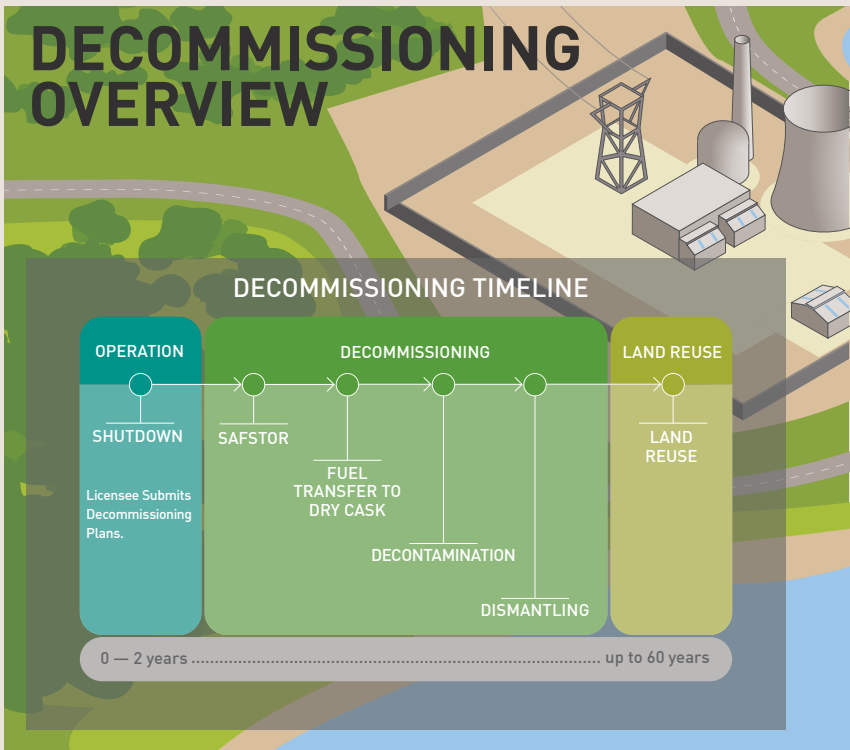
5 Radioactive Waste

To ensure the safe transportation of spent fuel and other nuclear materials, each year the NRC:

- conducts about 1,000 transportation safety inspections of fuel, reactor, and materials licensees
- reviews, evaluates, and certifies approximately 65 new, renewed, or amended transportation package design applications
- conducts 16 inspections of cask vendors and manufacturers to ensure the quality of dry cask design and fabrication
- reviews and evaluates approximately 150 license applications for the export or import of nuclear materials

Additional information on materials transportation is available on the NRC's Web site (see the Web Link Index).

Figure 40. Decommissioning Overview Timeline



Decommissioning

Decommissioning is the safe removal of a nuclear facility from service and the reduction of residual radioactivity to a level that permits release of the property and termination of the license. NRC rules establish site-release criteria and provide for unrestricted and (under certain conditions) restricted release of a site. The NRC also requires all licensees to maintain financial assurance that funds will be available when needed for decommissioning.

The NRC regulates the decontamination and decommissioning of nuclear power plants, materials and fuel cycle facilities, research and test reactors, and uranium recovery facilities, with the ultimate goal of license termination.

For commercial power reactors that have ceased operations, the decommissioning process may take up to 60 years. This may include extended periods of inactivity (called SAFSTOR), during which residual radioactivity is allowed to decay, making eventual cleanup easier and more efficient. A facility is said to be in “DECON” when active demolition and decontamination is underway. Active decommissioning of a nuclear power plant takes about 10 years on average.

During 2013, the Kewaunee, Crystal River, and San Onofre 2 and 3 nuclear power reactors permanently shut down and entered the decommissioning process. The Vermont Yankee reactor ceased operations at the end of 2014.

The NRC terminates approximately 150 materials licenses each year. Most of these license terminations are routine, and the sites require little or no cleanup to meet the NRC’s criteria for unrestricted access. The decommissioning program focuses on the termination of licenses for sites involving more complex decommissioning activities. (See Figures 40: Decommissioning Overview Timeline, 41: Power Reactors Decommissioning Status, and 42: Locations of NRC-Regulated Sites Undergoing Decommissioning.)

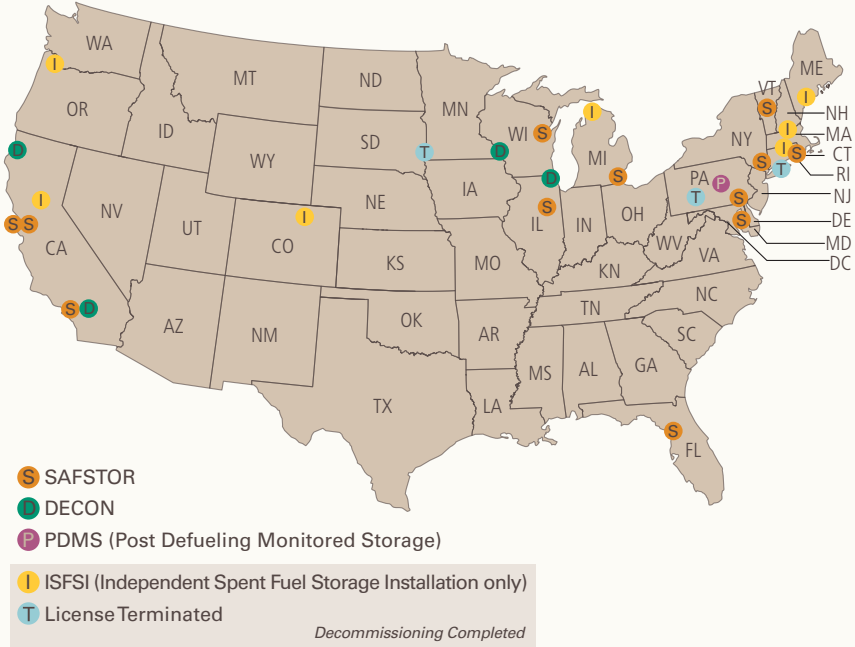
As of June 2015, the following facilities were undergoing decommissioning under NRC jurisdiction (see Figure 43: Facilities Undergoing Decommissioning under NRC Jurisdiction):

- 19 nuclear power and early demonstration reactors
- 15 complex material sites
- five RTRs
- two fuel cycle facilities
- 11 uranium recovery facilities

See Appendices C, K, and R for licensees undergoing decommissioning.

The “Status of the Decommissioning Program 2014 Annual Report” contains additional information on the decommissioning programs of the NRC and Agreement States. More information is on the NRC’s Web site (see the Web Link Index).

Figure 41. Power Reactors Decommissioning Status



CALIFORNIA

- S GE EVESR
- S GE VBWR
- D Humboldt Bay 3
- I Rancho Seco
- S San Onofre 1
- D San Onofre 2 and 3

COLORADO

- I Fort St. Vrain
(DOE License)

CONNECTICUT

- S Millstone 1
- I Haddam Neck

FLORIDA

- S Crystal River 3

ILLINOIS

- S Dresden 1
- D Zion 1 and 2

MARYLAND

- S N.S. Savannah

MASSACHUSETTS

- I Yankee Rowe

MAINE

- I Maine Yankee

MICHIGAN

- S Fermi 1
- I Big Rock Point

NEW YORK

- S Indian Point 1
- T Shoreham

OREGON

- I Trojan

PENNSYLVANIA

- T Saxton
- S Peach Bottom 1
- P Three Mile Island 2

SOUTH DAKOTA

- T Pathfinder

VERMONT

- S Vermont Yankee

WISCONSIN

- D LaCrosse
- S Kewaunee

Notes: GE Bonus, Hallam, and Piqua decommissioned reactor sites are part of the DOE Nuclear Legacy. For more information, visit DOE's Legacy Management web site at energy.gov/lm/sites/lm-sites. CVTR, Elk River, and Shippingport decommissioned reactor sites were either decommissioned before the formation of the NRC or were not licensed by the NRC.

NRC-abbreviated reactor names listed

Figure 42. Locations of NRC-Regulated Sites Undergoing Decommissioning

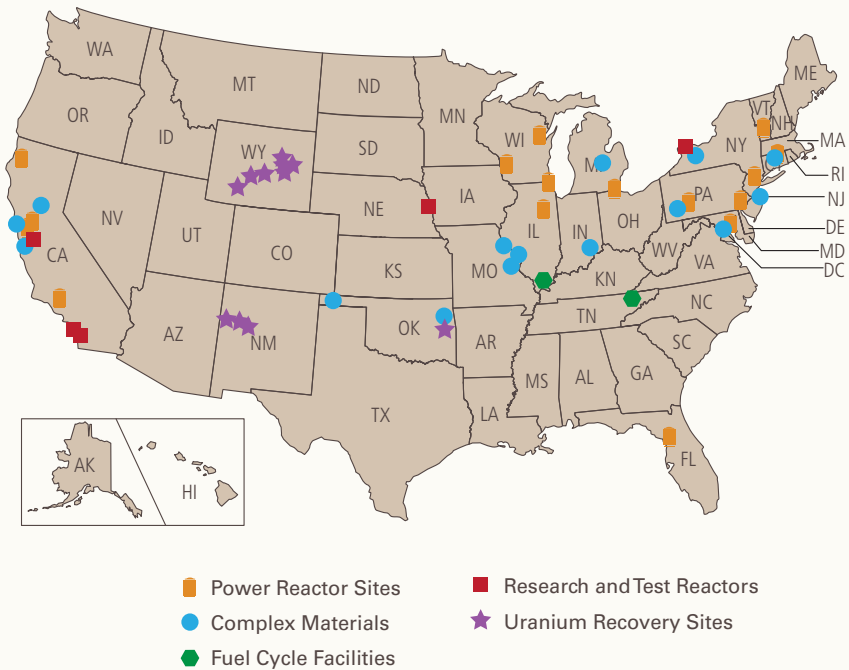


Figure 43. Facilities Undergoing Decommissioning under NRC Jurisdiction







Security and Emergency Preparedness

Overview

Nuclear security is a high priority for the NRC. For decades, effective NRC regulation and strong partnerships with Federal, State, Tribal, and local authorities have ensured effective implementation of security programs at nuclear facilities and radioactive materials sites across the country. In fact, nuclear power plants are likely the best-protected private sector facilities in the United States. However, given today's threat environment, the agency recognizes the need for continued vigilance and high levels of security.

In recent years, the NRC has made many enhancements to the security of nuclear power plants. Because nuclear power plants are inherently robust structures, these additional security upgrades (see Figure 44: Security Components) largely focus on:

- well-trained and armed security officers
- high-tech equipment and physical barriers
- greater standoff distances for vehicle checks
- intrusion detection and surveillance systems
- tested emergency preparedness and response plans
- restrictive site-access control, including background checks and fingerprinting of workers

The NRC also coordinates and shares threat information with DHS, the Federal Bureau of Investigation, intelligence agencies, the U.S. Department of Defense, and local law enforcement.

Facility Security

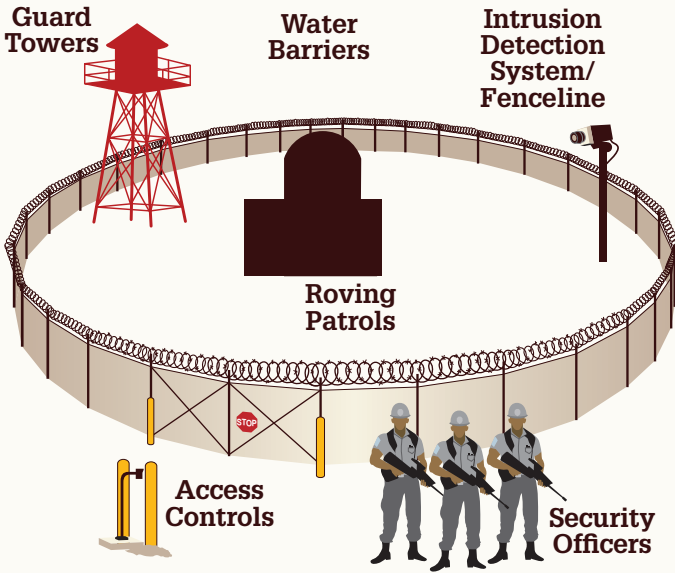
Under NRC regulations, nuclear power plants and fuel facilities that handle highly enriched uranium must be able to defend successfully against a set of threats the agency calls the design-basis threat (DBT). This includes threats to a plant's or facility's physical security, personnel security, and cyber security. The NRC does not make details of the DBT public because of security concerns. However, the agency continuously evaluates this set of threats against real-world intelligence to ensure the DBT remains current. To test the adequacy of a facility's defenses against the DBT, the NRC conducts rigorous "force-on-force" inspections at each facility every 3 years.



See Glossary for definitions of the categories of special nuclear material.

During these inspections, a highly trained mock adversary force "attacks" a nuclear facility. Beginning in 2004, the NRC made these exercises more realistic, more challenging, and more frequent.

Figure 44. Security Components



Protecting nuclear facilities requires all of the security features to come together and work as one.



Licenses are authorized to use deadly force while protecting nuclear facilities from intruders.

The NRC spends about 16,000 hours a year scrutinizing security at nuclear power plants and fuel fabrication facilities, including 8,000 hours of force-on-force inspections. Publicly available portions of security-related inspection reports are on the NRC's Web site (see the Web Link Index). For security reasons, inspection reports are not available for the NRC-licensed fuel facilities that handle highly enriched uranium.

Cyber Security

Nuclear facilities use digital and analog systems to monitor, control, and run various types of equipment, as well as to obtain and store vital information. Protecting these systems and the information they contain from sabotage or malicious use is called cyber security. The reactor control systems of nuclear plants are isolated from the Internet, but for added security, all nuclear power plants licensed by the NRC must have a cyber security program.

In 2013, the NRC began regular cyber security inspections of nuclear power plants under new regulations designed to guard against the cyber threat. The experience that the NRC gained in developing the cyber security requirements for nuclear power plants provided a basis for developing similar cyber security requirements for nonreactor licensees and nuclear facilities.

The NRC's cyber security team includes technology and threat experts who constantly evaluate and identify emerging cyber-related issues that could possibly



The NRC's cyber security staff includes technology and threat experts who constantly evaluate and identify emerging cyber-related issues.

endanger plant systems. The team also makes recommendations to other NRC offices and programs on cyber security issues.

In October 2014, the NRC joined other independent regulatory agencies to create a Cyber Security Forum for Independent and Executive Branch Regulators. The forum aims to “increase the overall effectiveness and consistency of regulatory authorities’ cyber security efforts pertaining to U.S. Critical Infrastructure, much of which is operated by industry and overseen by a number of federal regulatory authorities.”

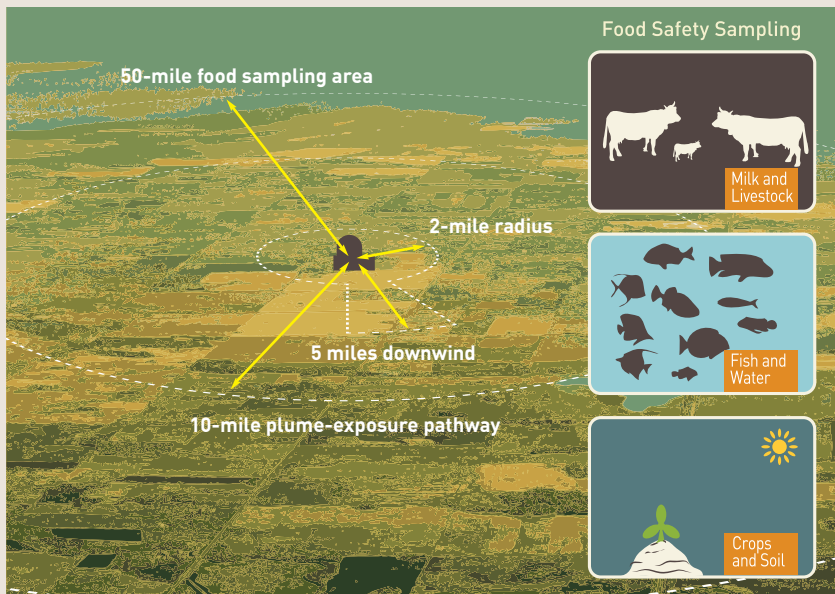
Materials Security

Radioactive materials must be secured to reduce the possibility that terrorists could use them to make a radiological dispersal device, sometimes called an RDD or a dirty bomb. The NRC has established rules to provide the requirements for the physical protection of certain types and quantities of radioactive material. Additionally, the NRC works with the Agreement States, other Federal agencies, the IAEA, and licensees to protect radioactive materials from theft and malicious use. In 2009, the NRC deployed the NSTS, designed to track the most risk-sensitive radioactive materials in sources. Other improvements allow U.S. Customs and Border Protection agents to promptly validate whether radioactive materials coming into the United States are properly licensed by the NRC or an Agreement State. In addition, the agency improved and upgraded the joint NRC-DOE database tracking the movement and location of certain forms and quantities of special nuclear material.

Emergency Preparedness

Operators of nuclear facilities are required to develop and maintain effective emergency plans and procedures to protect the public in the unlikely event of an emergency. Emergency preparedness plans include public information, preparations for evacuation, instructions for sheltering, and other actions to protect the residents near nuclear power plants in the event of a serious incident. The NRC includes emergency preparedness in its inspections and monitors performance indicators associated with emergency preparedness.

Nuclear power plant operators must conduct full-scale exercises with the NRC, the Federal Emergency Management Agency (FEMA), and State and local officials at least once every 2 years. Some of these exercises include security- and terrorism-based scenarios. These exercises test and maintain the skills of the emergency responders and identify any areas that need to be addressed. Nuclear power plant operators also conduct their own emergency response drills.

Figure 45. Emergency Planning Zones

Note: A 2-mile ring around the plant is identified for evacuation, along with a 5-mile zone downwind of the projected release path.

Emergency Planning Zones

The NRC defines two emergency planning zones (EPZs) around each nuclear power plant. The exact size and configuration of the zones vary from plant to plant, based on local emergency response needs and capabilities, population, land characteristics, access routes, and jurisdictional boundaries. The zone boundaries are flexible, and the NRC may expand these zones during an emergency if circumstances warrant. For a typical emergency planning zone around a nuclear plant, (see Figure 45: Emergency Planning Zones).

The two types of EPZs are the plume-exposure pathway and ingestion pathway. The plume-exposure pathway, covers a radius of about 10 miles (16 kilometers) from the plant and is the area of greatest concern about the public's exposure to and inhalation of airborne radioactive contamination. Research has shown the most significant impacts of an accident would be expected in the immediate vicinity of a plant, and any initial protective actions, such as evacuations or sheltering in place, should be focused there.

The ingestion pathway, or food safety sampling area. This zone extends to a radius of about 50 miles (80 kilometers) from the plant and is the area of greatest concern about the ingestion of food and liquid contaminated by radioactivity.

Protective Actions

During an actual nuclear power plant accident, the NRC would use radiation-dose projection models to predict the nature and extent of a radiation release. The dose calculations would account for weather conditions to project the extent of radiation exposure to the nearby population. The NRC would confer with appropriate State and county governments on its assessment results. Plant personnel would also provide assessments. State and local officials in communities within the EPZ have detailed plans to protect the public during a radiation release. These officials make their protective action decisions, including decisions to order evacuations, based on these and other assessments.

Evacuation, Sheltering, and the Use of Potassium Iodide

Protective actions considered for a radiological emergency include evacuation, sheltering, and the preventive use of potassium iodide (KI) supplements to protect the thyroid from radioactive iodine, which can cause thyroid cancer.

Under certain conditions, it may be preferable to evacuate the public away from further exposure to radioactive material. However, a complete evacuation of the 10-mile (16-kilometer) zone around a nuclear power plant is not likely to be needed in most cases. The release of radioactive material from a plant during a major incident would move with the wind, not in all directions surrounding the plant. The release would also expand and become less concentrated as it travels away from a plant. For these reasons, evacuations can be planned based on the anticipated path of the release.

Under some conditions, people may be instructed to take shelter in their homes, schools, or office buildings. Depending on the type of structure, sheltering can significantly reduce someone's dose when compared to staying outside. In certain situations, KI may be used as a supplement to sheltering. It may be appropriate to shelter when the release of radioactive material is known to be short term or is controlled by the nuclear power plant operator.

The risk of an offsite radiological release is significantly lower and the types of possible accidents significantly fewer at a nuclear power reactor that has

permanently ceased operations and removed fuel from the reactor vessel. Nuclear power plants that have begun decommissioning may therefore apply for exemptions from these FEMA and NRC emergency planning requirements. Once the exemptions are granted, State and local agencies may apply their comprehensive emergency plans—known as “all hazard” plans—to respond to incidents at the plant.

Additional information on emergency preparedness is available on the NRC’s Web site (see Web Link Index).

Incident Response

Quick communication among the NRC, other Federal and State agencies, and the nuclear industry is critical when responding to any incident. The NRC staff supports several Federal incident response centers where officials can coordinate assessments of event-related information. The NRC Headquarters Operations Center, located in the agency’s headquarters in Rockville, MD, is staffed around the clock to disseminate information and coordinate response activities. The NRC also reviews intelligence reports and assesses suspicious activity to keep licensees and other agencies up to date on current threats.

The NRC works within the National Response Framework to respond to events. The framework guides the Nation in its response to complex events that might involve a variety of agencies and hazards. Under this framework, the NRC retains its independent authority and ability to respond to emergencies involving NRC-licensed facilities or materials. The NRC may request support from DHS in responding to an emergency at an NRC-licensed facility or involving NRC-licensed materials. DHS may lead and manage the overall Federal



The NRC’s Headquarters Operations Center is located in the agency’s Three White Flint North headquarters building in Rockville, MD.

response to an event, according to Homeland Security Presidential Directive 5, “Management of Domestic Incidents.” In this case, the NRC would provide technical expertise and help share information among the various organizations and licensees.

In response to an incident involving possible radiation releases, the NRC activates its incident response program at its Headquarters Operations Center and one of its four Regional Incident Response Centers. Teams of specialists at these centers evaluate event information, independently assess the potential impact on public health and safety, and evaluate possible recovery strategies. The NRC staff provides expert consultation, support, and assistance to State and local public safety officials and keeps the public informed of agency actions. Meanwhile, other NRC experts evaluate the effectiveness of protective actions the licensee has recommended to State and local officials. If needed, the NRC will dispatch a team of technical experts from the responsible regional office to the site. This team would assist the NRC’s resident inspectors, who work at the plant. The Headquarters Operations Center would continue to provide around-the-clock communications, logistical support, and technical analysis throughout the response.



During an exercise in the agency’s Headquarters Operations Center, NRC Protective Measures team members look at simulated projected radiation doses to the public.

Emergency Classifications

Emergencies at nuclear facilities are classified according to the risk posed to the public. These classifications help guide first responders on the actions necessary to protect the population near the site.

Nuclear power plants use these four emergency classifications (see Figure 46: Emergency Classifications for Nuclear Reactor Events, 2014):

Notification of Unusual Event: Events that indicate a potential degradation in the level of safety of the plant are in progress or have occurred. No release of radioactive material requiring offsite response or monitoring is expected unless further degradation occurs.

Alert: Events that involve an actual or potential substantial degradation in the level of plant safety are in progress or have occurred. Any releases of radioactive material are expected to be limited to a small fraction of the limits set forth by the U.S. Environmental Protection Agency (EPA).

Site Area Emergency: Events that may result in actual or likely major failures of plant functions needed to protect the public are in progress or have occurred. Any releases of radioactive material are not expected to exceed the limits set forth by the EPA except near the site boundary.

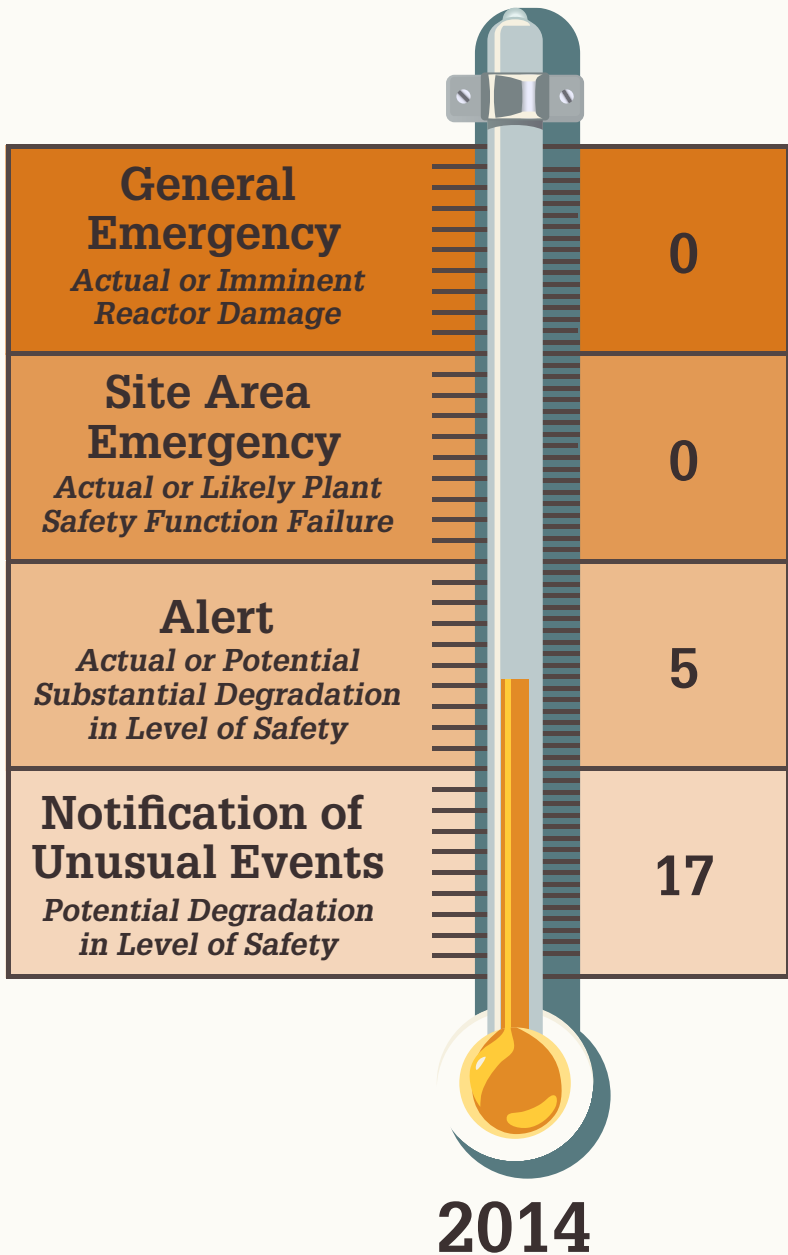
General Emergency: Events that involve actual or imminent substantial core damage or melting of reactor fuel with the potential for loss of containment integrity are in progress or have occurred. Radioactive releases can be expected to exceed the limits set forth by the EPA for more than the immediate site area.

Nuclear materials and fuel cycle facility licensees use these emergency classifications:

Alert: Events that could lead to a release of radioactive materials are in progress or have occurred. The release is not expected to require a response by an offsite response organization to protect residents near the site.

Site Area Emergency: Events that could lead to a significant release of radioactive materials are in progress or have occurred. The release could require a response by offsite response organizations to protect residents near the site.

Figure 46. Emergency Classifications for Nuclear Reactor Events, 2014

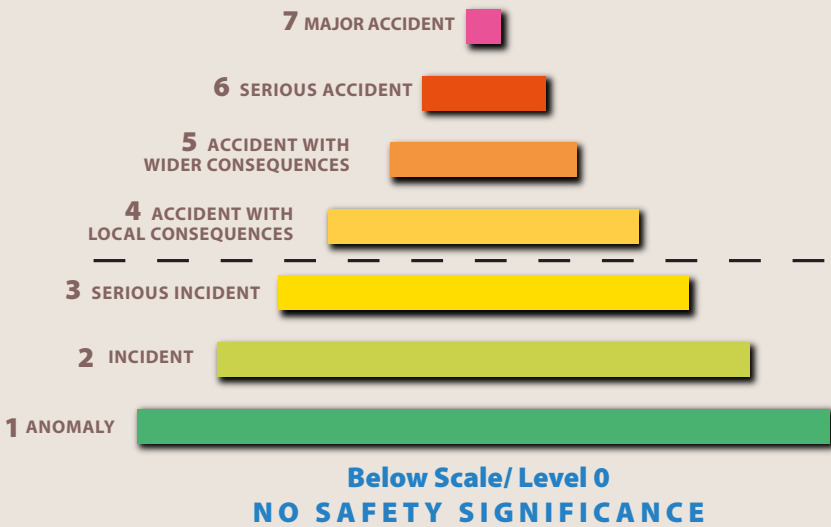


International Emergency Classifications

The IAEA uses the International Nuclear and Radiological Event Scale (INES) as a tool for promptly and consistently communicating to the public the safety significance of reported nuclear and radiological incidents and accidents worldwide (see Figure 47: The International Nuclear and Radiological Event Scale).

The scale can be applied to any event associated with nuclear facilities, as well as to the transport, storage, and use of radioactive material and radiation sources. Licensees are not required to classify events or provide offsite notifications using the INES scale. But the NRC has a commitment to transmit to the IAEA an INES-based rating for an applicable event occurring in the United States rated at Level 2 or above, or events attracting international public interest.

Figure 47. The International Nuclear and Radiological Event Scale



INES events are classified on the scale at 7 levels. Levels 1–3 are called “incidents” and Levels 4–7 “accidents.” The scale is designed so that the severity of an event is about 10 times greater for each increase in level on the scale. Events without safety significance are called “deviations” and are classified as Below Scale or at Level 0.



An NRC official, standing in front of a massive map projection, gives a presentation to public participants of a 2015 Headquarters Operations Center tour to understand the role and responsibilities of the NRC during an incident response.





Appendices

Abbreviations

ABWR	advanced boiling-water reactor	EPZ	emergency planning zone
AC	Allis Chalmers	ERO	emergency response organization
ACRS	Advisory Committee on Reactor Safeguards	ESBWR	Economic Simplified Boiling-Water Reactor
ADAMS	Agencywide Documents Access and Management System	ESP	early site permit
ADR	Alternative Dispute Resolution	EVESR	ESADA (Empire States Atomic Development Associates)
AEC	Atomic Energy Commission (U.S.)		Vallecos Experimental Superheat Reactor
AEP	American Electric Power Company	Exp. DATE	expiration date of operating license
AGN	solid homogeneous core (Aerojet-General Nucleonics)	FB	Federal Bureau of Investigation (U.S.)
ASLB	Atomic Safety and Licensing Board	FBR	fast breeder reactor
B&R	Burns & Roe	FEMA	Federal Emergency Management Agency
B&W	Babcock & Wilcox	FERC	Federal Energy Regulatory Commission
BALD	Baldwin Associates	FLUR	Fluor Pioneer
BECH	Bechtel	FOIA	Freedom of Information Act
BRRT	Brown & Root	FTE	full-time equivalent
BWR	boiling-water reactor	FW	Foster Wheeler
CE	Combustion Engineering	FY	fiscal year
CEUS	Central and Eastern United States Seismic Source	G&H	Gibbs & Hill
CFR	<i>Code of Federal Regulations</i>	GA	General Atomic
CO	Commission order	GCR	gas-cooled reactor
Co	company	GE	General Electric
COGGE	Committee on Geological and geotechnical Engineering	GEH	General Electric-Hitachi Nuclear Energy
COL	combined license	GEIS	generic environmental impact statement
COMM. OP.	date of commercial operation	GETR	General Electric Test Reactor
CON TYPE	containment type	GHDR	Gibbs & Hill & Durham & Richardson
DRYAMB	dry, ambient pressure	GIL	Gilbert Associates
DRYSUB	dry, subatmospheric	GL	general license
ICECND	wet, ice condenser	GPC	Georgia Power Company
MARK 1	wet, Mark I	GWe	gigawatt(s) electrical
MARK 2	wet, Mark II	GWh	gigawatts hours
MARK 3	wet, Mark III	HTG	high-temperature gas (reactor)
CP	civil penalty	HWR	heavy-water reactor
CP ISSUED	date of construction permit issuance	IAEA	International Atomic Energy Agency
CRGR	Committee To Review Generic Requirements	INES	International Nuclear Event Scale
CVP	civil penalties	ISFSI	independent spent fuel storage installation
CVTR	Carolas-Virginia Tube Reactor	ISR	in situ recovery
CWE	Commonwealth Edison Company	KAIS	Kaiser Engineers
CY	calendar year	KI	potassium iodide
DANI	Daniel International	kW	kilowatt(s)
DBDB	Duke & Bechtel	LES	Louisiana Energy Services
DBT	design-basis threat	LLP	B&W lowered loop
DC	design certification	LLW	low-level radioactive waste
DHS	Department of Homeland Security (U.S.)	LMFB	liquid metal fast breeder (reactor)
DOE	Department of Energy (U.S.)	LR ISSUED	license renewal issued
DOT	Department of Transportation (U.S.)	LWGR	light-water-cooled graphite moderated reactor
DUKE	Duke Power Company	MOX	mixed-oxide
EBSCO	Ebasco	MW	megawatt(s)
EIA	Energy Information Administration (DOE)	MWe	megawatt(s) electrical
EIS	environmental impact statement	MWh	megawatt-hour(s)
EPA	Environmental Protection Agency (U.S.)		
EPR	Evolutionary Power Reactor		

MWt	megawatt(s) thermal	RLP	B&W raised loop
NIAG	Niagara Mohawk Power Corporation	ROP	Reactor Oversight Process
NEA	Nuclear Energy Agency	RTR	research and test reactors
NMMSS	Nuclear Material Management and Safeguards System	S&L	Sargent & Lundy
NOV	notice(s) of violation	S&W	Stone & Webster
NOVF	notice(s) of violation associated with inspection findings	SAMG	Severe Accident Management Guidance
NOVSL	notice(s) of violation for severity level	SCF	sodium-cooled fast (reactor)
NRC	Nuclear Regulatory Commission (U.S.)	SDP	significance determination process
NSP	Northern States Power Company	SGEC	architect for Vogtle
NSSS	nuclear steam system supplier and design type	SI	système internationale (d'unités) (International System of Units)
GE 2	GE Type 2	SL	severity level
GE 3	GE Type 3	SOARCA	State-of-the-Art Consequence Analysis
GE 4	GE Type 4	SSI	Southern Services Incorporated
GE 5	GE Type 5	STARS	Strategic Teaming and Resource Sharing Group
GE 6	GE Type 6	STP	South Texas Project
WEST 2LP	Westinghouse Two-Loop	TMI-2	Three Mile Island Unit 2
WEST 3LP	Westinghouse Three-Loop	Sv	sievert
WEST 4LP	Westinghouse Four-Loop	TRACE	reactor systems codes
NSTS	National Source Tracking System	TRIGA	Training Reactor and Isotopes Production, General Atomics
OECD	Organisation for Economic Co-operation and Development	TVA	Tennessee Valley Authority
OL	operating license	UE&C	United Engineers & Constructors
OL ISSUED	date of latest full power operating license	USEC	U.S. Enrichment Corporation
PG&E	Pacific Gas & Electric Company	US-APWR	United States [version of] Advanced Pressurized-Water Reactor
PRA	probabilistic risk assessment	VBWR	Vallecitos Boiling-Water Reactor
PRIS	Power Reactor Information System	WDCO	Westinghouse Development Corporation
PSEG	Public Service Electric and Gas Company	WEST	Westinghouse Electric
PWR	pressurized-water reactor	WNA	World Nuclear Association
RIC	Regulatory Information Conference		

Alabama	AL	Kentucky	KY	Ohio	OH
Alaska	AK	Louisiana	LA	Oklahoma	OK
Arizona	AZ	Maine	ME	Oregon	OR
Arkansas	AR	Maryland	MD	Pennsylvania	PA
California	CA	Massachusetts	MA	Puerto Rico	PR
Colorado	CO	Michigan	MI	Rhode Island	RI
Connecticut	CT	Minnesota	MN	South Carolina	SC
Delaware	DE	Mississippi	MS	South Dakota	SD
District of Columbia	DC	Missouri	MO	Tennessee	TN
Florida	FL	Montana	MT	Texas	TX
Georgia	GA	Nebraska	NE	Utah	UT
Guam	GU	Nevada	NV	Vermont	VT
Hawaii	HI	New Hampshire	NH	Virgin Islands	VI
Idaho	ID	New Jersey	NJ	Virginia	VA
Illinois	IL	New Mexico	NM	Washington	WA
Indiana	IN	New York	NY	West Virginia	WV
Iowa	IA	North Carolina	NC	Wisconsin	WI
Kansas	KS	North Dakota	ND	Wyoming	WY

APPENDIX A
U.S. Commercial Nuclear Power Reactors
Operating Reactors

Plant Name, Unit Number Licensee Location Docket Number NRC Web Page Address	NRC Region	Con Type NSSS Architect Engineer Constructor	Licensed MWt	CP Issued OL Issued Comm. Op. LR Issued Exp. Date	2009– 2014* Capacity Factor (Percent)
Arkansas Nuclear One, Unit 1 Entergy Operations, Inc. London, AR (6 miles NW of Russellville, AR) 05000313 www.nrc.gov/info-finder/reactor/ano1.html	IV	PWR-DRYAMB	2,568	12/06/1968	99
		B&W LLP		05/21/1974	90
		BECH		12/19/1974	87
		BECH		06/20/2001	102
				05/20/2034	56
				98	
Arkansas Nuclear One, Unit 2 Entergy Operations, Inc. London, AR (6 miles NW of Russellville, AR) 05000368 www.nrc.gov/info-finder/reactor/ano2.html	IV	PWR-DRYAMB	3,026	12/06/1972	90
		CE		09/01/1978	97
		BECH		03/26/1980	90
		BECH		06/30/2005	93
				07/17/2038	91
				85	
Beaver Valley Power Station, Unit 1 FirstEnergy Nuclear Operating Co. Shippingport, PA (17 miles W of McCandless, PA) 05000334 www.nrc.gov/info-finder/reactor/bv1.html	I	PWR-DRYAMB	2,900	06/26/1970	92
		WEST 3LP		07/02/1976	91
		S&W		10/01/1976	101
		S&W		11/05/2009	92
				01/29/2036	86
				86	
Beaver Valley Power Station, Unit 2 FirstEnergy Nuclear Operating Co. Shippingport, PA (17 miles W of McCandless, PA) 05000412 www.nrc.gov/info-finder/reactor/bv2.html	I	PWR-DRYAMB	2,900	05/03/1974	87
		WEST 3LP		08/14/1987	84
		S&W		11/17/1987	102
		S&W		11/05/2009	91
				05/27/2047	97
				98	
Braidwood Station, Unit 1 Exelon Generation Co., LLC Braceville, IL (20 miles SW of Joliet, IL) 05000456 www.nrc.gov/info-finder/reactor/brai1.html	III	PWR-DRYAMB	3,645	12/31/1975	95
		WEST 4LP		07/02/1987	89
		S&L		07/29/1988	101
		CWE		N/A	91
				10/17/2026	95
				103	
Braidwood Station, Unit 2 Exelon Generation Co., LLC Braceville, IL (20 miles SW of Joliet, IL) 05000457 www.nrc.gov/info-finder/reactor/brai2.html	III	PWR-DRYAMB	3,645	12/31/1975	93
		WEST 4LP		05/20/1988	99
		S&L		10/17/1988	93
		CWE		N/A	93
				12/18/2027	98
				96	
Browns Ferry Nuclear Plant, Unit 1 Tennessee Valley Authority Athens (Limestone County), AL (32 miles W of Huntsville, AL) 05000259 www.nrc.gov/info-finder/reactor/bf1.html	II	BWR-MARK 1	3,458	05/10/1967	94
		GE 4		12/20/1973	86
		TVA		08/01/1974	91
		TVA		05/04/2006	88
				12/20/2033	94
				90	

APPENDIX A

U.S. Commercial Nuclear Power Reactors (continued) Operating Reactors

Plant Name, Unit Number Licensee Location Docket Number NRC Web Page Address	NRC Region	Con Type NSSS Architect Engineer Constructor	Licensed MWt	CP Issued OL Issued Comm. Op. LR Issued Exp. Date	2009– 2014* Capacity Factor (Percent)
Browns Ferry Nuclear Plant, Unit 2 Tennessee Valley Authority Athens (Limestone County), AL (32 miles W of Huntsville, AL) 05000260 www.nrc.gov/info-finder/reactor/bf2.html	II	BWR-MARK 1	3,458	05/10/1967	94
		GE 4		06/28/1974	91
		TVA		03/01/1975	80
		TVA		05/04/2006	99
				06/28/2034	79
Browns Ferry Nuclear Plant, Unit 3 Tennessee Valley Authority Athens (Limestone County), AL (32 miles W of Huntsville, AL) 05000296 www.nrc.gov/info-finder/reactor/bf3.html	II	BWR-MARK 1	3,458	07/31/1968	95
		GE 4		07/02/1976	81
		TVA		03/01/1977	87
		TVA		05/04/2006	83
				07/02/2036	89
Brunswick Steam Electric Plant, Unit 1 Duke Energy Progress, Inc. Southport, NC (30 miles S of Wilmington, NC) 05000325 www.nrc.gov/info-finder/reactor/bru1.html	II	BWR-MARK 1	2,923	02/07/1970	98
		GE 4		09/08/1976	83
		UE&C		03/18/1977	100
		BRRT		06/26/2006	77
				09/08/2036	92
Brunswick Steam Electric Plant, Unit 2 Duke Energy Progress, Inc. Southport, NC (30 miles S of Wilmington, NC) 05000324 www.nrc.gov/info-finder/reactor/bru2.html	II	BWR-MARK 1	2,923	02/07/1970	80
		GE 4		12/27/1974	99
		UE&C		11/03/1975	79
		BRRT		06/26/2006	98
				12/27/2034	73
Byron Station, Unit 1 Exelon Generation Co., LLC Byron, IL (17 miles SW of Rockford, IL) 05000454 www.nrc.gov/info-finder/reactor/byro1.html	III	PWR-DRYAMB	3,645	12/31/1975	94
		WEST 4LP		02/14/1985	101
		S&L		09/16/1985	88
		CWE		N/A	88
				10/31/2024	96
Byron Station, Unit 2 Exelon Generation Co., LLC Byron, IL (17 miles SW of Rockford, IL) 05000455 www.nrc.gov/info-finder/reactor/byro2.html	III	PWR-DRYAMB	3,645	12/31/1975	102
		WEST 4LP		01/30/1987	96
		S&L		08/02/1987	93
		CWE		N/A	94
				11/06/2026	86
Callaway Plant Union Electric Co. Fulton, MO (25 miles NE of Jefferson City, MO) 05000483 www.nrc.gov/info-finder/reactor/call.html	IV	PWR-DRYAMB	3,565	04/16/1976	98
		WEST 4LP		10/18/1984	86
		BECH		12/19/1984	90
		DANI		03/06/2015	103
				10/18/2044	77
			89		

APPENDIX A
U.S. Commercial Nuclear Power Reactors (continued)
Operating Reactors

Plant Name, Unit Number Licensee Location Docket Number NRC Web Page Address	NRC Region	Con Type NSSS Architect Engineer Constructor	Licensed MWt	CP Issued OL Issued Comm. Op. LR Issued Exp. Date	2009– 2014* Capacity Factor (Percent)
Calvert Cliffs Nuclear Power Plant, Unit 1 Calvert Cliffs Nuclear Power Plant, LLC Exelon Generation Co., LLC Lusby, MD (40 miles S of Annapolis, MD) 05000317 www.nrc.gov/info-finder/reactor/calv1.html	I	PWR-DRYAMB	2,737	07/07/1969	98
		CE		07/31/1974	90
		BECH		05/08/1975	101
		BECH		03/23/2000	81
				07/31/2034	97
					91
Calvert Cliffs Nuclear Power Plant, Unit 2 Calvert Cliffs Nuclear Power Plant, LLC Exelon Generation Co., LLC Lusby, MD (40 miles S of Annapolis, MD) 05000318 www.nrc.gov/info-finder/reactor/calv2.html	I	PWR-DRYAMB	2,737	07/07/1969	93
		CE		08/13/1976	97
		BECH		04/01/1977	92
		BECH		03/23/2000	101
				08/13/2036	81
					100
Catawba Nuclear Station, Unit 1 Duke Energy Carolinas, LLC York, SC (18 miles S of Charlotte, NC) 05000413 www.nrc.gov/info-finder/reactor/cat1.html	II	PWR-ICECND	3,411	08/07/1975	91
		WEST 4LP		01/17/1985	100
		DUKE		06/29/1985	89
		DUKE		12/05/2003	89
				12/05/2043	96
					86
Catawba Nuclear Station, Unit 2 Duke Energy Carolinas, LLC York, SC (18 miles S of Charlotte, NC) 05000414 www.nrc.gov/info-finder/reactor/cat2.html	II	PWR-ICECND	3,411	08/07/1975	90
		WEST 4LP		05/15/1986	92
		DUKE		08/19/1986	101
		DUKE		12/05/2003	92
				12/05/2043	86
					100
Clinton Power Station, Unit 1 Exelon Generation Co., LLC Clinton, IL (23 miles SSE of Bloomington, IL) 05000461 www.nrc.gov/info-finder/reactor/clin.html	III	BWR-MARK 3	3,473	02/24/1976	97
		GE 6		04/17/1987	92
		S&L		11/24/1987	93
		BALD		N/A	100
				09/29/2026	82
					97
Columbia Generating Station Energy Northwest Benton County, WA (12 miles NW of Richland, WA) 05000397 www.nrc.gov/info-finder/reactor/wash2.html	IV	BWR-MARK 2	3,486	03/19/1973	67
		GE 5		04/13/1984	95
		B&R		12/13/1984	50
		BECH		05/22/2012	97
				12/20/2043	80
					98
Comanche Peak Nuclear Power Plant, Unit 1 Luminant Generation Co., LLC Glen Rose, TX (40 miles SW of Fort Worth, TX) 05000445 www.nrc.gov/info-finder/reactor/cp1.html	IV	PWR-DRYAMB	3,612	12/19/1974	100
		WEST 4LP		04/17/1990	91
		G&H		08/13/1990	91
		BRRT		N/A	98
				02/08/2030	94
					85

APPENDIX A

U.S. Commercial Nuclear Power Reactors (continued) Operating Reactors

Plant Name, Unit Number Licensee Location Docket Number NRC Web Page Address	NRC Region	Con Type NSSS Architect Engineer Constructor	Licensed MWt	CP Issued OL Issued Comm. Op. LR Issued Exp. Date	2009– 2014* Capacity Factor (Percent)
Comanche Peak Nuclear Power Plant, Unit 2 Luminant Generation Company, LLC Glen Rose, TX (40 miles SW of Fort Worth, TX) 05000446 www.nrc.gov/info-finder/reactor/cp2.html	IV	PWR-DRYAMB WEST 4LP BECH BRRT	3,612	12/19/1974	94
				04/06/1993	104
				08/03/1993	92
				N/A	91
				02/02/2033	99
Cooper Nuclear Station Nebraska Public Power District Brownville, NE (23 miles S of Nebraska City, NE) 05000298 www.nrc.gov/info-finder/reactor/cns.html	IV	BWR-MARK 1 GE 4 B&R B&R	2,419	06/04/1968	72
				01/18/1974	100
				07/01/1974	86
				11/29/2010	87
				01/18/2034	97
88					
Davis-Besse Nuclear Power Station, Unit 1 FirstEnergy Nuclear Operating Co. Oak Harbor, OH (21 miles ESE of Toledo, OH) 05000346 www.nrc.gov/info-finder/reactor/davi.html	III	PWR-DRYAMB B&W RLP BECH B&W	2,817	03/24/1971	99
				04/22/1977	66
				07/31/1978	81
				N/A	91
				04/22/2017	95
74					
Diablo Canyon Nuclear Power Plant, Unit 1 Pacific Gas & Electric Co. Avila Beach, CA (12 miles SW of San Luis Obispo, CA) 05000275 www.nrc.gov/info-finder/reactor/diab1.html	IV	PWR-DRYAMB WEST 4LP PG&E PG&E	3,411	4/23/1968	84
				11/02/1984	88
				05/07/1985	100
				N/A	84
				11/02/2024	95
87					
Diablo Canyon Nuclear Power Plant, Unit 2 Pacific Gas & Electric Co. Avila Beach, CA (12 miles SW of San Luis Obispo, CA) 05000323 www.nrc.gov/info-finder/reactor/diab2.html	IV	PWR-DRYAMB WEST 4LP PG&E PG&E	3,411	12/09/1970	84
				08/26/1985	100
				03/13/1986	89
				N/A	97
				08/26/2025	82
86					
Donald C. Cook Nuclear Plant, Unit 1 Indiana Michigan Power Co. Bridgman, MI (13 miles S of Benton Harbor, MI) 05000315 www.nrc.gov/info-finder/reactor/cook1.html	III	PWR-ICECND WEST 4LP AEP AEP	3,304	03/25/1969	3
				10/25/1974	88
				08/28/1975	87
				08/30/2005	104
				10/25/2034	78
94					
Donald C. Cook Nuclear Plant, Unit 2 Indiana Michigan Power Co. Bridgman, MI (13 miles S of Benton Harbor, MI) 05000316 www.nrc.gov/info-finder/reactor/cook2.html	III	PWR-ICECND WEST 4LP AEP AEP	3,468	03/25/1969	87
				12/23/1977	84
				07/01/1978	104
				08/30/2005	91
				12/23/2037	85
101					

APPENDIX A
U.S. Commercial Nuclear Power Reactors (continued)
Operating Reactors

Plant Name, Unit Number Licensee Location Docket Number NRC Web Page Address	NRC Region	Con Type NSSS Architect Engineer Constructor	Licensed MWt	CP Issued OL Issued Comm. Op. LR Issued Exp. Date	2009– 2014* Capacity Factor (Percent)
Dresden Nuclear Power Station, Unit 2 Exelon Generation Co., LLC Morris, IL (25 miles SW of Joliet, IL) 05000237 www.nrc.gov/info-finder/reactor/dres2.html	III	BWR-MARK 1 GE 3 S&L UE&C	2,957	01/10/1966 02/20/1991 ^A 06/09/1970 10/28/2004 12/22/2029	91 102 95 104 85 98
Dresden Nuclear Power Station, Unit 3 Exelon Generation Co., LLC Morris, IL (25 miles SW of Joliet, IL) 05000249 www.nrc.gov/info-finder/reactor/dres3.html	III	BWR-MARK 1 GE 3 S&L UE&C	2,957	10/14/1966 01/12/1971 11/16/1971 10/28/2004 01/12/2031	97 90 99 91 89 95
Duane Arnold Energy Center NextEra Energy Duane Arnold, LLC Palo, IA (8 miles NW of Cedar Rapids, IA) 05000331 www.nrc.gov/info-finder/reactor/duan.html	III	BWR-MARK 1 GE 4 BECH BECH	1,912	06/22/1970 02/22/1974 02/01/1975 12/16/2010 02/21/2034	92 89 99 83 89 79
Edwin I. Hatch Nuclear Plant, Unit 1 Southern Nuclear Operating Co. Baxley, GA (20 miles S of Vidalia, GA) 05000321 www.nrc.gov/info-finder/reactor/hat1.html	II	BWR-MARK 1 GE 4 BECH GPC	2,804	09/30/1969 10/13/1974 12/31/1975 01/15/2002 08/06/2034	94 85 98 89 94 89
Edwin I. Hatch Nuclear Plant, Unit 2 Southern Nuclear Operating Co., Inc. Baxley, GA (20 miles S of Vidalia, GA) 05000366 www.nrc.gov/info-finder/reactor/hat2.html	II	BWR-MARK 1 GE 4 BECH GPC	2,804	12/27/1972 06/13/1978 09/05/1979 01/15/2002 06/13/2038	67 96 78 98 89 99
Fermi, Unit 2 DTE Electric Company Newport, MI (25 miles NE of Toledo, OH) 05000341 www.nrc.gov/info-finder/reactor/ferm2.html	III	BWR-MARK 1 GE 4 S&L DANI	3,486	09/26/1972 07/15/1985 01/23/1988 N/A 03/20/2025	75 80 94 54 62 82
Fort Calhoun Station, Unit 1 Omaha Public Power District Ft. Calhoun, NE (19 miles N of Omaha, NE) 05000285 www.nrc.gov/info-finder/reactor/fcs.html	IV	PWR-DRYAMB CE GHDR GHDR	1,500	06/07/1968 08/09/1973 09/26/1973 11/04/2003 08/09/2033	100 102 28 0 1 100

A: AEC issued a provisional OL on 12/22/1969, allowing commercial operation. The NRC issued a full-term OL on 02/20/1991.

APPENDIX A
U.S. Commercial Nuclear Power Reactors (continued)
Operating Reactors

Plant Name, Unit Number Licensee Location Docket Number NRC Web Page Address	NRC Region	Con Type NSSS Architect Engineer Constructor	Licensed MWt	CP Issued OL Issued Comm. Op. LR Issued Exp. Date	2009– 2014* Capacity Factor (Percent)
Grand Gulf Nuclear Station, Unit 1 Entergy Operations, Inc. Port Gibson, MS (20 miles S of Vicksburg, MS) 05000416 www.nrc.gov/info-finder/reactor/gg1.html	IV	BWR-MARK 3 GE 6 BECH BECH	4,408	09/04/1974 11/01/1984 07/01/1985 N/A 11/01/2024	100 88 94 70 86 82
H.B. Robinson Steam Electric Plant, Unit 2 Duke Energy Progress, Inc. Hartsville, SC (26 miles NW of Florence, SC) 05000261 www.nrc.gov/info-finder/reactor/rob2.html	II	PWR-DRYAMB WEST 3LP EBSO EBSO	2,339	04/13/1967 07/31/1970 03/07/1971 04/19/2004 07/31/2030	104 57 100 85 85 86
Hope Creek Generating Station, Unit 1 PSEG Nuclear, LLC Hancocks Bridge, NJ (18 miles SE of Wilmington, DE) 05000354 www.nrc.gov/info-finder/reactor/hope.html	I	BWR-MARK 1 GE 4 BECH BECH	3,840	11/04/1974 07/25/1986 12/20/1986 07/20/2011 04/11/2046	95 93 103 93 80 102
Indian Point Nuclear Generating, Unit 2 Entergy Nuclear Indian Point 2, LLC Buchanan, NY (24 miles N of New York, NY) 05000247 www.nrc.gov/info-finder/reactor/ip2.html	I	PWR-DRYAMB WEST 4LP UE&C WDCCO	3,216	10/14/1966 09/28/1973 08/01/1974 N/A 09/28/2013	98 82 98 90 77 93
Indian Point Nuclear Generating, Unit 3 Entergy Nuclear Indian Point 3, LLC Buchanan, NY (24 miles N of New York, NY) 05000286 www.nrc.gov/info-finder/reactor/ip3.html	I	PWR-DRYAMB WEST 4LP UE&C WDCCO	3,216	08/13/1969 12/12/1975 08/30/1976 N/A 12/12/2015	85 99 90 100 94 98
James A. FitzPatrick Nuclear Power Plant Entergy Nuclear FitzPatrick, LLC Scriba, NY (6 miles NE of Oswego, NY) 05000333 www.nrc.gov/info-finder/reactor/fitz.html	I	BWR-MARK 1 GE 4 S&W S&W	2,536	05/20/1970 10/17/1974 07/28/1975 09/08/2008 10/17/2034	99 85 97 84 89 79
Joseph M. Farley Nuclear Plant, Unit 1 Southern Nuclear Operating Co. Columbia, AL (18 miles E of Dothan, AL) 05000348 www.nrc.gov/info-finder/reactor/far1.html	II	PWR-DRYAMB WEST 3LP SSI DANI	2,775	08/16/1972 06/25/1977 12/01/1977 05/12/2005 06/25/2037	90 88 101 91 90 102

APPENDIX A
U.S. Commercial Nuclear Power Reactors (continued)
Operating Reactors

Plant Name, Unit Number Licensee Location Docket Number NRC Web Page Address	NRC Region	Con Type NSSS Architect Engineer Constructor	Licensed MWt	CP Issued OL Issued Comm. Op. LR Issued Exp. Date	2009– 2014* Capacity Factor (Percent)
Joseph M. Farley Nuclear Plant, Unit 2 Southern Nuclear Operating Co. Columbia, AL (18 miles E of Dothan, AL) 05000364 www.nrc.gov/info-finder/reactor/far2.html	II	PWR-DRYAMB WEST 3LP SSI BECH	2,775	08/16/1972	96
				03/31/1981	88
				07/30/1981	89
				05/12/2005	104
				03/31/2041	91
89					
LaSalle County Station, Unit 1 Exelon Generation Co., LLC Marseilles, IL (11 miles SE of Ottawa, IL) 05000373 www.nrc.gov/info-finder/reactor/lasa1.html	III	BWR-MARK 2 GE 5 S&L CWE	3,546	09/10/1973	99
				04/17/1982	94
				01/01/1984	101
				N/A	97
				04/17/2022	95
93					
LaSalle County Station, Unit 2 Exelon Generation Co., LLC Marseilles, IL (11 miles SE of Ottawa, IL) 05000374 www.nrc.gov/info-finder/reactor/lasa2.html	III	BWR-MARK 2 GE 5 S&L CWE	3,546	09/10/1973	93
				12/16/1983	101
				10/19/1984	96
				N/A	103
				12/16/2023	88
95					
Limerick Generating Station, Unit 1 Exelon Generation Co., LLC Limerick, PA (21 miles NW of Philadelphia, PA) 05000352 www.nrc.gov/info-finder/reactor/lim1.html	I	BWR-MARK 2 GE 4 BECH BECH	3,515	06/19/1974	101
				08/08/1985	91
				02/01/1986	96
				N/A	85
				10/26/2024	101
91					
Limerick Generating Station, Unit 2 Exelon Generation Co., LLC Limerick, PA (21 miles NW of Philadelphia, PA) 05000353 www.nrc.gov/info-finder/reactor/lim2.html	I	BWR-MARK 2 GE 4 BECH BECH	3,515	06/19/1974	94
				08/25/1989	99
				01/08/1990	90
				N/A	95
				06/22/2029	94
99					
McGuire Nuclear Station, Unit 1 Duke Energy Carolinas, LLC Huntersville, NC (17 miles N of Charlotte, NC) 05000369 www.nrc.gov/info-finder/reactor/mcg1.html	II	PWR-ICECND WEST 4LP DUKE DUKE	3,411	02/23/1973	104
				07/08/1981	92
				12/01/1981	94
				12/05/2003	105
				06/12/2041	82
82					
McGuire Nuclear Station, Unit 2 Duke Energy Carolinas, LLC Huntersville, NC (17 miles N of Charlotte, NC) 05000370 www.nrc.gov/info-finder/reactor/mcg2.html	II	PWR-ICECND WEST 4LP DUKE DUKE	3,411	02/23/1973	94
				05/27/1983	104
				03/01/1984	91
				12/05/2003	82
				03/03/2043	95
94					

APPENDIX A

U.S. Commercial Nuclear Power Reactors (continued) Operating Reactors

Plant Name, Unit Number Licensee Location Docket Number NRC Web Page Address	NRC Region	Con Type NSSS Architect Engineer Constructor	Licensed MWt	CP Issued OL Issued Comm. Op. LR Issued Exp. Date	2009– 2014* Capacity Factor (Percent)
Millstone Power Station, Unit 2 Dominion Nuclear Connecticut, Inc. Waterford, CT (3.2 miles SW of New London, CT) 05000336 www.nrc.gov/info-finder/reactor/mill2.html	I	PWR-DRYAMB CE BECH BECH	2,700	12/11/1970 09/26/1975 12/26/1975 11/28/2005 07/31/2035	81 97 87 83 95 85
Millstone Power Station, Unit 3 Dominion Nuclear Connecticut, Inc. Waterford, CT (3.2 miles SW of New London, CT) 05000423 www.nrc.gov/info-finder/reactor/mill3.html	I	PWR-DRYSUB WEST 4LP S&W S&W	3,650	08/09/1974 01/31/1986 04/23/1986 11/28/2005 11/25/2045	105 86 87 100 87 87
Monticello Nuclear Generating Plant, Unit 1 Northern States Power Company-Minnesota Monticello, MN (30 miles NW of Minneapolis, MN) 05000263 www.nrc.gov/info-finder/reactor/mont.html	III	BWR-MARK 1 GE 3 BECH BECH	2,004	06/19/1967 01/09/1981 ^B 06/30/1971 11/08/2006 09/08/2030	83 94 69 101 50 78
Nine Mile Point Nuclear Station, Unit 1 Nine Mile Point Nuclear Station, LLC Scriba, NY (6 miles NE of Oswego, NY) 05000220 www.nrc.gov/info-finder/reactor/nmp1.html	I	BWR-MARK 1 GE 2 NIAG S&W	1,850	04/12/1965 12/26/1974 ^C 12/01/1969 10/31/2006 08/22/2029	92 97 84 87 88 98
Nine Mile Point Nuclear Station, Unit 2 Nine Mile Point Nuclear Station, LLC Scriba, NY (6 miles NE of Oswego, NY) 05000410 www.nrc.gov/info-finder/reactor/nmp2.html	I	BWR-MARK 2 GE 5 S&W S&W	3,988	06/24/1974 07/02/1987 03/11/1988 10/31/2006 10/31/2046	99 89 95 83 99 87
North Anna Power Station, Unit 1 Virginia Electric & Power Co. Mineral (Louisa County), VA (40 miles NW of Richmond, VA) 05000338 www.nrc.gov/info-finder/reactor/na1.html	II	PWR-DRYSUB WEST 3LP S&W S&W	2,940	02/19/1971 04/01/1978 06/06/1978 03/20/2003 04/01/2038	92 86 78 89 89 100

B: AEC issued a provisional OL on 09/08/1970, allowing commercial operation. The NRC issued a full-term OL on 01/09/1981.

C: AEC issued a provisional OL on 08/22/1969, allowing commercial operation. The NRC issued a full-term OL on 12/26/1974.

APPENDIX A
U.S. Commercial Nuclear Power Reactors (continued)
Operating Reactors

Plant Name, Unit Number Licensee Location Docket Number NRC Web Page Address	NRC Region	Con Type NSSS Architect Engineer Constructor	Licensed MWt	CP Issued OL Issued Comm. Op. LR Issued Exp. Date	2009– 2014* Capacity Factor (Percent)
North Anna Power Station, Unit 2 Virginia Electric & Power Co. Mineral (Louisa County), VA (40 miles NW of Richmond, VA) 05000339 www.nrc.gov/info-finder/reactor/na2.html	II	PWR-DRYSUB WEST 3LP S&W S&W	2,940	02/19/1971 08/21/1980 12/14/1980 03/20/2003 08/21/2040	100 100 76 99 85 92
Oconee Nuclear Station, Unit 1 Duke Energy Carolinas, LLC Seneca, SC (30 miles W of Greenville, SC) 05000269 www.nrc.gov/info-finder/reactor/oco1.html	II	PWR-DRYAMB B&W LLP DBDB DUKE	2,568	11/06/1967 02/06/1973 07/15/1973 05/23/2000 02/06/2033	85 100 79 90 91 91
Oconee Nuclear Station, Unit 2 Duke Energy Carolinas, LLC Seneca, SC (30 miles W of Greenville, SC) 05000270 www.nrc.gov/info-finder/reactor/oco2.html	II	PWR-DRYAMB B&W LLP DBDB DUKE	2,568	11/06/1967 10/06/1973 09/09/1974 05/23/2000 10/06/2033	103 91 93 102 82 101
Oconee Nuclear Station, Unit 3 Duke Energy Carolinas, LLC Seneca, SC (30 miles W of Greenville, SC) 05000287 www.nrc.gov/info-finder/reactor/oco3.html	II	PWR-DRYAMB B&W LLP DBDB DUKE	2,568	11/06/1967 07/19/1974 12/16/1974 05/23/2000 07/19/2034	94 91 103 86 97 92
Oyster Creek Nuclear Generating Station Exelon Generation Co., LLC Forked River, NJ (9 miles S of Toms River, NJ) 05000219 www.nrc.gov/info-finder/reactor/oc.html	I	BWR-MARK 1 GE 2 B&R B&R	1,930	12/15/1964 07/02/1991 ^D 12/23/1969 04/08/2009 04/09/2029	92 85 98 88 106 90
Palisades Nuclear Plant Entergy Nuclear Operations, Inc. Covert, MI (5 miles S of South Haven, MI) 05000255 www.nrc.gov/info-finder/reactor/pali.html	III	PWR-DRYAMB CE BECH BECH	2,565.4	03/14/1967 02/21/1991 ^E 12/31/1971 01/17/2007 03/24/2031	90 92 96 74 85 86
Palo Verde Nuclear Generating Station, Unit 1 Arizona Public Service Company Wintersburg, AZ (50 miles W of Phoenix, AZ) 05000528 www.nrc.gov/info-finder/reactor/palo1.html	IV	PWR-DRYAMB CE80-2L BECH BECH	3,990	05/25/1976 06/01/1985 01/28/1986 04/21/2011 06/01/2045	101 81 83 100 85 90

D: AEC issued a provisional OL on 04/09/1969, allowing commercial operation. The NRC issued a full-term OL on 07/02/1991.

E: AEC issued a provisional OL on 03/24/1971, allowing commercial operation. The NRC issued a full-term OL on 02/21/1991.

APPENDIX A

U.S. Commercial Nuclear Power Reactors (continued) Operating Reactors

Plant Name, Unit Number Licensee Location Docket Number NRC Web Page Address	NRC Region	Con Type NSSS Architect Engineer Constructor	Licensed MWt	CP Issued OL Issued Comm. Op. LR Issued Exp. Date	2009– 2014* Capacity Factor (Percent)
Palo Verde Nuclear Generating Station, Unit 2 Arizona Public Service Company Wintersburg, AZ (50 miles W of Phoenix, AZ) 05000529 www.nrc.gov/info-finder/reactor/palo2.html	IV	PWR-DRYAMB CE80-2L BECH BECH	3,990	05/25/1976	83
				04/24/1986	101
				09/19/1986	91
				04/21/2011	90
				04/24/2046	91
Palo Verde Nuclear Generating Station, Unit 3 Arizona Public Service Company Wintersburg, AZ (50 miles W of Phoenix, AZ) 05000530 www.nrc.gov/info-finder/reactor/palo3.html	IV	PWR-DRYAMB CE80-2L BECH BECH	3,990	05/25/1976	83
				11/25/1987	89
				01/08/1988	97
				04/21/2011	88
				11/25/2047	79
Peach Bottom Atomic Power Station, Unit 2 Exelon Generation Co., LLC Delta, PA (17.9 miles S of Lancaster, PA) 05000277 www.nrc.gov/info-finder/reactor/pb2.html	I	BWR-MARK 1 GE 4 BECH BECH	3,514	01/31/1968	102
				10/25/1973	92
				07/05/1974	101
				05/07/2003	88
				08/08/2033	100
Peach Bottom Atomic Power Station, Unit 3 Exelon Generation Co., LLC Delta, PA (17.9 miles S of Lancaster, PA) 05000278 www.nrc.gov/info-finder/reactor/pb3.html	I	BWR-MARK 1 GE 4 BECH BECH	3,514	01/31/1968	89
				07/02/1974	100
				12/23/1974	90
				05/07/2003	103
				07/02/2034	85
Perry Nuclear Power Plant, Unit 1 FirstEnergy Nuclear Operating Co. Perry, OH (35 miles NE of Cleveland, OH) 05000440 www.nrc.gov/info-finder/reactor/perr1.html	III	BWR-MARK 3 GE 6 GIL KAIS	3,758	05/03/1977	67
				11/13/1986	98
				11/18/1987	79
				N/A	92
				03/18/2026	73
Pilgrim Nuclear Power Station Entergy Nuclear Operations, Inc. Plymouth, MA (38 miles SE of Boston, MA) 05000293 www.nrc.gov/info-finder/reactor/pilg.html	I	BWR-MARK 1 GE 3 BECH BECH	2,028	08/26/1968	90
				06/08/1972	99
				12/01/1972	85
				05/29/2012	98
				06/08/2032	74
Point Beach Nuclear Plant, Unit 1 NextEra Energy Point Beach, LLC Two Rivers, WI (13 miles NW of Manitowoc, WI) 05000266 www.nrc.gov/info-finder/reactor/poin1.html	III	PWR-DRYAMB WEST 2LP BECH BECH	1,800	07/19/1967	98
				10/05/1970	88
				12/21/1970	79
				12/22/2005	100
				10/05/2030	84
					90

APPENDIX A
U.S. Commercial Nuclear Power Reactors (continued)
Operating Reactors

Plant Name, Unit Number Licensee Location Docket Number NRC Web Page Address	NRC Region	Con Type NSSS Architect Engineer Constructor	Licensed MWt	CP Issued OL Issued Comm. Op. LR Issued Exp. Date	2009– 2014* Capacity Factor (Percent)
Point Beach Nuclear Plant, Unit 2 NextEra Energy Point Beach, LLC Two Rivers, WI (13 miles NW of Manitowoc, WI) 05000301 www.nrc.gov/info-finder/reactor/poin2.html	III	PWR-DRYAMB WEST 2LP BECH BECH	1,800	07/25/1968	84
				03/08/1973 ^F	96
				10/01/1972	67
				12/22/2005	89
				03/08/2033	93
90					
Prairie Island Nuclear Generating Plant, Unit 1 Northern States Power Co.—Minnesota Welch, MN (28 miles SE of Minneapolis, MN) 05000282 www.nrc.gov/info-finder/reactor/prai1.html	III	PWR-DRYAMB WEST 2LP FLUR NSP	1,677	06/25/1968	97
				04/05/1974 ^G	96
				12/16/1973	91
				06/27/2011	81
				08/09/2033	90
84					
Prairie Island Nuclear Generating Plant, Unit 2 Northern States Power Co.—Minnesota Welch, MN (28 miles SE of Minneapolis, MN) 05000306 www.nrc.gov/info-finder/reactor/prai2.html	III	PWR-DRYAMB WEST 2LP FLUR NSP	1,677	06/25/1968	75
				10/29/1974	86
				12/21/1974	99
				06/27/2011	74
				10/29/2034	59
101					
Quad Cities Nuclear Power Station, Unit 1 Exelon Generation Co., LLC Cordova, IL (20 miles NE of Moline, IL) 05000254 www.nrc.gov/info-finder/reactor/quad1.html	III	BWR-MARK 1 GE 3 S&L UE&C	2,957	02/15/1967	82
				12/14/1972	99
				02/18/1973	92
				10/28/2004	102
				12/14/2032	85
103					
Quad Cities Nuclear Power Station, Unit 2 Exelon Generation Co., LLC Cordova, IL (20 miles NE of Moline, IL) 05000265 www.nrc.gov/info-finder/reactor/quad2.html	III	BWR-MARK 1 GE 3 S&L UE&C	2,957	02/15/1967	91
				12/14/1972	92
				03/10/1973	104
				10/28/2004	92
				12/14/2032	91
90					
River Bend Station, Unit 1 Entergy Nuclear Operations, Inc. St. Francisville, LA (24 miles NW of Baton Rouge, LA) 05000458 www.nrc.gov/info-finder/reactor/rbs1.html	IV	BWR-MARK 3 GE 6 S&W S&W	3,091	03/25/1977	113
				11/20/1985	98
				06/16/1986	90
				N/A	91
				08/29/2025	84
96					

F: AEC issued a provisional OL on 11/18/1971. The NRC issued a full-term OL on 03/08/1973.

G: AEC issued a provisional OL on 08/09/1973. The NRC issued a full-term OL on 04/05/1974.

APPENDIX A

U.S. Commercial Nuclear Power Reactors (continued) Operating Reactors

Plant Name, Unit Number Licensee Location Docket Number NRC Web Page Address	NRC Region	Con Type NSSS Architect Engineer Constructor	Licensed MWt	CP Issued OL Issued Comm. Op. LR Issued Exp. Date	2009– 2014* Capacity Factor (Percent)
R.E. Ginna Nuclear Power Plant R.E. Ginna Nuclear Power Plant, LLC Ontario, NY (20 miles NE of Rochester, NY) 05000244 www.nrc.gov/info-finder/reactor/ginn.html	I	PWR-DRYAMB WEST 2LP GIL BECH	1,775	04/25/1966 09/19/1969 07/01/1970 05/19/2004 09/18/2029	91 97 84 90 93 91
St. Lucie Plant, Unit 1 Florida Power & Light Co. Jensen Beach, FL (10 miles SE of Ft. Pierce, FL) 05000335 www.nrc.gov/info-finder/reactor/stl1.html	II	PWR-DRYAMB CE EBSO EBSO	3,020	07/01/1970 03/01/1976 12/21/1976 10/02/2003 03/01/2036	100 72 85 72 74 101
St. Lucie Plant, Unit 2 Florida Power & Light Co. Jensen Beach, FL (10 miles SE of Ft. Pierce, FL) 05000389 www.nrc.gov/info-finder/reactor/stl2.html	II	PWR-DRYAMB CE EBSO EBSO	3,020	05/02/1977 06/10/1983 08/08/1983 10/02/2003 04/06/2043	80 100 66 68 91 82
Salem Nuclear Generating Station, Unit 1 PSE&G Nuclear, LLC Hancocks Bridge, NJ (18 miles SE of Wilmington, DE) 05000272 www.nrc.gov/info-finder/reactor/salm1.html	I	PWR-DRYAMB WEST 4LP PSEG UE&C	3,459	09/25/1968 12/01/1976 06/30/1977 06/30/2011 08/13/2036	99 85 86 97 88 86
Salem Nuclear Generating Station, Unit 2 PSE&G Nuclear, LLC Hancocks Bridge, NJ (18 miles SE of Wilmington, DE) 05000311 www.nrc.gov/info-finder/reactor/salm2.html	I	PWR-DRYAMB WEST 4LP PSEG UE&C	3,459	09/25/1968 05/20/1981 10/13/1981 06/30/2011 04/18/2040	93 98 89 88 100 73
Seabrook Station, Unit 1 NextEra Energy Seabrook, LLC Seabrook, NH (13 miles S of Portsmouth, NH) 05000443 www.nrc.gov/info-finder/reactor/seab1.html	I	PWR-DRYAMB WEST 4LP UE&C UE&C	3,648	07/07/1976 03/15/1990 08/19/1990 N/A 03/15/2030	81 100 77 75 100 93
Sequoyah Nuclear Plant, Unit 1 Tennessee Valley Authority Soddy-Daisy, TN (16 miles NE of Chattanooga, TN) 05000327 www.nrc.gov/info-finder/reactor/seq1.html	II	PWR-ICECND WEST 4LP TVA TVA	3,455	05/27/1970 09/17/1980 07/01/1981 N/A 09/17/2020	89 84 98 89 83 100

APPENDIX A
U.S. Commercial Nuclear Power Reactors (continued)
Operating Reactors

Plant Name, Unit Number Licensee Location Docket Number NRC Web Page Address	NRC Region	Con Type NSSS Architect Engineer Constructor	Licensed MWt	CP Issued OL Issued Comm. Op. LR Issued Exp. Date	2009– 2014* Capacity Factor (Percent)
Sequoyah Nuclear Plant, Unit 2 Tennessee Valley Authority Soddy-Daisy, TN (16 miles NE of Chattanooga, TN) 05000328 www.nrc.gov/info-finder/reactor/seq2.html	II	PWR-ICECND WEST 4LP TVA TVA	3,455	05/27/1970 09/15/1981 06/01/1982 N/A 09/15/2021	89 97 89 77 90 90
Shearon Harris Nuclear Power Plant, Unit 1 Duke Energy Progress, Inc. New Hill, NC (20 miles SW of Raleigh, NC) 05000400 www.nrc.gov/info-finder/reactor/har1.html	II	PWR-DRYAMB WEST 3LP EBSO DANI	2,900	01/27/1978 10/24/1986 05/02/1987 12/17/2008 10/24/2046	94 90 103 90 83 99
South Texas Project, Unit 1 STP Nuclear Operating Co. Bay City, TX (90 miles SW of Houston, TX) 05000498 www.nrc.gov/info-finder/reactor/stp1.html	IV	PWR-DRYAMB WEST 4LP BECH EBSO	3,853	12/22/1975 03/22/1988 08/25/1988 N/A 08/20/2027	90 101 94 93 91 81
South Texas Project, Unit 2 STP Nuclear Operating Co. Bay City, TX (90 miles SW of Houston, TX) 05000499 www.nrc.gov/info-finder/reactor/stp2.html	IV	PWR-DRYAMB WEST 4LP BECH EBSO	3,853	12/22/1975 03/28/1989 06/19/1989 N/A 12/15/2028	101 88 88 72 59 103
Surry Power Station, Unit 1 Virginia Electric and Power Co. Surry, VA (17 miles NW of Newport News, VA) 05000280 www.nrc.gov/info-finder/reactor/sur1.html	II	PWR-DRYSUB WEST 3LP S&W S&W	2,587	06/25/1968 05/25/1972 12/22/1972 03/20/2003 05/25/2032	94 89 101 92 91 99
Surry Power Station, Unit 2 Virginia Electric and Power Co. Surry, VA (17 miles NW of Newport News, VA) 05000281 www.nrc.gov/info-finder/reactor/sur2.html	II	PWR-DRYSUB WEST 3LP S&W S&W	2,587	06/25/1968 01/29/1973 05/01/1973 03/20/2003 01/29/2033	92 100 76 91 101 95
Susquehanna Steam Electric Station, Unit 1 PPL Susquehanna, LLC Berwick, PA (70 miles NE of Harrisburg, PA) 05000387 www.nrc.gov/info-finder/reactor/susq1.html	I	BWR-MARK 2 GE 4 BECH BECH	3,952	11/03/1973 07/17/1982 06/08/1983 11/24/2009 07/17/2042	101 80 86 70 87 83

APPENDIX A

U.S. Commercial Nuclear Power Reactors (continued) Operating Reactors

Plant Name, Unit Number Licensee Location Docket Number NRC Web Page Address	NRC Region	Con Type NSSS Architect Engineer Constructor	Licensed MWt	CP Issued OL Issued Comm. Op. LR Issued Exp. Date	2009– 2014* Capacity Factor (Percent)
Susquehanna Steam Electric Station, Unit 2 PPL Susquehanna, LLC Berwick, PA (70 miles NE of Harrisburg, PA) 05000388 www.nrc.gov/info-finder/reactor/susq2.html	I	BWR-MARK 2 GE 4 BECH BECH	3,952	11/03/1973 03/23/1984 02/12/1985 11/24/2009 03/23/2044	90 96 72 83 80 88
Three Mile Island Nuclear Station, Unit 1 Exelon Generation Co., LLC Middletown, PA (10 miles SE of Harrisburg, PA) 05000289 www.nrc.gov/info-finder/reactor/tmi1.html	I	PWR-DRYAMB B&W LLP GIL UE&C	2,568	05/18/1968 04/19/1974 09/02/1974 10/22/2009 04/19/2034	86 94 92 100 78 104
Turkey Point Nuclear Generating, Unit 3 Florida Power & Light Co. Homestead, FL (20 miles S of Miami, FL) 05000250 www.nrc.gov/info-finder/reactor/tp3.html	II	PWR-DRYAMB WEST 3LP BECH BECH	2,644	04/27/1967 07/19/1972 12/14/1972 06/06/2002 07/19/2032	86 88 96 40 81 84
Turkey Point Nuclear Generating, Unit 4 Florida Power & Light Co. Homestead, FL (20 miles S of Miami, FL) 05000251 www.nrc.gov/info-finder/reactor/tp4.html	II	PWR-DRYAMB WEST 3LP BECH BECH	2,644	04/27/1967 04/10/1973 09/07/1973 06/06/2002 04/10/2033	99 98 84 85 70 88
Virgil C. Summer Nuclear Station, Unit 1 South Carolina Electric & Gas Co. Jenkinsville, SC (26 miles NW of Columbia, SC) 05000395 www.nrc.gov/info-finder/reactor/sum.html	II	PWR-DRYAMB WEST 3LP GIL DANI	2,900	03/21/1973 11/12/1982 01/01/1984 04/23/2004 08/06/2042	81 100 88 86 93 81
Vogtle Electric Generating Plant, Unit 1 Southern Nuclear Operating Co., Inc. Waynesboro, GA (26 miles SE of Augusta, GA) 05000424 www.nrc.gov/info-finder/reactor/vog1.html	II	PWR-DRYAMB WEST 4LP SBEC GPC	3,625.6	06/28/1974 03/16/1987 06/01/1987 06/03/2009 01/16/2047	91 102 92 91 101 87
Vogtle Electric Generating Plant, Unit 2 Southern Nuclear Operating Co., Inc. Waynesboro, GA (26 miles SE of Augusta, GA) 05000425 www.nrc.gov/info-finder/reactor/vog2.html	II	PWR-DRYAMB WEST 4LP SBEC GPC	3,625.6	06/28/1974 03/31/1989 05/20/1989 06/03/2009 02/09/2049	101 93 94 102 87 92

APPENDIX A
U.S. Commercial Nuclear Power Reactors (continued)
Operating Reactors

Plant Name, Unit Number Licensee Location Docket Number NRC Web Page Address	NRC Region	Con Type NSSS Architect Engineer Constructor	Licensed MWt	CP Issued OL Issued Comm. Op. LR Issued Exp. Date	2009– 2014* Capacity Factor (Percent)
Waterford Steam Electric Station, Unit 3 Entergy Operations, Inc. Killona, LA (25 miles W of New Orleans, LA) 05000382 www.nrc.gov/info-finder/reactor/wat3.html	IV	PWR-DRYAMB COMB CE EBSO EBSO	3,716	11/14/1974 03/16/1985 09/24/1985 N/A 12/18/2024	87 100 82 77 89 90
Watts Bar Nuclear Plant, Unit 1 Tennessee Valley Authority Spring City, TN (60 miles SW of Knoxville, TN) 05000390 www.nrc.gov/info-finder/reactor/wb1.html	II	PWR-ICECND WEST 4LP TVA TVA	3,459	01/23/1973 02/07/1996 05/27/1996 N/A 11/09/2035	94 99 84 87 90 89
Wolf Creek Generating Station, Unit 1 Wolf Creek Nuclear Operating Corp. Burlington (Coffey County), KS (28 miles SE of Emporia, KS) 05000482 www.nrc.gov/info-finder/reactor/wc.html	IV	PWR-DRYAMB WEST 4LP BECH DANI	3,565	05/17/1977 06/04/1985 ^H 09/03/1985 11/20/2008 03/11/2045	86 86 72 80 65 83

H: The original OL (NPF-32) was issued on 03/11/1985. The license was superseded by OL (NPF-42), issued on 06/04/1985.

APPENDIX A

U.S. Commercial Nuclear Power Reactors (continued) Operating Reactors Under Active Construction or Deferred Policy

Plant Name, Unit Number Licensee Location Docket Number NRC Web Page Address	NRC Region	Con Type NSSS Architect Engineer Constructor	Licensed MWt	CP Issued OL Issued Comm. Op. LR Issued Exp. Date	2009– 2014* Capacity Factor (Percent)
Bellefonte Nuclear Power Station, Unit 1** Tennessee Valley Authority (6 miles NE of Scottsboro, AL) 05000438	II	PWR-DRYAMB B&W 205 TVA TVA	3,763	12/24/1974	N/A
Bellefonte Nuclear Power Station, Unit 2** Tennessee Valley Authority (6 miles NE of Scottsboro, AL) 05000439	II	PWR-DRYAMB B&W 205 TVA TVA	3,763	12/24/1974	N/A
Watts Bar Nuclear Plant, Unit 2*** Tennessee Valley Authority Spring City, TN (60 miles SW of Knoxville, TN) 05000391 www.nrc.gov/info-finder/reactor/wb/watts-bar.html	II	PWR-ICECND WEST 4LP TVA TVA	3,411	01/23/1973	N/A
Virgil C. Summer Nuclear Station, Unit 2 South Carolina Electric & Gas Co. South Carolina Public Service Auth. Jenkinsville (Fairfield County), SC (26 miles NW of Columbia, SC) 0500027	II	PWR AP1000 WEST SHAW	3,400	03/30/2012	N/A
Virgil C. Summer Nuclear Station, Unit 3 South Carolina Electric & Gas Co. South Carolina Public Service Auth. Jenkinsville (Fairfield County), SC (26 miles NW of Columbia, SC) 05200028	II	PWR AP1000 WEST SHAW	3,400	03/30/2012	N/A
Vogtle Electric Generating Plant, Unit 3 Southern Nuclear Operating Co., Inc. Waynesboro (Burke County), GA (26 miles SE of Augusta, GA) 05200025	II	PWR AP1000 WEST SHAW	3,400	02/10/2012	N/A
Vogtle Electric Generating Plant, Unit 4 Southern Nuclear Operating Co., Inc. Waynesboro, (Burke County), GA (26 miles SE of Augusta, GA) 05200026	II	PWR AP1000 WEST SHAW	3,400	02/10/2012	N/A
Fermi, Unit 3 DTE Electric Company Newport, MI (25 miles NE of Toledo, OH) 05200033	III	ESBWR GEH	4,500	05/01/2015	N/A

* Average capacity factor is listed in year order starting with 2009.

** Bellefonte Units 1 and 2 are under the Commission Policy Statement on Deferred Plants (52 FR 38077; October 14, 1987).

*** Watts Bar Unit 2 is currently under active construction.

Source: NRC, with some data compiled from EIA/DOE

Note: Plant names are as identified on the license as of August 2015.

APPENDIX B

U.S. New Nuclear Power Plant Licensing Applications

Applicant	Docket Number	Type	Submittal Date	Design	Site	State	Existing Plant?	Date Accepted	Status
Combined Operating License									
Nuclear Innovation North America, LLC (NINA)	05200012 & 05200013	COL	9/20/07	ABWR	South Texas Project, Units 3 and 4	TX	Yes	11/29/07	Scheduled
Tennessee Valley Authority (TVA)	05200014 & 05200015	COL	10/30/07	AP1000	Bellefonte, Units 3 and 4	AL	No	1/18/08	Suspended – 09/29/2010
Dominion Virginia Power	05200017	COL	11/27/07	ESBWR	North Anna, Unit 3	VA	Yes	01/28/08	Scheduled
Duke Energy Carolinas	05200018 & 05200019	COL	12/13/07	AP1000	Lee Nuclear Station, Units 3 and 4	SC	No	2/25/08	Scheduled
Progressive Energy	05200022 & 05200023	COL	2/19/08	AP1000	Shearon Harris, Units 2 and 3	NC	Yes	4/17/08	Suspended – 05/02/2013
Southern Nuclear Operating Co.	05200025 & 05200026	COL	3/31/08	AP1000	Vogtle, Units 3 and 4	GA	Yes	5/30/08	COL Issued 02/10/2012
South Carolina Electric and Gas	05200027 & 05200028	COL	3/31/08	AP1000	V.C. Summer, Units 2 and 3	SC	Yes	7/31/08	COL Issued 03/30/2012
AmerenUE	05200037	COL	7/24/08	U.S. EPR	Callaway, Unit 2	MO	Yes	12/12/08	Suspended
Duke Energy Florida	05200029 & 05200030	COL	7/30/08	AP1000	Levy County, Units 1 and 2	FL	No	10/6/08	Scheduled
DTE Electric Company	05200033	COL	9/18/08	ESBWR	Fermi, Unit 3	MI	Yes	11/25/08	COL Issued 05/01/2015
Luminant Generation Co.	05200034 & 05200035	COL	9/19/08	US APWR	Comanche Peak, Units 3 and 4	TX	Yes	12/2/08	Suspended – 03/31/2014
Entergy	05200036	COL	9/25/08	ESBWR	River Bend, Unit 3	LA	Yes	12/4/08	Suspended – 01/09/2009
PPL Bell Bend	05200039	COL	10/10/08	U.S. EPR	Bell Bend (1 Unit)	PA	Yes	12/19/08	Safety review on hold; Environmental review scheduled
Florida Power and Light	05200040 & 05200041	COL	6/30/09	AP1000	Turkey Point, Units 6 and 7	FL	Yes	9/4/09	Scheduled
Design Certification									
AREVA NP	05200020	DC	12/11/07	U.S. EPR	N/A	N/A	N/A	2/25/08	Suspended – 03/27/2015
Mitsubishi Heavy Industries	05200021	DC	12/31/07	US APWR	N/A	N/A	N/A	2/29/08	Applicant Delayed – Not Scheduled
Korea Electric Power Company and Korea Hydro and Nuclear Power	05200046	DC	12/23/14	APR 1400	N/A	N/A	N/A	3/4/15	Scheduled
Toshiba Corporation	05200044	DC	10/27/10	ABWR	N/A	N/A	N/A	12/14/10	Applicant Delayed – Not Scheduled
GE-Hitachi Nuclear Energy	05200045	DC	12/7/10	ABWR	N/A	N/A	N/A	2/14/11	Applicant Delayed – Not Scheduled
Early Site Permit									
PSEG Site	05200043	ESP	5/25/10	Not yet announced	PSEG Site	NJ	Yes	8/4/10	Scheduled

Note: Data is as of July 2015. Withdrawal was requested for Calvert Cliffs, Grand Gulf, Nine Mile Point and Victoria County (COL and ESP). NRC-abbreviated reactor names listed

APPENDIX C

U.S. Commercial Nuclear Power Reactors Undergoing Decommissioning and Permanently Shut Down Formerly Licensed To Operate

Unit Location Docket Number	Reactor Type MWT	NSSS Vendor	OL Issued Shut Down OL Terminated Closure Date Est.	Decommissioning Alternative Selected Current License Status
Big Rock Point Charlevoix, MI 05000155	BWR 240	GE	05/01/1964 08/29/1997 01/08/2007	DECON DECON Completed
Crystal River 3 Crystal River, FL 05000302	PWR 2,609	B&W LLP	12/03/1976 02/20/2013 2074	SAFSTOR SAFSTOR In Progress
Dresden 1 Morris, IL 0500010	BWR 700	GE	09/28/1959 10/31/1978 2036	SAFSTOR SAFSTOR
GE EVESR Sunol, CA 05000183	Experimental Superheat Reactor 12.5	GE	11/12/1963 2/1/1967 4/15/1970 1/1/2019	SAFSTOR Possession only license expires 1/2016
GE VBWR (Vallecitos) Sunol, CA 05000017	BWR 50	GE	08/31/1957 12/09/1963 2019	SAFSTOR SAFSTOR
Fermi 1 Newport, MI 05000016	SCF 200	CE	05/10/1963 09/22/1972 2032	SAFSTOR DECON
Fort St. Vrain Platteville, CO 05000267	HTG 842	GA	12/21/1973 08/18/1989 08/08/1997	DECON DECON Completed
Haddam Neck Meriden, CT 05000213	PWR 1,825	WEST	12/27/1974 12/05/1996 11/26/2007	DECON DECON Completed
Humboldt Bay 3 Eureka, CA 05000133	BWR 200	GE	08/28/1962 07/02/1976 2015	DECON DECON In Progress
Indian Point 1 Buchanan, NY 05000003	PWR 615	B&W	03/26/1962 10/31/1974 2026	SAFSTOR SAFSTOR

APPENDIX C

**U.S. Commercial Nuclear Power Reactors
Undergoing Decommissioning and Permanently Shut Down
Formerly Licensed To Operate (continued)**

Unit Location Docket Number	Reactor Type MWT	NSSS Vendor	OL Issued Shut Down OL Terminated Closure Date Est.	Decommissioning Alternative Selected Current License Status
Kewaunee Carlton, WI 05000305	PWR 1,772	WEST 2LP	12/21/1973 05/07/2013 2073	SAFSTOR SAFSTOR
La Crosse Genoa, WI 05000409	BWR 165	AC	07/03/1967 04/30/1987 TBD	DECON DECON in Progress
Maine Yankee Wiscasset, ME 05000309	PWR 2,700	CE	06/29/1973 12/06/1996 09/30/2005	DECON DECON Completed
Millstone 1 Waterford, CT 05000245	BWR 2,011	GE	10/31/1970 07/21/1998 12/31/2056	SAFSTOR SAFSTOR
Pathfinder Sioux Falls, SD 05000130	BWR 190	AC	03/12/1964 09/16/1967 07/27/2007	DECON DECON Completed
Peach Bottom 1 Delta, PA 05000171	HTG 115	GA	01/24/1966 10/31/1974 12/31/2034	SAFSTOR SAFSTOR
Rancho Seco Herald, CA 05000312	PWR 2,772	B&W	08/16/1974 06/07/1989 09/25/2009	DECON DECON Completed
San Onofre 1* San Clemente, CA 05000206	PWR 1,347	WEST	03/27/1967 11/30/1992 12/30/2030	DECON SAFSTOR
San Onofre 2* San Clemente, CA 05000361	PWR CE 3,438	CE	02/16/1982 06/12/2013 2030	DECON DECON in Progress
San Onofre 3 San Clemente, CA 05000362	PWR CE 3,438	CE	11/15/1982 06/12/2013 2030	DECON DECON in Progress
Savannah, N.S. Baltimore, MD 05000238	PWR 74	B&W	08/1965 11/1970 12/01/2031	SAFSTOR SAFSTOR

APPENDIX C

U.S. Commercial Nuclear Power Reactors Undergoing Decommissioning and Permanently Shut Down Formerly Licensed To Operate (continued)

Unit Location Docket Number	Reactor Type MwT	NSSS Vendor	OL Issued Shut Down OL Terminated Closure Date Est.	Decommissioning Alternative Selected Current License Status
Saxton Saxton, PA 05000146	PWR 23.5	WEST	11/15/1961 05/01/1972 11/07/2005	DECON DECON Completed
Shoreham Wading River, NY 05000322	BWR 2,436	GE	04/21/1989 06/28/1989 04/11/1995	DECON DECON Completed
Three Mile Island 2 Middletown, PA 05000320	PWR 2,770	B&W	02/08/1978 03/28/1979 12/31/2036	(1)
Trojan Rainier, OR 05000344	PWR 3,411	WEST	11/21/1975 11/09/1992 05/23/2005	DECON DECON Completed
Yankee-Rowe Rowe, MA 05000029	PWR 600	WEST	12/24/1963 10/01/1991 08/10/2007	DECON DECON Completed
Vermont Yankee Vernon, VT 05000271	BWR-MARK 1 1,912	GE 4	03/21/1972 12/29/2014 2073	SAFSTOR SAFSTOR In progress
Zion 1 Zion, IL 05000295	PWR 3,250	WEST	10/19/1973 02/21/1997	DECON DECON In Progress
Zion 2 Zion, IL 05000304	PWR 3,250	WEST	11/14/1973 09/19/1996 2020	DECON DECON In Progress

* Site has been dismantled and decontaminated with the exception of the reactor vessel in long-term storage.

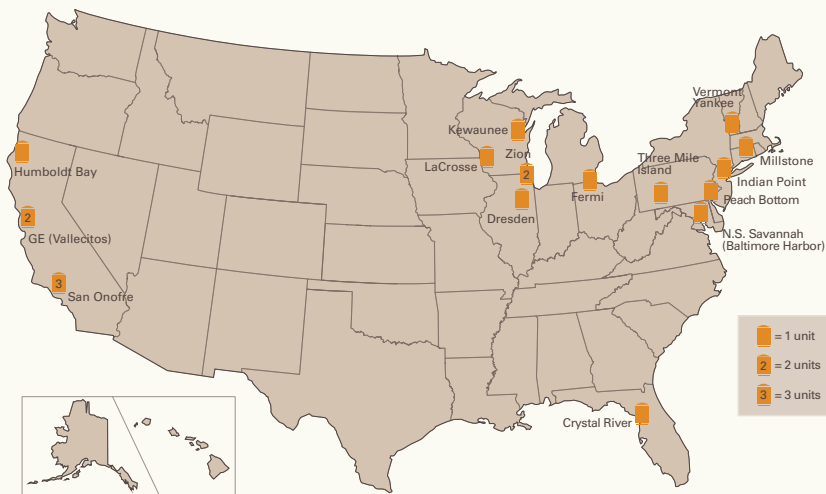
Notes: GE Bonus, Hallam, and Piqua decommissioned reactor sites are part of the DOE Nuclear Legacy. For more information, visit DOE's Legacy Management Web site at energy.gov/lm/sites/lm-sites. CVTR, Elk River, and Shippingport decommissioned reactor sites were either decommissioned prior to the formation of the NRC or were not licensed by the NRC. See Glossary for definitions of decommissioning alternatives (DECON, SAFSTOR).

(1) Three Mile Island Unit 2 has been placed in a postdefueling monitored storage mode until Unit 1 permanently ceases operation, at which time both units are planned to be decommissioned.

Source: DOE Integrated Database for 1990, "U.S. Spent Fuel and Radioactive Waste, Inventories, Projections, and Characteristics" (DOE/RW-0006, Rev. 6), and NRC, "Nuclear Power Plants in the World," Edition 6

APPENDIX C

U.S. Commercial Nuclear Power Reactors Undergoing Decommissioning and Permanently Shut Down Formerly Licensed To Operate (continued)



Reactor Decommissioning Sites

APPENDIX D

Cancelled U.S. Commercial Nuclear Power Reactors Part 50—Domestic Licensing of Production and Utilization Facilities

Unit Utility Location	Con Type MWe per Unit	Cancelled Date Status Docket Number
Allens Creek 1 Houston Lighting & Power Company 4 miles NW of Wallis, TX	BWR 1,150	1982 Under CP Review 05000466
Allens Creek 2 Houston Lighting & Power Company 4 miles NW of Wallis, TX	BWR 1,150	1976 Under CP Review 05000467
Atlantic 1 & 2 Public Service Electric & Gas Company Floating Plants off the Coast of NJ	PWR 1,150	1978 Under CP Review 05000477 & 478
Bailly 1 Northern Indiana Public Service Company 12 miles NNE of Gary, IN	BWR 645	1981 With CP 05000367
Barton 1 & 2 Alabama Power & Light 15 miles SE of Clanton, AL	BWR 1,159	1977 Under CP Review 05000524 & 525
Barton 3 & 4 Alabama Power & Light 15 miles SE of Clanton, AL	BWR 1,159	1975 Under CP Review 05000526 & 527
Black Fox 1 & 2 Public Service Company of Oklahoma 3.5 miles S of Inola, OK	BWR 1,150	1982 Under CP Review 05000556 & 557
Blue Hills 1 & 2 Gulf States Utilities Company SW tip of Toledo Bend Reservoir, TX	PWR 918	1978 Under CP Review 05000510 & 511
Callaway 2 Union Electric Company 25 miles ENE of Jefferson City, MO	PWR 1,150	1981 With CP 05000486
Cherokee 1 Duke Power Company 6 miles SSW of Blacksburg, SC	PWR 1,280	1983 With CP 05000491
Cherokee 2 & 3 Duke Power Company 6 miles SSW of Blacksburg, SC	PWR 1,280	1982 With CP 05000492 & 493
Clinch River Project Management Corp., DOE, TVA 23 miles W of Knoxville, in Oak Ridge, TN	LMFB 350	1983 Under CP Review 05000537
Clinton 2 Illinois Power Company 6 miles E of Clinton, IL	BWR 933	1983 With CP 05000462

APPENDIX D

Cancelled U.S. Commercial Nuclear Power Reactors

Part 50—Domestic Licensing of Production and Utilization Facilities (continued)

Unit Utility Location	Con Type MWe per Unit	Cancelled Date Status Docket Number
Davis-Besse 2 & 3 Toledo Edison Company 21 miles ESE of Toledo, OH	PWR 906	1981 Under CP Review 05000500 & 501
Douglas Point 1 & 2 Potomac Electric Power Company Charles County, MD	BWR 1,146	1977 Under CP Review 05000448 & 449
Erie 1 & 2 Ohio Edison Company Berlin, OH	PWR 1,260	1980 Under CP Review 05000580 & 581
Forked River 1 Jersey Central Power & Light Company 2 miles S of Forked River, NJ	PWR 1,070	1980 With CP 05000363
Fort Calhoun 2 Omaha Public Power District 19 miles N of Omaha, NE	PWR 1,136	1977 Under CP Review 05000548
Fulton 1 & 2 Philadelphia Electric Company 17 miles S of Lancaster, PA	HTG 1,160	1975 Under CP Review 05000463 & 464
Grand Gulf 2 Entergy Nuclear Operations, Inc. 20 miles SW of Vicksburg, MS	BWR 1,250	1990 With CP 05000417
Greene County Power Authority of the State of NY 20 miles N of Kingston, NY	PWR 1,191	1980 Under CP Review 05000549
Greenwood 2 & 3 Detroit Edison Company Greenwood Township, MI	PWR 1,200	1980 Under CP Review 05000452 & 453
Hartsville A1 & A2 Tennessee Valley Authority 5 miles SE of Hartsville, TN	BWR 1,233	1984 With CP 05000518 & 519
Hartsville B1 & B2 Tennessee Valley Authority 5 miles SE of Hartsville, TN	BWR 1,233	1982 With CP 05000520 & 521
Haven 1 (formerly Koshkonong) Wisconsin Electric Power Company 4.2 miles SSW of Fort Atkinson, WI	PWR 900	1980 Under CP Review 05000502
Haven 2 (formerly Koshkonong) Wisconsin Electric Power Company 4.2 miles SSW of Fort Atkinson, WI	PWR 900	1978 Under CP Review 05000503

APPENDIX D

Cancelled U.S. Commercial Nuclear Power Reactors

Part 50—Domestic Licensing of Production and Utilization Facilities (continued)

Unit Utility Location	Con Type MWe per Unit	Cancelled Date Status Docket Number
Hope Creek 2 Public Service Electric & Gas Company 18 miles SE of Wilmington, DE	BWR 1,067	1981 With CP 05000355
Jamesport 1 & 2 Long Island Lighting Company 65 miles E of New York City, NY	PWR 1,150	1980 With CP 05000516 & 517
Marble Hill 1 & 2 Public Service of Indiana 6 miles NE of New Washington, IN	PWR 1,130	1985 With CP 05000546 & 547
Midland 1 Consumers Power Company S of City of Midland, MI	PWR 492	1986 With CP 05000329
Midland 2 Consumers Power Company S of City of Midland, MI	PWR 818	1986 With CP 05000330
Montague 1 & 2 Northeast Nuclear Energy Company 1.2 miles SSE of Turners Falls, MA	BWR 1,150	1980 Under CP Review 05000496 & 497
New England 1 & 2 New England Power Company 8.5 miles E of Westerly, RI	PWR 1,194	1979 Under CP Review 05000568 & 569
New Haven 1 & 2 New York State Electric & Gas Corporation 3 miles NW of New Haven, NY	PWR 1,250	1980 Under CP Review 05000596 & 597
North Anna 3 Virginia Electric & Power Company 40 miles NW of Richmond, VA	PWR 907	1982 With CP 05000404
North Anna 4 Virginia Electric & Power Company 40 miles NW of Richmond, VA	PWR 907	1980 With CP 05000405
North Coast 1 Puerto Rico Water Resources Authority 4.7 miles ESE of Salinas, PR	PWR 583	1978 Under CP Review 05000376
Palo Verde 4 & 5 Arizona Public Service Company 36 miles W of Phoenix, AZ	PWR 1,270	1979 Under CP Review 05000592 & 593
Pebble Springs 1 & 2 Portland General Electric Company 55 miles WSW of Tri-Cities (Kenewick-Pasco-Richland, WA), OR	PWR 1,260	1982 Under CP Review 05000514 & 515

APPENDIX D

Cancelled U.S. Commercial Nuclear Power Reactors

Part 50—Domestic Licensing of Production and Utilization Facilities (continued)

Unit Utility Location	Con Type MWe per Unit	Cancelled Date Status Docket Number
Perkins 1, 2, & 3 Duke Power Company 10 miles N of Salisbury, NC	PWR 1,280	1982 Under CP Review 05000488 & 489 & 490
Perry 2 Cleveland Electric Illuminating Co. 35 miles NE of Cleveland, OH	BWR 1,205	1994 Under CP Review 05000441
Phipps Bend 1 & 2 Tennessee Valley Authority 15 miles SW of Kingsport, TN	BWR 1,220	1982 With CP 05000553 & 554
Pilgrim 2 Boston Edison Company 4 miles SE of Plymouth, MA	PWR 1,180	1981 Under CP Review 05000471
Pilgrim 3 Boston Edison Company 4 miles SE of Plymouth, MA	PWR 1,180	1974 Under CP Review 05000472
Quanicassee 1 & 2 Consumers Power Company 6 miles E of Essexville, MI	PWR 1,150	1974 Under CP Review 05000475 & 476
River Bend 2 Gulf States Utilities Company 24 miles NNW of Baton Rouge, LA	BWR 934	1984 With CP 05000459
Seabrook 2 Public Service Co. of New Hampshire 13 miles S of Portsmouth, NH	PWR 1,198	1988 With CP 05000444
Shearon Harris 2 Carolina Power & Light Company 20 miles SW of Raleigh, NC	PWR 900	1983 With CP 05000401
Shearon Harris 3 & 4 Carolina Power & Light Company 20 miles SW of Raleigh, NC	PWR 900	1981 With CP 05000402 & 403
Skagit/Hanford 1 & 2 Puget Sound Power & Light Company 23 miles SE of Bellingham, WA	PWR 1,277	1983 Under CP Review 05000522 & 523
Sterling Rochester Gas & Electric Corporation 50 miles E of Rochester, NY	PWR 1,150	1980 With CP 05000485
Summit 1 & 2 Delmarva Power & Light Company 15 miles SSW of Wilmington, DE	HTG 1,200	1975 Under CP Review 05000450 & 451

APPENDIX D

Cancelled U.S. Commercial Nuclear Power Reactors

Part 50—Domestic Licensing of Production and Utilization Facilities (continued)

Unit Utility Location	Con Type MWe per Unit	Cancelled Date Status Docket Number
Sundesert 1 & 2 San Diego Gas & Electric Company 16 miles SW of Blythe, CA	PWR 974	1978 Under CP Review 05000582 & 583
Surry 3 & 4 Virginia Electric & Power Company 17 miles NW of Newport News, VA	PWR 882	1977 With CP 05000434 & 435
Tyrone 1 Northern States Power Company 8 miles NE of Durond, WI	PWR 1,150	1981 Under CP Review 05000484
Tyrone 2 Northern States Power Company 8 miles NE of Durond, WI	PWR 1,150	1974 With CP 05000487
Vogtle 3 & 4 Georgia Power Company 26 miles SE of Augusta, GA	PWR 1,113	1974 With CP 050000426 & 427
Washington Nuclear 1 (WPPSS) Energy Northwest 10 miles E of Aberdeen, WA	PWR 1,266	1995 With CP 05000460
Washington Nuclear 3 (WPPSS) Energy Northwest 16 miles E of Aberdeen, WA	PWR 1,242	1995 With CP 05000508
Washington Nuclear 4 (WPPSS) Energy Northwest 10 miles E of Aberdeen, WA	PWR 1,218	1982 With CP 05000513
Washington Nuclear 5 (WPPSS) Energy Northwest 16 miles E of Aberdeen, WA	PWR 1,242	1982 With CP 05000509
Yellow Creek 1 & 2 Tennessee Valley Authority 15 miles E of Corinth, MS	BWR 1,285	1984 With CP 05000566 & 567
Zimmer 1 Cincinnati Gas & Electric Company 25 miles SE of Cincinnati, OH	BWR 810	1984 With CP 05000358

Note: Cancellation is defined as public announcement of cancellation or written notification to the NRC. Only NRC-docketed applications are included. Status is the status of the application at the time of cancellation.

Source: DOE/EIA Commercial Nuclear Power 1991 (DOE/EIA-0438), Appendix E (page 105), and the NRC

APPENDIX D

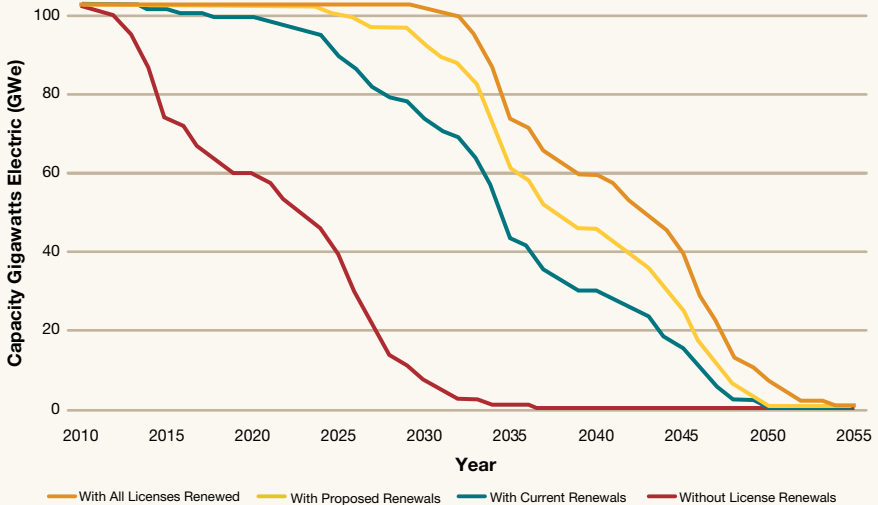
Cancelled U.S. Commercial Nuclear Power Reactors
Part 52— Licenses Certifications, and Approvals for Nuclear Power Plants

Unit Utility Location	Con Type MWe per Unit	Cancelled Date Status Docket Number
Calvert Cliffs, Unit 3 LLC and UniStar Nuclear Operating Services, LLC Near Lusby in Calvert County, Maryland	U.S. EPR 4,500	July 17, 2015 Withdrawn during COL Review 05200016
Victoria County Station, Units 1 and 2 Exelon Nuclear Texas Holdings, LLC near Victoria City in Victoria County, Texas	ESBWR 4,500	June 11, 2010 Withdrawn during COL Review 05200031 & 05200032
Grand Gulf, Unit 3 Entergy Operations, Inc. (EOI) near Port Gibson in Claiborne County, Mississippi	ESBWR 4,500	January 9, 2009 Withdrawn during COL Review 05200024
Nine Mile Point, Unit 3 LLC and UniStar Nuclear Operating Services, LLC 25 miles SE of Cincinnati, OH	ESBWR 4,500	January 9, 2009 Withdrawn during COL Review 05000358

Note: Data is as of June 2015. Withdrawal was requested for Calvert Cliffs, Grand Gulf, Nine Mile Point, and Victoria County (COL and ESP). NRC-abbreviated reactor names listed

APPENDIX E

Projected Electric Capacity Dependent on License Renewals



APPENDIX F

U.S. Commercial Nuclear Power Reactors by Parent Company

Utility	NRC-Abbreviated Reactor Unit Name
AmerenUE www.ameren.com	Callaway*
Arizona Public Service Company www.aps.com	Palo Verde 1, 2, & 3*
DTE Electric Company (DTE) www.dteenergy.com	Fermi 2
Dominion Generation www.dom.com	Millstone 2 & 3 North Anna 1 & 2 Surry 1 & 2
Duke Energy www.duke-energy.com	Brunswick 1 & 2 Catawba 1 & 2 Harris 1 McGuire 1 & 2 Oconee 1, 2, & 3 Robinson 2
Energy Northwest www.energy-northwest.com	Columbia
Entergy Nuclear Operations, Inc. www.entyerj-nuclear.com	Arkansas Nuclear One 1 & 2 FitzPatrick Grand Gulf 1 Indian Point 2 & 3 Palisades Pilgrim 1 River Bend 1 Waterford 3
Exelon Corporation, LLC www.exeloncorp.com	Braidwood 1 & 2 Byron 1 & 2 Calvert Cliffs 1 & 2 Clinton Dresden 2 & 3 Ginna LaSalle 1 & 2 Limerick 1 & 2 Nine Mile Point 1 & 2 Oyster Creek Peach Bottom 2 & 3 Quad Cities 1 & 2 Three Mile Island 1
First Energy Nuclear Operating Company www.firstenergycorp.com	Beaver Valley 1 & 2 Davis-Besse Perry 1

APPENDIX F

U.S. Commercial Nuclear Power Reactors by Parent Company
(continued)

Utility	NRC-Abbreviated Reactor Unit Name
NextEra Energy, Inc. with principal subsidiaries Florida Power & Light Co. and NextEra Energy Resources, LLC www.fplgroup.com	Duane Arnold Point Beach 1 & 2 Seabrook 1 St. Lucie 1 & 2 Turkey Point 3 & 4
Indiana Michigan Power Company www.indianamichiganpower.com	Cook 1 & 2
Luminant Generation Company, LLC www.luminant.com	Comanche Peak 1 & 2*
Nebraska Public Power District www.nppd.com	Cooper
Northern States Power Company Minnesota doing business as Xcel Energy www.xcelenergy.com	Monticello Prairie Island 1 & 2
Omaha Public Power District www.oppd.com	Fort Calhoun
Pacific Gas & Electric Company www.pge.com	Diablo Canyon 1 & 2*
PPL Susquehanna, LLC www.pplweb.com	Susquehanna 1 & 2
PSEG Nuclear, LLC www.pseg.com	Hope Creek 1 Salem 1 & 2
South Carolina Electric & Gas Company www.sceg.com	Summer
Southern Nuclear Operating Company www.southerncompany.com	Hatch 1 & 2 Farley 1 & 2 Vogtle 1 & 2
STP Nuclear Operating Company www.stpnoc.com	South Texas Project 1 & 2*
Tennessee Valley Authority www.tva.gov	Browns Ferry 1, 2, & 3 Sequoyah 1 & 2 Watts Bar 1
Wolf Creek Nuclear Operating Corporation www.wcnoc.com	Wolf Creek 1*

* These plants have a joint program called the Strategic Teaming and Resource Sharing (STARS) group. They share resources for refueling outages and develop some shared licensing applications.

APPENDIX G

U.S. Commercial Nuclear Power Reactor Operating Licenses— Issued by Year

1969 Dresden 2 Ginna Nine Mile Point 1 Oyster Creek	Calvert Cliffs 1 Cooper Cook 1 Duane Arnold FitzPatrick Hatch 1 Oconee 3 Peach Bottom 3 Prairie Island 1 Prairie Island 2 Three Mile Island 1	1980 North Anna 2 Sequoyah 1	1986 Catawba 2 Hope Creek 1 Millstone 3 Palo Verde 2 Perry 1
1970 Point Beach 1 Robinson 2		1981 Farley 2 McGuire 1 Salem 2 Sequoyah 2	1987 Beaver Valley 2 Braidwood 1 Byron 2 Clinton Harris 1 Nine Mile Point 2 Palo Verde 3 Vogtle 1
1971 Dresden 3 Monticello		1982 LaSalle 1 Summer Susquehanna 1	
1972 Palisades Pilgrim Quad Cities 1 Quad Cities 2 Surry 1 Turkey Point 3	1975 Millstone 2	1983 McGuire 2 St. Lucie 2	
1973 Browns Ferry 1 Fort Calhoun Indian Point 2 Oconee 1 Oconee 2 Peach Bottom 2 Point Beach 2 Surry 2 Turkey Point 4	1976 Beaver Valley 1 Browns Ferry 3 Brunswick 1 Calvert Cliffs 2 Indian Point 3 Salem 1 St. Lucie 1	1984 Callaway Columbia Diablo Canyon 1 Grand Gulf 1 LaSalle 2 Susquehanna 2	1988 Braidwood 2 South Texas Project 1
1974 Arkansas Nuclear 1 Browns Ferry 2 Brunswick 2	1977 Davis-Besse D.C. Cook 2 Farley 1	1985 Byron 1 Catawba 1 Diablo Canyon 2 Fermi 2 Limerick 1 Palo Verde 1 River Bend 1 Waterford 3 Wolf Creek 1	1989 Limerick 2 South Texas Project 2 Vogtle 2
	1978 Arkansas Nuclear 2 Hatch 2 North Anna 1		1990 Comanche Peak 1 Seabrook 1
			1993 Comanche Peak 2
			1996 Watts Bar 1

Note: List is limited to reactors licensed to operate. Year is based on the date the initial full-power operating license was issued. NRC-abbreviated reactor names are listed.

APPENDIX H

U.S. Commercial Nuclear Power Reactor Operating Licenses— Expiration by Year, 2013–2049

2013 Indian Point 2	2030 Comanche Peak 1 Monticello Point Beach 1 Robinson 2 Seabrook	Calvert Cliffs 1 Cook 1 Cooper Duane Arnold Hatch 1 FitzPatrick Oconee 3 Peach Bottom 3 Prairie Island 2 Three Mile Island 1	2041 Farley 2 McGuire 1
2015 Indian Point 3			2042 Summer Susquehanna 1
2017 Davis-Besse	2031 Dresden 3 Palisades		2043 Catawba 1 Catawba 2 Columbia McGuire 2 St. Lucie 2
2020 Sequoyah 1	2032 Pilgrim Quad Cities 1 Quad Cities 2 Surry 1 Turkey Point 3	2035 Millstone 2 Watts Bar 1	2044 Susquehanna 2 Callaway Millstone 3
2021 Sequoyah 2	2033 Browns Ferry 1 Comanche Peak 2 Fort Calhoun Oconee 1 Oconee 2 Peach Bottom 2 Point Beach 2 Prairie Island 1 Surry 2 Turkey Point 4	2036 Beaver Valley 1 Browns Ferry 3 Brunswick 1 Calvert Cliffs 2 St. Lucie 1 Salem 1	2045 Palo Verde 1 Wolf Creek 1
2022 LaSalle 1		2037 Cook 2 Farley 1	2046 Nine Mile Point 2 Harris 1 Hope Creek Palo Verde 2
2023 LaSalle 2	2034 Arkansas Nuclear 1 Browns Ferry 2 Brunswick 2	2038 Arkansas Nuclear 2 Hatch 2 North Anna 1	2047 Beaver Valley 2 Palo Verde 3 Vogtle 1
2024 Byron 1 Diablo Canyon 1 Grand Gulf 1 Limerick 1 Waterford 3		2040 North Anna 2 Salem 2	2049 Vogtle 2
2025 Diablo Canyon 2 Fermi 2 River Bend 1			
2026 Braidwood 1 Byron 2 Clinton Perry			
2027 Braidwood 2 South Texas Project 1			
2028 South Texas Project 2			
2029 Dresden 2 Ginna Limerick 2 Nine Mile Point 1 Oyster Creek			

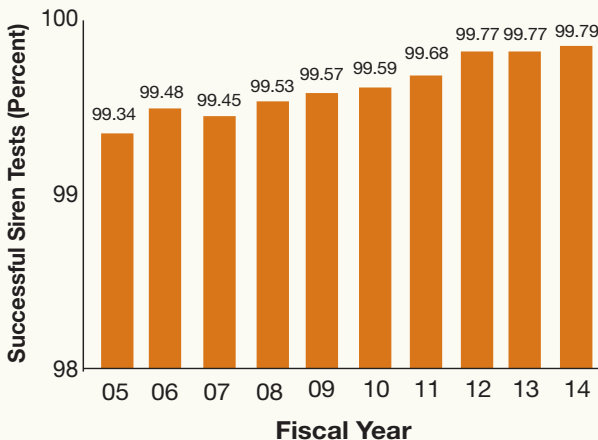
Note: This includes Indian Point 2, which entered timely renewal on Sept. 29, 2013. Limited to reactors licensed to operate. NRC-abbreviated reactor names listed. Data are as of June 2015.

APPENDIX I

Industry Performance Indicators: Industry Averages, FYs 2005–2014

Indicator	2005	2006	2007	2008	2009	2010	2011	2012	2013	2014
Alert and Notification	99.34	99.48	99.45	99.53	99.57	99.59	99.68	99.77	99.77	99.79
Automatic Scrams	0.47	0.32	0.48	0.29	0.36	0.44	0.45	0.30	0.38	0.39
Collective Radiation Exposure	117	93	109	96	87	91	90	72	77	72
Drill/Exercise Performance	95.82	96.06	96.19	96.22	97.06	96.89	97.30	97.51	97.47	97.86
Equipment-Forced Outage Rate	0.13	0.10	0.11	0.08	0.09	0.10	0.09	0.09	0.08	0.10
ERO Drill Participation	98.20	97.89	97.55	98.16	98.65	98.77	99.43	99.33	99.46	99.71
Forced Outage Rate	2.34	1.47	1.41	1.34	2.21	1.74	1.80	2.77	2.98	1.27
Safety System Actuations	0.38	0.22	0.25	0.14	0.23	0.18	0.19	0.17	0.29	0.20
Safety System Failures	0.98	0.60	0.68	0.71	0.71	0.93	0.94	0.87	0.92	0.87
Significant Events	0.05	0.03	0.02	0.03	0.02	0.10	0.13	0.10	0.07	0.01

Alert and Notification System (ANS) Reliability



This figure shows the percentage of ANS sirens that successfully operated during periodic tests in the previous year. The result is an indicator of the reliability of the ANS to alert the public in an emergency.

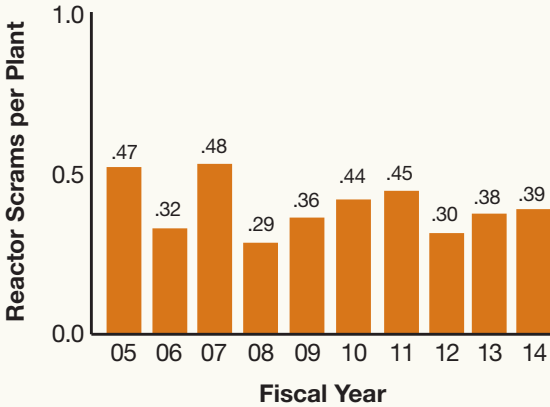
Note: Data represents annual industry averages for operating reactors. The data is continuously updated to incorporate recent information and any subsequent changes in the analysis.

Source: Licensee data as compiled by the NRC

APPENDIX I

Industry Performance Indicators: Industry Averages, FYs 2005–2014 (continued)

Automatic Scrams While Critical

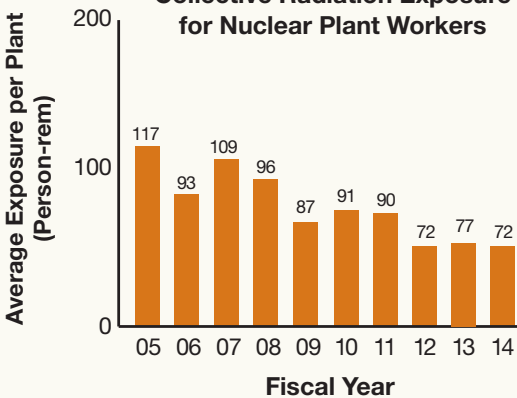


A reactor is considered “critical” when it achieves a self-sustaining nuclear chain reaction, such as when the reactor is operating. The sudden shutting down of a nuclear reactor by the rapid insertion of control rods, either automatically or manually by the reactor operator, is referred to as a “scram.” This indicator measures the number of unplanned automatic scrams that occurred while the reactor was critical.

Note: Data represents annual industry averages for operating reactors. The data is continuously updated to incorporate recent information and any subsequent changes in the analysis.

Source: Licensee data as compiled by the NRC

Collective Radiation Exposure for Nuclear Plant Workers



This indicator monitors the total radiation dose accumulated by plant personnel.

Further Explanation:

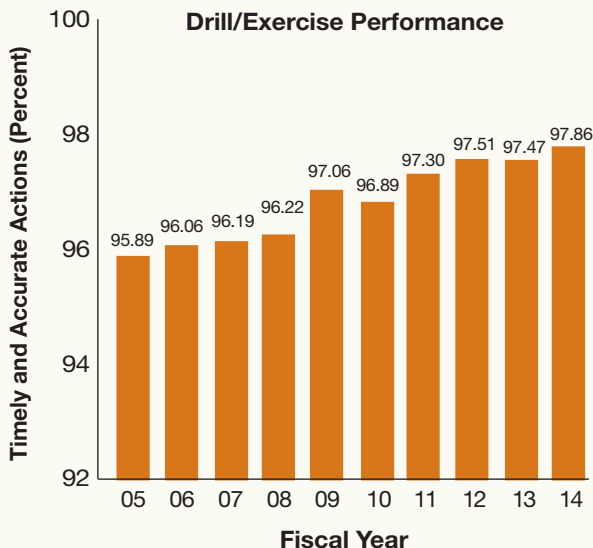
In 2014, those workers receiving a measurable dose of radiation received an average of about 0.1 rem. For comparison purposes, the average U.S. citizen receives 0.3 rem of radiation each year from natural sources (i.e., the everyday environment). See the definition of “exposure” in the Glossary.

Note: Data represents annual industry averages for operating reactors. The data is continuously updated to incorporate recent information and any subsequent changes in the analysis.

Source: Licensee data as compiled by the NRC

APPENDIX I

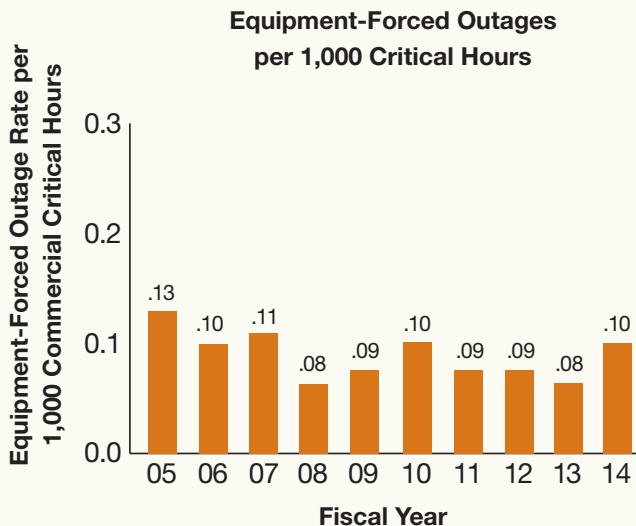
Industry Performance Indicators:
Industry Averages, FYs 2005–2014 (continued)



This indicator is the percentage of timely and accurate actions taken by plant personnel (emergency classifications, protective action recommendations, and notification to offsite authorities) in drills and actual events during the previous 2 years.

Note: Data represents annual industry averages for operating reactors. The data is continuously updated to incorporate recent information and any subsequent changes in the analysis.

Source: Licensee data as compiled by the NRC



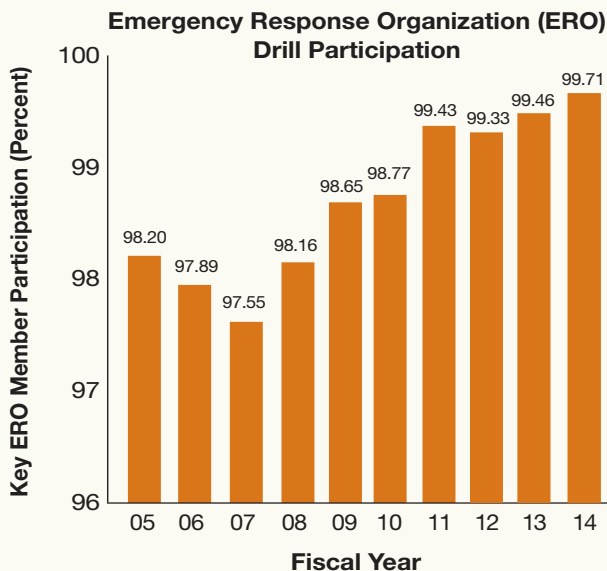
This indicator is the number of times the plant is forced to shut down because of equipment failures for every 1,000 hours that the plant is in operation and transmitting electricity.

Note: Data represents annual industry averages for operating reactors. The data is continuously updated to incorporate recent information and any subsequent changes in the analysis.

Source: Licensee data as compiled by the NRC

APPENDIX I

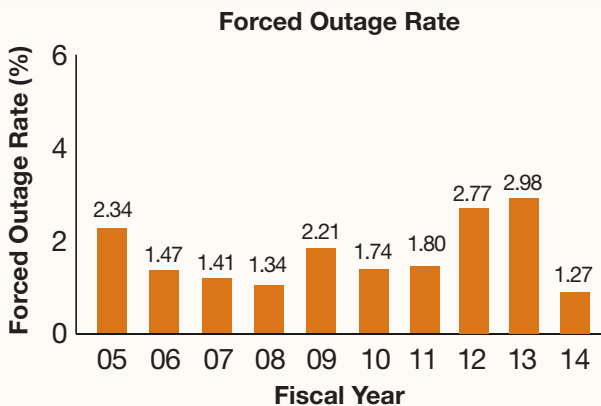
Industry Performance Indicators: Industry Averages, FYs 2005–2014 (continued)



This indicator is the percentage of participation by key plant personnel in drills or actual events in the previous 2 years, indicating proficiency and readiness to respond to emergencies.

Note: Data represents annual industry averages for operating reactors. The data is continuously updated to incorporate recent information and any subsequent changes in the analysis.

Source: Licensee data as compiled by the NRC



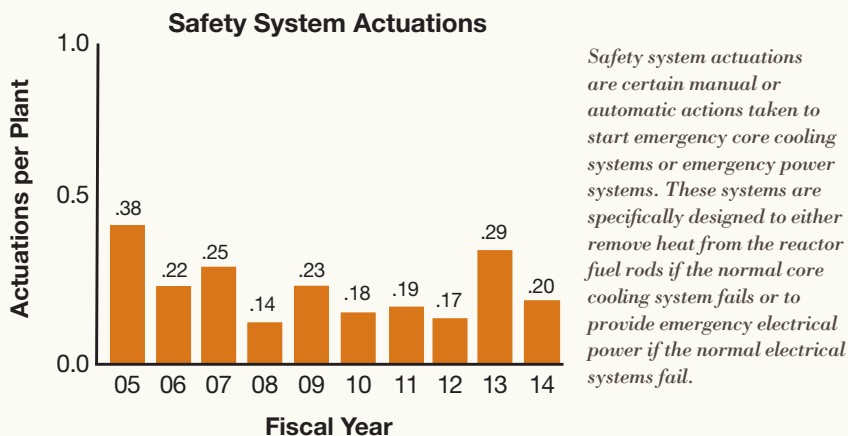
The forced outage rate is the number of hours that the plant is unable to operate (forced outage hours) divided by the sum of the hours that the plant is generating and transmitting electricity (unit service hours) and the hours that the plant is unable to operate (forced outage hours).

Note: Data represents annual industry averages for operating reactors. The data is continuously updated to incorporate recent information and any subsequent changes in the analysis.

Source: Licensee data as compiled by the NRC

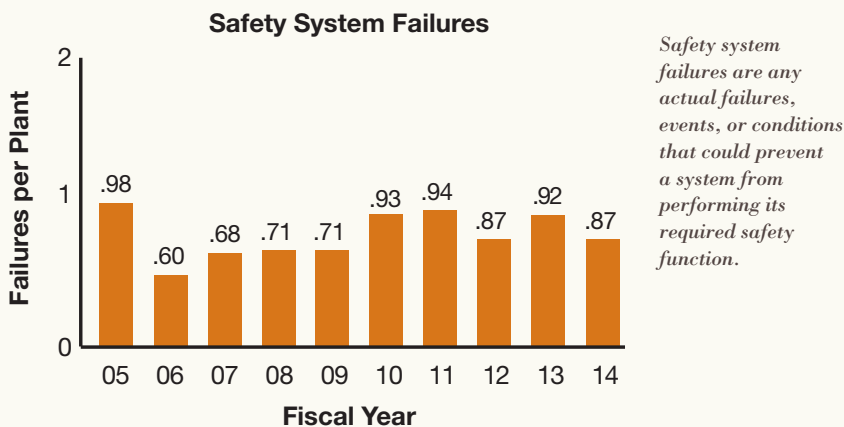
APPENDIX I

**Industry Performance Indicators:
Industry Averages, FYs 2005–2014 (continued)**



Note: Data represents annual industry averages for operating reactors. The data is continuously updated to incorporate recent information and any subsequent changes in the analysis.

Source: Licensee data as compiled by the NRC

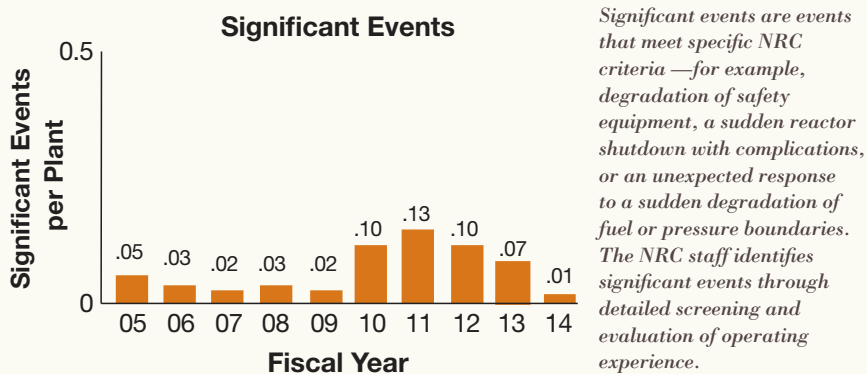


Note: Data represents annual industry averages for operating reactors. The data is continuously updated to incorporate recent information and any subsequent changes in the analysis.

Source: Licensee data as compiled by the NRC

APPENDIX I

Industry Performance Indicators: Industry Averages, FYs 2005–2014 (continued)



Note: Data represents annual industry averages for operating reactors. The data is continuously updated to incorporate recent information and any subsequent changes in the analysis.

Source: Licensee data as compiled by the NRC

APPENDIX J
Operating U.S. Nuclear Research and Test Reactors
Regulated by the NRC

Licensee Location	Reactor Type OL Issued	Power Level (kW)	Licensee Number Docket Number
Aerotest* San Ramon, CA	TRIGA (Indus) 07/02/1965	250	R-98 05000228
Armed Forces Radiobiology Research Institute Bethesda, MD	TRIGA 06/26/1962	1,100	R-84 05000170
Dow Chemical Company Midland, MI	TRIGA 07/03/1967	300	R-108 05000264
GE-Hitachi Sunol, CA	Tank 10/31/1957	100	R-33 05000073
Idaho State University Pocatello, ID	AGN-201 #103 10/11/1967	0.005	R-110 05000284
Kansas State University Manhattan, KS	TRIGA 10/16/1962	250	R-88 05000188
Massachusetts Institute of Technology Cambridge, MA	HWR Reflected 06/09/1958	6,000	R-37 05000020
Missouri University of Science and Technology Rolla, MO	Pool 11/21/1961	200	R-79 05000123
National Institute of Standards & Technology Gaithersburg, MD	Nuclear Test 05/21/1970	20,000	TR-5 05000184
North Carolina State University Raleigh, NC	Pulstar 08/25/1972	1,000	R-120 05000297
Ohio State University Columbus, OH	Pool 02/24/1961	500	R-75 05000150
Oregon State University Corvallis, OR	TRIGA Mark II 03/07/1967	1,100	R-106 05000243
Pennsylvania State University State College, PA	TRIGA 07/08/1955	1,100	R-2 05000005
Purdue University West Lafayette, IN	Lockheed 08/16/1962	1	R-87 05000182
Reed College Portland, OR	TRIGA Mark I 07/02/1968	250	R-112 05000288

* Permanent shutdown ordered July 24, 2013.

APPENDIX J
Operating U.S. Nuclear Research and Test Reactors
Regulated by the NRC (continued)

Licensee Location	Reactor Type OL Issued	Power Level (kW)	Licensee Number Docket Number
Rensselaer Polytechnic Institute Troy, NY	Critical Assembly 07/03/1964	0.1	CX-22 05000225
Rhode Island Atomic Energy Commission Narragansett, RI	GE Pool 07/23/1964	2,000	R-95 05000193
Texas A&M University College Station, TX	AGN-201M #106 08/26/1957	0.005	R-23 05000059
Texas A&M University College Station, TX	TRIGA 12/07/1961	1,000	R-83 05000128
U.S. Geological Survey Denver, CO	TRIGA Mark I 02/24/1969	1,000	R-113 05000274
University of California/Davis Sacramento, CA	TRIGA 08/13/1998	2,300	R-130 05000607
University of California/Irvine Irvine, CA	TRIGA Mark I 11/24/1969	250	R-116 05000326
University of Florida Gainesville, FL	Argonaut 05/21/1959	100	R-56 05000083
University of Maryland College Park, MD	TRIGA 10/14/1960	250	R-70 05000166
University of Massachusetts/Lowell Lowell, MA	GE Pool 12/24/1974	1,000	R-125 05000223
University of Missouri/Columbia Columbia, MO	Tank 10/11/1966	10,000	R-103 05000186
University of New Mexico Albuquerque, NM	AGN-201M #112 09/17/1966	0.005	R-102 05000252
University of Texas Austin, TX	TRIGA Mark II 01/17/1992	1,100	R-129 05000602
University of Utah Salt Lake City, UT	TRIGA Mark I 09/30/1975	100	R-126 05000407
University of Wisconsin Madison, WI	TRIGA 11/23/1960	1,000	R-74 05000156
Washington State University Pullman, WA	TRIGA 03/06/1961	1,000	R-76 05000027

APPENDIX K

U.S. Nuclear Research and Test Reactors
under Decommissioning Regulated by the NRC

Licensee Location	Reactor Type Power Level (kW)	OL Issued Shutdown	Decommissioning Alternative Selected Current Status
General Atomics San Diego, CA	TRIGA Mark F 1,500	07/01/60 09/07/94	DECON SAFSTOR
General Atomics San Diego, CA	TRIGA Mark I 250	05/03/58 12/17/96	DECON SAFSTOR
General Electric Company Sunol, CA	GETR (Tank) 50,000	01/07/59 06/26/85	SAFSTOR SAFSTOR
University of Buffalo Buffalo, NY	Pulstar 2,000	03/24/61 07/23/96	DECON DECON In Progress
Veterans Administration Omaha, NE	TRIGA 20	06/26/59 11/05/01	DECON SAFSTOR

APPENDIX L

U.S. Materials Licenses by State

State	Number of Licenses	
	NRC	Agreement States
Alabama	16	425
Alaska	69	0
Arizona	10	358
Arkansas	9	192
California	72	1,789
Colorado	17	335
Connecticut	157	0
Delaware	49	0
District of Columbia	37	0
Florida	25	1,648
Georgia	24	465
Hawaii	55	0
Idaho	71	0
Illinois	31	647
Indiana	252	0
Iowa	2	162
Kansas	15	280
Kentucky	12	388
Louisiana	11	509
Maine	3	111
Maryland	85	539
Massachusetts	35	444
Michigan	466	0
Minnesota	15	166
Mississippi	9	307
Missouri	256	0
Montana	86	0
Nebraska	10	145
Nevada	2	244
New Hampshire	8	80
New Jersey	37	637

State	Number of Licenses	
	NRC	Agreement States
New Mexico	11	202
New York	28	1,341
North Carolina	24	615
North Dakota	6	102
Ohio	44	610
Oklahoma	19	224
Oregon	6	396
Pennsylvania	50	665
Rhode Island	1	43
South Carolina	15	396
South Dakota	39	0
Tennessee	25	553
Texas	49	1,571
Utah	10	185
Vermont	33	0
Virginia	61	401
Washington	17	357
West Virginia	172	0
Wisconsin	11	305
Wyoming	88	0
Puerto Rico	131	0
Virgin Islands	9	0
Guam	5	0

Total number of materials licenses in Agreement State jurisdiction	17,837
Total number of materials licenses in NRC jurisdiction	2,800
Total number of materials licenses in the United States	20,637

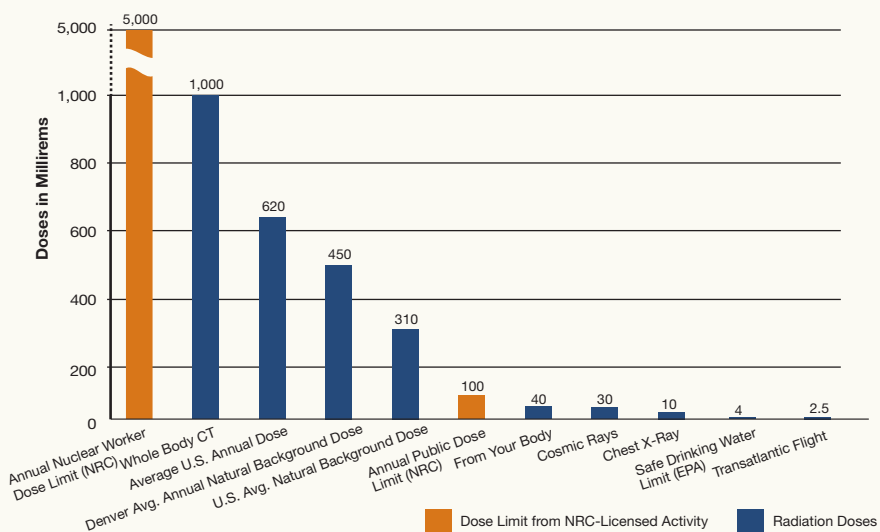
Agreement State
 Letter of intent

* Others include major U.S. territories.

Note: The NRC and Agreement State data is as of November 2014. These totals represent an estimate because the number of specific radioactive materials licenses per State may change on a daily basis.

The NRC licenses Federal agencies in Agreement States.

APPENDIX M Radiation Doses and Regulatory Limits



APPENDIX N

Major U.S. Fuel Cycle Facility Sites

Licensee	Location	Status	Docket #
Uranium Hexafluoride Conversion Facility			
Honeywell International, Inc.	Metropolis, IL	active	04003392
Uranium Fuel Fabrication Facilities			
Global Nuclear Fuel-Americas, LLC	Wilmington, NC	active	07001139
Westinghouse Electric Company, LLC Columbia Fuel Fabrication Facility	Columbia, SC	active	07109239
Nuclear Fuel Services, Inc.	Erwin, TN	active	07000143
Babcock & Wilcox Nuclear Operations Group	Lynchburg, VA	active	07000027
AREVA NP, Inc.	Richland, WA	active	07001257
Mixed Oxide Fuel Fabrication Facility			
Shaw AREVA MOX Services, LLC	Aiken, SC	under construction (operating license under review)	07003098
Gaseous Diffusion Uranium Enrichment Facilities			
USEC, United States Enrichment Corp. Paducah Gaseous Diffusion Plant	Paducah, KY	shut down, certificate termination pending	07007001
Gas Centrifuge Uranium Enrichment Facilities			
USEC, American Centrifuge Operating, LLC Lead Cascade: Test and Demonstration Facility	Piketon, OH	active	07007003
USEC, American Centrifuge Operating, LLC American Centrifuge Plant	Piketon, OH	license issued, construction halted	07007004
Louisiana Energy Services (URENCO-USA)	Eunice, NM	active*	07003103
AREVA Enrichment Services, LLC Eagle Rock Enrichment Facilities	Idaho Falls, ID	license issued, construction not started	07007015
Laser Separation Enrichment Facility			
GE-Hitachi	Wilmington, NC	license issued, construction not started	07007016
Uranium Hexafluoride Deconversion Facility			
International Isotopes	Hobbs, NM (Lea County)	license issued, construction not started	04009086

* Operating and producing enriched uranium while undergoing further phases of construction.

Note: The NRC regulates eleven other university and research facilities that possess special nuclear material (other than reactors) or process source material (other than uranium-recovery facilities) licensed under 10 CFR Part 70, "Domestic Licensing of Special Nuclear Material."

Data are as of May 2015.

APPENDIX O
Dry Spent Fuel Storage Designs:
NRC-Approved for Use by General Licensees

Vendor	Docket #	Storage Design Model
General Nuclear Systems, Inc.	07201000	CASTOR V/21
		CASTOR X/33
NAC International, Inc.	07201002	NAC S/T
	07201003	NAC-C28 S/T
	07201015	NAC-UMS
	07201025	NAC-MPC
	07201031	Magnastor
	07201013	NAC-STC
Holtec International	07201008	HI-STAR 100
	07201014	HI-STORM 100
	07201032	HI-STORM FW
	07201040	HI-STORM UMAX
Energy Solutions, Inc.	07201007	VSC-24
	07201026	Fuel Solutions™ (WSNF-220, -221, -223)
		W-150 Storage Cask
		W-100 Transfer Cask
		W-21, W-74 Canisters
Transnuclear, Inc.	07201005	TN-24
	07201027	TN-68
	07201021	TN-32, 32A, 32B
	07201004	Standardized NUHOMS®-24P, -24PHB, -24PTH, -32PT, -32PTH1, -37PTH, -52B, -61BT, -61BTH, -69BTH
	07201029	Standardized Advanced NUHOMS®-24PT1, -24PT4
	07201030	NUHOMS® HD-32PTH
	07201022	NUHOMS®-7P

Data are as of May 2015. (See latest list on the NRC Web site at www.nrc.gov/waste/spent-fuel-storage/designs.html.)

APPENDIX P

Dry Cask Spent Fuel Storage Licensees

Name Licensee	License Type	Vendor	Storage Model	Docket Number
Arkansas Nuclear Entergy Nuclear Operations, Inc.	GL	Energy Solutions, Inc. Holtec International	VSC-24 HI-STORM 100	07200013
Beaver Valley	GL	Transnuclear, Inc	NUHOMS®-37PTH	07201043
Big Rock Point Entergy Nuclear Operations, Inc.	GL	Energy Solutions, Inc.	Fuel Solutions™ W74	07200043
Braidwood Exelon Generation Co., LLC	GL	Holtec International	HI-STORM 100	07200073
Browns Ferry Tennessee Valley Authority	GL	Holtec International	HI-STORM 100S	07200052
Brunswick Carolina Power Co.	GL	Transnuclear, Inc.	NUHOMS®-HD-61BTH	07200006
Byron Exelon Generation Co., LLC	GL	Holtec International	HI-STORM 100	07200068
Callaway	GL	Holtec International	HI-STORM UMAX	07201045
Calvert Cliffs Calvert Cliffs Nuclear Power Plant, Inc.	SL	Transnuclear, Inc.	NUHOMS®-24P NUHOMS®-32P	07200008
Catawba Duke Energy Carolinas, LLC	GL	NAC International, Inc.	NAC-UMS	07200045
Columbia Generating Station Energy Northwest	GL	Holtec International	HI-STORM 100	07200035
Comanche Peak Luminant Generation Company, LLC	GL	Holtec International	HI-STORM 100	07200074
Cook Indiana/Michigan Power	GL	Holtec International	HI-STORM	07200072
Cooper Nuclear Station Nebraska Public Power District	GL	Transnuclear, Inc.	NUHOMS®-61BT	07200066
Davis-Besse FirstEnergy Nuclear Operating Company	GL	Transnuclear, Inc.	NUHOMS®-24P	07200014
Diablo Canyon Pacific Gas & Electric Co.	SL	Holtec International	HI-STORM 100	07200026
Dresden Exelon Generation Company, LLC	GL	Holtec International	HI-STAR 100 HI-STORM 100	07200037
Duane Arnold Next Era Energy Duane Arnold, LLC.	GL	Transnuclear, Inc.	NUHOMS®-61BT	07200032
Fermi	GL	Holtec International	HI-STORM 100	07200071

APPENDIX P
Dry Cask Spent Fuel Storage Licensees (continued)

Name Licensee	License Type	Vendor	Storage Model	Docket Number
Fort Calhoun Omaha Public Power District	GL	Transnuclear, Inc.	NUHOMS®-32PT	07200054
Fort St. Vrain* U.S. Department of Energy	SL	FW Energy Applications, Inc.	Modular Vault Dry Store	07200009
Ginna Constellation Energy	GL	Transnuclear, Inc.	NUHOMS®-32PT	07200067
Grand Gulf Entergy Nuclear Operations, Inc.	GL	Holtec International	HI-STORM 100S	07200050
H.B. Robinson Carolina Power & Light Company	SL GL	Transnuclear, Inc. Transnuclear, Inc.	NUHOMS®-7P NUHOMS®-24P	07200003 07200060
Haddam Neck CT Yankee Atomic Power	GL	NAC International, Inc.	NAC-MPC	07200039
Hatch Southern Nuclear Operating, Inc.	GL	Holtec International	HI-STAR 100 HI-STAR 100	07200036
Hope Creek/Salem PSEG Nuclear, LLC	GL	Holtec International	HI-STORM 100	07200048
Humboldt Bay Pacific Gas & Electric Co.	SL	Holtec International	HI-STORM 100HB	07200027
Idaho National Lab TMI-2 Fuel Debris, U.S. Department of Energy	SL	Transnuclear, Inc.	NUHOMS®-12T	07200020
Idaho Spent Fuel Facility Environmental Corp.	SL	Foster Wheeler	Concrete Vault	07200025
Indian Point Entergy Nuclear Operations, Inc.	GL	Holtec International	HI-STORM 100	07200051
James A. FitzPatrick Entergy Nuclear Operations, Inc.	GL	Holtec International	HI-STORM 100	07200012
Joseph M. Farley Southern Nuclear Operating Co.	GL	Transnuclear, Inc.	NUHOMS®-32PT	07200042
Kewaunee Northern States Power Co., Minnesota	GL	Transnuclear, Inc.	NUHOMS®-39PT	07200064
La Salle Exelon Generation Co., LLC	GL	Holtec International	HI-STORM100	07200070
Lacrosse Dairyland Power	GL	NAC International, Inc	NAC-MPC	07200046
Limerick Exelon Generation Co., LLC	GL	Transnuclear, Inc.	NUHOMS®-61BT	07200065

APPENDIX P

Dry Cask Spent Fuel Storage Licensees (continued)

Name Licensee	License Type	Vendor	Storage Model	Docket Number
Maine Yankee Maine Yankee Atomic Power Company	GL	NAC International, Inc.	NAC-UMS	07200030
McGuire Duke Energy, LLC	GL	Transnuclear, Inc.	TN-32	07200038
Millstone Dominion Generation	GL	Transnuclear, Inc.	NUHOMS®-32PT	07200047
Monticello Northern States Power Co., Minnesota	GL	Transnuclear, Inc.	NUHOMS®-61BT NUHOMS®-61BTH	07200058
Nine Mile Point Constellation Energy	GL	Transnuclear, Inc.	NUHOMS®-61BT	07201036
North Anna Virginia Electric & Power Company (Dominion Gen.)	SL GL	Transnuclear, Inc. Transnuclear, Inc.	TN-32 NUHOMS®HD32PTH	07200016 07200056
Oconee Duke Energy Company	SL GL	Transnuclear, Inc. Transnuclear, Inc.	NUHOMS®-24P NUHOMS®-24P	07200004 07200040
Oyster Creek AmerGen Energy Company, LLC.	GL	Transnuclear, Inc.	NUHOMS®-61BT	07200015
Palisades Energy Nuclear Operations, Inc.	GL	Energy Solutions, Inc.	VSC-24 NUHOMS®-32PT	07200007
Palo Verde Arizona Public Service Co.	GL	NAC International, Inc.	NAC-UMS	07200044
Peach Bottom Exelon Generation Company, LLC	GL	Transnuclear, Inc.	TN-68	07200029
Perry FirstEnergy	GL	Holtec International	HI-STORM	07200069
Pilgrim	GL	Holtec International	HI-STORM 100	07201044
Point Beach FLP Energy Point Beach, LLC	GL	Energy Solutions, Inc.	VSC-24 NUHOMS®-32PT	07200005
Prairie Island Northern States Power Co., Minnesota	SL	Transnuclear, Inc.	TN-40 HT TN-40	07200010
Private Fuel Storage Facility	SL	Holtec International	HI-STORM 100	07200022
Quad Cities Exelon Generation Company, LLC	GL	Holtec International	HI-STORM 100S	07200053
Rancho Seco Sacramento Municipal Utility District	SL	Transnuclear, Inc.	NUHOMS®-24P	07200011

APPENDIX P
Dry Cask Spent Fuel Storage Licensees (continued)

Name Licensee	License Type	Vendor	Storage Model	Docket Number
River Bend Entergy Nuclear Operations, Inc.	GL	Holtec International	HI-STORM 100S	07200049
Salem PSEG Nuclear	GL	Holtec International	HI-STORM	07200048
San Onofre Southern California Edison Company	GL	Transnuclear, Inc.	NUHOMS®-24PT	07200041
Seabrook FPL Energy	GL	Transnuclear, Inc.	NUHOMS®-HD-32PTH	07200061
Sequoyah Tennessee Valley Authority	GL	Holtec International	HI-STORM 100	07200034
St. Lucie Florida Power and Light Company	GL	Transnuclear, Inc.	NUHOMS®-HD-32PTH	07200061
Surry Virginia Electric & Power Company (Dominion Gen.)	SL	General Nuclear Systems, Inc. Transnuclear, Inc. General Nuclear Westinghouse, Inc.	CASTOR V/21 TN-32 NAC-128 CASTOR X/33 MC-10	07200002
Susquehanna PPL Susquehanna, LLC	GL	Transnuclear, Inc.	NUHOMS®HD32PTH NUHOMS®-52B NUHOMS®-61BT NUHOMS®-61BTH	07200028
Trojan Portland General Electric Corp.	SL	Holtec International	HI-STORM 100	07200017
Turkey Point ISFSI Florida Power and Light Company	GL	Transnuclear, Inc.	NUHOMS®-HD-32PTH	07200062
Vermont Yankee Entergy Nuclear Operations, Inc.	GL	Holtec International	HI-STORM100	07200059
Vogtle Southern Company	GL	Holtec International	HI-STORM 100S	07201039
Waterford Steam Electric Station Entergy Nuclear Operations, Inc.	GL	Holtec International	HI-STORM 100	07200075
Yankee Rowe Yankee Atomic Electric	GL	NAC International, Inc.	NAC-MPC	07200031
Zion Zion Solutions, LLC	GL	NAC International, Inc.	Magnastor	07201037

* Fort St. Vrain is undergoing decommissioning and was transferred to DOE on June 4, 1999.

Note: NRC-abbreviated unit names.

APPENDIX Q

U.S. Low-Level Radioactive Waste Disposal Compact Membership

Appalachian Compact

Delaware
Maryland
Pennsylvania
West Virginia

Atlantic Compact

Connecticut
New Jersey
South Carolina*

Central Compact

Arkansas
Kansas
Louisiana
Oklahoma

Central Midwest Compact

Illinois
Kentucky

Midwest Compact

Indiana
Iowa
Minnesota
Missouri
Ohio
Wisconsin

Northwest Compact

Alaska
Hawaii
Idaho
Montana
Oregon
Utah*
Washington*
Wyoming

Rocky Mountain Compact

Colorado
Nevada
New Mexico
(Northwest accepts Rocky Mountain waste as agreed between compacts)

Southeast Compact

Alabama
Florida
Georgia
Mississippi
Tennessee
Virginia

Southwestern Compact

Arizona
California
North Dakota
South Dakota

Texas Compact

Texas*
Vermont

Unaffiliated

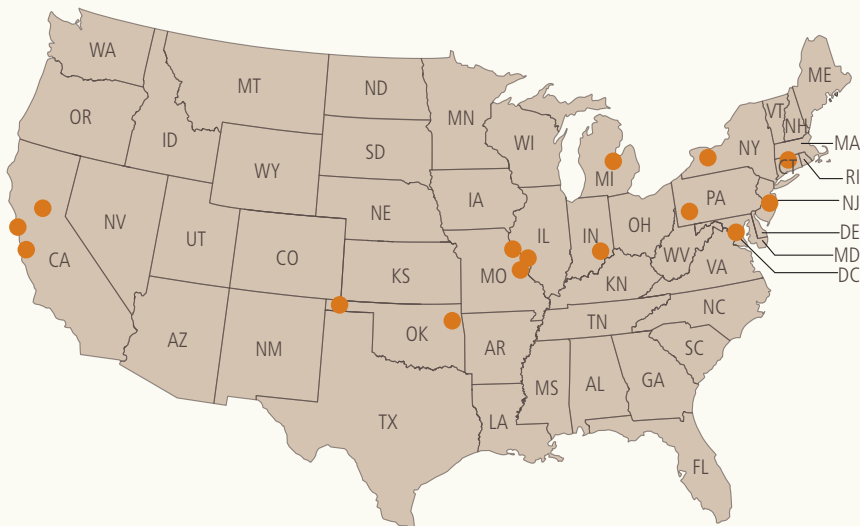
District of Columbia
Maine
Massachusetts
Michigan
Nebraska
New Hampshire
New York
North Carolina
Puerto Rico
Rhode Island

Closed Low-Level Waste Disposal Facility Sites Licensed by the NRC or Agreement States

Beatty, NV, closed 1993
Sheffield, IL, closed 1978
Maxey Flats, KY, closed 1977
West Valley, NY, closed 1975

* Site of an active LLW disposal facility.
Note: Data are as of May 2015.

APPENDIX R NRC-Regulated Complex Material Sites Undergoing Decommissioning



● NRC-regulated complex material sites (15)

Company	Location
AAR Manufacturing, Inc. (Brooks & Perkins)	Livonia, MI
Alameda Naval Air Station	Alameda, CA
Army, Department of, Jefferson Proving Ground	Madison, IN
Babcock & Wilcox SLDA	Vandergrift, PA
Beltsville Agricultural Research Center	Beltsville, MD
FMRI	Muskogee, OK
Hunter's Point Naval Shipyard	San Francisco, CA
Cimarron Environmental Trust	Cimarron, OK
Mallinckrodt Chemical, Inc.	St. Louis, MO
McClellan Air Force Base	Sacramento, CA
Sigma Aldrich	Maryland Heights, MO
Stepan Chemical Corporation	Maywood, NJ
UNC Naval Products	New Haven, CT
West Valley Demonstration Project	West Valley, NY
Westinghouse Electric Corporation—Hematite	Festus, MO

Note: Data are as of May 2015.

Picatinny Arsenal in NJ managed by Region I is not considered "complex."

APPENDIX S

Nuclear Power Units by Nation

Country	Nuclear Power Production GWh*	In Operation		Under Construction or on Order		Shutdown
		Number of Units	Capacity Net MWe	Number of Units	Capacity Net MWe	
Argentina	5,257	3	1625	1	25	0
Armenia	2,266	1	376	0	0	1 ^P
Belarus	0	0	0	2	2,218	0
Belgium	33,093	7	5,927	0	0	1 ^P
Brazil	15,385	2	1,884	1	1,245	0
Bulgaria	15,014	2	1,926	0	0	4 ^P
Canada	100,921	19	13,500	0	0	6 ^P
China	130,580	27	23,025	24	23,738	0
Czech Republic	28,636	6	3,904	0	0	0
Finland	22,654	4	2,752	1	1,600	0
France	415,900	58	63,130	1	1,600	12 ^P
Germany	91,783	9	12,074	0	0	27 ^P
Hungary	14,778	4	1,889	0	0	0
India	33,231	21	5,308	6	3,990	0
Iran	4,140	1	915	0	0	0
Italy	0	0	0	0	0	4 ^P
Japan	0	43	40,290	2	2,650	11 ^P & 1 ^L
Kazakhstan	0	0	0	0	0	1 ^P
Rep. Korea	149,165	24	21,667	4	5,420	0
Lithuania	0	0	0	0	0	2 ^P
Mexico	9,312	2	1,330	0	0	0
Netherlands	3,874	1	482	0	0	1 ^P
Pakistan	4,610	3	690	2	630	0
Romania	10,754	2	1,300	0	0	0
Russia	169,049	34	24,654	9	7,371	5 ^P
Slovakia	14,420	4	1,814	2	880	3 ^P
Slovenia	6,061	1	688	0	0	0
South Africa	14,749	2	1,860	0	0	0
Spain	54,832	7	7,121	0	0	1 ^L
Sweden	62,270	10	9,470	0	0	3 ^P

APPENDIX S

Nuclear Power Units by Nation (continued)

Country	Nuclear Power Production GWh*	In Operation		Under Construction or on Order		Shutdown
		Number of Units	Capacity Net MWe	Number of Units	Capacity Net MWe	
Switzerland	26,370	5	3,333	0	0	1 ^P
Ukraine	83,123	15	13,107	2	1,900	4 ^P
United Arab Emirates	0	0	0	3	4,035	0 ^P
United Kingdom	57,918	16	9,373	0	0	29 ^P
United States	797,067	99	98,639	5	5,633	33 ^P
Total	2,410,373*	438	379,261	67	65,482	150^P & 2^L

* Annual electrical power production for 2014. Total includes information on Taiwan.

P = Permanent Shutdown

L = Long-Term Shutdown

Note: Operable, under construction, or on order; country's short-form name used; rounded to the nearest whole number.

Sources: IAEA Power Reactor Information System Database; analysis compiled by the NRC, April 2015.

APPENDIX T

Nuclear Power Units by Reactor Type, Worldwide

Reactor Type	In Operation	
	Number of Units	Net MWe
Pressurized light-water reactors (PWR)	279	261,052
Boiling light-water reactors (BWR)	78	74,686
Heavy-water reactors, all types (HWR) (PHWR)	49	24,549
Light-water-cooled graphite-moderated reactor (LWGR)	15	10,219
Gas-cooled reactors, all types (GCR)	15	8,175
Liquid-metal-cooled fast-breeder reactors (FBR)	2	580
Total	438	379,261

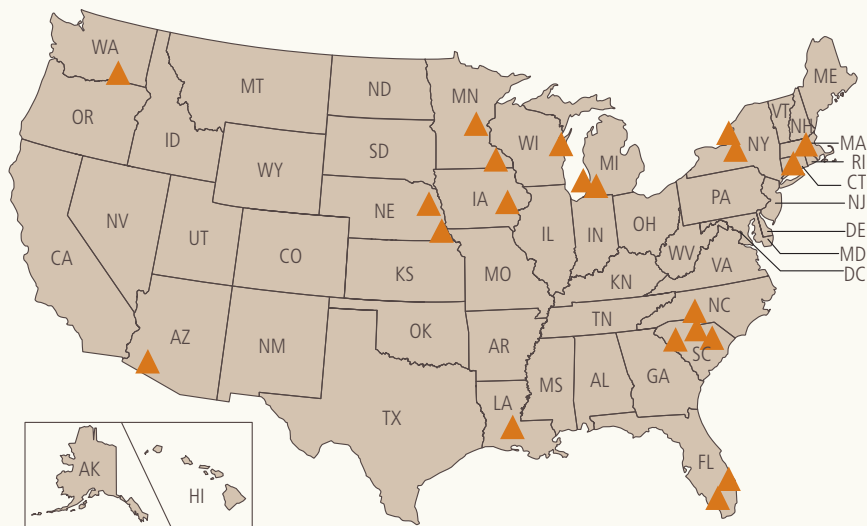
Note: MWe values rounded to the nearest whole number.

Source: IAEA Power Reactor Information System Database, www.iaea.org

Compiled by the NRC from data available as of June 2015.

APPENDIX U

Native American Reservations and Trust Lands within a 50-Mile Radius of a Nuclear Power Plant



ARIZONA

Palo Verde
Ak-Chin Indian Community
Tohono O'odham
Trust Land
Gila River Reservation
Maricopa Reservation

CONNECTICUT

Millstone
Mohegan Reservation
Mashantucket Pequot
Reservation
Narragansett
Reservation

FLORIDA

St. Lucie
Brighton Reservation
(Seminole Tribes
of Florida)
Fort Pierce Reservation
Turkey Point
Hollywood Reservation
(Seminole Tribes
of Florida)
Micosukee Reservation
Micosukee Trust Land

IOWA

Duane Arnold
Sac & Fox Trust Land
Sac & Fox Reservation

KANSAS

Iowa Reservation
Iowa Trust Land

LOUISIANA

River Bend
Tunica-Biloxi Reservation

MASSACHUSETTS

Pilgrim
Wampanoag
Tribe of Gay Head
(Aquinnah)
Trust Land

MICHIGAN

Palisades
Pottawatomi Reservation
Matcheбенашshewish
Band
Pokagon Reservation
Pokagon Trust Land*

DC Cook
Pokagon Reservation
Pokagon Trust Land

MINNESOTA

Monticello
Shakopee Community
Shakopee Trust Land
Mille Lacs Reservation

Prairie Island
Prairie Island Community*
Prairie Island Trust Land*
Shakopee Community
Shakopee Trust Land

NEBRASKA

Cooper
Sac & Fox Trust Land
Sac & Fox Reservation
Iowa Reservation
Kickapoo

Fort Calhoun
Omaha Reservation
Winnebago Reservation
Winnebago Trust Land

NEW YORK

FitzPatrick
Onondaga Reservation
Onida Reservation

Nine Mile Point
Onondaga Reservation
Onida Reservation

NORTH CAROLINA

McGuire
Catawba Reservation

SOUTH CAROLINA

Catawba
Catawba Reservation

Oconee
Eastern Cherokee
Reservation

Summer
Catawba Reservation

WASHINGTON

Columbia
Yakama Reservation
Yakama Trust Land

WISCONSIN

Point Beach
Oneida Trust Land
Oneida Reservation

* Tribe is located within the 10-mile emergency preparedness zone.

San Onofre and Kewaunee ceased operations.

Note: This table uses NRC-abbreviated reactor names and Native American Reservation and Trust land names.

There are no reservations or Trust lands within 50-miles of a reactor in Alaska or Hawaii. For more information on other Tribal concerns, go to the NRC Web site at: www.nrc.gov.

NRC-abbreviated reactor names listed

APPENDIX V

Regulatory Research Cooperative Agreements and Grants

American Nuclear Society	Support for the development and maintenance of probabilistic risk assessment (PRA)-related standards
American Society of Mechanical Engineers	Support in the following areas: Committee on Nuclear Risk Management on PRA standards, nuclear risk management, code comparison for the Multinational Design Evaluation Program, and harmonization of codes
Electric Power Research Institute	Research on irradiation-assisted stress-corrosion cracking and irradiation-assisted degradation of vessel internals materials
International Commission on Radiological Protection	Research on radiological protection standards
Kansas State University	Development of a High-Fidelity Model with Depleted Fuel for the Kansas State TRIGA Mark II Reactor
National Academy of Sciences	Support in the following areas: The Committee on Geological and Geotechnical Engineering, research to develop a consensus on the assessment of soil liquefaction potential and the related infrastructure consequences, research on porous rocks
National Council on Radiation Protection Measurements Inc.	Radiation Protection Guidance for the United States: SC 1-23 Guidance on Radiation Dose Limits for the Lens of the Eye and radiation protection guidance for the United States
Northwestern University	Service Lifetime Extension of Nuclear Power Plants: Prediction of Concrete Aging and Deterioration Through Accelerated Tests, Nondestructive Evaluation, and Stochastic Multiscale Computations
Ohio State University	Severe Accident Management Guidelines (SAMG) Validation within the Context of Severe Accident Uncertainties
Pennsylvania State University	Research on the Minimum Film Boiling Temperature for Nuclear Fuel Rods with Different Types of Fouling Layers and experiments on scalability of horizontal two-phase flow in small and large diameter pipes
Purdue University	Steel Plate Composite Walls: Behavior, Analysis and Design for Missile Impact
Texas A&M University	Research on bypass flow in prismatic reactor blocks and prolonged station blackout conditions
The Regents of the University of Colorado	Experimental and Numerical Investigation of Alkali Silica Reaction in Nuclear Reactors
University of California, Los Angeles	Methodological and Software Enhancements of Dynamic PRA Platforms for Event Assessment Applications
University of California-Berkeley	Work on ground motion prediction models for central and eastern North America and postliquefaction residual strength
University of Florida	Synergistic Effect of Thermal Aging and Low-Dose Irradiation on the Cast Stainless Steels and Stainless Steel Welds
University of Illinois	Validation of the PARCS/PATHS/SCALE for PWR Depletion Using the BEAVRS Benchmark
University of Maryland	Study of the Implications of Multi-Unit Accidents in the Context of NRC's Quantitative Health Objectives
University of Michigan	Multiscale thermal hydraulic tool for nuclear power plant safety analyses and assessment of Trace interfacial area transport equations against high resolution experiments of two-phase vertical flows
University of Tennessee	Research on seismic hazards and associated ground motion for the East Tennessee Seismic Zone
University of Toronto, Ontario	Research to develop a tool to confirm safety margins for modular steel-concrete composite constructions under seismic loads

APPENDIX W

Significant Enforcement Actions Issued, 2014

Significant (escalated) Enforcement Actions include notices of violation (NOVs) for severity level (SL) I, II, or III violations; NOVs associated with inspection findings that the significance determination process (SDP) categorizes as white, yellow, or red; civil penalties (CPs); and enforcement-related orders. The NRC Enforcement Policy also allows related violations to be categorized collectively as a single problem. Escalated enforcement actions are issued to reactor, materials, and individual licensees; nonlicensees; and fuel cycle facility licensees.

Action #	Name	Type	Issue Date	Enforcement Action
EA-12-140	South Carolina Electric & Gas Co. (Summer)	Reactor	03/10/2014	Confirmatory Order result of an ADR mediation
EA-13-026	University Nuclear and Diagnostics	Materials	05/13/2014	NOV SL III
EA-13-059	Centro De Medicina Nuclear CE	Materials	04/08/2014	Order Imposing CP of \$7,000
EA-13-076	Entergy Nuclear IP2 LLC (Indian Point)	Reactor	04/29/2014	NOV SL III
EA-13-078	Entergy Nuclear IP2 LLC (Indian Point)	Reactor	04/29/2014	NOV SL III
EA-13-105	Geisser Engineering Corporation	Materials	03/20/2014 07/31/2014	NOV/CP SL II - \$11,200 (CP subsequently reduced) Order Imposing CP of \$8,400
EA-13-136	Geisser Engineering Corporation	Materials	03/20/2014	NOV SL II
EA-13-196	Chicago Bridge and Iron Company (Lake Charles, LA)	Reactor	09/25/2014	Confirmatory Order result of an ADR mediation
EA-13-215	Valley Quarries, Inc.	Materials	02/06/2014	Problem/CP SL III - \$3,500
EA-13-222	Omaha Public Power District (Fort Calhoun)	Reactor	04/25/2014	NOV White SDP finding resulting in plant inspections
EA-13-223	FPL Energy Duane Arnold, LLC (Duane Arnold)	Reactor	02/11/2014	NOV White SDP finding resulting in plant inspections
EA-13-227	Tetra Tech, Inc.	Materials	02/24/2014	NOV SL III
EA-13-233	Entergy Operations, Inc. (Waterford 3)	Reactor	03/28/2014	NOV White SDP finding resulting in plant inspections
EA-13-240	Wittnauer Worldwide, LP	Materials	10/24/2014	NOV SL III
EA-13-244	Kuehne Company	Materials	03/20/2014	NOV SL III
EA-13-247	R.E. Ginna Nuclear Power Plant, LLC	Reactor	04/17/2014	NOV White SDP finding resulting in plant inspections
EA-13-251	ATC Group Services, Inc., d/b/a Car	Materials	11/19/2014	NOV/CP SL III - \$3,500
EA-14-001	City of Kirksville, Missouri	Materials	03/17/2014	NOV SL III
EA-14-005	Tennessee Valley Authority (Browns Ferry)	Reactor	04/30/2014 05/01/2014	NOV White SDP finding resulting in plant inspections Confirmatory Order result of an ADR mediation

APPENDIX W

Significant Enforcement Actions Issued, 2014 (continued)

Action #	Name	Type	Issue Date	Enforcement Action
EA-14-008	Entergy Operations, Inc. (Arkansas Nuclear One, Units 1 and 2)	Reactor	06/23/2014	2 NOV Yellow SDP findings resulting in plant inspections
EA-14-009	Entergy Operations, Inc. (River Bend)	Reactor	12/03/2014	Confirmatory Order, with a CP of \$70,000, result of ADR mediation
EA-14-013	Entergy Nuclear Operations, Inc. (Palisades)	Reactor	07/21/2014	Confirmatory Order result of an ADR mediation
EA-14-017	Southern Nuclear Operating Co., Inc. (Farley)	Reactor	02/14/2014	NOV White SDP finding resulting in plant inspections
EA-14-024	Wolf Creek Nuclear Operating Corp. (Wolf Creek)	Reactor	07/01/2014	NOV White SDP finding resulting in plant inspections
EA-14-026	Dominion NDT Services, Inc.	Materials	04/02/2014	NOV SL III
EA-14-028	IUPUI/Indiana University Medical Center	Materials	04/03/2014	NOV SL III
EA-14-029	ECS Carolinas, LLP	Materials	04/23/2014	Problem SL III
EA-14-030	Dominion Engineering Associates, Inc.	Materials	12/18/2014	NOV/CP SL III - \$3,500
EA-14-072	Metro Cardiovascular Diagnostics	Materials	09/30/2014	NOV/CP SL III - \$3,500
EA-14-075	ConAgra Foods	Materials	08/01/2014	Problem SL III
EA-14-089	Idahoan Foods	Materials	10/16/2014	Problem SL III
EA-14-091	Duke Energy Corp. (Oconee 1)	Reactor	08/12/2014	NOV White SDP finding resulting in plant inspections
EA-14-092	Dominion Nuclear Connecticut, Inc. (Millstone 3)	Reactor	10/20/2014	NOV White SDP finding resulting in plant inspections
EA-14-094	FirstEnergy Nuclear Operating Co. (Davis-Besse)	Reactor	06/30/2014	Confirmatory Order
EA-14-100	Calvert Cliffs Nuclear Power Plant, Inc. (Calvert Cliffs)	Reactor	10/27/2014	NOV White SDP finding resulting in plant inspections
EA-14-106	City of St. Peters, Missouri	Materials	08/19/2014	NOV SL III
EA-14-108	Diagnostic Imaging Centers, P.A.	Materials	08/26/2014	NOV SL III
EA-14-112	Southern Nuclear Operating Co., Inc. (Vogtle)	Reactor	08/06/2014	NOV White SDP finding resulting in plant inspections
EA-14-113	Kruger Technologies Inc.	Materials	10/28/2014	Confirmatory Order result of an ADR mediation

APPENDIX W

Significant Enforcement Actions Issued, 2014 (continued)

Action #	Name	Type	Issue Date	Enforcement Action
EA-14-115	Truman Medical Center	Materials	10/08/2014	NOV SL III
EA-14-116	Bradley D. Bastow, D.O.	Materials	11/06/2014	Problem/CP SL III - \$7,000
EA-14-167	Kim Engineering	Materials	12/23/2014	NOV SL III
EA-14-187	Omaha Public Power District (Fort Calhoun)	Reactor	11/25/2014	NOV White SDP finding resulting in plant inspections
IA-12-045	Mr. Michael P. Cooley	Reactor	03/10/2014	Prohibition Order
IA-13-012	Mr. Armando N. Clavero	Materials	05/13/2014	Prohibition Order
IA-13-033	Mr. George Geisser, III	Materials	07/17/2014	Prohibition Order
IA-13-055	Mr. John Amburgey	Reactor	01/13/2014	NOV SL III
IA-13-059	Mr. Richard B. Smith	Reactor	05/14/2014	Prohibition Order
IA-13-064	Mr. Daniel L. Wilson	Reactor	04/29/2014	Prohibition Order
IA-14-002	Mr. Lane McHugh	Reactor	02/24/2014	NOV SL III
IA-14-004	Mr. Douglas D. Stouffer	Reactor	04/14/2014	NOV SL III
IA-14-007	Mr. Trey Brattin	Materials	07/24/2014	NOV SL III
IA-14-014	Mr. Donald K. Brown	Reactor	07/09/2014	NOV SL III
IA-14-021	Mr. Gary Meekins	Reactor	07/09/2014	NOV SL III
IA-14-025	Mr. James Chaisson	Materials	07/11/2014	Prohibition Order

Note: Reactor facilities in a decommissioning status are listed as materials licensees. The NRC report on Issued Significant Enforcement Actions can be found on the NRC Web site at www.nrc.gov/about-nrc/regulatory/enforcement/current.html.

APPENDIX X

Quick-Reference Metric Conversion Tables

SPACE AND TIME

Quantity	From Inch-Pound Units	To Metric Units	Multiply by
Length	mi (statute)	km	1.609347
	yd	m	*0.9144
	ft (int)	m	*0.3048
	in	cm	*2.54
Area	mi ²	km ²	2.589998
	acre	m ²	4,046.873
	yd ²	m ²	0.8361274
	ft ²	m ²	*0.09290304
	in ²	cm ²	*6.4516
Volume	acre foot	m ³	1,233.489
	yd ³	m ³	0.7645549
	ft ³	m ³	0.02831685
	ft ³	L	28.31685
	gal	L	3.785412
	fl oz	mL	29.57353
	in ³	cm ³	16.38706
Velocity	mi/h	km/h	1.609347
	ft/s	m/s	*0.3048
Acceleration	ft/s ²	m/s ²	*0.3048

NUCLEAR REACTION AND IONIZING RADIATION

Quantity	From Inch-Pound Units	To Metric Units	Multiply by
Activity (of a radionuclide)	curie (Ci)	MBq	*37,000.0
	dpm	becquerel (Bq)	0.016667
Absorbed dose	rad	gray (Gy)	*0.01
	rad	cGy	*1.0
Dose equivalent	rem	sievert (Sv)	*0.01
	rem	mSv	*10.0
	mrem	mSv	*0.01
	mrem	μSv	*10.0
Exposure (X-rays and gamma rays)	roentgen (R)	C/kg (coulomb)	0.000258

APPENDIX X

Quick-Reference Metric Conversion Tables (continued)

HEAT

Quantity	From Inch-Pound Units	To Metric Units	Multiply by
Thermodynamic temperature	°F	K	*K = (°F + 459.67)/1.8
Celsius temperature	°F	°C	*°C = (°F - 32)/1.8
Linear expansion coefficient	1/°F	1/K or 1/°C	*1.8
Thermal conductivity	(Btu • in)/(ft ² • h • °F)	W/(m • °C)	0.1442279
Coefficient of heat transfer	Btu / (ft ² • h • °F)	W/(m ² • °C)	5.678263
Heat capacity	Btu/°F	kJ/°C	1.899108
Specific heat capacity	Btu/(lb • °F)	kJ/(kg • °C)	*4.1868
Entropy	Btu/°F	kJ/°C	1.899108
Specific entropy	Btu/(lb • °F)	kJ/(kg • °C)	*4.1868
Specific internal energy	Btu/lb	kJ/kg	*2.326

MECHANICS

Quantity	From Inch-Pound Units	To Metric Units	Multiply by
Mass (weight)	ton (short)	t (metric ton)	*0.90718474
	lb (avdp)	kg	*0.45359237
Moment of mass	lb • ft	kg • m	0.138255
Density	ton (short)/yd ³	t/m ³	1.186553
	lb/ft ³	g/m ³	16.01846
Concentration (mass)	lb/gal	g/L	119.8264
Momentum	lb • ft/s	kg • m/s	0.138255
Angular momentum	lb • ft ² /s	kg • m ² /s	0.04214011
Moment of inertia	lb • ft ²	kg • m ²	0.04214011
Force	kip (kilopound)	kN (kilonewton)	4.448222
	lbf	N (newton)	4.448222
Moment of force, torque	lbf • ft	N • m	1.355818
	lbf • in	N • m	0.1229848
Pressure	atm (std)	kPa (kilopascal)	*101.325
	bar	kPa	*100.0
	lbf/in ² (formerly psi)	kPa	6.894757
	inHg (32 °F)	kPa	3.38638
	ftH ₂ O (39.2 °F)	kPa	2.98898
	inH ₂ O (60 °F)	kPa	0.24884
	mmHg (0 °C)	kPa	0.133322

APPENDIX X

Quick-Reference Metric Conversion Tables (continued)

MECHANICS (continued)			
Quantity	From Inch-Pound Units	To Metric Units	Multiply by
Stress	kip/in ² (formerly ksi)	MPa	6.894757
	lbf/in ² (formerly psi)	MPa	0.006894757
	lbf/in ² (formerly psi)	kPa	6.894757
	lbf/ft ²	kPa	0.04788026
Energy, work	kWh	MJ	*3.6
	cal th	J (joule)	*4.184
	Btu	kJ	1.055056
	ft • lbf	J	1.355818
	therm (US)	MJ	105.4804
Power	Btu/s	kW	1.055056
	hp (electric)	kW	*0.746
	Btu/h	W	0.2930711

* Exact conversion factors

Note: The information contained in this table is intended to provide familiarization with commonly used SI units and provide a quick reference to aid in the understanding of documents containing SI units. The conversion factors provided have not been approved as NRC guidelines for the development of licensing actions, regulations, or policy.

To convert from metric units to inch-pound units, divide the metric unit by the conversion factor.

Sources: Federal Standard 376B (January 27, 1993), "Preferred Metric Units for General Use by the Federal Government," and International Commission on Radiation Units and Measurements, ICRU Report 33 (1980), "Radiation Quantities and Units"

APPENDIX Y

Progress List of Rulemaking for Nuclear Material and Waste

- | | |
|----|---|
| 1. | 10 CFR Part 20, "Standards for Protection Against Radiation" |
| 2. | 10 CFR Part 35, "Medical Use of Byproduct Material" |
| 3. | 10 CFR Part 37, "Physical Protection of Category 1 and Category 2 Quantities of Radioactive Material" |
| 4. | 10 CFR Part 40, "Domestic Licensing of Source Material" |
| 5. | 10 CFR Part 61, "Licensing Requirements for Land Disposal of Radioactive Waste" |
| 6. | 10 CFR Part 73, "Physical Protection of Plants and Materials" |
| 7. | 10 CFR Part 74, "Material Control and Accounting of Special Nuclear Material" |

APPENDIX Z

Laws Governing the U.S. Nuclear Regulatory Commission

1. Atomic Energy Act of 1954, as amended (Pub. L. 83–703)
2. Energy Reorganization Act of 1974, as amended (Pub. L. 93–438)
3. Reorganization Plan No. 1 of 1980, 5 U.S.C., App 1.
4. Uranium Mill Tailings Radiation Control Act of 1978, as amended (Pub. L. 95–604)
5. Nuclear Non-Proliferation Act of 1978 (Pub. L. 95–242)
6. West Valley Demonstration Project Act of 1980 (Pub. L. 96–368)
7. Nuclear Waste Policy Act of 1982, as amended (Pub. L. 97–425)
8. Low-Level Radioactive Waste Policy Amendments Act of 1985 (Pub. L. 99–240)
9. Energy Policy Act of 1992 (Pub. L. 102–486)
10. Energy Policy Act of 2005 (Pub. L. 109–58)

Fundamental Laws Governing Civilian Uses of Radioactive Materials

Nuclear Materials and Facilities

1. Atomic Energy Act of 1954, as amended
2. Energy Reorganization Act of 1974, as amended
3. Reorganization Plan No. 1 of 1980

Radioactive Waste

1. Nuclear Waste Policy Act of 1982, as amended
2. Low-Level Radioactive Waste Policy Amendments Act of 1985
3. Uranium Mill Tailings Radiation Control Act of 1978

Nonproliferation

1. Nuclear Non-Proliferation Act of 1978

Fundamental Laws Governing the Processes of Regulatory Agencies

1. Administrative Procedure Act (5 U.S.C. Chapters 5 through 8)
2. National Environmental Policy Act

APPENDIX AA International Activities

CONVENTIONS AND TREATIES

Nuclear Treaties—But Not Touching on Arms Control Agreements

1. Treaty on the Non-Proliferation of Nuclear Weapons, entry into force on March 5, 1970, United States (U.S.) is a party to the Treaty
2. Treaty for the Prohibition of Nuclear Weapons in Latin America (Tlatelolco Treaty), entry into force for each government individually, U.S. is a party to the specific protocols appended to the Treaty
3. Three to Four other treaties specifying nuclear weapons free zones in Africa, the South Pacific (Rarotonga), and Southeast Asia, including one being negotiated on the Middle East; the U.S. is only bound by specific protocols

Safety and Security Treaties, Conventions and Agreements under IAEA Auspices

1. Convention on Early Notification of a Nuclear Accident, entry into force 27 October 1986, U.S. is a party
2. Convention on Assistance in the Case of a Nuclear Accident or Radiological Emergency, entry into force 26 February 1987, U.S. is a party
3. Convention on Nuclear Safety, entry into force 24 October 1996, U.S. is a party
4. Joint Convention on the Safety of Spent Fuel Management and on the Safety of Radioactive Waste Management, entry into force 18 June 2001, U.S. is a party
5. Convention on the Physical Protection of Nuclear Material (CPPNM), entry into force 8 February 1987, U.S. is a party
6. Convention on the Prevention of Marine Pollution by Dumping of Wastes and Other Matter, entry into force 30 August 1975, U.S. is a party (also to amendments in 1978 (incineration), 1978 (disputes), 1980 (list of substances), 1989 (procedures), 1993 (banning dumping into sea of low-level radioactive wastes), 1996 (protocol to replace the 1972 Convention with a more restrictive text regulating the use of the sea as a depository for waste materials)
7. Convention on Supplementary Compensation for Nuclear Damage

Safeguards Treaties, Conventions, and Agreements under IAEA Auspices

1. Model Protocol Additional to the Agreement Between State(s) and the IAEA for the Application of Safeguards, entry into force 6 April 1989, U.S. is a party

APPENDIX AA
International Activities (continued)

LIST OF IMPORT AND EXPORT LICENSES ISSUED FOR 2014

License Number	Applicant	Docket Number
IW012/05	PermaFix/DSSI	11005322
IW016/02	Eastern Technologies, Inc.	11005602
IW022/02	Perma-Fix Northwest, Inc.	11005700
IW022/03	Perma-Fix Northwest, Inc.	11005700
IW032	Eastern Technologies, Inc.	11006100
XB1328	Humboldt Scientific, Inc.	11006050
XB1328/01	Humboldt Scientific, Inc.	11006050
XB1329	Rapiscan Systems, Inc.	11006059
XB1329/01	Rapiscan Systems, Inc.	11006059
XB1329/02	Rapiscan Systems, Inc.	11006059
XCOM1133/03	Global Nuclear Fuel - Americas, LLC	11005160
XCOM1222/01	The Barden Corporation	11006016
XCOM1231	Weir Valves & Controls USA Inc.	11005948
XCOM1232	ENERTECH, a business unit of Curtis-Wright Control Corporation	11005950
XCOM1237	Mytech Corporation	11005975
XCOM1238	Technetics Group Columbia	11005978
XCOM1239/01	Louisiana Energy Services, LLC	11005981
XCOM1240	Technetics Group Columbia	11005982
XCOM1240/01	Technetics Group Columbia	11005982
XCOM1242	Materion Brush Inc.	11005991
XCOM1245	Transco Products Inc.	11005999
XCOM1246	Westinghouse Electric Company LLC	11006001
XCOM1250	General Atomics	11006028
XCOM1252	Westinghouse Electric Company LLC.	11006040
XCOM1254	GE Oil and Gas-Dresser, Inc.	11006048
XCOM1255	Westinghouse Electric Company LLC	11006060
XCOM1256	Kingsbury, Inc.	11006071
XCOM1257	Pall Corporation	11006076
XCOM1259	ATI Wah Chang	11006079
XCOM1260	ATI Wah Chang	11006080
XCOM1261	ENERTECH, a business unit of Curtiss-Wright Flow Control Corporation	11006084
XCOM1262	Westinghouse Electric Company LLC	11006085
XCOM1263	Circor Aerospace, Inc.	11006087
XCOM1265	Materion Brush Inc.	11006094
XMAT413/01	Linde Electronics and Specialty Gases	11005877
XMAT422	Cambridge Isotope Laboratories, Inc.	11005997
XMAT423/01	Linde Electronics and Specialty Gases	11006029
XMAT425	Cambridge Isotope Laboratories, Inc.	11006062
XMAT426	Cambridge Isotope Laboratories, Inc.	11006063

APPENDIX AA

International Activities (continued)

LIST OF IMPORT AND EXPORT LICENSES ISSUED FOR 2014 (continued)

License Number	Applicant	Docket Number
XMAT427	Airgas Specialty Gases	11006098
XMAT428	Linde Gas North America LLC	11006093
XMAT429	Matheson Tri Gas, Inc.	11006116
XR135/07	GE-Hitachi Nuclear Energy Americas LLC	11001075
XSNM3066/05	Global Nuclear Fuel - Americas, L.L.C.	11005081
XSNM3135/04	Global Nuclear Fuel-Americas, L.L.C.	11005186
XSNM3622/02	Department of Energy - Oak Ridge	11006024
XSNM3627/02-R	Mitsui & Co. (U.S.A.), Inc.	11005848
XSNM3679	Transnuclear, Inc.	11005923
XSNM3708/01	Department of Energy - Oak Ridge	11005974
XSNM3716/01	Mitsui & Co. (U.S.A.), Inc.	11005996
XSNM3722	Transnuclear, Inc.	11006019
XSNM3725-R	Department of Energy - Savannah River	11006035
XSNM3726	Department of Energy - Oak Ridge	11006037
XSNM3727	Department of Energy - Oak Ridge	11006049
XSNM3728	AREVA NP Inc.	11006052
XSNM3729	Department of Energy - Oak Ridge	11006053
XSNM3730	Department of Energy - Oak Ridge	11006054
XSNM3731	Mitsui & Co. (U.S.A.), Inc.	11006056
XSNM3732	Transport Logistics International, Inc.	11006057
XSNM3733	AREVA NP Inc.	11006065
XSNM3734	Mitsui & Co. (U.S.A.), Inc.	11006066
XSNM3735	Transnuclear Inc.	11006067

Non-Appendix P Components Guide

(XSNM) denotes export of special nuclear material (plutonium, uranium-233, or uranium enriched above 0.711 percent, by weight, in the isotope uranium-235).

(XCOM) denotes export of minor reactor COMponents or other nuclear facility (e.g., nuclear fabrication) components under NRC jurisdiction (refer to Title 10 of the *Code of Federal Regulations* Part 110, Export and Import of Nuclear Equipment and Material, Appendix A, Items (5)–(9) for minor reactor components and Appendices B–K and N–O for other nuclear facility components).

(XSOU) denotes export of source material (natural or depleted uranium; thorium; a mixture of uranium and thorium other than special nuclear material; or certain ores [e.g., tantalum and niobium that contain, by weight, 0.05 percent or more of the aforementioned materials for nonnuclear end use]).

(XB) denotes export of byproduct material, 10 CFR Part 110, Appendix L, for an illustrative list of byproduct materials under NRC jurisdiction.

(XR) denotes export of reactor facilities, 10 CFR Part 110, Appendix A, Items (1) – (4).

(IW) denotes import of radioactive waste.

(XW) denotes export of radioactive waste.

APPENDIX AA

International Activities (continued)

LIST OF IMPORT AND EXPORT LICENSES ISSUED FOR 2014 (continued)

License Number	Applicant	Docket Number
XSNM3736	Westinghouse Electric Company LLC	11006069
XSNM3737	AREVA NP Inc.	11006072
XSNM3738	Transport Logistics International	11006076
XSNM3739	Department of Energy - Oak Ridge	11006078
XSNM3740	Transport Logistics International	11006081
XSNM3741	AREVA NP Inc.	11006083
XSNM3742	AREVA NP Inc.	11006086
XSNM3743-R	Transport Logistics International, Inc.	11006095
XSNM3744	Transport Logistics International, Inc.	11006097
XSNM3745	Department of Energy - Oak Ridge	11006098
XSNM3746	Transnuclear Inc.	11006109
XSNM3747	AREVA NP Inc.	11006110
XSNM3748	Transport Logistics International, Inc.	11006111
XSOU8789/06	ConverDyn	11005360
XSOU8798/04	RSB Logistics	11005445
XSOU8828/01	Global Advanced Metals USA, Inc.	11006003
XW008/04	Diversified Scientific Services, Inc. (DSSI)	11005323
XW012/02	Perma-Fix Northwest, Inc.	11005699
XW012/03	Perma-Fix Northwest, Inc.	11005699
XW016/01	Eastern Technologies, Inc.	11005825
XW020	EnergySolutions	11006061
XW021	Eastern Technologies, Inc.	11006101

Appendix P to 10 CFR Part 110 Components Guide

Appendix P licenses support the use of radioactive sealed sources for a variety of medical, industrial, research, and educational activities. Some applicants have previously obtained a combined export/import license to allow export or import, use, resale, and import or export back to the supplier for recycling. These combined licenses are no longer appropriate and can no longer be amended going forward, given the authorization for imports of Appendix P materials under a general license (see 10 CFR 110.27, "General License for Import"). These combined import/export licenses needing amendment are converted to export-only licenses. The 2010 changes to 10 CFR Part 110 generally necessitate specific licenses for only Appendix P, Category 1 and 2 exports.

APPENDIX AA**International Activities (continued)****LIST OF THE NRC'S PARTICIPATION WITH MULTILATERAL ORGANIZATIONS****International Commission on Radiological Protection****International Atomic Energy Agency**

- Commission on Safety Standards
 - Nuclear Safety Standards Committee
 - Radiation Safety Standards Committee
 - Transport Safety Standards Committee
 - Waste Safety Standards Committee
- Nuclear Security Guidance Committee

International Nuclear Regulators Association**Multinational Design Evaluation Program****Nuclear Energy Agency**

- NEA Steering Committee for Nuclear Energy
- Committee on Nuclear Regulatory Activities
 - Working Group on Inspection Practices
 - Working Group on Operating Experience
 - Working Group on Public Communication of Nuclear Regulatory Organisations
 - Working Group on the Regulation of New Reactors
 - Safety of Research Reactors Task Group
 - Senior-Level Task Group on Impacts of the Fukushima Accident
 - Task Group of Nonconforming, Counterfeit, Fraudulent, and Suspect Items
 - Task Group on Accident Management
- Committee on Radiation Protection and Public Health
 - Expert Group on the Implications of ICRP Recommendations
 - Expert Group on Occupational Exposure
 - Expert Group on the Public Health Perspective in Radiological Protection
 - Expert Group on the Radiological Protection Aspects of the Fukushima Accident
 - Expert Group on Radiological Protection Sciences
 - Working Party on Nuclear Emergency Matters

APPENDIX AA

International Activities (continued)

LIST OF THE NRC'S PARTICIPATION WITH MULTILATERAL ORGANIZATIONS

- Committee on the Safety of Nuclear Installations
 - CSNI Program Review Group
 - Working Group on Integrity and Aging of Components and Structures
 - Working Group on Analysis and Management of Accidents
 - Working Group on Risk Assessment
 - Working Group on Human and Organisational Factors
 - Working Group on Fuel Safety
 - Working Group on Fuel Cycle Safety
- Radioactive Waste Management Committee
 - Forum on Stakeholder Confidence
 - Integration Group for the Safety Case of Radioactive Waste Repositories
 - Regulators' Forum
 - Working Party on Decommissioning and Dismantling

United Nations Scientific Committee on the Effects of Atomic Radiation

BILATERAL INFORMATION EXCHANGE AND COOPERATION AGREEMENTS WITH THE NRC

Agreement Country

Argentina	Finland	Korea, Rep. of	Spain
Armenia	France	Lithuania	Sweden
Australia	Germany	Mexico	Switzerland
Belgium	Greece	Netherlands	Tecro (Taiwan)
Brazil	Hungary	Peru	Thailand
Bulgaria	India	Philippines	Turkey
Canada	Indonesia	Poland	Ukraine
China	Israel	Romania	United Arab Emirates
Croatia	Italy	Russia	United Kingdom
Czech Republic	Japan	Slovakia	Vietnam
Egypt	Jordan	Slovenia	
EURATOM	Kazakhstan	South Africa	

Note: The country's short-form name is used. NRC's Technical Arrangements are initiated and renewed for 5-year terms.

EURATOM—The European Atomic Energy Community

Tecro (Taiwan) is the Taipei Economic and Cultural Representative Office in the United States.





Glossary

Glossary (Abbreviations and Terms Defined)

Agreement State

A U.S. State that has signed an agreement with the U.S. Nuclear Regulatory Commission authorizing the State to regulate certain uses of radioactive materials within the State.

Atomic energy

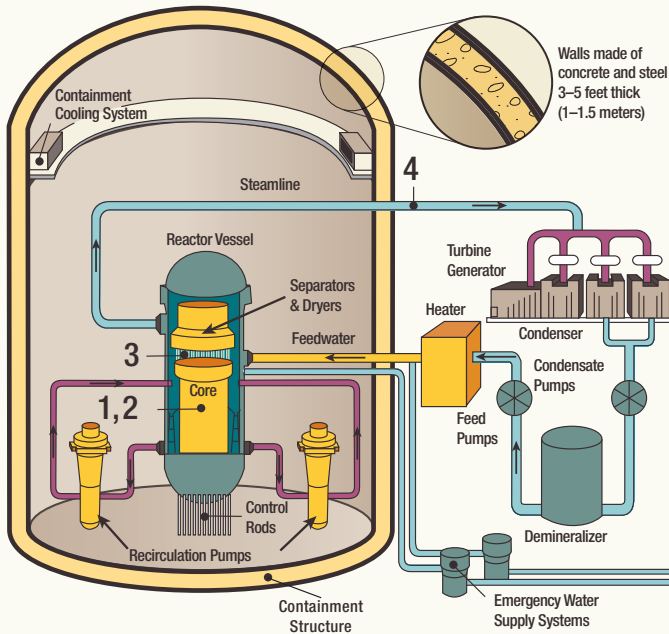
The energy that is released through a nuclear reaction or radioactive decay process. One kind of nuclear reaction is fission, which occurs in a nuclear reactor and releases energy, usually in the form of heat and radiation. In a nuclear power plant, this heat is used to boil water to produce steam that can be used to drive large turbines. The turbines drive generators to produce electrical power.

Background radiation

The natural radiation that is always present in the environment. It includes cosmic radiation that comes from the sun and stars, terrestrial radiation that comes from the Earth, and internal radiation that exists in all living things and enters organisms by ingestion or inhalation. The typical average individual exposure in the United States from natural background sources is about 310 millirems per year.

Boiling-water reactor (BWR)

A nuclear reactor in which water is boiled using heat released from fission. The resulting steam released by boiling then drives turbines and generators to produce electrical power. BWRs operate similarly to electrical plants using fossil fuel, except that the BWRs are heated by nuclear fission in the reactor core.



How Nuclear Reactors Work

In a typical design concept of a commercial BWR, the following process occurs:

1. The nuclear fuel core inside the reactor vessel creates heat from nuclear fission.
2. A steam-water mixture is produced when very pure water (reactor coolant) moves upward through the core, absorbing heat.
3. The steam-water mixture leaves the top of the core and enters the two stages of moisture separation where water droplets are removed before the steam is allowed to enter the steamline.
4. The steam is piped to the main turbine, causing it to turn the turbine generator, which produces electricity.
5. The steam is exhausted to the condenser, where it is condensed into water. The resulting water is pumped out of the condenser with a series of pumps and pumped back to the reactor vessel.

The reactor's core contains fuel assemblies that are cooled by water circulated using electrically powered pumps. These pumps and other operating systems in the plant receive their power from the electrical grid. If offsite power is lost, emergency cooling water is supplied by other pumps, which can be powered by onsite diesel generators. Other safety systems, such as the containment cooling system, also need electric power. BWRs contain between 370–800 fuel assemblies.

Brachytherapy

A medical procedure during which a sealed radioactive source (or sources) is implanted directly into a person being treated for cancer (usually of the mouth, breast, lung, prostate, ovaries, or uterus). The radioactive implant may be temporary or permanent, and the radiation kills cells in the tumor as long as the device remains in place and emits radiation. Brachytherapy uses radioisotopes, such as iridium-192 or iodine-125, which are regulated by the NRC and Agreement States.

Byproduct material

As defined by NRC regulations, byproduct material includes any radioactive material (except enriched uranium or plutonium) produced by a nuclear reactor. It also includes the tailings or wastes produced by the extraction or concentration of uranium or thorium or the fabrication of fuel for nuclear reactors, or through the use of a particle accelerator or any discrete source of radium-226 used for a commercial, medical, or research activity. It also includes the tailings or wastes produced by the extraction or concentration of uranium or thorium or the fabrication of fuel for nuclear reactors. In addition, the NRC, in consultation with the U.S. Environmental Protection Agency, U.S. Department of Energy, U.S. Department of Homeland Security, and others, can designate as byproduct material any source of naturally occurring radioactive material, other than source material, that it determines would pose a threat to public health and safety or the common defense and security of the United States.

Canister

See *Dry cask storage*.

Capability

The maximum load that a generating unit, generating station, or other electrical apparatus can carry under specified conditions for a given period of time without exceeding approved limits of temperature and stress.

Capacity

The amount of electric power that a generator, turbine transformer, transmission, circuit, or system is able to produce, as rated by the manufacturer.

Capacity factor

The ratio of the available capacity (the amount of electrical power actually produced by a generating unit) to the theoretical capacity (the amount of electrical power that could theoretically have been produced if the generating unit had operated continuously at full power) during a given time period.

Capacity utilization

A percentage that a generating unit fulfilled its capacity in generating electric power over a given time period. This percentage is defined as the margin between the unit's available capacity (the amount of electrical power the unit actually produced) and its theoretical capacity (the amount of electrical power that could have been produced if the unit had operated continuously at full power) during a certain time period. Capacity utilization is computed by dividing the amount of power actually produced by the theoretical capacity and multiplying by 100.

Cask

A heavily shielded container used for the dry storage or shipment (or both) of radioactive materials such as spent nuclear fuel or other high-level radioactive waste (HLW). Casks are often made from lead, concrete, or steel. Casks must meet regulatory requirements and are not intended for long-term disposal in a repository.

Category of radiation sources

The categories for radiation sources are defined by NRC requirements and the IAEA's Code of Conduct on the Safety and Security of Radioactive Sources. This helps ensure that sufficient controls are being used to achieve safety and security.

- Category 1 sources, if not safely or securely managed, would be likely to cause permanent injury to a person who handled them or was otherwise in contact with them for more than a few minutes. It would probably be fatal to be close to this amount of unshielded material for a period of a few minutes to an hour. These sources are typically used in radiothermal generators, irradiators, and radiation teletherapy.
- Category 2 sources, if not safely or securely managed, could cause permanent injury to a person who handled them or was otherwise in contact with them for a short time (minutes to hours). It could possibly be fatal to be close to this amount of unshielded radioactive material for a period of hours to days. These sources are typically used in industrial gamma radiography, high- and medium-dose rate brachytherapy and radiography.

- Category 3 sources, if not safely or securely managed, could cause permanent injury to a person who handled them or was otherwise in contact with them for hours. It could possibly—although it is unlikely—be fatal to be close to this amount of unshielded radioactive material for a period of days to weeks. These sources are typically used in fixed industrial gauges such as level gauges, dredger gauges, conveyor gauges, and spinning pipe gauges and well logging.

Categories of special nuclear material

The NRC categorizes special nuclear materials and the facilities that possess them into three categories based upon the materials' potential for use in nuclear weapons, or their "strategic significance." The three categories are:

- Category I: High strategic significance;
- Category II: Moderate strategic significance; and
- Category III: Low strategic significance.

The NRC's physical security and safeguards requirements differ by category, with Category I facilities subject to more stringent requirements because they pose greater security and safeguards risks.

Classified information

Information that has been determined pursuant to executive order to require protection against unauthorized disclosure and is marked to indicate its classified status when in documentary form. The NRC has two types of classified information. The first type, known as national security information, is information that is classified by an Executive order. Its release would damage national security. The second type, known as restricted data, would assist individuals or organizations in designing, manufacturing, or using nuclear weapons. Access to both types of information is restricted to authorized persons who have been properly cleared and have a "need to know" the information to accomplish their official duties.

Combined license (COL)

An NRC-issued license that authorizes a licensee to construct and (with certain specified conditions) operate a nuclear power facility, such as a nuclear plant at a specific site.

Commercial Irradiator

A facility that uses high doses of radiation to sterilize and treat products, such as food and spices, medical supplies, and wood flooring. Irradiation can be used to eliminate harmful bacteria, germs, and insects or for hardening or other purposes. The radiation does not leave radioactive residue or make the treated products radioactive. Radiation sources include radioactive materials (e.g., cobalt-60), an x-ray machine, or an electron beam.

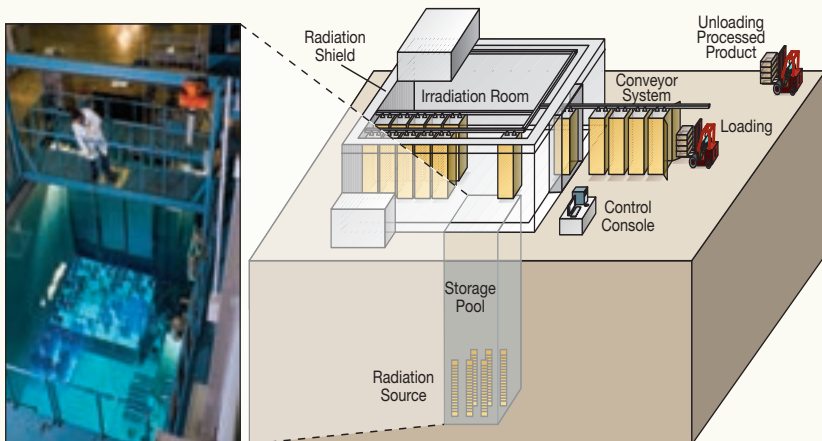


Photo courtesy: Nordion

Compact

A group of two or more U.S. States that have formed alliances to dispose of low-level radioactive waste (LLW).

Construction recapture

The maximum number of years that could be added to a nuclear power plant's license expiration date to recapture the period between the date the NRC issued the plant's construction permit and the date it granted an operating license. A licensee must submit an application to request this extension.

Containment structure

A gas-tight resilient shell or other enclosure around a nuclear reactor to confine fission products that otherwise might be released to the atmosphere in the event of a severe reactor accident. Such enclosures are usually dome-shaped and made of steel-reinforced concrete.

Contamination

Undesirable radiological, chemical, or biological material (with a potentially harmful effect) that is either airborne or deposited in (or on the surface of) structures, objects, soil, water, or living organisms.

Criticality

The condition involving fission of nuclear materials when the number of neutrons produced equals or exceeds the nuclear containment. During normal reactor operations, nuclear fuel sustains a fission chain reaction. A reactor achieves criticality (and is said to be critical) when each fission event releases a sufficient number of neutrons to sustain an ongoing series of reactions.

Decommissioning

The process of safely closing a nuclear power plant (or other facility where nuclear materials are handled) to retire it from service after its useful life has ended. This process primarily involves decontaminating the facility to reduce residual radioactivity and then releasing the property for unrestricted or (under certain conditions) restricted use. This often includes dismantling the facility or dedicating it to other purposes. See *ENTOMB* and *SAFSTOR*.

DECON

A phase of reactor decommissioning in which structures, systems, and components that contain radioactive contamination are removed from a site and safely disposed of at a commercially operated low-level waste (LLW) disposal facility or decontaminated to a level that permits the site to be released for unrestricted use.



Decontamination

A process used to reduce, remove, or neutralize radiological, chemical, or biological contamination to reduce the risk of exposure. Decontamination may be accomplished by cleaning or treating surfaces to reduce or remove the contamination, filtering contaminated air or water, or subjecting contamination to evaporation and precipitation. The process can also simply allow adequate time for radioactive decay to decrease the radioactivity.

Defense in depth

An approach to designing and operating nuclear facilities that prevents and mitigates accidents that release radiation or hazardous materials. The key is creating multiple independent and redundant layers of controls or design features to compensate for potential human and mechanical failures so that no single control, no matter how robust, is exclusively relied upon to achieve safety or security. Defense in depth includes the use of access controls, physical barriers, redundant and diverse key safety functions, and emergency response measures.

Depleted uranium

Uranium with a percentage of uranium-235 lower than the 0.7 percent (by mass) contained in natural uranium. Depleted uranium is the byproduct of the uranium enrichment process. Depleted uranium can be blended with highly enriched uranium, such as that from weapons, to make reactor fuel.

Design-basis threat (DBT)

A description of the type, composition, and capabilities of an adversary that a security system is designed to protect against. The NRC uses the DBT as a basis for designing safeguards systems to protect against acts of radiological sabotage and to prevent the theft of special nuclear material. Certain nuclear facility licensees are expected to demonstrate that they can defend against the DBT.

Design certification

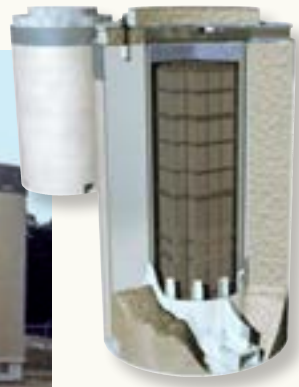
Certification and approval by the NRC of a standard nuclear power plant design independent of a specific site or an application to construct or operate a plant. A design certification is valid for 15 years from the date of issuance but can be renewed for an additional 10 to 15 years.

Dose (Radiation)

The National Council on Radiation Protection and Measurements estimates that an average person in the United States receives a total annual dose of about 0.62 rem (620 millirem) from all radiation sources, a level that has not been shown to cause humans any harm. Of this total, natural background sources of radiation—including radon and thoron gas, natural radiation from soil and rocks, radiation from space, and radiation sources that are found naturally within the human body—account for approximately 50 percent. Medical procedures such as computed tomography (CT scans) and nuclear medicine account for approximately another 48 percent. Other small contributors of exposure to the U.S. population include consumer products and activities, industrial and research uses, and occupational tasks. The maximum permissible yearly dose for a person working with or around nuclear material is 5 rem.

Dry cask storage

A method for storing spent nuclear fuel in special containers known as dry casks. After fuel has been cooled in a spent fuel pool, dry cask storage allows spent fuel assemblies to be sealed in casks and surrounded by inert gas. They are welded or bolted closed, and each cask includes steel, concrete, lead, or other material to provide leak-tight containment and radiation shielding. The casks may store fuel horizontally or vertically.



Early site permit (ESP)

A permit granted by the NRC to approve one or more proposed sites for a nuclear power facility, independent of a specific nuclear plant design or an application for a construction permit or COL. An ESP is valid for 10 to 20 years but can be renewed for an additional 10 to 20 years.

Economic Simplified Boiling-Water Reactor (ESBWR)

A 4,500-megawatt thermal nuclear reactor design, which has passive safety features and uses natural circulation (with no recirculation pumps or associated piping) for normal operation. The NRC certified the ESWBR standard design submitted by GE-Hitachi Nuclear Energy (GEH) on October 15, 2014.

Efficiency, plant

The percentage of the total energy content of a power plant's thermal energy that is converted into electricity. The remaining energy is lost to the environment as heat.

Electric power grid

A system of synchronized power providers and consumers, connected by transmission and distribution lines and operated by one or more control centers. In the continental United States, the electric power grid consists of three systems—the Eastern Interconnect, the Western Interconnect, and the Texas Interconnect. In Alaska and Hawaii, several systems encompass areas smaller than the State.

Electric utility

A corporation, agency, authority, person, or other legal entity that owns or operates facilities within the United States, its territories, or Puerto Rico for the generation, transmission, distribution, or sale of electric power (primarily for use by the public). Facilities that qualify as cogenerators or small power producers under the Public Utility Regulatory Policies Act are not considered electric utilities.

Emergency classifications

Sets of plant conditions that indicate various levels of risk to the public and that might require response by an offsite emergency response organization to protect citizens near the site.

Emergency preparedness (EP)

The programs, plans, training, exercises, and resources used to prepare to rapidly identify, evaluate, and respond to emergencies, including those arising from terrorism or natural events such as hurricanes. EP strives to ensure that operators of nuclear power plants and certain fuel cycle facilities can implement measures to protect public health and safety in the event of a radiological emergency. Licensees who operate certain nuclear facilities, such as nuclear power plants, must develop and maintain EP plans that meet NRC requirements.

Energy Information Administration (EIA)

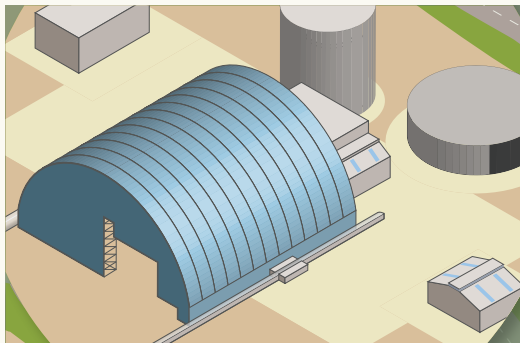
The agency, within the U.S. Department of Energy, that provides policy-neutral statistical data, forecasts, and analyses to promote sound policymaking, efficient markets, and public understanding regarding energy and its interaction with the economy and the environment.

Enrichment

See *Uranium enrichment*.

ENTOMB

A method of decommissioning a nuclear power plant, in which radioactive contaminants are encased in a structurally long-lived material, such as concrete. The entombed structure is maintained and surveillance is continued until the radioactive waste decays to a level permitting termination of the license and unrestricted release of the property.

**Event Notification System**

An automated system used by the NRC to document incoming notifications of significant nuclear events with an actual or potential effect on the health and safety of the public and the environment. Significant events are reported to the NRC by licensees, Agreement States, other Federal agencies, the public, and other countries.

Exposure (Radiation)

Absorption of ionizing radiation or the amount of a hazardous substance that has been ingested, inhaled, or contacted the skin. Acute exposure is a large exposure received over a short period of time. Chronic exposure is exposure received over a long period of time, such as during a lifetime. (See *Occupational dose*.)

Federal Emergency Management Agency (FEMA)

A component of the U.S. Department of Homeland Security responsible for protecting the Nation and reducing the loss of life and property from all hazards, such as natural disasters and acts of terrorism. FEMA leads and supports a risk-based, comprehensive emergency management system of preparedness, protection, response, recovery, and mitigation.

Federal Energy Regulatory Commission (FERC)

An independent agency that regulates the interstate transmission of electricity, natural gas, and oil. FERC also regulates and oversees hydropower projects and the construction of liquefied natural gas terminals and interstate natural gas pipelines. FERC protects the economic, environmental, and safety interests of the American public, while working to ensure abundant, reliable energy in a fair, competitive market.

Fiscal year (FY)

The 12-month period from October 1 through September 30 used by the Federal Government for budget formulation and execution. The FY is designated by the calendar year in which it ends; for example, FY 2016 runs from October 1, 2015, through September 30, 2016.

Fissile material

A nuclide that is capable of undergoing fission after capturing neutrons. Although sometimes used as a synonym for fissionable material, this term has acquired its more restrictive interpretation with the limitation that the nuclide must be fissionable by thermal neutrons. With that interpretation, the three primary fissile materials are uranium-233, uranium-235, and plutonium-239. This definition excludes natural uranium and depleted uranium that have not been irradiated or have only been irradiated in thermal reactors.

Fission

The splitting of an atom, which releases a considerable amount of energy (usually in the form of heat). Fission may be spontaneous but is usually caused by the nucleus of an atom becoming unstable (or “heavy”) after capturing or absorbing a neutron. During fission, the nucleus splits into roughly equal parts, producing the nuclei of at least two lighter elements. In addition to energy, this reaction usually releases gamma radiation and two or more daughter neutrons.

Force on force

A type of security exercise designed to evaluate and improve the effectiveness of a security strategy. For the NRC, force-on-force exercises are used to assess the ability of the licensee to defend a nuclear power plant and other nuclear facilities against a design-Basis threat (DBT).

Foreign Assignee Program

An on-the-job training program, sponsored by the NRC for assignees from other countries, usually under bilateral information exchange arrangements with their respective regulatory organizations. The assignee’s regulatory authorities generally identify the individuals participating and pay their salaries.

Freedom of Information Act (FOIA)

A Federal law that requires Federal agencies to provide, upon written request, access to records or information. Some material is exempt from FOIA, and FOIA does not apply to records that are maintained by State and local governments, Federal contractors, grantees, or private organizations or businesses.

Fuel assembly (fuel bundle, fuel element)

A structured group of fuel rods (long, slender, metal tubes containing pellets of fissionable material, which provide fuel for nuclear reactors). Depending on the design, each reactor core may have dozens of fuel assemblies (also known as fuel bundles), each of which contains dozens of fuel rods.

Fuel cycle

The series of steps involved in supplying fuel for nuclear power reactors. The uranium fuel cycle includes the following:

- uranium recovery to extract and concentrate the uranium to produce yellowcake
- conversion of yellowcake into uranium hexafluoride (UF_6)
- enrichment to increase the concentration of uranium-235 in UF_6
- fuel fabrication to convert enriched UF_6 into fuel for nuclear reactors
- use of the fuel in reactors (nuclear power, research, or naval propulsion)
- interim storage of spent nuclear fuel
- reprocessing of spent fuel to recover the fissionable material remaining in the spent fuel (currently not done in the United States)
- final disposition (disposal) of HLW
- transportation of the uranium in all forms, including spent fuel

The NRC regulates these processes, as well as the fabrication of mixed oxide (MOX) nuclear fuel, which is a combination of uranium and plutonium oxides.

Fuel reprocessing (recycling)

The processing of reactor fuel to separate the unused fissionable material from waste material. Reprocessing extracts uranium and plutonium from spent nuclear fuel so they can be used again as reactor fuel. Commercial reprocessing is not practiced in the United States, although it has been in the past. However, the DOE operates reprocessing facilities such as in Hanford, WA, and Savannah River, SC, for national defense purposes.

Fuel rod

A long, slender, zirconium metal tube containing pellets of fissionable material, which provide fuel for nuclear reactors. Fuel rods are assembled into bundles called fuel assemblies, which are loaded individually into the reactor core.

Full-time equivalent (FTE)

A human resources measurement equal to one person working full time for 1 year.

Gas centrifuge

Uranium enrichment technology that uses many rotating cylinders that are connected in long lines to increase the concentration of uranium-235. Gas is placed in the cylinder, which spins at a high speed, creating a strong centrifugal force. Heavier gas molecules move to the cylinder wall, while lighter molecules collect near the center. The stream, slightly enriched, is fed into the next cylinder. The depleted stream is recycled back into the previous cylinder.

Gas chromatography

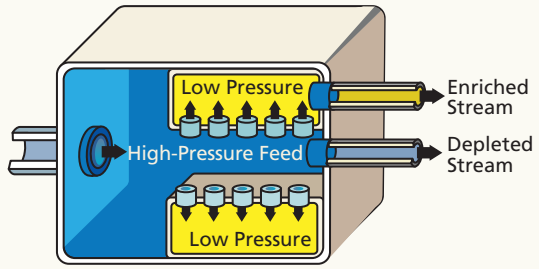
An analytical technique for separating chemical substances from a mixed sample by passing the sample, carried by a moving stream of gas, through a tube packed with a finely divided solid that may be coated with a liquid film. Gas chromatography devices are used to analyze air pollutants, blood alcohol content, essential oils, and food products.

Gaseous diffusion

A uranium enrichment process used to increase the concentration of uranium-235 in uranium for use in fuel for nuclear reactors by separating its isotopes (as gases) based on their slight difference in mass. (Lighter isotopes diffuse faster through a porous membrane or vessel than do heavier isotopes.)

This process involves filtering UF_6 gas to separate uranium-234 and uranium-235 from uranium-238, increasing the percentage of uranium-235.

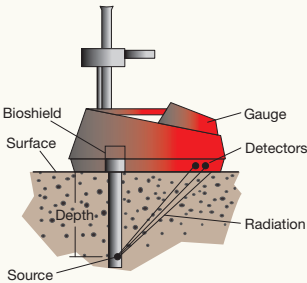
In May 2013, the last remaining gaseous diffusion plant in operation in the United States in Paducah, KY, shut down. A similar plant near Piketon, OH, was closed in March 2001. Another plant in Oak Ridge, TN, closed years ago and was not regulated by the NRC.



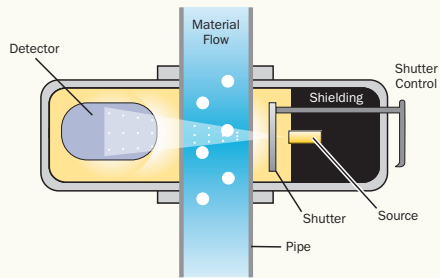
Gauging devices

Devices used to measure, monitor, and control the thickness of sheet metal, textiles, paper napkins, newspaper, plastics, photographic film, and other products as they are manufactured. Gauges mounted in fixed locations are designed for measuring or controlling material density, flow, level, thickness, or weight. The gauges contain sealed sources that radiate through the substance being measured to a readout or controlling device. Portable gauging devices, such as moisture density gauges, are used at field locations. These gauges contain a gamma-emitting sealed source, usually cesium-137, or a sealed neutron source, usually americium-241 and beryllium.

Direct Transmission



Fixed Fluid



Generation (gross)

The total amount of electric energy produced by a power generating station, as measured at the generator terminals.

Generation (net)

The gross amount of electric energy produced by a generating station, minus the amount used to operate the station. Net generation is usually measured in wathours.

Generator capacity

The maximum amount of electric energy that a generator can produce (from the mechanical energy of the turbine), adjusted for ambient conditions. Generator capacity is commonly expressed in megawatts (MW).

Geological repository

An excavated, underground facility that is designed, constructed, and operated for safe and secure permanent disposal of high-level waste (HLW). A geological repository uses an engineered barrier system and a portion of the site's natural geology, hydrology, and geochemical systems to isolate the radioactivity of the waste.

Gigawatt (GW)

A unit of power equivalent to one billion (1,000,000,000) watts.

Gigawatthour (GWh)

One billion (1,000,000,000) wathours.

Grid

See *Electric power grid*.

Half-life (radiological)

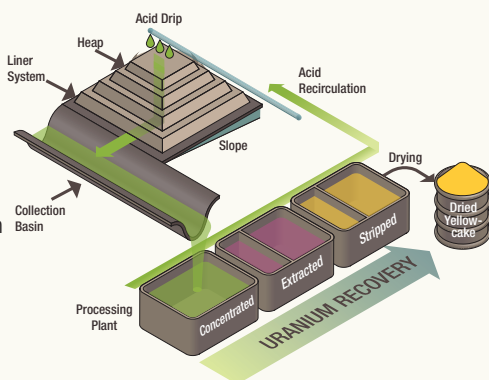
The time required for half the atoms of a particular radioactive material. A specific half-life is a characteristic property of each radioisotope. Measured half-lives range from millionths of a second to billions of years, depending on the stability of the nucleus. Radiological half-life is related to, but different from biological half-life and effective half-life.

Health physics

The science concerned with recognizing and evaluating the effects of ionizing radiation on the health and safety of people and the environment, monitoring radiation exposure, and controlling the associated health risks and environmental hazards to permit the safe use of technologies that produce ionizing radiation.

Heap leach recovery process

A method for extracting uranium from ore. The ore is placed in piles or heaps on top of liners. The liners prevent uranium and other chemicals from moving into the ground. Sulfuric acid is dripped onto the heap and dissolves uranium as it moves through the ore. Uranium solution drains into collection basins, where it is piped to a processing plant. At the plant, uranium is extracted, concentrated, and dried to form yellowcake.



High-level radioactive waste (HLW)

The highly radioactive materials produced as byproducts of fuel reprocessing or of the reactions that occur inside nuclear reactors. HLW includes the following:

- irradiated spent nuclear fuel discharged from commercial nuclear power reactors
- highly radioactive liquid and solid materials resulting from the reprocessing of spent nuclear fuel, which contain fission products in concentration including some reprocessed HLW from defense activities and a small quantity of reprocessed commercial HLW)
- other highly radioactive materials that the Commission may determine require permanent isolation

Highly (or High-) enriched uranium

Uranium enriched to at least 20 percent uranium-235 (a higher concentration than exists in natural uranium ore).

In situ recovery (ISR)

A common method currently used to extract uranium from ore bodies without physical excavation of the ore. ISR is also known as “solution mining” or in situ leaching.

Incident response

Activities that address the short-term, direct effects of a natural or human-caused event and require an emergency response to protect life or property.

Independent spent fuel storage installation (ISFSI)

A complex designed and constructed for the interim storage of spent nuclear fuel; solid, reactor-related, greater than Class C waste; and other associated radioactive materials. A spent fuel storage facility may be considered independent, even if it is located on the site of another NRC-licensed facility.

International Atomic Energy Agency (IAEA)

A United Nations agency established in 1957 to serve as a world center of cooperation in the nuclear field. The agency works with its 154 member States and multiple partners worldwide to promote safe, secure, and peaceful nuclear technology.

International Nuclear Regulators Association

An association established in January 1997 to give international nuclear regulators from advanced nuclear programs a forum to discuss nuclear safety. Countries represented include Canada, France, Japan, the Republic of South Korea, Spain, Sweden, the United Kingdom, and the United States.

Irradiation

Exposure to ionizing radiation. Irradiation may be intentional, such as in cancer treatments or in sterilizing medical instruments. Irradiation may also be accidental, such as from exposure to an unshielded source. Irradiation does not usually result in radioactive contamination, but damage can occur, depending on the dose received.

Isotope

Two or more forms (or atomic configurations) of a given element that have identical atomic numbers (the same number of protons in their nuclei) and the same or very similar chemical properties but different atomic masses (different numbers of neutrons in their nuclei) and distinct physical properties. Thus, carbon-12, carbon-13, and carbon-14 are isotopes of the element carbon, and the numbers denote the approximate atomic masses. Among their distinct physical properties, some isotopes (known as radioisotopes) are radioactive, because their nuclei are unstable and emit radiation as they decay spontaneously toward a more stable nuclear configuration. For example, carbon-12 and carbon-13 are stable, but carbon-14 is unstable and radioactive.

Kilowatt (kW)

A unit of power equivalent to 1,000 watts.

Licensed material

Source material, byproduct material, or special nuclear material that is received, possessed, used, transferred, or disposed of under a general or specific license issued by the NRC or Agreement States and is not otherwise exempt from regulation.

Licensee

A company, organization, institution, or other entity to which the NRC or an Agreement State has granted a general or specific license to construct or operate a nuclear facility, or to receive, possess, use, transfer, or dispose of source, byproduct, or special nuclear material.

Licensing basis

The collection of documents or technical criteria that provides the basis upon which the NRC issues a license to construct or operate a nuclear facility; to conduct operations involving the emission of radiation; or to receive, possess, use, transfer, or dispose of source, byproduct, or special nuclear material.

Light-water reactor

A term used to describe reactors using ordinary water as a moderated coolant, including boiling-water reactors (BWRs) and pressurized-water reactors (PWRs), the most common types used in the United States.

Low-level radioactive waste (LLW)

A general term for a wide range of waste that is contaminated with radioactive material or has become radioactive through exposure to neutron radiation. A variety of industries, hospitals and medical institutions, educational and research institutions, private or government laboratories, and nuclear fuel cycle facilities generate LLW. Some examples include radioactively contaminated protective shoe covers and clothing; cleaning rags, mops, filters, and reactor water treatment residues; equipment and tools; medical tubes, swabs, and hypodermic syringes; and carcasses and tissues from laboratory animals.

Megawatt (MW)

A unit of power equivalent to 1,000,000 watts.

Metric ton

Approximately 2,200 pounds.

Mill tailings

Primarily, the solid residue from a conventional uranium recovery facility in which uranium or thorium ore is crushed and processed mechanically or chemically to recover the uranium, thorium, or other valuable materials. This naturally radioactive ore residue contains the radioactive decay products from the uranium chains (mainly the uranium-238 chain). Although the milling process recovers about 93 percent of the uranium, the "tailings" contain several naturally occurring radioactive elements, including uranium, thorium, radium, polonium, and radon as well as heavy metals and other constituents.

Mixed-oxide (MOX) fuel

A type of nuclear reactor fuel that contains plutonium oxide mixed with either natural or depleted uranium oxide, in ceramic pellet form. This differs from conventional nuclear fuel, which is made of uranium oxide before it is irradiated in a reactor. Using plutonium reduces the amount of enriched uranium needed to produce a controlled reaction in commercial light-water reactors. However, plutonium exists only in trace amounts in nature and, therefore, must be produced by neutron irradiation of uranium-238 or obtained from other manufactured sources. As directed by Congress, the NRC regulates the fabrication of MOX fuel by DOE, a program that is intended to dispose of plutonium from excess nuclear weapons.

Monitoring of radiation

Periodic or continuous determination of the amount of ionizing radiation or radioactive contamination in a region. Radiation monitoring is a safety measure to protect the health and safety of the public and the environment through the use of bioassay, alpha scans, and other radiological survey methods to monitor air, surface water and ground water, soil and sediment, equipment surfaces, and personnel.

National Response Framework

The guiding principles, roles, and structures that enable all domestic incident response partners to prepare for and provide a unified national response to disasters and emergencies. It describes how the Federal Government, States, Tribes, communities, and the private sector work together to coordinate a national response. The framework, which became effective March 22, 2008, builds upon the National Incident Management System, which provides a template for managing incidents.

National Source Tracking System (NSTS)

A secure, Web-based data system that helps the NRC and its Agreement States track and regulate the medical, industrial, and academic uses of certain nuclear materials, from the time they are manufactured or imported to the time of their disposal or exportation. This information enhances the ability of the NRC and Agreement States to conduct inspections and investigations, communicate information to other government agencies, and verify the ownership and use of nationally tracked sources.

Natural uranium

Uranium containing the relative concentrations of isotopes found in nature; 0.7 percent uranium-235, 99.3 percent uranium-238, and a trace amount of uranium-234 by mass. In terms of radioactivity, however, natural uranium contains approximately 2.2 percent uranium-235, 48.6 percent uranium-238, and 49.2 percent uranium-234. Natural uranium can be used as fuel in nuclear reactors or are feedstock for uranium enrichment facilities.

Net electric generation

The gross amount of electric energy produced by a generating station, minus the amount used to operate the station. Note: Electricity required for pumping at pumped-storage plants is regarded as electricity for station operation and is deducted from gross generation. Net electric generation is measured in watthours, except as otherwise noted.

Nonpower reactor (research and test reactor)

A nuclear reactor that is used for research, training, or development purposes (which may include producing radioisotopes for medical and industrial uses) but has no role in producing electrical power. These reactors, which are also known as research and test reactors, contribute to almost every field of science, including physics, chemistry, biology, medicine, geology, archeology, and ecology.

NRC Headquarters Operations Center

The primary center of communication and coordination among the NRC, its licensees, State and Tribal agencies, and other Federal agencies regarding operating events involving nuclear reactors or materials. Located in Rockville, MD, the Headquarters Operations Center is staffed 24 hours a day by employees trained to receive and evaluate event reports and coordinate incident response activities.

Nuclear energy

See *Atomic energy*.

Nuclear Energy Agency (NEA)

A specialized agency within the Organisation for Economic Co-operation and Development (OECD), which was created to assist its member countries in maintaining and further developing the scientific, technological, and legal bases for safe, environmentally friendly, and economical use of nuclear energy for peaceful purposes. The NEA's current membership consists of 31 countries in Europe, North America, and the Asia-Pacific region, which account for approximately 86 percent of the world's installed nuclear capacity.

Nuclear fuel

Fissionable material that has been enriched to a composition that will support a self-sustaining fission chain reaction when used to fuel a nuclear reactor, thereby releasing energy (usually in the form of heat or useful radiation) for use in other processes.

Nuclear materials

See *Special nuclear material*, *Source material*, and *Byproduct material*.

Nuclear Material Management and Safeguards System (NMMSS)

A centralized U.S. Government database used to track and account for source and special nuclear material. The system contains current and historical data on the possession, use, and shipment of source and special nuclear material within the United States, as well as all exports and imports of such material. The database is jointly funded by the NRC and DOE and is operated under a DOE contract.

Nuclear poison (or neutron poison)

In reactor physics, a substance (other than fissionable material) that has a large capacity for absorbing neutrons in the vicinity of the reactor core. This effect may be undesirable in some reactor applications, because it may prevent or disrupt the fission chain reaction, thereby affecting normal operation. However, neutron-absorbing materials (commonly known as “poisons”) are intentionally inserted into some types of reactors to decrease the reactivity of their initial fresh fuel load for fuel intended to achieve higher burnup levels during the fuel cycle. Adding poisons, such as control rods or boron, is described as adding “negative reactivity” to the reactor.

Nuclear power plant

A thermal power plant, in which the energy (heat) released by the fissioning of nuclear fuel is used to boil water to produce steam. The steam spins the propeller-like blades of a turbine that turns the shaft of a generator to produce electricity. Of the various nuclear power plant designs, pressurized-water reactors (PWRs) and boiling-water reactors (BWRs) are in commercial operation in the United States. These facilities generate about 20 percent of U.S. electrical power.

Nuclear and Radiological Incident Annex

An annex to the National Response Framework, which provides for a timely, coordinated response by Federal agencies to nuclear or radiological accidents or incidents within the United States. This annex covers radiological dispersal devices and improvised nuclear devices, as well as accidents involving commercial reactors or weapons production facilities, lost radioactive sources, transportation accidents involving radioactive material, and foreign accidents involving nuclear or radioactive material.

Nuclear reactor

The heart of a nuclear power plant or nonpower reactor, in which nuclear fission may be initiated and controlled in a self-sustaining chain reaction to generate energy or produce useful radiation. Although there are many types of nuclear reactors, they all incorporate certain essential features, including the use of fissionable material as fuel, a moderator (such as water) to increase the likelihood of fission (unless reactor operation relies on fast neutrons), a reflector to conserve escaping neutrons, coolant provisions for heat removal, instruments for monitoring and controlling reactor operation, and protective devices (such as control rods and shielding).

Nuclear waste

A subset of radioactive waste that includes unusable byproducts produced during the various stages of the nuclear fuel cycle, including extraction, conversion, and enrichment of uranium; fuel fabrication; and use of the fuel in nuclear reactors. Specifically, these stages produce a variety of nuclear waste materials, including uranium mill tailings, depleted uranium, and spent (depleted) fuel, all of which are regulated by the NRC. (By contrast, “radioactive waste” is a broader term, which includes all wastes that contain radioactivity, regardless of how they are produced. It is not considered “nuclear waste,” because it is not produced through the nuclear fuel cycle and is generally not regulated by the NRC.)

Occupational dose

The internal and external dose of ionizing radiation received by workers in the course of employment in such areas as fuel cycle facilities, industrial radiography, nuclear medicine, and nuclear power plants. These workers are exposed to varying amounts of radiation, depending on their jobs and the sources with which they work. The NRC requires its licensees to limit occupational exposure to 5,000 mrem (50 millisievert) per year. Occupational dose does not include the dose received from natural background sources, doses received as a medical patient or participant in medical research programs, or “second-hand doses” to members of the public received through exposure to patients treated with radioactive materials.

Organisation for Economic Co-operation and Development (OECD)

An intergovernmental organization (based in Paris, France) that provides a forum for discussion and cooperation among the governments of industrialized countries committed to democracy and the market economy. The primary goal of OECD and its member countries is to support sustainable economic growth, boost employment, raise living standards, maintain financial stability, assist other countries’ economic development, and contribute to growth in world trade. In addition, OECD is a reliable source of comparable statistics and economic and social data. OECD also monitors trends, analyzes and forecasts economic developments, and researches social changes and evolving patterns in trade, environment, agriculture, technology, taxation, and other areas.

Orphan sources (unwanted radioactive material)

Sealed sources of radioactive material contained in a small volume (but not radioactively contaminated soils and bulk metals) in any one or more of the following conditions:

- an uncontrolled condition that requires removal to protect public health and safety from a radiological threat
- a controlled or uncontrolled condition, for which a responsible party cannot be readily identified
- a controlled condition, compromised by an inability to ensure the continued safety of the material (e.g., the licensee may have few or no options to provide for safe disposition of the material)
- an uncontrolled condition, in which the material is in the possession of a person who did not seek, and is not licensed, to possess it

- an uncontrolled condition, in which the material is in the possession of a State radiological protection program solely to mitigate a radiological threat resulting from one of the above conditions, and for which the State does not have the necessary means to provide for the appropriate disposition of the material

Outage

The period during which a generating unit, transmission line, or other facility is out of service. Outages may be forced or scheduled and full or partial.

Outage (forced)

The shutdown of a generating unit, transmission line, or other facility for emergency reasons, or a condition in which the equipment is unavailable as a result of an unanticipated breakdown. An outage (whether full, partial, or attributable to a failed start) is considered “forced” if it could not reasonably be delayed beyond 48 hours from identification of the problem, if there had been a strong commercial desire to do so. In particular, the following problems may result in forced outages:

- any failure of mechanical, fuel handling, or electrical equipment or controls within the generator’s ownership or direct responsibility (i.e., from the point the generator is responsible for the fuel through to the electrical connection point)
- a failure of a mine or fuel transport system dedicated to that power station with a resulting fuel shortage that cannot be economically managed
- inadvertent or operator error
- limitations caused by fuel quality

Forced outages do not include scheduled outages for inspection, maintenance, or refueling.

Outage (full forced)

A forced outage that causes a generating unit to be removed from the committed state (when the unit is electrically connected and generating or pumping) or the available state (when the unit is available for dispatch as a generator or pump but is not electrically connected and not generating or pumping). Full-forced outages do not include failed starts.

Outage (scheduled)

The shutdown of a generating unit, transmission line, or other facility for inspection, maintenance, or refueling, which is scheduled well in advance (even if the schedule changes). Scheduled outages do not include forced outages and could be deferred if there were a strong commercial reason to do so.

Pellet, fuel

A thimble-sized ceramic cylinder (approximately 3/8-inch in diameter and 5/8-inch in length), consisting of uranium (typically uranium oxide), which has been enriched to increase the concentration of uranium-235 (U-235) to fuel a nuclear reactor. Modern reactor cores in PWRs and BWRs may contain up to 10 million pellets, stacked in the fuel rods that form fuel assemblies.



Performance-based regulation

A regulatory approach that focuses on desired, measurable outcomes, rather than prescriptive processes, techniques, or procedures. Performance-based regulation leads to defined results without specific direction regarding how those results are to be obtained. At the NRC, performance-based regulatory actions focus on identifying performance measures that ensure an adequate safety margin and offer incentives for licensees to improve safety without formal regulatory intervention by the agency.

Performance indicator

A quantitative measure of a particular attribute of licensee performance that shows how well a plant is performing when measured against established thresholds. Licensees submit their data quarterly; the NRC regularly conducts inspections to verify the submittals and then uses its own inspection data plus the licensees' submittals to assess each plant's performance.

Possession-only license

A license, issued by the NRC, that authorizes the licensee to possess specific nuclear material but does not authorize its use or the operation of a nuclear facility.

Power uprate

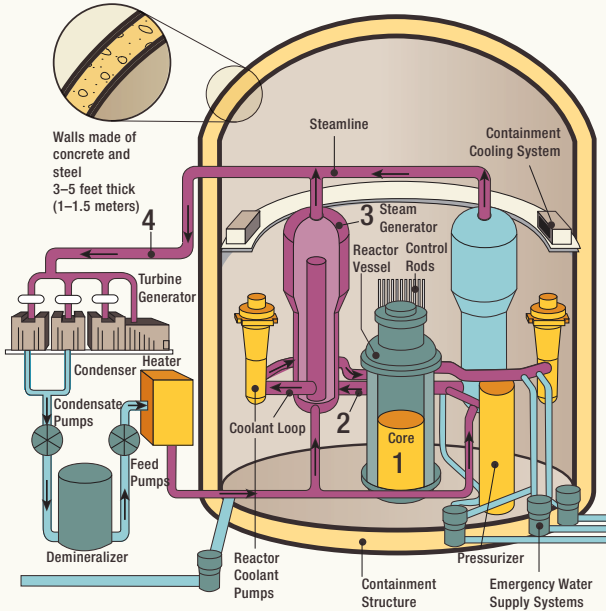
The process of increasing the maximum power level at which a commercial nuclear power plant may operate. This power level, regulated by the NRC, is included in the plant's operating license and technical specifications. A licensee may only change its maximum power output after the NRC approves an uprate application. The NRC analyses must demonstrate that the plant could continue to operate safely with its proposed new configuration. When all requisite conditions are fulfilled, the NRC may grant the power uprate by amending the plant's operating license and technical specifications.



Transmission lines distribute electricity generated by nuclear power plants to the power grid.

Pressurized-water reactor (PWR)

A common nuclear power reactor design in which very pure water is heated to a very high temperature by fission, kept under high pressure (to prevent it from boiling), and converted to steam by a steam generator (rather than by boiling, as in a BWR). The resulting steam is used to drive turbines, which activate generators to produce electrical power. A PWR essentially operates like a pressure cooker, where a lid is tightly placed over a pot of heated water, causing the pressure inside to increase as the temperature increases (because the steam cannot escape) but keeping the water from boiling at the usual 212 degrees Fahrenheit (100 degrees Celsius). About two-thirds of the operating nuclear reactor power plants in the United States are PWRs.



How Nuclear Reactors Work

In a typical design concept of a commercial PWR, the following process occurs:

1. The core inside the reactor vessel creates heat.
2. Pressurized water in the primary coolant loop carries the heat to the steam generator.
3. Inside the steam generator, heat from the primary coolant loop vaporizes the water in a secondary loop, producing steam.
4. The steamline directs the steam to the main turbine, causing it to turn the turbine generator, which produces electricity.

The unused steam is exhausted to the condenser, where it is condensed into water. The resulting water is pumped out of the condenser with a series of pumps, reheated, and pumped back to the steam generator. The reactor's core contains fuel assemblies that are cooled by water circulated using electrically powered pumps. These pumps and other operating systems in the plant receive their power from the electrical grid. If offsite power is lost, emergency cooling water is supplied by other pumps, which can be powered by onsite diesel generators. Other safety systems, such as the containment cooling system, also need electric power. PWRs contain between 150–200 fuel assemblies.

Probabilistic risk assessment (PRA)

A systematic method for assessing three questions that the NRC uses to define “risk.” These questions consider (1) what can go wrong, (2) how likely it is to happen, and (3) what the consequences might be. These questions allow the NRC to understand likely outcomes, sensitivities, areas of importance, system interactions, and areas of uncertainty, which the staff can use to identify risk-significant scenarios. The NRC uses PRA to determine a numeric estimate of risk to provide insights into the strengths and weaknesses of the design and operation of a nuclear power plant.

Production expense

Production expense is one component of the cost of generating electric power, which includes costs associated with fuel, as well as plant operation and maintenance.

Rad (radiation absorbed dose)

One of the two units used to measure the amount of radiation absorbed by an object or person, known as the “absorbed dose,” which reflects the amount of energy that radioactive sources deposit in materials through which they pass. The radiation-absorbed dose (rad) is the amount of energy (from any type of ionizing radiation) deposited in any medium (e.g., water, tissue, air). An absorbed dose of 1 rad means that 1 gram of material absorbed 100 ergs of energy (a small but measurable amount) as a result of exposure to radiation. The related international system unit is the gray (Gy), where 1 Gy is equivalent to 100 rad.

Radiation, ionizing

A form of radiation, which includes alpha particles, beta particles, gamma rays and x-rays, neutrons, high-speed electrons, and high-speed protons. Compared to nonionizing radiation, such as found in ultraviolet light or microwaves, ionizing radiation is considerably more energetic. When ionizing radiation passes through material such as air, water, or living tissue, it deposits enough energy to break molecular bonds and displace (or remove) electrons. This electron displacement may lead to changes in living cells. Given this ability, ionizing radiation has a number of beneficial uses, including treating cancer or sterilizing medical equipment. However, ionizing radiation is potentially harmful if not used correctly, and high doses may result in severe skin or tissue damage. It is for this reason that the NRC strictly regulates commercial and institutional uses of the various types of ionizing radiation.

Radiation, nuclear

Energy given off by matter in the form of tiny fast-moving particles (alpha particles, beta particles, and neutrons) or pulsating electromagnetic rays or waves (gamma rays) emitted from the nuclei of unstable radioactive atoms. All matter is composed of atoms, which are made up of various parts; the nucleus contains minute particles called protons and neutrons, and the atom’s outer shell contains other particles called electrons. The nucleus carries a positive electrical charge, while the electrons carry a negative electrical charge. These forces work toward a strong, stable balance by getting rid of excess atomic energy (radioactivity). In that process, unstable radioactive nuclei may emit energy, and this spontaneous emission is called nuclear



radiation. All types of nuclear radiation are also ionizing radiation, but the reverse is not necessarily true; for example, x-rays are a type of ionizing radiation, but they are not nuclear radiation, because they do not originate from atomic nuclei. In addition, some elements are naturally radioactive, as their nuclei emit nuclear radiation as a result of radioactive decay, but others become radioactive by being irradiated in a reactor. Naturally occurring nuclear radiation is indistinguishable from induced radiation.

Radiation source

A radioactive material or byproduct that is specifically manufactured or obtained for the purpose of using the emitted radiation. Such sources are commonly used in teletherapy or industrial radiography; in various types of industrial gauges, irradiators, and gamma knives; and as power sources for batteries (such as those used in spacecraft). These sources usually consist of a known quantity of radioactive material, which is encased in a manmade capsule, sealed between layers of nonradioactive material, or firmly bonded to a nonradioactive substrate to prevent radiation leakage. Other radiation sources include devices such as accelerators and x-ray generators.

Radiation standards

Exposure limits; permissible concentrations; rules for safe handling; and regulations regarding receipt, possession, use, transportation, storage, disposal, and industrial control of radioactive material.

Radiation therapy (radiotherapy)

The therapeutic use of ionizing radiation to treat disease in patients. Although most radiotherapy procedures are intended to kill cancerous tissue or reduce the size of a tumor, therapeutic doses may also be used to reduce pain or treat benign conditions. For example, intravascular brachytherapy uses radiation to treat clogged blood vessels. Other common radiotherapy procedures include gamma stereotactic radiosurgery (gamma knife), teletherapy, and iodine treatment to correct an overactive thyroid gland. These procedures use radiation sources, regulated by the NRC and its Agreement States, that may be applied either inside or outside the body. In either case, the goal of radiotherapy is to deliver the required therapeutic or pain-relieving dose of radiation with high precision and for the required length of time, while preserving the surrounding healthy tissue.

Radiation warning symbol

An officially prescribed magenta or black trefoil on a yellow background, which must be displayed where certain quantities of radioactive materials are present or where certain doses of radiation could be received.



Radioactive contamination

Undesirable radioactive material (with a potentially harmful effect) that is either airborne or deposited in (or on the surface of) structures, objects, soil, water, or living organisms (people, animals, or plants) in a concentration that may harm people, equipment, or the environment.

Radioactive decay

The spontaneous transformation of one radioisotope into one or more different isotopes (known as “decay products” or “daughter products”), accompanied by a decrease in radioactivity (compared to the parent material). This transformation takes place over a defined period of time (known as a “half-life”), as a result of electron capture; fission; or the emission of alpha particles, beta particles, or photons (gamma radiation or x-rays) from the nucleus of an unstable atom. Each isotope in the sequence (known as a “decay chain”) decays to the next until it forms a stable, less energetic end product. In addition, radioactive decay may refer to gamma-ray and conversion electron emission, which only reduces the excitation energy of the nucleus.

Radioactivity

The property possessed by some elements (such as uranium) of spontaneously emitting energy in the form of radiation as a result of the decay (or disintegration) of an unstable atom. Radioactivity is also the term used to describe the rate at which radioactive material emits radiation. Radioactivity is measured in units of becquerels or disintegrations per second.

Radiography

The use of sealed sources of ionizing radiation for nondestructive examination of the structure of materials. When the radiation penetrates the material, it produces a shadow image by blackening a sheet of photographic film that has been placed behind the material, and the differences in blackening suggest flaws and unevenness in the material.

Radioisotope (radionuclide)

An unstable isotope of an element that decays or disintegrates spontaneously, thereby emitting radiation. Approximately 5,000 natural and artificial radioisotopes have been identified.

Radiopharmaceutical

A pharmaceutical drug that emits radiation and is used in diagnostic or therapeutic medical procedures. Radioisotopes that have short half-lives are generally preferred to minimize the radiation dose to the patient and the risk of prolonged exposure. In most cases, these short-lived radioisotopes decay to stable elements within minutes, hours, or days, allowing patients to be released from the hospital in a relatively short time.

Reactor core

The central portion of a nuclear reactor, which contains the fuel assemblies, water, and control mechanisms, as well as the supporting structure. The reactor core is where fission takes place.

Reactor Oversight Process (ROP)

The process by which the NRC monitors and evaluates the performance of commercial nuclear power plants. Designed to focus on those plant activities that are most important to safety, the ROP uses inspection findings and performance indicators to assess each plant’s safety performance.

Regulation

The governmental function of controlling or directing economic entities through the process of rulemaking and adjudication.

Regulatory Information Conference

An annual NRC conference that brings together NRC staff, regulated utilities, materials users, and other interested stakeholders to discuss nuclear safety topics and significant and timely regulatory activities through informal dialogue to ensure an open regulatory process.

Rem (roentgen equivalent man)

One of the two standard units used to measure the dose equivalent (or effective dose), which combines the amount of energy (from any type of ionizing radiation) that is deposited in human tissue with the biological effects of the given type of radiation. For beta and gamma radiation, the dose equivalent is the same as the absorbed dose. By contrast, the dose equivalent is larger than the absorbed dose for alpha and neutron radiation, because these types of radiation are more damaging to the human body. Thus, the dose equivalent (in rems) is equal to the absorbed dose (in rads) multiplied by the quality factor of the type of radiation (Title 10 of the *Code of Federal Regulations* (10 CFR) 20.1004, "Units of Radiation Dose"). The related international system unit is the sievert (Sv), where 100 rem is equivalent to 1 Sv.

Renewable resources

Natural, but limited, energy resources that can be replenished, including biomass, hydro, geothermal, solar, and wind. These resources are virtually inexhaustible but limited in the amount of energy that is available per unit of time. In the future, renewable resources could also include the use of ocean thermal, wave, and tidal action technologies. Utility renewable resource applications include bulk electricity generation, onsite electricity generation, distributed electricity generation, nongrid-connected generation, and demand-reduction (energy efficiency) technologies.

Risk

The combined answer to three questions that consider (1) what can go wrong, (2) how likely it is to occur, and (3) what the consequences might be. These three questions allow the NRC to understand likely outcomes, sensitivities, areas of importance, system interactions, and areas of uncertainty, which can be used to identify risk-significant scenarios.

Risk-based decisionmaking

An approach to regulatory decisionmaking that considers only the results of a probabilistic risk assessment.

Risk-informed decisionmaking

An approach to regulatory decisionmaking, in which insights from probabilistic risk assessment are considered with other engineering insights.

Risk-informed regulation

An approach to regulation taken by the NRC, which incorporates an assessment of safety significance or relative risk. This approach ensures that the regulatory burden imposed by an individual regulation or process is appropriate to its importance in protecting the health and safety of the public and the environment.

Risk significant

“Risk significant” can refer to a facility’s system, structure, component, or accident sequence that exceeds a predetermined limit for contributing to the risk associated with the facility. The term also describes a level of risk exceeding a predetermined “significance” level.

Safeguards

The use of material control and accounting programs to verify that all special nuclear material is properly controlled and accounted for, as well as the physical protection (or physical security) equipment and security forces. As used by IAEA, this term also means verifying that the peaceful use commitments made in binding nonproliferation agreements, both bilateral and multilateral, are honored.

Safeguards Information

A special category of sensitive unclassified information that must be protected. Safeguards Information concerns the physical protection of operating power reactors, spent fuel shipments, strategic special nuclear material, or other radioactive material.

Safety related

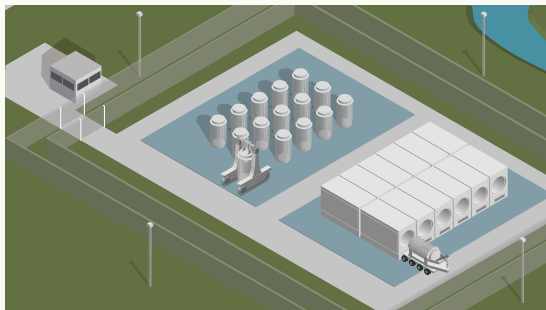
In the regulatory arena, this term applies to systems, structures, components, procedures, and controls (of a facility or process) that are relied upon to remain functional during and following design-basis events. Their functionality ensures that key regulatory criteria, such as levels of radioactivity released, are met. Examples of safety-related functions include shutting down a nuclear reactor and maintaining it in a safe-shutdown condition.

Safety significant

When used to qualify an object, such as a system, structure, component, or accident sequence, this term identifies that object as having an impact on safety, whether determined through risk analysis or other means, that exceeds a predetermined significance criterion.

SAFSTOR

A long-term storage condition for a permanently shut-down nuclear power plant. During SAFSTOR, radioactive contamination decreases substantially, making subsequent decontamination and demolition easier and reducing the amount of low-level waste requiring disposal.



Scram

The sudden shutting down of a nuclear reactor, usually by rapid insertion of control rods, either automatically or manually by the reactor operator (also known as a “reactor trip”).

Sensitive unclassified nonsafeguards information

Information that is generally not publicly available and that encompasses a wide variety of categories, such as proprietary information, personal and private information, or information subject to attorney-client privilege.

Shutdown

A decrease in the rate of fission (and heat or energy production) in a reactor (usually by the insertion of control rods into the core).

Source material

Uranium or thorium, or any combination thereof, in any physical or chemical form, or ores that contain, by weight, 1/20 of 1 percent (0.05 percent) or more of (1) uranium, (2) thorium, or (3) any combination thereof. Source material does not include special nuclear material.

Special nuclear material

Plutonium, uranium-233, or uranium enriched in the isotopes uranium-233 or uranium-235.

Spent fuel pool

An underwater storage and cooling facility for spent (depleted) fuel assemblies that have been removed from a reactor.

Spent (depleted or used) nuclear fuel

Nuclear reactor fuel that has been used to the extent that it can no longer effectively sustain a chain reaction.

Subcriticality

The condition of a nuclear reactor system, in which nuclear fuel no longer sustains a fission chain reaction (that is, the reaction fails to initiate its own repetition, as it would in a reactor’s normal operating condition). A reactor becomes subcritical when its fission events fail to release a sufficient number of neutrons to sustain an ongoing series of reactions, possibly as a result of increased neutron leakage or poisons.

Teletherapy

Treatment in which the source of the therapeutic radiation is at a distance from the body. Because teletherapy is often used to treat malignant tumors deep within the body by bombarding them with a high-energy beam of gamma rays (from a radioisotope such as cobalt-60) projected from outside the body, it is often called “external beam radiotherapy.”

Title 10 of the Code of Federal Regulations (10 CFR)

Four volumes of the Code of Federal Regulations (CFR) address energy-related topics. Parts 1 to 199 contain the regulations (or rules) established by the NRC. These regulations govern the transportation and storage of nuclear materials; use of radioactive materials at nuclear power plants, research and test reactors, uranium recovery facilities, fuel cycle facilities, waste repositories, and other nuclear facilities; and use of nuclear materials for medical, industrial, and academic purposes.

Transient

A change in the reactor coolant system temperature, pressure, or both, attributed to a change in the reactor's power output. Transients can be caused by (1) adding or removing neutron poisons, (2) increasing or decreasing electrical load on the turbine generator, or (3) accident conditions.

Transuranic waste

Material contaminated with transuranic elements—artificially made, radioactive elements, such as neptunium, plutonium, americium, and others—that have atomic numbers higher than uranium in the periodic table of elements. Transuranic waste is primarily produced from recycling spent fuel or using plutonium to fabricate nuclear weapons.

Tritium

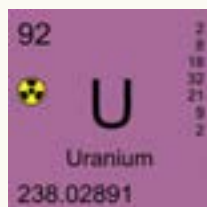
A radioactive isotope of hydrogen. Because it is chemically identical to natural hydrogen, tritium can easily be taken into the body by any ingestion path. It decays by emitting beta particles and has a half-life of about 12.5 years.

Uprate

See *Power uprate*.

Uranium

A radioactive element with the atomic number 92 and, as found in natural ores, an atomic weight of approximately 238. The two principal natural isotopes are uranium-235 (which comprises 0.7 percent of natural uranium), which is fissile, and uranium-238 (99.3 percent of natural uranium), which is fissionable by fast neutrons and is fertile, meaning that it becomes fissile after absorbing one neutron. Natural uranium also includes a minute amount of uranium-234.

**Uranium enrichment**

The process of increasing the percentage of uranium-235 from 0.7 percent in natural uranium to about 3 to 5 percent for use in fuel for nuclear reactors. Enrichment can be done through gaseous diffusion, gas centrifuges, or laser isotope separation.

Uranium fuel fabrication facility

A facility that converts enriched UF₆ into fuel for commercial light-water power reactors, research and test reactors, and other nuclear reactors. The UF₆, in solid form in containers, is heated to a gaseous form and then chemically processed to form uranium dioxide (UO₂) powder. This powder is then processed into ceramic pellets and loaded into metal tubes, which are subsequently bundled into fuel assemblies. Fabrication also can involve MOX fuel, which contains plutonium oxide mixed with either natural or depleted uranium oxide, in ceramic pellet form.

Uranium hexafluoride production facility (or uranium conversion facility)

A facility that receives natural uranium in the form of ore concentrate (known as yellowcake) and converts it into UF₆, in preparation for fabricating fuel for nuclear reactors.

U.S. Department of Energy (DOE)

The Federal agency established by Congress to advance the national, economic, and energy security of the United States, among other missions.

U.S. Department of Homeland Security (DHS)

The Federal agency responsible for leading the unified national effort to secure the United States against those who seek to disrupt the American way of life. DHS is also responsible for preparing for and responding to all hazards and disasters and includes the formerly separate FEMA, the Coast Guard, and the Secret Service.

U.S. Environmental Protection Agency (EPA)

The Federal agency responsible for protecting human health and safeguarding the environment. The EPA leads the Nation's environmental science, research, education, and assessment efforts to ensure that attempts to reduce environmental risk are based on the best available scientific information. The EPA also ensures that environmental protection is an integral consideration in U.S. policies.

Viability assessment

A decisionmaking process used by DOE to assess the prospects for safe and secure permanent disposal of high-level waste (HLW) in an excavated, underground facility, known as a geologic repository. This decisionmaking process is based on (1) specific design work on the critical elements of the repository and waste package, (2) a total system performance assessment that will describe the probable behavior of the repository, (3) a plan and cost estimate for the work required to complete the license application, and (4) an estimate of the costs to construct and operate the repository.

Waste, radioactive

Radioactive materials at the end of their useful life or in a product that is no longer useful and requires proper disposal (see high-level waste, low-level waste and spent nuclear fuel).

Waste classification (classes of waste)

Classification of low-level waste (LLW) according to its radiological hazard. The classes include Class A, B, and C, with Class A being the least hazardous and accounting for 96 percent of LLW. As the waste class and hazard increase, the regulations established by the NRC require progressively greater controls to protect the health and safety of the public and the environment.

Watt

A unit of power (in the international system of units) defined as the consumption or conversion of 1 joule of energy per second. In electricity, a watt is equal to current (in amperes) multiplied by voltage (in volts).





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www.nrc.gov/reading-rm/doc-collections/nuregs/staff/sr1614

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www.nrc.gov/about-nrc/governing-laws.html

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Public Involvement

www.nrc.gov/public-involve.html

Freedom of Information Act (FOIA)

www.nrc.gov/reading-rm/foia/foia-privacy.html

Rulemaking Actions

www.regulations.gov

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www.nrc.gov/reading-rm/doc-collections/enforcement/actions/

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www.nrc.gov/about-nrc/organization/nrcorg.pdf

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NRC Regions

www.nrc.gov/about-nrc/locations.html

NRC Budget

Performance Budget: Fiscal Year 2016 (NUREG-1100)

www.nrc.gov/reading-rm/doc-collections/nuregs/staff/sr1100

U.S. and Worldwide Nuclear Energy

U.S. Electricity

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Official Energy Statistics from the U.S. Government

www.eia.doe.gov

Worldwide Electricity Generated by Commercial Nuclear Power

International Atomic Energy Agency (IAEA)

www.iaea.org

IAEA Power Reactor Information System (PRIS)

www.iaea.org/programmes/a2

Nuclear Energy Agency (NEA)

www.nea.fr/

World Nuclear Association (WNA)

www.world-nuclear.org/

World Nuclear Power Reactors and Uranium Requirements

www.world-nuclear.org/info/reactors.html

WNA Reactor Database

www.world-nuclear.org/nucleardatabase/default.aspx

NRC Office of International Programs

www.nrc.gov/about-nrc/organization/oipfuncdesc.html

NRC Regulatory Information Conference (RIC)

www.nrc.gov/public-involve/conference-symposia/ric/index.html

International Activities

Treaties and Conventions

www.nrc.gov/about-nrc/ip/treaties-conventions.html

Code of Conduct on the Safety and Security of Radioactive Sources

www.iaea.org/publications/documents/infcircs/code-conduct-safety-and-security-radioactive-sources-and

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www.nrc.gov/info-finder/reactor/

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www.nrc.gov/NRR/OVERSIGHT/ASSESS/index.html

NUREG-1649, "Reactor Oversight Process"

www.nrc.gov/reading-rm/doc-collections/nuregs/staff/sr1649

ROP Performance Indicators Summary

www.nrc.gov/NRR/OVERSIGHT/ASSESS/pi_summary.html

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www.nrc.gov/reactors/new-reactor-op-lic/licensing-process.html#licensing

New Reactors

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