

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555-0001

September 3, 2015

Mr. Bryan C. Hanson Senior Vice President Exelon Generation Company, LLC President and Chief Nuclear Officer Exelon Nuclear 4300 Winfield Road Warrenville, IL 60555

SUBJECT: LASALLE COUNTY STATION, UNITS 1 AND 2 – INTERIM STAFF RESPONSE TO REEVALUATED FLOOD HAZARDS SUBMITTED IN RESPONSE TO 10 CFR 50.54(f) INFORMATION REQUEST – FLOOD-CAUSING MECHANISM REEVALUATION (TAC NOS. MF3655 AND MF3656)

Dear Mr. Hanson:

The purpose of this letter is to provide a summary of the U.S. Nuclear Regulatory Commission (NRC) staff's assessment of the re-evaluated flood-causing mechanisms described in the March 12, 2014 (Agencywide Document Access and Management System (ADAMS) Accession No. ML14079A417), flood hazard reevaluation report (FHRR) submitted by Exelon Generation Company, LLC (Exelon, the licensee) for LaSalle County, Units 1 and 2 (LaSalle), as well as supplemental information resulting from requests for additional information and audits.

By letter dated March 12, 2012, the NRC issued a request for information pursuant to Title 10 of the *Code of Federal Regulations*, Section 50.54(f) (hereafter referred to as the 50.54(f) letter) (ADAMS Accession No. ML12053A340). The request was issued as part of implementing lessons-learned from the accident at the Fukushima Dai-ichi nuclear power plant. Enclosure 2 to the 50.54(f) letter requested licensees to re-evaluate flood-causing mechanisms using present-day methodologies and guidance. Concurrently, with the reevaluation of flooding hazards, licensees were required to develop and implement mitigating strategies in accordance with NRC Order EA-12-049, "Requirements for Mitigation Strategies for Beyond-Design-Basis External Events" (ADAMS Accession No. ML12054A735). On March 30, 2015, the Commission provided Staff Requirements Memoranda (SRM) (ADAMS Accession No. ML15089A236) to COM-SECY-14-0037, "Integration of Mitigating Strategies for Beyond-Design-Basis External Events and the Reevaluation of Flooding Hazards," dated November 21, 2014 (ADAMS Accession No. ML14309A256), affirming that licensees need to address the reevaluated flooding hazards within their mitigating strategies for beyond-design-basis external events.

The NRC staff has reviewed the flood hazard information, submitted by the licensee, and has summarized the results of the review in the tables provided as an Enclosure to this letter. Table 1 provides the current design-basis flood hazard mechanisms. Table 2 provides the reevaluated flood hazard Mechanisms; however, the reevaluated flood hazard mechanisms bounded by the current design basis (Table 1) are not included.

B. Hanson

The NRC staff has concluded that the licensee's reevaluated flood hazards information, as summarized in the Enclosure, is suitable for the assessment of mitigating strategies developed in response to Order EA-12-049 (I.e., defines the mitigating strategies flood hazard information described in guidance documents currently being finalized by the industry and NRC staff), for LaSalle. Further, the NRC staff has concluded that the licensee's reevaluated flood hazard information is a suitable input for other assessments associated with Near-Term Task Force Recommendation 2.1 "Flooding". The NRC staff plans to issue a staff assessment documenting the basis for these conclusions at a later time.

In addition, NEI 12-06 "Diverse and Flexible Coping Strategies (FLEX) implementation Guide" is currently being revised. This revision will include a methodology to perform a Mitigating Strategies Assessment (MSA) with respect to the reevaluated flood hazards. Once this methodology is endorsed by the NRC, flood event duration parameters and applicable flood associated effects should be considered as part of the LaSalle MSA. The NRC staff will evaluate the flood event duration parameters (including warning time and period of inundation) and flood-related associated effects developed by the licensee during the NRC staff's review of the MSA.

As stated above, Table 2 of the enclosure to this letter describes the reevaluated flood hazards that exceed the current design-basis. In order to complete its response to the information requested by Enclosure 2 to the 50.54(f) letter, the licensee is expected to submit an integrated assessment or a focused evaluation, as appropriate, to address these reevaluated flood hazards, as described in NRC letter, "Coordination of Request for Information Regarding Flooding Hazard Reevaluation and Mitigating Strategies for Beyond Design Bases External Events" (ADAMS Accession No. ML15174A257). This letter describes the changes in the NRC's approach to the flood hazard reevaluations that were approved by the Commission in its SRM to COMSECY-15-0019, "Closure Plan for the Reevaluation of Flooding Hazards for Operating Nuclear Power Plants" (ADAMS Accession No. ML15209A682).

B. Hanson

If you have any questions, please contact me at (301) 415-6197 or e-mail at Tekia.Govan@nrc.gov.

Sincerely,

Tekia Govan, Project Manager Hazards Management Branch Japan Lessons-Learned Division Office of Nuclear Reactor Regulation

Docket Nos. 50-373 and 50-374

Enclosure: Summary of Results of Flooding Hazard Re-Evaluation Report

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ENCLOSURE:

SUMMARY TABLES OF REEVALUATED FLOOD HAZARD LEVELS

Table 1.	Current Desig	n Basis Flood Hazards	for Use in the MSA

Mechanism	Stillwater Elevation	Waves/ Runup	Design Basis Hazard Elevation	Reference
Local Intense Precipitation				
LIP Zone 1 (North portion of site)	710.1 ft MSL	Not applicable	710.1 ft MSL	FHRR Section 2.2.1
LIP Zone 2 (South portion of site)	710.3 ft MSL	Not applicable	710.3 ft MSL	FHRR Section 2.2.1
Streams and Rivers				
Riverine	521.8 ft MSL	0.7 ft	522.5 ft MSL	USFAR Revision 19 Section 2.4.3 FHRR Section 2.2.2
Failure of Dams and Onsite Water Control/Storage Structures	No impact on the site identified	No impact on the site identified	No impact on the site identified	FHRR Section 2.2.3
Storm Surge			1	
PMF on Lake at Dike	704.3 ft MSL	2.9 ft	707.2 ft MSL	FHRR Section 2.2.4
PMF on Cooling Lake at Plant	704.3 ft MSL	1.3 ft	705.6 ft MSL	FHRR Section 2.2.4
PMF on Cooling Lake at Lake Screen House	704.3 ft MSL	1.8 ft	706.1 ft MSL	FHRR Section 2.2.4
Seiche				
	Not included in DB	Not included in DB	Not included in DB	FHRR Section 2.2.5
Tsunami	Not included in DB	Not included in DB	Not included in DB	FHRR Section 2.2.6

Mechanism	Stillwater Elevation	Waves/ Runup	Design Basis Hazard Elevation	Reference
Ice-Induced Flooding	No impact on the site identified	No impact on the site identified	No impact on the site identified	FHRR Section 2.2.7
Channel Migrations/Diversions	No impact on the site identified	No impact on the site identified	No impact on the site identified	FHRR Section 2.2.8

Table 1. Current Design Basis Flood Hazards for Use in the MSA

Note: Reported values are rounded to the nearest one-tenth of a foot.

Mechanism	Stillwater Elevation	Waves/ Runup	Reevaluated Hazard Elevation	Reference
Local Intense Precipitation	710.8 MSL	Minimal	710.8 MSL	FHRR Section 3.1
Storm Surge Lake screen house	701.0 MSL	9.6 ft	710.6 MSL	FHRR Section 3.4.4
Inlet structure	701.0 MSL	11.0 ft	712.0 MSL	FHRR Section 3.4.4

Table 2. Reevaluated Flood Hazards for Flood-Causing Mechanisms for Use in the MSA

Note 1: The licensee is expected to develop flood event duration parameters and applicable flood associated effects to conduct the MSA. The staff will evaluate the flood event duration parameters (including warning time and period of inundation) and flood associated effects during its review of the MSA.

Note 2: Reevaluated hazard mechanisms bounded by the current design basis (see Table 1) are not included in this table.

Note 3: Reported values are rounded to the nearest one-tenth of a foot.

B. Hanson

If you have any questions, please contact me at (301) 415-6197 or e-mail at Tekia.Govan@nrc.gov.

Sincerely,

/RA/

Tekia Govan, Project Manager Hazards Management Branch Japan Lessons-Learned Division Office of Nuclear Reactor Regulation

Docket Nos. 50-373 and 50-374

Enclosure: Summary of Results of Flooding Hazard Re-Evaluation Report

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ADAMS Accession Nos.	: Pkg	. ML15232A190;	LTR. ML15211A482; ENCL ML15219A661	*via email
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