



Exelon Generation®

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10 CFR 72.44 (d)(3)
Technical Specification 6.9.1.d

RA-15-035

April 30, 2015

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555 - 0001

Oyster Creek Nuclear Generating Station
Renewed Facility Operating License No. DPR-16
NRC Docket No. 50-219

Independent Spent Fuel Storage Facility
NRC Docket No. 72-15

Subject: Annual Radioactive Effluent Release Report for 2014

Enclosed with this cover letter is the Annual Radioactive Effluent Release Report for the period January 1 to December 31, 2014. This report includes the Oyster Creek Nuclear Generating Station Independent Spent Fuel Storage Facility.

If any further information or assistance is needed, please contact John Renda, Chemistry Manager, at 609-971-2572.

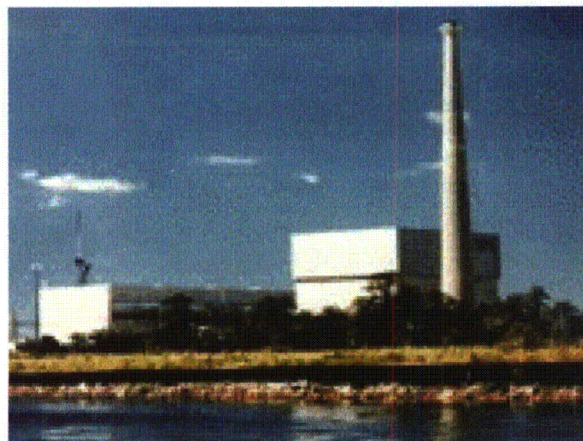
Sincerely,

Jeffrey Dostal
Plant Manager – Oyster Creek Nuclear Generating Station

Enclosure: 2014 Annual Radioactive Effluent Release Report

cc: Administrator, USNRC Region I (w/o attachment)
USNRC Senior Project Manager, Oyster Creek (w/o attachment)
USNRC Senior Resident Inspector, Oyster Creek (w/o attachment)
Craig Stewart, American Nuclear Insurers

IE48
NMSS26



Annual Radioactive Effluent Release Report

2014

Oyster Creek Generating Station

Oyster Creek 2014 Annual Radioactive Effluent Release Report

ANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT

January 1, 2014 through December 31, 2014

EXELON GENERATION COMPANY, LLC

OYSTER CREEK GENERATING STATION

DOCKET NO. 50-219 (Oyster Creek Generating Station)

DOCKET NO. 72-15 (Independent Spent Fuel Storage Facility)

**Submitted to
The United States Nuclear Regulatory Commission
Pursuant to
Renewed Facility Operating License DPR-16**

Oyster Creek 2014 Annual Radioactive Effluent Release Report

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EXECUTIVE SUMMARY

Effluents are strictly monitored to ensure that radioactivity released to the environment is as low as reasonably achievable and does not exceed regulatory limits. Effluent control includes the operation of monitoring systems, in-plant and environmental sampling and analyses programs, quality assurance programs for the effluent and environmental programs, and procedures covering all aspects of effluent and environmental monitoring.

Both radiological environmental and effluent monitoring indicate that the operation of Oyster Creek Generating Station (OCGS) does not result in significant radiation exposure to the people or the environment surrounding OCGS and is well below the applicable levels set by the Nuclear Regulatory Commission (NRC) and the Environmental Protection Agency (EPA).

There were liquid radioactive effluent releases during 2014 of concentrations of tritium too low to detect at an LLD of 200 picocuries per liter (pCi/L) at the New Jersey Pollution Discharge Elimination System (NJPDDES) permitted main condenser outfall. The releases were part of nearly continuous pumping of groundwater at approximately 70 gpm containing low levels of tritium and no detectable gamma. Exelon and the State of New Jersey Department of Environmental Protection (NJDEP) agreed to this remediation action instead of natural attenuation to address concentrations of tritium in groundwater. Well 73 and supporting equipment and piping were installed to pump groundwater to the intake structure at the inlet of the main circulating water pumps. Provisions were established for both batch and continuous releases of groundwater. Continuous releases occurred approximately 230 days in 2014. The nearly continuous releases occurred from March 12, 2014 through December 31, 2014 with a total of 2.31E+07 gallons of groundwater pumped resulting in 1.74E-01 Ci of tritium released to the discharge canal. The dose to the most limiting member of the public due to the release of groundwater was 8.28E-07 mrem.

There were no liquid abnormal releases during 2014.

There were two gaseous abnormal releases during 2014. During maintenance of the Old Radwaste Building exhaust fan EF-1-17 the side panel was removed to access the fan when positive pressure was noted coming from the opening. The side panel was immediately replaced and sealed. During maintenance of the New Radwaste Building exhaust fan HV-S-54A, two slits were identified in the expansion joint and repaired.

The maximum calculated organ dose (Bone) from iodines, tritium, carbon-14 (C-14), and particulates to any individual due to gaseous effluents was 4.78E-01 mrem, which was approximately 3.19E+00 percent of the annual limit of 15 mrem. The majority of organ dose from gaseous effluents was due to C-14. The maximum calculated gamma air dose in the UNRESTRICTED AREA due to noble gas effluents was 1.62E-02 mrad, which was 1.62E-01 percent of the annual 10 CFR 50 Appendix I, As Low As Reasonably Achievable (ALARA) limit of 10 mrad.

For comparison, the background radiation dose averages approximately 300 mrem per year in the Central New Jersey area, which includes approximately 200 mrem from naturally occurring radon gas and 100 mrem from background radiation.

The Independent Spent Fuel Storage Installation (ISFSI) is a closed system and the only exposure

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is due to direct radiation. Based on offsite TLD readings, dose due to direct radiation from the ISFSI was less than 1 mrem for 2014. Because it is a sealed unit, no radioactive material was released.

Comparison of environmental sampling results to iodine and particulate gaseous effluents released, showed no radioactivity attributable to the operation of OCGS. Both elevated and ground-level release paths were considered in this review, with total iodines released of $5.29\text{E-}03$ Ci and total particulates with half-lives greater than 8 days less C-14 released of $3.33\text{E-}02$ Ci.

Joint Frequency Tables of meteorological data, per Stability Classification Category, as well as for all stability classes, are included. All data was collected from the on-site Meteorological Facility. Data recoveries for the 380-foot data and the 33-foot data were 99.9 percent and 99.9 percent, respectively. The UFSAR commits to Regulatory Guide (RG) 1.23 for Meteorological Facility data recovery. RG 1.23 requires data recovery of at least 90% on an annual basis.

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1 Introduction

In accordance with the reporting requirements of Technical Specification 6.9.1.d applicable during the reporting period, this report summarizes the effluent release data for OCGS for the period January 1, 2014 through December 31, 2014. This submittal complies with the format described in Regulatory Guide 1.21, "Measuring, Evaluating and Reporting Radioactivity in Solid Wastes and Releases of Radioactive Materials in Liquid and Gaseous Effluents from Light-Water-Cooled Nuclear Power Plants", Revision 1, June 1974.

Meteorological data was reported in the format specified in Regulatory Guide 1.23, Revision 1, "Meteorological Monitoring Programs for Nuclear Power Plants".

All vendor results were received and included in the report calculations. Therefore the 2014 report is complete.

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2 Supplemental Information

Oyster Creek Generating Station

Exelon Generation Company, LLC

A. Regulatory Limits:

	Limit	Units	Receptor	ODCM and 10 CFR 50, Appendix I Design Objective Limits
1. Noble Gases:				
a.	≤ 500	mrem/yr	Total Body	ODCM Control 3.11.2.1
	≤ 3000	mrem/yr	Skin	
b.	≤ 5	mrad	Air Gamma	Quarterly air dose limits ODCM Control 3.11.2.2
	≤ 10	mrad	Air Beta	
c.	≤ 10	mrad	Air Gamma	Yearly air dose limits ODCM Control 3.11.2.2
	≤ 20	mrad	Air Beta	
d.	< 5	mrem	Total Body (Gamma)	10 CFR 50, Appendix I, Section II.B.2(b)
	< 15	mrem	Skin (Beta)	
2. Iodines, Tritium, Particulates with Half Life > 8 days:				
a.	≤ 1500	mrem/yr	Any Organ	ODCM Control 3.11.2.1
b.	≤ 7.5	mrem	Any Organ	Quarterly dose limits ODCM Control 3.11.2.3
c.	≤ 15	mrem	Any Organ	Yearly dose limits ODCM Control 3.11.2.3
3. Liquid Effluents				
a.	Concentration 10 CFR 20, Appendix B, Table 2 Column 2			ODCM Control 3.11.1.1
b.	≤ 1.5	mrem	Total Body	Quarterly dose limits ODCM Control 3.11.1.2
	≤ 5	mrem	Any Organ	
c.	≤ 3	mrem	Total Body	Yearly dose limits ODCM Control 3.11.1.2
	≤ 10	mrem	Any Organ	

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B. Effluent Concentration Limits:

Gaseous dose rates rather than effluent concentrations are used to calculate permissible release rates for gaseous releases. The maximum permissible dose rates for gaseous releases are defined in ODCM Controls 3.11.2.1.

The Effluent Concentration Limit (ECL) specified in 10 CFR 20, Appendix B, Table 2, Column 2 for identified nuclides, were used to calculate permissible release rates and concentrations for liquid release per ODCM Controls 3.11.1.1. The total activity concentration at the Route 9 bridge for all dissolved or entrained gases was limited to $< 2E-04 \mu\text{Ci/ml}$.

C. Average Energy (\bar{E}):

The Oyster Creek ODCM limits the instantaneous dose equivalent rates due to the release of noble gases to less than or equal to 500 mrem/year to the total body and less than or equal to 3000 mrem/year to the skin. The average beta and gamma energies (\bar{E}) of the radionuclide mixture in releases of fission and activation gases as described in Regulatory Guide 1.21, "Measuring, Evaluating and Reporting Radioactivity in Solid Wastes and Releases of Radioactive Materials in Liquid and Gaseous Effluents from Light-Water-Cooled Nuclear Power Plant", may be used to calculate doses in lieu of more sophisticated software. The Oyster Creek radioactive effluent program employs the methodologies presented in U.S. NRC Regulatory Guide 1.109 "Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents for the Purpose of Evaluating Compliance with 10 CFR Part 50, Appendix I," Revision 1, October 1977. Therefore, average energy (\bar{E}) as described in Regulatory Guide 1.21 is not applicable to Oyster Creek.

D. Measurements and Approximations of Total Radioactivity:

1. Fission and Activation Gases

The method used for Gamma Isotopic Analysis is the Canberra Gamma Spectroscopy System with a gas Marinelli beaker. Airborne effluent gaseous activity was continuously monitored and recorded in accordance with the Off Site Dose Calculation Manual (ODCM) Table 4.11.2.1.2-1. Additional grab samples were taken from the stack Radioactive and Gaseous Effluent Monitoring System (RAGEMS) sample point and ground-level release sample points and analyzed at least monthly to determine the isotopic mixture of noble gas activity released for the month. If activity was found in the grab isotopic analysis, the results are entered into Simplified Environmental Effluent Dosimetry System (SEEDS) to calculate dose and dose rates. If no activity is detected in the stack grab samples, post treatment or Off Gas Isotopic Analysis data may be used.

2. Iodines

The method used for Gamma Isotopic Analysis is the Canberra Gamma Spectroscopy System with a charcoal cartridge. Iodine activity was continuously sampled and analyzed in accordance with ODCM Table 4.11.2.1.2-1. Charcoal samples are taken from the stack RAGEMS sample point and

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ground-level release sample points and analyzed at least weekly to determine the total activity released from the plant based on the average vent flow rates recorded for the sampling period.

3. Particulates (half-lives > 8 days)

The method used for Gamma Isotopic Analysis is the Canberra Gamma Spectroscopy System with a particulate filter (47 mm). Particulate activity was continuously sampled and analyzed in accordance with ODCM Table 4.11.2.1.2-1. Particulate samples are taken from the stack RAGEMS sample point and ground-level release sample points and analyzed at least weekly to determine the total activity released from the plant based on the average vent flow rates recorded for the sampling period.

4. Tritium

A. Gaseous Effluents

Air from stack and vent effluents was passed through a desiccant column and distilled to remove the moisture collected. An aliquot of the water from the distillate was analyzed for tritium using a liquid scintillation counter.

B. Liquid Effluents

Water from liquid effluents was analyzed for tritium using a liquid scintillation counter.

5. Gross Alpha

Gross alpha was measured by an off-site vendor for both the gas and liquid effluent composite samples.

6. Hard-To-Detects

Hard-To-Detects was measured by an off-site vendor for one set of gas monthly composites. The analysis included Fe-55, I-129, Ni-59, Ni-63, Tc-99, Am-241, Cm-242, Cm-243/244, Pu-238, Pu-239/240 and Pu-241. Fe-55 and Ni-63 have been added to the routine monthly composite analysis schedule based on previous sample results for Hard-To-Detects.

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7. Carbon-14 (C-14)

The amount of C-14 (Ci) released was estimated using the guidance from EPRI Technical Report 1021106, Estimation of Carbon-14 in Nuclear Power Plant Gaseous Effluents. The C-14 was released primarily through the stack (97%) with a small amount (3%) released through plant vents. The activity in liquid effluents was determined to not be significant.

The offsite dose from C-14 was calculated using SEEDS, which uses approved ODCM methodologies. The resulting annual dose to a child from gaseous releases of C-14 is about 4.53E-01 mrem to the bone.

8. Liquid Effluents

Groundwater containing tritium was released during 2014. For continuous releases, tritium and principal gamma emitters were determined for a composite sample daily. The concentration of tritium is limited to ensure concentrations were less than 200 pCi/l in the discharge canal. The gamma emitters were limited to less than detectable concentrations. Gross alpha and Hard-to-detect analyses (Fe-55, Ni-63, Sr-89 and Sr-90) were determined for monthly composite samples for each type of release (batch or continuous).

The leaks into the groundwater were reported in the 2009 Annual Radioactive Effluent Release Report as abnormal releases. Estimates of the curies of the tritium releases were reported. Doses due to the release of the groundwater to the discharge canal were included in the report. To ensure that amount of activity discharge is accurate and limiting, the activity and doses as a result of discharges during 2014 from the groundwater remediation project are included in this report.

9. Estimated Total Error Present

Procedure CY-AA-170-2100, Estimated Errors of Effluent Measurements, provides the methodology to obtain an overall estimate of the error associated with radioactive effluents.

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10. Composite Samples and Lower Limit of Detection (LLD)

Particulate air samples were composited monthly and analyzed for gross alpha, Sr-89, Sr-90, Fe-55 and Ni-63. Groundwater batch and continuous releases were composited at least monthly and analyzed for gross alpha, Sr-89, Sr-90, Fe-55 and Ni-63. These composites are submitted to an offsite vendor laboratory for analysis. The ODCM required LLD for liquid and airborne releases are as follows:

Liquid:	LLD
Principal Gamma Emitters (Mn-54, Fe-59, Co-58, Co-60, Zn-65, Mo-99, I-131, Ce-141, Cs-134, Cs-137)	5E-07 µCi/ml
Principal Gamma Emitters (Ce-144)	5E-06 µCi/ml
Dissolved and Entrained Gases	1E-05 µCi/ml
H-3	1E-05 µCi/ml
Gross Alpha	1E-07 µCi/ml
Sr-89 and Sr-90	5E-08 µCi/ml
Fe-55 and Ni-63	1E-06 µCi/ml
Airborne	LLD
Principal Gamma Emitters (Kr-87, Kr-88, Xe-133, Xe-133m, Xe-135, Xe-138)	1E-04 µCi/ml
H-3	1E-06 µCi/ml
I-131	1E-12 µCi/ml
I-133	1E-10 µCi/ml
Principal Gamma Emitters (Mn-54, Fe-59, Co-58, Co-60, Zn-65, Cs-134, Cs-137, Ce-141)	1E-11 µCi/ml
Principal Gamma Emitters (Mo-99, Ce-144)	1E-10 µCi/ml
Gross Alpha	1E-11 µCi/ml
Sr-89, Sr-90	1E-11 µCi/ml

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E. Batch Releases:

1. Liquid

There were no batch releases of liquid effluents during 2014.

2. Gaseous

There were no batch releases of gaseous effluents during 2014.

F. Abnormal Releases:

There were no abnormal liquid releases during 2014.

There were two abnormal gaseous releases during 2014.

During the performance of Work Order C2031047, the side panel of exhaust fan EF-1-17 was removed to access the fan when positive pressure was noted coming from the opening. The side panel was immediately replaced and sealed. A tent had been built around the work area but there was no HEPA filter in place to prevent any releases from the fan from going to the atmosphere. Note that the exhaust from EF-1-17 is normally to the atmosphere but through the monitored stack effluent release point. Engineering performed a calculation to determine a release rate that encompasses the maximum release rate through this opening.

The following assumptions were made when calculating Curies released and dose:

- Though the panel was off for less than 15 minutes, 15 minutes was used as the release period.
- The activity due to particulates identified from the sample at the time of the release at the opening was released as a ground level release from the exhaust fan opening at the flow rate calculated by Engineering.

	Units	Quarter 1	Quarter 2	Quarter 3	Quarter 4
Total Release	Ci	0.00e0	0.00e0	0.00e0	1.29e-07
Organ Dose	mrem	0.00e0	0.00e0	0.00e0	1.35E-07

During the performance of Work Order R2227008, which is to perform an annual inspection of the New Radwaste (NRW) exhaust fan HV-S-54A, two slits were discovered in the expansion joint. This is one of the exhaust fans for the New Radwaste Building to the Main Stack. Note that the exhaust from HV-S-54A is normally to the atmosphere but through the monitored stack effluent release point. Engineering performed a calculation to determine a release rate that encompasses the maximum release rate for both slits. The activity to inspect the exhaust fan is an annual activity.

The following assumptions were made when calculating Curies released and dose:

- The exhaust fan has been leaking since the date of the last inspection until the time of the repair, 8/20/13 through 7/25/14.

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- All of the activity due to particulates reported from the Main Stack was coming from the New Radwaste Building and therefore being released as a ground level release from the exhaust fan leaks at the flow rate calculated by Engineering.
- HV-S-54A was in operation the entire time since the last inspection was performed.

	Units	Quarter 1	Quarter 2	Quarter 3	Quarter 4
Total Release	Ci	8.81E-05	8.90E-05	2.41E-05	0.00e0
Organ Dose	mrem	4.78E-04	4.63E-04	1.56E-04	0.00e0

G. Revisions to the ODCM:

Revision 6 of the ODCM, CY-OC-170-301 was implemented 12/16/2014. A list of the major changes is included below. See the complete copy of CY-OC-170-301 Revision 6 attached as part of this report.

- Meteorological data was updated resulting in changes to X/Q and D/Q values. These changes were also used to update the A γ Si, P γ Si and DFSi dose factors.
- Distances to the site boundary were updated.
- AOG/NRW CCW Service Water Effluent was removed from Table 4.11.1.1.1-1, Radioactive Liquid Waste Sampling and Analysis Program

H. Radiation Effluent Monitors Out of Service More Than 30 Days

Per ODCM Control 3.3.3.10, "Radioactive Liquid Effluent Monitoring Instrumentation" and 3.3.3.11, Radioactive Gaseous Effluent Monitoring Instrumentation requires:

With less than the minimum number of radioactive liquid/gaseous effluent monitoring instrumentation channels OPERABLE, take the ACTION shown in Table 3.3.3.10-1/3.3.3.11-1. Make every reasonable effort to return the instrument to OPERABLE status within 30 days and, if unsuccessful, explain in the next Radioactive Effluent Release Report why the inoperability was not corrected in a timely manner.

The following is a discussion of instrumentation out of service for greater than 30 days:

1. There was no instrumentation out of service for greater than 30 days.

I. Releases from the Independent Spent Fuel Storage Facility:

The ISFSI is a closed system and the only exposure would be due to direct radiation. This includes iodines, particulates, and noble gases. Based on offsite

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TLD readings, dose due to direct radiation from the ISFSI was less than 1 mrem for 2014. Because it is a sealed unit, no radioactive material was released.

J. Program Deviations:

1. The groundwater remediation composite sampler was found with no composite collected from August 26, 2014 at 3:47 AM through August 26, 2014 at 10:00 AM (approximately 6.25 hours). The composite sampler is required by ODCM Table 4.11.1.1.1-1, Radioactive Liquid Waste Sampling and Analysis Program and is set to obtain a composite sample once per hour. This issue occurred due to Operations starting the remediation pump per the section of the procedure for by-passing the sample station. This issue was entered into our Corrective Action Program (CAP) and corrective actions taken have been documented per process.
2. The New Radwaste (NRW)/Augmented Offgas (AOG) Service Water Effluent sample was not collected January 1, 2014 through November 13, 2014. The New Radwaste/Augmented Offgas Service Water Effluent sample was required by Revision 5 of CY-OC-170-301, Offsite Dose Calculation Manual for Oyster Creek Generating Station Table 4.11.1.1.1-1, Radioactive Liquid Waste Sampling and Analysis Program and was required to be sampled weekly for principal gamma emitters and I-131. This requirement was removed in Revision 6 of CY-OC-170-301. This issue occurred from the inclusion of this requirement in revision 4 of the ODCM due to a misunderstanding of the system flow path. It was believed that all service water exited the site from the same release point which was evident by the fact that the change paperwork included in the description of this change as being consistent with the practices at Oyster Creek and there was no modification to the sampling schedule. The NRW and AOG Closed Cooling Water systems were reviewed as part of NRC Bulletin IE 80-10. At that time NRW Closed Cooling Water system was determined to be a potentially contaminated system and was added to the sampling program for NRC Bulletin IE 80-10. The sampling program for IE 80-10 requires all potentially contaminated systems to be sampled quarterly for gamma emitters. AOG Closed Cooling Water system was also reviewed but determined to not be a potentially contaminated system. Even though AOG was found not to be a potentially contaminated system, sampling is still performed quarterly for gamma emitters. The potential pathway for a release from NRW/AOG Service Water Effluent would be from a heat exchanger leak from the NRW Closed Cooling Water system into the Service Water side of the heat exchanger. The system is designed to maintain the Service Water side of the heat exchanger at a higher pressure than the Closed Cooling Water side so that if any leaks would develop in the heat exchanger it would be Service Water leaking into the Closed Cooling Water side. A review was performed for both NRW and AOG quarterly Closed Cooling Water samples for the time that the required samples were not obtained and all samples were less than MDA for gamma emitters for that entire time period.

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Appendix A Effluent and Waste Disposal Summary

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Table A-1: Gaseous Effluents - Summary Of All Releases

Period: January 1, 2014 through December 31, 2014

Unit: Oyster Creek

A. Fission & Activation Gases	Units	Quarter 1	Quarter 2	Quarter 3	Quarter 4	Est. Total Error %
1. Total Release	Ci	1.95E+01	2.07E+01	1.45E+01	7.76E+00	25.00%
2. Average Release Rate for Period	µCi/sec	2.51E+00	2.63E+00	1.83E+00	9.77E-01	
3. Gamma Air Dose	mrad	9.68E-04	1.30E-02	2.86E-03	4.97E-04	
4. Beta Air Dose	mrad	3.97E-04	1.63E-02	3.47E-03	4.43E-05	
5. Percent of ODCM Limit						
- Gamma Air Dose	%	1.94E-02	2.60E-01	5.72E-02	9.94E-03	
- Beta Air Dose	%	3.97E-03	1.63E-01	3.47E-02	4.43E-04	
B. Iodines						
1. Total – I-131	Ci	3.88E-04	6.19E-04	6.32E-04	7.18E-05	25.00%
2. Average Release Rate for Period	µCi/sec	4.99E-05	7.87E-05	7.95E-05	9.04E-06	
3. Percent of ODCM limit	%	*	*	*	*	
C. Particulate						
1. Particulates with T 1/2 > 8 days	Ci	8.98E-03	6.33E-03	1.30E-02	5.02E-03	25.00%
2. Average Release Rate for Period	µCi/sec	1.16E-03	8.05E-04	1.63E-03	6.32E-04	
3. Percent of ODCM limit	%	*	*	*	*	
D. Tritium						
1. Total Release	Ci	6.44E+00	1.00E+01	7.93E+00	4.57E+00	25.00%
2. Average Release Rate for Period	µCi/sec	8.28E-01	1.27E+00	9.98E-01	5.75E-01	
3. Percent of ODCM limit	%	*	*	*	*	
E. Gross Alpha						
1. Total Release	Ci	<LLD	<LLD	<LLD	<LLD	25.00%
2. Average Release Rate for Period	µCi/sec	<LLD	<LLD	<LLD	<LLD	
3. Percent of ODCM limit	%	*	*	*	*	
F. Carbon-14						
1. Total Release	Ci	2.39E+00	2.40E+00	1.81E+00	2.04E+00	
2. Average Release Rate for Period	µCi/sec	3.08E-01	3.06E-01	2.28E-01	2.57E-01	
3. Percent of ODCM limit	%	*	*	*	*	
G. Iodine 131 & 133, Tritium & Particulate						
1. Organ Dose	mrem	3.86E-02	1.57E-01	1.67E-01	1.18E-01	
2. Percent of ODCM Limit	%	5.15E-01	2.09E+00	2.23E+00	1.57E+00	

* ODCM Limit is for combined Iodine, tritium, Carbon-14 and particulate only, which is shown in Item G.

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Table A-2: Gaseous Effluents Release Point: Elevated Release

Period: January 1, 2014 through December 31, 2014

Unit: Oyster Creek

Nuclides Released	Unit	Continuous Mode				Batch Mode			
		Quarter 1	Quarter 2	Quarter 3	Quarter 4	Quarter 1	Quarter 2	Quarter 3	Quarter 4
1. Fission gases									
Kr- 85	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Kr- 85m	Ci	1.90E+00	1.51E+00	8.55E-01	4.38E-01	<LLD	<LLD	<LLD	<LLD
Kr-87	Ci	3.24E+00	1.08E+00	2.39E+00	<LLD	<LLD	<LLD	<LLD	<LLD
Kr-88	Ci	2.13E+00	2.26E+00	5.80E-01	1.98E+00	<LLD	<LLD	<LLD	<LLD
Xe-133	Ci	1.45E+00	1.70E+00	1.06E+00	5.25E-01	<LLD	<LLD	<LLD	<LLD
Xe-135	Ci	5.96E+00	1.11E+01	8.71E+00	4.82E+00	<LLD	<LLD	<LLD	<LLD
Xe-135m	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Xe-137	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Xe-138	Ci	4.83E+00	1.71E+00	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Ar-41	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Total for Period	Ci	1.95E+01	1.94E+01	1.36E+01	7.76E+00	<LLD	<LLD	<LLD	<LLD
2. Iodines									
I-131	Ci	3.88E-04	6.18E-04	6.19E-04	7.03E-05	<LLD	<LLD	<LLD	<LLD
I-133	Ci	8.14E-04	1.76E-03	9.50E-04	5.20E-05	<LLD	<LLD	<LLD	<LLD
I-135	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Total for Period	Ci	1.20E-03	2.38E-03	1.57E-03	1.22E-04	<LLD	<LLD	<LLD	<LLD
3. Particulates									
Sr-89	Ci	6.68E-04	1.57E-04	2.93E-04	6.45E-05	<LLD	<LLD	<LLD	<LLD
Sr-90	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Cs-134	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Cs-137	Ci	1.92E-05	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Ba-140	Ci	1.13E-03	4.17E-04	7.12E-04	3.70E-04	<LLD	<LLD	<LLD	<LLD
La-140	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Cr-51	Ci	<LLD	<LLD	7.77E-05	2.09E-05	<LLD	<LLD	<LLD	<LLD
Mn-54	Ci	5.40E-04	4.16E-04	1.14E-03	3.98E-04	<LLD	<LLD	<LLD	<LLD
Co-58	Ci	2.41E-03	1.97E-03	3.96E-03	2.00E-03	<LLD	<LLD	<LLD	<LLD
Co-60	Ci	3.04E-03	2.30E-03	5.01E-03	1.88E-03	<LLD	<LLD	<LLD	<LLD
Ni-63	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Mo-99	Ci	2.46E-04	2.25E-04	3.52E-04	5.20E-05	<LLD	<LLD	<LLD	<LLD
Ag-110m	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Ce-141	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Ce-144	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Fe-55	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Fe-59	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Zn-65	Ci	8.40E-04	7.41E-04	1.38E-03	2.36E-04	<LLD	<LLD	<LLD	<LLD
Total for Period	Ci	8.89E-03	6.23E-03	1.29E-02	5.02E-03	<LLD	<LLD	<LLD	<LLD
4. Tritium									
H-3	Ci	6.17E+00	9.80E+00	7.49E+00	4.40E+00	<LLD	<LLD	<LLD	<LLD
5. Gross Alpha									
Gross Alpha	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
6. Carbon-14									
C-14	Ci	2.32E+00	2.33E+00	1.76E+00	1.98E+00	<LLD	<LLD	<LLD	<LLD

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table A-3: Gaseous Effluent Release Point: Ground Level Releases

Period: January 1, 2014 through December 31, 2014

Unit: Oyster Creek

Nuclides Released	Unit	Continuous Mode				Batch Mode			
		Quarter 1	Quarter 2	Quarter 3	Quarter 4	Quarter 1	Quarter 2	Quarter 3	Quarter 4
1. Fission gases									
Kr- 85	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Kr- 85m	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Kr-87	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Kr-88	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Xe-133	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Xe-133m	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Xe-135	Ci	<LLD	1.34E+00	9.26E-01	<LLD	<LLD	<LLD	<LLD	<LLD
Xe-135m	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Xe-138	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Ar-41	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Total for Period	Ci	<LLD	1.34E+00	9.26E-01	<LLD	<LLD	<LLD	<LLD	<LLD
2. Iodines									
I-131	Ci	<LLD	9.91E-07	1.30E-05	1.52E-06	<LLD	<LLD	<LLD	<LLD
I-133	Ci	<LLD	4.86E-06	1.94E-06	<LLD	<LLD	<LLD	<LLD	<LLD
I-135	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Total for Period	Ci	<LLD	5.85E-06	1.49E-05	1.52E-06	<LLD	<LLD	<LLD	<LLD
3. Particulates									
Sr-89	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Sr-90	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Cs-134	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Cs-137	Ci	8.29E-07	8.38E-07	2.27E-07	<LLD	<LLD	<LLD	<LLD	<LLD
Ba-140	Ci	1.19E-05	1.20E-05	3.26E-06	<LLD	<LLD	<LLD	<LLD	<LLD
La-140	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Cr-51	Ci	6.72E-06	6.79E-06	1.84E-06	<LLD	<LLD	<LLD	<LLD	<LLD
Mn-54	Ci	8.03E-06	8.12E-06	2.20E-06	<LLD	<LLD	<LLD	<LLD	<LLD
Co-58	Ci	2.30E-05	2.36E-05	6.65E-06	7.76E-07	<LLD	<LLD	<LLD	<LLD
Co-60	Ci	2.81E-05	3.78E-05	1.94E-05	2.18E-06	<LLD	<LLD	<LLD	<LLD
Ni-63	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Mo-99	Ci	2.21E-06	2.24E-06	6.06E-07	<LLD	<LLD	<LLD	<LLD	<LLD
Ag-110m	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Ce-141	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Ce-144	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Fe-55	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Fe-59	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Zn-65	Ci	8.71E-06	8.81E-06	2.38E-06	<LLD	<LLD	<LLD	<LLD	<LLD
Am-241	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Total for Period	Ci	8.95E-05	1.00E-04	3.66E-05	2.96E-06	<LLD	<LLD	<LLD	<LLD
4. Tritium									
H-3	Ci	2.66E-01	2.03E-01	4.40E-01	1.70E-01	<LLD	<LLD	<LLD	<LLD
5. Gross Alpha									
Gross Alpha	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
6. Carbon-14									
C-14	Ci	7.20E-02	7.21E-02	5.44E-02	6.13E-02	<LLD	<LLD	<LLD	<LLD

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table A-4: Liquid Effluents - Summary Of All Releases

Period: January 1, 2014 through December 31, 2014

Unit: Oyster Creek

A. Fission & Activation Products	Units	Quarter 1	Quarter 2	Quarter 3	Quarter 4	Est. Total Error %
1. Total Release not including tritium, gases, alpha	Ci	<LLD	<LLD	<LLD	<LLD	25.00%
2. Average Diluted concentration during period	µCi/ml	<LLD	<LLD	<LLD	<LLD	
3. Total Body Dose	mrem	7.70E-08	3.26E-07	2.82E-07	1.43E-07	
4. Organ Dose	mrem	7.70E-08	3.26E-07	2.82E-07	1.43E-07	
3. Percent of ODCM Limit						
-Total Body Dose	%	5.13E-06	2.18E-05	1.88E-05	9.53E-06	
-Organ Dose	%	1.54E-06	6.53E-06	5.64E-06	2.86E-06	
B. Tritium						
	Units	Quarter 1	Quarter 2	Quarter 3	Quarter 4	Est. Total Error %
1. Total Release	Ci	1.59E-02	6.95E-02	5.85E-02	3.05E-02	25.00%
2. Average diluted concentration during period	µCi/ml	1.53E-10	1.43E-10	1.43E-10	1.39E-10	
3. Percent of 10CFR20 limit	%	1.53E-05	1.43E-05	1.43E-05	1.39E-05	
C. Dissolved and Entrained Gases						
	Units	Quarter 1	Quarter 2	Quarter 3	Quarter 4	Est. Total Error %
1. Total Release	Ci	<LLD	<LLD	<LLD	<LLD	25.00%
2. Average diluted concentration	µCi/ml	<LLD	<LLD	<LLD	<LLD	
3. Percent of ODCM limit	%	<LLD	<LLD	<LLD	<LLD	
D. Gross Alpha Activity						
	Units	Quarter 1	Quarter 2	Quarter 3	Quarter 4	Est. Total Error %
1. Total Release	Ci	<LLD	<LLD	<LLD	<LLD	25.00%
E. Volume of Waste Released prior to dilution						
	Liters	7.44E+06	3.50E+07	2.95E+07	1.54E+07	
F. Volume of Dilution Water Used During Period						
	Liters	1.04E+11	4.85E+11	4.09E+11	2.19E+11	

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table A-5: Liquid Release Point: Groundwater Remediation

Period: January 1, 2014 through December 31, 2014

Unit: Oyster Creek

Nuclides Released	Unit	Continuous Mode				Batch Mode			
		Quarter 1	Quarter 2	Quarter 3	Quarter 4	Quarter 1	Quarter 2	Quarter 3	Quarter 4
Fission & Activation Products									
Sr-89	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Sr-90	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Cs-134	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Cs-137	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
I-131	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Co-58	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Co-60	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Ni-63	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Fe-59	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Zn-65	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Mn-54	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Cr-51	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Zr-95	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Nb-95	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Mo-99	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Tc-99m	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Ba-140	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
La-140	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Ce-141	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Ag-110m	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Fe-55	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Ce-144	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Total for Period		<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Dissolved Entrained Gases									
Xe-133	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Xe-135	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Tritium									
H-3	Ci	1.59E-02	6.95E-02	5.85E-02	3.05E-02	<LLD	<LLD	<LLD	<LLD
Gross Alpha									
Gross Alpha	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Appendix B Solid Waste and Irradiated Fuel Shipments

Oyster Creek 2014 Annual Radioactive Effluent Release Report

A. Solid waste shipped offsite for burial or disposal (not irradiated fuel)

1. Type of waste

Types of Waste	Total Quantity (m ³)	Total Activity (Ci)	Period	Est. Total Error%
a. Spent resins, filter sludges, evaporator bottom, etc	4.71E+01	1.55E+02	2014	2.50E+01
b. Dry compressible waste, contaminated equip, etc	5.61E+02	1.09E+00	2014	2.50E+01
c. Irradiated components, control rods, etc	1.66E-01	4.24E+03	2014	2.50E+01
d. Other	1.10E+02	5.06E-01	2014	2.50E+01

Oyster Creek 2014 Annual Radioactive Effluent Release Report

1. Estimate of Major Nuclide Composition (By Waste Type)

Category A – Spent Resin, Filters, Sludges, Evaporator Bottoms, etc.

Isotope	Waste Class A		Waste Class B		Waste Class C	
	Curies	Percent	Curies	Percent	Curies	Percent
H-3	5.13E-02	8.87E-02	4.55E-03	4.68E-03		
C-14	4.83E-02	8.35E-02	1.03E-03	1.06E-03		
Cr-51						
P-32						
Mn-54	3.05E+00	5.27E+00	2.72E-01	2.80E-01		
Fe-55	3.05E+01	5.27E+01	6.30E+01	6.48E+01		
Fe-59						
Co-57	1.78E-03	3.08E-03				
Co-58	1.46E-02	2.52E-02				
Co-60	2.22E+01	3.84E+01	2.24E+01	2.30E+01		
Ni-59			2.21E-02	2.27E-02		
Ni-63	5.28E-01	9.13E-01	1.58E+00	1.63E+00		
Zn-65	6.02E-01	1.04E+00	3.14E-02	3.23E-02		
Sr-89	1.12E-05	1.94E-05				
Sr-90	3.30E-03	5.71E-03	8.85E-02	9.10E-02		
Nb-95						
Tc-99			2.72E-03	2.80E-03		
Ag-110m						
Sb-125	1.50E-02	2.59E-02				
Cs-134						
Cs-137	7.54E-01	1.30E+00	9.73E+00	1.00E+01		
Ce-144	3.45E-02	5.97E-02	2.04E-02	2.10E-02		
Pu-238	1.16E-04	2.01E-04	1.71E-03	1.76E-03		
Pu-239	1.78E-05	3.08E-05	5.55E-04	5.71E-04		
Pu-240	1.78E-05	3.08E-05				
Pu-241	2.06E-02	3.56E-02	5.56E-02	5.72E-02		
Am-241	8.05E-05	1.39E-04	2.09E-03	2.15E-03		
Cm-242	1.59E-05	2.75E-05				
Cm-243	3.69E-05	6.38E-05	1.40E-03	1.44E-03		
Cm-244	1.67E-05	2.89E-05	1.30E-03	1.34E-03		
Totals	5.78E+01	1.00E+02	9.72E+01	1.00E+02	0.00E+00	0.00E+00

Note: Grey fields are where results were not reported in the NRC Regulatory Guide 1.21 Report

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Category B – Dry Compressible Waste, Contaminated Equipment, etc.

Isotope	Waste Class A	
	Curies	Percent
H-3		
C-14	7.28E-06	6.70E-04
P-32	6.10E-05	5.61E-03
Mn-54	3.67E-02	3.38E+00
Fe-55	7.99E-01	7.35E+01
Co-57	9.24E-05	8.50E-03
Co-58	1.80E-03	1.66E-01
Co-60	1.90E-01	1.75E+01
Ni-63	6.93E-03	6.38E-01
Zn-65	1.82E-02	1.67E+00
Tc-99		
Sb-125		
Sr-89	1.50E-04	1.38E-02
Sr-90	1.89E-04	1.74E-02
Cs-137	3.29E-02	3.03E+00
Ce-144	5.00E-04	4.60E-02
Pu-238	6.22E-06	5.72E-04
Pu-239	1.90E-06	1.75E-04
Pu-241	2.19E-04	2.02E-02
Am-241	7.52E-06	6.92E-04
Cm-242	1.24E-06	1.14E-04
Cm-243	2.43E-06	2.24E-04
Cm-244	2.43E-06	2.24E-04
Totals	1.09E+00	1.00E+02

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Category C – Irradiated components, control rods, etc.

Isotope	Waste Class C	
	Curies	Percent Abundance %
H-3	1.06E-02	2.50E-04
C-14	1.16E-01	2.73E-03
Cr-51	6.22E-02	1.47E-03
Mn-54	1.83E+01	4.31E-01
Fe-55	4.77E+02	1.12E+01
Fe-59	3.27E-02	7.71E-04
Co-58	1.19E+00	2.81E-02
Co-60	2.75E+03	6.48E+01
Ni-59	2.92E-01	6.88E-03
Ni-63	3.62E+01	8.53E-01
Zn-65	2.83E-01	6.67E-03
Sr-90	2.79E-05	6.58E-07
Zr-95	1.37E+02	3.23E+00
Nb-94	2.60E-03	6.13E-05
Mo-93	2.91E-05	6.86E-07
Tc-99	3.32E-05	7.83E-07
Sb-125	8.16E+02	1.92E+01
I-129	1.70E-09	4.01E-11
Cs-137	4.52E-05	1.07E-06
Ce-144	1.31E-07	3.09E-09
Hf-181	9.68E-01	2.28E-02
Ta-182	4.35E+00	1.03E-01
U-235	3.97E-11	9.36E-13
Np-237	9.79E-10	2.31E-11
Pu-238	1.35E-05	3.18E-07
Totals	4.24E+03	1.00E+02

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Category D - Other - Scrap Metal

Isotope	Waste Class A	
	Curies	Percent
H-3	4.70E-01	9.28E+01
C-14	2.42E-07	4.78E-05
P-32	1.82E-06	3.59E-04
Mn-54	1.23E-03	2.43E-01
Fe-55	2.66E-02	5.25E+00
Co-57	3.27E-06	6.46E-04
Co-58	6.11E-05	1.21E-02
Co-60	6.34E-03	1.25E+00
Ni-63	2.30E-04	4.54E-02
Zn-65	6.11E-04	1.21E-01
Sr-85	5.73E-13	1.13E-10
Sr-89	5.08E-06	1.00E-03
Sr-90	6.29E-06	1.24E-03
Y-88	9.40E-08	1.86E-05
Cd-109	8.87E-07	1.75E-04
Sn-113	2.62E-08	5.17E-06
Cs-137	1.33E-03	2.63E-01
Ba-133	1.15E-05	2.27E-03
Ce-139	3.82E-08	7.54E-06
Ce-144	1.67E-05	3.30E-03
Hg-203	8.55E-11	1.69E-08
Pu-238	2.07E-07	4.09E-05
Pu-239	6.30E-08	1.24E-05
Pu-241	7.26E-06	1.43E-03
Am-241	9.89E-06	1.95E-03
Cm-242	4.16E-08	8.21E-06
Cm-243	8.07E-08	1.59E-05
Cm-244	8.06E-08	1.59E-05
Totals	5.06E-01	1.00E+02

Note: Grey fields are where results were not reported in the NRC Regulatory Guide 1.21 Report

2. Solid Waste (Disposition)

Number of Shipments	Mode of Transportation	Destination
9	Hittman Transport Co.	Barnwell Disposal Facility
16	Hittman Transport Co.	Energy Solutions Services, Inc.

B. Irradiated Fuel Shipments (disposition).

There were no irradiated fuel shipments.

Oyster Creek 2014 Annual Radioactive Effluent Release Report

C. Changes to the Process Control Program

Revision 10 of the Process Control Program, RW-AA-100 was implemented August 26, 2014. See the complete copy of RW-AA-100 Revision 10 attached as part of this report.

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Appendix C Radiological Impact to Man

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Per ODCM Administrative Control 6.2, an assessment of radiation doses to the likely most exposed MEMBER OF THE PUBLIC from reactor releases and other nearby uranium fuel cycle sources (including doses from primary effluent pathways and direct radiation) for the previous calendar year must be made to show conformance with 40 CFR Part 190, Environmental Radiation Protection Standards for Nuclear Power Operation. For purposes of this calculation the following assumptions were made:

Gaseous

- Nearest member of the public was W sector at 483 meters.
- Actual 2014 meteorology and measured gaseous effluent releases were used.
- All significant pathways were assumed to be present.
- Occupancy factor was considered 22.8% (40 hours/week for 50 weeks).

Liquid

- Doses calculated in the discharge canal at the Route 9 Bridge.
- Fish, shellfish and shoreline pathways doses calculated.

40 CFR Part 190 Compliance

- Dosimetry measurements (minus average of control stations) measured direct radiation for the nearest member of the public. The nearest member of the public for direct radiation is considered an individual that works in the warehouse west of the site. As a worker, the individual is assumed to work 2,000 hours per year at this location. Note that for the warehouse worker total dose calculations the full year (8760 hours) values for Iodine, Particulate, Carbon-14, Tritium and Liquid are used.
- Nearest resident was at SE sector at 937 meters.
- The highest calculated dose for gamma air dose and liquid total body were summed for total body dose.
- The highest calculated dose for gamma air dose, child bone and liquid organ were summed for organ dose.
- The limits for Kr-85, I-129, Pu-239 and other alpha-emitting transuranic radionuclides with half-lives greater than one year were not exceeded.

The ODCM does not require total body doses to the population and average doses to individuals in the population from gaseous effluents to a distance of 50 miles from the site to be calculated.

Oyster Creek 2014 Annual Radioactive Effluent Release Report

A summary of gaseous and liquid radiation doses to most likely exposed MEMBER OF THE PUBLIC was as follows:

Effluent	Applicable Organ	Estimated Dose	Age Group	Location		% of Applicable Limit	Limit	Unit
				Distance (meters)	Direction (toward)			
Noble Gas	Gamma - Air Dose	1.62E-02	All	405	E	1.62E-01	10	mrad
Noble Gas	Beta - Air Dose	1.91E-02	All	406	E	9.55E-02	20	mrad
Noble Gas	Total Body (Gamma)	2.18E-03	All	972	ESE	4.36E-02	5	mrem
Noble Gas	Skin (Beta)	4.86E-03	All	972	ESE	3.24E-02	15	mrem
Iodine, Particulate, Carbon-14 & Tritium	Bone	4.78E-01	Child	937	SE	3.19E+00	15	mrem
Liquid	Total body	8.28E-07	All	South Route 9 Bridge		2.76E-05	3	mrem
Liquid	Organ	8.28E-07	All			8.28E-06	10	mrem
Direct Radiation	Total Body	5.09E+00	All	483	W	2.04E+01	25	mrem
Direct Radiation	Total Body	<LLD	All	937	SE	<LLD	25	mrem
40 CFR Part 190 Compliance								
Warehouse Worker								
Total Dose	Total Body	5.09E+00	All	483	W	2.04E+01	25	mrem
Total Dose	Bone	5.57E+00	All	483	W	2.23E+01	25	mrem
Total Dose	Thyroid	5.09E+00	All	483	W	6.79E+00	75	mrem
Nearest Resident								
Total Dose	Total Body	1.62E-02	All	937	SE	6.48E-02	25	mrem
Total Dose	Bone	4.94E-01	All	937	SE	1.98E+00	25	mrem
Total Dose	Thyroid	1.62E-02	All	937	SE	2.16E-02	75	mrem

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Appendix D Meteorological Data

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Table D – 1 Wind Speed by Direction Measured at 33 Feet for various Stability Classes for the Oyster Creek Generating Station, January – March, 2014

Oyster Creek Alpha							
Period of Record: January - March 2014							
Stability Class - Extremely Unstable - 150Ft-33Ft Delta-T (F)							
Winds Measured at 33 Feet							
Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	0	4	0	0	0	0	4
NNE	0	1	0	0	0	0	1
NE	0	4	11	0	0	0	15
ENE	0	8	7	2	0	0	17
E	0	2	0	0	0	0	2
ESE	0	8	0	0	0	0	8
SE	0	5	5	0	0	0	10
SSE	0	1	4	0	0	0	5
S	0	0	7	5	0	0	12
SSW	0	5	3	5	0	0	13
SW	0	4	6	0	0	0	10
WSW	0	6	9	4	0	0	19
W	0	8	22	8	0	0	38
WNW	0	10	36	20	0	0	66
NW	0	9	28	10	0	0	47
NNW	0	8	17	0	0	0	25
Variable	0	0	0	0	0	0	0
Total	0	83	155	54	0	0	292

Hours of calm in this stability class: 0
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 4

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Table D – 1 Wind Speed by Direction Measured at 33 Feet for various Stability Classes for the Oyster Creek Generating Station, January – March, 2014

Oyster Creek Alpha

Period of Record: January - March 2014
Stability Class - Moderately Unstable - 150Ft-33Ft Delta-T (F)
Winds Measured at 33 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	0	7	0	0	0	0	7
NNE	0	1	0	0	0	0	1
NE	0	3	2	0	0	0	5
ENE	0	3	5	0	0	0	8
E	0	4	0	0	0	0	4
ESE	0	1	0	0	0	0	1
SE	0	4	1	0	0	0	5
SSE	0	1	2	0	0	0	3
S	0	0	5	2	0	0	7
SSW	0	2	4	1	0	0	7
SW	0	2	0	0	0	0	2
WSW	0	4	6	0	0	0	10
W	0	4	5	5	0	0	14
WNW	0	10	4	3	0	0	17
NW	0	8	7	2	0	0	17
NNW	0	4	0	0	0	0	4
Variable	0	0	0	0	0	0	0
Total	0	58	41	13	0	0	112

Hours of calm in this stability class: 0
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 4

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Table D – 1 Wind Speed by Direction Measured at 33 Feet for various Stability Classes for the Oyster Creek Generating Station, January – March, 2014

Oyster Creek Alpha

Period of Record: January - March 2014
Stability Class - Slightly Unstable - 150Ft-33Ft Delta-T (F)
Winds Measured at 33 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	0	0	0	0	0	0	0
NNE	0	1	0	0	0	0	1
NE	0	3	0	0	0	0	3
ENE	0	2	0	0	0	0	2
E	0	1	0	0	0	0	1
ESE	0	2	0	0	0	0	2
SE	0	4	0	0	0	0	4
SSE	0	0	0	0	0	0	0
S	0	1	1	0	0	0	2
SSW	0	1	2	0	0	0	3
SW	0	2	0	0	0	0	2
WSW	0	2	0	0	0	0	2
W	0	1	3	1	0	0	5
WNW	0	2	6	2	0	0	10
NW	0	4	2	4	0	0	10
NNW	0	3	1	0	0	0	4
Variable	0	0	0	0	0	0	0
Total	0	29	15	7	0	0	51

Hours of calm in this stability class: 0
Hours of missing wind measurements in this stability class: 0
Hours of missing stability measurements in all stability classes: 4

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 1 Wind Speed by Direction Measured at 33 Feet for various Stability Classes for the Oyster Creek Generating Station, January – March, 2014

Oyster Creek Alpha

Period of Record: January - March 2014
Stability Class - Neutral - 150Ft-33Ft Delta-T (F)
Winds Measured at 33 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	4	21	17	0	0	0	42
NNE	3	32	33	2	0	0	70
NE	8	33	16	4	0	0	61
ENE	3	19	13	0	0	0	35
E	2	2	1	0	0	0	5
ESE	2	5	0	0	0	0	7
SE	1	5	3	0	0	0	9
SSE	1	7	1	0	0	0	9
S	1	7	6	0	0	0	14
SSW	1	6	12	15	0	0	34
SW	2	7	7	1	0	0	17
WSW	4	13	5	0	0	0	22
W	5	18	13	5	0	0	41
WNW	5	23	21	19	0	0	68
NW	4	31	25	5	0	0	65
NNW	5	33	40	0	0	0	78
Variable	0	0	0	0	0	0	0
Total	51	262	213	51	0	0	577

Hours of calm in this stability class: 0
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 4

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 1 Wind Speed by Direction Measured at 33 Feet for various Stability Classes for the Oyster Creek Generating Station, January – March, 2014

Oyster Creek Alpha

Period of Record: January - March 2014
Stability Class - Slightly Stable - 150Ft-33Ft Delta-T (F)
Winds Measured at 33 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	3	14	2	0	0	0	19
NNE	4	13	0	0	0	0	17
NE	3	27	2	1	0	0	33
ENE	8	10	5	0	0	0	23
E	5	2	0	0	0	0	7
ESE	2	1	0	0	0	0	3
SE	0	8	1	0	0	0	9
SSE	2	6	9	1	0	0	18
S	5	15	11	12	0	0	43
SSW	5	16	40	11	0	0	72
SW	10	34	6	0	0	0	50
WSW	10	28	3	0	0	0	41
W	9	34	3	1	0	0	47
WNW	13	49	27	1	0	0	90
NW	13	46	22	0	0	0	81
NNW	11	17	13	0	0	0	41
Variable	0	0	0	0	0	0	0
Total	103	320	144	27	0	0	594

Hours of calm in this stability class: 0
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 4

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 1 Wind Speed by Direction Measured at 33 Feet for various Stability Classes for the Oyster Creek Generating Station, January – March, 2014

Oyster Creek Alpha

Period of Record: January - March 2014
Stability Class - Moderately Stable - 150Ft-33Ft Delta-T (F)
Winds Measured at 33 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	8	1	0	0	0	0	9
NNE	2	1	0	0	0	0	3
NE	5	0	0	0	0	0	5
ENE	2	1	0	0	0	0	3
E	1	0	0	0	0	0	1
ESE	2	0	0	0	0	0	2
SE	2	3	0	0	0	0	5
SSE	0	1	0	0	0	0	1
S	2	1	0	0	0	0	3
SSW	2	6	0	0	0	0	8
SW	6	6	0	0	0	0	12
WSW	13	14	0	0	0	0	27
W	28	16	0	0	0	0	44
WNW	11	13	0	0	0	0	24
NW	7	15	0	0	0	0	22
NNW	7	5	0	0	0	0	12
Variable	0	0	0	0	0	0	0
Total	98	83	0	0	0	0	181

Hours of calm in this stability class: 1
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 4

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 1 Wind Speed by Direction Measured at 33 Feet for various Stability Classes for the Oyster Creek Generating Station, January – March, 2014

Oyster Creek Alpha							
Period of Record: January - March 2014							
Stability Class - Extremely Stable - 150Ft-33Ft Delta-T (F)							
Winds Measured at 33 Feet							
Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	4	0	0	0	0	0	4
NNE	4	0	0	0	0	0	4
NE	6	0	0	0	0	0	6
ENE	3	0	0	0	0	0	3
E	3	0	0	0	0	0	3
ESE	4	0	0	0	0	0	4
SE	3	2	0	0	0	0	5
SSE	4	0	0	0	0	0	4
S	5	0	0	0	0	0	5
SSW	9	0	0	0	0	0	9
SW	27	1	0	0	0	0	28
WSW	72	10	0	0	0	0	82
W	76	18	0	0	0	0	94
WNW	38	3	0	0	0	0	41
NW	31	15	0	0	0	0	46
NNW	2	0	0	0	0	0	2
Variable	0	0	0	0	0	0	0
Total	291	49	0	0	0	0	340

Hours of calm in this stability class: 8
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 4

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Table D – 2 Wind Speed by Direction Measured at 380 Feet for various Stability Classes for the Oyster Creek Generating Station, January – March, 2014

Oyster Creek Alpha

Period of Record: January - March 2014
 Stability Class - Extremely Unstable - 380Ft-33Ft Delta-T (F)
 Winds Measured at 380 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	0	0	0	0	0	0	0
NNE	0	0	0	0	0	0	0
NE	0	0	0	2	0	0	2
ENE	0	0	0	0	0	0	0
E	0	0	0	0	0	0	0
ESE	0	0	0	0	0	0	0
SE	0	0	0	0	0	0	0
SSE	0	0	0	0	0	0	0
S	0	0	0	0	0	0	0
SSW	0	0	0	0	0	0	0
SW	0	0	0	0	0	0	0
WSW	0	0	0	0	0	1	1
W	0	0	0	1	1	1	3
WNW	0	0	0	2	2	4	8
NW	0	0	1	2	0	3	6
NNW	0	0	0	0	1	0	1
Variable	0	0	0	0	0	0	0
Total	0	0	1	7	4	9	21

Hours of calm in this stability class: 0
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 4

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 2 Wind Speed by Direction Measured at 380 Feet for various Stability Classes for the Oyster Creek Generating Station, January – March, 2014

Oyster Creek Alpha

Period of Record: January - March 2014
Stability Class - Moderately Unstable - 380Ft-33Ft Delta-T (F)
Winds Measured at 380 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	0	0	1	0	0	0	1
NNE	0	0	0	0	0	0	0
NE	0	0	0	4	3	0	7
ENE	0	0	0	0	1	1	2
E	0	0	0	0	0	0	0
ESE	0	0	0	0	0	0	0
SE	0	0	1	0	0	0	1
SSE	0	0	0	1	0	0	1
S	0	0	0	1	0	0	1
SSW	0	0	0	0	2	0	2
SW	0	0	1	1	0	0	2
WSW	0	0	0	3	2	2	7
W	0	0	0	1	2	4	7
WNW	0	0	2	9	3	4	18
NW	0	0	1	10	4	5	20
NNW	0	0	2	4	3	0	9
Variable	0	0	0	0	0	0	0
Total	0	0	8	34	20	16	78

Hours of calm in this stability class: 0
Hours of missing wind measurements in this stability class: 0
Hours of missing stability measurements in all stability classes: 4

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 2 Wind Speed by Direction Measured at 380 Feet for various Stability Classes for the Oyster Creek Generating Station, January – March, 2014

Oyster Creek Alpha

Period of Record: January - March 2014
Stability Class - Slightly Unstable - 380Ft-33Ft Delta-T (F)
Winds Measured at 380 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	0	0	1	0	0	0	1
NNE	0	1	0	0	0	0	1
NE	0	2	2	3	1	0	8
ENE	0	2	6	0	1	0	9
E	0	0	0	0	0	0	0
ESE	0	1	1	0	0	0	2
SE	0	0	5	0	0	0	5
SSE	0	0	0	3	0	0	3
S	0	0	0	1	0	0	1
SSW	0	0	0	2	3	1	6
SW	0	2	1	1	0	0	4
WSW	0	0	4	5	1	0	10
W	0	1	2	10	4	4	21
WNW	0	1	7	7	12	5	32
NW	0	0	10	8	6	5	29
NNW	0	0	6	7	2	0	15
Variable	0	0	0	0	0	0	0
Total	0	10	45	47	30	15	147

Hours of calm in this stability class: 0
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 4

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 2 Wind Speed by Direction Measured at 380 Feet for various Stability Classes for the Oyster Creek Generating Station, January – March, 2014

Oyster Creek Alpha

Period of Record: January - March 2014
Stability Class - Neutral - 380Ft-33Ft Delta-T (F)
Winds Measured at 380 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	0	5	17	16	19	1	58
NNE	0	3	5	48	24	0	80
NE	0	8	21	35	12	5	81
ENE	0	1	7	27	24	1	60
E	0	6	4	5	1	0	16
ESE	0	7	2	0	1	0	10
SE	0	1	6	5	1	0	13
SSE	0	0	4	3	0	0	7
S	0	2	12	8	0	1	23
SSW	0	0	5	19	22	21	67
SW	0	1	13	8	8	1	31
WSW	0	3	11	14	8	0	36
W	1	3	17	20	11	13	65
WNW	0	4	10	28	26	34	102
NW	0	13	26	28	34	32	133
NNW	1	10	18	31	27	12	99
Variable	0	0	0	0	0	0	0
Total	2	67	178	295	218	121	881

Hours of calm in this stability class: 0
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 4

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 2 Wind Speed by Direction Measured at 380 Feet for various Stability Classes for the Oyster Creek Generating Station, January – March, 2014

Oyster Creek Alpha							
Period of Record: January - March 2014							
Stability Class - Slightly Stable - 380Ft-33Ft Delta-T (F)							
Winds Measured at 380 Feet							
Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	0	1	2	4	1	0	8
NNE	0	1	5	11	0	0	17
NE	0	1	9	5	0	7	22
ENE	1	0	4	10	6	2	23
E	0	2	2	0	0	0	4
ESE	1	3	0	0	0	0	4
SE	0	5	2	4	1	2	14
SSE	1	2	2	0	3	10	18
S	0	8	2	2	3	12	27
SSW	0	0	6	15	32	20	73
SW	0	0	7	9	23	4	43
WSW	0	0	7	17	14	3	41
W	0	1	7	23	13	0	44
WNW	0	4	4	40	38	3	89
NW	0	1	5	36	38	4	84
NNW	0	1	5	18	5	0	29
Variable	0	0	0	0	0	0	0
Total	3	30	69	194	177	67	540

Hours of calm in this stability class: 0
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 4

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 2 Wind Speed by Direction Measured at 380 Feet for various Stability Classes for the Oyster Creek Generating Station, January – March, 2014

Oyster Creek Alpha							
Period of Record: January - March 2014							
Stability Class - Moderately Stable - 380Ft-33Ft Delta-T (F)							
Winds Measured at 380 Feet							
Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	0	0	2	8	8	1	19
NNE	0	0	1	1	0	0	2
NE	1	2	1	2	0	0	6
ENE	0	2	2	5	0	0	9
E	1	1	4	0	0	0	6
ESE	2	1	3	2	0	0	8
SE	1	2	3	3	1	0	10
SSE	0	2	6	0	1	3	12
S	1	5	4	2	2	0	14
SSW	2	1	5	1	1	1	11
SW	0	4	2	6	5	5	22
WSW	0	1	4	9	2	5	21
W	0	1	3	16	8	3	31
WNW	0	0	7	21	23	2	53
NW	0	0	6	18	23	0	47
NNW	0	0	4	10	9	1	24
Variable	0	0	0	0	0	0	0
Total	8	22	57	104	83	21	295

Hours of calm in this stability class: 0
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 4

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 2 Wind Speed by Direction Measured at 380 Feet for various Stability Classes for the Oyster Creek Generating Station, January – March, 2014

Oyster Creek Alpha

Period of Record: January - March 2014
Stability Class - Extremely Stable - 380Ft-33Ft Delta-T (F)
Winds Measured at 380 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	2	1	4	1	4	3	15
NNE	1	2	6	2	0	0	11
NE	2	1	1	0	0	0	4
ENE	1	5	0	0	0	0	6
E	3	2	0	0	0	0	5
ESE	2	2	0	0	0	0	4
SE	2	5	3	1	0	0	11
SSE	0	6	4	2	4	1	17
S	1	4	4	5	2	1	17
SSW	0	2	3	4	1	0	10
SW	0	2	4	1	5	2	14
WSW	1	1	7	7	0	0	16
W	0	1	1	4	2	2	10
WNW	0	0	4	8	4	2	18
NW	3	4	4	4	5	0	20
NNW	2	2	3	5	2	0	14
Variable	0	0	0	0	0	0	0
Total	20	40	48	44	29	11	192

Hours of calm in this stability class: 2
Hours of missing wind measurements in this stability class: 0
Hours of missing stability measurements in all stability classes: 4

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 3 Wind Speed by Direction Measured at 33 Feet for various Stability Classes for the Oyster Creek Generating Station, April – June, 2014

Oyster Creek Alpha							
Period of Record: April - June 2014							
Stability Class - Extremely Unstable - 150Ft-33Ft Delta-T (F)							
Winds Measured at 33 Feet							
Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	0	7	4	0	0	0	11
NNE	0	3	1	0	0	0	4
NE	0	13	8	0	0	0	21
ENE	0	20	33	0	0	0	53
E	1	24	9	0	0	0	34
ESE	0	28	19	0	0	0	47
SE	0	20	42	0	0	0	62
SSE	0	3	39	1	0	0	43
S	0	4	41	13	0	0	58
SSW	0	5	3	6	2	0	16
SW	0	5	2	0	0	0	7
WSW	0	6	11	0	0	0	17
W	0	9	13	0	0	0	22
WNW	0	3	17	11	0	0	31
NW	0	14	50	9	0	0	73
NNW	0	5	18	0	0	0	23
Variable	0	0	0	0	0	0	0
Total	1	169	310	40	2	0	522

Hours of calm in this stability class: 0
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 1

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 3 Wind Speed by Direction Measured at 33 Feet for various Stability Classes for the Oyster Creek Generating Station, April – June, 2014

Oyster Creek Alpha

Period of Record: April - June 2014
Stability Class - Moderately Unstable - 150Ft-33Ft Delta-T (F)
Winds Measured at 33 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	0	1	1	0	0	0	2
NNE	1	6	0	0	0	0	7
NE	0	11	0	0	0	0	11
ENE	0	13	3	4	0	0	20
E	0	5	0	0	0	0	5
ESE	1	5	0	0	0	0	6
SE	0	14	2	0	0	0	16
SSE	0	5	3	1	0	0	9
S	0	6	3	2	0	0	11
SSW	0	2	1	2	0	0	5
SW	0	3	0	1	0	0	4
WSW	1	3	4	1	0	0	9
W	0	5	4	0	0	0	9
WNW	0	3	3	1	0	0	7
NW	0	4	7	2	0	0	13
NNW	0	11	1	0	0	0	12
Variable	0	0	0	0	0	0	0
Total	3	97	32	14	0	0	146

Hours of calm in this stability class: 0
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 1

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 3 Wind Speed by Direction Measured at 33 Feet for various Stability Classes for the Oyster Creek Generating Station, April – June, 2014

Oyster Creek Alpha							
Period of Record: April - June 2014							
Stability Class - Slightly Unstable - 150Ft-33Ft Delta-T (F)							
Winds Measured at 33 Feet							
Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	0	1	0	0	0	0	1
NNE	2	0	0	0	0	0	2
NE	0	3	0	0	0	0	3
ENE	0	2	2	1	0	0	5
E	1	2	0	0	0	0	3
ESE	0	1	1	0	0	0	2
SE	0	3	1	0	0	0	4
SSE	0	6	1	0	0	0	7
S	0	4	4	0	0	0	8
SSW	0	1	0	2	0	0	3
SW	0	1	0	0	0	0	1
WSW	0	0	1	0	0	0	1
W	0	0	1	0	0	0	1
WNW	1	3	0	1	0	0	5
NW	0	1	0	0	0	0	1
NNW	1	0	0	0	0	0	1
Variable	0	0	0	0	0	0	0
Total	5	28	11	4	0	0	48

Hours of calm in this stability class: 0
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 1

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 3 Wind Speed by Direction Measured at 33 Feet for various Stability Classes for the Oyster Creek Generating Station, April – June, 2014

Oyster Creek Alpha

Period of Record: April - June 2014
Stability Class - Neutral - 150Ft-33Ft Delta-T (F)
Winds Measured at 33 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	2	10	1	0	0	0	13
NNE	4	4	0	0	0	0	8
NE	5	48	12	0	0	0	65
ENE	7	38	33	18	0	0	96
E	2	22	5	3	0	0	32
ESE	5	9	4	2	0	0	20
SE	0	20	2	2	0	0	24
SSE	1	21	15	9	0	0	46
S	2	28	15	4	4	0	53
SSW	0	8	8	19	1	0	36
SW	0	7	0	0	0	0	7
WSW	1	3	4	0	0	0	8
W	3	8	5	0	0	0	16
WNW	3	10	10	1	0	0	24
NW	3	16	7	3	0	0	29
NNW	4	20	8	1	0	0	33
Variable	0	0	0	0	0	0	0
Total	42	272	129	62	5	0	510

Hours of calm in this stability class: 0
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 1

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 3 Wind Speed by Direction Measured at 33 Feet for various Stability Classes for the Oyster Creek Generating Station, April – June, 2014

Oyster Creek Alpha

Period of Record: April - June 2014
Stability Class - Slightly Stable - 150Ft-33Ft Delta-T (F)
Winds Measured at 33 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	7	3	0	0	0	0	10
NNE	7	2	0	0	0	0	9
NE	11	10	2	0	0	0	23
ENE	5	10	7	0	0	0	22
E	5	7	2	0	0	0	14
ESE	4	6	1	0	0	0	11
SE	9	10	1	0	0	0	20
SSE	14	16	4	3	0	0	37
S	19	41	6	1	0	0	67
SSW	12	26	20	9	0	0	67
SW	6	16	1	0	0	0	23
WSW	6	33	2	0	0	0	41
W	13	19	2	0	0	0	34
WNW	11	20	4	0	0	0	35
NW	16	22	8	0	0	0	46
NNW	5	7	2	0	0	0	14
Variable	1	0	0	0	0	0	1
Total	151	248	62	13	0	0	474

Hours of calm in this stability class: 0
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 1

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 3 Wind Speed by Direction Measured at 33 Feet for various Stability Classes for the Oyster Creek Generating Station, April – June, 2014

Oyster Creek Alpha

Period of Record: April - June 2014
Stability Class - Moderately Stable - 150Ft-33Ft Delta-T (F)
Winds Measured at 33 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	2	0	0	0	0	0	2
NNE	2	0	0	0	0	0	2
NE	2	1	0	0	0	0	3
ENE	3	0	0	0	0	0	3
E	2	1	0	0	0	0	3
ESE	0	0	0	0	0	0	0
SE	4	0	0	0	0	0	4
SSE	9	0	0	0	0	0	9
S	9	1	0	0	0	0	10
SSW	7	0	0	0	0	0	7
SW	12	5	0	0	0	0	17
WSW	7	17	0	0	0	0	24
W	14	6	0	0	0	0	20
WNW	10	25	0	0	0	0	35
NW	13	7	0	0	0	0	20
NNW	6	0	0	0	0	0	6
Variable	0	0	0	0	0	0	0
Total	102	63	0	0	0	0	165

Hours of calm in this stability class: 0
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 1

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 3 Wind Speed by Direction Measured at 33 Feet for various Stability Classes for the Oyster Creek Generating Station, April – June, 2014

Oyster Creek Alpha

Period of Record: April - June 2014
Stability Class - Extremely Stable - 150Ft-33Ft Delta-T (F)
Winds Measured at 33 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	2	0	0	0	0	0	2
NNE	0	1	0	0	0	0	1
NE	1	0	0	0	0	0	1
ENE	2	0	0	0	0	0	2
E	1	0	0	0	0	0	1
ESE	0	0	0	0	0	0	0
SE	0	0	0	0	0	0	0
SSE	1	0	0	0	0	0	1
S	3	0	0	0	0	0	3
SSW	11	0	0	0	0	0	11
SW	13	1	0	0	0	0	14
WSW	48	8	0	0	0	0	56
W	94	4	0	0	0	0	98
WNW	56	9	0	0	0	0	65
NW	34	6	0	0	0	0	40
NNW	15	1	0	0	0	0	16
Variable	0	0	0	0	0	0	0
Total	281	30	0	0	0	0	311

Hours of calm in this stability class: 7
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 1

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 4 Wind Speed by Direction Measured at 380 Feet for various Stability Classes for the Oyster Creek Generating Station, April – June, 2014

Oyster Creek Alpha

Period of Record: April - June 2014
Stability Class - Extremely Unstable - 380Ft-33Ft Delta-T (F)
Winds Measured at 380 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	0	0	0	0	0	0	0
NNE	0	0	0	0	0	0	0
NE	0	0	0	0	5	0	5
ENE	0	0	3	6	1	0	10
E	0	0	3	0	0	0	3
ESE	0	0	2	0	0	0	2
SE	0	0	2	1	0	0	3
SSE	0	0	0	2	0	0	2
S	0	0	0	13	4	0	17
SSW	0	0	0	0	0	0	0
SW	0	0	0	0	0	0	0
WSW	0	0	2	1	0	0	3
W	0	0	0	2	0	0	2
WNW	0	0	0	2	3	5	10
NW	0	0	2	15	3	8	28
NNW	0	0	0	5	0	0	5
Variable	0	0	0	0	0	0	0
Total	0	0	14	47	16	13	90

Hours of calm in this stability class: 0
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 1

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 4 Wind Speed by Direction Measured at 380 Feet for various Stability Classes for the Oyster Creek Generating Station, April – June, 2014

Oyster Creek Alpha

Period of Record: April - June 2014
Stability Class - Moderately Unstable - 380Ft-33Ft Delta-T (F)
Winds Measured at 380 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	0	0	0	4	0	0	4
NNE	0	0	1	0	1	0	2
NE	0	0	1	2	1	0	4
ENE	0	0	11	4	9	0	24
E	0	0	9	0	0	0	9
ESE	0	4	13	1	0	0	18
SE	0	0	16	4	0	0	20
SSE	0	0	4	6	0	0	10
S	0	0	2	11	3	0	16
SSW	0	0	1	3	3	4	11
SW	0	0	2	0	0	0	2
WSW	0	0	2	3	1	0	6
W	0	0	2	2	1	0	5
WNW	0	0	2	6	0	1	9
NW	0	0	5	18	5	3	31
NNW	0	0	2	6	1	0	9
Variable	0	0	0	0	0	0	0
Total	0	4	73	70	25	8	180

Hours of calm in this stability class: 0
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 1

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 4 Wind Speed by Direction Measured at 380 Feet for various Stability Classes for the Oyster Creek Generating Station, April – June, 2014

Oyster Creek Alpha

Period of Record: April - June 2014
Stability Class - Slightly Unstable - 380Ft-33Ft Delta-T (F)
Winds Measured at 380 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	0	1	3	1	0	0	5
NNE	0	2	0	1	0	0	3
NE	0	2	5	1	1	0	9
ENE	0	2	9	5	4	0	20
E	0	1	12	0	0	0	13
ESE	0	6	7	0	0	0	13
SE	1	2	11	0	0	0	14
SSE	0	3	12	5	0	0	20
S	0	0	6	8	1	0	15
SSW	0	0	3	2	1	4	10
SW	0	1	4	0	0	0	5
WSW	0	1	1	3	1	0	6
W	0	0	5	6	1	0	12
WNW	0	0	1	5	1	2	9
NW	0	0	5	12	1	2	20
NNW	0	0	3	4	2	0	9
Variable	0	0	0	0	0	0	0
Total	1	21	87	53	13	8	183

Hours of calm in this stability class: 0
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 1

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 4 Wind Speed by Direction Measured at 380 Feet for various Stability Classes for the Oyster Creek Generating Station, April – June, 2014

Oyster Creek Alpha

Period of Record: April - June 2014
Stability Class - Neutral - 380Ft-33Ft Delta-T (F)
Winds Measured at 380 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	1	3	11	7	0	0	22
NNE	1	6	8	1	0	0	16
NE	1	16	33	21	2	2	75
ENE	1	11	25	37	30	30	134
E	0	15	14	9	12	6	56
ESE	0	15	11	7	0	2	35
SE	2	11	19	4	1	0	37
SSE	1	10	41	9	6	9	76
S	0	5	37	29	3	7	81
SSW	1	4	24	25	7	32	93
SW	0	2	14	0	0	0	16
WSW	0	4	3	7	4	2	20
W	0	5	5	8	11	0	29
WNW	0	4	10	8	9	8	39
NW	0	5	17	9	14	4	49
NNW	1	7	10	11	9	5	43
Variable	0	0	0	0	0	0	0
Total	9	123	282	192	108	107	821

Hours of calm in this stability class: 0
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 1

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 4 Wind Speed by Direction Measured at 380 Feet for various Stability Classes for the Oyster Creek Generating Station, April – June, 2014

Oyster Creek Alpha

Period of Record: April - June 2014
Stability Class - Slightly Stable - 380Ft-33Ft Delta-T (F)
Winds Measured at 380 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	3	2	3	6	0	0	14
NNE	0	1	6	2	0	0	9
NE	0	6	4	4	1	0	15
ENE	2	2	4	7	2	0	17
E	0	3	10	4	0	1	18
ESE	0	2	5	4	1	2	14
SE	1	1	16	1	2	2	23
SSE	0	0	10	5	8	7	30
S	0	3	13	23	3	3	45
SSW	0	3	17	34	18	4	76
SW	0	4	10	11	12	0	37
WSW	0	1	1	10	14	2	28
W	0	1	3	16	8	1	29
WNW	0	3	6	12	17	1	39
NW	0	3	6	14	18	2	43
NNW	1	3	6	10	4	0	24
Variable	0	0	0	0	0	0	0
Total	7	38	120	163	108	25	461

Hours of calm in this stability class: 0
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 1

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 4 Wind Speed by Direction Measured at 380 Feet for various Stability Classes for the Oyster Creek Generating Station, April – June, 2014

Oyster Creek Alpha

Period of Record: April - June 2014
Stability Class - Moderately Stable - 380Ft-33Ft Delta-T (F)
Winds Measured at 380 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	0	2	4	9	8	0	23
NNE	0	0	0	1	0	0	1
NE	0	0	5	1	0	0	6
ENE	0	0	3	0	0	0	3
E	0	0	0	0	0	0	0
ESE	0	0	0	0	0	0	0
SE	1	1	1	0	0	0	3
SSE	1	0	1	0	0	0	2
S	0	2	2	5	4	0	13
SSW	0	2	4	9	2	0	17
SW	0	1	4	9	3	0	17
WSW	2	1	3	6	10	11	33
W	0	0	5	8	5	0	18
WNW	0	2	4	6	9	0	21
NW	0	0	4	11	26	0	41
NNW	1	1	1	7	17	1	28
Variable	0	0	0	0	0	0	0
Total	5	12	41	72	84	12	226

Hours of calm in this stability class: 0
Hours of missing wind measurements in this stability class: 0
Hours of missing stability measurements in all stability classes: 1

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 4 Wind Speed by Direction Measured at 380 Feet for various Stability Classes for the Oyster Creek Generating Station, April – June, 2014

Oyster Creek Alpha

Period of Record: April - June 2014
Stability Class - Extremely Stable - 380Ft-33Ft Delta-T (F)
Winds Measured at 380 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	0	1	7	11	1	0	20
NNE	0	3	6	6	3	0	18
NE	0	4	4	4	0	0	12
ENE	1	1	0	0	0	0	2
E	3	2	0	0	0	0	5
ESE	1	0	0	0	0	0	1
SE	3	3	3	0	0	0	9
SSE	1	1	1	0	0	0	3
S	0	2	3	0	0	0	5
SSW	0	1	8	10	0	0	19
SW	0	1	1	5	4	0	11
WSW	1	6	3	5	8	3	26
W	0	2	0	4	6	0	12
WNW	0	1	4	6	3	0	14
NW	0	4	11	8	4	2	29
NNW	0	1	5	14	15	1	36
Variable	0	0	0	0	0	0	0
Total	10	33	56	73	44	6	222

Hours of calm in this stability class: 0
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 1

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 5 Wind Speed by Direction Measured at 33 Feet for various Stability Classes for the Oyster Creek Generating Station, July – September, 2014

Oyster Creek Alpha							
Period of Record: July - September 2014							
Stability Class - Extremely Unstable - 150Ft-33Ft Delta-T (F)							
Winds Measured at 33 Feet							
Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	0	5	0	0	0	0	5
NNE	0	5	0	0	0	0	5
NE	0	16	2	0	0	0	18
ENE	0	37	8	0	0	0	45
E	0	19	5	0	0	0	24
ESE	0	19	2	0	0	0	21
SE	0	12	24	0	0	0	36
SSE	0	5	18	0	0	0	23
S	0	2	36	17	0	0	55
SSW	0	8	12	2	0	0	22
SW	0	11	19	0	0	0	30
WSW	0	3	1	0	0	0	4
W	0	4	9	0	0	0	13
WNW	0	5	4	0	0	0	9
NW	0	13	13	0	0	0	26
NNW	0	5	3	0	0	0	8
Variable	0	0	0	0	0	0	0
Total	0	169	156	19	0	0	344

Hours of calm in this stability class: 0
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 0

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 5 Wind Speed by Direction Measured at 33 Feet for various Stability Classes for the Oyster Creek Generating Station, July – September, 2014

Oyster Creek Alpha

Period of Record: July - September 2014
Stability Class - Moderately Unstable - 150Ft-33Ft Delta-T (F)
Winds Measured at 33 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	0	4	1	0	0	0	5
NNE	0	7	0	0	0	0	7
NE	1	6	4	0	0	0	11
ENE	0	12	3	0	0	0	15
E	0	6	0	0	0	0	6
ESE	0	7	0	0	0	0	7
SE	0	7	9	0	0	0	16
SSE	0	4	2	0	0	0	6
S	0	4	12	1	0	0	17
SSW	1	3	6	0	0	0	10
SW	0	7	4	0	0	0	11
WSW	0	12	0	0	0	0	12
W	0	3	3	0	0	0	6
WNW	0	9	2	0	0	0	11
NW	0	7	2	0	0	0	9
NNW	0	13	1	0	0	0	14
Variable	0	0	0	0	0	0	0
Total	2	111	49	1	0	0	163

Hours of calm in this stability class: 0
Hours of missing wind measurements in this stability class: 0
Hours of missing stability measurements in all stability classes: 0

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 5 Wind Speed by Direction Measured at 33 Feet for various Stability Classes for the Oyster Creek Generating Station, July – September, 2014

Oyster Creek Alpha

Period of Record: July - September 2014
Stability Class - Slightly Unstable - 150Ft-33Ft Delta-T (F)
Winds Measured at 33 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	0	3	0	0	0	0	3
NNE	2	1	0	0	0	0	3
NE	0	8	1	0	0	0	9
ENE	0	6	3	0	0	0	9
E	0	2	0	0	0	0	2
ESE	0	2	0	0	0	0	2
SE	2	4	3	0	0	0	9
SSE	0	2	1	0	0	0	3
S	1	0	11	0	0	0	12
SSW	0	2	3	0	0	0	5
SW	1	6	0	0	0	0	7
WSW	0	4	1	0	0	0	5
W	0	4	0	0	0	0	4
WNW	0	6	1	0	0	0	7
NW	0	3	1	0	0	0	4
NNW	0	5	0	0	0	0	5
Variable	0	0	0	0	0	0	0
Total	6	58	25	0	0	0	89

Hours of calm in this stability class: 0
Hours of missing wind measurements in this stability class: 0
Hours of missing stability measurements in all stability classes: 0

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 5 Wind Speed by Direction Measured at 33 Feet for various Stability Classes for the Oyster Creek Generating Station, July – September, 2014

Oyster Creek Alpha

Period of Record: July - September 2014
Stability Class - Neutral - 150Ft-33Ft Delta-T (F)
Winds Measured at 33 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	4	6	0	0	0	0	10
NNE	5	12	5	0	0	0	22
NE	4	27	20	0	0	0	51
ENE	4	28	23	0	0	0	55
E	0	9	7	1	0	0	17
ESE	5	11	0	0	0	0	16
SE	1	16	7	1	0	0	25
SSE	0	17	3	0	0	0	20
S	1	18	16	0	0	0	35
SSW	0	11	17	3	0	0	31
SW	2	13	4	0	0	0	19
WSW	6	11	4	0	0	0	21
W	2	9	3	0	0	0	14
WNW	2	12	2	0	0	0	16
NW	3	17	4	0	0	0	24
NNW	4	19	3	0	0	0	26
Variable	0	0	0	0	0	0	0
Total	43	236	118	5	0	0	402

Hours of calm in this stability class: 0
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 0

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 5 Wind Speed by Direction Measured at 33 Feet for various Stability Classes for the Oyster Creek Generating Station, July – September, 2014

Oyster Creek Alpha

Period of Record: July - September 2014
Stability Class - Slightly Stable - 150Ft-33Ft Delta-T (F)
Winds Measured at 33 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	8	11	3	0	0	0	22
NNE	3	4	0	0	0	0	7
NE	15	33	6	0	0	0	54
ENE	4	18	17	0	0	0	39
E	7	7	9	0	0	0	23
ESE	2	8	1	0	0	0	11
SE	4	19	5	3	0	0	31
SSE	13	8	1	0	0	0	22
S	15	59	0	0	0	0	74
SSW	19	51	18	0	0	0	88
SW	18	36	4	0	0	0	58
WSW	22	21	2	0	0	0	45
W	10	12	0	0	0	0	22
WNW	7	21	0	0	0	0	28
NW	9	19	2	0	0	0	30
NNW	10	25	2	0	0	0	37
Variable	0	0	0	0	0	0	0
Total	166	352	70	3	0	0	591

Hours of calm in this stability class: 2
Hours of missing wind measurements in this stability class: 0
Hours of missing stability measurements in all stability classes: 0

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 5 Wind Speed by Direction Measured at 33 Feet for various Stability Classes for the Oyster Creek Generating Station, July – September, 2014

Oyster Creek Alpha

Period of Record: July - September 2014
Stability Class - Moderately Stable - 150Ft-33Ft Delta-T (F)
Winds Measured at 33 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	5	0	0	0	0	0	5
NNE	2	0	0	0	0	0	2
NE	0	1	0	0	0	0	1
ENE	1	1	0	0	0	0	2
E	1	1	0	0	0	0	2
ESE	2	0	0	0	0	0	2
SE	2	0	0	0	0	0	2
SSE	6	0	0	0	0	0	6
S	6	3	0	0	0	0	9
SSW	10	3	0	0	0	0	13
SW	15	1	0	0	0	0	16
WSW	23	8	0	0	0	0	31
W	18	3	0	0	0	0	21
WNW	17	10	0	0	0	0	27
NW	13	3	0	0	0	0	16
NNW	7	6	0	0	0	0	13
Variable	0	0	0	0	0	0	0
Total	128	40	0	0	0	0	168

Hours of calm in this stability class: 1
Hours of missing wind measurements in this stability class: 0
Hours of missing stability measurements in all stability classes: 0

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 5 Wind Speed by Direction Measured at 33 Feet for various Stability Classes for the Oyster Creek Generating Station, July – September, 2014

Oyster Creek Alpha

Period of Record: July - September 2014
Stability Class - Extremely Stable - 150Ft-33Ft Delta-T (F)
Winds Measured at 33 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	1	0	0	0	0	0	1
NNE	1	2	0	0	0	0	3
NE	0	0	0	0	0	0	0
ENE	1	0	0	0	0	0	1
E	0	0	0	0	0	0	0
ESE	1	0	0	0	0	0	1
SE	1	1	0	0	0	0	2
SSE	3	0	0	0	0	0	3
S	11	0	0	0	0	0	11
SSW	7	0	0	0	0	0	7
SW	15	0	0	0	0	0	15
WSW	75	8	0	0	0	0	83
W	149	1	0	0	0	0	150
WNW	64	3	0	0	0	0	67
NW	62	5	0	0	0	0	67
NNW	22	3	0	0	0	0	25
Variable	1	0	0	0	0	0	1
Total	414	23	0	0	0	0	437

Hours of calm in this stability class: 11
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 0

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 6 Wind Speed by Direction Measured at 380 Feet for various Stability Classes for the Oyster Creek Generating Station, July – September, 2014

Oyster Creek Alpha

Period of Record: July - September 2014
Stability Class - Extremely Unstable - 380Ft-33Ft Delta-T (F)
Winds Measured at 380 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	0	0	0	0	0	0	0
NNE	0	0	0	0	0	0	0
NE	0	0	0	0	0	0	0
ENE	0	0	0	0	0	0	0
E	0	0	0	0	0	0	0
ESE	0	0	0	0	0	0	0
SE	0	0	0	0	0	0	0
SSE	0	0	0	0	0	0	0
S	0	0	0	6	7	0	13
SSW	0	0	0	0	0	0	0
SW	0	0	0	0	0	0	0
WSW	0	0	0	0	0	0	0
W	0	0	0	0	0	0	0
WNW	0	0	0	0	0	0	0
NW	0	0	0	0	0	0	0
NNW	0	0	0	0	0	0	0
Variable	0	0	0	0	0	0	0
Total	0	0	0	6	7	0	13

Hours of calm in this stability class: 0
Hours of missing wind measurements in this stability class: 0
Hours of missing stability measurements in all stability classes: 0

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 6 Wind Speed by Direction Measured at 380 Feet for various Stability Classes for the Oyster Creek Generating Station, July – September, 2014

Oyster Creek Alpha

Period of Record: July - September 2014
Stability Class - Moderately Unstable - 380Ft-33Ft Delta-T (F)
Winds Measured at 380 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	0	0	0	0	0	0	0
NNE	0	0	0	0	0	0	0
NE	0	0	2	5	0	0	7
ENE	0	0	10	0	0	0	10
E	0	0	5	0	0	0	5
ESE	0	0	2	0	0	0	2
SE	0	0	5	0	0	0	5
SSE	0	0	2	2	0	0	4
S	0	0	1	11	5	0	17
SSW	0	0	0	5	3	0	8
SW	0	0	3	11	2	0	16
WSW	0	0	0	4	0	0	4
W	0	0	0	0	1	0	1
WNW	0	0	0	0	0	0	0
NW	0	0	0	6	0	0	6
NNW	0	0	1	1	0	0	2
Variable	0	0	0	0	0	0	0
Total	0	0	31	45	11	0	87

Hours of calm in this stability class: 0
Hours of missing wind measurements in this stability class: 0
Hours of missing stability measurements in all stability classes: 0

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 6 Wind Speed by Direction Measured at 380 Feet for various Stability Classes for the Oyster Creek Generating Station, July – September, 2014

Oyster Creek Alpha

Period of Record: July - September 2014
Stability Class - Slightly Unstable - 380Ft-33Ft Delta-T (F)
Winds Measured at 380 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	0	0	4	1	0	0	5
NNE	0	2	4	0	0	0	6
NE	0	0	16	5	0	0	21
ENE	0	4	16	8	0	0	28
E	0	1	8	0	0	0	9
ESE	0	3	9	0	0	0	12
SE	0	1	15	4	0	0	20
SSE	0	0	9	4	0	0	13
S	0	0	2	9	3	0	14
SSW	0	2	7	6	2	0	17
SW	0	0	10	3	0	0	13
WSW	0	0	1	1	0	0	2
W	0	0	2	7	2	0	11
WNW	0	0	3	5	0	0	8
NW	0	0	13	7	1	0	21
NNW	0	2	4	2	0	0	8
Variable	0	0	0	0	0	0	0
Total	0	15	123	62	8	0	208

Hours of calm in this stability class: 0
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 0

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 6 Wind Speed by Direction Measured at 380 Feet for various Stability Classes for the Oyster Creek Generating Station, July – September, 2014

Oyster Creek Alpha							
Period of Record: July - September 2014							
Stability Class - Neutral - 380Ft-33Ft Delta-T (F)							
Winds Measured at 380 Feet							
Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	0	5	7	6	6	0	24
NNE	1	17	17	8	4	0	47
NE	1	17	34	33	22	7	114
ENE	0	9	22	32	22	15	100
E	0	9	18	2	14	8	51
ESE	0	11	18	4	0	1	34
SE	4	13	20	29	1	3	70
SSE	0	4	20	9	1	0	34
S	0	5	21	30	4	0	60
SSW	0	2	15	35	26	2	80
SW	1	11	9	20	1	0	42
WSW	0	13	16	8	4	0	41
W	0	9	11	10	2	0	32
WNW	0	4	22	11	0	0	37
NW	1	9	10	9	4	0	33
NNW	1	7	18	17	0	2	45
Variable	0	0	0	0	0	0	0
Total	9	145	278	263	111	38	844

Hours of calm in this stability class: 0
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 0

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 6 Wind Speed by Direction Measured at 380 Feet for various Stability Classes for the Oyster Creek Generating Station, July – September, 2014

Oyster Creek Alpha

Period of Record: July - September 2014
Stability Class - Slightly Stable - 380Ft-33Ft Delta-T (F)
Winds Measured at 380 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	0	1	4	9	2	0	16
NNE	0	3	8	2	0	0	13
NE	2	6	3	1	1	0	13
ENE	1	4	4	11	2	0	22
E	0	3	5	4	1	0	13
ESE	0	3	11	1	0	1	16
SE	0	4	8	7	0	0	19
SSE	0	4	12	5	0	1	22
S	1	8	8	19	1	0	37
SSW	2	2	21	68	10	0	103
SW	0	2	16	41	18	2	79
WSW	1	4	9	17	10	0	41
W	1	1	9	6	2	0	19
WNW	0	2	4	4	11	2	23
NW	1	2	3	12	10	3	31
NNW	1	1	3	14	9	0	28
Variable	0	0	0	0	0	0	0
Total	10	50	128	221	77	9	495

Hours of calm in this stability class: 0
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 0

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 6 Wind Speed by Direction Measured at 380 Feet for various Stability Classes for the Oyster Creek Generating Station, July – September, 2014

Oyster Creek Alpha

Period of Record: July - September 2014
Stability Class - Moderately Stable - 380Ft-33Ft Delta-T (F)
Winds Measured at 380 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	0	0	3	10	7	0	20
NNE	1	1	4	5	0	0	11
NE	0	1	4	1	0	0	6
ENE	2	3	5	0	1	0	11
E	3	2	0	0	0	0	5
ESE	0	1	1	2	0	0	4
SE	1	4	3	1	0	0	9
SSE	1	3	5	0	0	0	9
S	1	3	8	7	1	0	20
SSW	0	2	7	10	0	0	19
SW	0	5	6	10	4	0	25
WSW	1	5	8	15	11	0	40
W	1	4	3	8	5	0	21
WNW	0	0	1	5	6	1	13
NW	2	2	5	11	6	0	26
NNW	0	2	4	5	17	0	28
Variable	0	0	0	0	0	0	0
Total	13	38	67	90	58	1	267

Hours of calm in this stability class: 2
Hours of missing wind measurements in this stability class: 0
Hours of missing stability measurements in all stability classes: 0

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 6 Wind Speed by Direction Measured at 380 Feet for various Stability Classes for the Oyster Creek Generating Station, July – September, 2014

Oyster Creek Alpha

Period of Record: July - September 2014
Stability Class - Extremely Stable - 380Ft-33Ft Delta-T (F)
Winds Measured at 380 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	1	1	5	17	9	0	33
NNE	0	9	2	1	1	0	13
NE	1	5	7	6	0	0	19
ENE	1	4	3	2	0	0	10
E	3	4	0	2	0	0	9
ESE	0	3	0	0	0	0	3
SE	0	1	3	0	0	0	4
SSE	1	5	8	0	0	0	14
S	0	4	6	2	0	0	12
SSW	3	4	6	5	0	0	18
SW	0	2	9	8	3	0	22
WSW	0	3	10	14	6	3	36
W	0	5	9	7	3	1	25
WNW	2	1	4	9	1	0	17
NW	1	5	7	13	1	0	27
NNW	1	0	4	18	6	0	29
Variable	0	0	0	0	0	0	0
Total	14	56	83	104	30	4	291

Hours of calm in this stability class: 1
Hours of missing wind measurements in this stability class: 0
Hours of missing stability measurements in all stability classes: 0

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 7 Wind Speed by Direction Measured at 33 Feet for various Stability Classes for the Oyster Creek Generating Station, October – December, 2014

Oyster Creek Alpha

Period of Record: October – December 2014
Stability Class - Extremely Unstable - 150Ft-33Ft Delta-T (F)
Winds Measured at 33 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	0	0	0	0	0	0	0
NNE	0	0	0	0	0	0	0
NE	0	2	0	0	0	0	2
ENE	0	4	0	0	0	0	4
E	0	1	0	0	0	0	1
ESE	0	1	0	0	0	0	1
SE	0	1	1	0	0	0	2
SSE	0	0	3	1	0	0	4
S	0	0	3	4	0	0	7
SSW	0	1	7	0	0	0	8
SW	0	4	4	0	0	0	8
WSW	0	9	12	0	0	0	21
W	0	9	11	2	0	0	22
WNW	0	3	26	3	0	0	32
NW	0	4	33	5	0	0	42
NNW	0	4	3	0	0	0	7
Variable	0	0	0	0	0	0	0
Total	0	43	103	15	0	0	161

Hours of calm in this stability class: 0
Hours of missing wind measurements in this stability class: 0
Hours of missing stability measurements in all stability classes: 5

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 7 Wind Speed by Direction Measured at 33 Feet for various Stability Classes for the Oyster Creek Generating Station, October – December, 2014

Oyster Creek Alpha

Period of Record: October – December 2014
 Stability Class - Moderately Unstable - 150Ft-33Ft Delta-T (F)
 Winds Measured at 33 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	0	1	4	0	0	0	5
NNE	0	2	0	0	0	0	2
NE	0	3	0	0	0	0	3
ENE	0	0	0	0	0	0	0
E	0	1	0	0	0	0	1
ESE	0	3	0	0	0	0	3
SE	0	8	1	0	0	0	9
SSE	0	1	3	0	0	0	4
S	0	1	6	1	0	0	8
SSW	0	3	2	0	0	0	5
SW	0	3	3	0	0	0	6
WSW	0	2	2	0	0	0	4
W	0	7	4	0	0	0	11
WNW	0	7	14	0	0	0	21
NW	0	4	11	1	0	0	16
NNW	0	4	1	0	0	0	5
Variable	0	0	0	0	0	0	0
Total	0	50	51	2	0	0	103

Hours of calm in this stability class: 0
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 5

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 7 Wind Speed by Direction Measured at 33 Feet for various Stability Classes for the Oyster Creek Generating Station, October – December, 2014

Oyster Creek Alpha

Period of Record: October – December 2014
Stability Class - Slightly Unstable - 150Ft-33Ft Delta-T (F)
Winds Measured at 33 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	0	4	4	0	0	0	8
NNE	0	0	1	0	0	0	1
NE	0	2	0	0	0	0	2
ENE	0	1	0	0	0	0	1
E	0	0	0	0	0	0	0
ESE	0	0	0	0	0	0	0
SE	0	0	0	0	0	0	0
SSE	0	0	4	0	0	0	4
S	0	0	0	0	0	0	0
SSW	0	0	2	0	0	0	2
SW	0	1	0	0	0	0	1
WSW	1	3	3	0	0	0	7
W	0	2	2	0	0	0	4
WNW	0	7	6	2	0	0	15
NW	0	1	5	1	0	0	7
NNW	0	2	0	1	0	0	3
Variable	0	0	0	0	0	0	0
Total	1	23	27	4	0	0	55

Hours of calm in this stability class: 0
Hours of missing wind measurements in this stability class: 0
Hours of missing stability measurements in all stability classes: 5

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 7 Wind Speed by Direction Measured at 33 Feet for various Stability Classes for the Oyster Creek Generating Station, October – December, 2014

Oyster Creek Alpha

Period of Record: October – December 2014
Stability Class - Neutral - 150Ft-33Ft Delta-T (F)
Winds Measured at 33 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	3	26	30	2	0	0	61
NNE	3	26	26	2	0	0	57
NE	3	16	19	0	0	0	38
ENE	1	5	6	0	0	0	12
E	0	9	1	0	0	0	10
ESE	1	2	0	0	0	0	3
SE	2	10	3	0	0	0	15
SSE	3	12	19	3	0	0	37
S	3	5	13	5	0	0	26
SSW	4	7	9	5	0	0	25
SW	2	18	4	0	0	0	24
WSW	2	28	9	0	0	0	39
W	4	20	11	0	0	0	35
WNW	4	31	32	3	0	0	70
NW	7	44	48	3	0	0	102
NNW	7	13	15	0	0	0	35
Variable	0	0	0	0	0	0	0
Total	49	272	245	23	0	0	589

Hours of calm in this stability class: 1
Hours of missing wind measurements in this stability class: 0
Hours of missing stability measurements in all stability classes: 5

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 7 Wind Speed by Direction Measured at 33 Feet for various Stability Classes for the Oyster Creek Generating Station, October – December, 2014

Oyster Creek Alpha							
Period of Record: October – December 2014							
Stability Class - Slightly Stable - 150Ft-33Ft Delta-T (F)							
Winds Measured at 33 Feet							
Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	15	21	10	0	0	0	46
NNE	6	8	0	0	0	0	14
NE	13	20	2	0	0	0	35
ENE	7	7	1	0	0	0	15
E	1	1	0	0	0	0	2
ESE	1	2	0	0	0	0	3
SE	5	10	3	0	0	0	18
SSE	7	24	11	0	0	0	42
S	0	15	19	0	0	0	34
SSW	2	48	15	2	0	0	67
SW	5	41	15	0	0	0	61
WSW	18	46	1	0	0	0	65
W	17	49	5	0	0	0	71
WNW	15	84	20	0	0	0	119
NW	20	35	22	0	0	0	77
NNW	16	32	16	1	0	0	65
Variable	0	0	0	0	0	0	0
Total	148	443	140	3	0	0	734

Hours of calm in this stability class: 2
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 5

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 7 Wind Speed by Direction Measured at 33 Feet for various Stability Classes for the Oyster Creek Generating Station, October – December, 2014

Oyster Creek Alpha

Period of Record: October – December 2014
Stability Class - Moderately Stable - 150Ft-33Ft Delta-T (F)
Winds Measured at 33 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	12	1	0	0	0	0	13
NNE	2	1	0	0	0	0	3
NE	3	0	0	0	0	0	3
ENE	1	0	0	0	0	0	1
E	2	0	0	0	0	0	2
ESE	0	0	0	0	0	0	0
SE	3	0	0	0	0	0	3
SSE	1	0	0	0	0	0	1
S	4	1	0	0	0	0	5
SSW	4	10	0	0	0	0	14
SW	5	14	0	0	0	0	19
WSW	17	12	0	0	0	0	29
W	20	26	0	0	0	0	46
WNW	18	13	0	0	0	0	31
NW	17	6	0	0	0	0	23
NNW	9	2	0	0	0	0	11
Variable	0	0	0	0	0	0	0
Total	118	86	0	0	0	0	204

Hours of calm in this stability class: 3
Hours of missing wind measurements in this stability class: 0
Hours of missing stability measurements in all stability classes: 5

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 7 Wind Speed by Direction Measured at 33 Feet for various Stability Classes for the Oyster Creek Generating Station, October – December, 2014

Oyster Creek Alpha							
Period of Record: October – December 2014							
Stability Class - Extremely Stable - 150Ft-33Ft Delta-T (F)							
Winds Measured at 33 Feet							
Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	3	0	0	0	0	0	3
NNE	1	0	0	0	0	0	1
NE	1	0	0	0	0	0	1
ENE	1	0	0	0	0	0	1
E	2	0	0	0	0	0	2
ESE	0	0	0	0	0	0	0
SE	0	0	0	0	0	0	0
SSE	4	0	0	0	0	0	4
S	7	1	0	0	0	0	8
SSW	5	1	0	0	0	0	6
SW	16	3	0	0	0	0	19
WSW	64	4	0	0	0	0	68
W	133	15	0	0	0	0	148
WNW	37	8	0	0	0	0	45
NW	21	5	0	0	0	0	26
NNW	7	3	0	0	0	0	10
Variable	0	0	0	0	0	0	0
Total	302	40	0	0	0	0	342

Hours of calm in this stability class: 9
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 5

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 8 Wind Speed by Direction Measured at 380 Feet for various Stability Classes for the Oyster Creek Generating Station, October – December, 2014

Oyster Creek Alpha

Period of Record: October – December 2014
 Stability Class - Extremely Unstable - 380Ft-33Ft Delta-T (F)
 Winds Measured at 380 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	0	0	0	0	0	0	0
NNE	0	0	0	0	0	0	0
NE	0	0	0	0	0	0	0
ENE	0	0	0	0	0	0	0
E	0	0	0	0	0	0	0
ESE	0	0	0	0	0	0	0
SE	0	0	0	0	0	0	0
SSE	0	0	0	0	0	0	0
S	0	0	0	0	0	0	0
SSW	0	0	0	0	0	0	0
SW	0	0	0	0	0	0	0
WSW	0	0	0	0	0	0	0
W	0	0	0	0	0	0	0
WNW	0	0	0	0	0	0	0
NW	0	0	0	0	1	0	1
NNW	0	0	0	0	0	0	0
Variable	0	0	0	0	0	0	0
Total	0	0	0	0	1	0	1

Hours of calm in this stability class: 0
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 5

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 8 Wind Speed by Direction Measured at 380 Feet for various Stability Classes for the Oyster Creek Generating Station, October – December, 2014

Oyster Creek Alpha

Period of Record: October – December 2014
 Stability Class - Moderately Unstable - 380Ft-33Ft Delta-T (F)
 Winds Measured at 380 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	0	0	0	0	0	0	0
NNE	0	0	0	0	0	0	0
NE	0	0	0	0	0	0	0
ENE	0	0	1	0	0	0	1
E	0	0	0	0	0	0	0
ESE	0	0	0	0	0	0	0
SE	0	0	0	0	0	0	0
SSE	0	0	0	0	0	0	0
S	0	0	0	0	0	0	0
SSW	0	0	0	2	0	0	2
SW	0	0	0	0	0	0	0
WSW	0	0	0	2	0	0	2
W	0	0	1	4	0	1	6
WNW	0	0	0	2	3	2	7
NW	0	0	1	6	7	2	16
NNW	0	0	0	0	0	0	0
Variable	0	0	0	0	0	0	0
Total	0	0	3	16	10	5	34

Hours of calm in this stability class: 0
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 5

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 8 Wind Speed by Direction Measured at 380 Feet for various Stability Classes for the Oyster Creek Generating Station, October – December, 2014

Oyster Creek Alpha

Period of Record: October – December 2014
Stability Class - Slightly Unstable - 380Ft-33Ft Delta-T (F)
Winds Measured at 380 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	0	0	0	0	0	0	0
NNE	0	0	0	0	0	0	0
NE	0	1	3	0	0	0	4
ENE	0	1	1	0	0	0	2
E	0	0	2	0	0	0	2
ESE	0	0	0	0	0	0	0
SE	0	0	0	1	0	0	1
SSE	0	0	0	2	1	0	3
S	0	0	0	4	5	0	9
SSW	0	0	1	2	0	0	3
SW	0	0	3	4	0	0	7
WSW	0	0	5	10	0	0	15
W	0	0	4	9	1	1	15
WNW	0	0	4	10	15	2	31
NW	0	0	1	9	10	3	23
NNW	0	0	4	0	2	0	6
Variable	0	0	0	0	0	0	0
Total	0	2	28	51	34	6	121

Hours of calm in this stability class: 0
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 5

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 8 Wind Speed by Direction Measured at 380 Feet for various Stability Classes for the Oyster Creek Generating Station, October – December, 2014

Oyster Creek Alpha

Period of Record: October – December 2014
Stability Class - Neutral - 380Ft-33Ft Delta-T (F)
Winds Measured at 380 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	0	5	26	11	31	16	89
NNE	1	6	23	15	18	10	73
NE	1	5	6	14	16	3	45
ENE	1	2	1	1	3	3	11
E	0	4	4	3	0	0	11
ESE	1	2	3	2	0	0	8
SE	0	4	11	2	2	0	19
SSE	0	2	6	15	21	3	47
S	1	1	2	8	5	2	19
SSW	1	7	7	12	11	6	44
SW	0	4	17	19	6	1	47
WSW	1	4	9	25	7	0	46
W	0	4	25	23	14	2	68
WNW	0	4	19	29	43	16	111
NW	0	5	15	48	60	9	137
NNW	0	3	21	2	12	14	52
Variable	0	0	0	0	0	0	0
Total	7	62	195	229	249	85	827

Hours of calm in this stability class: 0
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 5

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 8 Wind Speed by Direction Measured at 380 Feet for various Stability Classes for the Oyster Creek Generating Station, October – December, 2014

Oyster Creek Alpha							
Period of Record: October – December 2014							
Stability Class - Slightly Stable - 380Ft-33Ft Delta-T (F)							
Winds Measured at 380 Feet							
Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	1	3	22	19	1	0	46
NNE	1	8	10	13	0	0	32
NE	0	4	13	16	2	1	36
ENE	1	4	2	10	6	0	23
E	0	3	4	1	2	0	10
ESE	1	2	1	0	1	0	5
SE	0	1	5	8	0	2	16
SSE	0	1	4	12	16	3	36
S	2	2	9	5	19	1	38
SSW	2	1	4	25	38	7	77
SW	0	1	4	16	38	10	69
WSW	2	1	5	17	19	1	45
W	0	5	12	28	22	1	68
WNW	0	1	7	38	55	6	107
NW	1	0	6	32	39	3	81
NNW	0	4	8	14	5	0	31
Variable	0	0	0	0	0	0	0
Total	11	41	116	254	263	35	720

Hours of calm in this stability class: 1
Hours of missing wind measurements in this stability class: 0
Hours of missing stability measurements in all stability classes: 5

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 8 Wind Speed by Direction Measured at 380 Feet for various Stability Classes for the Oyster Creek Generating Station, October – December, 2014

Oyster Creek Alpha

Period of Record: October – December 2014
Stability Class - Moderately Stable - 380Ft-33Ft Delta-T (F)
Winds Measured at 380 Feet

Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	1	1	6	19	6	0	33
NNE	0	1	2	5	0	0	8
NE	0	3	5	3	0	0	11
ENE	0	0	3	1	0	0	4
E	0	1	3	1	0	0	5
ESE	0	1	1	0	0	0	2
SE	1	0	4	4	0	0	9
SSE	0	0	3	1	1	1	6
S	0	0	1	2	1	0	4
SSW	0	2	0	5	3	0	10
SW	0	2	0	2	20	1	25
WSW	0	2	6	5	10	2	25
W	0	1	3	12	16	3	35
WNW	0	0	5	14	31	6	56
NW	0	2	6	12	19	3	42
NNW	0	0	4	7	1	0	12
Variable	0	0	0	0	0	0	0
Total	2	16	52	93	108	16	287

Hours of calm in this stability class: 0
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 5

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 8 Wind Speed by Direction Measured at 380 Feet for various Stability Classes for the Oyster Creek Generating Station, October – December, 2014

Oyster Creek Alpha							
Period of Record: October – December 2014							
Stability Class - Extremely Stable - 380Ft-33Ft Delta-T (F)							
Winds Measured at 380 Feet							
Wind Direction	Wind Speed (in mph)						Total
	1-3	4-7	8-12	13-18	19-24	> 24	
N	0	8	1	11	1	0	21
NNE	1	1	1	0	3	0	6
NE	1	2	0	0	0	0	3
ENE	0	0	0	0	0	0	0
E	0	0	0	0	0	0	0
ESE	2	1	0	0	0	0	3
SE	0	5	6	10	2	0	23
SSE	1	4	3	2	0	0	10
S	1	2	2	1	1	0	7
SSW	0	2	4	3	4	0	13
SW	0	0	3	8	4	1	16
WSW	0	1	2	5	0	1	9
W	0	1	1	6	4	1	13
WNW	1	0	3	10	7	5	26
NW	2	0	1	15	18	3	39
NNW	0	2	7	10	4	0	23
Variable	0	0	0	0	0	0	0
Total	9	29	34	81	48	11	212

Hours of calm in this stability class: 0
 Hours of missing wind measurements in this stability class: 0
 Hours of missing stability measurements in all stability classes: 5

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 9 Wind Speed by Direction Measured at 33 Feet for various Stability Classes for the Oyster Creek Generating Station, January – December, 2014

Oyster Creek Alpha

Period of Record: January - December 2014
Stability Class - All Stabilities - 150Ft-33Ft Delta-T (F)
Winds Measured at 33 Feet

Wind Direction Sector	Wind Speed (in m/s)											Total
	<0.50	0.5-1	1.1-1.5	1.6-2	2.1-3	3.1-4	4.1-5	5.1-6	6.1-8	8.1-10	>10.00	
N	3	45	34	39	91	44	53	4	0	0	0	313
NNE	3	20	29	40	77	49	30	9	0	0	0	257
NE	2	28	49	85	162	104	46	4	3	0	0	483
ENE	1	23	24	36	148	140	80	22	18	0	0	492
E	1	13	20	25	76	48	12	10	2	0	0	207
ESE	2	9	18	24	76	48	3	3	0	0	0	183
SE	1	13	22	29	127	108	36	6	3	0	0	345
SSE	5	26	32	31	87	93	65	21	11	0	0	371
S	2	38	46	64	133	94	99	83	34	4	0	597
SSW	1	40	55	44	146	117	63	64	60	4	0	594
SW	4	67	76	65	160	88	21	5	2	0	0	488
WSW	5	142	228	139	157	83	31	6	2	0	0	793
W	0	234	331	147	160	93	44	25	16	0	0	1050
WNW	3	135	163	120	234	144	104	65	48	0	0	1016
NW	4	88	163	140	180	203	116	49	36	0	0	979
NNW	4	46	80	95	139	83	69	18	1	0	0	535
Tot	41	967	1370	1123	2153	1539	872	394	236	8	0	8703
Hours of Calm					45							
Hours of Variable Direction					2							
Hours of Valid Data					8750							
Hours of Missing Data					10							
Hours in Period					8760							

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 9 Wind Speed by Direction Measured at 33 Feet for various Stability Classes for the Oyster Creek Generating Station, January – December, 2014

Oyster Creek Alpha

Period of Record: January - December 2014
Stability Class - Extremely Unstable - 150Ft-33Ft Delta-T (F)
Winds Measured at 33 Feet

Wind Direction Sector	Wind Speed (in m/s)											Total
	<0.50	0.5-1	1.1-1.5	1.6-2	2.1-3	3.1-4	4.1-5	5.1-6	6.1-8	8.1-10	>10.00	
N	0	0	0	0	13	6	1	0	0	0	0	20
NNE	0	0	0	1	8	0	1	0	0	0	0	10
NE	0	0	0	4	21	19	11	1	0	0	0	56
ENE	0	0	0	2	41	50	21	5	0	0	0	119
E	0	0	1	2	31	27	0	0	0	0	0	61
ESE	0	0	0	1	39	36	1	0	0	0	0	77
SE	0	0	0	0	23	67	20	0	0	0	0	110
SSE	0	0	0	0	4	38	26	7	0	0	0	75
S	0	0	0	2	1	18	44	46	21	0	0	132
SSW	0	0	0	0	13	17	8	10	8	3	0	59
SW	0	0	0	0	16	24	12	3	0	0	0	55
WSW	0	0	0	0	14	23	19	3	2	0	0	61
W	0	0	0	1	16	42	20	8	8	0	0	95
WNW	0	0	0	1	8	43	35	27	24	0	0	138
NW	0	0	0	0	26	71	49	21	21	0	0	188
NNW	0	0	0	0	15	29	15	4	0	0	0	63
Tot	0	0	1	14	289	510	283	135	84	3	0	1319
Hours of Calm					0							
Hours of Variable Direction					0							
Hours of Valid Data					1319							
Hours of Missing Data					10							
Hours in Period					8760							

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 9 Wind Speed by Direction Measured at 33 Feet for various Stability Classes for the Oyster Creek Generating Station, January – December, 2014

Oyster Creek Alpha

Period of Record: January - December 2014
Stability Class - Moderately Unstable - 150Ft-33Ft Delta-T (F)
Winds Measured at 33 Feet

Wind Direction Sector	Wind Speed (in m/s)											Total
	<0.50	0.5-1	1.1-1.5	1.6-2	2.1-3	3.1-4	4.1-5	5.1-6	6.1-8	8.1-10	>10.00	
N	0	0	0	4	9	2	4	0	0	0	0	19
NNE	0	0	1	4	12	0	0	0	0	0	0	17
NE	0	0	1	4	16	7	2	0	0	0	0	30
ENE	0	0	0	3	19	9	7	1	4	0	0	43
E	0	0	0	4	9	3	0	0	0	0	0	16
ESE	0	0	1	3	12	1	0	0	0	0	0	17
SE	0	0	0	1	27	11	6	1	0	0	0	46
SSE	0	0	0	0	6	7	7	1	1	0	0	22
S	0	0	0	1	6	12	13	10	1	0	0	43
SSW	0	0	0	3	4	8	5	4	3	0	0	27
SW	0	0	0	3	9	9	1	0	1	0	0	23
WSW	0	0	1	4	10	16	2	2	0	0	0	35
W	0	0	0	6	9	10	9	1	5	0	0	40
WNW	0	0	0	2	21	11	15	4	3	0	0	56
NW	0	0	0	3	18	11	14	7	2	0	0	55
NNW	0	0	0	4	27	2	2	0	0	0	0	35
Tot	0	0	4	49	214	119	87	31	20	0	0	524
Hours of Calm												0
Hours of Variable Direction												0
Hours of Valid Data												524
Hours of Missing Data												10
Hours in Period												8760

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 9 Wind Speed by Direction Measured at 33 Feet for various Stability Classes for the Oyster Creek Generating Station, January – December, 2014

Oyster Creek Alpha												
Period of Record: January - December 2014												
Stability Class - Slightly Unstable - 150Ft-33Ft Delta-T (F)												
Winds Measured at 33 Feet												
Wind Direction Sector	Wind Speed (in m/s)											Total
	<0.50	0.5-1	1.1-1.5	1.6-2	2.1-3	3.1-4	4.1-5	5.1-6	6.1-8	8.1-10	>10.00	
N	0	0	0	2	4	2	4	0	0	0	0	12
NNE	0	0	3	2	1	0	1	0	0	0	0	7
NE	0	0	0	8	8	0	1	0	0	0	0	17
ENE	0	0	0	2	9	0	4	1	1	0	0	17
E	0	0	1	1	4	0	0	0	0	0	0	6
ESE	0	0	0	1	3	2	0	0	0	0	0	6
SE	0	0	1	1	10	3	2	0	0	0	0	17
SSE	0	0	0	0	3	7	2	2	0	0	0	14
S	0	0	1	1	3	8	7	2	0	0	0	22
SSW	0	0	0	1	2	2	3	4	1	0	0	13
SW	0	0	1	3	6	1	0	0	0	0	0	11
WSW	0	0	1	4	4	6	0	0	0	0	0	15
W	0	0	0	1	4	3	2	3	1	0	0	14
WNW	0	0	1	6	10	7	5	6	2	0	0	37
NW	0	0	0	2	5	7	3	1	4	0	0	22
NNW	0	0	1	2	7	1	1	0	1	0	0	13
Tot	0	0	10	37	83	49	35	19	10	0	0	243
Hours of Calm												0
Hours of Variable Direction												0
Hours of Valid Data												243
Hours of Missing Data												10
Hours in Period												8760

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 9 Wind Speed by Direction Measured at 33 Feet for various Stability Classes for the Oyster Creek Generating Station, January – December, 2014

Oyster Creek Alpha												
Period of Record: January - December 2014												
Stability Class – Neutral - 150Ft-33Ft Delta-T (F)												
Winds Measured at 33 Feet												
Wind Direction Sector	Wind Speed (in m/s)											Total
	<0.50	0.5-1	1.1-1.5	1.6-2	2.1-3	3.1-4	4.1-5	5.1-6	6.1-8	8.1-10	>10.00	
N	1	5	7	12	39	26	32	4	0	0	0	126
NNE	2	0	12	21	41	44	28	9	0	0	0	157
NE	0	2	18	28	76	57	30	1	3	0	0	215
ENE	0	5	7	16	49	61	34	13	13	0	0	198
E	0	1	2	12	22	13	8	4	2	0	0	64
ESE	0	3	9	8	15	7	1	3	0	0	0	46
SE	0	0	4	11	35	11	7	4	1	0	0	73
SSE	0	1	4	10	45	16	21	7	8	0	0	112
S	0	3	4	6	42	31	19	13	6	4	0	128
SSW	0	1	4	5	19	25	18	18	35	1	0	126
SW	0	2	4	8	29	17	5	1	1	0	0	67
WSW	0	2	10	18	30	24	5	1	0	0	0	90
W	0	1	12	9	34	27	12	9	2	0	0	106
WNW	0	6	8	15	48	33	28	22	18	0	0	178
NW	0	2	12	27	60	65	31	14	9	0	0	220
NNW	0	2	15	32	45	36	33	9	0	0	0	172
Tot	3	36	132	238	629	493	312	132	98	5	0	2078
Hours of Calm					1							
Hours of Variable Direction					0							
Hours of Valid Data					2079							
Hours of Missing Data					10							
Hours in Period					8760							

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 9 Wind Speed by Direction Measured at 33 Feet for various Stability Classes for the Oyster Creek Generating Station, January – December, 2014

Oyster Creek Alpha												
Period of Record: January - December 2014												
Stability Class – Slightly Stable - 150Ft-33Ft Delta-T (F)												
Winds Measured at 33 Feet												
Wind Direction Sector	Wind Speed (in m/s)											Total
	<0.50	0.5-1	1.1-1.5	1.6-2	2.1-3	3.1-4	4.1-5	5.1-6	6.1-8	8.1-10	>10.00	
N	0	16	16	19	26	8	12	0	0	0	0	97
NNE	0	11	9	10	12	5	0	0	0	0	0	47
NE	1	15	25	40	39	21	2	2	0	0	0	145
ENE	0	13	10	12	28	20	14	2	0	0	0	99
E	1	7	10	5	8	5	4	6	0	0	0	46
ESE	0	1	6	11	7	2	1	0	0	0	0	28
SE	1	5	10	13	29	16	1	1	2	0	0	78
SSE	0	12	19	19	29	25	9	4	2	0	0	119
S	0	11	24	45	79	25	16	12	6	0	0	218
SSW	0	11	25	27	96	65	29	28	13	0	0	294
SW	0	11	25	26	89	37	3	1	0	0	0	192
WSW	0	9	40	44	80	14	5	0	0	0	0	192
W	0	12	31	42	73	11	1	4	0	0	0	174
WNW	0	11	32	37	115	49	21	6	1	0	0	272
NW	0	16	36	46	62	49	19	6	0	0	0	234
NNW	1	21	19	34	44	15	18	5	0	0	0	157
Tot	4	182	337	430	816	367	155	77	24	0	0	2392
Hours of Calm											4	
Hours of Variable Direction											1	
Hours of Valid Data											2397	
Hours of Missing Data											10	
Hours in Period											8760	

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 9 Wind Speed by Direction Measured at 33 Feet for various Stability Classes for the Oyster Creek Generating Station, January – December, 2014

Oyster Creek Alpha

Period of Record: January - December 2014
Stability Class – Moderately Stable - 150Ft-33Ft Delta-T (F)
Winds Measured at 33 Feet

Wind Direction Sector	Wind Speed (in m/s)											Total
	<0.50	0.5-1	1.1-1.5	1.6-2	2.1-3	3.1-4	4.1-5	5.1-6	6.1-8	8.1-10	>10.00	
N	1	17	9	2	0	0	0	0	0	0	0	29
NNE	0	4	4	2	0	0	0	0	0	0	0	10
NE	1	3	5	1	2	0	0	0	0	0	0	12
ENE	0	2	4	1	2	0	0	0	0	0	0	9
E	0	2	3	1	2	0	0	0	0	0	0	8
ESE	1	2	1	0	0	0	0	0	0	0	0	4
SE	0	4	7	1	2	0	0	0	0	0	0	14
SSE	2	4	9	2	0	0	0	0	0	0	0	17
S	0	8	12	5	2	0	0	0	0	0	0	27
SSW	0	10	13	8	11	0	0	0	0	0	0	42
SW	0	15	19	19	11	0	0	0	0	0	0	64
WSW	0	22	34	37	18	0	0	0	0	0	0	111
W	0	18	57	35	21	0	0	0	0	0	0	131
WNW	1	11	39	35	31	0	0	0	0	0	0	117
NW	0	6	38	30	7	0	0	0	0	0	0	81
NNW	0	6	22	13	1	0	0	0	0	0	0	42
Tot	6	134	276	192	110	0	0	0	0	0	0	718

Hours of Calm 5
Hours of Variable Direction 0
Hours of Valid Data 723
Hours of Missing Data 10
Hours in Period 8760

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 9 Wind Speed by Direction Measured at 33 Feet for various Stability Classes for the Oyster Creek Generating Station, January – December, 2014

Oyster Creek Alpha

Period of Record: January - December 2014
 Stability Class – Extremely Stable - 150Ft-33Ft Delta-T (F)
 Winds Measured at 33 Feet

Wind Direction Sector	Wind Speed (in m/s)										Total	
	<0.50	0.5-1	1.1-1.5	1.6-2	2.1-3	3.1-4	4.1-5	5.1-6	6.1-8	8.1-10		>10.00
N	1	7	2	0	0	0	0	0	0	0	0	10
NNE	1	5	0	0	3	0	0	0	0	0	0	9
NE	0	8	0	0	0	0	0	0	0	0	0	8
ENE	1	3	3	0	0	0	0	0	0	0	0	7
E	0	3	3	0	0	0	0	0	0	0	0	6
ESE	1	3	1	0	0	0	0	0	0	0	0	5
SE	0	4	0	2	1	0	0	0	0	0	0	7
SSE	3	9	0	0	0	0	0	0	0	0	0	12
S	2	16	5	4	0	0	0	0	0	0	0	27
SSW	1	18	13	0	1	0	0	0	0	0	0	33
SW	4	39	27	6	0	0	0	0	0	0	0	76
WSW	5	109	142	32	1	0	0	0	0	0	0	289
W	0	203	231	53	3	0	0	0	0	0	0	490
WNW	2	107	83	24	1	1	0	0	0	0	0	218
NW	4	64	77	32	2	0	0	0	0	0	0	179
NNW	3	17	23	10	0	0	0	0	0	0	0	53
Tot	28	615	610	163	12	1	0	0	0	0	0	1429
Hours of Calm					35							
Hours of Variable Direction					1							
Hours of Valid Data					1465							
Hours of Missing Data					10							
Hours in Period					8760							

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 10 Wind Speed by Direction Measured at 380 Feet for various Stability Classes for the Oyster Creek Generating Station, January – December, 2014

Oyster Creek Alpha												
Period of Record: January - December 2014												
Stability Class – All Stabilities - 380Ft-33Ft Delta-T (F)												
Winds Measured at 380 Feet												
Wind Direction Sector	Wind Speed (in m/s)											Total
	<0.50	0.5-1	1.1-1.5	1.6-2	2.1-3	3.1-4	4.1-5	5.1-6	6.1-8	8.1-10	>10.00	
N	0	2	6	7	25	34	62	80	127	77	57	477
NNE	0	1	6	9	40	54	43	61	85	48	22	369
NE	0	2	6	13	53	75	78	96	107	65	40	535
ENE	1	5	5	6	32	56	59	78	122	89	88	541
E	0	3	8	13	33	48	59	20	20	25	26	255
ESE	0	4	5	10	37	60	38	21	15	4	6	200
SE	0	5	12	7	37	69	86	51	68	13	10	358
SSE	0	3	4	6	30	63	74	54	59	54	52	399
S	0	1	6	7	32	55	61	108	145	78	32	525
SSW	0	4	7	6	21	38	64	108	226	168	147	789
SW	0	0	1	7	23	41	65	88	135	131	74	565
WSW	0	1	6	10	37	38	46	86	149	112	75	560
W	0	1	2	5	29	40	43	96	184	126	76	602
WNW	0	2	1	3	22	30	62	81	233	242	221	897
NW	1	5	5	6	34	57	74	106	289	300	186	1063
NNW	0	3	5	6	30	57	63	60	173	136	76	609
Tot	2	42	85	121	515	815	977	1194	2137	1668	1188	8744
Hours of Calm					6							
Hours of Variable Direction					0							
Hours of Valid Data					8750							
Hours of Missing Data					10							
Hours in Period					8760							

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 10 Wind Speed by Direction Measured at 380 Feet for various Stability Classes for the Oyster Creek Generating Station, January – December, 2014

Oyster Creek Alpha

Period of Record: January - December 2014
Stability Class – Extremely Unstable - 380Ft-33Ft Delta-T (F)
Winds Measured at 380 Feet

Wind Direction Sector	Wind Speed (in m/s)											Total	
	<0.50	0.5-1	1.1-1.5	1.6-2	2.1-3	3.1-4	4.1-5	5.1-6	6.1-8	8.1-10	>10.00		
N	0	0	0	0	0	0	0	0	0	0	0	0	0
NNE	0	0	0	0	0	0	0	0	0	0	0	0	0
NE	0	0	0	0	0	0	0	2	0	4	1	7	
ENE	0	0	0	0	0	0	1	2	5	2	0	10	
E	0	0	0	0	0	1	2	0	0	0	0	3	
ESE	0	0	0	0	0	0	1	1	0	0	0	2	
SE	0	0	0	0	0	0	1	2	0	0	0	3	
SSE	0	0	0	0	0	0	0	0	2	0	0	2	
S	0	0	0	0	0	0	0	4	13	13	0	30	
SSW	0	0	0	0	0	0	0	0	0	0	0	0	
SW	0	0	0	0	0	0	0	0	0	0	0	0	
WSW	0	0	0	0	0	0	2	0	0	1	1	4	
W	0	0	0	0	0	0	0	0	2	2	1	5	
WNW	0	0	0	0	0	0	0	0	4	4	10	18	
NW	0	0	0	0	0	0	1	2	15	4	13	35	
NNW	0	0	0	0	0	0	0	0	4	2	0	6	
Tot	0	0	0	0	0	1	8	13	45	32	26	125	

Hours of Calm 0
Hours of Variable Direction 0
Hours of Valid Data 125
Hours of Missing Data 10
Hours in Period 8760

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 10 Wind Speed by Direction Measured at 380 Feet for various Stability Classes for the Oyster Creek Generating Station, January – December, 2014

Oyster Creek Alpha

Period of Record: January - December 2014
 Stability Class – Moderately Unstable - 380Ft-33Ft Delta-T (F)
 Winds Measured at 380 Feet

Wind Direction Sector	Wind Speed (in m/s)											Total
	<0.50	0.5-1	1.1-1.5	1.6-2	2.1-3	3.1-4	4.1-5	5.1-6	6.1-8	8.1-10	>10.00	
N	0	0	0	0	0	0	1	0	4	0	0	5
NNE	0	0	0	0	0	0	1	0	0	1	0	2
NE	0	0	0	0	0	0	1	10	2	4	1	18
ENE	0	0	0	0	0	2	12	10	2	9	2	37
E	0	0	0	0	0	2	11	1	0	0	0	14
ESE	0	0	0	0	0	7	10	3	0	0	0	20
SE	0	0	0	0	0	3	14	7	2	0	0	26
SSE	0	0	0	0	0	0	4	5	6	0	0	15
S	0	0	0	0	0	0	1	7	17	9	0	34
SSW	0	0	0	0	0	0	1	2	8	5	7	23
SW	0	0	0	0	0	1	2	6	6	5	0	20
WSW	0	0	0	0	0	0	1	3	7	6	2	19
W	0	0	0	0	0	0	1	2	7	3	6	19
WNW	0	0	0	0	0	0	0	6	11	7	10	34
NW	0	0	0	0	0	0	2	8	34	14	15	73
NNW	0	0	0	0	0	0	2	7	5	5	1	20
Tot	0	0	0	0	0	15	64	77	111	68	44	379
Hours of Calm												0
Hours of Variable Direction												0
Hours of Valid Data												379
Hours of Missing Data												10
Hours in Period												8760

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 10 Wind Speed by Direction Measured at 380 Feet for various Stability Classes for the Oyster Creek Generating Station, January – December, 2014

Oyster Creek Alpha												
Period of Record: January - December 2014												
Stability Class – Slightly Unstable - 380Ft-33Ft Delta-T (F)												
Winds Measured at 380 Feet												
Wind Direction Sector	Wind Speed (in m/s)											Total
	<0.50	0.5-1	1.1-1.5	1.6-2	2.1-3	3.1-4	4.1-5	5.1-6	6.1-8	8.1-10	>10.00	
N	0	0	0	0	0	2	6	2	1	0	0	11
NNE	0	0	0	0	3	3	3	0	1	0	0	10
NE	0	0	0	0	3	9	14	11	3	2	0	42
ENE	0	0	0	0	3	15	15	15	6	4	1	59
E	0	0	0	0	1	11	11	1	0	0	0	24
ESE	0	0	0	0	4	13	9	1	0	0	0	27
SE	0	0	1	0	3	6	19	8	3	0	0	40
SSE	0	0	0	0	3	0	14	13	8	1	0	39
S	0	0	0	0	0	0	3	8	19	9	0	39
SSW	0	0	0	0	0	2	6	8	8	5	7	36
SW	0	0	0	0	1	5	11	4	8	0	0	29
WSW	0	0	0	0	1	0	6	8	15	3	0	33
W	0	0	0	0	1	4	2	14	23	8	7	59
WNW	0	0	0	0	1	1	9	9	22	15	23	80
NW	0	0	0	0	0	6	17	16	24	14	16	93
NNW	0	0	0	0	0	8	4	10	10	3	3	38
Tot	0	0	1	0	24	85	149	128	151	64	57	659
Hours of Calm 0												
Hours of Variable Direction 0												
Hours of Valid Data 659												
Hours of Missing Data 10												
Hours in Period 8760												

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 10 Wind Speed by Direction Measured at 380 Feet for various Stability Classes for the Oyster Creek Generating Station, January – December, 2014

Oyster Creek Alpha												
Period of Record: January - December 2014												
Stability Class – Neutral - 380Ft-33Ft Delta-T (F)												
Winds Measured at 380 Feet												
Wind Direction Sector	Wind Speed (in m/s)											Total
	<0.50	0.5-1	1.1-1.5	1.6-2	2.1-3	3.1-4	4.1-5	5.1-6	6.1-8	8.1-10	>10.00	
N	0	0	1	2	10	19	28	35	24	32	42	193
NNE	0	1	2	5	18	23	22	36	49	38	22	216
NE	0	1	2	4	32	40	34	51	73	48	30	315
ENE	0	0	1	2	16	26	16	35	74	60	75	305
E	0	0	0	7	18	25	17	9	13	21	24	134
ESE	0	0	1	7	19	26	12	9	8	2	3	87
SE	0	2	4	2	16	35	24	15	30	8	3	139
SSE	0	0	1	2	10	31	34	18	26	24	18	164
S	0	0	1	1	9	20	34	48	47	12	11	183
SSW	0	1	1	2	9	15	19	35	70	53	79	284
SW	0	0	1	3	9	13	22	39	28	15	6	136
WSW	0	0	1	3	20	12	14	28	36	25	4	143
W	0	0	1	3	14	16	24	32	48	29	27	194
WNW	0	0	0	3	10	17	27	30	57	54	91	289
NW	0	0	1	4	17	32	25	42	62	96	73	352
NNW	0	1	2	4	16	25	36	19	49	34	53	239
Tot	0	6	20	54	243	375	388	481	694	551	561	3373
Hours of Calm												0
Hours of Variable Direction												0
Hours of Valid Data												3373
Hours of Missing Data												10
Hours in Period												8760

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 10 Wind Speed by Direction Measured at 380 Feet for various Stability Classes for the Oyster Creek Generating Station, January – December, 2014

Oyster Creek Alpha

Period of Record: January - December 2014
Stability Class – Slightly Stable - 380Ft-33Ft Delta-T (F)
Winds Measured at 380 Feet

Wind Direction Sector	Wind Speed (in m/s)											Total
	<0.50	0.5-1	1.1-1.5	1.6-2	2.1-3	3.1-4	4.1-5	5.1-6	6.1-8	8.1-10	>10.00	
N	0	0	3	2	4	8	12	22	25	8	0	84
NNE	0	0	1	2	10	16	7	14	20	1	0	71
NE	0	0	0	3	11	12	15	15	16	6	8	86
ENE	1	3	1	2	4	5	6	11	30	12	10	85
E	0	0	0	1	6	7	13	7	5	4	2	45
ESE	0	1	1	1	6	12	4	5	4	2	3	39
SE	0	0	1	3	5	10	18	8	18	2	7	72
SSE	0	0	1	1	3	14	10	11	14	24	28	106
S	0	0	2	3	10	17	14	24	34	25	18	147
SSW	0	1	3	1	3	10	20	40	110	89	52	329
SW	0	0	0	0	5	6	19	23	56	76	43	228
WSW	0	1	1	3	3	6	7	22	45	37	30	155
W	0	0	1	0	6	7	9	21	61	48	7	160
WNW	0	0	0	0	7	5	11	14	78	100	43	258
NW	0	1	1	1	4	5	5	18	80	96	28	239
NNW	0	0	1	1	7	9	10	8	45	28	3	112
Tot	1	7	17	24	94	149	180	263	641	558	282	2216
Hours of Calm					1							
Hours of Variable Direction					0							
Hours of Valid Data					2217							
Hours of Missing Data					10							
Hours in Period					8760							

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 10 Wind Speed by Direction Measured at 380 Feet for various Stability Classes for the Oyster Creek Generating Station, January – December, 2014

Oyster Creek Alpha

Period of Record: January - December 2014
 Stability Class – Moderately Stable - 380Ft-33Ft Delta-T (F)
 Winds Measured at 380 Feet

Wind Direction Sector	Wind Speed (in m/s)											Total
	<0.50	0.5-1	1.1-1.5	1.6-2	2.1-3	3.1-4	4.1-5	5.1-6	6.1-8	8.1-10	>10.00	
N	0	1	0	0	3	0	8	12	38	22	11	95
NNE	0	0	1	0	0	2	4	8	7	0	0	22
NE	0	1	0	1	2	8	9	3	5	0	0	29
ENE	0	1	1	1	1	6	7	4	5	1	0	27
E	0	1	2	3	1	2	5	1	1	0	0	16
ESE	0	0	1	1	3	2	2	2	3	0	0	14
SE	0	1	3	1	4	5	3	5	8	1	0	31
SSE	0	1	1	2	3	4	7	5	0	2	4	29
S	0	1	1	1	5	9	5	9	11	8	1	51
SSW	0	1	1	0	5	5	11	7	16	10	1	57
SW	0	0	0	4	5	8	6	9	17	24	16	89
WSW	0	0	2	4	5	9	9	7	29	26	28	119
W	0	1	0	1	4	5	5	14	31	26	18	105
WNW	0	0	0	0	2	4	8	12	35	50	32	143
NW	0	1	1	0	2	5	12	13	41	53	28	156
NNW	0	1	0	0	3	7	4	5	22	39	11	92
Tot	0	11	14	19	48	81	105	116	269	262	150	1075
Hours of Calm					2							
Hours of Variable Direction					0							
Hours of Valid Data					1077							
Hours of Missing Data					10							
Hours in Period					8760							

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Table D – 10 Wind Speed by Direction Measured at 380 Feet for various Stability Classes for the Oyster Creek Generating Station, January – December, 2014

Oyster Creek Alpha

Period of Record: January - December 2014
Stability Class – Extremely Stable - 380Ft-33Ft Delta-T (F)
Winds Measured at 380 Feet

Wind Direction Sector	Wind Speed (in m/s)											Total
	<0.50	0.5-1	1.1-1.5	1.6-2	2.1-3	3.1-4	4.1-5	5.1-6	6.1-8	8.1-10	>10.00	
N	0	1	2	3	8	5	7	9	35	15	4	89
NNE	0	0	2	2	9	10	6	3	8	8	0	48
NE	0	0	4	5	5	6	5	4	8	1	0	38
ENE	0	1	2	1	8	2	2	1	0	1	0	18
E	0	2	6	2	7	0	0	1	1	0	0	19
ESE	0	3	2	1	5	0	0	0	0	0	0	11
SE	0	2	3	1	9	10	7	6	7	2	0	47
SSE	0	2	1	1	11	14	5	2	3	3	2	44
S	0	0	2	2	8	9	4	8	4	2	2	41
SSW	0	1	2	3	4	6	7	16	14	6	1	60
SW	0	0	0	0	3	8	5	7	20	11	9	63
WSW	0	0	2	0	8	11	7	18	17	14	10	87
W	0	0	0	1	4	8	2	13	12	10	10	60
WNW	0	2	1	0	2	3	7	10	26	12	12	75
NW	1	3	2	1	11	9	12	7	33	23	13	115
NNW	0	1	2	1	4	8	7	11	38	25	5	102
Tot	1	18	33	24	106	109	83	116	226	133	68	917
Hours of Calm					3							
Hours of Variable Direction					0							
Hours of Valid Data					920							
Hours of Missing Data					10							
Hours in Period					8760							

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Appendix E ODCM Revisions

See attached copy of CY-OC-170-301 Revision 6

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Appendix F ERRATA

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Corrections to 2013 ARERR

1. A Program Deviation was not reported as required in the 2013 Annual Radiological Effluent release Report in Section J. Program Deviations. A turbine building (TB) effluent noble gas grab sample was not collected for July 2013.
2. A Program Deviation was not reported as required in the 2013 Annual Radiological Effluent release Report in Section J. Program Deviations. The New Radwaste/Augmented Offgas Service Water Effluent sample was not obtained January 1, 2013 through December 31, 2013.
3. An abnormal release was discovered in 2014 after the 2013 ARERR was submitted that was determined to impact the 2013 ARERR. There were multiple pages impacted by this abnormal release and all of the pages are included in this errata section.

Oyster Creek 2014 Annual Radioactive Effluent Release Report

ANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT

January 1, 2013 through December 31, 2013

EXELON GENERATION COMPANY, LLC

OYSTER CREEK GENERATING STATION

DOCKET NO. 50-219 (Oyster Creek Generating Station)

DOCKET NO. 72-15 (Independent Spent Fuel Storage Facility)

**Submitted to
The United States Nuclear Regulatory Commission
Pursuant to
Renewed Facility Operating License DPR-16**

Oyster Creek 2014 Annual Radioactive Effluent Release Report

ORIGINAL PAGE Oyster Creek 2013 Annual Radioactive Effluent Release Report

H. Radiation Effluent Monitors Out of Service More Than 30 Days

Per ODCM Control 3.3.3.10, "Radioactive Liquid Effluent Monitoring Instrumentation" and 3.3.3.11, "Radioactive Gaseous Effluent Monitoring Instrumentation" requires:

With less than the minimum number of radioactive liquid/gaseous effluent monitoring instrumentation channels OPERABLE, take the ACTION shown in Table 3.3.3.10-1/3.3.3.11-1. Make every reasonable effort to return the instrument to OPERABLE status within 30 days and, if unsuccessful, explain in the next Radioactive Effluent Release Report why the inoperability was not corrected in a timely manner.

The following is a discussion of instrumentation out of service for greater than 30 days:

1. The Offgas Building Exhaust Gaseous Effluent Radioactive Noble Gas Monitor was out of service from 5/20/2013 through 8/9/2013. The monitor was taken out of service 5/20/2013 for failing the monitor functional test. The original issue for the monitor failing the functional test was a faulty pushbutton on the alarm panel causing the alarm not to function. Parts were ordered and replaced on 6/18/2013 but the alarm still did not function as expected. After multiple troubleshooting attempts it was determined that the annunciator window card wiring was the issue. After changing the wiring in the alarm panel, all alarms responded as expected. The functional test was reperformed 8/9/2013 satisfactorily and the monitor was returned to service. This issue was entered into our Corrective Action Program (CAP) and corrective actions taken have been documented per process. All ODCM required compensatory measures were met.

I. Releases from the Independent Spent Fuel Storage Facility:

The ISFSI is a closed system and the only exposure would be due to direct radiation. This includes iodines, particulates, and noble gases. Based on onsite TLD readings, dose due to direct radiation from the ISFSI was less than 1 mrem for 2013. Because it is a sealed unit, no radioactive material was released.

J. Program Deviations:

1. The groundwater remediation composite sampler was found with no composite collected on December 27, 2013. The composite sampler is required by ODCM Table 4.11.1.1.1-1, Radioactive Liquid Waste Sampling and Analysis Program. A manual sample was obtained per SP 10-003. Operations department was notified that composite sampler was not working and Operations secured discharging from W-73 and Chemistry secured the composite sampler. This issue was entered into our Corrective Action Program (CAP) and corrective actions taken have been documented per process.

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H. Radiation Effluent Monitors Out of Service More Than 30 Days

Per ODCM Control 3.3.3.10, "Radioactive Liquid Effluent Monitoring Instrumentation" and 3.3.3.11, Radioactive Gaseous Effluent Monitoring Instrumentation requires:

With less than the minimum number of radioactive liquid/gaseous effluent monitoring instrumentation channels OPERABLE, take the ACTION shown in Table 3.3.3.10-1/3.3.3.11-1. Make every reasonable effort to return the instrument to OPERABLE status within 30 days and, if unsuccessful, explain in the next Radioactive Effluent Release Report why the inoperability was not corrected in a timely manner.

The following is a discussion of instrumentation out of service for greater than 30 days:

1. The Offgas Building Exhaust Gaseous Effluent Radioactive Noble Gas Monitor was out of service from 5/20/2013 through 8/9/2013. The monitor was taken out of service 5/20/2013 for failing the monitor functional test. The original issue for the monitor failing the functional test was a faulty pushbutton on the alarm panel causing the alarm not to function. Parts were ordered and replaced on 6/18/2013 but the alarm still did not function as expected. After multiple troubleshooting attempts it was determined that the annunciator window card wiring was the issue. After changing the wiring in the alarm panel, all alarms responded as expected. The functional test was reperformed 8/9/2013 satisfactorily and the monitor was returned to service. This issue was entered into our Corrective Action Program (CAP) and corrective actions taken have been documented per process. All ODCM required compensatory measures were met.

I. Releases from the Independent Spent Fuel Storage Facility:

The ISFSI is a closed system and the only exposure would be due to direct radiation. This includes iodines, particulates, and noble gases. Based on offsite TLD readings, dose due to direct radiation from the ISFSI was less than 1 mrem for 2013. Because it is a sealed unit, no radioactive material was released.

J. Program Deviations:

1. The groundwater remediation composite sampler was found with no composite collected on December 27, 2013. The composite sampler is required by ODCM Table 4.11.1.1.1-1, Radioactive Liquid Waste Sampling and Analysis Program. A manual sample was obtained per SP 10-003. Operations department was notified that composite sampler was not working and Operations secured discharging from W-73 and Chemistry secured the composite sampler. This issue was entered into our Corrective Action Program (CAP) and corrective actions taken have been documented per process.

2. A turbine building (TB) effluent noble gas grab sample was not collected for July 2013. This sample is a requirement of Table 4.11.2.1.2-1, Radioactive Gaseous Waste Sampling and Analysis Program, of CY-OC-170-301, ODCM for Oyster Creek. The July sample was originally scheduled to be collected 7/5/13, but was rescheduled to occur the following week so it could be collected in July. The

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sample was not rescheduled and tracked by the Chemistry Supervisor adequately, and a sample was not collected until the scheduled August sample 8/1/13. Historical data from July's TB RAGEMS low range monitor was trended against the data from June and August with no discrepancies noted. There were no isotopes identified in the Noble Gas Grab Samples from June and August. This issue was entered into our Corrective Action Program (CAP) and corrective actions taken have been documented per process.

3. The New Radwaste (NRW)/Augmented Offgas (AOG) Service Water Effluent sample was not collected January 1, 2013 through December 31, 2013. The New Radwaste/Augmented Offgas Service Water Effluent sample was required by CY-OC-170-301, Offsite Dose Calculation Manual for Oyster Creek Generating Station Table 4.11.1.1.1-1, Radioactive Liquid Waste Sampling and Analysis Program and was required to be sampled weekly for principal gamma emitters and I-131. This issue occurred from the inclusion of this requirement in revision 4 of the ODCM due to a misunderstanding of the system flow path. It was believed that all service water exited the site from the same release point which was evident by the fact that the change paperwork included in the description of this change as being consistent with the practices at Oyster Creek and there was no modification to the sampling schedule. The NRW and AOG Closed Cooling Water systems were reviewed as part of NRC Bulletin IE 80-10. At that time NRW Closed Cooling Water system was determined to be a potentially contaminated system and was added to the sampling program for NRC Bulletin IE 80-10. The sampling program for IE 80-10 requires all potentially contaminated systems to be sampled quarterly for gamma emitters. AOG Closed Cooling Water system was also reviewed but determined to not be a potentially contaminated system. Even though AOG was found not to be a potentially contaminated system sampling is still performed quarterly for gamma emitters. The potential pathway for a release from NRW/AOG Service Water Effluent would be from a heat exchanger leak from the NRW Closed Cooling Water system into the Service Water side of the heat exchanger. The system is designed to maintain the Service Water side of the heat exchanger at a higher pressure than the Closed Cooling Water side so that if any leaks would develop in the heat exchanger it would be Service Water leaking into the Closed Cooling Water side. A review was performed for both NRW and AOG quarterly Closed Cooling Water samples for the time that the required samples were not obtained and all samples were less than MDA for gamma emitters for that entire time period.

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Effluents are strictly monitored to ensure that radioactivity released to the environment is as low as reasonably achievable and does not exceed regulatory limits. Effluent control includes the operation of monitoring systems, in-plant and environmental sampling and analyses programs, quality assurance programs for the effluent and environmental programs, and procedures covering all aspects of effluent and environmental monitoring.

Both radiological environmental and effluent monitoring indicate that the operation of Oyster Creek Generating Station (OCGS) does not result in significant radiation exposure to the people or the environment surrounding OCGS and is well below the applicable levels set by the Nuclear Regulatory Commission (NRC) and the Environmental Protection Agency (EPA).

There were liquid radioactive effluent releases during 2013 of concentrations of tritium too low to detect at an LLD of 200 picocuries per liter (pCi/L) at the New Jersey Pollution Discharge Elimination System (NJPDDES) permitted main condenser outfall. The releases were part of nearly continuous pumping of groundwater at approximately 70 gpm containing low levels of tritium and no detectable gamma. Exelon and the State of New Jersey Department of Environmental Protection (NJDEP) agreed to this remediation action instead of natural attenuation to address concentrations of tritium in groundwater. Well 73 and supporting equipment and piping were installed to pump groundwater to the intake structure at the inlet of the main circulating water pumps. Provisions were established for both batch and continuous releases of groundwater. Continuous releases occurred approximately 361 days in 2013. The Continuous releases occurred from January 1, 2013 through December 27, 2013 with a total of 3.50E+07 gallons of groundwater pumped resulting in 2.82E-01 Ci of tritium released to the discharge canal. The dose to the most limiting member of the public due to the release of groundwater was 1.37E-06 mrem.

There were no liquid abnormal releases during 2013.

There was one gaseous abnormal release during 2013. During the annual inspection of the Old Radwaste Building exhaust fan connections to the main stack it was discovered that there were three slits in the top boot connection of exhaust fan EF-1-16 and a bolt missing in the connection of exhaust fan EF-1-17.

The maximum calculated organ dose (Bone) from iodines, tritium, carbon-14 (C-14), and particulates to any individual due to gaseous effluents was 4.93E-01 mrem, which was approximately 3.29E+00 percent of the annual limit of 15 mrem. The majority of organ dose from gaseous effluents was due to C-14. The maximum calculated gamma air dose in the UNRESTRICTED AREA due to noble gas effluents was 3.60E-03 mrad, which was 3.60E-02 percent of the annual 10 CFR 50 Appendix I, As Low As Reasonably Achievable (ALARA) limit of 10 mrad.

For comparison, the background radiation dose averages approximately 300 mrem per year in the Central New Jersey area, which includes approximately 200 mrem from naturally occurring radon gas and 100 mrem from background radiation.

The Independent Spent Fuel Storage Installation (ISFSI) is a closed system and the only

Oyster Creek 2014 Annual Radioactive Effluent Release Report

CORRECTED PAGE Oyster Creek 2013 Annual Radioactive Effluent Release Report EXECUTIVE SUMMARY

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There were no liquid abnormal releases during 2013.

There were two gaseous abnormal releases during 2013. During the annual inspection of the Old Radwaste Building exhaust fan connections to the main stack it was discovered that there were three slits in the top boot connection of exhaust fan EF-1-16 and a bolt missing in the connection of exhaust fan EF-1-17. During maintenance of the New Radwaste Building exhaust fan HV-S-54A, two slits were identified in the expansion joint and repaired.

The maximum calculated organ dose (Bone) from iodines, tritium, carbon-14 (C-14), and particulates to any individual due to gaseous effluents was $4.94E-01$ mrem, which was approximately $3.29E+00$ percent of the annual limit of 15 mrem. The majority of organ dose from gaseous effluents was due to C-14. The maximum calculated gamma air dose in the UNRESTRICTED AREA due to noble gas effluents was $3.60E-03$ mrad, which was $3.60E-02$ percent of the annual 10 CFR 50 Appendix I, As Low As Reasonably Achievable (ALARA) limit of 10 mrad.

For comparison, the background radiation dose averages approximately 300 mrem per year in the Central New Jersey area, which includes approximately 200 mrem from naturally occurring radon gas and 100 mrem from background radiation.

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exposure is due to direct radiation. Based on offsite TLD readings, dose due to direct radiation from the ISFSI was less than 1 mrem for 2013. Because it is a sealed unit, no radioactive material was released.

Comparison of environmental sampling results to iodine and particulate gaseous effluents released, showed no radioactivity attributable to the operation of OCGS. Both elevated and ground-level release paths were considered in this review, with total iodines released of $3.12\text{E-}03$ Ci and total particulates with half-lives greater than 8 days less C-14 released of $3.82\text{E-}02$ Ci.

Joint Frequency Tables of meteorological data, per Stability Classification Category, as well as for all stability classes, are included. All data was collected from the on-site Meteorological Facility. Data recoveries for the 380-foot data and the 33-foot data were 99.4 percent and 99.4 percent, respectively. The UFSAR commits to Regulatory Guide (RG) 1.23 for Meteorological Facility data recovery. RG 1.23 requires data recovery of at least 90% on an annual basis.

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Comparison of environmental sampling results to iodine and particulate gaseous effluents released, showed no radioactivity attributable to the operation of OCGS. Both elevated and ground-level release paths were considered in this review, with total iodines released of 3.12E-03 Ci and total particulates with half-lives greater than 8 days less C-14 released of 3.84E-02 Ci.

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E. Batch Releases:

1. Liquid

There were no batch releases of liquid effluents during 2013.

2. Gaseous

There were no batch releases of gaseous effluents during 2013.

F. Abnormal Releases:

There were no abnormal liquid releases during 2013.

There was one abnormal gaseous release during 2013. During the performance of Work Order R2205683, which is to perform an annual inspection of the stack pad exhaust fans, three slits were discovered in the top boot of exhaust fan EF-1-16 and a bolt was missing from exhaust fan EF-1-17. These are the two exhaust fans for the Old Radwaste Building to the Main Stack. One of these two exhaust fans is in operation at all times to maintain a negative pressure in the Old Radwaste Building. At no times do both of these fans run simultaneously as each fan is designed to provide 100% of the required flow. Engineering performed a calculation to determine a release rate that encompasses the maximum release rate regardless of which exhaust fan is in service. The activity to inspect the exhaust fans is an annual activity.

The following assumptions were made when calculating Curies released and dose:

- Both exhaust fans have been leaking since the date of the last inspection until the time of the last repair, 7/29/12 through 7/25/13.
- All of the activity due to particulates reported from the Main Stack was coming from the Old Radwaste Building and therefore being released as a groundlevel release from the exhaust fan leaks at the flow rate calculated by Engineering.

	Units	Quarter 1	Quarter 2	Quarter 3	Quarter 4
Total Release	Ci	4.94E-05	5.00E-05	1.37E-05	0.00E+00

G. Revisions to the ODCM:

There were no revisions to the ODCM during 2013.

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E. Batch Releases:

1. Liquid

There were no batch releases of liquid effluents during 2013.

2. Gaseous

There were no batch releases of gaseous effluents during 2013.

F. Abnormal Releases:

There were no abnormal liquid releases during 2013.

There were two abnormal gaseous releases during 2013.

During the performance of Work Order R2205683, which is to perform an annual inspection of the stack pad exhaust fans, three slits were discovered in the top boot of exhaust fan EF-1-16 and a bolt was missing from exhaust fan EF-1-17. These are the two exhaust fans for the Old Radwaste Building to the Main Stack. One of these two exhaust fans is in operation at all times to maintain a negative pressure in the Old Radwaste Building. At no times do both of these fans run simultaneously as each fan is designed to provide 100% of the required flow. Engineering performed a calculation to determine a release rate that encompasses the maximum release rate regardless of which exhaust fan is in service. The activity to inspect the exhaust fans is an annual activity.

The following assumptions were made when calculating Curies released and dose:

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- All of the activity due to particulates reported from the Main Stack was coming from the Old Radwaste Building and therefore being released as a ground level release from the exhaust fan leaks at the flow rate calculated by Engineering.

	Units	Quarter 1	Quarter 2	Quarter 3	Quarter 4
Total Release	Ci	4.94E-05	5.00E-05	1.37E-05	0.00E+00

During the performance of Work Order R2227008, which is to perform an annual inspection of the New Radwaste (NRW) exhaust fan HV-S-54A, two slits were discovered in the expansion joint. This is one of the exhaust fans for the New Radwaste Building to the Main Stack. Note that the exhaust from HV-S-54A is normally to the atmosphere but through the monitored stack effluent release point. Engineering performed a calculation to determine a release rate that encompasses the maximum release rate for both slits. The activity to inspect the exhaust fan is an annual activity.

The following assumptions were made when calculating Curies released and dose:

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- The exhaust fan has been leaking since the date of the last inspection until the time of the repair, 8/20/13 through 7/25/14.
- All of the activity due to particulates reported from the Main Stack was coming from the New Radwaste Building and therefore being released as a groundlevel release from the exhaust fan leaks at the flow rate calculated by Engineering.
- HV-S-54A was in operation the entire time since the last inspection was performed.

	Units	Quarter 1	Quarter 2	Quarter 3	Quarter 4
Total Release	Ci	0.00e0	0.00e0	4.08E-05	9.00E-05

G. Revisions to the ODCM:

There were no revisions to the ODCM during 2013.

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Table A-1: Gaseous Effluents - Summary Of All Releases

Period: January 1, 2013 through December 31, 2013

Unit: Oyster Creek

A. Fission & Activation Gases	Units	Quarter 1	Quarter 2	Quarter 3	Quarter 4	Est. Total Error %
1. Total Release	Ci	2.06E+01	1.40E+01	7.04E+00	8.02E+00	25.00%
2. Average Release Rate for Period	µCi/sec	2.62E+00	1.79E+00	8.85E-01	1.01E+00	
3. Gamma Air Dose	mrad	2.60E-03	1.13E-03	3.99E-04	8.36E-04	
4. Beta Air Dose	mrad	1.87E-04	6.44E-04	2.77E-04	2.79E-04	
5. Percent of ODCM Limit						
- Gamma Air Dose	%	5.20E-02	2.26E-02	7.98E-03	1.67E-02	
- Beta Air Dose	%	1.87E-03	6.44E-03	2.77E-03	2.79E-03	
B. Iodines						
1. Total - I-131	Ci	3.78E-04	1.27E-04	2.57E-04	5.94E-04	25.00%
2. Average Release Rate for Period	µCi/sec	4.81E-05	1.62E-05	3.23E-05	7.47E-05	
3. Percent of ODCM limit	%	*	*	*	*	
C. Particulate						
1. Particulates with T 1/2 > 8 days	Ci	1.32E-02	7.61E-03	8.44E-03	8.97E-03	25.00%
2. Average Release Rate for Period	µCi/sec	1.69E-03	9.67E-04	1.06E-03	1.13E-03	
3. Percent of ODCM limit	%	*	*	*	*	
D. Tritium						
1. Total Release	Ci	8.78E+00	6.40E+00	3.89E+00	5.82E+00	25.00%
2. Average Release Rate for Period	µCi/sec	1.12E+00	8.14E-01	4.90E-01	7.32E-01	
3. Percent of ODCM limit	%	*	*	*	*	
E. Gross Alpha						
1. Total Release	Ci	<LLD	<LLD	<LLD	<LLD	25.00%
2. Average Release Rate for Period	µCi/sec	<LLD	<LLD	<LLD	<LLD	
3. Percent of ODCM limit	%	*	*	*	*	
F. Carbon-14						
1. Total Release	Ci	2.46E+00	2.49E+00	2.53E+00	2.53E+00	
2. Average Release Rate for Period	µCi/sec	3.13E-01	3.17E-01	3.18E-01	3.18E-01	
3. Percent of ODCM limit	%	*	*	*	*	
G. Iodine 131 & 133, Tritium & Particulate						
1. Organ Dose	mrem	5.00E-02	1.40E-01	1.64E-01	1.49E-01	
2. Percent of ODCM Limit	%	6.67E-01	1.87E+00	2.19E+00	1.99E+00	

* ODCM Limit is for combined iodine, tritium, Carbon-14 and particulate only, which is shown in Item G.

Oyster Creek 2014 Annual Radioactive Effluent Release Report

CORRECTED PAGE Oyster Creek 2013 Annual Radioactive Effluent Release Report Table A-1: Gaseous Effluents - Summary Of All Releases

Period: January 1, 2013 through December 31, 2013

Unit: Oyster Creek

A. Fission & Activation Gases	Units	Quarter 1	Quarter 2	Quarter 3	Quarter 4	Est. Total Error %
1. Total Release	Ci	2.06E+01	1.40E+01	7.04E+00	8.02E+00	25.00%
2. Average Release Rate for Period	µCi/sec	2.62E+00	1.79E+00	8.85E-01	1.01E+00	
3. Gamma Air Dose	mrad	2.60E-03	1.13E-03	3.99E-04	8.36E-04	
4. Beta Air Dose	mrad	1.87E-04	6.44E-04	2.77E-04	2.79E-04	
5. Percent of ODCM Limit						
- Gamma Air Dose	%	5.20E-02	2.26E-02	7.98E-03	1.67E-02	
- Beta Air Dose	%	1.87E-03	6.44E-03	2.77E-03	2.79E-03	
B. Iodines						
1. Total – I-131	Ci	3.78E-04	1.27E-04	2.57E-04	5.94E-04	25.00%
2. Average Release Rate for Period	µCi/sec	4.81E-05	1.62E-05	3.23E-05	7.47E-05	
3. Percent of ODCM limit	%	*	*	*	*	
C. Particulate						
1. Particulates with T 1/2 > 8 days	Ci	1.32E-02	7.61E-03	8.48E-03	9.06E-03	25.00%
2. Average Release Rate for Period	µCi/sec	1.69E-03	9.67E-04	1.07E-03	1.14E-03	
3. Percent of ODCM limit	%	*	*	*	*	
D. Tritium						
1. Total Release	Ci	8.78E+00	6.40E+00	3.89E+00	5.82E+00	25.00%
2. Average Release Rate for Period	µCi/sec	1.12E+00	8.14E-01	4.90E-01	7.32E-01	
3. Percent of ODCM limit	%	*	*	*	*	
E. Gross Alpha						
1. Total Release	Ci	<LLD	<LLD	<LLD	<LLD	25.00%
2. Average Release Rate for Period	µCi/sec	<LLD	<LLD	<LLD	<LLD	
3. Percent of ODCM limit	%	*	*	*	*	
F. Carbon-14						
1. Total Release	Ci	2.46E+00	2.49E+00	2.53E+00	2.53E+00	
2. Average Release Rate for Period	µCi/sec	3.13E-01	3.17E-01	3.18E-01	3.18E-01	
3. Percent of ODCM limit	%	*	*	*	*	
G. Iodine 131 & 133, Tritium & Particulate						
1. Organ Dose	mrem	5.00E-02	1.40E-01	1.64E-01	1.49E-01	
2. Percent of ODCM Limit	%	6.67E-01	1.87E+00	2.19E+00	1.99E+00	

* ODCM Limit is for combined iodine, tritium, Carbon-14 and particulate only, which is shown in Item G.

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Table A-3: Gaseous Effluent Release Point: Ground Level Releases

Period: January 1, 2013 through December 31, 2013

Unit: Oyster Creek

Nuclides Released	Unit	Continuous Mode				Batch Mode			
		Quarter 1	Quarter 2	Quarter 3	Quarter 4	Quarter 1	Quarter 2	Quarter 3	Quarter 4
1. Fission gases									
Kr- 85	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Kr- 85m	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Kr-87	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Kr-88	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Xe-133	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Xe-133m	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Xe-135	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Xe-135m	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Xe-138	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Ar-41	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Total for Period	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
2. Iodines									
I-131	Ci	9.94E-08	<LLD	3.73E-08	8.18E-07	<LLD	<LLD	<LLD	<LLD
I-133	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
I-135	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Total for Period	Ci	9.94E-08	<LLD	3.73E-08	8.18E-07	<LLD	<LLD	<LLD	<LLD
3. Particulates									
Sr-89	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Sr-90	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Cs-134	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Cs-137	Ci	2.57E-07	2.59E-07	7.13E-08	<LLD	<LLD	<LLD	<LLD	<LLD
Ba-140	Ci	2.62E-05	2.65E-05	7.28E-06	<LLD	<LLD	<LLD	<LLD	<LLD
La-140	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Cr-51	Ci	5.30E-06	5.35E-06	1.47E-06	<LLD	<LLD	<LLD	<LLD	<LLD
Mn-54	Ci	4.78E-06	5.64E-06	1.22E-06	1.80E-05	<LLD	<LLD	<LLD	<LLD
Co-58	Ci	3.86E-06	1.33E-05	4.02E-06	3.51E-07	<LLD	<LLD	<LLD	<LLD
Co-60	Ci	1.17E-05	2.86E-05	1.69E-05	8.87E-05	<LLD	<LLD	<LLD	<LLD
Ni-63	Ci	2.74E-05	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Mo-99	Ci	7.96E-07	8.05E-07	2.21E-07	<LLD	<LLD	<LLD	<LLD	<LLD
Ag-110m	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Ce-141	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Ce-144	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Fe-55	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Fe-59	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Zn-65	Ci	2.09E-06	2.11E-06	5.81E-07	<LLD	<LLD	<LLD	<LLD	<LLD
Am-241	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Total for Period	Ci	8.24E-05	8.26E-05	3.18E-05	1.07E-04	<LLD	<LLD	<LLD	<LLD
4. Tritium									
H-3	Ci	2.88E-01	1.22E-01	2.93E-01	2.05E-01	<LLD	<LLD	<LLD	<LLD
5. Gross Alpha									
Gross Alpha	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
6. Carbon-14									
C-14	Ci	7.40E-02	7.49E-02	7.57E-02	7.57E-02	<LLD	<LLD	<LLD	<LLD

Oyster Creek 2014 Annual Radioactive Effluent Release Report

CORRECTED PAGE Oyster Creek 2013 Annual Radioactive Effluent Release Report
Table A-3: Gaseous Effluent Release Point: Ground Level Releases

Period: January 1, 2013 through December 31, 2013

Unit: Oyster Creek

Nuclides Released	Unit	Continuous Mode				Batch Mode			
		Quarter 1	Quarter 2	Quarter 3	Quarter 4	Quarter 1	Quarter 2	Quarter 3	Quarter 4
1. Fission gases									
Kr- 85	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Kr- 85m	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Kr-87	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Kr-88	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Xe-133	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Xe-133m	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Xe-135	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Xe-135m	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Xe-138	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Ar-41	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Total for Period	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
2. Iodines									
I-131	Ci	9.94E-08	<LLD	3.73E-08	8.18E-07	<LLD	<LLD	<LLD	<LLD
I-133	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
I-135	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Total for Period	Ci	9.94E-08	<LLD	3.73E-08	8.18E-07	<LLD	<LLD	<LLD	<LLD
3. Particulates									
Sr-89	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Sr-90	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Cs-134	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Cs-137	Ci	2.57E-07	2.59E-07	4.55E-07	8.47E-07	<LLD	<LLD	<LLD	<LLD
Ba-140	Ci	2.62E-05	2.65E-05	1.28E-05	1.22E-05	<LLD	<LLD	<LLD	<LLD
La-140	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Cr-51	Ci	5.30E-06	5.35E-06	4.58E-06	6.86E-06	<LLD	<LLD	<LLD	<LLD
Mn-54	Ci	4.78E-06	5.64E-06	4.94E-06	2.62E-05	<LLD	<LLD	<LLD	<LLD
Co-58	Ci	3.86E-06	1.33E-05	1.47E-05	2.38E-05	<LLD	<LLD	<LLD	<LLD
Co-60	Ci	1.17E-05	2.86E-05	2.93E-05	1.16E-04	<LLD	<LLD	<LLD	<LLD
Ni-63	Ci	2.74E-05	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Mo-99	Ci	7.96E-07	8.05E-07	1.25E-06	2.26E-06	<LLD	<LLD	<LLD	<LLD
Ag-110m	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Ce-141	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Ce-144	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Fe-55	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Fe-59	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Zn-65	Ci	2.09E-06	2.11E-06	4.62E-06	8.90E-06	<LLD	<LLD	<LLD	<LLD
Am-241	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
Total for Period	Ci	8.24E-05	8.26E-05	7.26E-05	1.97E-04	<LLD	<LLD	<LLD	<LLD
4. Tritium									
H-3	Ci	2.88E-01	1.22E-01	2.93E-01	2.05E-01	<LLD	<LLD	<LLD	<LLD
5. Gross Alpha									
Gross Alpha	Ci	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD	<LLD
6. Carbon-14									
C-14	Ci	7.40E-02	7.49E-02	7.57E-02	7.57E-02	<LLD	<LLD	<LLD	<LLD

Oyster Creek 2014 Annual Radioactive Effluent Release Report

ORIGINAL PAGE Oyster Creek 2013 Annual Radioactive Effluent Release Report

Effluent	Applicable Organ	Estimated Dose	Age Group	Location		% of Applicable Limit	Limit	Unit
				Distance (meters)	Direction (toward)			
Noble Gas	Gamma - Air Dose	3.60E-03	All	500	ESE	3.60E-02	10	mrad
Noble Gas	Beta - Air Dose	8.44E-04	All	500	ESE	4.22E-03	20	mrad
Noble Gas	Total Body (Gamma)	1.42E-03	All	937	SE	2.84E-02	5	mrem
Noble Gas	Skin (Beta)	1.77E-03	All	972	ESE	1.18E-02	15	mrem
Iodine, Particulate, Carbon-14 & Tritium	Bone	4.93E-01	Child	937	SE	3.29E+00	15	mrem
Liquid	Total body	1.37E-06	All	South Route 9 Bridge		4.57E-05	3	mrem
Liquid	Organ	1.37E-06	All			1.37E-05	10	mrem
Direct Radiation	Total Body	5.39E+00	All	483	W	2.16E+01	25	mrem
Direct Radiation	Total Body	<LLD	All	937	SE	<LLD	25	mrem
40 CFR Part 190 Compliance								
Warehouse Worker								
Total Dose	Total Body	5.39E+00	All	483	W	2.16E+01	25	mrem
Total Dose	Bone	5.50E+00	All	483	W	2.20E+01	25	mrem
Total Dose	Thyroid	5.39E+00	All	483	W	7.19E+00	75	mrem
Nearest Resident								
Total Dose	Total Body	3.60E-03	All	937	SE	1.44E-02	25	mrem
Total Dose	Bone	4.97E-01	All	937	SE	1.99E+00	25	mrem
Total Dose	Thyroid	3.60E-03	All	937	SE	4.80E-03	75	mrem

Oyster Creek 2014 Annual Radioactive Effluent Release Report

CORRECTED PAGE Oyster Creek 2013 Annual Radioactive Effluent Release Report

Effluent	Applicable Organ	Estimated Dose	Age Group	Location		% of Applicable Limit	Limit	Unit
				Distance (meters)	Direction (toward)			
Noble Gas	Gamma - Air Dose	3.60E-03	All	500	ESE	3.60E-02	10	mrad
Noble Gas	Beta - Air Dose	8.44E-04	All	500	ESE	4.22E-03	20	mrad
Noble Gas	Total Body (Gamma)	1.42E-03	All	937	SE	2.84E-02	5	mrem
Noble Gas	Skin (Beta)	1.77E-03	All	972	ESE	1.18E-02	15	mrem
Iodine, Particulate, Carbon-14 & Tritium	Bone	4.94E-01	Child	937	SE	3.29E+00	15	mrem
Liquid	Total body	1.37E-06	All	South Route 9 Bridge		4.57E-05	3	mrem
Liquid	Organ	1.37E-06	All			1.37E-05	10	mrem
Direct Radiation	Total Body	5.39E+00	All	483	W	2.16E+01	25	mrem
Direct Radiation	Total Body	<LLD	All	937	SE	<LLD	25	mrem
40 CFR Part 190 Compliance								
Warehouse Worker								
Total Dose	Total Body	5.39E+00	All	483	W	2.16E+01	25	mrem
Total Dose	Bone	5.50E+00	All	483	W	2.20E+01	25	mrem
Total Dose	Thyroid	5.39E+00	All	483	W	7.19E+00	75	mrem
Nearest Resident								
Total Dose	Total Body	3.60E-03	All	937	SE	1.44E-02	25	mrem
Total Dose	Bone	4.98E-01	All	937	SE	1.99E+00	25	mrem
Total Dose	Thyroid	3.60E-03	All	937	SE	4.80E-03	75	mrem

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Corrections to 2012 ARERR

1. A Program Deviation was not reported as required in the 2012 Annual Radiological Effluent release Report in Section J. Program Deviations. The New Radwaste/Augmented Offgas Service Water Effluent sample was not obtained January 1, 2012 through December 31, 2012.

Oyster Creek 2014 Annual Radioactive Effluent Release Report

ANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT

January 1, 2012 through December 31, 2012

EXELON GENERATION COMPANY, LLC

OYSTER CREEK GENERATING STATION

DOCKET NO. 50-219 (Oyster Creek Generating Station)

DOCKET NO. 72-15 (Independent Spent Fuel Storage Facility)

**Submitted to
The United States Nuclear Regulatory Commission
Pursuant to
Renewed Facility Operating License DPR-16**

Oyster Creek 2014 Annual Radioactive Effluent Release Report

ORIGINAL PAGE Oyster Creek 2012 Annual Radioactive Effluent Release Report

J. Program Deviations:

1. A turbine building (TB) effluent noble gas grab sample was not collected for September 2012. This sample is a requirement of Table 4.11.2.1.2-1, Radioactive Gaseous Waste Sampling and Analysis Program, of CY-OC-170-301, ODCM for Oyster Creek. The September sample was originally scheduled to be collected 8/31/12, but was rescheduled to occur the following week so it could be collected in September. The sample was not rescheduled and tracked by the Chemistry Supervisor adequately, and a sample was not collected until the scheduled October sample 10/5/12. Historical data from September's TB RAGEMS low range monitor was trended against the data from August and October with no discrepancies noted. There were no isotopes identified in the Noble Gas Grab Samples from August and October. This issue was entered into our Corrective Action Program (CAP) and corrective actions taken have been documented per process.
2. The groundwater remediation composite sampler was found out of service for less than eight hours on October 19, 2012. The composite sampler is required by ODCM Table 4.11.1.1.1-1, Radioactive Liquid Waste Sampling and Analysis Program. There was enough sample in the composite sampler to perform the required analyses and the composite sampler was immediately returned to service. This issue was entered into our Corrective Action Program (CAP) and corrective actions taken have been documented per process.

Oyster Creek 2014 Annual Radioactive Effluent Release Report

CORRECTED PAGE Oyster Creek 2012 Annual Radioactive Effluent Release Report

J. Program Deviations:

1. A turbine building (TB) effluent noble gas grab sample was not collected for September 2012. This sample is a requirement of Table 4.11.2.1.2-1, Radioactive Gaseous Waste Sampling and Analysis Program, of CY-OC-170-301, ODCM for Oyster Creek. The September sample was originally scheduled to be collected 8/31/12, but was rescheduled to occur the following week so it could be collected in September. The sample was not rescheduled and tracked by the Chemistry Supervisor adequately, and a sample was not collected until the scheduled October sample 10/5/12. Historical data from September's TB RAGEMS low range monitor was trended against the data from August and October with no discrepancies noted. There were no isotopes identified in the Noble Gas Grab Samples from August and October. This issue was entered into our Corrective Action Program (CAP) and corrective actions taken have been documented per process.
2. The groundwater remediation composite sampler was found out of service for less than eight hours on October 19, 2012. The composite sampler is required by ODCM Table 4.11.1.1.1-1, Radioactive Liquid Waste Sampling and Analysis Program. There was enough sample in the composite sampler to perform the required analyses and the composite sampler was immediately returned to service. This issue was entered into our Corrective Action Program (CAP) and corrective actions taken have been documented per process.
3. The New Radwaste (NRW)/Augmented Offgas (AOG) Service Water Effluent sample was not collected January 1, 2012 through December 31, 2012. The New Radwaste/Augmented Offgas Service Water Effluent sample was required by CY-OC-170-301, Offsite Dose Calculation Manual for Oyster Creek Generating Station Table 4.11.1.1.1-1, Radioactive Liquid Waste Sampling and Analysis Program and was required to be sampled weekly for principal gamma emitters and I-131. This issue occurred from the inclusion of this requirement in revision 4 of the ODCM due to a misunderstanding of the system flow path. It was believed that all service water exited the site from the same release point which was evident by the fact that the change paperwork included in the description of this change as being consistent with the practices at Oyster Creek and there was no modification to the sampling schedule. The NRW and AOG Closed Cooling Water systems were reviewed as part of NRC Bulletin IE 80-10. At that time NRW Closed Cooling Water system was determined to be a potentially contaminated system and was added to the sampling program for NRC Bulletin IE 80-10. The sampling program for IE 80-10 requires all potentially contaminated systems to be sampled quarterly for gamma emitters. AOG Closed Cooling Water system was also reviewed but determined to not be a potentially contaminated system. Even though AOG was found not to be a potentially contaminated system sampling is still performed quarterly for gamma emitters. The potential pathway for a release from NRW/AOG Service Water Effluent would be from a heat exchanger leak from the NRW Closed Cooling Water system into the Service Water side of the heat exchanger. The system is designed to

Oyster Creek 2014 Annual Radioactive Effluent Release Report

maintain the Service Water side of the heat exchanger at a higher pressure than the Closed Cooling Water side so that if any leaks would develop in the heat exchanger it would be Service Water leaking into the Closed Cooling Water side. A review was performed for both NRW and AOG quarterly Closed Cooling Water samples for the time that the required samples were not obtained and all samples were less than MDA for gamma emitters for that entire time period.

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Corrections to 2011 ARERR

1. A Program Deviation was not reported as required in the 2011 Annual Radiological Effluent release Report in Section J. Program Deviations. The New Radwaste/Augmented Offgas Service Water Effluent sample was not obtained January 1, 2011 through December 31, 2011.

Oyster Creek 2014 Annual Radioactive Effluent Release Report

ANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT

January 1, 2011 through December 31, 2011

EXELON GENERATION COMPANY, LLC

OYSTER CREEK GENERATING STATION

DOCKET NO. 50-219 (Oyster Creek Generating Station)

DOCKET NO. 72-15 (Independent Spent Fuel Storage Facility)

**Submitted to
The United States Nuclear Regulatory Commission
Pursuant to
Renewed Facility Operating License DPR-16**

Oyster Creek 2014 Annual Radioactive Effluent Release Report

ORIGINAL PAGE Oyster Creek 2011 Annual Radioactive Effluent Release Report

J. Program Deviations:

1. During Augmented Off Gas (AOG) sample line pressure testing conducted February 21, 2011, it was discovered that the sample line was not able to maintain pressure due to the filter assembly not being tight. The filter assembly was tightened and the pressure test was completed SAT. This issue was entered into our Corrective Action Program (CAP) and corrective actions implemented to ensure filter assemblies are tight when installed for sampling. The weekly sample results were evaluated against previous sample results for impact to sampling due to the assembly not being tight with no impact identified.
2. The groundwater remediation composite sampler was found out of service for less than one day on March 29, 2011. The composite sampler is required by ODCM Table 4.11.1.1.1-1, Radioactive Liquid Waste Sampling and Analysis Program. There was enough sample in the composite sampler to perform the required analyses and the composite sampler was immediately returned to service.
3. The groundwater remediation composite sampler was found out of service for for approximately 4 hours on August 21, 2011. The composite sampler is required by ODCM Table 4.11.1.1.1-1, Radioactive Liquid Waste Sampling and Analysis Program. The composite sampler was immediately returned to service and enough sample was collected in the composite sampler to perform the required analyses.

Oyster Creek 2014 Annual Radioactive Effluent Release Report

CORRECTED PAGE Oyster Creek 2011 Annual Radioactive Effluent Release Report

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2. The groundwater remediation composite sampler was found out of service for less than one day on March 29, 2011. The composite sampler is required by ODCM Table 4.11.1.1.1-1, Radioactive Liquid Waste Sampling and Analysis Program. There was enough sample in the composite sampler to perform the required analyses and the composite sampler was immediately returned to service.
3. The groundwater remediation composite sampler was found out of service for for approximately 4 hours on August 21, 2011. The composite sampler is required by ODCM Table 4.11.1.1.1-1, Radioactive Liquid Waste Sampling and Analysis Program. The composite sampler was immediately returned to service and enough sample was collected in the composite sampler to perform the required analyses.
4. The New Radwaste (NRW)/Augmented Offgas (AOG) Service Water Effluent sample was not collected January 1, 2011 through December 31, 2011. The New Radwaste/Augmented Offgas Service Water Effluent sample was required by CY-OC-170-301, Offsite Dose Calculation Manual for Oyster Creek Generating Station Table 4.11.1.1.1-1, Radioactive Liquid Waste Sampling and Analysis Program and was required to be sampled weekly for principal gamma emitters and I-131. This issue occurred from the inclusion of this requirement in revision 4 of the ODCM due to a misunderstanding of the system flow path. It was believed that all service water exited the site from the same release point which was evident by the fact that the change paperwork included in the description of this change as being consistent with the practices at Oyster Creek and there was no modification to the sampling schedule. The NRW and AOG Closed Cooling Water systems were reviewed as part of NRC Bulletin IE 80-10. At that time NRW Closed Cooling Water system was determined to be a potentially contaminated system and was added to the sampling program for NRC Bulletin IE 80-10. The sampling program for IE 80-10 requires all potentially contaminated systems to be sampled quarterly for gamma emitters. AOG Closed Cooling Water system was also reviewed but determined to not be a potentially contaminated system. Even though AOG was found not to be a potentially contaminated system sampling is still performed quarterly for gamma emitters. The potential pathway for a release from NRW/AOG Service Water Effluent would be from a heat exchanger leak from the NRW Closed Cooling Water system into the

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Service Water side of the heat exchanger. The system is designed to maintain the Service Water side of the heat exchanger at a higher pressure than the Closed Cooling Water side so that if any leaks would develop in the heat exchanger it would be Service Water leaking into the Closed Cooling Water side. A review was performed for both NRW and AOG quarterly Closed Cooling Water samples for the time that the required samples were not obtained and all samples were less than MDA for gamma emitters for that entire time period.

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Corrections to 2010 ARERR

1. A Program Deviation was not reported as required in the 2010 Annual Radiological Effluent release Report in Section J. Program Deviations. The New Radwaste/Augmented Offgas Service Water Effluent sample was not obtained January 1, 2010 through December 31, 2010.

Oyster Creek 2014 Annual Radioactive Effluent Release Report

ANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT

January 1, 2010 through December 31, 2010

EXELON GENERATION COMPANY, LLC

OYSTER CREEK GENERATING STATION

DOCKET NO. 50-219 (Oyster Creek Generating Station)

DOCKET NO. 72-15 (Independent Spent Fuel Storage Facility)

**Submitted to
The United States Nuclear Regulatory Commission
Pursuant to
Renewed Facility Operating License DPR-16**

Oyster Creek 2014 Annual Radioactive Effluent Release Report

ORIGINAL PAGE Oyster Creek 2010 Annual Radioactive Effluent Release Report

J. Program Deviations:

1. Procedure 621.3.037, Turbine Building RAGEMS Noble Gas Monitor – Functional Test was not performed at the frequency required by ODCM Table 4.3.3.11-1. The test was required to be performed by November 13, 2010 and was completed on December 2, 2010.
2. The groundwater remediation composite sampler was found out of service for less than one day on November 30, 2010. The composite sampler is required by ODCM Table 4.11.1.1.1-1, Radioactive Liquid Waste Sampling and Analysis Program. A grab sample was taken as a compensatory measure and the composite sample was immediately returned to service.
3. On December 18, 2010 at 04:00 the stack monitor noble gas monitor count rate was discovered to be much lower than expected. A valve was found to be out of position causing a dilution in the sample concentration. Based on the stack monitor noble gas monitor readings, the valve had been open for 15 hours. The valve was placed in the correct, closed position, immediately and the count rate returned to normal.
4. The stack sample line was discovered separated on April 7, 2010. A root cause analysis concluded that the sample line was first separated about January 1, 2006. The sample line was returned to service on April 20, 2010.

Oyster Creek 2014 Annual Radioactive Effluent Release Report

CORRECTED PAGE Oyster Creek 2010 Annual Radioactive Effluent Release Report

J. Program Deviations:

1. Procedure 621.3.037, Turbine Building RAGEMS Noble Gas Monitor – Functional Test was not performed at the frequency required by ODCM Table 4.3.3.11-1. The test was required to be performed by November 13, 2010 and was completed on December 2, 2010.
2. The groundwater remediation composite sampler was found out of service for less than one day on November 30, 2010. The composite sampler is required by ODCM Table 4.11.1.1.1-1, Radioactive Liquid Waste Sampling and Analysis Program. A grab sample was taken as a compensatory measure and the composite sample was immediately returned to service.
3. On December 18, 2010 at 04:00 the stack monitor noble gas monitor count rate was discovered to be much lower than expected. A valve was found to be out of position causing a dilution in the sample concentration. Based on the stack monitor noble gas monitor readings, the valve had been open for 15 hours. The valve was placed in the correct, closed position, immediately and the count rate returned to normal.
4. The stack sample line was discovered separated on April 7, 2010. A root cause analysis concluded that the sample line was first separated about January 1, 2006. The sample line was returned to service on April 20, 2010.
5. The New Radwaste (NRW)/Augmented Offgas (AOG) Service Water Effluent sample was not collected January 1, 2010 through December 31, 2010. The New Radwaste/Augmented Offgas Service Water Effluent sample was required by CY-OC-170-301, Offsite Dose Calculation Manual for Oyster Creek Generating Station Table 4.11.1.1.1-1, Radioactive Liquid Waste Sampling and Analysis Program and was required to be sampled weekly for principal gamma emitters and I-131. This issue occurred from the inclusion of this requirement in revision 4 of the ODCM due to a misunderstanding of the system flow path. It was believed that all service water exited the site from the same release point which was evident by the fact that the change paperwork included in the description of this change as being consistent with the practices at Oyster Creek and there was no modification to the sampling schedule. The NRW and AOG Closed Cooling Water systems were reviewed as part of NRC Bulletin IE 80-10. At that time NRW Closed Cooling Water system was determined to be a potentially contaminated system and was added to the sampling program for NRC Bulletin IE 80-10. The sampling program for IE 80-10 requires all potentially contaminated systems to be sampled quarterly for gamma emitters. AOG Closed Cooling Water system was also reviewed but determined to not be a potentially contaminated system. Even though AOG was found not to be a potentially contaminated system sampling is still performed quarterly for gamma emitters. The potential pathway for a release from NRW/AOG Service Water Effluent would be from a heat exchanger leak from the NRW Closed Cooling Water system into the Service Water side of the heat exchanger. The system is designed to maintain the Service Water side of the heat exchanger at a higher pressure than the Closed Cooling Water side so that if any leaks would develop in the

Oyster Creek 2014 Annual Radioactive Effluent Release Report

heat exchanger it would be Service Water leaking into the Closed Cooling Water side. A review was performed for both NRW and AOG quarterly Closed Cooling Water samples for the time that the required samples were not obtained and all samples were less than MDA for gamma emitters for that entire time period.

Oyster Creek 2014 Annual Radioactive Effluent Release Report

Corrections to 2009 ARERR

1. A Program Deviation was not reported as required in the 2009 Annual Radiological Effluent release Report in Section J. Program Deviations. The New Radwaste/Augmented Offgas Service Water Effluent sample was not obtained September 29, 2009 through December 31, 2009.

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2009

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Oyster Creek Generating Station

Revision 2

Issued 2012

Oyster Creek 2014 Annual Radioactive Effluent Release Report

ANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT

January 1, 2009 through December 31, 2009

EXELON GENERATION COMPANY, LLC

OYSTER CREEK GENERATING STATION

DOCKET NO. 50-219 (Oyster Creek Generating Station)

DOCKET NO. 72-15 (Independent Spent Fuel Storage Facility)

**Submitted to
The United States Nuclear Regulatory Commission
Pursuant to
Renewed Facility Operating License DPR-16**

Oyster Creek 2014 Annual Radioactive Effluent Release Report

ORIGINAL PAGE Oyster Creek 2009 Annual Radioactive Effluent Release Report

The maximum hypothetical calculated organ dose from iodines, tritium and particulates to any individual due to gaseous effluents was $3.38\text{E-}02$ mrem , which was approximately $2.25\text{E-}01$ percent of the annual limit. The maximum calculated gamma air dose in the UNRESTRICTED AREA due to noble gas effluents was $4.55\text{E-}03$ mrem, which was $4.55\text{E-}02$ percent of the annual limit.

For comparison, the background radiation dose averages approximately 300 mRem per year in the Central New Jersey area, which includes approximately 200 mrem per year from naturally occurring radon gas and 100 mrem per year from background radiation.

The Independent Spent Fuel Storage Installation (ISFSI) is a closed system and the only exposure would be due to direct radiation. This includes iodines, particulates and noble gases. Based on offsite TLD readings, dose due to direct radiation from the ISFSI was less than 1 mRem for 2009. Because it is a sealed unit, no radioactive material was released.

Additionally, comparison of environmental sampling results to iodine and particulate gaseous effluents released, showed no radioactivity attributable to the operation of OCGS. Both elevated and ground-level release paths were considered in this review, with total iodines released of $1.78\text{E-}02$ Ci and total particulates with half-lives greater than 8 days released of $2.50\text{E-}02$ Ci.

Joint Frequency Tables of meteorological data, per Pasquill Category, as well as for all stability classes, are included. All data was collected from the on-site Meteorological Facility. Data recoveries for the 380-foot data and the 33-foot data were 97.8 percent and 97.8 percent, respectively. The UFSAR commits to Regulatory Guide (RG) 1.23 for Meteorological Facility data recovery. RG 1.23 requires data recovery of at least 90% on an annual basis.

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CORRECTED PAGE Oyster Creek 2009 Annual Radioactive Effluent Release Report

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The New Radwaste (NRW)/Augmented Offas (AOG) Service Water Effluent sample was not collected September 29, 2009 through December 31, 2009. The New Radwaste/Augmented Offas Service Water Effluent sample was required by CY-OC-170-301, Offsite Dose Calculation Manual for Oyster Creek Generating Station Table 4.11.1.1.1-1, Radioactive Liquid Waste Sampling and Analysis Program and was required to be sampled weekly for principal gamma emitters and I-131. This issue occurred from the inclusion of this requirement in revision 4 of the ODCM due to a misunderstanding of the system flow path. It was believed that all service water exited the site from the same release point which was evident by the fact that the change paperwork included in the description of this change as being consistent with the practices at Oyster Creek and there was no modification to the sampling schedule. The NRW and AOG Closed Cooling Water systems were reviewed as part of NRC Bulletin IE 80-10. At that time NRW Closed Cooling Water system was determined to be a potentially contaminated system and was added to the sampling program for NRC Bulletin IE 80-10. The sampling program for IE 80-10 requires all potentially contaminated systems to be sampled quarterly for gamma emitters. AOG Closed Cooling Water system was also reviewed but determined to not be a potentially contaminated system. Even though AOG was found not to be a potentially contaminated system sampling is still performed quarterly for gamma emitters. The potential pathway for a release from NRW/AOG Service Water Effluent would be from a heat exchanger leak from the NRW Closed Cooling Water system into the Service Water side of the heat exchanger. The system is designed to maintain the Service Water side of the heat exchanger at a higher pressure than the Closed Cooling Water side so that if any leaks would

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develop in the heat exchanger it would be Service Water leaking into the Closed Cooling Water side. A review was performed for both NRW and AOG quarterly Closed Cooling Water samples for the time that the required samples were not obtained and all samples were less than MDA for gamma emitters for that entire time period.

OFFSITE DOSE CALCULATION MANUAL

FOR

OYSTER CREEK GENERATING STATION

Revision of this document requires PORC approval and changes are controlled by
CY-AA-170-3100

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OYSTER CREEK GENERATING STATION OFF SITE DOSE CALCULATION MANUAL

INTRODUCTION

The Oyster Creek Off Site Dose Calculation Manual (ODCM) is an implementing document to the Oyster Creek Technical Specifications. The previous Limiting Conditions for Operations that were contained in the Radiological Effluent Technical Specifications (RETS) are now included in the ODCM as Radiological Effluent Controls (REC). The ODCM contains two parts: Part I – Radiological Effluent Controls, and Part II – Calculational Methodologies.

Part I includes the following:

- The Radiological Effluent Controls and the Radiological Environmental Monitoring Programs required by Technical Specifications 6.8.4
- Descriptions of the information that should be included in the Annual Radioactive Effluent Release Report and the Annual Radiological Environmental Operating Report required by Technical Specifications 6.9.1.d and 6.9.1.e, respectively.

Part II describes methodologies and parameters used for:

- The calculation of radioactive liquid and gaseous effluent monitoring instrumentation alarm/trip set points; and
- The calculation of radioactive liquid and gaseous concentrations, dose rates, cumulative yearly doses, and projected doses.

Part II also contains a list and graphical description of the specific sample locations for the radiological environmental monitoring program (REMP), and the liquid and gaseous waste treatment systems and discharge points.

PART I - RADIOLOGICAL EFFLUENT CONTROLS

1.0 DEFINITIONS

The following terms are defined so that uniform interpretation of these CONTROLS may be achieved. The defined terms appear in capitalized type and are applicable throughout these CONTROLS.

1.1 OPERABLE – OPERABILITY

A system, subsystem, train, component or device shall be OPERABLE or have OPERABILITY when it is capable of performing its specified function(s). Implicit in the definition shall be the assumption that all necessary attendant instrumentation, controls, normal and emergency electrical power sources, cooling or seal water, lubrication or other auxiliary equipment that are required for the system, subsystem, train, component or device to perform its function(s) are also capable of performing their related support function(s).

A verification of OPERABILITY is an administrative check, by examination of appropriate plant records (logs, surveillance test records) to determine that a system, subsystem, train, component or device is not inoperable. Such verification does not preclude the demonstration (testing) of a given system, subsystem, train, component or device to determine OPERABILITY.

1.2 ACTION

ACTION shall be that part of a CONTROL that prescribes remedial measures required under designated conditions.

1.3 CHANNEL CALIBRATION

A CHANNEL CALIBRATION shall be the adjustment, as necessary, of the channel output such that it responds, with acceptable range and accuracy, to known values of the parameter that the channel monitors. The CHANNEL CALIBRATION shall encompass the entire channel, including equipment actuation, alarm, or trip.

1.4 CHANNEL CHECK

A CHANNEL CHECK shall be a qualitative determination of acceptable operability by observation of channel behavior during operation. This determination shall include, where possible, comparison of the channel with other independent channels measuring the same variable.

1.5 CHANNEL FUNCTIONAL TEST

CHANNEL FUNCTIONAL TEST shall be the injection of a simulated signal into the channel to verify its proper response including, where applicable, alarm and/or trip initiating actions.

1.6 CONTROL

The Limiting Conditions for Operation (LCOs) that were contained in the Radiological Effluent Technical Specifications were transferred to the OFF SITE DOSE CALCULATION MANUAL (ODCM) and were renamed CONTROLS. This is to distinguish between those LCOs that were retained in the Technical Specifications and those LCOs or CONTROLS that were transferred to the ODCM.

1.7 FREQUENCY NOTATION

The FREQUENCY NOTATION specified for the performance of Surveillance Requirements shall correspond to the intervals defined in Table 1.1.

1.8 REPORTABLE EVENT

A REPORTABLE EVENT shall be any of those conditions specified Section 50.73 to 10CFR Part 50.

1.9 SOURCE CHECK

SOURCE CHECK shall be the qualitative assessment of channel response when the channel sensor is exposed to a source of increased radioactivity.

1.10 AUGMENTED OFF GAS SYSTEM (AOG)

The AUGMENTED OFF GAS SYSTEM is designed and installed to holdup and/or process radioactive gases from the main condenser off gas system for the purpose of reducing the radioactive material content of the gases before release to the environs.

1.11 MEMBER (S) OF THE PUBLIC

MEMBER (S) OF THE PUBLIC shall include all persons who are not occupationally associated with Exelon Generation and who do not normally frequent the Oyster Creek Generating Station site. This category does not include employees of the utility, its contractors, contractor employees, vendors, or persons who enter the site to make deliveries, to service equipment, work on site or for other purposes associated with plant functions. This category does include persons who use portions of the site for recreational, occupational, or other

purposes not associated with the plant. An individual is not a member of the public during any period in which the individual receives an occupational dose.

1.12 OFF SITE DOSE CALCULATION MANUAL (ODCM)

The OFF SITE DOSE CALCULATION MANUAL (ODCM) shall contain the methodology and parameters used in the calculation of Off Site doses due to radioactive gaseous and liquid effluents, in the calculation of gaseous and liquid effluent monitoring Alarm/Trip Set points, and in the conduct of the Radiological Environmental Monitoring Program. The ODCM shall also contain (1) the Radioactive Effluent Controls and Radiological Environmental Monitoring Programs required by Technical Specification Section 6.8.4 and (2) descriptions of the information that should be included in the Annual Radioactive Effluent Release Report AND Annual Radiological Environmental Operating Report required by Technical Specification Sections 6.9.1.d and 6.9.1.e, respectively.

1.13 PURGE – PURGING

PURGE or PURGING shall be the controlled process of discharging air or gas from a confinement and replacing it with air or gas.

1.14 SITE BOUNDARY

The SITE BOUNDARY shall be the perimeter line around OCGS beyond which the land is neither owned, leased, nor otherwise subject to control by Exelon Generation. The area outside the SITE BOUNDARY is termed OFF SITE or UNRESTRICTED AREA.

1.15 OFF SITE

The area that is beyond the site boundary where the land is neither owned, leased nor otherwise subject to control by Exelon Generation. Can be interchanged with UNRESTRICTED AREA.

1.16 UNRESTRICTED AREA

An UNRESTRICTED AREA shall be any area at or beyond the SITE BOUNDARY, access to which is not controlled by the licensee for purposes of protection of individuals from exposure to radiation and radioactive materials, or any area within the SITE BOUNDARY used for residential quarters or for industrial, commercial, institutional, and/or recreational purposes. Can be interchanged with OFF SITE.

1.17 DOSE EQUIVALENT I-131

DOSE EQUIVALENT I-131 shall be that concentration of I-131 (micro curies per gram), which alone would produce the same thyroid dose as the quantity and isotopic mixture of I-131, I-132, I-133, I-134, and I-135 actually present. The thyroid dose conversion factors used for this calculation shall be those listed in Table E-7 of Regulatory Guide 1.109, "Calculation of Annual Doses to Man from Routine Releases of Reactor Effluences for the Purpose of Evaluating Compliance with 10CFR Part 40 Appendix I."

1.18 DEPOSITION (D/Q)

The direct removal of gaseous and particulate species on land or water surfaces. DEPOSITION is expressed as a quantity of material per unit area (e.g. m^{-2}).

1.19 DOSE CONVERSION FACTOR (DCF)

A parameter calculated by the methods of internal dosimetry, which indicates the committed dose equivalent (to the whole body or organ) per unit activity inhaled or ingested. This parameter is specific to the isotope and the dose pathway. DOSE CONVERSION FACTORS are commonly tabulated in units of mrem/hr per picocurie/ m^3 in air or water. They can be found in Reg Guide 1.109 appendices.

1.20 EFFLUENT CONCENTRATION (EC)

The liquid and air concentration levels which, if inhaled or ingested continuously over the course of a year, would produce a total effective dose equivalent of 0.05 rem. LEC refers to liquid EFFLUENT CONCENTRATION.

1.21 ELEVATED (STACK) RELEASE

An airborne effluent plume whose release point is higher than twice the height of the nearest adjacent solid structure and well above any building wake effects so as to be essentially unentrained. Regulatory Guide 1.111 is the basis of the definition of an ELEVATED RELEASE. Elevated releases generally will not produce any significant ground level concentrations within the first few hundred yards of the source. ELEVATED RELEASES generally have less dose consequence to the public due to the greater downwind distance to the ground concentration maximum compared to ground releases. All main stack releases at the OCGS are ELEVATED RELEASES.

1.22 FINITE PLUME MODEL

Atmospheric dispersion and dose assessment model which is based on the assumption that the horizontal and vertical dimensions of an effluent plume are not necessarily large compared to the distance that gamma rays can travel in air. It is more realistic than the semi-infinite plume model because it considers the

finite dimensions of the plume, the radiation build-up factor, and the air attenuation of the gamma rays coming from the cloud. This model can estimate the dose to a receptor who is not submerged in the radioactive cloud. It is particularly useful in evaluating doses from an elevated plume or when the receptor is near the effluent source.

1.23 GROUND LEVEL (VENT) RELEASE

An airborne effluent plume which contacts the ground essentially at the point of release either from a source actually located at ground elevation or from a source well above the ground elevation which has significant building wake effects to cause the plume to be entrained in the wake and driven to the ground elevation. GROUND LEVEL RELEASES are treated differently than ELEVATED RELEASES in that the X/Q calculation results in significantly higher concentrations at the ground elevation near the release point.

1.24 OCCUPATIONAL DOSE

The dose received by an individual in a RESTRICTED AREA or in the course of employment in which the individual's assigned duties involve exposure to radiation and to radioactive material from licensed and unlicensed sources of radiation, whether in the possession of the licensee or other person. Occupational dose does not include dose received from background radiation, as a patient from medical practices, from voluntary participation in medical research programs, or as a member of the general public

1.25 "OPEN DOSE"

A routine effluent dosimetry computer program that uses Reg. Guides 1.109 and 1.111 methodologies.

1.26 RAGEMS (RADIOACTIVE EMISSIONS MONITORING SYSTEM)

A plant system that monitors gaseous effluent releases from monitored release points. There is a RAGEMS system for the main stack (RAGEMS I) and one for the turbine building (RAGEMS II). They monitor particulates, iodine's, and noble gases.

1.27 SEMI-INFINITE PLUME MODEL

Dose assessment model with the following assumptions. The ground is considered to be an infinitely large flat plate and the receptor is located at the origin of a hemispherical cloud of infinite radius. The radioactive cloud is limited to the space above the ground plane. The semi-infinite plume model is limited to immersion dose calculations.

1.28 SOURCE TERM

The activity release rate, or concentration of an actual release or potential release. The common units for the source term are curies, curies per second, and curies per cubic centimeter, or multiples thereof (e.g., micro curies).

1.29 X/Q - ("CHI over Q")

The dispersion factor of a gaseous release in the environment calculated by a point source Gaussian dispersion model. Normal units of X/Q are sec/m^3 . The X/Q is used to determine environmental atmospheric concentrations by multiplying the source term, represented by Q (in units of $\mu\text{Ci}/\text{sec}$ or Ci/sec). Thus, the plume dispersion, X/Q (seconds/cubic meter) multiplied by the source term, Q ($\mu\text{Ci}/\text{seconds}$) yields an environmental concentration, X ($\mu\text{Ci}/\text{m}^3$). X/Q is a function of many parameters including wind speed, stability class, release point height, building size, and release velocity.

1.30 SEEDS (Simplified Effluent Environmental Dosimetry System)

A routine effluent dosimetry computer program that uses Reg. Guides 1.109 and 1.111 methodologies.

TABLE 1.1: SURVEILLANCE FREQUENCY NOTATION *

<u>NOTATION</u>	<u>FREQUENCY</u>
S	At least once per 12 hours.
D	At least once per 24 hours.
W	At least once per 7 days.
M	At least once per 31 days.
Q	At least once per 92 days.
SA	At least once per 184 days.
A	At least once per 366 days.
R	At least once per 18 months (550 days).
1/24	At least once per 24 months (refueling cycle)
S/U	Prior to each reactor startup.
P	Prior to each radioactive release.
N.A.	Not applicable.

* Each surveillance requirement shall be performed within the specified time interval with a maximum allowable extension not to exceed 25% of the surveillance interval.

3/4 CONTROLS AND SURVEILLANCE REQUIREMENTS

3/4.0 APPLICABILITY

CONTROLS

- 3.0.1 Compliance with the CONTROLS contained in the succeeding CONTROLS is required during the OPERATIONAL CONDITIONS or other conditions specified therein; except that upon failure to meet the CONTROL, the associated ACTION requirements shall be met.
- 3.0.2 Noncompliance with a CONTROL shall exist when the requirements of the CONTROL and associated ACTION requirements are not met within the specified time intervals. If the CONTROL is restored prior to expiration of the specified time intervals, completion of the ACTION requirements is not required.
- 3.0.3 Except as provided in the associated ACTION requirements, when a CONTROL is not met or the associated ACTION requirements cannot be satisfied, action shall be initiated to place the unit into COLD SHUTDOWN within the following 30 hours.

Where corrective measures are completed that permit operation under the ACTION requirements, the ACTION may be taken in accordance with the specified time limits as measured from the time of failure to meet the CONTROL. Exceptions to these requirements are stated in the individual CONTROLS.

This CONTROL is not applicable in COLD SHUTDOWN or REFUELING.

- 3.0.4 Entry into an OPERATIONAL CONDITION or other specified condition shall not be made when the conditions of the CONTROLS are not met and the associated ACTION requires a shutdown if they are not met within a specified time interval. Entry into an OPERATIONAL CONDITION or other specified condition may be made in accordance with ACTION requirements when conformance to them permits continued operation of the facility for an unlimited period of time. This provision shall not prevent passage through or to OPERATIONAL CONDITIONS as required to comply with ACTION requirements. Exceptions to these requirements are stated in the individual CONTROLS.
- 3.0.5 Equipment removed from service or declared inoperable to comply with ACTIONS may be returned to service under administrative control solely to perform testing to demonstrate its OPERABILITY or the OPERABILITY of other equipment. This is an exception to CONTROL 3.0.2 for the system returned to service under administrative control to perform the testing required to demonstrate OPERABILITY.

3 /4.0 APPLICABILITY

SURVEILLANCE REQUIREMENTS

- 4.0.1 Surveillance Requirements shall be met during the OPERATIONAL CONDITIONS or other conditions specified for individual CONTROLS unless otherwise stated in an individual Surveillance Requirement.
- 4.0.2 Each Surveillance Requirement shall be performed within the specified surveillance interval with a maximum allowable extension not to exceed 25 percent of the specified surveillance interval.
- 4.0.3 Failure to perform a Surveillance Requirement within the allowed surveillance interval, defined by CONTROL 4.0.2, shall constitute a failure to meet the OPERABILITY requirements for a CONTROL. The time limits of the ACTION requirements are applicable at the time it is identified that a Surveillance Requirement has not been performed. The ACTION requirements may be delayed for up to 24 hours to permit the completion of the surveillance when the allowed outage time limits of the ACTION requirements are less than 24 hours. Surveillance Requirements do not have to be performed on inoperable equipment.
- 4.0.4 Entry into an OPERATIONAL CONDITION or other specified applicable condition shall not be made unless the Surveillance Requirement(s) associated with the CONTROLS have been performed within the applicable surveillance interval or as otherwise specified. This provision shall not prevent passage through or to OPERATIONAL CONDITIONS as required to comply with ACTION requirements.

3/4.3 INSTRUMENTATION

3/4.3.3.10 RADIOACTIVE LIQUID EFFLUENT MONITORING INSTRUMENTATION

CONTROLS

3.3.3.10 In accordance with Oyster Creek Technical Specifications 6.8.4.a.1, the radioactive liquid effluent monitoring instrumentation channels shown in Table 3.3.3.10-1 shall be OPERABLE with their Alarm/Trip set points set to ensure that the limits of CONTROL 3.11.1.1 are not exceeded. The Alarm/Trip set points of these channels shall be determined and adjusted in accordance with the methodology and parameters in the ODCM Part II section 1.2.1.

APPLICABILITY: During all liquid releases via these pathways.

ACTION:

- a. With a radioactive liquid effluent monitoring instrumentation channel Alarm/Trip set point less conservative than required by the above CONTROL, immediately suspend the release of radioactive liquid effluents monitored by the affected channel, or declare the channel inoperable, or change the set point so it is acceptably conservative, or provide for manual initiation of the Alarm/Trip function(s).
- b. With less than the minimum number of radioactive liquid effluent monitoring instrumentation channels OPERABLE, take the ACTION shown in Table 3.3.3.10-1. Make every reasonable effort to return the instrument to OPERABLE status within 30 days and, if unsuccessful, explain in the next Radioactive Effluent Release Report pursuant to Technical Specification 6.9.1.d why the inoperability was not corrected in a timely manner.
- c. The provisions of CONTROL 3.0.3 and 3.0.4 are not applicable. Report all deviations in the Radioactive Effluent Release Report.

SURVEILLANCE REQUIREMENTS

4.3.3.10 Each radioactive liquid effluent monitoring instrumentation channel shall be demonstrated OPERABLE by performance of the CHANNEL CHECK, SOURCE CHECK, CHANNEL CALIBRATION, and CHANNEL FUNCTIONAL TEST at the Frequencies shown in Table 4.3.3.10-1.

TABLE 3.3.3.10-1: RADIOACTIVE LIQUID EFFLUENT MONITORING INSTRUMENTATION

INSTRUMENT	MINIMUM CHANNELS OPERABLE	ACTION
1. RADIOACTIVITY MONITORS PROVIDING ALARM AND AUTOMATIC TERMINATION OF RELEASE		
a. Liquid Radwaste Effluent Line (DELETED)	N/A	N/A
b. Turbine Building Sump No. 1-5 (DELETED)	N/A	N/A
2. RADIOACTIVITY MONITORS PROVIDING ALARM BUT NOT PROVIDING AUTOMATIC TERMINATION OF RELEASE		
a. Reactor Building Service Water System Effluent Line	1	112
3. FLOW RATE MEASUREMENT DEVICES		
a. Liquid Radwaste Effluent Line (DELETED)	N/A	N/A
b. Groundwater Remediation Release Path	1	115

TABLE 3.3.3.10-1 (Continued)

TABLE NOTATIONS

- ACTION 110 With no channels OPERABLE, effluent releases via this pathway may continue provided that:
- a. At least two independent samples are taken, one prior to discharge and one near the completion of discharge and analyzed in accordance with SURVEILLANCE REQUIREMENT 4.11.1.1.1.
 - b. Before initiating a release, at least two technically qualified members of the Facility Staff independently verify the release rate calculations and discharge line valving;
- Otherwise, suspend release of radioactive effluents via this pathway.
- ACTION 112 With no channels OPERABLE, effluent releases via this pathway may continue provided that, at least once per 24 hours, grab samples are collected and analyzed for radioactivity at a limit of detection specified by Table 4.11.1.1.1-1.
- ACTION 113 With no channel OPERABLE, effluent releases via the affected pathway may continue provided the flow is estimated with the pump curve or change in tank level, at least once per batch during a release.
- ACTION 114 With no channel OPERABLE effluent may be released provided that before initiating a release:
- a. A sample is taken and analyzed in accordance with SURVEILLANCE REQUIREMENT 4.11.1.1.1.
 - b. Qualified personnel determine and independently verify the acceptable release rate.
- ACTION 115 With no channel OPERABLE, effluent releases via this pathway may continue provided the flow rate is estimated at least once per 4 hours during actual releases. Pump performance curves may be used to estimate flow.

TABLE 4.3.3.10-1: RADIOACTIVE LIQUID EFFLUENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS^a

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>SOURCE CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>CHANNEL FUNCTIONAL TEST</u>
1. RADIOACTIVITY MONITORS PROVIDING ALARM AND AUTOMATIC TERMINATION OF RELEASE				
a. Liquid Radwaste Effluent Line (DELETED)	N/A	N/A	N/A	N/A
b. Turbine Building Sump No. 1-5 (DELETED)	N/A	N/A	N/A	N/A
2. RADIOACTIVITY MONITORS PROVIDING ALARM BUT NOT PROVIDING AUTOMATIC TERMINATION OF RELEASE				
a. Reactor Building Service Water System Effluent Line	D	M	R ^e	Q ^d
3. FLOW RATE MEASUREMENT DEVICES				
a. Liquid Radwaste Effluent Line (DELETED)	N/A	N/A	N/A	N/A
b. Groundwater Remediation Release path	D ^f	N/A	R ^f	N/A

TABLE 4.3.3.10-1 (Continued)

TABLE NOTATIONS

- a. Instrumentation shall be **OPERABLE** and in service except that a channel may be taken out of service for the purpose of a check, calibration, test or maintenance without declaring it to be inoperable.

- d. The **CHANNEL FUNCTIONAL TEST** shall also demonstrate that Control Room alarm annunciation occurs if any of the following conditions exists:
 - 1. Instrument indicates measured levels above the alarm set point.
 - 2. Instrument indicates a downscale failure.
 - 3. Instrument controls not set in operate mode.
 - 4. Instrument electrical power loss.

- e. The **CHANNEL CALIBRATION** shall be performed according to established calibration procedures.

- f. While actively discharging through this pathway.

3/4.3 INSTRUMENTATION

3/4.3.3.11 RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION

CONTROLS

3.3.3.11 In accordance with Oyster Creek Technical Specifications 6.8.4.a.1, the radioactive gaseous effluent monitoring instrumentation channels shown in Table 3.3.3.11-1 shall be OPERABLE with their alarm/trip set points set to ensure that the limits of CONTROL 3.11.2.1 are not exceeded. The alarm/trip set points of these channels meeting CONTROLS 3.11.2.1 shall be determined and adjusted in accordance with the methodology and parameters in the ODCM Part II Section 2.2.

APPLICABILITY: As shown in Table 3.3.3.11-1

ACTION:

- a. With a radioactive gaseous effluent monitoring instrumentation channel alarm/trip set point less conservative than required by the above CONTROL, immediately suspend the release of radioactive gaseous effluents monitored by the affected channel, or declare the channel inoperable, or change the set point so it is acceptably conservative.
- b. With less than the minimum number of radioactive gaseous effluent monitoring instrumentation channels OPERABLE, take the ACTION shown in Table 3.3.3.11-1. Exert best efforts to return the instrument to OPERABLE status within 30 days and, if unsuccessful, explain in the next Radioactive Effluent Release Report pursuant to Technical Specification 6.9.1.d why this inoperability was not corrected in a timely manner.
- c. The provisions of CONTROLS 3.0.3 and 3.0.4 are not applicable. Report all deviations in the Radioactive Effluent Release Report.

SURVEILLANCE REQUIREMENTS

4.3.3.11 Each radioactive gaseous effluent monitoring instrumentation channel shall be demonstrated OPERABLE by performance of the CHANNEL CHECK, SOURCE CHECK, CHANNEL CALIBRATION, and CHANNEL FUNCTIONAL TEST operations at the frequencies shown in Table 4.3.3.11-1.

TABLE 3.3.3.11-1: RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>MINIMUM CHANNELS OPERABLE^a</u>	<u>APPLICABILITY</u>	<u>ACTION</u>
1. DELETED			
2. STACK MONITORING SYSTEM			
a. Radioactive Noble Gas Monitor (Low Range)	1	b,e	124
b. Iodine Sampler	1	b,e	127
c. Particulate Sampler	1	b,e	127
d. Effluent Flow Measuring Device	1	b	122
e. Sample Flow Measuring Device	1	b	128
3. TURBINE BUILDING VENTILATION MONITORING SYSTEM			
a. Radioactive Noble Gas Monitor (Low Range)	1	b	123
b. Iodine Sampler	1	b	127
c. Particulate Sampler	1	b	127
d. Effluent Flow Measuring Device	1	b	122
e. Sample Flow Measuring Device	1	b	128

TABLE 3.3.3.11-1(Continued)

RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>MINIMUM CHANNELS OPERABLE^a</u>	<u>APPLICABILITY</u>	<u>ACTION</u>
4. AUGMENTED OFF GAS BUILDING EXHAUST VENTILATION MONITORING SYSTEM			
a. Radioactive Noble Gas Monitor	1	b	123
b. Iodine Sampler	1	b	127
c. Particulate Sampler	1	b	127
d. Sample Flow Measuring Device	1	b	128

TABLE 3.3.3.11-1 (Continued)

TABLE NOTATIONS

- a. Channels shall be OPERABLE and in service as indicated except that a channel may be taken out of service for the purpose of a check, calibration, test maintenance or sample media change without declaring the channel to be inoperable.
- b. During releases via this pathway
- e. Monitor / sampler or an alternate shall be OPERABLE to monitor / sample Stack effluent whenever the drywell is being purged.

- ACTION 122 With no channel OPERABLE, effluent releases via this pathway may continue provided the flow rate is estimated whenever the exhaust fan combination in this system is changed.
- ACTION 123 With no channel OPERABLE, effluent releases via this pathway may continue provided a grab sample is taken at least once per 48 hours and is analyzed for gross radioactivity within 24 hours thereafter or provided an alternate monitoring system with local display is utilized.
- ACTION 124 With no channel OPERABLE, effluent releases via this pathway may continue provided a grab sample is taken at least once per 8 hours and analyzed for gross radioactivity within 24 hours or provided an alternate monitoring system with local display is utilized. Drywell purge is permitted only when the radioactive noble gas monitor is operating.
- ACTION 127 With no channel OPERABLE, effluent releases via this pathway may continue provided the required sampling is initiated with auxiliary sampling equipment as soon as reasonable after discovery of inoperable primary sampler(s).
- ACTION 128 With no channel OPERABLE, effluent releases via the sampled pathway may continue provided the sampler air flow is estimated and recorded at least once per day.

TABLE 4.3.3.11-1: RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>SOURCE CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>MODES IN WHICH SURVEILLANCE REQUIRED^a</u>
1. DELETED					
2. MAIN STACK MONITORING SYSTEM					
a. Radioactive Noble Gas Monitor (Low Range)	D	M	1/24 ^f	Q ^h	b
b. Iodine Sampler	W	N.A.	N.A.	N.A.	b
c. Particulate Sampler	W	N.A.	N.A.	N.A.	b
d. Effluent Flow Measuring Device	D	N.A.	1/24	Q	b
e. Sample Flow Measuring Device	D	N.A.	R	Q	b
3. TURBINE BUILDING VENTILATION MONITORING SYSTEM					
a. Radioactive Noble Gas Monitor (Low Range)	D	M	1/24 ^f	Q ⁱ	b
b. Iodine Sampler	W	N.A.	N.A.	N.A.	b
c. Particulate Sampler	W	N.A.	N.A.	N.A.	b
d. Effluent Flow Measuring Device	D	N.A.	1/24	Q	b
e. Sample Flow Measuring Device	D	N.A.	R	Q	b

TABLE 4.3.3.11-1(Continued)RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>SOURCE CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>MODES IN WHICH SURVEILLANCE IS REQUIRED^a</u>
4. AUGMENTED OFF GAS BUILDING EXHAUST VENTILATION MONITORING SYSTEM					
a. Radioactive Noble Gas Monitor	D	M	R ^f	Q ^e	b
b. Iodine Sampler	W	N.A.	N.A.	N.A.	b
c. Particulate Sampler	W	N.A.	N.A.	N.A.	b
d. Sample Flow Measuring Device	D	N.A.	R	N.A.	b

TABLE 4.3.3.11-1 (Continued)

TABLE NOTATIONS

- a. Instrumentation shall be OPERABLE and in service except that a channel may be taken out of service for the purpose of a check calibration, test or maintenance without declaring it to be inoperable.
- b. During releases via this pathway.
- e. The CHANNEL FUNCTIONAL TEST shall also demonstrate that control room alarm annunciation occurs if any of the following conditions exist:
 - 1. Instrument indicates measured levels above the alarm set point.
 - 2. Instrument indicates a downscale failure.
 - 3. Instrument controls not set in operate mode.
 - 4. Instrument electrical power loss.
- f. The CHANNEL CALIBRATION shall be performed according to established calibration procedures.
- h. The CHANNEL FUNCTIONAL TEST shall also demonstrate that control room alarm annunciation occurs if any of the following conditions exist:
 - 1. Instrument indicates measured levels above the alarm set point.
 - 2. Instrument indicates a low count rate/monitor failure.
 - 3. Switch cover alarm shall be verified to alarm when the cover is opened; and clear when the cover is closed after the faceplate switches are verified in their correct positions.
- i. The CHANNEL FUNCTIONAL TEST shall also demonstrate that control room alarm annunciation occurs if any of the following conditions exist:
 - 1. Instrument indicates measured levels above the alarm set point.
 - 2. Instrument indicates a low count rate/monitor failure.

3/4.11 RADIOACTIVE EFFLUENTS

3/4.11.1 LIQUID EFFLUENTS

3/4.11.1.1 CONCENTRATION

CONTROLS

3.11.1.1 In accordance with the Oyster Creek Technical Specifications 6.8.4.a.2 and 3, the concentration of radioactive material, other than noble gases, in liquid effluent in the discharge canal at the Route 9 bridge (See Figure E-4) shall not exceed the concentrations specified in 10 CFR Part 20, Appendix B, Table 2, Column 2. The concentration of noble gases dissolved or entrained in liquid effluent in the discharge canal at the Route 9 bridge shall not exceed 2E-4 microcuries/milliliter.

APPLICABILITY: At all times.

ACTION:

- a. In the event the concentration of radioactive material in liquid effluent released into the Off Site area beyond the Route 9 bridge exceeds either of the concentration limits above, reduce the release rate without delay to bring the concentration below the limit.
- b. The provisions of CONTROLS 3.0.3, 3.0.4 and Technical Specification 6.9.2 are not applicable.

SURVEILLANCE REQUIREMENTS

4.11.1.1.1 Radioactive liquid wastes shall be sampled and analyzed according to the sampling and analysis program in Table 4.11.1.1.1-1.

Alternately, pre-release analysis of batches(es) of radioactive liquid waste may be by gross beta or gamma counting provided a maximum concentration limit of 1E-8 $\mu\text{Ci/ml}$ in the discharge canal at the Route 9 bridge is applied.

4.11.1.1.2 The results of the radioactivity analyses shall be used in accordance with the methodology and parameters in the ODCM Part II Section 1.2 to assure that the concentrations at the point of release are maintained within the limits of CONTROL 3.11.1.1 and 3.11.1.2.

4.11.1.1.3 The alarm or trip set point of each radioactivity monitoring channel in Table 3.3.3.10-1 shall be determined on the basis of sampling and analyses results obtained according to Table 4.11.1.1.1-1 and the set point method in ODCM Part II 1.2.1 and set to alarm or trip before exceeding the limits of CONTROL 3.11.1.1.

TABLE 4.11.1.1.1-1: RADIOACTIVE LIQUID WASTE SAMPLING AND ANALYSIS PROGRAM

Liquid Release Type	Sampling Frequency	Minimum Analysis Frequency	Type of Activity Analysis	Lower Limit Detection ^a (LLD) ($\mu\text{Ci/ml}$)
1. Batch Waste Release Tanks	P Each Batch ^b	P ^c Each Batch	Principal Gamma Emitters	5E-07
			I-131	5E-07
	P One Batch/M ^b	M	Dissolved and Entrained Gases (Gamma Emitters)	1E-05
	P Each Batch ^b	M Composite ^d	H-3	1E-05
			Gross Alpha	1E-07
	P Each Batch ^b	Q Composite ^d	Sr-89, Sr-90	5E-08
Fe-55			1E-06	
2. Continuous Release a. Reactor Building Service Water Effluent	W Grab Sample ^e	W	Principal Gamma Emitters	5E-07
			I-131	5E-07
	(note f)	M Composite ^g	H-3	1E-05
			Gross Alpha	1E-07
			Fe-55	1E-06
3. Groundwater Remediation Pathway a. Continuous	Continuous ^h	D Composite	Principal Gamma Emitters	5E-07
			H-3	1E-05
	M	M Composite ^g	Gross Alpha	1E-07
			Sr-89, Sr-90	5E-08
			Fe-55	1E-06
			Ni-63	1E-06
b. Batch Release Tank	P Each Batch ^b	P Each Batch	Principal Gamma Emitters	5E-07
			H-3	1E-05
	P Each Batch ^b	M Composite ^d	Gross Alpha	1E-07
			Sr-89, Sr-90	5E-08
			Fe-55	1E-06
		Ni-63	1E-06	

TABLE 4.11.1.1-1 (CONTINUED)

TABLE NOTATIONS

- a. The Lower Limit of Detection (LLD) is defined, for purposes of these specifications, as the smallest concentration of radioactive material in a sample that will yield a net count, above system background, that will be detected with 95 percent probability with only 5 percent probability of falsely concluding that a blank observation represents a "real" signal.

The LLD is applicable to the capability of a measurement system under typical conditions and not as a limit for the measurement of a particular sample in the radioactive liquid waste sampling and analyses program.

For a particular measurement system, which may include radiochemical separation:

$$LLD = \frac{4.66 * S_b}{E * V * 2.22E6 * Y * \exp(-\lambda\Delta t)}$$

Where:

LLD is the lower limit of detection as defined above (microcurie per unit mass or volume),

S_b is the standard deviation of the background counting rate or of the counting rate of a blank sample as appropriate (counts per minute),

E is the counting efficiency,

V is the sample size (units of mass or volume),

2.22E+6 is the number of disintegrations per minute per microcurie,

Y is the fractional radiochemical yield, when applicable,

λ is the radioactive decay constant for the particular radionuclide, and

Δt is the elapsed time between the end of the sample collection and the time of counting.

Analyses shall be performed in such a manner that the stated LLDs will be achieved under routine conditions with typical values of E, V, Y, and t for the radionuclides Mn-54, Fe-59, Co-58, Co-60, Zn-65, Mo-99, Ce-141, Cs-134, Cs-137; and an LLD of 5E-6 $\mu\text{Ci/ml}$ should typically be achieved for Ce-144.

TABLE 4.11.1.1.1-1 (CONTINUED)

TABLE NOTATIONS

Occasionally, background fluctuations, interfering radionuclides, or other uncontrollable circumstances may render these LLD's unachievable.

When calculating the LLD for a radionuclide determined by gamma ray spectrometry, the background may include the typical contributions of other radionuclides normally present in the sample. The background count rate of a semiconductor detector (e.g. HPGe) is determined from background counts that are determined to be within the full width of the specific energy band used for the quantitative analysis for the radionuclide.

The principal gamma emitters for which the LLD specification will apply are exclusively the following radionuclides: Mn-54, Fe-59, Co-58, Co-60, Zn-65, Mo-99, Cs-134, Cs-137, Ce-141, and Ce-144. This list does not mean that only these nuclides are to be detected and reported. Other peaks that are measurable and identifiable, together with the above nuclides, shall be identified and reported. The LLD for Ce-144 is $5E-6$ $\mu\text{Ci/mL}$ whereas the LLD for Mo-99 and the other gamma emitters is $5E-7$ $\mu\text{Ci/mL}$. Nuclides that are below the LLD for the analysis should not be reported.

- b. A batch release is the discharge of liquid wastes of a discrete volume. Before sampling for analysis, each batch should be thoroughly mixed.
- c. In the event a gross radioactivity analysis is performed in lieu of an isotopic analysis before a batch is discharged, a sample will be analyzed for principal gamma emitters afterwards.
- d. A composite sample is one in which the quantity of liquid sampled is proportional to the quantity of liquid waste discharged and in which the method of sampling employed results in a specimen which is representative of the liquids released.
- e. Analysis may be performed after release.
- f. In the event a grab sample contains more than $5E-7$ $\mu\text{Ci/mL}$ of I-131 and principal gamma emitters or in the event the Reactor Building Service Water radioactivity monitor indicates more than $5E-7$ $\mu\text{Ci/mL}$ radioactivity in the effluent, as applicable, sample the elevated activity effluent daily until analysis confirms the activity concentration in the effluent does not exceed $5E-7$ $\mu\text{Ci/mL}$. In addition a composite sample must be made up for further analysis for all samples taken when the activity was $> 5E-7$ $\mu\text{Ci/mL}$.
- g. A composite sample is produced combining grab samples, each having a defined volume, collected routinely from the sump or stream being sampled
- h. Aliquot sampling shall be at least hourly to be representative of the quantities and concentrations of radioactive materials in liquid effluent. Prior to analyses, all samples taken for the composite shall be thoroughly mixed in order for the composite samples to be representative of the effluent release.

3/ 4.11 RADIOACTIVE EFFLUENTS

3/ 4.11.1.2 DOSE

CONTROLS

- 3.11.1.2 In accordance with Oyster Creek Technical Specifications 6.8.4.a.4 and 5, the dose or dose commitment to a MEMBER OF THE PUBLIC from radioactive materials in liquid effluents released to UNRESTRICTED AREAS (see Figure E-4) shall be limited:
- a. During any calendar quarter to less than or equal to 1.5 mrem to the Total Body and to less than or equal to 5 mrem to any body organ, and
 - b. During any calendar year to less than or equal to 3 mrem to the Total Body and to less than or equal to 10 mrem to any body organ.

APPLICABILITY: At all times.

ACTION:

- a. With the calculated dose from the release of radioactive materials in liquid effluents exceeding any of the above limits, prepare and submit to the Commission within 30 days from the end of the quarter, pursuant to Technical Specification 6.9.2, a Special Report that identifies the cause(s) for exceeding the limit(s) and defines the corrective actions that have been taken and/or will be taken to reduce the releases and the proposed corrective actions to be taken to assure that subsequent releases will be in compliance with the above limits.
- b. The provisions of CONTROL 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

- 4.11.1.2 Cumulative dose contributions from liquid effluents for the current calendar quarter and the current calendar year shall be determined in accordance with the methodology and parameters in the ODCM Part II Section 1.5 at least once per 31 days in accordance with Technical Specification 6.8.4.a.5.

3/4.11 RADIOACTIVE EFFLUENTS

3/4.11.1.3 LIQUID WASTE TREATMENT SYSTEM

CONTROLS

3.11.1.3 In accordance with the Oyster Creek Technical Specifications 6.8.4.a.6, the liquid radwaste treatment system shall be OPERABLE and appropriate portions of the system shall be used to reduce releases of radioactivity when projected doses due to the liquid effluent to UNRESTRICTED AREAS (see Figure E-4) would exceed 0.06 mrem to the Total Body or 0.2 mrem to any organ in a 31 day period.

APPLICABILITY: At all times.

ACTION:

- a. With radioactive liquid waste being discharged without treatment and in excess of the above, prepare and submit to the Commission within 30 days, pursuant to Technical Specification 6.9.2, a Special Report that includes the following information:
 1. Explanation of why liquid radwaste was being discharged without treatment, identification of any inoperable equipment or subsystems, and the reason for the inoperability,
 2. Action(s) taken to restore the inoperable equipment to OPERABLE status, and
 3. Summary description of action(s) taken to prevent a recurrence.
- b. The provisions of CONTROL 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.11.1.3.1 Doses due to liquid releases to UNRESTRICTED AREAS shall be projected at least once per 31 days in accordance with the methodology and parameters in the ODCM Part II Section 1.5 in accordance with Technical Specifications 6.8.4.a.5.

4.11.1.3.2 The installed liquid radwaste treatment system shall be demonstrated OPERABLE by meeting CONTROLS 3.11.1.1, 3.11.1.2, and 3.11.1.3.

3/4.11 RADIOACTIVE EFFLUENTS

3/4.11.2 GASEOUS EFFLUENTS

3/4.11.2.1 DOSE RATE

CONTROLS

3.11.2.1 In accordance with the Oyster Creek Technical Specifications 6.8.4.a.5 and 7, the dose rate due to radioactive materials released in gaseous effluents in the UNRESTRICTED AREA (see Figure E-4) shall be limited to the following:

- a. For noble gases: Less than or equal to 500 mrem/yr to the total body and less than or equal to 3000 mrem/yr to the skin, and
- b. For iodine-131, iodine-133, tritium, and for all radionuclides in particulate form with half-lives greater than 8 days: Less than or equal to 1500 mrem/yr to any body organ.

APPLICABILITY: At all times.

ACTION:

- a. With the dose rate(s) exceeding the above limits, immediately restore the release rate to within the above limit(s).
- b. If the gaseous effluent release rate cannot be reduced to meet the above limits, the reactor shall be in at least SHUTDOWN CONDITION within 48 hours unless corrective actions have been completed and the release rate restored to below the above limit.

SURVEILLANCE REQUIREMENTS

4.11.2.1.1 The dose rate due to noble gases in gaseous effluents shall be determined to be within the above limits in accordance with the methodology and parameters in the ODCM Part II Section 2.3.1.

4.11.2.1.2 The dose rate due to iodine-131, iodine-133, tritium, and all radionuclides in particulate form with half-lives greater than 8 days in gaseous effluents shall be determined to be within the above limits in accordance with the methodology and parameters in the ODCM Part II Section 2.3.2 by obtaining representative samples and performing analyses in accordance with the sampling and analysis program specified in Table 4.11.2.1.2-1.

4.11.2.1.3 Dose rates due to tritium, Sr-89, Sr-90, and alpha-emitting radionuclides are averaged over no more than 3 months and the dose rate due to other radionuclides is averaged no more than 31 days.

TABLE 4.11.2.1.2-1: RADIOACTIVE GASEOUS WASTE SAMPLING AND ANALYSIS PROGRAM

Gaseous Release Type	Sampling Frequency	Minimum Analysis Frequency	Type of Activity Analysis	Lower Limit Detection ^a (LLD) ($\mu\text{Ci/ml}$)
Stack; Turbine Building Exhaust Vents; Augmented Off gas Building Vent	Q Grab Sample ^f	Q	H-3	1E-06
	M Grab Sample c,d,f	M	Principal Gamma Emitters ^b (Noble Gases)	1E-04
	Continuous ^f	W Charcoal Sample	I-131	1E-12
			I-133	1E-10
	Continuous ^f	W Particulate Sample	Principal Gamma Emitters ^b (particulates)	1E-11
	Continuous ^f	M ^e Composite Particulate Sample	Gross Alpha	1E-11
	Continuous	Q ^e Composite Particulate Sample	Sr-89, Sr-90	1E-11
	Continuous	Noble Gas Monitor	Noble Gases Gamma Radioactivity	1E-06

TABLE 4.11.2.1.2-1 (Continued)

TABLE NOTATIONS

- a. The Lower Limit of Detection (LLD) is defined, for purposes of these specifications, as the smallest concentration of radioactive material in a sample that will yield a net count, above system background, that will be detected with 95 percent probability with only 5 percent probability of falsely concluding that a blank observation represents a "real" signal.

The LLD is applicable to the capability of a measurement system under typical conditions and not as a limit for the measurement of a particular sample in the radioactive liquid waste sampling and analyses program.

For a particular measurement system which may include radiochemical separation:

$$LLD = \frac{4.66 * S_b}{E * V * 2.22E6 * Y * \exp(-\lambda \Delta t)}$$

Where:

LLD is the lower limit of detection as defined above (microcurie per unit mass or volume),

S_b is the standard deviation of the background counting rate or of the counting rate of a blank sample as appropriate (counts per minute),

E is the counting efficiency,

V is the sample size (units of mass or volume),

2.22E+6 is the number of disintegrations per minute per microcurie,

Y is the fractional radiochemical yield, when applicable,

λ is the radioactive decay constant for the particular radionuclide, and

Δt is the elapsed time between the end of the sample collection and the time of counting.

Analyses shall be performed in such a manner that the stated LLD's will be achieved under routine conditions with typical values of E, V, Y, and t for the radionuclides Mn-54, Fe-59, Co-58, Co-60, Zn-65, Cs-134, Cs-137, and Ce-141. Occasionally background fluctuations or other uncontrollable circumstances may render these LLD's unachievable.

When calculating the LLD for a radionuclide determined by gamma ray spectrometry, the background may include the typical contributions of other radionuclides normally present in the samples. The background count rate of a HpGe detector is determined from background

TABLE 4.11.2.1.2-1 (Continued)TABLE NOTATIONS

counts that are determined to be within the full width of the specific energy band used for the quantitative analysis for that radionuclide

- b. The principal gamma emitters for which the LLD specification applies exclusively are the following radionuclides: Kr-87, Kr-88, Xe-133, Xe-133m, Xe-135 and Xe-138 for gaseous emissions and Mn-54, Fe-59, Co-58, Co-60, Zn-65, Mo-99, Cs-134, Cs-137, Ce-141, and Ce-144 for particulate emissions. This list does not mean that only these nuclides are to be considered. Other gamma peaks that are identifiable, together with those of the above nuclides, shall also be analyzed and reported in the Annual Radioactive Effluent Release Report consistent with CONTROL 3.11.2.1. The LLD for Mo-99 and Ce-144 is $1\text{E-}10$ $\mu\text{Ci/ml}$ whereas the LLD for other principal gamma emitting particulates is $1\text{E-}11$ $\mu\text{Ci/ml}$. Radionuclides which are below the LLD for the analysis should not be reported.
- c. The noble gas radionuclides in gaseous effluent may be identified by taking a grab sample of effluent and analyzing it.
- d. In the event the reactor power level increases more than 15 percent in one hour and the Stack noble gas radioactivity monitor shows an activity increase of more than a factor of three after factoring out the effect due to the change in reactor power, a grab sample of Stack effluent shall be collected and analyzed.
- e. A composite particulate sample shall include an equal fraction of at least one particulate sample collected during each week of the compositing period.
- f. In the event a sample is collected for 24 hours or less, the LLD may be increased by a factor of 10.

3/4.11 RADIOACTIVE EFFLUENTS

3/4.11.2.2 DOSE - NOBLE GASES

CONTROLS

- 3.11.2.2 In accordance with the Oyster Creek Technical Specification 6.8.4.a.5 and 8, the air dose due to noble gases released in gaseous effluents in the UNRESTRICTED AREA (see Figure E-4) shall be limited to the following:
- a. During any calendar quarter: Less than or equal to 5 mrad for gamma radiation and less than or equal to 10 mrad for beta radiation and,
 - b. During any calendar year: Less than or equal to 10 mrad for gamma radiation and less than or equal to 20 mrad for beta radiation.

APPLICABILITY: At all times.

ACTION:

- a. With the calculated air dose from radioactive noble gases in gaseous effluents exceeding any of the above limits, prepare and submit to the Commission within 30 days from the end of the quarter during which the release occurred, pursuant to Technical Specification 6.9.2, a Special Report that identifies the cause(s) for exceeding the limit(s) and defines the corrective actions that have been taken to reduce the release and the proposed corrective actions to be taken to assure that subsequent releases will be in compliance with the above limits.
- b. The provisions of CONTROLS 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

- 4.11.2.2 Cumulative dose contributions for the current calendar quarter and current calendar year for noble gases shall be determined in accordance with the methodology and parameters in the ODCM Part II Section 2.4.1 at least once per 31 days in accordance with Technical Specification 6.8.4.a.5.

3/4.11 RADIOACTIVE EFFLUENTS

3/4.11.2.3 DOSE - IODINE-131, IODINE-133, TRITIUM, AND RADIONUCLIDES IN PARTICULATE FORM

CONTROLS

- 3.11.2.3 In accordance with Oyster Creek Technical Specification 6.8.4.a.5 and 9, the dose to a MEMBER OF THE PUBLIC from iodine-131, iodine-133, tritium, and all radionuclides in particulate form with half-lives greater than 8 days in gaseous effluents released in the UNRESTRICTED AREA (see Figure E-4) shall be limited to the following:
- a. During any calendar quarter: Less than or equal to 7.5 mrem to any body organ and,
 - b. During any calendar year: Less than or equal to 15 mrem to any body organ.

APPLICABILITY: At all times.

ACTION:

- a. With the calculated dose from the release of iodine-131, iodine-133 and radionuclides in particulate form with half-lives greater than 8 days, in gaseous effluents exceeding any of the above limits, prepare and submit to the Commission within 30 days, pursuant to Technical Specification 6.9.2, a Special Report that identifies the cause(s) for exceeding the limit and defines the corrective actions that have been taken to reduce the releases and the proposed corrective actions to be taken to assure that subsequent releases will be in compliance with the above limits.
- b. The provisions of CONTROLS 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

- 4.11.2.3 Cumulative dose contributions for the current calendar quarter and current calendar year for iodine-131, iodine-133, tritium, and radionuclides in particulate form with half-lives greater than 8 days shall be determined in accordance with the methodology and parameters in the ODCM Part II Section 2.5 at least once per 31 days in accordance with Technical Specification 6.8.4.a.5.

3/4.11 RADIOACTIVE EFFLUENTS

3/4.11.2.4 GASEOUS RADWASTE TREATMENT SYSTEM

CONTROLS

3.11.2.4 The AUGMENTED OFF GAS SYSTEM shall be in operation to reduce releases of radioactivity when the projected doses in 31 days due to gaseous effluent releases to areas at and beyond the SITE BOUNDARY (see Figure E-4) would exceed:

- a. 0.2 mrad to air from gamma radiation, or
- b. 0.4 mrad to air from beta radiation, or
- c. 0.3 mrem to any body organ

APPLICABILITY: Whenever the main condenser steam jet air ejector is in operation except during startup or shutdown with reactor power less than 40 percent of rated. In addition, the AUGMENTED OFF GAS SYSTEM need not be in operation during end of cycle coast-down periods when the system can no longer function due to low off gas flow.

ACTION:

- a. Every reasonable effort shall be made to maintain and operate charcoal absorbers in the AUGMENTED OFF GAS SYSTEM to treat radioactive gas from the main condenser air ejectors.
- b. With gaseous radwaste from the main condenser air ejector system being discharged without treatment for more than 30 consecutive days and either CONTROL 3.11.2.1 or 3.11.2.4 exceeded, prepare and submit to the Commission within 30 days from the end of the quarter during which release occurred, pursuant to Technical Specification 6.9.2, a Special Report that includes the following information:
 - 1. Identification of any inoperable equipment or subsystems, and the reason for the inoperability,
 - 2. Action(s) taken to restore the inoperable equipment to OPERABLE status, and
 - 3. Summary description of action(s) taken to prevent a recurrence.
- c. The provisions of CONTROL 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

- 4.11.2.4.1 Operation of the Augmented Off gas System charcoal absorbers shall be verified by verifying the AOG System bypass valve, V-7-31, alignment or alignment indication closed at least once every 12 hours whenever the main condenser air ejector is operating.
- 4.11.2.4.2 Doses due to gaseous releases to UNRESTRICTED AREAS shall be projected at least once per 31 days in accordance with the methodology and parameters in the ODCM Part II Section 1.5 in accordance with Technical Specifications 6.8.4.a.5.

3/4.11 RADIOACTIVE EFFLUENTS

3/4.11.3 MARK I CONTAINMENT

CONTROLS

3.11.3.1 Venting or purging of the containment Drywell may be through normal Reactor Building Ventilation if the following requirements are met:

APPLICABILITY: If the Station year-to-date radiological effluent releases (either iodine or noble gas) are less than 10% of the ODCM limit, then Standby Gas Treatment is NOT required for purging the contents of the Drywell.

ACTION:

- a. If the Station year-to-date radiological effluent releases (either iodine or noble gas) are greater than 10% of the ODCM limit, then the Standby Gas Treatment System must be used for purging the contents of the Drywell.

SURVEILLANCE REQUIREMENTS

4.11.3.1 The Standby Gas Treatment System is OPERABLE and available whenever the purge system is in use.

3/4.11 RADIOACTIVE EFFLUENTS

3/4.11.4 TOTAL DOSE

CONTROLS

3.11.4.1 In accordance with Oyster Creek Technical Specifications 6.8.4.a.10, the annual (calendar year) dose commitment to any MEMBER OF THE PUBLIC due to radioactive material in the effluent and direct radiation from the OCGS in the UNRESTRICTED AREA shall be limited to less than or equal to 75 mrem to the thyroid or less than or equal to 25 mrem to the total body or any other organ.

APPLICABILITY: At all times

ACTION:

- a. With the calculated doses from the release of radioactive materials in liquid or gaseous effluents exceeding twice the limits of CONTROLS 3.11.1.2a, 3.11.1.2b, 3.11.2.2a, 3.11.2.2b, 3.11.2.3a, or 3.11.2.3b, perform an assessment to determine whether the limits of CONTROL 3.11.4.1 have been exceeded. If such is the case, prepare and submit to the Commission within 30 days, pursuant to Technical Specification 6.9.2, a Special Report that defines the corrective action to be taken to reduce subsequent releases to prevent recurrence of exceeding the above limits and includes the schedule for achieving conformance with the above limits. This Special Report shall include information specified in 10CFR20.2203. If the estimated dose(s) exceeds the above limits, and if the release condition resulting in violation of 40 CFR Part 190 has not already been corrected, the Special Report shall include a request for a variance in accordance with the provisions of 40 CFR Part 190. Submittal of the report is considered a timely request, and a variance is granted until staff action on the request is complete.
- b. The provisions of CONTROLS 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

- 4.11.4.1 Cumulative dose contributions from liquid and gaseous effluents shall be determined in accordance with SURVEILLANCE REQUIREMENT 4.11.1.2, 4.11.2.2, 4.11.2.3, and in accordance with the methodology and parameters in the ODCM Part II Section 3.0 at least once per year.
- 4.11.4.2 Cumulative dose contributions from direct radiation from the facility shall be determined in accordance with the methodology and parameters in the ODCM Part II Section 3.2. This requirement is applicable only under conditions set forth in CONTROL 3.11.4, ACTION a.

3/4.12 RADIOLOGICAL ENVIRONMENTAL MONITORING

3/4.12.1 MONITORING PROGRAM

CONTROLS

3.12.1. In accordance with Oyster Creek Technical Specifications 6.8.4.b, the radiological environmental monitoring program shall be conducted as specified in Table 3.12.1-1. For specific sample locations see Table E-1. Revisions to the non-ODCM required portions of the program may be implemented at any time. Non-ODCM samples are those taken in addition to the minimum required samples listed in Table 3.12.1-1.

APPLICABILITY: At all times.

ACTION:

- a. With the radiological environmental monitoring program not being conducted as specified in Table 3.12.1-1, prepare and submit to the Commission, in the Annual Radiological Environmental Operating Report required by Technical Specification 6.9.1.e, a description of the reasons for not conducting the program as required and the plans for preventing a recurrence.
- b. With the level of radioactivity as the result of plant effluents in an environmental sampling medium at a specified location exceeding the reporting levels of Table 3.12.1-2 when averaged over any calendar quarter, prepare and submit to the Commission within 60 days of the end of the quarter, pursuant to Technical Specification 6.9.2, a Special Report that identifies the cause(s) for exceeding the limit(s) and defines the corrective actions to be taken to reduce radioactive effluents so that the potential annual dose* to a MEMBER OF THE PUBLIC is less than the calendar year limits of CONTROLS 3.11.1.2, 3.11.2.2, and 3.11.2.3. When more than one of the radionuclides in Table 3.12.1-2 are detected in the sampling medium, this report shall be submitted if:

$$\frac{\text{concentration (1)}}{\text{reporting level (1)}} + \frac{\text{concentration (2)}}{\text{reporting level (2)}} + \dots \geq 1.0$$

When radionuclides other than those in Table 3.12.1-2 are detected and are the result of plant effluents, this report shall be submitted if the potential annual dose* to a MEMBER OF THE PUBLIC from all radionuclides is equal to or greater than the calendar year limits of CONTROLS 3.11.1.2, 3.11.2.2, and 3.11.2.3. This report is not required if the measured level of radioactivity was not the result of plant effluents; however, in such an event, the condition shall be reported and described in the Annual Radiological Environmental Operating Report pursuant to Section 6.1.2.1.

*The methodology used to estimate the potential annual dose to a MEMBER OF THE PUBLIC shall be indicated in this report.

3/4.12 RADIOLOGICAL ENVIRONMENTAL MONITORING

3/4.12.1 MONITORING PROGRAM

CONTROLS (Continued)

ACTION: (Continued)

- c. If garden vegetation samples are unobtainable due to any legitimate reason, it is NOT ACCEPTABLE to substitute vegetation from other sources. The missed sample will be documented in the annual report, with no further actions necessary. If a permanent sampling location becomes unavailable, follow Table 3.12.1-1 Table Notation (1) to replace the location.
- d. The provisions of CONTROLS 3.0.3, 3.0.4 and Technical Specification 6.9.2 are not applicable.

SURVEILLANCE REQUIREMENTS

- 4.12.1 The radiological environmental monitoring samples shall be collected pursuant to Table 3.12.1-1 from the specific locations given in Table E-1, and shall be analyzed pursuant to the requirements of Table 3.12.1-1, and the detection capabilities required by Table 4.12.1-1.

TABLE 3.12.1-1: RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM

<u>EXPOSURE PATHWAY AND/OR SAMPLE</u>	<u>NUMBER OF REPRESENTATIVE SAMPLES AND SAMPLE LOCATIONS⁽¹⁾</u>	<u>SAMPLING AND COLLECTION FREQUENCY</u>	<u>TYPE AND FREQUENCY OF ANALYSIS</u>
1. DIRECT RADIATION ⁽²⁾	<p>Routine monitoring stations with two or more dosimeters placed as follows:</p> <p>An inner ring of stations one in each meteorological sector in the general area of the SITE BOUNDARY (At least 16 locations);</p> <p>An outer ring of stations, one in each land-based meteorological sector in the approximately 6- to 8-km range from the site (At least 14 locations); and</p> <p>At least 8 stations to be placed in special interest areas such as population centers, nearby residences, schools, and in one or two areas to serve as control stations.</p>	Quarterly	Gamma dose quarterly

TABLE 3.12.1-1(Cont'd)

RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM

<u>EXPOSURE PATHWAY AND/OR SAMPLE</u>	<u>NUMBER OF REPRESENTATIVE SAMPLES AND SAMPLE LOCATIONS⁽¹⁾</u>	<u>SAMPLING AND COLLECTION FREQUENCY</u>	<u>TYPE AND FREQUENCY OF ANALYSIS</u>
2. AIRBORNE	<p>Samples from 5 locations:</p> <p>Three samples from close to the SITE BOUNDARY in different sectors of the highest calculated annual average ground-level D/Q</p> <p>One sample from the vicinity of a community having the highest calculated annual average ground-level D/Q; and</p> <p>One sample from a control location, as for example 15-30 km distant and in the least prevalent wind direction ⁽⁶⁾</p>	<p>Continuous sampler operation with sample collection weekly or more frequently if required by dust loading</p>	<p><u>Radioiodine Canister:</u> I-131 analysis weekly</p> <p><u>Particulate Sampler</u> Gross beta radioactivity analysis following filter change⁽³⁾;</p> <p>Gamma isotopic analysis⁽⁴⁾ of composites (by location) quarterly</p>

TABLE 3.12.1-1(Cont'd)

RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM

<u>EXPOSURE PATHWAY AND/OR SAMPLE</u>	<u>NUMBER OF REPRESENTATIVE SAMPLES AND SAMPLE LOCATIONS⁽¹⁾</u>	<u>SAMPLING AND COLLECTION FREQUENCY</u>	<u>TYPE AND FREQUENCY OF ANALYSIS</u>
3. WATERBORNE			
a. Surface	One sample upstream One sample downstream	Grab sample weekly, Combine into monthly composite.	Gamma isotopic and tritium analysis ⁽⁴⁾
b. Ground ⁽⁵⁾	Samples from one or two sources if likely to be affected	Grab sample quarterly	Gamma isotopic and tritium analysis ⁽⁴⁾
c. Drinking	1 sample of each of 1 to 3 of the nearest water supplies that could be affected by its discharge One sample from a background location	Grab sample weekly, combine into a 2-week composite if I-131 analysis is required; composite monthly otherwise	Gross beta, gamma isotopic and tritium analysis monthly ⁽⁴⁾⁽⁷⁾

TABLE 3.12.1-1(Cont'd)

RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM

<u>EXPOSURE PATHWAY AND/OR SAMPLE</u>	<u>NUMBER OF REPRESENTATIVE SAMPLES AND SAMPLE LOCATIONS⁽¹⁾</u>	<u>SAMPLING AND COLLECTION FREQUENCY</u>	<u>TYPE AND FREQUENCY OF ANALYSIS</u>
d. Sediment	One sample from downstream area with existing or potential recreational value	Semiannually	Gamma isotopic analysis ⁽⁴⁾ semiannually

TABLE 3.12.1-1(Cont'd)

RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM

<u>EXPOSURE PATHWAY AND/OR SAMPLE</u>	<u>NUMBER OF REPRESENTATIVE SAMPLES AND SAMPLE LOCATIONS⁽¹⁾</u>	<u>SAMPLING AND COLLECTION FREQUENCY</u>	<u>TYPE AND FREQUENCY OF ANALYSIS</u>
4. INGESTION			
a. Milk ⁽⁶⁾	<p>No milking animals</p> <p>If milk animals are identified: Samples from milking animals in three locations within 5km having the highest dose potential. If there are none, then one sample from milking animals in each of three areas between 5 an 8 km distant where doses are calculated to be greater than 1 mrem per year. One sample from milking animal at a control location 15 to 30 km distant and in the least prevalent wind direction</p>	<p>Semimonthly when on pasture; monthly at other times</p>	<p>Gamma isotopic ⁽⁴⁾ and Iodine -131 semimonthly when animals are on pasture; monthly at other times</p>
b. Fish	<p>One sample of available species consumed by man in plant discharge canal</p> <p>One sample of available species consumed by man not influenced by plant discharge</p>	<p>Semiannually, when available</p>	<p>Gamma isotopic analysis ⁽⁴⁾ on edible portions</p>

TABLE 3.12.1-1(Cont'd)

RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM

<u>EXPOSURE PATHWAY AND/OR SAMPLE</u>	<u>NUMBER OF REPRESENTATIVE SAMPLES AND SAMPLE LOCATIONS⁽¹⁾</u>	<u>SAMPLING AND COLLECTION FREQUENCY</u>	<u>TYPE AND FREQUENCY OF ANALYSIS</u>
c. Clams	<p>One sample of available species consumed by man within the influence of the facility discharge.</p> <p>One sample of available species consumed by man not influenced by plant discharge.</p>	Semiannually, when available	Gamma isotopic analysis ⁽⁴⁾ on edible portions.
d. Vegetation ⁽⁸⁾	<p>3 samples of broad leaf vegetation grown nearest each of two different Off Site locations of highest predicted annual average combined elevated and ground level release D/Q</p> <p>One sample of each of the similar broad leaf vegetation grown at least 15 to 30 km (9.3-18.6 miles) distant in the least prevalent wind direction.</p>	Monthly during growing season	Gamma isotopic analysis ⁽⁴⁾ and I-131 on edible portion.

TABLE 3.12.1-1 (Continued)

TABLE NOTATIONS

- 1 Specific parameters of distance and direction sector from the centerline of the reactor, and additional description where pertinent, are provided for each and every sample location in Table 3.12.1-1 and Table E-1. Deviations are permitted from the required sampling schedule if specimens are unobtainable due to circumstances such as hazardous conditions, seasonal unavailability, malfunction of automatic sampling equipment, and other legitimate reasons. If specimens are unobtainable due to sampling equipment malfunction, effort shall be made to complete corrective action prior to the end of the next sampling period. All deviations from the sampling schedule shall be documented in the Annual Radiological Environmental Operating Report pursuant to CONTROL 6.1.2.4. It is recognized that, at times, it may not be possible or practicable to continue to obtain samples of the media of choice at the most desired location or time. In these instances suitable specific alternative media and locations may be chosen for the particular pathway in question and appropriate substitutions made within 30 days in the Radiological Environmental Monitoring Program given in the ODCM. Pursuant to Technical specification 6.19, submit in the next Radioactive Effluent Release Report documentation for a change in the ODCM including revised figure(s) and table for the ODCM reflecting the new location(s) with supporting information identifying the cause of the unavailability of samples for the pathway and justifying the selection of the new location(s) for obtaining samples. This applies to changes/deletions/additions of permanent sampling locations. This does not apply to one-time deviations from the sampling schedule. In those cases, it is NOT ACCEPTABLE to substitute sample media from other sources. The missed sample will be documented in the annual report, with no further actions necessary.
- 2 One or more instruments, such as pressurized ion chamber, for measuring and recording dose rate continuously may be used in place of, or in addition to, integrating dosimeters. The number of direct radiation monitoring stations has been reduced from the NUREG 1302 recommendation due to geographical limitations; e.g., some sectors are over water and some sectors cannot be reached due to lack of highway access, therefore the number of dosimeters has been reduced accordingly.
- 3 Airborne particulate sample filters shall be analyzed for gross beta radioactivity 24 hours or more after sampling to allow for radon and thoron daughter decay. If gross beta activity in air particulate is greater than ten times the yearly mean of control samples, gamma isotopic analysis shall be performed on the individual samples.
- 4 Gamma isotopic analysis means the identification and quantification of gamma-emitting radionuclides that may be attributable to the effluents from the facility.
- 5 Groundwater samples shall be taken when this source is tapped for drinking or irrigation purposes in areas where the hydraulic gradient or recharge properties are suitable for contamination. Extensive studies of geology and groundwater in the vicinity of the OCGS (Reference 21 and 31) have demonstrated that there is no plausible pathway for effluents from the facility to contaminate offsite groundwater, including the local

drinking water supplies. Samples of groundwater, including local drinking water wells, are collected in order to provide assurance to the public that these water resources are not impacted.

- 6 The purpose of this sample is to obtain background information. If it is not practical to establish control locations in accordance with the distance and wind direction criteria, other sites which provide valid background data may be substituted
- 7 I-131 analysis on each composite when the dose calculated for the consumption of the water is greater than 1 mrem per year. If garden vegetation samples are unobtainable due to any legitimate reason (see (1) above), it is NOT ACCEPTABLE to substitute vegetation from other sources. The missed sample will be documented in the annual report, with no further actions necessary.

TABLE 3.12.1-2: REPORTING LEVELS FOR RADIOACTIVITY CONCENTRATIONS IN ENVIRONMENTAL SAMPLES - REPORTING LEVELS

Analysis	Surface and Ground Water(pCi/l)	Airborne Particulate and Iodine (pCi/m ³)	Fish (pCi/Kg, wet)	Milk (pCi/l)	Vegetation (pCi/Kg, wet)
H-3	20000*				
Mn-54	1000		30000		
Fe-59	400		10000		
Co-58	1000		30000		
Co-60	300		10000		
Zn-65	300		20000		
Zr-Nb-95	400				
I-131	2**	0.9		3	100
Cs-134	30	10	1000	60	1000
Cs-137	50	20	2000	70	2000
Ba-La-140	200			300	

*For drinking water samples (this is the 40 CFR Part 141 value).
If no drinking water pathway exists, a value of 30,000 pCi/L may be used.

**If no drinking water pathway exists, a value of 20 pCi/L may be used.

TABLE 4.12.1-1: DETECTION CAPABILITIES FOR ENVIRONMENTAL SAMPLE ANALYSIS^{(1),(2)} - LOWER LIMITS OF DETECTION (LLD)⁽³⁾

Analysis	Surface and Ground Water (pCi/l)	Air Particulate and Air Iodine (pCi/m ³)	Vegetation (pCi/Kg, wet)	Sediment (pCi/Kg, dry)	Fish, Clams and Crabs (pCi/Kg, wet)
Gross Beta	4	0.01			
H-3	2000 ⁽⁴⁾				
Mn-54	15				130
Fe-59	30				260
Co-58, 60	15				130
Zn-65	30				260
Zr-95	30				
Nb-95	15				
I-131	1 ⁽⁴⁾	0.07 ⁽⁵⁾	60		
Cs-134	15	0.05 ⁽⁶⁾	60	150	130
Cs-137	18	0.06 ⁽⁶⁾	80	180	150
La-140	15				
Ba-140	60 ⁽⁷⁾				

TABLE 4.12.1-1 (Continued)

TABLE NOTATIONS

1. This list does not mean that only these nuclides are to be considered. Other peaks that are identifiable, together with those of the above nuclides, shall also be analyzed and reported in the Annual Radiological Environmental Operating Report pursuant to CONTROL 6.1.2.3.
2. Required detection capabilities for dosimeters used for environmental measurements shall be in accordance with the recommendations of Regulatory Guide 4.13.
3. The LLD is defined, for purposes of these CONTROLS as the smallest concentration of radioactive material in a sample that will yield a net count, above system background, that will be detected with 95% probability with only 5% probability of falsely concluding that a blank observation represents a "real" signal.

For a particular measurement system, which may include radiochemical separation:

$$LLD = \frac{4.66 * Sb}{E * V * 2.22 * Y * \exp(-\lambda\Delta t)}$$

Where:

LLD is the "a priori" lower limit of detection as defined above, as picocuries per unit mass or volume,

Sb is the standard deviation of the background counting rate or of the counting rate of a blank sample as appropriate, as counts per minute,

E is the counting efficiency, as counts per disintegration,

V is the sample size in units of mass or volume,

2.22 is the number of disintegrations per minute per Pico curie,

Y is the fractional radiochemical yield, when applicable,

λ is the radioactive decay constant for the particular radionuclide (sec^{-1}), and

Δt for environmental samples is the elapsed time between sample collection, or end of the sample collection period, and time of counting (sec).

Typical values of E, V, Y, and Δt should be used in the calculation.

TABLE 4.12.1-1 (Continued)

TABLE NOTATIONS

It should be recognized that the LLD is defined as an a priori (before the fact) limit representing the capability of a measurement system and not as an a posteriori (after the fact) limit for a particular measurement. Analyses shall be performed in such a manner that the stated LLDs will be achieved under routine conditions. Occasionally, background fluctuations, unavoidable small sample sizes, the presence of interfering nuclides, or other uncontrollable circumstances may render these LLDs unachievable. In such cases, the contributing factors shall be identified and described in the Annual Radiological Environmental Operating Report pursuant Technical Specification 6.9.1.e and Control 6.1.2.6.4.

4. If no drinking water pathway exists, a value of 3000 pCi/L for tritium and 15 pCi/L for iodine-131 may be used.
5. For the air iodine sample
6. For the air particulate sample
7. Ba-140 and La-140 are in equilibrium

3/4.12 RADIOLOGICAL ENVIRONMENTAL MONITORING

3/4.12.2 LAND USE CENSUS

CONTROLS

3.12.2 In accordance with the Oyster Creek Technical Specifications 6.8.4.b, a land use census shall be conducted and shall identify within a distance of 5 miles the location in each of the 16 meteorological sectors of the nearest milk animal, the nearest residence and the nearest garden* of greater than 500 ft² producing broad leaf vegetation. The census shall also identify within a distance of 3 miles the location in each of the 16 meteorological sectors all milk animal and all gardens greater than 500 square feet producing broadleaf vegetation.

APPLICABILITY: At all times.

ACTION:

- a. With a land use census identifying a location(s) that yields a calculated dose or dose commitment greater than the values currently being calculated in SURVEILLANCE REQUIREMENT 4.11.2.3, identify the new location(s) in the next Radioactive Effluent Release Report, pursuant to Control 6.2.2.4.
- b. With a land use census identifying a location(s) that yields a calculated dose or dose commitment (via the same exposure pathway) 20 percent greater than at a location from which samples are currently being obtained in accordance with CONTROL 3.12.1, add the new location(s) to the radiological environmental monitoring program within 30 days. The sampling location(s), excluding the control station location, having the lowest calculated dose or dose commitment(s), via the same exposure pathway, may then be deleted from this monitoring. Pursuant to CONTROL 6.2.2.4, identify the new location(s) in the next Radioactive Effluent Release Report and also include in the report a revised figure(s) and table for the ODCM reflecting the new location(s).
- c. The provisions of CONTROLS 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.12.2 The land use census shall be conducted during the growing season at least once per 12 months using that information that will provide the best results, such as by door-to-door survey, visual survey, aerial survey, or by consulting local agriculture authorities. The results of the land use census shall be included in the Annual Radiological Environmental Operating Report pursuant to CONTROL 6.1.2.2.

*Broad leaf vegetation sampling of at least three different kinds of vegetation may be performed at the SITE BOUNDARY in each of two different direction sectors with the highest predicted elevated release D/Q's in lieu of the garden census. Controls for broadleaf vegetation sampling in Table 3.12.1-1, Part 4.c shall be followed, including analysis of control samples.

3/4.12 RADIOLOGICAL ENVIRONMENTAL MONITORING

3/4.12.3 INTERLABORATORY COMPARISON PROGRAM

CONTROLS

3.12.3 In accordance with Oyster Creek Technical Specifications 6.8.4.b.3, analyses shall be performed on radioactive materials supplied as part of an interlaboratory comparison program which has been approved by the Commission.

APPLICABILITY: At all times.

ACTION:

- a. With analyses not being performed as required above, report the reason and corrective actions taken to prevent a recurrence to the Commission in the Annual Radiological Environmental Operating Report pursuant to CONTROL 6.1.2.6.3.
- b. The provisions of CONTROLS 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.12.3 A summary of the results obtained as part of the above-required Interlaboratory Comparison Program shall be included in the Annual Radiological Environmental Operating Report pursuant to CONTROL 6.1.2.6.3.

3 /4 .12 RADIOLOGICAL ENVIRONMENTAL MONITORING

3/4.12.4 METEOROLOGICAL MONITORING PROGRAM

CONTROLS

3.12.4 The meteorological monitoring instrumentation channels shown in Table 3.12.4.-1 shall be operable.

APPLICABILITY: At all times.

ACTION:

- a. With less than the minimum required instrumentation channels OPERABLE for more than 7 days, initiate an Issue Report outlining the cause of the malfunction and the plans for restoring the instrumentation to OPERABLE status.
- b. The provisions of CONTROLS 3.0.3 and 3.0.4 are not applicable.

TABLE 3.12.4-1

METEOROLOGICAL MONITORING INSTRUMENTATION

INSTRUMENT	ELEVATION	MINIMUM INSTRUMENT OPERABLE
1. Wind Speed		
a.	380 feet	1
b.	150 feet	1
c.	33 feet	1
2. Wind Direction		
a.	380 feet	1
b.	150 feet	1
c.	33 feet	1
3. ΔT		
a.	380-33	1
b.	150-33	1

BASES FOR SECTIONS 3.0 AND 4.0

CONTROLS AND SURVEILLANCE REQUIREMENTS

NOTE: The BASES contained in the succeeding pages summarize the reasons for the CONTROLS of Sections 3.0 and 4.0, but are not considered a part of these CONTROLS.

3/4.3 INSTRUMENTATION

BASES

3/4.3.3.10 RADIOACTIVE LIQUID EFFLUENT MONITORING INSTRUMENTATION

The radioactive liquid effluent instrumentation is provided to monitor and control, as applicable, the releases of radioactive materials in liquid effluents during actual or potential releases of liquid effluents. The reactor service water system discharge line radioactivity monitor initiates an alarm in the Control Room when the alarm set point is exceeded. The alarm/trip set points for these instruments shall be calculated and adjusted in accordance with the methodology and parameters in the ODCM to ensure that the alarm/trip will occur prior to exceeding the limits of 10 CFR Part 20. The OPERABILITY and use of this instrumentation is consistent with the requirements of General Design Criteria 60, 63, and 64 of Appendix A to 10 CFR Part 50.

3/4.3.3.11 RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION

The radioactive gaseous effluent instrumentation is provided to monitor and control, as applicable, the releases of radioactive materials in gaseous effluents during actual or potential releases of gaseous effluents. The alarm/trip set points for each of the noble gas monitors shall be calculated and adjusted in accordance with the methodology and parameters in the ODCM. This will ensure the alarm/trip will occur prior to exceeding the limits of 10 CFR Part 20. The radioactive gas monitors for the stack effluent and the Augmented Off gas Building exhaust ventilation have alarms which report in the Reactor Control Room. The OPERABILITY and use of this instrumentation is consistent with the requirements of General Design Criteria 60, 63, and 64 of Appendix A to 10 CFR Part 50.

3/4.11 RADIOACTIVE EFFLUENTS

BASES

3/4.11.1 LIQUID EFFLUENTS

3/4.11.1.1 CONCENTRATION

This CONTROL is provided to ensure that the concentration of radioactive materials released in liquid waste effluents to UNRESTRICTED AREAS will be less than the concentration levels specified in 10 CFR Part 20, Appendix B, Table 2, Column 2. This limitation provides additional assurance that the levels of radioactive materials in bodies of water in UNRESTRICTED AREAS will result in exposures within (1) the Section II.A design objectives of Appendix I, 10 CFR Part 50, to a MEMBER OF THE PUBLIC and (2) the limits of 10 CFR Part 20.106(a) to the population. The concentration limit for dissolved or entrained noble gases is based upon the assumption that Xe-135 is the controlling radioisotope and its concentration limit in air (submersion) was converted to an equivalent concentration in water using the methods described in International Commission on Radiological Protection (ICRP) Publication 2.

The value 1E-8 is the limit for unidentified gross gamma or beta releases as per 10 CFR 20 Appendix B, Table 2, Column 2 "any single radionuclide... other than alpha or spontaneous fission ... half life greater than 2 hours". This provides operational flexibility while providing reasonable assurance that dose will remain less than 0.1 rem/yr.

The required detection capabilities for radioactive materials in liquid waste samples are tabulated in terms of the lower limits of detection (LLDs). Detailed discussion of the LLD, and other detection limits can be found in references 25, 26, and 27.

Weekly grab samples for Service Water Effluent are composited for monthly tritium and gross alpha analysis and quarterly Sr-89/90 and Fe-55 analysis if activity is detected.

While discharging groundwater via the continuous release remediation pathway, the composite sample will be analyzed daily for principal gamma emitters and tritium. A monthly composite sample is analyzed for gross alpha, Sr-89/90, Fe-55, and Ni-63.

While discharging groundwater via the batch release mode remediation pathway, a grab sample is collected from each tank and analyzed for principal gamma emitters and tritium, a composite sample is analyzed monthly for gross alpha, Sr-89/90, Fe-55, and Ni-63.

Circulating Water Effluent is not included in Table 4.11.1.1.1-1, Radioactive Liquid Waste Sampling and Analysis Program since the Circulating Water is sampled as part of the Radiological Environmental Monitoring Program, Table 3.12.1-1, 3a, Waterborne, Surface downstream sample.

3/4.11.1.2 DOSE

This CONTROL is provided to implement the requirements of Sections II.A, III.A, and IV.A of Appendix I, 10 CFR Part 50. The CONTROL implements the guides set forth in Section II.A of Appendix I. The ACTION statements provide the required operating flexibility and at the same time implement the guides set forth in Section IV.A of Appendix I to assure that the releases of radioactive material in liquid effluents to UNRESTRICTED AREAS will be kept "as low as is reasonably achievable." The dose calculation methodology and parameters in the ODCM implement the requirements in Section III.A of Appendix I that conformance with the guides of Appendix I be shown by calculational procedures based on models and data, such that the actual exposure of a MEMBER OF THE PUBLIC through appropriate pathways is unlikely to be substantially underestimated. The equations specified in the ODCM for calculating the doses due to the actual release rates of radioactive materials in liquid effluents are consistent with the methodology provided in Regulatory Guide 1.109 and Regulatory Guide 1.113.

3/4.11.1.3 LIQUID RADWASTE TREATMENT

The OPERABILITY of the liquid radwaste treatment system ensures that this system will be available for use whenever liquid effluents require treatment prior to their release to the environment. The requirement that the appropriate portions of this system be used, when specified, provides assurance that the releases of radioactive materials in liquid effluents will be kept "as low as is reasonably achievable". This CONTROL implements the requirements of General Design Criterion 60 of Appendix A to 10 CFR Part 50 and the design objective given in Section II.D of Appendix I to 10 CFR Part 50. The specified limits governing the use of appropriate portions of the liquid radwaste treatment system were specified as a suitable fraction of the dose design objectives set forth in Section II.A of Appendix I, 10 CFR Part 50, for liquid effluents. Figure D-1-1a, Liquid Radwaste Treatment Chem Waste and Floor Drain System and Figure D-1-1b, Liquid Radwaste Treatment – High Purity and Equipment Drain System provides details of the Liquid Radwaste Treatment system.

3/4.11.2 GASEOUS EFFLUENTS

3/4.11.2.1 DOSE RATE

This CONTROL is provided to ensure that the dose at any time at and beyond the SITE BOUNDARY from gaseous effluents will be within the annual dose limits of 10 CFR Part 20 to UNRESTRICTED AREAS. The annual dose limits are the doses associated with the concentrations of 10 CFR Part 20, Appendix B, Table 2, Column 1. These limits provide reasonable assurance that radioactive material discharged in gaseous effluents will not result in the exposure of a MEMBER OF THE PUBLIC in an UNRESTRICTED AREA either within or outside the SITE BOUNDARY, to annual average concentrations exceeding the limits specified in Appendix B, Table 2 of 10 CFR Part 20. For MEMBERS OF THE PUBLIC who may at times be within the SITE BOUNDARY, the occupancy of the individual will usually be sufficiently low to

compensate for any increase in the atmospheric diffusion factor above that for the SITE BOUNDARY. Examples of calculations for such MEMBERS OF THE PUBLIC with the appropriate occupancy factors shall be given in the ODCM. The specified release rate limits restrict, at all times, the corresponding gamma and beta dose rates above background to a MEMBER OF THE PUBLIC at or beyond the SITE BOUNDARY to less than or equal to 500 mrem/year to the total body or to less than or equal to 3000 mrem/yr to the skin. These release rate limits also restrict, at all times, the corresponding thyroid dose rate above background to a child via the inhalation pathway to less than or equal to 1500 mrem/year.

The required detection capabilities for radioactive materials in liquid waste samples are tabulated in terms of the lower limits of detection (LLDs). Detailed discussion of the LLD, and other detection limits can be found in references 25, 26 and 27.

Tritium is sampled quarterly for gaseous effluents. Based on the consistency of the data from the quarterly sampling, the sampling frequency is adequate.

3/4.11.2.2 DOSE - NOBLE GASES

This CONTROL is provided to implement the requirements of Section II.B, III.A and IV.A of Appendix I, 10 CFR Part 50. The CONTROL implements the guides set forth in Section II.B of Appendix I. The ACTION statements provide the required operating flexibility and at the same time implement the guides set forth in Section IV.A of Appendix I to assure that the releases of radioactive material in gaseous effluents will be kept "as low as is reasonably achievable." The SURVEILLANCE REQUIREMENTS implement the requirements in Section III.A of Appendix I that conformance with the guides of Appendix I be shown by calculational procedures based on models and data such that the actual exposure of a MEMBER OF THE PUBLIC through appropriate pathways is unlikely to be substantially underestimated. The dose calculation methodology and parameters established in the ODCM for calculating the doses due to the actual release rates of radioactive noble gases in gaseous effluents are consistent with the methodology provided in Regulatory Guide 1.109 and Regulatory Guide 1.111. The ODCM equations provided for determining the air doses at and beyond the SITE BOUNDARY are based upon the historical average atmospheric conditions.

3/4.11.2.3 DOSE - IODINE-131, IODINE-133, TRITIUM, AND RADIONUCLIDES IN PARTICULATE FORM

This CONTROL is provided to implement the requirements of Section II.C, III.A and IV.A of Appendix I, 10 CFR Part 50. The CONTROLS are the guides set forth in Section II.C of Appendix I. The ACTION statements provide the required operating flexibility and at the same time implement the guides set forth in Section IV.A of Appendix I to assure that the releases of radioactive material in gaseous effluents to UNRESTRICTED AREAS will be kept "as low as is reasonably achievable." The ODCM calculational methods specified in SURVEILLANCE REQUIREMENTS implement the requirements in Section III.A of Appendix I that conformance with the guides of Appendix I be shown by calculational procedures based on models and data

such that the actual exposure of a MEMBER OF THE PUBLIC through appropriate pathways is unlikely to be substantially underestimated. The ODCM calculational methodology and parameters for calculating the doses due to the actual release rates of the subject materials are consistent with the methodology provided in Regulatory Guide 1.109, and Regulatory Guide 1.111. These equations also provide for determining the actual doses based upon the historical average atmospheric conditions. The release rate controls for iodine-131, iodine-133, tritium, and radionuclides in particulate form with half-life greater than 8 days are dependent on the existing radionuclide pathways to man, in the areas at and beyond the SITE BOUNDARY. The pathways that were examined in the development of these calculations were: 1) individual inhalation of airborne radionuclides, 2) deposition of radionuclides onto green leafy vegetation with subsequent consumption by man, 3) deposition onto grassy areas where milk animals and meat producing animals graze with consumption of the milk and meat by man, and 4) deposition on the ground with subsequent exposure of man.

3/4.11.2.4 GASEOUS RADWASTE TREATMENT SYSTEM

The OPERABILITY of the AUGMENTED OFF GAS TREATMENT SYSTEM (AOG) ensures that the system will be available for use whenever gaseous effluents require treatment prior to release to the environment. The requirement that the appropriate portions of these systems be used, when specified, provides reasonable assurance that the releases of radioactive materials in gaseous effluents will be kept "as low as is reasonably achievable." This CONTROL implements the requirements of General Design Criterion 60 of Appendix A to 10 CFR Part 50, and the design objectives given in Section II.D of Appendix I to 10 CFR Part 50. The specified limits governing the use of appropriate portions of the systems were specified as a suitable fraction of the dose design objectives set forth in Section II.B and II.C of Appendix I, 10 CFR Part 50, for gaseous effluents. Figure D-2-1, Gaseous Radwaste Treatment – Augmented Off gas System, Figure D-2-2, Ventilation System provide details of the Augmented Off Gas Treatment System and Figure D-2-3, AOG Ventilation System.

3/4.11.4 TOTAL DOSE

This CONTROL is provided to meet the dose limitations of 40 CFR Part 190 that have now been incorporated into 10 CFR Part 20 by 46 FR 18525. The CONTROL requires the preparation and submittal of a Special Report whenever the calculated doses from plant radioactive effluents exceed 25 mrem to the total body or any organ, except the thyroid, which shall be limited to less than or equal to 75 mrem. It is highly unlikely that the resultant dose to a MEMBER OF THE PUBLIC will exceed the dose limits of 40 CFR Part 190 if the doses remain within twice the dose design objectives of Appendix I, and if direct radiation doses from the unit, including outside storage tanks, etc. are kept small. The Special Report will describe a course of action that should result in the limitation of the annual dose to a MEMBER OF THE PUBLIC to within the 40 CFR Part 190 limits. For purposes of the Special Report, it may be assumed that the dose commitment to the MEMBER OF THE PUBLIC from other uranium fuel cycle sources is negligible. If the dose to any MEMBER OF THE PUBLIC is estimated to exceed the requirements of 40 CFR Part 190, the Special Report with a request for a

variance (provided the release conditions resulting in violation of 40 CFR Part 190 have not already been corrected), in accordance with the provisions of 40 CFR Part 190 and 10 CFR Part 20, is considered to be a timely request and fulfills the requirements of 40 CFR Part 190 until NRC staff action is completed. The variance only relates to the limits of 40 CFR Part 190, and does not apply in any way to the other requirements for dose limitation of 10 CFR Part 20, as addressed in CONTROLS 3.11.1.1 and 3.11.2.1. An individual is not considered a MEMBER OF THE PUBLIC during any period in which he/she is engaged in carrying out any operation that is part of the nuclear fuel cycle.

3/4.12 RADIOLOGICAL ENVIRONMENTAL MONITORING

BASES

3/4.12.1 MONITORING PROGRAM

The radiological environmental monitoring program required by this CONTROL provides representative measurements of radiation and of radioactive materials in those exposure pathways and for those radionuclides that lead to the highest potential radiation exposures of MEMBERS OF THE PUBLIC resulting from the station operation. This monitoring program implements Section IV.B.2 of Appendix I to 10 CFR Part 50 and thereby supplements the radiological effluent monitoring program by verifying that the measurable concentrations of radioactive materials and levels of radiation are not higher than expected on the basis of the effluent measurements and the modeling of the environmental exposure pathways. Guidance for this monitoring program is provided by the Radiological Assessment Branch Position on Environmental Monitoring, Revision 1, November 1979.

The required detection capabilities for environmental sample analyses are tabulated in terms of the lower limits of detection (LLDs). The LLDs required by Table 4.12.1-1 are considered optimum for routine environmental measurements in industrial laboratories. It should be recognized that the LLD is defined as an a priori (before the fact) limit representing the capability of a measurement system and not as an a posteriori (after the fact) limit for a particular measurement.

Detailed discussion of the LLD, and other detection limits in references 25, 26, and 27.

Site-specific research, which included the installation of a groundwater monitoring well network, has demonstrated that the groundwater pathway is not a potential pathway to man from the OCGS. The surface water into which the OCGS discharges is a marine estuary containing saline water that is not used as drinking water or irrigation water by man.

3/4.12.2 LAND USE CENSUS

This CONTROL is provided to ensure that changes in the use of areas at and beyond the SITE BOUNDARY are identified and that modifications to the radiological environmental monitoring program are made if required by the results of this census. The best information from the door-to-door survey, from aerial survey, from visual survey or consulting with local agricultural authorities shall be used. This census satisfies the requirements of Section IV.B.3 of Appendix I to 10 CFR Part 50. Restricting the census to gardens of greater than 50 m² (500 ft²) provides assurance that significant exposure pathways via leafy vegetables will be identified and monitored since a garden of this size is the minimum required to produce the quantity (26 kg/year) of leafy vegetables assumed in Regulatory Guide 1.109 for consumption by a child. To determine this minimum garden size, the following assumptions were made: 1) 20% of the garden was used for growing broad leaf vegetation (i.e., similar to lettuce and cabbage), and 2) a vegetation yield of 2 kg/m².

3/4.12 RADIOLOGICAL ENVIRONMENTAL MONITORING

BASES

3/4.12.3 INTERLABORATORY COMPARISON PROGRAM

The requirement for participation in an approved Interlaboratory Comparison Program is provided to ensure that independent checks on the precision and accuracy of the measurements of radioactive material in environmental sample matrices are performed as part of the quality assurance program for environmental monitoring in order to demonstrate that the results are valid for the purposes of Section IV.B.2 of Appendix I to 10 CFR Part 50.5.0

5.0 DESIGN FEATURES / SITE MAP

(Provided FOR INFORMATION ONLY. Technical Specifications are controlling.)

- 5.1 Site map which will allow identification of structures and release points shall be as shown in Figure E-4.

6.0 ADMINISTRATIVE CONTROLS

6.1 ANNUAL RADIOLOGICAL ENVIRONMENTAL OPERATING REPORT (AREOR)

6.1.1 In accordance with Oyster Creek Technical Specifications 6.9.1.e, a routine radiological environmental operating report covering the operation of the unit during the previous calendar year shall be submitted prior to May 1 of the following year.

6.1.2 The Annual Radiological Environmental Operating Reports shall include:

6.1.2.1 Summaries, interpretations, and an analysis of trends of the results of the radiological environmental surveillance activities (Radiological Environmental Monitoring Program – REMP) for the report period. This will include a comparison with preoperational studies, with operational controls (as appropriate), and with previous environmental surveillance reports, and an assessment of the observed impacts of the plant operation on the environment.

6.1.2.2 The reports shall also include the results of land use censuses required by CONTROL 3.12.2. If harmful effects or evidence of irreversible damage are detected by the monitoring, the report shall provide an analysis of the problem and a planned course of action to alleviate the problem.

6.1.2.3 The Annual Radiological Environmental Operating Reports shall include summarized and tabulated results similar in format to that in Regulatory Guide 4.8, December 1975 of all the radiological environmental samples taken during the report period.

6.1.2.4 Deviations from the sampling program identified in CONTROL 3.12.1 shall be reported.

6.1.2.5 In the event that some individual results are not available for inclusion with the report, the report shall be submitted noting and explaining the reasons for the missing results. The missing data shall be submitted as soon as possible in a supplementary report.

6.1.2.6 The reports shall also include the following:

- a. A summary description of the radiological environmental monitoring Program;
- b. Map(s), covering sampling locations, keyed to a table giving distances and directions from the reactor;
- c. The results of licensee participation in the Inter-laboratory Comparison Program, as required by CONTROL 3.12.3;
- d. Identification of environmental samples analyzed when the analysis instrumentation was not capable of meeting the detection capabilities in Table 4.12.1-1.

- 6.2 ANNUAL ROUTINE RADIOACTIVE EFFLUENT RELEASE REPORT (ARERR)
- 6.2.1 Routine radioactive effluent release reports covering the operation of the unit shall be submitted prior to May 1 of each year and in accordance with the requirements of 10CFR50.36a and section IV.B.1 of 10CFR 50 Appendix I.
- 6.2.2 The Radioactive Effluent Release Report shall include:
- 6.2.2.1 A summary of the quantities of radioactive liquid and gaseous effluents and solid waste released from the unit as outlined in Regulatory Guide 1.21. "Measuring, Evaluating, and Reporting Radioactivity in Solid Wastes and Releases of Radioactive Materials in Liquid and Gaseous Effluents from Light-Water-Cooled Nuclear Power Plants," Revision 1, June 1974, with data summarized on a quarterly basis following the format of Appendix B thereof.
- 6.2.2.2 An annual summary of hourly meteorological data collected over the previous year. This annual summary may be in the form of joint frequency distributions of wind speed, wind direction, and atmospheric stability. Alternatively, summary meteorological data may be retained and made available to the NRC upon request.
- 6.2.2.3 An assessment of the radiation doses due to the radioactive liquid and gaseous effluents released from the unit or station during the previous calendar year. The historical annual average meteorology or the meteorological conditions concurrent with the time of release of radioactive materials in gaseous effluents (as determined by sampling frequency and measurement) shall be used for determining the gaseous pathway doses. The assessment of radiation doses shall be performed in accordance with this OFF SITE DOSE CALCULATION MANUAL (ODCM).
- 6.2.2.4 Identify those radiological environmental sample parameters and locations where it is not possible or practicable to continue to obtain samples of the media of choice at the most desired location or time. In addition, the cause of the unavailability of samples for the pathway and the new location(s) for obtaining replacement samples should be identified. The report should also include a revised figure(s) and table(s) for the ODCM reflecting the new location(s).
- 6.2.2.5 An assessment of radiation doses to the likely most exposed MEMBER OF THE PUBLIC from reactor releases and other nearby uranium fuel cycle sources (including doses from primary effluent pathways and direct radiation) for the previous calendar year to show conformance with 40 CFR Part 190, Environmental Radiation Protection Standards for Nuclear Power Operation. The assessment of radiation doses shall be performed in accordance with this OFF SITE DOSE CALCULATION MANUAL (ODCM) Part II Sections 1.5, 2.4, 2.5 and 3.2.
- 6.2.2.6 The Radioactive Effluent Release Reports shall include the following information for each class of solid waste (as defined by 10 CFR Part 61) shipped Off Site during the report period (see Figure D-1-2):
- a. Total volume shipped
 - b. Total curie quantity (specify whether determined by measurement or estimate),
 - c. Principal radionuclides (specify whether determined by measurement or estimate),

- d. Type of waste (e.g., dewatered spent resin, compacted dry waste, evaporator bottoms)
- 6.2.2.7 Unplanned releases from the site to UNRESTRICTED AREAS of radioactive materials in gaseous and liquid effluents on a quarterly basis.
- 6.2.2.8 Changes to the PROCESS CONTROL PROGRAM (PCP)
- 6.2.2.9 Changes to the OFF SITE DOSE CALCULATION MANUAL (ODCM) in the form of a complete, legible copy of the ODCM.
- 6.3 RESPONSIBILITIES:
 - 6.3.1 Chemistry / Radwaste - Responsible for:
 - 6.3.1.1 Implementing approval.
 - 6.3.1.2 Compliance with specifications regarding routine dose assessment.
 - 6.3.1.3 Radiological Environmental Monitoring Program
 - 6.3.1.4 Technical consultation and review
 - 6.3.2 Operations - Responsible for compliance with specifications regarding operation of the OCGS.
 - 6.3.3 Engineering - Responsible for compliance with specifications regarding set point determination and implementation
 - 6.3.4 Radiological Engineering - Responsible for technical consultation and review.

PART II - CALCULATIONAL METHODOLOGIES

1.0 LIQUID EFFLUENTS

1.1 RADIATION MONITORING INSTRUMENTATION AND CONTROLS

The liquid effluent monitoring instrumentation and controls at Oyster Creek for controlling and monitoring normal radioactive material releases in accordance with the Oyster Creek Radiological Effluent Technical Specifications are summarized as follows:

Reactor Building Service Water Effluent - The Reactor Building Service Water Effluent Line Monitor provides an alarm function only for releases into the environment.

Liquid radioactive waste flow diagrams are presented in Figures D-1-1a, D-1-1b, and D-1-1c.

1.2 LIQUID EFFLUENT MONITOR SET POINT DETERMINATION

Per the requirements of CONTROL 3.3.3.10, alarm set points shall be established for the liquid monitoring instrumentation to ensure that the release concentration limits of CONTROL 3.11.1.1 are met (i.e., the concentration of radioactive material released in liquid effluents to UNRESTRICTED AREAS at the U.S. route 9 bridge over the discharge canal shall not exceed the concentrations specified in 10 CFR 20 Appendix B, Table 2, Column 2, for radionuclides and 2E-04 $\mu\text{Ci/ml}$ for dissolved or entrained noble gases).

1.2.1 LIQUID EFFLUENT MONITORS

The set points for the liquid effluent monitors at the Oyster Creek Generating Station are determined by the following equation:

$$S = \frac{A}{FLEC} g \frac{F_2}{F_1} + BKG$$

Where:

S = radiation monitor alarm set point (cpm)

A = activity concentration ($\mu\text{Ci/ml}$) of sample in laboratory: $A = \sum C_i$

g = the primary conversion factor for the instrument – the ratio of effluent radiation monitor counting rate to laboratory activity concentration in a sample of liquid (cpm per $\mu\text{Ci/mL}$).

F₁ = flow in the batch release line (e.g. gal/min). Value not greater than the discharge line flow alarm maximum set point.

- F_2 = flow in the discharge canal (e.g. gal/min). Value not less than the discharge canal minimum flow.
- BKG = Monitoring instrument background (cpm)
- FLEC = fraction or multiple of unrestricted area LEC in aqueous effluent based on sample analysis. FLEC is the ratio between the LEC_i and C_i . FLEC is unitless. For example: LEC for Co-60 is $3E-6 \mu\text{Ci/mL}$. If the concentration in a expected release is $6E-6 \mu\text{Ci/mL}$; then FLEC is $6E-6/3E-6 = 2$.
The term $\frac{A}{FLEC}$ represents the count rate of a solution having the same nuclide distribution as the sample and the LEC of that mixture.
- C_i = concentration of radionuclide i in effluent, i.e., in a liquid radwaste sample tank, in reactor building service water ($\mu\text{Ci/mL}$).
- LEC_i = the unrestricted area liquid effluent concentration (LEC) of radionuclide i , i.e., 10 CFR 20, Appendix B, Table 2, Column 2 quantity for radionuclide i ($\mu\text{Ci/mL}$).
- In the event gross radioactivity analysis alone is used to determine the radioactivity in an effluent stream or batch, FLEC is $C/1E-8$ (see 4.11.1.1.1),

Where:

- C = the gross radioactivity concentration in effluent ($\mu\text{Ci/mL}$).
- $1E-8$ = the unrestricted area LEC for unidentified radionuclides ($\mu\text{Ci/mL}$) from 4.11.1.1.1.

If the gross activity concentration, C , is below the lower limit of detection for gross activity, the value, $1E-8 \mu\text{Ci/mL}$, or the equivalent counting rate (cpm/mL) may be substituted for the factor $\frac{A}{FLEC}$

$$\frac{A}{FLEC} = 1E-8 \mu\text{Ci/mL}$$

1.2.2 SAMPLE RESULT SET POINTS

Usually, when the concentration of specific radionuclides is determinable in a sample(s), i.e., greater than the LLD, the alarm/trip set point of each liquid effluent radioactivity monitor is based upon the measurement of radioactive material in a batch of liquid to be released or in a continuous aqueous discharge.

1.2.3 ASSUMED DISTRIBUTION SET POINTS

Alternatively, a radionuclide distribution that represents the distribution expected to be in the effluent if the concentration were high enough to be detectable, i.e., greater than the LLD, may be assumed. The representative distribution may be based upon past measurements of the effluent stream or upon a computed distribution.

1.3 BATCH RELEASES

A sample of each batch of liquid radwaste is analyzed for I-131 and other principal gamma emitters or for gross beta or gross gamma activity before release. The result of the analysis is used to calculate the trip set point of the radioactivity monitor on the liquid radwaste effluent line to apply to release of the batch.

1.4 CONTINUOUS RELEASES

The Reactor Building Service Water Effluent is sampled and analyzed weekly for I-131, other principal gamma emitters. Results of analyses for the preceding week or for a period as long as the preceding 3 months are used to calculate the alarm/trip set point of the corresponding effluent radioactivity monitor in order to determine a representative value. In each case, whether batch or continuous, the monitor alarm/trip set point may be set at lower activity concentration than the calculated set point.

1.5 LIQUID EFFLUENT DOSE CALCULATION - 10 CFR 50

Doses resulting from the release of radioiodines and particulates must be calculated to show compliance with Appendix I of 10CFR50. Calculations will be performed at least monthly for all liquid effluents as stated in SURVEILLANCE REQUIREMENT 4.11.1.2 and SURVEILLANCE REQUIREMENT 4.11.1.3.1 to verify that the dose to MEMBERS OF THE PUBLIC is maintained below the limits specified in CONTROL 3.11.1.2

The maximum dose to an individual from radioiodines, tritium, and radioactive particulates with half-lives of greater than eight days in liquid effluents released to unrestricted areas is determined as described in Reg. Guide 1.109. Environmental pathways that radioiodine, tritium, and particulates in liquid effluent follow to the maximally exposed MEMBER OF THE PUBLIC are assumed to be: exposure to shoreline deposits, ingestion of fish, and ingestion of shellfish. To assess compliance with CONTROL 3.11.1.2, the dose due to radioactive iodine, tritium, and particulates in liquid effluent is calculated to a person at the Route 9 bridge who consumes fish and shellfish harvested at that location.

1.5.1 MEMBER OF THE PUBLIC DOSE - LIQUID EFFLUENTS

CONTROL 3.11.1.2 limits the dose or dose commitment to MEMBERS OF THE PUBLIC from radioactive materials in liquid effluents from Oyster Creek Generating Station to those listed in Table 1.5.1-1.

TABLE 1.5.1-1 LIQUID PATHWAY DOSE LIMITS

<u>During Any Calendar Quarter</u>	<u>During Any Calendar Year</u>
≤ 1.5 mrem to total body	≤ 3.0 mrem to total body
≤ 5.0 mrem to any organ	≤ 10.0 mrem to any organ

Per the SURVEILLANCE REQUIREMENTS of 4.11.1.2, the following calculation methods shall be used for determining the dose or dose commitment due to the liquid radioactive effluents from Oyster Creek. Applicable liquid pathways to man for Oyster Creek include shoreline exposure, and ingestion of saltwater fish and shellfish. The receptor location is provided in Table A-4.

1.5.2 SHORELINE DEPOSIT DOSE

The shoreline exposure pathway dose is calculated generally in the form (based on Reg. Guide 1.109):

$$Rapj = 110000 \frac{UapWM}{F} \sum_i QiTiDaipj (1 - \exp(-\lambda iTb))$$

Where:

110000 = a constant that accounts for time and flow conversions

Rapj = the annual dose to organ j (including the total body), through pathway p, to age group a

Uap = the age dependent usage factor for the specific pathway. Usage factors for shoreline exposure are residence time on the shoreline (hours). Usage factors are provided in Reg. Guide 1.109 Table E-5. Usage factors specifically selected for Oyster Creek are presented in Table B-1.

W = the shore width factor. This adjusts the infinite plane gamma or beta dose factors for the finite size and shape of the shoreline. Different factors apply to different bodies of water. A factor of 0.1 is used for OC for 'discharge canal bank'.

M = the recirculation factor. The recirculation factor is a multiplier of 3.76 to account for recirculation of discharge water back into the intake. Although this occurs infrequently, it is assumed to occur for each liquid release.

- F = the flow rate in the discharge canal in cubic feet per second
- Qi = the activity of the ith isotope in the release in curies
- Ti = the half life of the ith isotope in days
- Daipj = the age a, isotope i, pathway p, and organ j, specific dose conversion factor. Pathway, isotope, age, and organ specific dose factors are obtained from Regulatory Guide 1.109 Appendix E, Tables E-6 through E-14
- λ_i = the decay constant of the ith isotope in years
- Tb = the long term buildup time, assumed to be 30 years

Note: λ_i and Tb can use any time units as long as they are both the same. No transit delay (Tp from Reg. Guide 1.109) is assumed.

1.5.3 SHORELINE DOSE EXAMPLE 1

The following provides an example of the liquid dose calculation:

Initial parameters:

Canal flow rate 1E6 gpm (typical of normal full power operation)

Release: 10,000 gallons of water at 1E-3 μ Ci/ml Co-60

Problem: calculate shoreline whole body dose

- Uap = 67 (teenager) hours
- W = 0.1
- M = 3.76
- F = 2228 [1E6 gpm * 3785 ml / gal / (60 sec/min * 28316 ml/ft³) = 2228 CFS]
- Qi = 0.03785 Ci [1E-3 μ Ci/ml * 10000gal * 3785ml/gal / 1E6 uCi/Ci = 0.03785 Ci]
- Ti = 1920 [5.27 years*365 days/yr = 1.92E3 days]
- λ_i = 1.31E-1 [0.693 / (5.27 yrs)]
- Tb = 30 years
- Daipj = 1.7E-8 mrem/hr / pCi/m² Gamma dose factor

Calculate Rapj for a = Teen, j = total body, p = shoreline dose for one isotope

$$Rapj = 110000 \frac{67 * 0.1 * 3.76}{2228} \sum_i 0.03785 * 1920 * 1.7E-8 * (1 - \exp(-1.3E-1 * 30))$$

$$Rapj = 1.51E-3 \text{ mrem:teen:wholebody}$$

1.5.3.1 SHORELINE DOSE EXAMPLE 2

The following provides an example of the liquid dose calculation for groundwater remediation discharge:

Initial parameters:

Canal flow rate 1E6gpm (typical of normal full power operation)
Release: 50 gpm for 7 days at 1E-8 µCi/ml Co-60

Problem: Calculate shoreline whole body dose

Uap = 67 (teenager) hours
W = 0.1
M = 3.76
F = 2228 [1E6 gpm * 3785 ml / gal / (60 sec/min * 28316 ml/ft³) = 2228 CFS]
Qi = 1.91E-5 Ci [1E-08 µCi/ml*50gpm*7days*24hr/day*60 min/hr*3785ml/gal/1E+6 µCi/Ci]
Ti = 1920 [5.27 years*365 days/yr = 1.92E+03 days]
λi = 1.31E-1 [0.693 / (5.27 yrs)]
Tb = 30 years
Daipj = 1.7E-8 mrem/hr / pCi/m² External Dose factor

Calculate Rapj for a = Teen, j = total body, p = shoreline dose for one isotope

$$Rapj = 110000 \frac{67 * 0.1 * 3.76}{2228} \sum_i 1.91E-5 * 1920 * 1.7E-8 * (1 - \exp(-1.31E-1 * 30))$$

$$Rapj = 7.60E-7 \text{ mrem:teen:wholebody}$$

1.5.4 INGESTION DOSE - LIQUID

Ingestion dose pathway calculations are similar to those for the shoreline dose, with minor changes in constants, removal of the shore width factor, and inclusion of the bioaccumulation factor:

$$Rapj = 1100 \frac{UapM}{F} \sum_i QiBipDaipj$$

Where:

Bip = the stable element bioaccumulation factor for pathway p for the ith isotope

No transit delay is assumed

Pathway, isotope, age, and organ specific dose factors are obtained from Regulatory Guide 1.109 Appendix E Tables E-7 through E-14. Bioaccumulation factors are

provided in Reg. Guide 1.109 Table A-1. Usage factors are provided in Reg. Guide 1.109 Table E-5. Usage factors specifically selected for Oyster Creek are presented in Table B-1.

The radionuclides included in the periodic dose assessment per the requirements of CONTROL 3/4.11.1.2 are those as identified by gamma spectral analysis of the liquid waste samples collected and analyzed per the requirements of CONTROL 3/4.11.1.1, Table 4.11.1.1.1-1.

Radionuclides requiring radiochemical analysis (e.g., Sr-89 and Sr-90) will be added to the dose analysis at a frequency consistent with the required minimum analysis frequency of Table 4.11.1.1.1-1.

1.5.5 INGESTION DOSE CALCULATION EXAMPLE 1

The following provides an example of the liquid dose calculation:

Initial parameters:

Canal flow rate 1E6 gpm (typical of normal full power operation)

Release: 10000 gallons of water at 1E-3 $\mu\text{Ci/mL}$ Co-60

Problem: calculate teen whole body dose from saltwater fish ingestion

$$Rapj = 1100 \frac{UapM}{F} \sum_i QiBipDaipj$$

Uap = 16 (teenager) Kg

M = 3.76

F = 2228 [1E6 gpm * 3785 ml / gal / (60 sec/min * 28316 ml/ft³) = 2228 CFS]

Qi = 0.03785 Ci [1E-3 $\mu\text{Ci/mL}$ * 10000gal * 3785mL/gal / 1E6 $\mu\text{Ci/Ci}$ = 0.03785 Ci]

Bip = 100

Daipj = 6.33E-6 mrem / pCi

Calculate Rapj for a = Teen, j = total body, p = fish ingestion dose for one isotope

$$Rapj = 1100 \frac{16 * 3.76}{2228} \sum_i 0.03785 * 100 * 6.33 E - 6$$

$$Rapj = 7.12 E - 4 \text{ mrem} : \text{teen} : \text{wholebody}$$

1.5.5.1 INGESTION DOSE CALCULATION EXAMPLE 2

The following provides an example of the liquid dose calculation for ground water remediation discharge:

Initial parameters:

Canal flow rate 1E6 gpm (typical of normal full power operation)
Release: 50 gpm for 7 days at 700,000 pCi/l H-3

Problem: calculate teen whole body dose from saltwater fish ingestion

$$Rapj = 1100 \frac{UapM}{F} \sum_i QiBipDaipj$$

Uap = 16 (teenager) Kg
M = 3.76
F = 2228 [1E6 gpm * 3785 ml / gal / (60 sec/min * 28316 ml/ft³) = 2228 CFS]
Qi = 1.34 Ci [7E+5 pCi/l * 50gpm * 7days * 24hr/day * 60 min/hr * 3.785 l/gal / 1E+12 pCi/Ci]
Bip = 0.90 (Regulatory Guide 1.109, Table A-1)
Daipj = 1.06E-07 mrem / pCi

Calculate Rapj for a = Teen, j = total body, p = fish ingestion dose for one isotope

$$Rapj = 1100 \frac{16 * 3.76}{2228} \sum_i 1.34 * 0.9 * 1.06 E - 7$$

$$Rapj = 3.80 E - 6 \text{ mrem :teen : wholebody}$$

1.5.6 PROJECTED DOSE – LIQUID

The projected doses in a 31 day period are equal to the calculated doses from the current 31 day period.

1.6 REPRESENTATIVE SAMPLES

A sample should be representative of the bulk stream or volume of effluent from which it is taken. Prior to sampling, large volumes of liquid waste should be mixed in as short a time interval as practicable to assure that any sediments or particulate solids are distributed uniformly in the waste mixture. Recirculation pumps for liquid waste tanks (collection or sample test tanks) should be capable of recirculating at a rate of not less than two tank volumes in eight hours. Minimum recirculation times and methods of recirculation are controlled by specific plant procedures.

2.0 GASEOUS EFFLUENTS

2.1 RADIATION MONITORING INSTRUMENTATION AND CONTROLS

The gaseous effluent monitoring instrumentation and controls at Oyster Creek for controlling and monitoring normal radioactive material releases in accordance with the Radiological Effluent CONTROLS are summarized as follows:

(1) Main Stack

The main stack receives normal ventilation flow from the reactor building, new radwaste, old radwaste, process discharge flow from the augmented off gas system (AOG), condenser off gas flow if AOG is not in service, and normal ventilation flow from portions of the turbine building, predominantly the condenser bay area. Reactor building and turbine building flow is not normally processed or filtered. Reactor Building flow may be manually or automatically directed through the Standby Gas Treatment System (SBGTS) which has particulate and charcoal filtration. Off gas flow is processed through AOG or through a 30-minute delay pipe prior to release. Flow from the 'new' and 'old' radwaste buildings is HEPA filtered. Releases through the main stack are monitored for noble gases using the RAGEMS I system and sampled for iodine, particulates and tritium. The plant stack is considered to be a true elevated release point.

(2) Turbine Building Vent

The Turbine building vent is monitored for noble gases by the RAGEMS II system and sampled for iodine, particulates and tritium. It discharges on the west side of the turbine building approximately at roof height and is considered to be a ground level release. It ventilates the turbine floor and other areas of the turbine building. Flow through this release point is not filtered.

(3) Feed Pump Room Vent

The feed pump room vent is monitored by RAGEMS II. It discharges on the east side of the turbine building below roof height and is considered to be a ground level release. It ventilates the reactor feed pump room. Flow through this release point is not filtered.

(4) Augmented Off Gas Building Vent

Off gas Building HVAC is released through a ground level release from the building. Off Gas process flow is not released through the building ventilation, but is routed to the stack plant. A ventilation monitoring system monitors for noble gas and samples for particulate and iodine.

(5) Isolation Condensers

The isolation condensers are a ground level release. The predominant isotope through this potential release point is tritium as a consequence of the forced evaporation of condensate transfer water when the isolation condensers are initiated. Releases are neither monitored nor is the release process flow sampled. Releases of tritium are evaluated based on liquid samples of the input and the volume used.

Gaseous radioactive waste flow diagrams with the applicable, associated radiation monitoring instrumentation controls are presented in Figures D-2-1 and D-2-2.

2.2 GASEOUS EFFLUENT MONITOR SET POINT DETERMINATION

2.2.1 PLANT VENT

Per the requirements of CONTROL 3.3.3.11, alarm set points shall be established for the gaseous effluent monitoring instrumentation to ensure that the release rate of noble gases does not exceed the limits of CONTROL 3.11.2.1, which corresponds to a dose rate at the SITE BOUNDARY of 500 mrem/year to the total body or 3000 mrem/year to the skin. Based on a grab sample analysis of the applicable release (i.e., of the Stack, Off gas process flow, etc.), the radiation monitoring alarm set points may be established by the following calculation methods. A set point of a monitor of an elevated release, e.g., from the stack, may be calculated using the equation:

$$S = 1.06 \left(\frac{h}{f} \right) \frac{\sum C_i}{\sum (C_i D F S_i)} + Bkg$$

Where:

S = the alarm set point (cpm)

h = primary conversion factor of the instrument - monitor response to activity concentration of effluent being monitored, cpm/($\mu\text{Ci}/\text{cm}^3$). Each monitoring channel has a unique response, h, which is determined by the instrument calibration.

C_i = relative concentration of noble gas radionuclide i in effluent at the point of monitoring ($\mu\text{Ci}/\text{cm}^3$)

1.06 = 500 mrem/year /472 (conversion of cfm to cc/sec)

$D F S_i$ = factor converting elevated release rate of radionuclide i to total body dose equivalent rate at the location of potential exposure. Units are: mrem/(yr($\mu\text{Ci}/\text{sec}$)). From Table A-1.

f = flow of gaseous effluent stream being monitored, i.e., stack flow, vent flow, etc. (ft^3/min)

BKG = Monitoring instrument background (cpm or mR/hr)

2.2.2 OTHER RELEASE POINTS

The set point of a monitor of a ground-level or vent release, e.g., from the turbine building vent or the AOG building, may be calculated with the equation:

$$S = 1.06 \left(\frac{h}{f \frac{X}{Q}} \right) \frac{\sum C_i}{\sum (C_i DFV_i)} + Bkg$$

Where:

DFVi = factor converting ground-level or vent release of radionuclide i to the total body dose equivalent rate at the location of potential exposure. Units are: mrem-m³/μCi-year From Table A-1.

X/Q = atmospheric dispersion from point of ground-level or vent release to the location of potential exposure (sec/m³) from Table 2.2.2-1.

The atmospheric dispersion, X/Q, and the dose conversion factor, DFSi, depend upon local conditions. For the purpose of calculating radioactive noble gas effluent monitor alarm set points appropriate for the OCGS, the locations of maximum potential Off Site exposure and the reference atmospheric dispersion factors applicable to the derivation of set points are given in Table 2.2.2-1.

Symbols for this equation were defined in Section 2.2.1.

TABLE 2.2.2-1 RECEPTOR LOCATIONS AND DISPERSION FOR GASEOUS MONITOR SET POINTS

Discharge Point	Receptor Location		Atm. Dispersion (sec/m ³)
	Sector	Distance(m)	
Ground-level or vent	ENE	338	4.59 E-5
Stack	SW	229	N/A

2.2.3 RADIONUCLIDE MIX FOR SET POINTS

For the purpose of deriving a set point, the distribution of radioactive noble gases in an effluent stream may be determined in one of the following ways:

- 2.2.3.1 Preferably, the radionuclide distribution is obtained by gamma isotopic analysis of identifiable noble gases in effluent gas samples. Results of the analyses of one or more samples may be averaged to obtain a representative spectrum.
- 2.2.3.2 In the event a representative distribution is unobtainable from recent measurements by the radioactive gaseous waste sampling and analysis program, it may be based upon past measurements.
- 2.2.3.3 Alternatively, the total activity concentration of radioactive noble gases may be assumed to be Xenon-133 as found in Reg Guide 1.97.

2.3 GASEOUS EFFLUENT INSTANTANEOUS DOSE RATE CALCULATIONS - 10
CFR 20

2.3.1 SITE BOUNDARY DOSE RATE - NOBLE GASES

CONTROL 3.11.2.1a limits the dose rate at the SITE BOUNDARY due to noble gas releases to ≤ 500 mrem/yr, total body and ≤ 3000 mrem/yr, skin. Radiation monitor alarm set points are established to ensure that these release limits are not exceeded. In the event any gaseous releases from the station results in an alarm set point (as determined in Section 2.2) being exceeded, an evaluation of the SITE BOUNDARY dose rate resulting from the release shall be performed.

2.3.1.1 TOTAL BODY DOSE RATE

The total body dose equivalent rate from radioactive noble gases discharged from an elevated point (stack above building wake) is calculated with the equation:

$$DG = \sum_i Q_i P_{\gamma} S_i$$

From a ground-level release (building vent) the total body dose equivalent rate is:

$$DG = \frac{X}{Q_v} \sum_i Q_i P_{\gamma} V_i$$

Where:

DG = total body dose equivalent rate due to irradiation by radioactive noble gas (mrem/hr)

Q_i = average discharge rate of noble gas radionuclide i released during the averaging time (μ Ci/hr)

P _{γ} V_i = factor converting time integrated ground-level concentration of noble gas nuclide i to total body dose $\frac{\text{mrem} \cdot \text{m}^3}{\mu\text{Ci} \cdot \text{sec}}$. See Table A-2.

$\frac{X}{Q_v}$ = atmospheric dispersion factor from the OCGS to the Off Site location of interest (sec/m^3) from Table 2.3.1.3-1

$P_{\gamma Si}$ = factor converting unit noble gas nuclide i stack release to total body dose at ground level received outdoors from the overhead plume (mrem/ μ Ci). See Table A-2

The noble gas plume gamma-to-total body dose factors, $P_{\gamma Si}$ at designated locations are derived from meteorological dispersion data with the USNRC RABFIN software computer code or similar computer program implementing Reg Guide 1.109, Appendix B. The noble gas semi-infinite cloud gamma-to-total body dose factors, $P_{\gamma Vi}$, are derived from Reg Guide 1.109, Revision 1, Table B-1, Column 5.

2.3.1.2 EXAMPLE TOTAL BODY DOSE RATE

Calculate the dose from a release of 100 Ci of Xe133 in 1 hour from a ground level release

$$DG = \frac{X}{Q_v} \sum_i Q_i P_{\gamma Vi}$$

X/Q_v = 4.59E-5 sec/m³ (Table 2.3.1.3-1)
 Q_i = 1E8 μ Ci/hr [100Ci*1E6 μ Ci/Ci]
 $P_{\gamma Vi}$ = 9.33E-6 mrem-m³ / μ Ci-sec

$$DG = 4.59 E - 5 \sum_i 1 E 8 * 9.33 E - 6$$

$$DG = 0.043 \text{ mrem / hr}$$

2.3.1.3 SKIN DOSE RATE

The dose equivalent rate to skin from radioactive noble gases is calculated by assuming a person at ground level is immersed in and irradiated by a semi-infinite cloud of the noble gases originating in airborne effluent. It is calculated for each air effluent discharge point with the equation:

$$DB = \frac{X}{Q} \sum_i Q_i (SB_i + 1.11 A \gamma Vi)$$

where:

DB = dose rate to skin from radioactive noble gases (mrem/hr)

$\frac{X}{Q}$ = Atmospheric dispersions from gaseous effluent discharge point to ground-level location of interest (sec/m³) from Table 2.3.1.3-1.

Q_i = discharge rate of noble gas radionuclide i (μ Ci/hr)

- S_{Bi} = factor converting time integrated ground-level concentration of noble gas radionuclide i to skin dose from beta radiation $\frac{\text{mrem} \cdot \text{m}^3}{\mu\text{Ci} \cdot \text{sec}}$ from Table A-2.
- $A_{\gamma Vi}$ = factor for converting time integrated, semi-infinite concentration of noble gas radionuclide i to air dose from its gamma $\frac{\text{mrad} \cdot \text{m}^3}{\mu\text{Ci} \cdot \text{sec}}$ from Table A-2.

The noble gas beta radiation-to-skin-dose factors, S_{Bi} and the noble gas gamma-to-air dose factors, $A_{\gamma Vi}$, are derived from Reg Guide 1.109, Revision 1, Table B-1, columns 3 and 4 respectively. A tabulation of these factors used to compute noble gas-to-dose equivalent rate at 338 meters ENE for ground-level or vent and 544 meters SE for stack from the OCGS is in Table A-2.

The dose equivalent rate is calculated with the meteorological dispersion data given in Table 2.3.1.3-1.

TABLE 2.3.1.3-1 RECEPTOR LOCATIONS AND DISPERSION FOR SITE BOUNDARY DOSE RATES

Discharge Point	Receptor Location Sector	Distance (m)	Atm. Dispersion (sec/m ³)
Ground Level or Vent	ENE	338	4.59 E-5
Stack	SE	544	1.05 E-8

Alternatively, an approved computer code (e.g., "SEEDS" or "Open EMS") that implements the requirements of Regulatory Guide 1.109 may be used.

Actual meteorological conditions concurrent with the release period or the default, annual average dispersion parameters as presented above may be used for evaluating the gaseous effluent dose rate.

2.3.1.4 EXAMPLE SKIN DOSE RATE

Calculate the skin dose from a release of 100 Ci of Xe133 in 1 hour from a ground level release:

$$DB = \frac{X}{Q} \sum_i Q_i (S_{Bi} + 1.11 A_{\gamma Vi})$$

- X/Q = 4.59 E-5 sec/m³
 Q_i = 1E8 $\mu\text{Ci/hr}$
 S_{Bi} = 9.71E-6 mrem/ $\mu\text{Ci}/ \text{m}^3/\text{sec}$
 $A_{\gamma Vi}$ = 1.12E-5 mrad/yr per $\mu\text{Ci}/\text{m}^3$

$$DB = 4.59E - 5 \sum_i 1E8(9.71E - 6 + 1.11 * 1.12E - 5)$$

$$DB = 0.102 \text{ mrad / hr}$$

2.3.2 SITE BOUNDARY DOSE RATE - RADIOIODINE AND PARTICULATES

2.3.2.1 METHOD - SITE BOUNDARY DOSE RATE - RADIOIODINE AND PARTICULATES

The dose rate Off Site due to the airborne release of I-131, I-133, tritium, and particulates with half-lives greater than 8 days is limited to no more than 1500 mrem/yr to any organ in CONTROL 3.11.2.1b. Evaluation of compliance with CONTROL 3.11.2.1b is based on the sampling and analyses specified in TABLE 4.11.2.1.2-1. Since the dose rate cannot be resolved within less than the sample integration or compositing time, the contribution of each radionuclide to the calculated dose rate will be averaged no more than 3 months for H-3, Sr-89, Sr-90, and alpha-emitting radionuclides and no more than 31 days for other radionuclides. These are their usual sample integration or compositing times. The equation used to assess compliance of radioiodine, tritium, and radioactive particulate releases with the dose rate limit is:

$$DR_p = 1E6 \sum_e \sum_i^n Ra D F A_{ija} Q_{ei} \frac{\overline{X}}{Q_e}$$

Where:

1E6 = conversion pCi/μCi

DR_p = the average dose rate to an organ via exposure pathway, p (mrem/yr).

DFA_{ija} = inhalation dose factors due to intake of radionuclide i, to organ j age group a (mrem/pCi) from Reg. Guide 1.109 Appendix E.

Ra = age group dependent inhalation respiratory rate (usage factor) m³/yr from Table B-1

$\frac{\overline{X}}{Q_e}$ = annual average relative airborne concentration at an Off Site location due to a release from either the Stack or a vent, i.e. release point, e (sec/m³) from Table 2.3.2.1-1.

Q_{ei} = release rate of radionuclide i from release point, e during the period of interest (μCi/sec).

For real-time meteorology and on an annual average basis, the location of the maximum ground-level concentration originating from a vent release will differ from the maximum ground-level concentration from a stack release. When assessing

compliance with CONTROL 3.11.2.1b for tritium, iodine, and particulate, the air dispersion (X/Q) values are provided in Table 2.3.2.1-1.

TABLE 2.3.2.1-1 LOCATION OF MAXIMUM EXPOSURE RE BY INHALATION

Discharge Point	Receptor Location		Atm. Dispersion (sec/m ³)
	Sector	Distance (m)	
Ground Level or Vent	ENE	338	4.59 E-5
Stack	SE	937	1.25 E-8
Alternatively, inhalation exposure to effluent from the stack may be evaluated at the closest hypothetical individual located at:			
Stack	SE	805	1.29 E-8

Alternatively, an approved computer code (e.g., "SEEDS" or "Open EMS") that implements the methods of Regulatory Guide 1.109, may be used.

2.3.2.2 EXAMPLE IODINE AND PARTICULATES DOSE RATE CALCULATION

Calculate the child thyroid dose rate from a release of 100 $\mu\text{Ci/hr}$ of I131 from a ground level release:

$$DR_p = 1E6 \sum_e \sum_i^n Ra D F A_{ij} Q_{ei} \frac{\bar{X}}{Q_e}$$

$$Ra = 3700 \text{ m}^3/\text{yr}$$

$$DFA_{ij} = 4.39E-3 \text{ mrem/pCi}$$

$$Q_{ei} = 0.028 \text{ } \mu\text{Ci/sec} \quad [100 \mu\text{Ci/hr} / 3600 \text{ sec/hr} = 0.02778]$$

$$X/Q_e = 4.59 \text{ E-5 sec/m}^3$$

$$DR_p = 1E6 \sum_i^n 3700 * 4.39E-3 * 0.028 * 4.59E-5$$

$$DR_p = 20.9 \text{ mrem/yr}$$

2.4 NOBLE GAS EFFLUENT DOSE CALCULATIONS - 10 CFR 50

Doses resulting from the release of noble gases must be calculated to show compliance with Appendix I of 10CFR50. Calculations will be performed at least monthly for all gaseous effluents as stated in SURVEILLANCE REQUIREMENT 4.11.2.2 to verify that the dose to air is kept below the limits specified in CONTROL 3.11.2.2 and the dose to MEMBERS OF THE PUBLIC is maintained below the limits specified in CONTROL 3.11.2.3.

2.4.1 UNRESTRICTED AREA DOSE - NOBLE GASES

CONTROL 3.11.2.2 requires a periodic assessment of releases of noble gases to evaluate compliance with the quarterly air dose limits shown in Table 2.4.1-1.

TABLE 2.4.1-1 ANNUAL AIR DOSE LIMITS

<u>During any calendar quarter</u>	<u>During any calendar year</u>
≤ 5 mrad gamma-air	≤ 10 mrad gamma-air
≤ 10 mrad beta-air	≤ 20 mrad beta-air

The method used to calculate the air dose at the critical location due to noble gas is described by the following equations. The limits are provided in CONTROL 3.11.2.2 for air dose Off Site due to gamma and beta radiations from effluent noble gas.

2.4.1.1 AIR DOSE METHOD

For Gamma Radiation:

$$Dose \gamma = \sum_{i=1}^n A \gamma V_i \frac{\bar{X}}{Q} v Q_{vi} + A \gamma S_i Q_{si}$$

For Beta Radiation

$$Dose \beta = \sum_e \sum_{i=1}^n A \beta_i \frac{\bar{X}}{Q} e Q_{ei}$$

Where:

Dose γ = the gamma dose during any specified time period (mrad).

Dose β = the beta dose during any specified time period (mrad).

$A \gamma V_i$ = the air dose factor due to ground level gamma emissions for each identified noble gas radionuclide, i; (mrad/yr per $\mu\text{Ci}/\text{m}^3$). Table A-2

$A \gamma S_i$ = the factor for air dose at ground level due to irradiation for an airborne plume resulting from a Stack release (mrad per μCi), Table A-3.

$A \beta_i$ = the air dose factor due to beta emissions for each identified noble gas radionuclide, i (mrad/yr per $\mu\text{Ci}/\text{m}^3$). Table A-3

$\frac{\bar{X}}{Q_e}$ $\frac{\bar{X}}{Q_v}$ = the annual average relative concentration for areas at or beyond the site boundary for releases from either the Stack or ground vent at the critical location (sec/m^3), Table 2.4.1.1-1

- Qvi = amount of radionuclide i released from vents (μCi).
- Qsi = amount of radionuclide i released from the Stack (μCi).
- Qei = amount of radionuclide i released from release point e(μCi).

Noble gases may be released from the ground level vents and stack. The quantity of noble gas radionuclides released will be determined from the continuous noble gas monitors and periodic isotopic analyses. The maximum Off Site gamma radiation dose rate to air from noble gases discharged from either the stack or from building vents occurs at 805 meters SE of the OCGS for the stack and 338 meters ENE of the OCGS for building vents. Values of $A_{\gamma Si}$ depend upon the meteorological conditions and the location of exposure and are calculated using the NRC RABFIN code or similar one in accordance with Reg. Guide 1.109, Appendix B, Section 1. $A_{\gamma Vi}$ and ABi are derived from Reg. Guide 1.109, Table B-1 for a semi-infinite cloud, independent of meteorology or location. Values of $A_{\gamma Si}$, $A_{\gamma Vi}$ and ABi used to calculate the noble gas radiation dose to air at 805 meters SE of the OCGS for the stack and 338 meters ENE of the OCGS for building vents are in Table A-3. Reference atmospheric dispersion from the OCGS to 805 meters SE for the stack and 338 meters ENE for building vents is given in Table 2.4.1.1-1.

TABLE 2.4.1.1-1 RECEPTOR LOCATIONS AND DISPERSION FOR AIR DOSE

Discharge Point	Receptor Location		Atm. Dispersion (sec/m ³)
	Sector	Distance (m)	
Ground Level or Vent	ENE	338	4.59 E-5
Stack	SE	805	1.29 E-8

Alternatively, an approved computer code (e.g., "SEEDS" or "Open EMS") that implements the requirements of Reg. Guide 1.109 may be used.

2.4.1.2 EXAMPLE NOBLE GAS AIR DOSE CALCULATION

Calculate the gamma air dose from a release of 1 Ci per hour of Xe133 for 10 hours from a ground level release and 100Ci per hour for 10 hours from an elevated release:

$$Dose_{\gamma} = \sum_{i=1}^n A_{\gamma Vi} \frac{\bar{X}}{Q} v Q_{vi} + A_{\gamma Si} Q_{si}$$

- $A_{\gamma Vi}$ = 1.12E-5 mrad – m³ / μCi – sec
- X/Q = 4.59 E-5 sec/m³
- Q_{vi} = 1E7 μCi [1Ci/hr*10hrs*1E6 μCi/Ci]
- $A_{\gamma Si}$ = 1.03E-12 mrad / μCi
- Q_{si} = 1E9 μCi [100Ci/hr*10hrs*1E6 μCi/Ci]

$$Dose_{\gamma} = \sum_{i=1}^n 1.12E-5 * 4.59E-5 * 1E7 + 1.03E-12 * 1E9$$

$$Dose_{\gamma} = \sum_{i=1}^n 5.14E-3 + 1.03E-3$$

$$Dose_{\gamma} = 6.17E-3 \text{ mrad}$$

Note how the ground level portion has a higher dose contribution per unit activity than the elevated portion.

2.4.1.3 INDIVIDUAL PLUME DOSE METHOD

The method for dose to an individual from noble gases is essentially identical with the air dose method except that different dose factors apply. Also, since dose to the skin combines the contribution from gamma and beta emissions, the gamma dose must be added to the beta dose to obtain a total skin dose.

For Total Body:

$$Dose(t) = \sum_{i=1}^n P_{\gamma} V_i \frac{\bar{X}}{Q} v Q_{vi} + P_{\gamma} S_i Q_{si}$$

For Skin

$$Dose(s) = \sum_e \sum_{i=1}^n S_{\beta i} \frac{\bar{X}}{Q} e Q_{ei} + Dose(t)$$

Where:

Dose(t) = the total body dose during any specified time period (mrem).

Dose(s) = the skin dose during any specified time period (mrem).

$P_{\gamma} V_i$ = the plume dose factor due to ground level gamma emissions for each identified noble gas radionuclide, i; (mrad/yr per $\mu\text{Ci}/\text{m}^3$). Table A-5

$P_{\gamma} S_i$ = the factor for plume dose at ground level due to irradiation for an airborne plume resulting from a Stack release (mrad per μCi), Table A-5.

$S_{\beta i}$ = the skin dose factor due to beta emissions for each identified noble gas radionuclide, i (mrad/yr per $\mu\text{Ci}/\text{m}^3$) from Table A-5.

$\frac{\bar{X}_e}{Q}$ $\frac{\bar{X}_v}{Q}$ = the annual average relative concentration for areas at or beyond the site boundary for releases from either the Stack or ground vent at the critical location (sec/m³) from Table 2.5.1.

Q_{vi} = amount of radionuclide i released from vents (μCi).

Q_{si} = amount of radionuclide i released from the Stack (μCi).

Q_{ei} = amount of radionuclide i released from release point e (μCi).

2.5 RADIOIODINE, PARTICULATE AND OTHER RADIONUCLIDES DOSE CALCULATIONS - 10 CFR 50

Doses resulting from the release of radioiodines and particulates must be calculated to show compliance with Appendix I of 10CFR50. Calculations will be performed at least monthly for all gaseous effluents as stated in SURVEILLANCE REQUIREMENT 4.11.2.2 and SURVEILLANCE REQUIREMENT 4.11.2.3 to verify that the dose to air is kept below the limits specified in CONTROL 3.11.2.2 and the dose to MEMBERS OF THE PUBLIC is maintained below the limits specified in CONTROL 3.11.2.3.

The maximum dose to an individual from radioiodines, tritium, and radioactive particulates with half-lives of greater than eight days in gaseous effluents released to unrestricted areas is determined as described in Reg. Guide 1.109. Environmental pathways that radioiodine, tritium, and particulates in airborne effluent follow to the maximally exposed MEMBER OF THE PUBLIC as determined by the annual land use survey and reference meteorology will be evaluated. The seasonality of exposure pathways may be considered. For instance, if the most exposed receptor has a garden, fresh and stored vegetables are assumed to be harvested and eaten during April through October. Fresh vegetables need not be considered as an exposure pathway during November through March. To assess compliance with CONTROL 3.11.2.3, the dose due to radioactive iodine, tritium, and particulates in airborne effluent is calculated to a person residing 972 meters ESE of the OCGS for ground-level or vent and 937 meters SE of the OCGS for stack. Reference atmospheric dispersion and deposition factors are given in Table 2.5-1.

TABLE 2.5-1 DISPERSION FOR 10CFR50 DOSES

Discharge Point	Dispersion X/Q (sec/m ³)	Deposition D/Q(1/m ²)
Ground Level or Vent	5.13 E-6	1.68 E-8
Stack	1.25 E-8	2.39 E-9

The environmental pathways of exposure to be evaluated are: inhalation, irradiation from ground deposition, and ingestion of milk (cow and goat are treated separately), meat, and vegetables. Eight organs are considered: Bone, Liver, Total Body, Thyroid, Kidney, Lung, GI-LLI (Gastro-Intestinal tract / Lower Large Intestine), and Skin. Four different age groups are considered: Infants, Children, Teens, and Adults. Doses are calculated to a 'receptor' – a person who inhales the airborne activity and resides in a location with ground deposition, and eats and drinks the foodstuffs produced. The maximally exposed individual is conservatively assumed to reside at the location of the highest sum of the inhalation and ground plane doses, while eating and drinking foodstuffs transported from the locations that are highest for those pathways. Receptor locations are provided in Table A-4.

Alternatively, an approved computer code (e.g., "SEEDS" or "Open EMS") that implements the requirements of Reg Guide 1.109 may be used.

2.5.1 INHALATION OF RADIOIODINES, TRITIUM, PARTICULATES, AND OTHER RADIONUCLIDES.

Dose from the inhalation pathway is generally in the form:

$$D_{ja} = RaT \sum_i \frac{X}{Q} Q_i D_{FAija} \text{Exp}(-\lambda_i Tr)$$

Where:

D_{ja} = the dose to the organ j (of eight) of age group a (of four)

Ra = the respiration rate for age group a from Table B-1

T = the duration of the release in fraction of a year

$\frac{X}{Q}$ = The atmospheric dispersion to the point of interest (the 'receptor') in sec/m^3 from Table 2.5-1

Q_i = The release rate of radionuclide i (pCi/sec)

D_{FAija} = The inhalation dose conversion factor (mrem per pCi) for radionuclide i to organ j of age group a from Reg. Guide 1.109 Appendix E.

λ_i = decay constant of isotope i: $0.693/\text{Half life in years}$

Tr = plume transit time from release to receptor in years

λ_i and Tr may be in any time units as long as they are the same

Note that a 'depleted X/Q' (dX/Q) is applicable to particulates only, which accounts for the natural settling and lack of surface reflection of particulates to estimate the downwind concentration accounting for these removal processes. Depleted X/Q will be slightly smaller than the X/Q. This is not used in the ODCM for simplicity. Using the X/Q is therefore slightly conservative compared to the dX/Q .

2.5.2 EXAMPLE CALCULATION - INHALATION OF RADIOIODINES, TRITIUM, PARTICULATES, AND OTHER RADIONUCLIDES

Calculate the dose to child lung from inhalation from a ground level release of 100 μCi of Co-60 in 10 hours. Plume transit decay time is ignored ($\exp(-\lambda_i T) = 1$).

$$D_{ja} = RaT \sum_i \frac{X}{Q} Q_i D_{FAi} j_a$$

D_{ja}	= the dose to the organ j (of eight) of age group a (of four)
Ra	= 3700 m^3/yr
T	= 0.00114 yrs [10 hrs / 8760 hrs / yr]
$\frac{X}{Q}$	= 5.13 E-6 sec/ m^3
Q_i	= 2.78E3 pCi/sec [100 μCi * 1E6 pCi/ μCi / (10 hrs*3600 sec / hr)]
	$D_{FAi} j_a = 1.91\text{E-}3 \text{ mrem / pCi}$

$$D_{ja} = 3700 * 0.00114 * 5.13\text{E-}6 * 2.78\text{E}3 * 1.91\text{E-}3$$

$$D_{ja} = 1.15\text{E-}4 \text{ mrem}$$

2.5.3 INGESTION OF RADIOIODINES, PARTICULATES AND OTHER RADIONUCLIDES

Dose from the ingestion pathways is more complex and is broken out here into multiple steps:

2.5.3.1 CONCENTRATION OF THE RADIONUCLIDE IN ANIMAL FORAGE AND VEGETATION – OTHER THAN TRITIUM

The concentration of a radionuclide in a foodstuff (other than tritium – see section 2.5.3.3 for tritium) is dependent on the atmospheric deposition, the biological uptake into the food, various decay times (plume travel, harvest to table, etc.) and is generally of the form:

Where:

C_{iv} = the concentration (pCi/kg) of radionuclide i in vegetation

Q_i = the release rate of isotope i in pCi/hr

$\frac{D}{Q}$ = The atmospheric deposition to the point of interest (the 'receptor') in $1/\text{m}^2$ from

Table 2.5-1.

$$C_{iv} = \frac{D}{Q} Q_i \left\{ \frac{r(1 - EXP(-\lambda E_i T_e))}{Y_v \lambda E_i} + \frac{B_{iv}(1 - EXP(-\lambda_i T_b))}{P \lambda_i} \right\} EXP(-\lambda_i T_h) EXP(-\lambda_i T_r)$$

- r = the retention coefficient for deposition onto vegetation surfaces (1.0 for iodines, 0.2 for particulates)
- λ_i = the decay constant of radionuclide i; 0.693/half life in hours
- λE_i = the effective removal constant which is the sum of $\lambda_i + \lambda_w$ where λ_w is the weathering constant, 0.0021/hr
- T_e = duration of crop exposure during the growing season in hours. This is not the entire duration of the growing season, and is different for leafy vegetable and fruit/grain/vegetables. Provided in Table E-15 of Reg. Guide 1.109 or Table B-1.
- Y_v = agricultural yield Kg of vegetation per m^2 , typically 0.7 kg/m^2
- B_{iv} = soil uptake concentration factor for transfer of the radionuclide i from the soil to the vegetation through normal root uptake processes in pCi/kg in vegetation per pCi/Kg in soil. Values are provided in Reg. Guide 1.109 Table E-1.
- T_b = the length of time the soil is exposed to contaminated inputs – nominally 30 years (2.63E5 hr)
- P = effective soil density in kg/m^2 normally 240 kg/m^2
- T_h = holdup time, the time the foodstuff is in transit between harvest and consumption in hours
- T_r = plume transit time from release to receptor in hours

2.5.3.2 EXAMPLE CALCULATION OF CONCENTRATION OF THE RADIONUCLIDE IN ANIMAL FORAGE AND VEGETATION – OTHER THAN TRITIUM.

Calculate the forage and vegetation concentration from a ground level release of 100 μCi of Co-60 in 10 hours (plume transit time is ignored $T_r=0$, $EXP(-\lambda_i T_r)=1$):

$$C_{iv} = \frac{D}{Q} Q_i \left\{ \frac{r(1 - EXP(-\lambda E_i T_e))}{Y_v \lambda E_i} + \frac{B_{iv}(1 - EXP(-\lambda_i T_b))}{P \lambda_i} \right\} EXP(-\lambda_i T_h) EXP(-\lambda_i T_r)$$

- D/Q = 1.67E-8 m^2
- Q_i = 1E7 pCi/hr [100 μCi * 1E6 pCi/ μCi / 10 hr]
- r = 0.2
- λ_i = 1.5E-5/hr [0.693 / (5.27yr * 8760 hr/yr)]
- λE_i = 2.12E-3 /hr [1.5E-5 + 0.0021]
- T_e = 720 hr [grass-cow-milk-man pathway value]

Yv = 0.7 kg/m²
Biv = 9.4E-3
Tb = 2.63E5 hr
P = 240 kg/m²
Th = 24.1 hours

$$Civ = 1.67E-8 * 1E7 \left\{ \frac{0.2 * (1 - EXP(-2.12E-3 * 720))}{0.7 * 2.12E-3} + \frac{9.4E-3 * (1 - EXP(-1.5E-5 * 2.63E5))}{240 * 1.5E-5} \right\} EXP(-1.5E-5 * 0)$$

$$Civ = 1.67E-8 * 1E7 \left\{ \frac{0.2 * (1 - EXP(-1.53))}{1.48E-3} + \frac{9.4E-3 * (1 - EXP(-3.95))}{3.6E-3} \right\} EXP(-0)$$

$$Civ = 1.67E-1 \left\{ \frac{105.9}{2.56} + \right\} * 1$$

$$Civ = 18.0 \text{ pCi / Kg}$$

2.5.3.3 CONCENTRATION OF TRITIUM IN ANIMAL FORAGE AND VEGETATION

Since tritium is assumed to be released as tritiated water (HTO), the concentration of tritium in a foodstuff is dependent on atmospheric dispersion like a gas, rather than particulate deposition as for other radionuclides for foodstuff uptake. Further, the concentration of tritium in food is assumed to be based on equilibrium between the concentration of the tritium in the atmospheric water and the concentration of tritium in the water in the food. Concentration of tritium in vegetation can be calculated generally in the form (a plume transit decay term: EXP(-λiTr) is ignored since plume travel times are very short compared to the half life):

$$Ctv = 1000 Q_t \frac{X}{Q} * 0.75 * \frac{0.5}{H}$$

Where:

Ctv = the concentration (pCi/kg) of tritium in vegetation

1000 = g per kg

Qt = the release rate of the tritium in pCi/ sec

- X/Q = the atmospheric dispersion at the vegetation point, sec/m³ from Table 2.5-1
- 0.75 = the fraction of vegetation that is water
- 0.5 = the effective ratio between the atmospheric water concentration and the vegetation concentration
- H = the absolute humidity g/m³. Absolute humidity is seasonally dependent, varying from as little as 1 in the winter to as much as 20 in the summer. Monthly average values derived from historical data are provided in Table B-2.

2.5.3.4 EXAMPLE CALCULATION OF CONCENTRATION OF TRITIUM IN ANIMAL FORAGE AND VEGETATION.

Calculate the forage and vegetation concentration from a ground level release of 100 μCi of H-3 in 10 hours. Plume transit decay time is ignored (exp(-λiTr)=1):

$$C_{tv} = 1000Q_t \frac{X}{Q} * 0.75 * \frac{0.5}{H}$$

- Qt = 2778 pCi/sec [100uCi * 1E6 pCi/uCi / (10hrs*3600sec/hr)]
- X/Q = 5.13E-6 sec/m³
- H = 5 g/m³ (assumed for this example)

$$C_{tv} = 2778 * 1000 * 5.13E - 6 * 0.75 * \frac{0.5}{5}$$

$$C_{tv} = 1.07 \text{ pCi / kg}$$

2.5.3.5 CONCENTRATION OF THE RADIONUCLIDE IN MILK AND MEAT

Meat and milk animals are assumed to eat both pasture grass and stored feed. During a fraction of the year, they may be assumed to be exclusively on stored feed, outside of the growing season. If using annual average release, the fraction of stored and fresh feed must be accounted for with fractions, otherwise (as in this ODCM), the fresh pasture pathway is turned on or off depending on the growing season.

The concentration of a radionuclide in the animal feed is calculated as follows:

$$C_{iv} = F_p C_{is} + (1 - F_p) C_{is} (1 - F_s) + C_{ip} F_s (1 - F_p)$$

Where:

- Fp = the growing season pasture factor: 1 if not growing season, 0 if in growing season

- Fs = the fraction of the daily feed from fresh pasture from Table B-1 or Exhibit E-15 from Reg. Guide 1.109.
- Cip = the concentration in the fresh pasture feed (Civ from section 2.5.3.2 with Th = 0 for immediate consumption)
- Cis = the concentration in stored feed (Civ from section 2.5.3.2 with Th = 90 days)

The concentration in the milk is then based on this feed concentration:

$$C_{im} = F_m C_{iv} Q_f \text{EXP}(-\lambda_i T_f)$$

Where:

- Cim = the concentration in milk pCi/l
- Fm = the transfer coefficient of intake to concentration in the milk (d/l) from Reg. Guide 1.109 Table E-1.
- Qf = feed intake rate Kg/d from Reg. Guide 1.109 Table E-3.
- λ_i = radionuclide i decay constant in 1/days
- Tf = transport time from milk production to consumption (2 days for milk)

The Goat milk pathway may be similarly evaluated:

$$C_{im} = F_g C_{iv} Q_f \text{EXP}(-\lambda_i T_f)$$

Where:

- Fg = the transfer coefficient of intake to concentration in the milk (d/l) for goats from Reg. Guide 1.109 Table E-2.

And for meat:

$$C_{if} = F_f C_{iv} Q_f \text{EXP}(-\lambda_i T_s)$$

Where:

- Ff = the transfer coefficient of intake to concentration in the meat d/kg from Reg. Guide 1.109 Table E-1.
- Ts = the transport time from slaughter to consumption (20 days)

2.5.3.6 EXAMPLE CALCULATION OF CONCENTRATION OF THE RADIONUCLIDE IN MILK AND MEAT

Calculate the concentration in cow milk from a ground level release of 100 μCi of Co-60 in 10 hours. Plume transit decay time is ignored ($\exp(-\lambda_i T_r)=1$):

$$C_{iv} = F_p C_{is} + (1 - F_p) C_{is} (1 - F_s) + C_{ip} F_s (1 - F_p)$$

Assume animals are on pasture and receive half of their food from stored feed.

C_{ip}	= 18.0 pCi/kg as previously calculated in section 2.5.3.2
F_p	= 0
F_s	= 0.5

C_{is} is calculated by applying a 90 day decay term to the C_{ip} value previously calculated, since the previous decay correction was for 0 time as shown in 2.5.3.2.

$$C_{is} = 18.0 * (\exp(-0.693 * 90 / (5.27 * 365)))$$

$$C_{is} = 17.4 \text{ pCi / kg}$$

C_{iv} is then:

$$C_{iv} = 0 * 17.4 + (1 - 0) 17.4 * (1 - 0.5) + 18.0 * 0.5 * (1 - 0)$$

$$C_{iv} = 17.7 \text{ pCi / kg}$$

The concentration in milk is given by:

$$C_{im} = F_m C_{iv} Q_f \text{EXP}(-\lambda_i T_f)$$

F_m	= 1.0E-3 d/l
Q_f	= 50 Kg/d
λ_i	= 3.6E-4/d [0.693 / (5.27 yrs*365 days/yr)]

$$C_{im} = 1.0E - 3 * 17.7 * 50 * \text{EXP}(-3.6E - 4 * 2)$$

$$C_{im} = 0.88 \text{ pCi / l}$$

The concentration in meat given by:

$$C_{if} = F_f C_{iv} Q_f \text{EXP}(-\lambda_i T_s)$$

F_f	= 1.3E-2 d/kg
-------	---------------

Qf = 50 Kg/d
λi = 3.6E-4/d

$$C_{if} = 1.3E-2 * 17.7 * 50 * EXP(-3.6E-4 * 20)$$

$$C_{if} = 11.5 \text{ pCi/kg}$$

2.5.3.7 DOSE FROM CONSUMPTION OF MILK, MEAT, AND VEGETABLES

The environmental pathway ingestion dose is the sum of the milk, meat, and vegetation ingestion pathways. There are two separate pathways for vegetation: fresh leafy vegetables and a combination of fruits, non-leafy vegetables, and grains. These differ only in the decay and buildup processes applied to account for the environmental exposure, and transportation delay decay represented by Te and Th as shown in section 2.5.3.1. For long half-life isotopes (e.g. Co-60) the decay differences have little impact on the dose.

Dose from the environmental ingestion pathways is generally of the form:

$$D_{ja} = T \sum_i DF_{Iija} [U_{av} F_g C_{iv} + U_{am} C_{im} + U_{af} C_{if} + U_{al} F_l C_{il}]$$

Where:

Dja = the dose to organ j of age group a - mrem

T = fraction of year of release duration

DFIija = the ingestion dose factor for isotope i to organ j for age group a - mrem/pCi from Reg. Guide 1.109 Appendix E

Uav = Ingestion rate (usage factor) for non-leafy vegetables, grains, and fruits for age group a from Reg. Guide 1.109 Table E-5 or Table B-1.

Fg = the fraction of vegetables, grains, and fruits from the location of interest : 0.76 in Reg. Guide 1.109.

Civ = the concentration of isotope i in the vegetables, fruits, and grains calculated from section 2.5.3.2.

Uam = Ingestion rate (usage factor) for milk for age group a: from Table B-1 or Reg. Guide 1.109 Table E-5.

Cim = the concentration of isotope i in milk calculated from section 2.5.3.5.

Uaf = the ingestion rate for meat for age group a: from Table B-1 or Reg. Guide 1.109 Table E-5.

- Cif = the concentration of isotope i in meat calculated from section 2.5.3.2.
- Ual = the ingestion rate for leafy vegetables for age group a: from Table B-1 or Reg. Guide 1.109 Table E-5.
- FI = the fraction of annual leafy vegetable ingestion from the location of interest : 1.0 in Reg. Guide 1.109.
- Cil = concentration of isotope i in the leafy vegetables for direct human consumption: Civ calculated from section 2.5.3.2 with Th=0.

2.5.3.8 EXAMPLE CALCULATION - DOSE FROM CONSUMPTION OF MILK, MEAT, AND VEGETABLES

Calculate the ingestion dose to child whole body from a ground level release of 100 μCi of Co-60 in 10 hours. Plume transit decay time is ignored ($\exp(-\lambda_i T_r)=1$):

$$D_{ja} = T \sum_i DF_{Iija} [U_{av} F_g C_{iv} + U_{am} C_{im} + U_{af} C_{if} + U_{al} F_l C_{il}]$$

Where:

- T = 0.00114 yr [10hrs / 8760 hrs/yr]
DFIija = 1.56E-5 mrem/pCi
Uav = 520
Fg = 0.76
Civ = 17.6 [18.0*EXP(- λ *60) using 60 day delay for ingestion]
Uam = 330
Cim = 0.88
Uaf = 41
Cif = 11.5
Ual = 26
Fl = 1
Cil = 17.7

$$D_{ja} = .00114 \sum_i 1.56E - 5 [520 * 0.76 * 17.6 + 330 * 0.88 + 41 * 11.5 + 26 * 1 * 17.7]$$

$$D_{ja} = .00114 \sum_i 1.56E - 5 [6956 + 290 + 472 + 460]$$

$$D_{ja} = 1.45E - 4 \text{ mrem: child : wholebody}$$

2.5.4 GROUND PLANE DEPOSITION IRRADIATION

Dose from ground plane deposition is estimated by determining the surface activity resulting from the release.

2.5.4.1 GROUND PLANE CONCENTRATION

The ground surface activity is estimated as:

$$C_{ig} = \frac{D}{Q} \frac{Q_i}{\lambda_i} (1 - EXP(-\lambda_i T_b))$$

Where:

C_{ig} = ground plane concentration of radionuclide i in pCi/m²

$\frac{D}{Q}$ = local atmospheric release deposition factor in 1/m² from Table 2.5-1

Q_i = release rate in pCi/sec

λ_i = radiological decay constant in 1/sec

T_b = long term buildup time 30 years (9.46E8 sec)

Note: Q_i , λ_i and T_b can utilize any time units as long as they are all the same

2.5.4.2 EXAMPLE GROUND PLANE CONCENTRATION CALCULATION

Calculate the ground plane concentration from a 100 μCi release of Co-60 over 10 hours from a ground level release point.

$$C_{ig} = \frac{D}{Q} \frac{Q_i}{\lambda_i} (1 - EXP(-\lambda_i T_b))$$

$\frac{D}{Q}$ = 1.67E-8 /m²

Q_i = 2778 pCi/sec [100μCi/10hrs/3600sec/hr]

λ_i = 4.17E-9/sec [0.693/(5.27yr*8760hr/yr*3600sec/hr)]

T_b = 9.46E8 sec

$$C_{ig} = 1.67E-8 \frac{2778}{4.17E-9} (1 - EXP(-4.17E-9 * 9.46E8))$$

$$C_{ig} = 1.09E4 pCi / m^2$$

2.5.4.3 GROUND PLANE DOSE

Annual dose from the ground plane deposition is of the form:

$$D_{jg} = 8760 * T * Sf \sum_i C_{ig} DFG_{ij}$$

Where:

D_{jg} = the annual dose (mrem) from ground plane pathway (g) to the total body or skin (j)

8760 = hours in a year

T = fraction of year release is in progress

Sf = shielding factor accounting for shielding from dwelling from Table B-1

DFG_{ij} = Ground plane dose factor for skin or total body (j) for radionuclide i from Table E-6 of Reg. Guide 1.109 in mrem/hr / pCi/m².

2.5.4.4 EXAMPLE GROUND PLANE DOSE

Calculate the ground plane Total Body dose from a 100 μCi release of Co-60 over 10 hours from a ground level release point.

$$D_{jg} = 8760 * T * Sf \sum_i C_{ig} DFG_{ij}$$

T = 0.00114 [10/8760]

Sf = 0.7

DFG_{ij} = 1.7E-8

C_{ig} = 1.09E4

$$D_{jg} = 8760 * 0.00114 * 0.7 \sum_i 1.09E4 * 1.7E-8$$

$$D_{jg} = 1.30E-3 \text{ mrem Total Body}$$

2.6 PROJECTED DOSES – GASEOUS

The projected doses in a 31 day period are equal to the calculated doses from the current 31 day period.

3.0 TOTAL DOSE TO MEMBERS OF THE PUBLIC - 40 CFR 190

The Radiological Environmental Monitoring Report (REMP) submitted by May 1st of each year shall include an assessment of the radiation dose to the likely most exposed MEMBER OF THE PUBLIC for reactor releases and other nearby uranium fuel cycle

sources (including dose contributions from effluents and direct radiation from on-site sources). For the likely most exposed MEMBER OF THE PUBLIC in the vicinity of Oyster Creek, the sources of exposure need only consider the Oyster Creek Generating Station. No other fuel cycle facilities would contribute significantly to the MEMBER OF THE PUBLIC dose for the Oyster Creek vicinity, however, both plant operation and ISFSI sources must be included in the dose assessment.

To assess compliance with CONTROL 3.11.4, calculated organ and total body doses from effluents from liquid pathways and atmospheric releases as well as any dose from direct radiation will be summed.

As appropriate for demonstrating/evaluating compliance with the limits of CONTROL 3.11.4 (40 CFR 190), the results of the environmental monitoring program may be used for providing data on actual measured levels of radiation and / or radioactive material and resultant dose to the MEMBER OF THE PUBLIC in the actual pathways of exposure.

3.1 EFFLUENT DOSE CALCULATIONS

For purposes of implementing the surveillance requirements of CONTROL 3/4.11.4 and the reporting requirements of Technical Specification 6.9.1.d (ARERR), dose calculations for the Oyster Creek Generating Station may be performed using the calculation methods contained within the ODCM; the conservative controlling pathways and locations from the ODCM or the actual pathways and locations as identified by the land use census (CONTROL 3/4.12.1) may be used. Average annual meteorological dispersion parameters provided herein or meteorological conditions concurrent with the release period under evaluation may be used.

3.2 DIRECT EXPOSURE DOSE DETERMINATION

Any potentially significant direct exposure contribution to off-site individual doses may be evaluated based on the results of environmental measurements (e.g., dosimeter) and/or by the use of radiation transport and shielding calculation methodologies.

4.0 RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM

The operational phase of the Radiological Environmental Monitoring Program (REMP) is conducted in accordance with the requirements of CONTROL 3.12.1. The objectives of the program are:

- To determine whether any significant increases occur in the concentration of radionuclides in the critical pathways of exposure in the vicinity of Oyster Creek
- To determine if the operation of the Oyster Creek Generating Station has resulted in any increase in the inventory of long lived radionuclides in the environment;
- To detect any changes in the ambient gamma radiation levels; and
- To verify that OCGS operations have no detrimental effects on the health and safety of the public or on the environment.

The REMP sample locations are presented in Appendix E.

APPENDIX A - DERIVED DOSE FACTORS AND RECEPTOR LOCATIONS

Table A-1 Dose Conversion Factors for Deriving Radioactive Noble Gas Radionuclide-to-Dose Equivalent Rate Factors*

<u>Radionuclide</u>	<u>Factor DFSi for Stack Release*</u>	<u>Factor DFVi for Ground-level or Vent Release**</u>
	<u>mrem-sec</u> <u>μCi-year</u>	<u>mrem-m³</u> <u>μCi-year</u>
Kr83m	9.21E-10	7.56E-02
Kr85m	1.46E-04	1.17E+03
Kr85	2.58E-06	1.61E+01
Kr87	8.65E-04	5.92E+03
Kr88	2.16E-03	1.47E+04
Kr89	2.06E-03	1.66E+04
Kr90	-	1.56E+04
Xe131m	3.13E-05	9.15E+01
Xe133m	2.50E-05	2.51E+02
Xe133	2.15E-05	2.94E+02
Xe135m	4.81E-04	3.12E+03
Xe135	2.51E-04	1.81E+03
Xe137	1.79E-04	1.42E+03
Xe138	1.37E-03	8.83E+03
Xe139	2.14E-04	5.02E+03
Ar41	1.67E-03	8.84E+03

* Based on meteorology applicable at 229 meters SW of stack.

** For exposure to a semi-infinite cloud of noble gas.

Table A-2 Noble Gas Radionuclide-to-Dose Equivalent Rate Factors*

Radionuclide	$P_{\gamma}Si^{**}$ <u>mrem</u> μCi	$P_{\gamma}Vi^{***}$ <u>mrem-m³</u> $\mu Ci\text{-sec} (K_i)$	$A_{\gamma}Vi^{***}$ <u>mrad-m³</u> $\mu Ci\text{-sec} (M_i)$	SBi^{***} <u>mrem-m³</u> $\mu Ci\text{-sec} (L_i)$
Kr83m	2.92E-17	2.40E-09	6.13E-07	-
Kr85m	4.64E-12	3.71E-05	3.90E-05	4.63E-05
Kr85	8.18E-14	5.11E-07	5.46E-07	4.25E-05
Kr87	2.74E-11	1.88E-04	1.96E-04	3.09E-04
Kr88	6.84E-11	4.67E-04	4.83E-04	7.52E-05
Kr89	6.53E-11	5.27E-04	5.49E-04	3.21E-04
Kr90	-	4.95E-04	5.17E-04	2.31E-04
Xe131m	9.92E-13	2.90E-06	4.95E-06	1.51E-05
Xe133m	7.94E-13	7.97E-06	1.04E-05	3.16E-05
Xe133	6.83E-13	9.33E-06	1.12E-05	9.71E-06
Xe135m	1.53E-11	9.90E-05	1.07E-04	2.26E-05
Xe135	7.97E-12	5.75E-05	6.10E-05	5.90E-05
Xe137	5.69E-12	4.51E-05	4.79E-05	3.87E-04
Xe138	4.34E-11	2.80E-04	2.92E-04	1.31E-04
Xe139	6.79E-12	-	-	-
Ar41	5.30E-11	2.81E-04	2.95E-04	8.54E-05

* All of these dose factors apply out-of-doors.

** Based on meteorology at 229 meters SW of effluent stack.

*** Derived from Reg Guide 1.109, Revision 1, Table B-1.

Table A-3 Air Dose Conversion Factors for Effluent Noble Gas

<u>Radionuclide</u>	$A_{\gamma Si}^{**}$ <u>mrad</u> μCi	$A_{\gamma Vi}^{***}$ <u>mrad-m3</u> $\mu Ci\text{-sec}(M_i)$	$A_{\beta i}^{***}$ <u>mrad-m3</u> $\mu Ci\text{-sec}(N_i)$
Kr83m	1.33E-16	6.13E-07	9.14E-06
Kr85m	6.89E-12	3.90E-05	6.25E-05
Kr85	1.24E-13	5.46E-07	6.19E-05
Kr87	4.13E-11	1.96E-04	3.27E-04
Kr88	1.03E-10	4.83E-04	9.30E-05
Kr89	9.82E-11	5.49E-04	3.37E-04
Kr90	-	5.17E-04	2.49E-04
Xe131m	1.50E-12	4.95E-06	3.52E-05
Xe133m	1.23E-12	1.04E-05	4.70E-05
Xe133	1.03E-12	1.12E-05	3.33E-05
Xe135m	2.31E-11	1.07E-04	2.35E-05
Xe135	1.20E-11	6.10E-05	7.81E-05
Xe137	8.59E-12	4.79E-05	4.03E-04
Xe138	6.51E-11	2.92E-04	1.51E-04
Xe139	1.02E-11	-	-
Ar41	7.94E-11	2.95E-04	1.04E-04

** Based on meteorology at 229 meters SW of effluent stack.

*** Derived from Reg Guide 1.109, Revision 1, Table B-1.

Table A-4 Locations Associated with Maximum Exposure of a Member of the Public*

<u>Effluent</u>	<u>Distance</u> (meters)	<u>Location</u>	<u>Direction</u> (to)
Liquid		U.S. Route 9 Bridge at Discharge Canal	
Airborne Iodine and Particulates	937		SE
Tritium	937		SE
Noble Gases	937		SE
Irradiation by OCGS		Site Boundary	All
Noble Gas g Air Dose	937		SE
Noble Gas B Air Dose	937		SE

Note: the nearby resident experiencing the maximum exposure to airborne effluent from the Station is located 937 meters SE of the OCGS. The nearby resident (part-time) experiencing the maximum exposure to gamma radiation directly from the Station is located 618 meters WSW of the OCGS. The most exposed member of the public is assumed to be exposed by irradiation from the OCGS, by inhaling airborne effluent, by irradiation by the airborne effluent, by irradiation by the airborne plume of the noble gas, by radionuclides deposited onto the ground, by irradiation by shoreline deposits, and by eating fish and shellfish caught in the discharge canal.

*The age group of the most exposed member of the public is based on Reg. Guide 1.109, Revision 1.

Table A-5 Critical Receptor Noble Gas Dose Conversion Factors*

Radionuclide	$P_{\gamma}Si^{**}$ $\frac{mrem}{\mu Ci}$	$P_{\gamma}Vi^{***}$ $\frac{mrem \cdot m^3}{\mu Ci \cdot sec(K_i)}$	$A_{\gamma}Vi^{***}$ $\frac{mrad \cdot m^3}{\mu Ci \cdot sec(M_i)}$	$A_{\gamma}Si^{**}$ $\frac{mrad}{\mu Ci}$	SBi^{***} $\frac{mrem \cdot m^3}{\mu Ci \cdot sec(L_i)}$
Kr83m	4.61E-17	2.40E-09	6.13E-07	1.77E-14	-
Kr85m	2.13E-12	3.71E-05	3.90E-05	3.16E-12	4.63E-05
Kr85	3.38E-14	5.11E-07	5.46E-07	5.12E-14	4.25E-05
Kr87	1.08E-11	1.88E-04	1.96E-04	1.63E-11	3.09E-04
Kr88	2.76E-11	4.67E-04	4.83E-04	4.14E-11	7.52E-05
Kr89	2.02E-11	5.27E-04	5.49E-04	3.03E-11	3.21E-04
Kr90	-	4.95E-04	5.17E-04	-	2.31E-04
Xe131m	5.05E-13	2.90E-06	4.95E-06	8.03E-13	1.51E-05
Xe133m	3.95E-13	7.97E-06	1.04E-05	6.50E-13	3.16E-05
Xe133	3.88E-13	9.33E-06	1.12E-05	6.13E-13	9.71E-06
Xe135m	5.82E-12	9.90E-05	1.07E-04	8.80E-12	2.26E-05
Xe135	3.51E-12	5.75E-05	6.10E-05	5.25E-12	5.90E-05
Xe137	1.74E-12	4.51E-05	4.79E-05	2.64E-12	3.87E-04
Xe138	1.72E-11	2.80E-04	2.92E-04	2.58E-11	1.31E-04
Xe139	9.30E-13	-	-	1.40E-12	-
Ar41	2.07E-11	2.81E-04	2.95E-04	3.10E-11	8.54E-05

* All of these dose factors apply out-of-doors.

** Based on meteorology at 937 meters SE of effluent stack.

*** Derived from Reg Guide 1.109, Revision 1, Table B-1

APPENDIX B - MODELING PARAMETERS

Table B-1- OCGS Usage Factors For Individual Dose Assessment

<u>Effluent Ingestion Parameters</u>	<u>Usage Factor</u>
Fraction Of Produce From Local Garden	7.6E-1
Soil Density In Plow Layer (Kg/m ²)	2.4E+2
Fraction Of Deposited Activity Retained On Vegetation	2.5E-1
Shielding Factor For Residential Structures	7.0E-1
Period Of Buildup Of Activity In Soil (hr)	1.31E+5
Period of Pasture Grass Exposure to Activity (hr)	7.2E+2
Period Of Crop Exposure to Activity (hr)	1.44E+3
Delay Time For Ingestion Of Stored Feed By Animals (hr)	2.16E+3
Delay Time For Ingestion Of Leafy Vegetables By Man (hr)	2.4E+1
Delay Time For Ingestion Of Other Vegetables By Man (hr)	1.44E+3
Transport Time Milk-Man (hr)	4.8E+1
Time Between Slaughter and Consumption of Meat Animal (hr)	4.8E+2
Grass Yield Wet Weight (Kg/m ²)	7.0E-1
Other Vegetation Yield Wet-Weight (Kg/m ²)	2.0
Weathering Rate Constant For Activity on Veg. (hr ⁻¹)	2.1E-3
Milk Cow Feed Consumption Rate (Kg/day)	5.0E+1
Goat Feed Consumption Rate (Kg/day)	6.0
Beef Cattle Feed Consumption Rate (Kg/day)	5.0E+1
Milk Cow Water Consumption Rate (L/day)	6.0E+1
Goat Water Consumption Rate (L/day)	8.0
Beef Cattle Water Consumption Rate (L/day)	5.0E+1
Environmental Transit Time For Water Ingestion (hr)	1.2E+1
Environmental Transit Time For Fish Ingestion (hr)	2.4E+1
Environmental Transit Time For Shore Exposure (hr)	0
Environmental Transit Time For Invertebrate Ingestion (hr)	2.4E+1

Table B-1(Continued)
OCGS Usage Factors for Individual Dose Assessment

<u>Effluent Ingestion Parameters</u>	<u>Usage Factor</u>
Water Ingestion (L/yr)	
a. Adult	7.3E+2
b. Teen	5.1E+2
c. Child	5.1E+2
d. Infant	3.3E+2
Shore Exposure (hr/yr)	
a. Adult	1.2E+1
b. Teen	6.7E+1
c. Child	1.4E+1
d. Infant	0
Salt Water Sport Fish Ingestion (Kg/yr)	
a. Adult	2.1E+1
b. Teen	1.6E+1
c. Child	6.9
d. Infant	0
Salt Water Commercial Fish Ingestion (Kg/yr)	
a. Adult	2.1E+1
b. Teen	1.6E+1
c. Child	6.9
d. Infant	0
Salt Water Invertebrate Ingestion (Kg/yr)	
a. Adult	5.0
b. Teen	3.8
c. Child	1.7
d. Infant	0
Irrigated Leafy Vegetable Ingestion (Kg/yr)	
a. Adult	6.4E+1
b. Teen	4.2E+1
c. Child	2.6E+1
d. Infant	0

Table B-1 (Continued)
OCGS Usage Factors for Individual Dose Assessment

<u>Effluent Ingestion Parameters</u>	<u>Usage Factor</u>
Irrigated Other Vegetable Ingestion (Kg/yr)	
a. Adult	5.2E+2
b. Teen	6.3E+2
c. Child	5.2E+2
d. Infant	0
Irrigated Root Vegetable Ingestion (Kg/yr)	
a. Adult	5.2E+2
b. Teen	6.3E+2
c. Child	5.2E+2
d. Infant	0
Irrigated Cow and Goat Milk Ingestion (L/yr)	
a. Adult	3.1E+2
b. Teen	4.0E+2
c. Child	3.3E+2
d. Infant	3.3E+2
Irrigated Beef Ingestion (Kg/yr)	
a. Adult	1.1E+2
b. Teen	6.5E+1
c. Child	4.1E+1
d. Infant	0
Inhalation (m ³ /yr)	
a. Adult	8.0E+3
b. Teen	8.0E+3
c. Child	3.7E+3
d. Infant	1.4E+3
Cow and Goat Milk Ingestion (L/yr)	
a. Adult	3.1E+2
b. Teen	4.0E+2
c. Child	3.3E+2
d. Infant	3.3E+2
Meat Ingestion (Kg/yr)	
a. Adult	1.1E+2
b. Teen	6.5E+1
c. Child	4.1E+1
d. Infant	0

Table B-1 (Continued)
OCGS Usage Factors for Individual Dose Assessment

<u>Effluent Ingestion Parameters</u>	<u>Usage Factor</u>
Leafy Vegetable Ingestion (Kg/yr)	
a. Adult	6.4E+1
b. Teen	4.2E+1
c. Child	2.6E+1
d. Infant	0
Fruits, Grains, & Other Vegetable Ingestion (Kg/yr)	
a. Adult	5.2E+2
b. Teen	6.3E+2
c. Child	5.2E+2
d. Infant	0

Table B-2 Monthly Average Absolute Humidity g/m³
(derived from historical climatological data)

<u>Month</u>	<u>Average Absolute Humidity (g/m³)</u>
January	3.3
February	3.3
March	4.5
April	6.1
May	9.4
June	12.8
July	15.2
August	15.6
September	12.4
October	7.9
November	5.9
December	3.8

APPENDIX C - REFERENCES

Table C-1 - REFERENCES

- 1) Oyster Creek Updated Final Safety Analysis Report
- 2) Oyster Creek Facility Description and Safety Analysis Report
- 3) Oyster Creek Operating License and Technical Specifications
- 4) NUREG 1302 "Off Site Dose Calculation Manual Guidance: Standard Radiological Effluent Controls for Boiling Water Reactors" – Generic Letter 89-10, Supplement No. 1, April 1991
- 5) Reg Guide 1.21 "Measuring, Evaluating, and Reporting Radioactivity in Solid Wastes and Releases of radioactive materials in Liquid and Gaseous Effluents from Light-Water-Cooled Nuclear Power Plants" Rev.1, June 1974
- 6) Reg Guide 1.23
- 7) Reg Guide 1.97
- 8) Reg Guide 1.109 "Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents for the Purpose of Evaluating Compliance With 10 CFR 50, Appendix I", Rev 1, October, 1977
- 9) Reg Guide 1.111 "Methods for Estimating Atmospheric Transport and Dispersion of Gaseous Effluents in Routine Releases From Light-Water-Cooled Reactors", Rev.1, July, 1977
- 10) Reg Guide 4.8 " Environmental Technical Specifications for Nuclear Power Plants"
- 11) NRC Radiological Assessment Branch Technical Position, Rev 1, November 1979 (Appendix A to NUREG1302)
- 12) NUREG-0016
- 13) NUREG-0133
- 14) Licensing Application, Amendment 13, Meteorological Radiological Evaluation for the Oyster Creek Nuclear Power Station Site.
- 15) Licensing Application, Amendment 11, Question IV-8.
- 16) Evaluation of the Oyster Creek Nuclear Generating Station to Demonstrate Conformance to the Design Objectives of 10CFR50, Appendix I, May, 1976, Tables 3-10
- 17) XOQDOQ Output Files for Oyster Creek Meteorology, Murray and Trettle, Inc.

- 18) Hydrological Information and Liquid Dilution Factors Determination to Conform with Appendix I Requirements: Oyster Creek, correspondence from T. Potter, Pickard, Lowe and Garrick, Inc. to Oyster Creek, July, 1976.
- 19) Carpenter, J. J. "Recirculation and Effluent Distribution for Oyster Creek Site", Pritchard-Carpenter Consultants, Baltimore, Maryland, 1964.
- 20) Nuclear Regulatory Commission, Generic Letter 89-01, "Implementation of Programmatic Controls for Radiological Effluent Technical Specifications in the Administrative Controls Section and Relocation of the Procedural Details of RETS to the ODCM or PCP", January, 1989.
- 21) Ground Water Monitoring System (Final Report), Woodward-Clyde Consultants, March, 1984.
- 22) Meteorology and Atomic Energy, Department of Energy, 1981.
- 23) SEEDS Code Documentation through V & V of Version 98.8F (Radiological Engineering Calculation No. 2820-99-005, Dated 3/23/99)
- 24) Lynch, Giuliano, and Associates, Inc., Drawing Entitled, "Minor Subdivision, Lots 4 and 4.01 Block 1001", signed 13 Sep 99.
- 25) Currie, L. A., "Lower Limit of Detection: Definition and Elaboration of a Proposed Position for Radiological Effluent and Environmental Measurements".
- 26) NUREG/CR-4007 (September 1984).
- 27) HASL Procedures Manual, HASL-300 (revised annually).
- 28) Regulatory Guide 1.113, "Estimating Aquatic Dispersion of Effluents from Accidental and Routine Reactor Releases for the Purposes of Implementing Appendix I," April 1977
- 29) Reg. Guide 4.13
- 30) 10CFR20, Appendix B, Table 2, Annual Limits on Intake (ALIs) and Derived Air Concentrations (DACs) of Radionuclides for Occupational Exposure; Effluent Concentrations; Concentrations for Release to Sewerage
- 31) Conestoga Rovers and Associates, Hydrogeologic Investigation Report, Fleet wide Assessment, Oyster Creek Generating Station, Forked River, New Jersey, Ref. No. 045136(18), September, 2006.
- 32) Letter date April 23, 2013 from Murray and Trettel, Incorporated
- 33) Letter dated January 10, 2013 titled "Meteorology and Dose Factor Update – ODCM Revision 6"

APPENDIX D – SYSTEM DRAWINGS

FIGURE D-1-1a: LIQUID RADWASTE TREATMENT CHEM WASTE AND FLOOR DRAIN SYSTEM

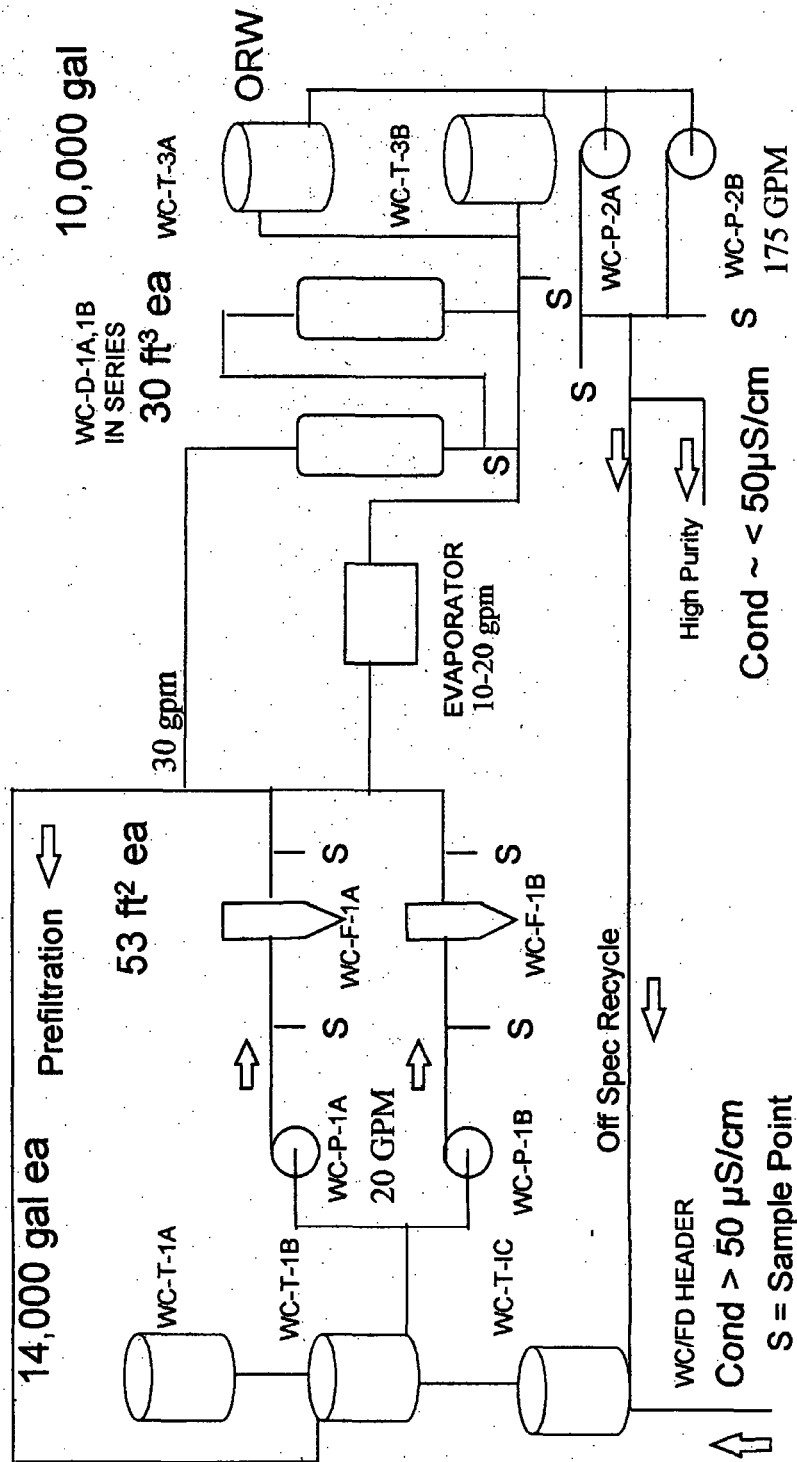


FIGURE D-1-1b: LIQUID RADWASTE TREATMENT - HIGH PURITY AND EQUIPMENT DRAIN SYSTEM

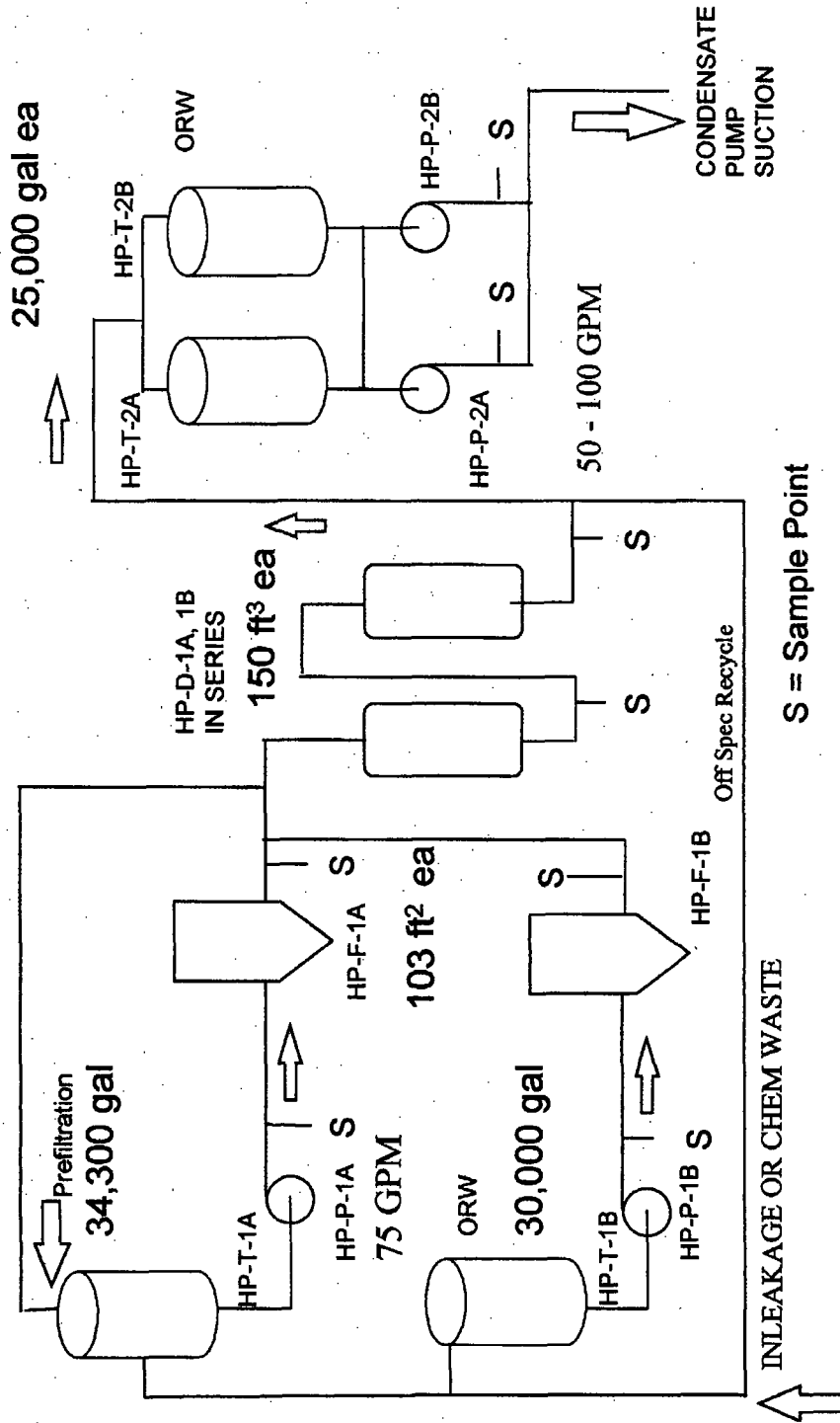


FIGURE D-1-1c: GROUNDWATER REMEDIATION SYSTEM

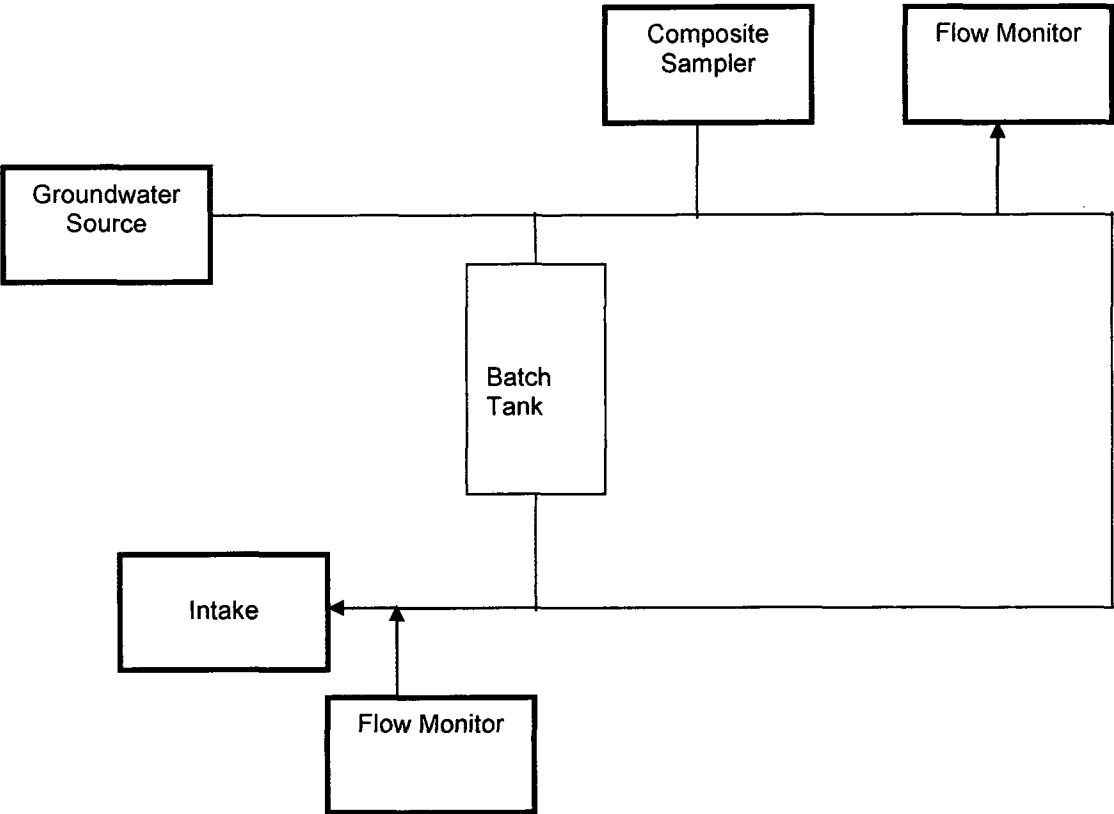


FIGURE D-1-2: SOLID RADWASTE PROCESSING SYSTEM

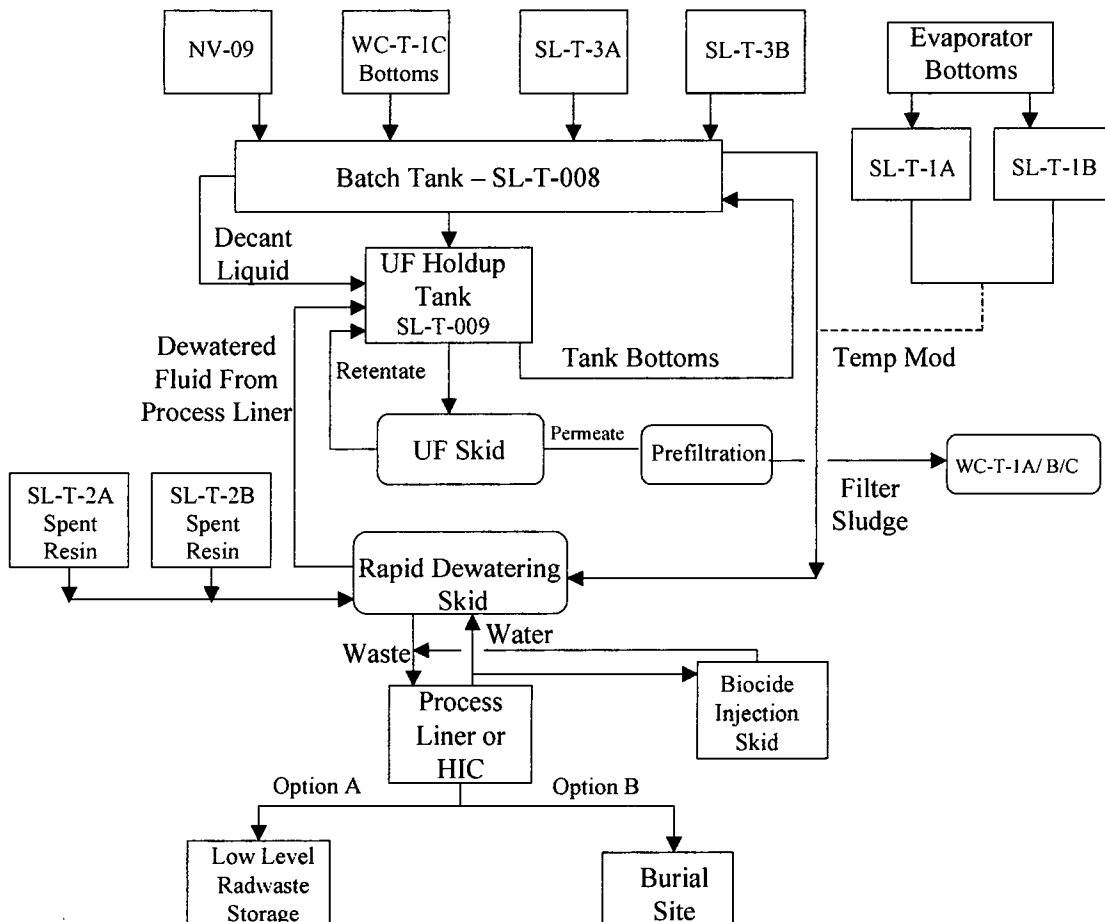


FIGURE D-2-2: VENTILATION SYSTEM

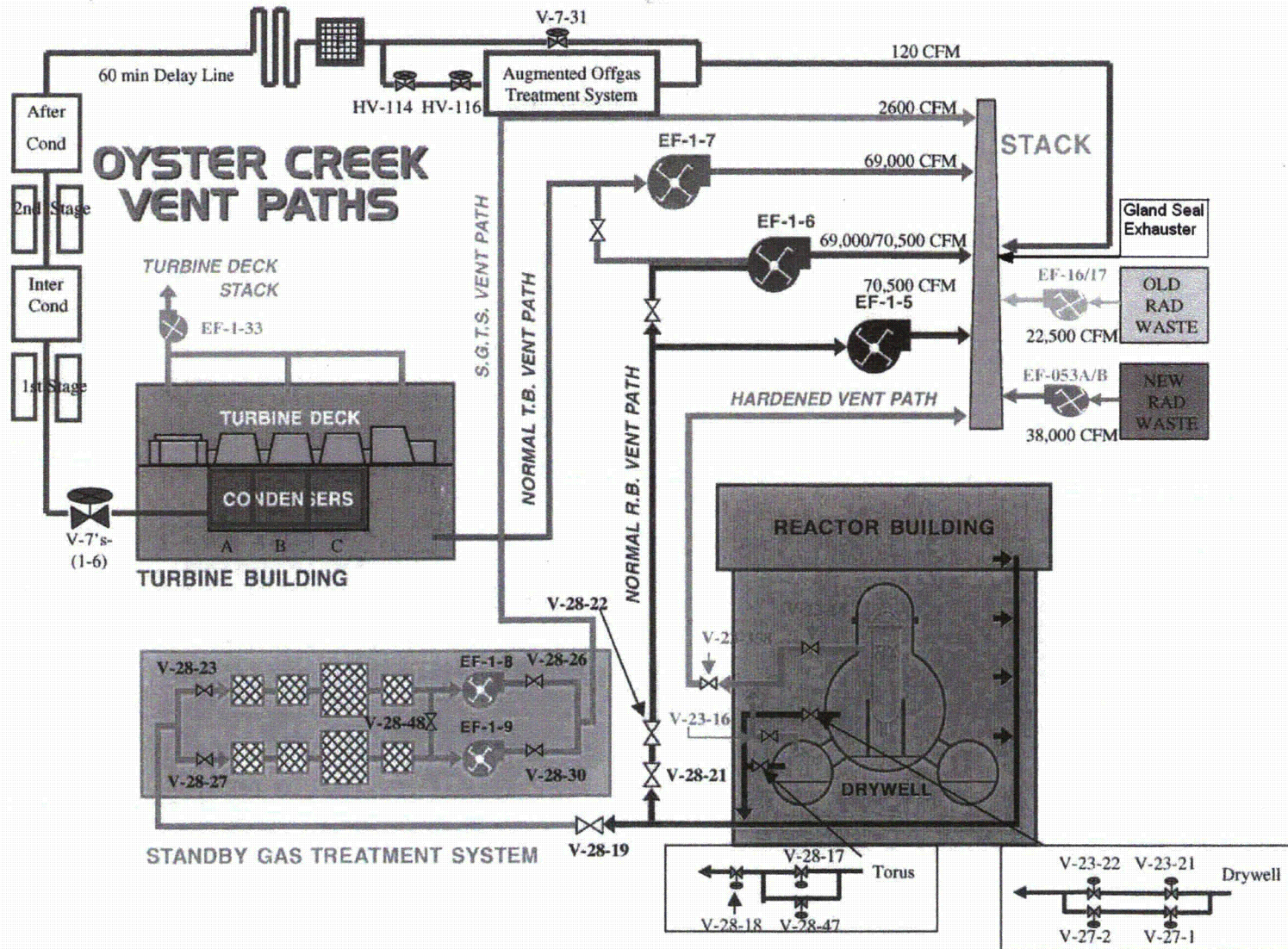
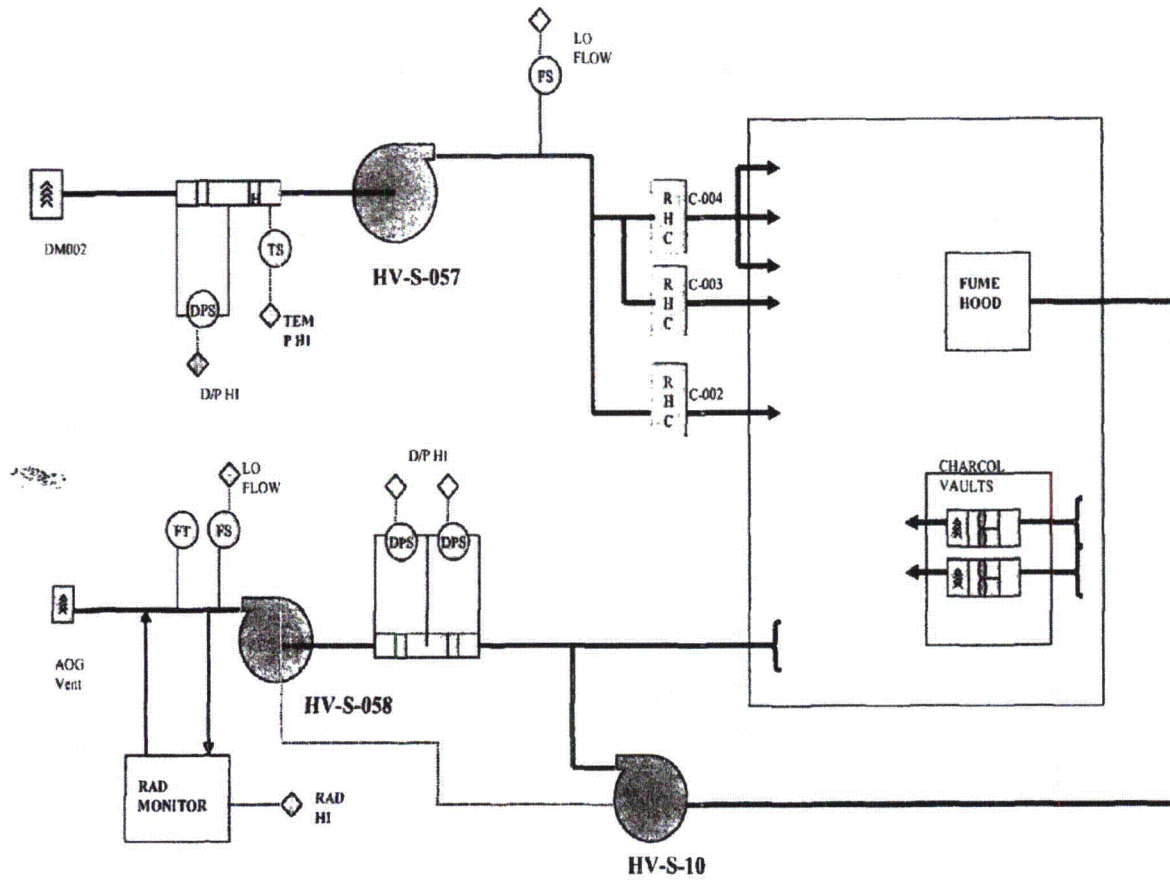


Figure D-2-3 AOG Ventilation System



APPENDIX E - RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM - SAMPLE TYPE AND LOCATION

All sampling locations and specific information about the individual locations are given in Table E-1. Figures E-1, E-2 and E-3 show the locations of sampling stations with respect to the site. Figure E-4 shows the site layout.

TABLE E-1: REMP SAMPLE LOCATIONS⁽¹⁾

1. Direct Radiation

DOS - Inner Ring at or near site boundary

<u>Code</u>	<u>(miles)</u>	<u>(degrees)</u>	<u>Description</u>
1	0.4	219	SW of site at OCGS Fire Pond, Forked River, NJ
51	0.4	358	North of site, on the access road to Forked River Site, Forked River, NJ
52	0.3	333	NNW of site, on the access road to Forked River Site, Forked River, NJ
53	0.3	309	NW of site, at sewage lift station on the access road to the Forked River Site, Forked River, NJ
54	0.3	288	WNW of site, on the access road to Forked River Site, Forked River, NJ
55	0.3	263	West of site, on Southern Area Stores security fence, west of OCGS Switchyard, Forked River, NJ
56	0.3	249	WSW of site, on utility pole east of Southern Area Stores, west of the OCGS Switchyard, Forked River, NJ
57	0.2	206	SSW of site, on Southern Area Stores access road, Forked River, NJ
58	0.2	188	South of site, on Southern Area Stores access road, Forked River, NJ
59	0.3	166	SSE of site, on Southern Area Stores access road, Waretown, NJ
61	0.3	104	ESE of site, on Route 9 south of OCGS Main Entrance, Forked River, NJ
62	0.2	83	East of site, on Route 9 at access road to OCGS Main Gate, Forked River, NJ
63	0.2	70	ENE of site, on Route 9, between main gate and OCGS North Gate access road, Forked River, NJ
64	0.3	42	NE of site, on Route 9 North at entrance to Finninger Farm, Forked River, NJ
65	0.4	19	NNE of site, on Route 9 at Intake Canal Bridge, Forked River, NJ
66	0.4	133	SE of site, east of Route 9 and south of the OCGS Discharge Canal, inside fence, Waretown, NJ
112	0.2	178	S of site, along Southern access road, Lacey Township, NJ

TABLE E-1: REMP SAMPLE LOCATIONS (CONTINUED)

1. Direct Radiation (Continued)

DOS - Inner Ring at or near site boundary

<u>Code</u>	<u>(miles)</u>	<u>(degrees)</u>	<u>Description</u>
113	0.3	90	E of site, along Rt. 9 North, Lacey Township, NJ
T1	0.4	219	SW of site, at OCGS Fire Pond, Lacey Township, NJ

DOS - Outer Ring at 6 - 8 km

4	4.6	213	SSW of Site, Garden State Parkway and Route 554, Barnegat, NJ
5	4.2	353	North of Site, Garden State Parkway Rest Area, Forked River, NJ
6	2.1	13	NNE of site, Lane Place, behind St. Pius Church, Forked River, NJ
8	2.3	177	South of site, Route 9 at the Waretown Substation, Waretown, NJ
9	2.0	230	SW of site, where Route 532 and the Garden State Parkway meet, Waretown, NJ
22	1.6	145	SE of site, on Long John Silver Way, Skippers Cove, Waretown, NJ
46	5.6	323	NW of Site, on Lacey Road adjacent to Utility Pole BT 259 65
47	4.6	26	NNE of Site, Route 9 and Harbor Inn Road, Berkeley Township, NJ
48	4.5	189	South of Site, Intersection of Brook and School Streets, Barnegat, NJ
68	1.3	266	West of site, on Garden State Parkway North at mile marker 71.7, Lacey Township, NJ
73	1.8	108	ESE of site, on Bay Parkway, Sands Point Harbor, Waretown, NJ
74	1.8	88	East of site, Orlando Drive and Penguin Court, Forked River, NJ
75	2.0	71	ENE of site, Beach Blvd. and Maui Drive, Forked River, NJ
78	1.8	2	North of site, 1514 Arient Road, Forked River, NJ
79	2.9	160	SSE of site, Hightide Drive and Bonita Drive, Waretown, NJ
82	4.4	36	NE of site, Bay Way and Clairmore Avenue, Lanoka Harbor, NJ
84	4.4	332	NNW of site, on Lacey Road, 1.3 miles west of the Garden State Parkway on siren pole, Lacey Township, NJ

TABLE E-1: REMP SAMPLE LOCATIONS (CONTINUED)

1. Direct Radiation (continued)

DOS - Outer Ring at 6 - 8 km (continued)

<u>Code</u>	<u>(miles)</u>	<u>(degrees)</u>	<u>Description</u>
85	3.9	250	WSW of site, on Route 532, just east of Wells Mills Park, Waretown, NJ
86	5.0	224	SW of site, on Route 554, 1 mile west of the Garden State Parkway, Barnegat, NJ
98	1.6	318	NW of site, on Garden State Parkway at mile marker 73.0, Lacey Township, NJ
99	1.5	310	NW of site, on Garden State Parkway at mile marker 72.8, Lacey Township, NJ
100	1.4	43	NE of site, Yacht Basin Plaza South off Lakdeside Dr., Lacey Township, NJ
101	1.7	49	NE of site, end of Lacey Rd., East, Lacey Township, NJ
102	1.6	344	NNW of site, end of Sheffield Dr., Barnegat Pines, Lacey Township, NJ
103	2.4	337	NNW of site, Llewellyn Parkway, Barnegat Pines, Lacey Township, NJ
104	1.8	221	SW of site, Rt. 532 West, before Garden State Parkway, Ocean Township, NJ
105	2.8	222	SW of site, Garden State Parkway North, beside mile marker 69.6, Ocean Township, NJ
106	1.2	288	NW of site, Garden State Parkway North, beside mile marker 72.2 Lacey Township, NJ
107	1.3	301	NW of Site, Garden State Parkway North, beside mile marker 72.5, Lacey Township, NJ
109	1.2	141	SE of site, Lighthouse Dr., Waretown, Ocean Township, NJ
110	1.5	127	SE of site, Tiller Drive and Admiral Way, Waretown, Ocean Township, NJ

DOS - Special Interest

11	8.2	152	SSE of site, 80 th and Anchor Streets, Harvey Cedars, NJ
71	1.6	164	SSE of site, on Route 532 at the Waretown Municipal Building, Waretown, NJ
72	1.9	25	NNE of site, on Lacey Road at Knights of Columbus Hall, Forked River, NJ

TABLE E-1: REMP SAMPLE LOCATIONS (CONTINUED)

1. Direct Radiation (continued)

DOS - Special Interest (continued)

<u>Code</u>	<u>(miles)</u>	<u>(degrees)</u>	<u>Description</u>
81	3.5	201	SSW of site, on Rose Hill Road at intersection with Barnegat Boulevard, Barnegat, NJ
88	6.6	125	SE of site, eastern end of 3 rd Street, Barnegat Light, NJ
89	6.1	108	ESE of site, Job Francis residence, Island Beach State Park
90	6.3	75	ENE of site, parking lot A-5, Island Beach State Park
92	9.0	46	NE of site, at Guard Shack/Toll Booth, Island Beach State Park
3	6.0	97	East of site, near old Coast Guard Station, Island Beach State Park Special Interest Area

DOS - Background

C	24.7	313	NW of site, JCP&L office in rear parking lot, Cookstown, NJ
14	20.8	2	North of site, Larrabee Substation on Randolph Road, Lakewood, NJ

2. Airborne - Radioiodines and Particulates

APT, AIO - At or near site boundary in highest D/Q Sectors

20	0.7	95	East of site, on Finninger Farm on south side of access road, Forked River, NJ
66	0.4	133	SE of site, east of Route 9 and south of the OCGS Discharge Canal, inside fence, Waretown, NJ
111	0.3	64	ENE of site, Finninger Farm property along access road, Lacey Township, NJ

APT, AIO -Special Interest

71	1.6	164	SSE of site, on Route 532 at the Waretown Municipal Building, Waretown, NJ
72	1.9	25	NNE of site, on Lacey Road at Knights of Columbus Hall, Forked River, NJ
73	1.8	108	ESE of site, on Bay Parkway, Sands Point Harbor, Waretown, NJ

APT, AIO - Background

C	24.7	313	NW of site, JCP&L office in rear parking lot, Cookstown, NJ
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TABLE E-1: REMP SAMPLE LOCATIONS (CONTINUED)

2. Airborne - Radioiodines and Particulates (continued)

APT, AIO – Background(continued)

<u>Code</u>	<u>(miles)</u>	<u>(degrees)</u>	<u>Description</u>
3	6.0	97	East of site, near old Coast Guard Station, Island Beach State Park Special Interest Area

3. Waterborne

SWA - Surface

23	3.6	64	ENE of site, Barnegat Bay off Stouts Creek, approximately 400 yards SE of "Flashing Light 1"
24	2.1	101	East of site, Barnegat Bay, approximately 250 yards SE of "Flashing Light 3"
33	0.4	123	ESE of site, east of Route 9 Bridge in OCGS Discharge Canal

SWA - Background

94	20.0	198	SSW of site, in Great Bay/Little Egg Harbor
----	------	-----	---

GW - Ground

W-3C	0.4	112	ESE of site on Finninger Farm adjacent to Station 35, Lacey Township, NJ
MW-24-3A	0.8	97	E of site on Finninger Farm on South side of access road, Lacey Township, NJ

DW - Drinking

1S	0.1	209	On-site southern domestic well at OCGS, Forked River, NJ
1N	0.2	349	On-site northern domestic well at OCGS, Forked River, NJ
38	1.6	197	SSW of Site, on Route 532, at Ocean Township MUA Pumping Station, Waretown, NJ

DW - Background

37	2.2	18	NNE of Site, off Boox Road at Lacey MUA Pumping Station, Forked River, NJ
39	3.5	353	N of Site, Trenton Ave. off Lacey Road Lacey Twp., MUA Pump Station, Forked River, NJ

AQS - Sediment

23	3.6	64	ENE of site, Barnegat Bay off Stouts Creek, approximately 400 yards SE of "Flashing Light 1"
24	2.1	101	East of site, Barnegat Bay, approximately 250 yards SE of "Flashing Light 3"
33	0.4	123	ESE of site, east of Route 9 Bridge in OCGS Discharge Canal

TABLE E-1: REMP SAMPLE LOCATIONS (CONTINUED)

3. Waterborne (continued)

AQS - Background

<u>Code</u>	<u>(miles)</u>	<u>(degrees)</u>	<u>Description</u>
94	20.0	198	SSW of site, in Great Bay/Little Egg Harbor

4. Ingestion

FISH - Fish

93	0.1	242	WSW of site, OCGS Discharge Canal between Pump Discharges and Route 9, Forked River, NJ
----	-----	-----	---

FISH - Background

94	20.0	198	SSW of site, in Great Bay/Little Egg Harbor
----	------	-----	---

CLAM - Clams

23	3.6	64	ENE of site, Barnegat Bay off Stouts Creek, approximately 400 yards SE of "Flashing Light 1"
24	2.1	101	East of site, Barnegat Bay, approximately 250 yards SE of "Flashing Light 3"

CLAM - Background

94	20.0	198	SSW of site, in Great Bay/Little Egg Harbor
----	------	-----	---

CRAB - Crabs

33	0.4	123	ESE of site, east of Route 9 Bridge in OCGS Discharge Canal
93	0.1	242	WSW of site, OCGS Discharge Canal between Pump Discharges and Route 9, Forked River, NJ

VEG - Vegetation

35	0.4	111	ESE of site, east of Route 9 and north of the OCGS Discharge Canal, Forked River, NJ
66	0.4	133	SE of site, east of Route 9 and south of the OCGS Discharge Canal, inside fence, Waretown, NJ
115	0.3	96	East of Site, on Finninger Farm

VEG - Background

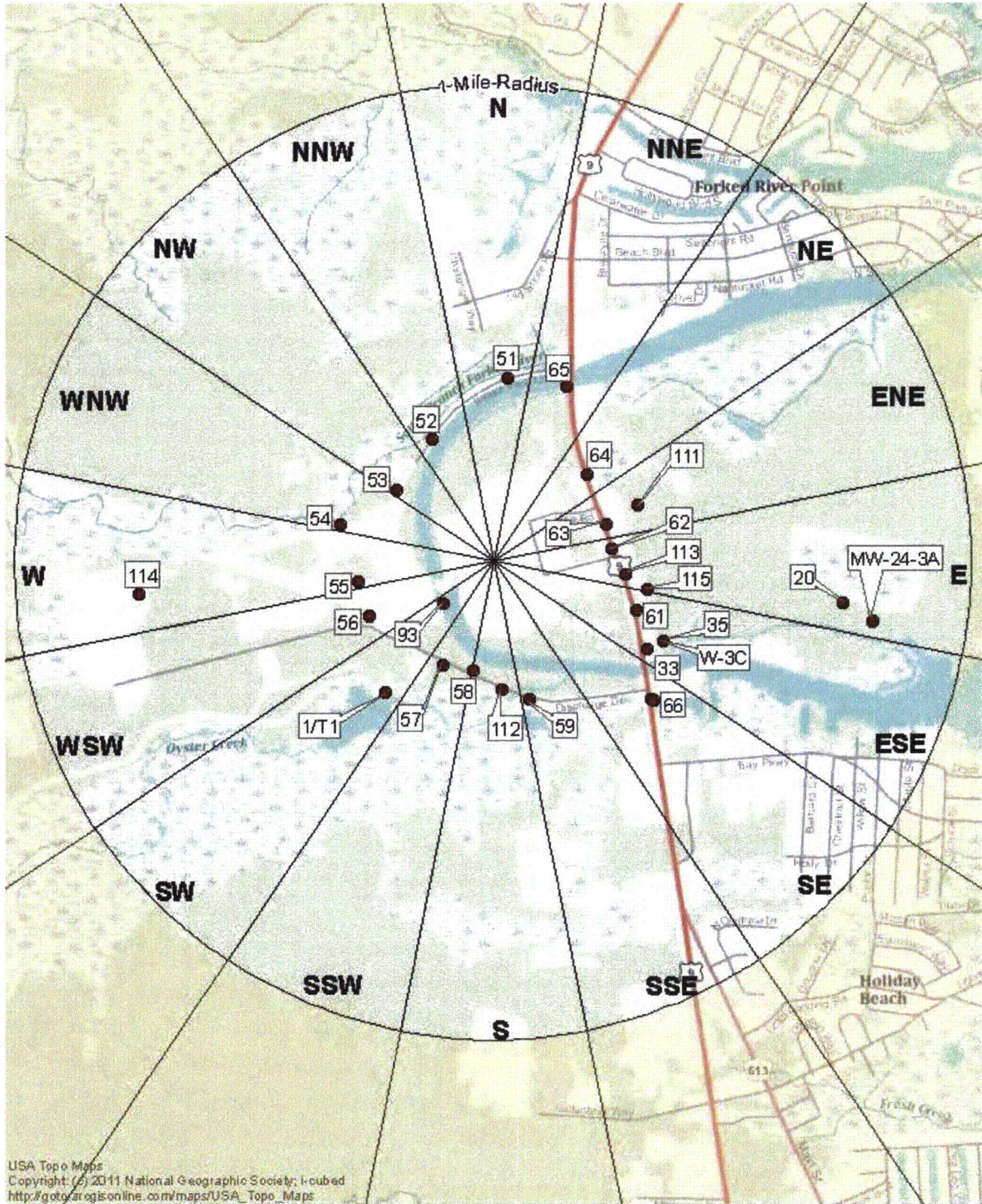
36	23.1	319	NW of site, at "U-Pick" Farm, New Egypt, NJ
----	------	-----	---

SAMPLE MEDIUM IDENTIFICATION KEY

APT = Air Particulate	SWA = Surface Water	DOS = Dosimeter
AIO = Air Iodine	AQS = Aquatic Sediment	FISH = Fish
	CLAM = Clams	CRAB =Crab
VEG = Vegetables	DW = Drinking Water	GW = Ground Water

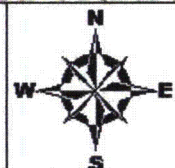
(1) Samples may not be collected from some locations listed in this table, as long as the minimum number of samples listed in Table 3.12.1-1 is collected.

FIGURE E-1



**Oyster Creek Generating Station
REMP Sample Locations
within a 1 Mile Radius**

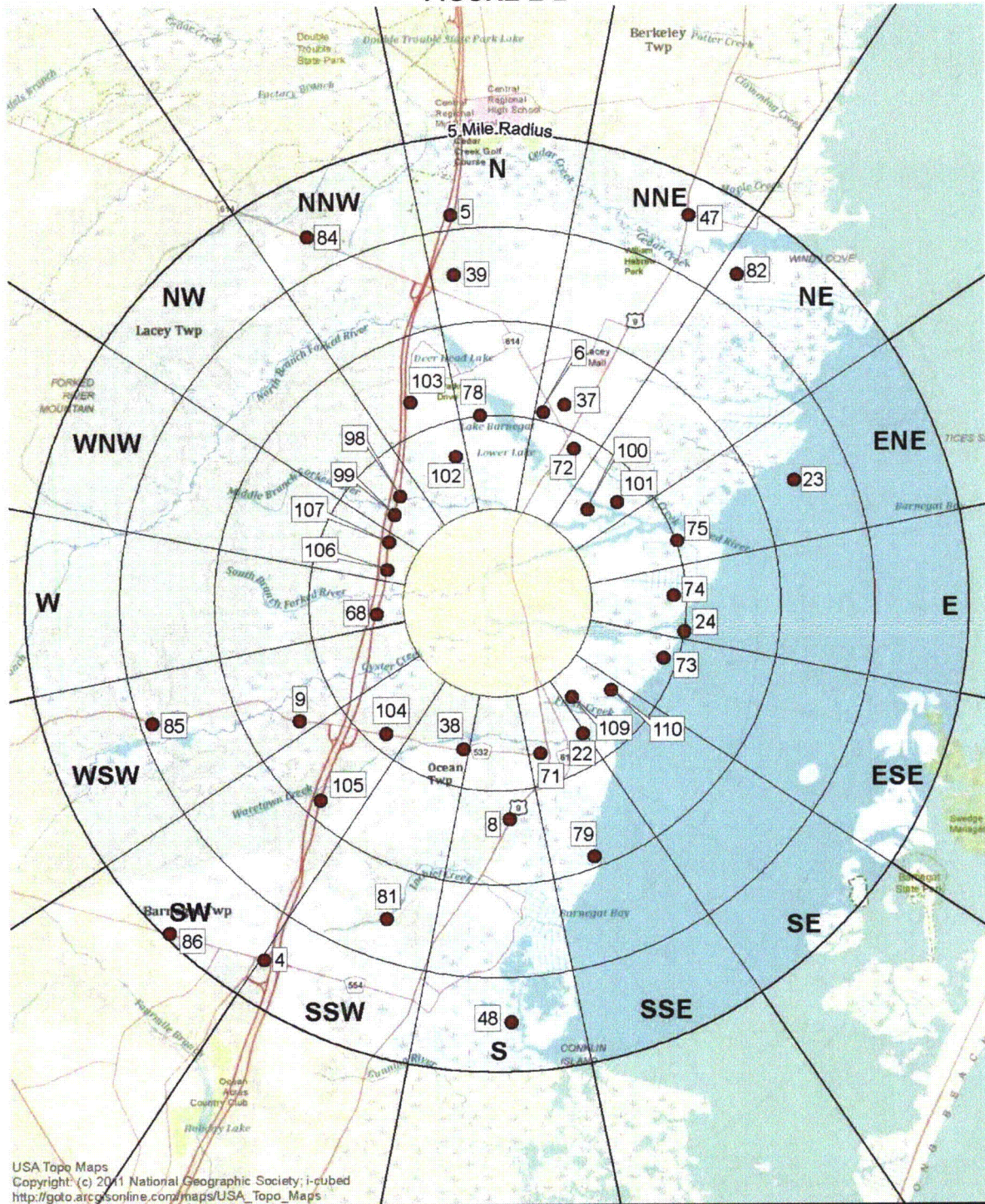
0.3 0.15 0 0.3 Miles



Date: 09/04/12

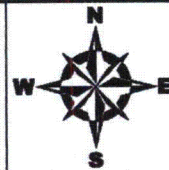
122014

FIGURE E-2



USA Topo Maps
 Copyright (c) 2011 National Geographic Society; i-cubed
http://gto.arc.gov/online/maps/USA_Topo_Maps

Oyster Creek Generating Station
 REMP Sample Locations
 within a 1 to 5 Mile Radius



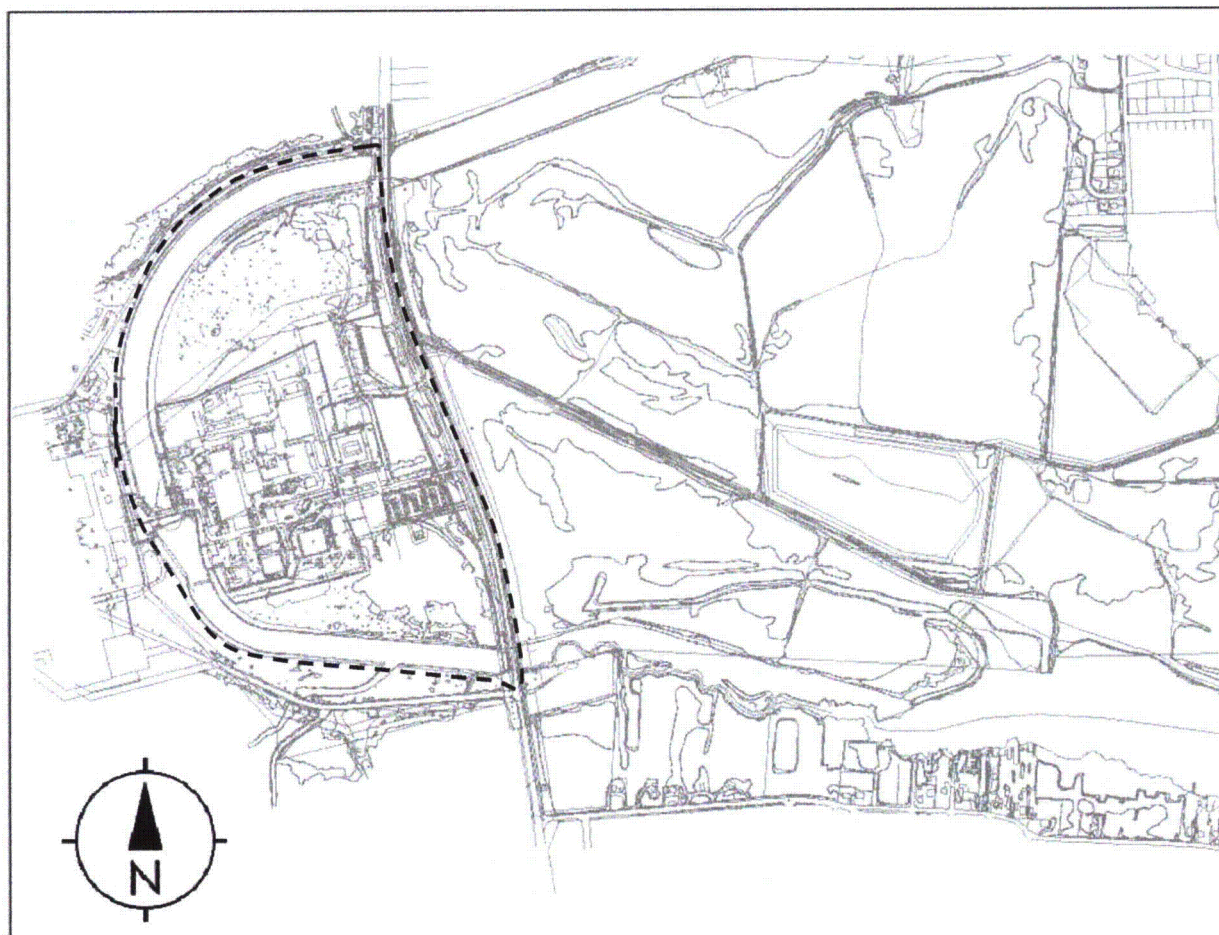
Date: 08/27/12

122014

FIGURE E-4

AREA PLOT PLAN OF SITE

SITE MAP DEFINING UNRESTRICTED AREAS AND SITE BOUNDARY FOR RADIOACTIVE
 GASEOUS AND LIQUID EFFLUENTS



Site Boundary Distances

Sector	Distance in meters	Sector	Distance in meters
S	348	N	584
SSW	291	NNE	621
SW	229	NE	373
WSW	260	ENE	338
W	239	E	360
WNW	284	ESE	491
NW	364	SE	544
NNW	474	SSE	395

PROCESS CONTROL PROGRAM FOR RADIOACTIVE WASTES

1. PURPOSE

- 1.1. The purpose of the Process Control Program (PCP) is to:
 - 1.1.1. Establish the process and boundary conditions for the preparation of specific procedures for processing, sampling, analysis, packaging, storage, and shipment of solid radwaste in accordance with local, state, and federal requirements. **(CM-1)**
 - 1.1.2. Establish parameters which will provide reasonable assurance that all Low Level Radioactive Wastes (LLRW), processed by the in-plant waste process systems on-site OR by on-site vendor supplied waste processing systems, meet the acceptance criteria to a Licensed Burial Facility, as required by 10CFR Part 20, 10CFR Part 61, 10CFR Part 71, 49CFR Parts 171-172, "Technical Position on Waste Form (Revision 1)" [1/91], "Low-Level Waste Licensing Branch Technical Position on Radioactive Waste Classification" [5/83], and the Station Technical Specifications, as applicable.
 - 1.1.3. Provide reasonable assurance that waste placed in "on-site storage" meets the requirements as addressed within the Safety Analysis Reports for the low level radwaste storage facilities for dry and/or processed wet waste.

2. TERMS AND DEFINITIONS

- 2.1. **Process Control Program (PCP):** The program which contains the current formulas, sampling, analysis, tests, and determinations to be made to ensure that processing and packaging of solid radioactive waste based on demonstrated processing of actual or simulated wet solid wastes will be accomplished in such a way as to assure the waste meets the stabilization criteria specified in 10CFR Parts 20, 61 and 71, state regulations, and burial site requirements.
- 2.2. **Solidification:** Liquid waste processed to either an unstable or stable form per 10CFR61 requirements. Waste solidified does not have to meet the 300-year free standing monolith criteria. Approved formulas, samples and tests do not have to meet NRC approval for wastes solidified in a container meeting stability criteria (e.g. High Integrity Container).
- 2.3. **Stabilization:** Liquid waste processed to a "stable state" per 10CFR61 Requirements. Established formulas, samples, and tests shall be approved by the NRC in order to meet solidification "stabilization" criteria. This processing method is currently not available, because the NRC recognizes that waste packed in a High Integrity Container meets the 300-year stabilization criteria. In the event that this processing method becomes an acceptable method, then the NRC shall approve the stabilization formulas, samples, tests, etc.

- 2.4. **Solidification Media:** An approved media (e.g. Barnwell - vinyl ester styrene, cement, bitumen) when waste containing nuclides with greater than 5-year half lives is solidified in a container with activity greater than 1 micro curie/cc. Waste solidified in a HIC is approved by the commission meeting the 10CFR61 stabilization criteria, including 1% free standing liquids by volume when the waste is packaged to a "stable" form and $\leq 0.5\%$ when waste is packaged to an "unstable" form. The formulas, sampling, analysis, and test do not require NRC approval, because the HIC meets the stability criteria.
- 2.4.1. Solidification to an unstable or stable state is performed by vendors, when applicable. Liquid waste solidified to meet stabilization criteria (10CFR61 and 01-91 Branch Technical Requirements) shall have documentation available that demonstrates that the process is approved by the NRC or disposal facility.
- 2.5. **Dewatering:** The process of removing fluids from liquid waste streams to produce a waste form that meets the requirements of 10CFR Part 61 and applicable burial site criteria, $\leq 0.5\%$ by volume when the waste is packaged to an "unstable" state, or $\leq 1\%$ by volume when the waste is packaged to a "stable" form.
- 2.6. **High Integrity Container (HIC):** A disposable container that is approved to the Requirements of 10CFR61. The use of HIC's is an alternative to solidification or encapsulation in a steel container to meet burial stability. HIC's are used to package dewatered liquid wastes, (e.g. filter cartridges, filter media, resin, sludges, etc), or dry active waste.
- 2.7. **Liquid Waste Processing Systems:** In-plant or vendor supplied processing systems consisting of equipment utilized for evaporation, filtration, demineralization, dewatering, compression dewatering, solidification, or reverse osmosis (RO) for the treatment of liquid wastes (such as Floor Drains, Chemical Drains and Equipment Drain inputs).
- 2.8. **Incineration, RVR, and/or Glass Vitrification of Liquid or Solid:** Dry or wet waste processed via incineration and/or thermal processing where the volume is reduced by thermal means meets 10CFR61 requirements.
- 2.9. **Compaction:** When dry wastes such as paper, wood, plastic, cardboard, incinerator ash, and etc. are volume reduced through the use of a compactor.

- 2.10. **Waste Streams:** Consist of but are not limited to
- Filter media (powdered, bead resin and fiber),
 - Filter cartridges,
 - Pre-coat body feed material,
 - Contaminated charcoal,
 - Fuel pool activated hardware,
 - Oil Dry absorbent material added to a container to absorb liquids
 - Fuel Pool Crud
 - Sump and tank sludges,
 - High activity filter cartridges,
 - Concentrated liquids,
 - Contaminated waste oil,
 - Dried sewage or wastewater plant waste,
 - Dry Active Waste (DAW): Waste such as filters, air filters, low activity cartridge filters, paper, wood, glass, plastic, cardboard, hoses, cloth, and metals, etc, which have become contaminated as a consequence of normal operating, housekeeping and maintenance activities.
 - Other radioactive waste generated from cleanup of inadvertent contamination.
- 2.11. **Concentration Averaging:** Concentration averaging is either: a) the mathematical averaging of waste concentrations, based on the size, geometry, type of radioactive emission, and observed dose rates, or b) the combining of radioactive components in a single container and how their radioactivity may be averaged over the volume of the container. Concentration averaging is subject to constraints identified in the NRC's Branch Technical Position on Concentration Averaging and Encapsulation and may also be constrained by Agreement States or Disposal Facilities.
- 2.12. **Encapsulation:** Encapsulation is the surrounding of a radioactive source or component with a nonradioactive material. Encapsulation involves a radioactive core surrounded by a non-radioactive matrix.
- 2.13. **Blending:** The intentional mixing of different, but miscible waste streams (such as resins, filter media, etc.) from different batches or systems for the purpose of operational efficiency or ALARA. Blending applies to LLRW streams only. The addition of non-radioactive materials or fillers is not considered blending.
3. **RESPONSIBILITIES**
- 3.1. Implementation of this Process Control Program (PCP) is described in procedures at each station and is the responsibility of the each site to implement.

4. **MAIN BODY**

4.1. **Process Control Program Requirements**

4.1.1. A change to this PCP (Radioactive Waste Treatment Systems) may be made provided that the change is reported as part of the annual radioactive effluent release report, Regulatory Guide 1.21, and is approved by the Plant Operations Review Committee (PORC).

4.1.2. Changes become effective upon acceptance per station requirements.

4.1.3. A solidification media, approved by the burial site, may be **REQUIRED** when liquid radwaste is solidified to a stable/unstable state.

4.1.4. **When** processing liquid radwaste to meet solidification stability using a vendor supplied solidification system:

1. **If** the vendor has its own Quality Assurance (QA) Program, **then** the vendor shall **ADHERE** to its own QA Program and shall have **SUBMITTED** its process system topical report to the NRC or agreement state.
2. **If** the vendor does **not HAVE** its own Quality Assurance Program, **then** the vendor shall **ADHERE** to an approved Quality Assurance Topical Report standard belonging to the Station or to another approved vendor.

4.1.5. The vendor processing system(s) is/are controlled per the following:

1. A commercial vendor supplied processing system(s) may be **USED** for the processing of LLRW streams.
2. Vendors that process liquid LLRW at the sites shall **MEET** applicable Quality Assurance Topical Report and Augmented Quality Requirements.

4.1.6. Vendor processing system(s) operated at the site shall be **OPERATED and CONTROLLED** in accordance with vendor approved procedures or station procedures based upon vendor approved documents.

4.1.7. All waste streams processed for burial or long term on-site storage shall **MEET** the waste classification and characteristics specified in 10CFR Part 61.55, Part 61.56, the 5-83 Branch Technical Position for waste classification, and the applicable burial site acceptance criteria (for any burial site operating at the time the waste was processed).

4.1.8. An Exelon Nuclear plant may store waste at another Exelon Nuclear plant, provided formal NRC approval has been **RECEIVED** for the transfer of waste.

4.2. General Waste Processing Requirements

NOTE: On-site resin processing involves tank mixing and settling, transferring to the station or vendor processing system via resin water slurry or vacuuming into approved waste containers, and, when applicable, dewatering for burial.

- 4.2.1. Vendor resin beds may be **USED** for decontamination of plant systems, such as, SFP (Spent Fuel Pool), RWCU (reactor water cleanup), and SDC (Shut Down Cooling). These resins are **then PROCESSED** via the station or vendor processing system.
- 4.2.2. Various drains and sump discharges will be **COLLECTED** in tanks or suitable containers for processing treatment. Water from these tanks may be **SENT** through a filter, demineralizer, concentrator or vendor supplied processing systems.
- 4.2.3. Process waste (e.g. filter media, sludges, resin, etc) will be periodically **DISCHARGED** to the station or vendor processing system for onsite waste treatment **or PACKAGED** in containers for shipment to offsite vendor for volume reduction processing.
- 4.2.4. Process water (e.g. chemical, floor drain, equipment drain, etc.) may be **SENT** to either the site waste processing systems or vendor waste processing systems for further filtration, demineralization for plant re-use, or discharge.
- 4.2.5. All dewatering and solidification/stabilization will be **PERFORMED** by either utility site personnel or by on-site vendors **or will be PACKAGED and SHIPPED** to an off-site vendor low-level radwaste processing facility.
- 4.2.6. Dry Active Waste (DAW) will be **HANDLED and PROCESSED** per the following:
 1. DAW will be **COLLECTED and SURVEYED and** may be **SORTED** for compactable and non-compactable wastes.
 2. DAW may be packaged in containers to facilitate on-site pre-compaction and/or off-site vendor contract requirements.
 3. DAW items may be **SURVEYED** for release onsite or offsite when applicable.
 4. Contaminated filter cartridges will be **PLACED** into a HIC **or will be ENCAPSULATED** in an in-situ liner for disposal **or SHIPPED** to an offsite waste processor in drums, boxes or steel liners per the vendor site criteria for processing and disposal.

- 4.2.7. Filtering devices using pre-coat media may be **USED** for the removal of suspended solids from liquid waste streams. The pre-coat material or cartridges from these devices may be routinely **REMOVED** from the filter vessel and discharged to a Filter Sludge Tank or Liner/HIC. Periodically, the filter sludge may be **DISCHARGED** to the vendor processing system for waste treatment onsite **or PACKAGED** in containers for shipment to offsite vendor for volume reduction processing.
- 4.2.8. Activated hardware stored in the Spent Fuel Pools will be **PROCESSED** periodically using remote handling equipment **and** may then be **PUT** into a container for shipment or storage in the pool or loading the processed activated hardware into the Dry Cask storage system.
- 4.2.9. High Integrity Containers (HIC):
1. For disposal at Barnwell, vendors supplying HIC's to the station shall **PROVIDE** a copy of the HIC Certificate of Compliance, which details specific limitations on use of the HIC.
 2. For disposal at Clive or WCS, vendors supplying HIC's to the station shall **PROVIDE** a copy of the HIC Certificate of Conformance, which details specific limitations on use of the HIC.
 3. Vendors supplying HIC's to the station shall **PROVIDE** a handling procedure which establishes guidelines for the utilization of the HIC. These guidelines serve to protect the integrity of the HIC and ensure the HIC is handled in accordance with the requirements of the Certificate of Compliance or Certificate of Conformance.
- 4.2.10. Lubricants and oils contaminated as a consequence of normal operating and maintenance activities may be **PROCESSED** on-site (by incineration, for oils meeting 10CFR20.2004 and applicable state requirements, or by an approved vendor process) **or SHIPPED** offsite (for incineration or other acceptable processing method).
- 4.2.11. Former in-plant systems GE or Stock Drum Transfer Cart and Drum Storage Areas may be **USED** for higher dose DAW storage at Clinton, Dresden, Quad Cities, Braidwood and Byron.
- 4.2.12. Certain waste, including flowable solids from holding pond, oily waste separator, cooling tower basin and emergency spray pond, may be disposed of onsite under the provisions of a 10CFR20.2002 permit. Specific requirements associated with the disposal shall be incorporated into station implementing procedures. **(CM-2)**

- 4.2.13. Concentration averaging may be **PERFORMED** to combine LLRW having different concentrations of radionuclides to form a homogeneous mixture in accordance with the guidance in the NRC's Branch Technical Position on Concentration Averaging and Encapsulation-1995:
- For homogeneous waste types such as resins and filter media, the concentration of the mixture for classification purposes may be based on either the highest radionuclide concentration in any of the individual waste types contributing to the mixture or the volumetric or weight-averaged nuclide concentrations in the mixture provided that the concentrations of the individual waste type contributors to the mixture are within a factor of 10 of the average concentration of the resulting mixture. (NOTE: a designed collection of homogeneous waste types (from different sources within a facility) is not considered 'mixing' and the concentration for classification purposes may be the average concentration of the combination).
 - For non-homogeneous waste types such as activated metals, cartridge filters or components incorporating radioactivity in their design, the concentration should be determined from the total weight or displaced volume (excluding major void spaces) of the component. Mixtures of components in a disposal container is permissible. Concentration averaging of a mixture of components of similar types can be performed in accordance with the NRC's Branch Technical Position on Concentration Averaging and Encapsulation and any State or Disposal Site specific requirements.
- 4.2.14. Blending may be **PERFORMED** for routine LLRW such as resins and filter media in accordance with the guidance in the NRC's Branch Technical Position on Concentration Averaging and Encapsulation as further clarified in SECY 2010-0043. The concentration of the mixture may be determined based on the total activity of all components in the mixture divided by the total volume or mass of the mixture. Reasonable effort should be made to mix blended LLRW so that activity is evenly distributed.
- 4.2.15. Encapsulation may be **PERFORMED** for routine wastes such as filters, filter cartridges, or sealed sources centered in an encapsulated mass, in accordance with the guidance in the NRC's Branch Technical Position on Concentration Averaging and Encapsulation. Classification may be based on the overall volume of the final solidified mass provided that;
- The minimum solidified volume or mass should be reasonably difficult to move by hand.
 - The maximum solidified volume or mass used for determining concentration for any single discrete source should be no more than 0.2 m³ or 500Kg (typically 55-gallon drum).
 - The maximum amount of gamma-emitting radioactivity or radioactive material is <0.02 mrem/hr on the surface of the encapsulation over a 500-year decay period.

- The maximum amount of any radionuclide in a single encapsulation, when averaged over the waste and encapsulating media, does not exceed the maximum concentration limits for Class C waste.
- Written procedures should be established to ensure that the radiation source(s) is reasonably centered (or distributed) within the encapsulating media.
- All other disposal facility requirements for encapsulated material are met.

4.3. Burial Site Requirements

- 4.3.1. Waste sent directly to burial shall **COMPLY** with the applicable parts of 49CFR171-172, 10CFR61, 10CFR71, and the acceptance criteria for the applicable burial site.

4.4. Shipping and Inspection Requirements

- 4.4.1. All shipping/storage containers shall be **INSPECTED**, as required by station procedures, for compliance with applicable requirements (Department of Transportation (DOT), Nuclear Regulatory Commission (NRC), station, on-site storage, and/or burial site requirements) prior to use.
- 4.4.2. Containers of solidified liquid waste shall be **INSPECTED** for solidification quality and/or dewatering requirements per the burial site, offsite vendor acceptance, or station acceptance criteria, as applicable.
- 4.4.3. Shipments sent to an off site processor shall be **INSPECTED** to ensure that the applicable processor's waste acceptance criteria are being met.
- 4.4.4. Shipments sent for off site storage shall **MEET** the storage site's waste acceptance criteria.

4.5. Inspection and Corrective Action

- 4.5.1. Inspection results that indicate non-compliance with applicable NRC, State, vendor, or site requirements shall be IDENTIFIED and TRACKED through the Corrective Action Program.
- 4.5.2. Administrative controls for preventing unsatisfactory waste forms from being released for shipment are described in applicable station procedures. If the provisions of the Process Control Program are not satisfied, then **SUSPEND** shipments of defectively packaged radioactive waste from the site. **(CM-1)**
- 4.5.3. If freestanding water or solidification **not** meeting program requirements is observed, then samples of the particular series of batches shall be **TAKEN** to determine the cause. Additional samples shall be **TAKEN**, as warranted, to ensure that **no** freestanding water is present and solidification requirements are maintained.

4.6. Procedure and Process Reviews

- 4.6.1. The Exelon Nuclear Process Control Program and subsequent changes (other than editorial/minor changes) shall be **REVIEWED and APPROVED** in accordance with the station procedures, plant-specific Technical Specifications (Tech Spec), Technical Requirements Manual (T&RM), Operation Requirements Manual (ORM), as applicable, for the respective station and LS-AA-106. Changes to the Licensees Controlled Documents, UFSAR, ORM, or TRM are controlled by the provisions of 10CFR 50.59.
- 4.6.2. Any changes to the PCP shall be reviewed to determine if reportability is required in the Annual Radiological Effluent Release Report (ARERR). The Radwaste Specialist shall ensure correct information is **SUBMITTED** to the ODCM program owner prior to submittal of the ARERR.
- 4.6.3. Procedures shall be **IMPLEMENTED** as follows:
- Station processes or other vendor waste processing/operating procedures shall be technically reviewed and approved per RM-AA-102-1006.
 - Procedures related to waste manifests, shipment inspections, and container activity determinations are **CONTROLLED** by Radiation Protection Standard Procedures (RP-AA-600 Series).
 - Site waste processing **IS CONTROLLED** by site operating procedures.
 - Liquid processed by vendor equipment shall be **PERFORMED** in accordance with vendor procedures.
 - The dewatering procedures implemented by Vendor for the purpose of compliance to the Process Control Program **SHALL BE REVIEWED and APPROVED** in accordance with the plant specific TRM or ORM (either Current Technical Specifications (CTS) or Improved Technical Specifications (ITS), as applicable for the respective stations).

4.7. Waste Types, Point of Generation, and Processing Method

Methods of processing and individual vendors may **CHANGE** due to changing financial and regulatory options. The table below is a representative sample. It is **not** intended to be all encompassing.

WASTE STREAM	POINTS OF GENERATION	AVAILABLE WASTE PROCESSING METHODS
Bead Resin	Systems - Fuel Pool, Condensate, Reactor Water Cleanup, Blowdown, Equipment Drain, Chemical and Volume Control Systems, Floor Drain, Maximum Recycle, Blowdown, Boric Acid Recycling System, Vendor Supplied Processing Systems, and Portable Demin System	Dewatering, solidification to an unstable/stable state Thermal Processing Free Release to a Land Fill
Powdered Resin	Systems - (Condensate System, Floor Drain/Equipment Drain filtration, Fuel Pool)	Dewatering, solidification to an unstable/stable state Thermal Processing
Concentrated Waste	Waste generated from Site Evaporators resulting typically from the Floor Drain and Equipment Drain Systems	Solidification to an unstable/stable state Thermal Processing
Sludge	Sedimentation resulting from various sumps, condensers, tanks, cooling tower, emergency spray pond, holding pond, and oily waste separators	Dewatering, solidification to an unstable/stable state Thermal Processing Evaporation on-site or at an offsite processor On-site disposal per 10CFR20.2002 permit
Filter cartridges	Systems - Floor/Equipment Drains, Fuel Pool; cartridge filters are typically generated from clean up activities within the fuel pool, torus, etc	Dewatering, solidification to an unstable/stable state Processed by a vendor for volume reduction
Dry Active Waste	Paper, wood, plastic, rubber, glass, metal, and etc. resulting from daily plant activities	Decon/Sorting for Free Release Compaction/Super-compaction Thermal Processing by Incineration or glass vitrification Sorting for Free Release Metal melting to an ingot
Contaminated Oil	Oil contaminated with radioactive materials from any in-plant system.	Solidification unstable state Thermal Processing by Incineration Free Release for recycling
Drying Bed Sludge	Sewage Treatment and Waste Water Treatment Facilities	Free release to a landfill or burial
Metals	See DAW	See DAW
Irradiated Hardware	Fuel Pool, Reactor Components	Volume Reduction for packaging efficiencies

5. **DOCUMENTATION**

- 5.1. Records of reviews performed shall be retained for the duration of the unit operating license. This documentation shall contain:
1. Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change, and
 2. A determination which documents that the change will maintain the overall conformance of waste products to Federal (10CFR61 and the Branch Technical Position), State, or other applicable requirements, including applicable burial site criteria.

6. **REFERENCES**

6.1. Technical Specifications:

- 6.1.1. The details contained in Current Tech Specs (CTS) or Improved Technical Specifications (ITS), as applicable, in regard to the Process Control Program (PCP), are to be relocated to the Licensee Controlled Documents. Some facilities have elected to relocate these details into the Operational Requirements Manual (ORM). Relocation of the description of the PCP from the CTS or ITS does **not** affect the safe operation of the facility. Therefore, the relocation details are **not** required to be in the CTS or the ITS to provide adequate protection of the public health and safety.

6.2. Writers' References:

- 6.2.1. Code of Federal Regulations: 10 CFR Part 20, Part 61, Part 71, 49 CFR Parts 171-172
- 6.2.2. Low Level Waste Licensing Branch Technical Position on Radioactive Waste Classification, May 1983
- 6.2.3. Technical Position on Waste Form (Revision 1), January 1991
- 6.2.4. USNRC Branch Technical Position on Concentration Averaging and Encapsulation, January 1995
- 6.2.5. Regulatory Guide 1.21, Measuring Evaluating, and Reporting Radioactivity in Solid Wastes and Releases of Radioactive materials in Liquid and Gaseous Effluents from Light-Water-Cooled Nuclear Power Plants
- 6.2.6. I.E. Circular 80.18, 10CFR 50.59 Safety Evaluation for Changes to Radioactive Waste Treatment Systems
- 6.2.7. Amendment No. 202 to Facility Operating License No. NPF-11 and Amendment No. 189 to Facility Operating License (FOL) No. NPF-18 for the LaSalle County Station (LSCS), Units 1 and 2

6.2.8. NRC Branch Technical Position on Blending of Low-Level Radioactive Waste, SECY-10-0043

6.3. Users' References:

6.3.1. Quality Assurance Program (QATR)

6.3.2. LS-AA-106, Plant Operations Review Committee

6.3.3. RM-AA-102-1006, Processing Vendor Documents

6.3.4. RP-AA-600 Series, Radioactive Material/Waste Shipments

6.3.5. CY-AA-170-2000, Annual Radioactive Effluent Release Report

6.4. Station Commitments:

6.4.1. Peach Bottom

CM-1, T03819, Letter from G.A. Hunger, Jr., dated Sept. 29 1994, transmitting TSCR 93-16 (Improved Technical Specifications). (Step 1.1.1, 4.5.2)

6.4.2. Limerick

CM-2, T03896, 10CFR20.2002 permit granted to Limerick via letter dated July 10, 1996. (Step 4.2.12)

7. **ATTACHMENTS** - None