

Tennessee Valley Authority, 1101 Market Street, Chattanooga, Tennessee 37402

CNL-15-037

February 27, 2015

10 CFR 2.202 10 CFR 50.4

ATTN: Document Control Desk U.S. Nuclear Regulatory Commission Washington, D.C. 20555-0001

> Sequoyah Nuclear Plant, Units 1 and 2 Facility Operating License Nos. DPR-77 and DPR-79 NRC Docket Nos. 50-327 and 50-328

- Subject: Fourth Six-Month Status Report in Response to the March 12, 2012, Commission Order Modifying Licenses with Regard to Requirements for Mitigation Strategies for Beyond-Design-Basis External Events (Order Number EA-12-049) for Sequoyah Nuclear Plant (TAC Nos. MF0864 and MF0865)
- References: 1. Letter from TVA to NRC, "Tennessee Valley Authority (TVA) Overall Integrated Plan in Response to the March 12, 2012, Commission Order Modifying Licenses with Regard to Requirements for Mitigation Strategies for Beyond-Design-Basis External Events (Order Number EA-12-049) for Sequeyah Nuclear Plant," dated February 28, 2013 (ML13063A183)
 - Letter from TVA to NRC, "First Six-Month Status Report in Response to March 12, 2012, Commission Order Modifying Licenses with Regard to Requirements for Mitigation Strategies for Beyond-Design-Basis External Events (Order Number EA-12-049) for Sequoyah Nuclear Plant," dated August 28, 2013 (ML13247A286)
 - Letter from NRC to TVA, "Sequoyah Nuclear Plant, Units 1 and 2 Interim Staff Evaluation Relating to Overall Intergraded Plan in Response to Order EA-12-049 (Mitigation Strategies) (TAC Nos. MF0864 and MF0865)," dated February 19, 2014 (ML14002A109)

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- 4. Letter from TVA to NRC, "Second Six-Month Status Report and Revised Overall Integrated Plan in Response to the March 12, 2012, Commission Order Modifying Licenses with Regard to Requirements for Mitigation Strategies for Beyond-Design-Basis External Events (Order-EA-12-049) for Sequoyah Nuclear Plant," dated February 28, 2014 (ML14064A181)
- Letter from TVA to NRC, "Third Six-Month Status Report in Response to the March 12, 2012, Commission Order Modifying Licenses with Regard to Requirements for Mitigation Strategies for Beyond-Design-Basis External Events (Order Number EA-12-049) for Sequoyah Nuclear Plant (TAC Nos. MF0864 and MF0865)," dated August 28, 2014 (ML14247A644)

On February 28, 2013, the Tennessee Valley Authority (TVA) submitted an Overall Integrated Plan (OIP) in response to the March 12, 2012, Commission Order modifying licenses with regards to requirements for mitigation strategies for beyond-design-basis external events, Order number EA-12-049, for the Sequoyah Nuclear Plant (SQN), Units 1 and 2 (Reference 1). On August 28, 2013, TVA provided the first six-month status report to the OIP (Reference 2).

The OIP submitted in Reference 1 employed a strategy using reactor coolant pump (RCP) low leakage seals. TVA revised its strategy to use the existing conventional RCP seals. This change in RCP seals required a revision to the OIP submitted by Reference 1. Based on a review of TVA's plan, including the first six-month update, and information obtained through the mitigation strategies audit process, the NRC concluded in its Interim Staff Evaluation that the plan, when properly implemented, will meet the requirements of Order EA-12-049 at SQN, Units 1 and 2 (Reference 3). The Interim Staff Evaluation included open item 3.2.1.6.A. This open item required revision to the Sequence of Events due to use of the conventional RCP seals for reanalysis by the NRC. On February 28, 2014, TVA provided the second six-month status report and revised OIP (Reference 4) which included the required revision to the Sequence of Events and RCP seals.

On August 28, 2014, TVA provided the third six-month status report (Reference 5). Reference 5 noted changes to the storage locations for the current 3 MW FLEX Diesel Generators and the change in the site location for the FLEX Equipment Storage Building. In addition, Reference 5 noted that TVA was evaluating the potential to qualify the existing Condensate Storage Tanks (CSTs) as a primary water source in lieu of the Auxiliary Feedwater Supply Tank (AFWST) and any changes resulting from this reevaluation will be provided in the fourth six-month status update.

The purpose of this letter is to provide the fourth six-month status report pursuant to Section IV, Condition C.2, of Reference 1, that delineates progress made in implementing the requirements of Reference 1. This status report also provides results of reevaluating the existing CSTs as a primary water source. Specifically, the Enclosure of this letter provides fourth six-month status report. As discussed above, TVA has completed evaluation of the CSTs and the CSTs are being modified to provide missile protection and seismic qualification. The CSTs will be used as the primary water source in lieu of the AFWST.

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In addition to the changes described previously, the Open Items table in the Enclosure has been updated. Open Item 14 has been re-opened as indicated in the Enclosure. Open Items 1, 2, 9, 10, 16 - 20 and 22 - 25 are closed. The milestone target completion dates have also been updated as shown in the Enclosure.

There are no new regulatory commitments in this letter. If you have any questions regarding this report, please contact Zachary Kitts at (423) 843-7018.

I declare under penalty of perjury that the foregoing is true and correct. Executed on the 27th day of February 2014.

Respectfully,

J. W. Shea Vice President, Nuclear Licensing

Enclosure:

Tennessee Valley Authority Sequoyah Nuclear Plant's Fourth Six-Month Status Report for the Implementation of Order EA-12-049, Order Modifying Licenses with Regard to Requirements for Mitigations Strategies for Beyond-Design-Basis External Events

cc (Enclosure):

NRR Director - NRC Headquarters NRO Director - NRC Headquarters NRR JLD Director - NRC Headquarters NRC Regional Administrator - Region II NRR Project Manager - Sequoyah Nuclear Plant NRC Senior Resident Inspector - Sequoyah Nuclear Plant

ENCLOSURE

TENNESSEE VALLEY AUTHORITY SEQUOYAH NUCLEAR PLANT FOURTH SIX-MONTH STATUS REPORT FOR THE IMPLEMENTATION OF ORDER EA-12-049, ORDER MODIFYING LICENSES WITH REGARD TO REQUIREMENTS FOR MITIGATION STRATEGIES FOR BEYOND-DESIGN-BASIS EXTERNAL EVENTS

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ENCLOSURE

TENNESSEE VALLEY AUTHORITY SEQUOYAH NUCLEAR PLANT FOURTH SIX MONTH STATUS REPORT FOR THE IMPLEMENTATION OF ORDER EA-12-049,ORDER MODIFYING LICENSES WITH REGARD TO REQUIREMENTS FOR MITIGATION STRATEGIES FOR BEYOND-DESIGN-BASIS EXTERNAL EVENTS

Introduction

Tennessee Valley Authority (TVA) developed an Overall Integrated Plan (OIP) (Reference 1 in Reference section of this enclosure), for Sequoyah Nuclear Plant (SQN), Units 1 and 2, documenting the diverse and flexible strategies (FLEX), in response to Reference 2. TVA provided the first 6-month status report on August 28, 2013 (Reference 3), a revised OIP on February 28, 2014 (Reference 4), and the third 6-month status report on August 28, 2014 (Reference 5). This attachment provides an update of milestone accomplishments since submittal of the third 6-month status report (Reference 5), including any changes to the compliance method or schedule.

Milestone Accomplishments

The following milestones have been completed since submittal of the revised OIP (Reference 4) and submittal of the third 6-month status report, and are current as of February 6, 2015.

- Perform Staffing Analysis
- Procure On-Site Equipment
- Identify Off-Site Delivery Stations
- Create Site Specific FSIs
- Create Maintenance Procedures
- Develop Training Plan

Milestone Schedule

The following provides an update to Attachment 2 of the OIP. The activity status of each item is provided, as well as any change to the expected completion date has changed. The dates are planning dates subject to change as design and implementation details are developed.

The revised milestone target completion dates do not impact the order implementation date.

Activity	Target Completion Date	Activity Status	Revised Target Completion Date
Submit Overall Integrated Plan	Feb 2013	Complete	
Submit 6 Month Updates:			
Update 1	Aug 2013	Complete	
Update 2	Feb 2014	Complete	
Update 3	Aug 2014	Complete	
Update 4	Feb 2015	Complete	
Update 5	Aug 2015	Not Started	
Update 6	Feb 2016	Not Started	
Update 7	Aug 2016	Not Started	
FLEX Strategy Evaluation	Jun 2013	Complete	
Walk-throughs or Demonstration	May 2015	Not Started	Jan 2016
Perform Staffing Analysis	Jan 2015	Complete	
Modifications:			
Modifications Evaluation	Oct 2013	Complete	
Unit 1 N-1 Walkdown	Oct 2013	Complete	
Unit 1 Design Engineering	Nov 2014	In-progress	Mar 2015
Unit 1 Implementation Outage	May 2015	Started	
Unit 2 N-1 Walkdown	Apr 2014	Complete	
Unit 2 Design Engineering	Nov 2014	In-progress	Mar 2015
Unit 2 Implementation Outage	Dec 2015	Not Started	
Storage:			
Storage Design Engineering	Oct 2014	In-progress	Mar 2015
Storage Implementation	May 2015	In-progress	
FLEX Equipment:			
Procure On-Site Equipment	Jan 2015	Complete	
Develop Strategies with NSRC	Dec 2013	Complete	
Identify Off-Site Delivery Stations	Mar 2014	Complete	
Procedures:			
PWROG issues FSG guidelines	Jun 2013	Complete	
Create Site Specific FSIs	Nov 2014	Complete	
Create Maintenance Procedures	Nov 2014	Complete	
Training:			
Develop Training Plan	Nov 2014	Complete	
Implement Training	May 2015	In-progress	
Unit 1 FLEX Implementation	May 2015	Started	
Unit 2 FLEX Implementation	Dec 2015	Started	
Full Site FLEX Implementation	Dec 2015	Not Started	
Submit Completion Report	Jan 2016	Not Started	

FLEX Mitigation Strategy Update Summary

The following is a list of updates made to the information provided since the third 6-month status report (Reference 5). A copy of the latest version of the OIP incorporating these changes has been uploaded to the ePortal. Additional discussion and detail are provided in the OIP. With exception of the identified alternate for using pre-staged diesel generators and pumps described below, these changes meet the NEI 12-06 compliance method.

General Integrated Plan Elements:

- The Seismic Augmented Approach provides additional requirements for plants to address seismic robustness of FLEX equipment. These requirements are captured in the SQN Design Criteria for the FLEX Response System. This ensures that FLEX credited equipment (both currently installed and new) retains function during and after a beyond design basis seismic event using seismic margins assessment criteria by calculating a High Confidence of Low Probability of Failure (HCLPF) seismic capacity and comparing that to the seismic demand of the Review Level Ground Motion (RLGM). For this margin assessment the RLGM is capped at 2X SSE from 1 to 10 Hz.
- In accordance with NEI 12-06, the liquefaction potential of FLEX deployment routes has been evaluated. This evaluation found that neither soil liquefaction or lateral spreading would prevent deployment of FLEX equipment after a beyond-design-basis seismic event. The evaluation was performed in accordance with ASCE 7-10.
- Sequoyah Nuclear Plant is using pre-staged 480v (225KVA) and 6900v (3 MW) FLEX Diesel Generators and pre-staged pumps that will be powered through the existing electrical distribution system as a part of the mitigation strategy integrated plan. This is identified as an alternative approach from the strategies identified in NEI 12-06, as endorsed by NRC in JLD-ISG-2012-01, due to reliance on permanently installed plant structures and systems (i.e., electrical distribution system) and components (pre-staged diesel generators and pumps) in lieu of reliance on complete deployment and alignment of portable generators and diesel driven pumps to accomplish an ELAP event mitigation. Sequoyah Nuclear Plant plans to comply with the guidance in JLD-ISG-2012-01 and NEI 12-06 in implementing FLEX strategies for the SQN site except for the alternatives to the guidance as stated above.
- Several updates made to the Sequence of Events discussion and to Attachment 1A, Sequence of Events Timeline for both Non-Flood and Flood scenarios, including: RCP seal leakage rates based on PWROG/Westinghouse calculations, SG pressure plateaus for RCS cooldown and depressurization, 480v FLEX DG refuel strategy, and recognition that hardened and seismically qualified CSTs will be the initial source of SG makeup.
- Sequoyah Nuclear Plant will pre-stage FLEX Flood Mode equipment based on a 25 year flood warning from TVA's River Operations forecasting group. Concurrent with full FLEX implementation at Sequoyah, River Operations procedure RvM-SOP-10.05.06, "Nuclear Notifications and Flood Warning Procedure," and AOP-N.03, "External Flooding," will be revised to provide the notification and direct the pre-staging of FLEX equipment.

 Sequoyah Nuclear Plant has updated the description for the National SAFER Response Center plan for phase 3 response as follows: Equipment will be moved from the NSRC to a SQN Staging Area, established by the Strategic Alliance for FLEX Emergency Response (SAFER)/NSRC team and TVA. Staging area B is on the Sequoyah Training Center upper parking lot. Staging area C is the Cleveland Regional Jetport located 52 driving miles from SQN. Staging area D is the Chattanooga Airport (Lovell Field) which is 28 driving miles from SQN. Communications will be established between SQN and the SAFER/NSRC team and required equipment moved to the site as needed. First arriving equipment, as established in the 'SAFER Response Plan for Sequoyah Nuclear', will be delivered to the site within 24 hours from the initial request. Once the equipment arrives onsite SQN will utilize it based on plant conditions and need. Details for activation, delivery and operational capability of the Phase 3 equipment can be found in the 'SAFER Response Plan for Sequoyah Nuclear'.

Maintain Core Cooling & Heat Removal:

- Identified hardened and seismically qualified CSTs as the primary source of SG makeup water.
- Provided discussion for FLEX mitigation strategy options available for SG makeup during Non-Flood and Flood Event scenarios.
- Added discussion for use of Mode 5 & 6 IP FLEX Pumps for Core Cooling with SGs not Available events.
- Updated the discussions in the Storage/Protection of Equipment subsections for: Seismic, Flooding, Severe Storms with High Winds, Snow, Ice and Extreme Cold and High Temperatures.

Maintain RCS Inventory Control:

- Identified revised RCP seal leakage rates based on PWROG/Westinghouse calculations.
- Identified revised SG pressure plateaus for RCS cooldown and depressurization.
- Provided discussion for FLEX mitigation strategy options available for RCS inventory makeup during Non-Flood and Flood Event scenarios.
- Updated the discussions in the Storage/Protection of Equipment subsections for: Seismic, Flooding, Severe Storms with High Winds, Snow, Ice and Extreme Cold and High Temperatures.

Maintain Containment:

• Sequoyah Nuclear Plant has performed a containment analysis based on the boundary conditions described in Section 2 of NEI 12-06. Based on the results of this analysis, required actions to ensure maintenance of containment integrity and required instrumentation function have been developed.

- In an ELAP event at SQN and adhering to SQN's Mitigation Strategy, Safety Injection Pump (SIP) operation is initiated at ~ T+5 hours to recover RCS inventory lost through RCP seal leakage and shrinkage due to a RCS cooldown and depressurization. The Ice Condenser doors open. Calculation LTR-ISENG-14-2 Revision 1, demonstrates that the containment pressure at T+72 hours is well below design pressure. The highest temperature to which the containment vessel is exposed to occurs in the upper containment compartment (140°F at 72 hours) and is also well below the design limit (220°F). The pressures and temperatures are not stabilized and continue to increase but the rate of increase is modest and conditions in the containment expected to remain benign until the ice bed is depleted. This is expected to occur approximately 6 days from the event initiation (T+~6 days).
- Provided discussion of long term containment cooling strategy options.
- Updated the discussions in the Storage/Protection of Equipment subsections for: Seismic, Flooding, Severe Storms with High Winds, Snow, Ice and Extreme Cold and High Temperatures.

Maintain Spent Fuel Pool Cooling:

 Two independent SFP level instruments have been added to facilitate remote monitoring of SFP level and SQN with the requirements of NRC Order EA 12-051 and NEI 12-02.

These SFP Level instruments are powered from the 120v AC Vital Power System. The primary power supply to Spent Fuel Level Continuous Monitoring Loop 1 (0-LI-78-43) is from 120v AC Vital Power Board 1-III with its individual power supply battery backup (0-BAT-78-43). The primary power supply to Spent Fuel Level Continuous Monitoring Loop 2 (0-LI-78-44) is from 120v AC Vital Power Board 2-IV with its individual battery backup power supply (0-BAT-78-44). The 120v AC Vital Power Boards are powered by 120v AC Vital Inverters fed by its 125v DC Vital Battery Board. (DCN D23195A).

 Sequoyah Nuclear Plant has performed a SFP analysis calculation CN-SEE-II-13-9, R0, demonstrating the following:

Considering no reduction of coolant inventory due to sloshing, the time to boil is 11.77 hours for a seismic event assuming the minimum critical damping and an initial bulk water temperature in the pool of 127°F. This time to boil is calculated using the normal credible decay heat load. For maximum credible heat load, the time to boil is 5.39 hours.

Considering no reduction in initial SFP water inventory, starting from nominal pool level, an initial bulk water temperature of 127°F and assuming the normal operating decay heat load results in a time when boil off decreases the water level to 10 feet above the SFP racks of approximately 58.83 hours for an SSE seismic event.

For the maximum credible heat load and an initial water temperature in the pool of 127°F, the time when boil off decreases the water level to 10 feet above the SFP racks is approximately 27.08 hours.

Initial SFP makeup should be from the Demineralized Water Head Tank until it is depleted. In order to keep the pool at a constant level of coolant (thus covering the top of the spent fuel), the deployed LP FLEX Pumps will pressurize the ERCW headers and provide makeup to the SFP.

• Updated the discussions in the Storage/Protection of Equipment subsections for: Seismic, Flooding, Severe Storms with High Winds, Snow, Ice and Extreme Cold and High Temperatures.

Safety Functions Support:

- The 6900v FLEX DGs are pre-staged in the Additional Diesel Generator Building (ADGB).
- Fuel for the 480v FLEX DGs will be provided by each 480v FLEX DG's day tank. Refueling operations will be required at an approximate 10 hour frequency for the duration of 480v FLEX DG operation. The sequence of refuel operations would be:
 - The 480v FLEX DGs refuel source (tanker trailer or fuel cube) should be filled from a selected EDG 7-day tank or diesel fuel storage tank, if it survived the initiating event and transported to the refuel staging area located on the South side of Unit 2 near the Auxiliary Building (AB).
 - Fuel transfer pump would be staged and refuel hose connections completed for ground level and AB roof refuel operations.
 - ➢ With ground level to Auxiliary Building (AB) roof and AB roof to ground level communications established, refuel the 480v FLEX DGs day tanks.

The Condensate Storage Tanks (CSTs) are currently being hardened by providing missile protection and seismic qualification and they will be used as the primary water source for SG makeup.

• This section addresses Habitability and Operations:

Operating Conditions - A loss of ventilation analyses was performed to quantify the maximum steady state temperatures expected in specific areas related to FLEX mitigation strategy implementation to ensure the environmental conditions remain acceptable for personnel habitability and within equipment qualification limits. (FLEX Implementation HVAC ELAP Analysis, Project No. 12938-017, SL-012415, Rev. 0, June 11, 2014, Letter No. SL-TVA-365).

Lighting - In an ELAP event initial lighting during Phase 1 of the response the MCR and Shutdown Board Room areas are provided by the plant designed 125v DC powered emergency lighting system, designated by the LD prefix. This system utilizes LED light bulbs. The Auxiliary Control Room (ACR), access and egress routes and areas that must be attended for safe shutdown operations are provided with 8 hour emergency battery lighting (EBL) units. The EBL units that support safe shutdown and emergency access and egress are routinely referred to as Appendix 'R' battery packs. Traveling to and from the various areas necessary to implement the FLEX mitigation strategies, making required mechanical connections, operating electrical disconnects and breakers, monitoring instrumentation and component manipulations are similar to tasks previously walked down for B.5.b and Appendix 'R' Safe Shutdown operations.

Communications - Sequoyah Nuclear Plant, Units 1 & 2 communications systems and equipment are designed and installed to ensure reliability of onsite and offsite communications in the event of a design basis or BDBEE. SQN's ELAP mitigation capability benefits from a previously planned upgrade of SQN's radio communication system. DCN D23096 has converted the analog Nextel Radio System to a trunked VHF and UHF digital system with new multi-ban handheld radios. The new radio system hardware (cabinets, repeaters, hand held radios, etc.) are provided by the Harris Corporation.

SQN maintains a sound powered phone system. There are four plant sound powered phone sub-divisions: Backup Control Center System, Plant Operations System, Health Physics System and Diesel Building to Main Control Room System. The sound-powered telephone system is a communication system which utilizes telephone instruments in which the transmitters and receivers are passive transducers; external power is not required since operating power is obtained from the speech input only.

TVA purchased 17 IsatPhone PRO global handheld satellite phones for Sequoyah. The SatPhone Battery Life is Talk time: Up to 8 Hours; Standby Time: Up to 100 Hours. These phones are deployed in the MCR at the Shift Manager - Senior Reactor Operator's (SRO) desk, in the TSC, Central Alarm Station (CAS), Environmental Monitoring Vans, with individuals on-site, and in the Emergency Planning (EP) office area. In addition spare batteries are kept fully charged that are available at the TSC, CAS, and EP offices. This gives the individual phones 16 hours of talk time before recharging is needed. The phone in the vans is charged using the standard 12V adapter and will remain available throughout the duration of the event.

• Updated the discussions in the Storage/Protection of Equipment subsections for: Seismic, Flooding, Severe Storms with High Winds, Snow, Ice and Extreme Cold and High Temperatures.

Need for Relief/Relaxation and Basis for the Relief/Relaxation

Currently, TVA expects to comply with the order implementation date and no relief/relaxation is required at this time.

Open Items from Overall Integrated Plan and NRC Evaluation

The following tables provide a summary of the open items documented in the OIP or the NRC Evaluation and the status of each item.

Open Item	Description	Status
1	The current Condensate Storage Tanks (CSTs) are a non-seismic tank that is not missile protected.	Closed
	The site is currently pursuing two options; the	The Condensate Storage
	qualification and hardening of the existing CST or	Tanks (CSTs) are being
	the construction of a new seismically qualified and	modified to provide missile
	missile protected CST. One of these options must	protection and seismic
	be completed before the volume of the CST can be	qualification.
	credited.	DCNs 23191 & 23376 in
	Liquefaction of hour routes for ELEX will be	progress.
	analyzed.	Closed
		Report of Geotechnical
		Exploration, Deployment
		Paths Analysis and Condenante Storage
		Tanks TVA Sequevab
		Nuclear Plant AMEC
		Project 3050140254 and
		Seguovah Nuclear Plant -
		Liquefaction Induced
		Settlement of FLEX
		Deployment Paths - White
		Paper.
3	No detailed analysis has been provided regarding	Closed
	initial FLEX fuel supplies to determine a need time	
	for access to 7 day tank supplies or resupply of the	Fuel consumption
	/ day tanks. It is assumed that each FLEX	spreadsheet completed to
	component is stored with a minimum supply of	snow that fuel supply of
	o nours of fuel at constant operation. This	dove
	ELEX equipment has been purchased and	(EDMS W50 140715 007)
	equipment specifications are known.	
4	No need time has been identified for action to	Closed
	protect containment. This includes actions to	
	mitigate pressurization of containment due to	Westinghouse Letter,
	steaming when reactor coolant system (RCS) vent	LTR-ISENG-14-2, Revision
	paths have been established or actions to mitigate	0, Containment Pressures
	temperature effects associated with equipment	and Temperatures for
	survivability. An evaluation will be provided to	Sequoyah Units 1 and 2
	prove indefinite containment coping.	during an ELAP Calculated
		August 8, 2014.

Description	Status
The Phase 3 equipment staging area has not been determined.	Closed
	Areas are identified and
	are included in the
	Center (NSRC) playbook.
A strategy for clearing and removing debris will be determined.	Closed
	Debris removal equipment
	has been identified and
	determined
A thorough analysis of the makeup flow rate	Started
requirements and other equipment characteristics	
will be finalized during the detailed design phase of FLEX.	
The need time for spent fuel pool (SFP) cooling	Closed
actions (deployment of nose, venting, and	Westinghouse Calculation
case heat loads. This item will continue to be	Note CN-SEE-II-13-9
assessed and later action times may be	Revision 0, Determination
acceptable. Note that the timing for this step	of the Time to Boil in the
during an outage is different, but resources will be	Sequoyah Units 1 & 2
available to complete the required actions.	Spent Fuel Pool after an
Functional requirements for each of the Phase 3	Closed
strategies, equipment and components will be	
completed at a later time and will be provided in	Functional requirements
the six month updates to the February 28, 2013	and equipment for Phase 3
submittal.	strategies are identified
	Center (NSRC) playbook
	Description The Phase 3 equipment staging area has not been determined. A strategy for clearing and removing debris will be determined. A thorough analysis of the makeup flow rate requirements and other equipment characteristics will be finalized during the detailed design phase of FLEX. The need time for spent fuel pool (SFP) cooling actions (deployment of hose, venting, and alignment of makeup) was determined using worst case heat loads. This item will continue to be assessed and later action times may be acceptable. Note that the timing for this step during an outage is different, but resources will be available to complete the required actions. Functional requirements for each of the Phase 3 strategies, equipment and components will be completed at a later time and will be provided in the six month updates to the February 28, 2013 submittal.

Open Item Number	Description	Status
10	Containment temperature instrumentation is only available until flood waters enter the technical	Closed
	support center (TSC) inverter or station battery	SQN procedure FSI-7,
	rooms. A method to monitor containment	Loss of Vital
	temperature, post flood, will be developed.	Instrumentation or Control
		Power, provides the
		vehicle to attain/monitor
		post flood A pyrometer
		will be used to monitor
		containment wall
		temperature.
		Westinghouse Letter, LIR-
		ISENG-14-2, REVISION 0, Containment Pressures
		and Temperatures for SQN
		Units 1 and 2 during an
		ELAP Calculated with
		MAAP 4.07,
		August 8, 2014.
11	The heating, ventilation and air conditioning	Closed
	(Invac) analysis is preliminary, and has not been finalized	FLEX Implementation
		HVAC Analysis Impact
		Study, Project No. 12938-
		012, January 31, 2013.
		Calculation ID:
		NUC-SQN-MEB-
		MDQ0009992013000085,
		Rev 001, SQN ELAP
		January 23, 2014 &
		FLEX Implementation
		HVAC ELAP Analysis,
		Project No. 12938-017,
		SL-012415, Rev. 0,
		June 11, 2014,
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Open Item Number	Description	Status
12	Verify ability to deploy FLEX equipment to provide core cooling in Modes 5 and 6 with steam	Closed
	generators (SGs) unavailable.	Westinghouse Calculation Note, CN-SEE-II-13-37-
		Redacted, Revision 0, Sequoyah Unit 1 and Unit
		2 Reactor Coolant System FLEX Evaluation with
		Standard Reactor Coolant Pump Seals.
		March 6, 2014.
		Alignment for 5000GPM
		Portable Diesel Pump, Revision 0.
13	An evaluation of the impact of FLEX response actions on design basis flood mode preparations	Open
	will be performed. This evaluation will include the	AOP-N.03, External
	Changes which affect the Integrated Plan will be	revision to integrate FLEX
	included in the six month update.	strategies.
14	The purpose of this analysis is to examine options	Open
	to utilize alternate water sources to provide	Westinghouse Calculation
	functions.	Revision 3-A (draft)
		Sequoyah FLEX Alternate Cooling Evaluation Input
		Auxiliary Feedwater Usage is in review.
15	Perform conceptual hydraulic performance analyses. The purpose of this analysis is to	Open
	conservatively evaluate hydraulic performance of FLEX systems.	Westinghouse Calculation Note, CN-FSE-14-48 ,
		Revision 0-A (draft),
		As-Built FLEX System
		Fathom Model is in review.
16	The purpose of this report is to summarize the	Ciosea
	mechanical conceptual design of the FLEX	DAR-FSE-13-3, Revision
	שמו מושעות אוזע ועבוונויץ מוזע ופעעוופע וווטעוווטמנוטווצ.	Conceptual Design Report
		for the Sequoyah Unit 1 and Unit 2 Nuclear Plant
		October 2014.

Open Item	Description	Status
17	Develop a electrical conceptual design report. The	Closed
	conceptual design of the FLEX strategies and identify any required modifications.	See SE Tracker Item 61-B Response Closeout Notes
18	Perform an RCS makeup analysis. The purpose of this analysis is to define FLEX RCS inventory and	Closed
	shutdown margin for Sequoyah.	Westinghouse Calculation Note, CN-SEE-II-13-37- Redacted, Revision 0, Sequoyah Unit 1 and Unit 2 Reactor Coolant System FLEX Evaluation with
		Pump Seals, March 6, 2014.
19	Perform an SFP evaluation. The purpose of this analysis is to evaluate the impact of sloshing and time-to-boil in the SFP after an earthquake.	Closed Westinghouse Calculation Note, CN-SEE-II-13-9, Revision 0, Determination of the Time to Boil in the Sequoyah Units 1 & 2 Spent Fuel Pool after an Earthquake, April 24, 2014, TVA-14-35.
20	Perform a timing and deployment evaluation. The purpose of this analysis is to summarize the FLEX timeline for Sequoyah, identify time constraints and provide for the safety function needs.	Closed NEI 12-01 Phase 2 Extended Loss of AC Power (ELAP) ERO Staffing Analysis Report, Revision 0, January 21, 2015.
21	Develop a programmatic control report. The purpose of this report is to summarize the need to implement programmatic control of the FLEX	Open Program Document to
	program.	complete December, 2015.

Open Item Number	Description	Status
22	Evaluate the existing extreme hazard analysis and planned Near-Term Task Force (NTTF) Tier 1	Closed
	activities on FLEX strategies to summarize on-	NEI 12-06, Revision 0,
	going industry activities and the potential to impact	Diverse and Flexible
	the developed FLEX strategies.	Coping Strategies (FLEX)
		Implementation Guide,"
		August 2012.
		NEI 12-02, Revision 1.
		'Industry Guidance for
		Compliance with NRC
		Order EA-12-051, "To
		Modify Licenses with
		Regard to Reliable Spent
		Fuel Pool Instrumentation,
		August, 2012.
23	I he time at which the Forebay volume depletes	Closed
	needs to be evaluated to determine the time at	
	Which repienishment is required. Based on	See SE Tracker Item 23-C
	in the Eeroboy Based on the alternate easiling	Response Closeout Notes.
	in the Forebay. Based on the alternate cooling	
	are required at 72 hours post ELAD. Therefore, it	
	is expected the Forebay volume will supply suction	
	to the TDAFWP for greater than 72 hours following	
	the ELAP event and replenishment will be required	
	during Phase 3.	
24	Further analysis will be performed to determine the	Closed
	required timeline for implementing the 6.9 KV	
	FLEX DGs as an alternate power source for the	NEI 12-01 Phase 2
	loads supplied by the 480v FLEX DGs.	Extended Loss of AC
		Power (ELAP) ERO
		Staffing Analysis Report.
25	Complete battery calculations to document Vital	Closed
	Battery life of 8 nours after loss of all AC. A battery	SON Coloulations : SON
	of similar design	
		Vital Control Power
		System Loading Channel L
		II III IV and Continuous
		I oading Evaluation of
		Protective Devices in the
		120VAC Vital Instrument
		Power Boards.
	I	i ottor Dourdo.

Open Item	Description	Status
26	The CETs are only available until water enters the	Closed
	auxiliary instrument room. A method to monitor	
	CET, post flood, will be evaluated and developed, if	The CETs indications are
	required.	only available until flood
		waters enter the Auxiliary
		Instrument Room
		(elevation 685). The
		validating indicator for
		CETs is the RCS Wide
		Range (WR) T-hot
		indicators. SQN procedure
		FSI-7, Loss of Vital
		Instrumentation or Control
		Power provides the vehicle
		to attain/monitor RCS WR
		T-hot indication. Flood
		protected instrumentation
		is available in the Auxiliary
		Control Room (ACR).
		Appendix B, Establishing
		Methods for Monitoring
		Parameters During Flood
		provides listing of RCS WR
		T-hot instruments that
		would be available in the
07		ACR for Units 1 & 2.
21	Strategies to address extreme cold conditions on	Closed
	the refueling water storage tank (RVVST) and/or	
	boric acid tanks (BATS), including potential need to	See SE Tracker Item 27-C
	reenergize neaters have not been finalized.	Responses Closeout
20	Establish a contract with the SAEED team in	
20	Establish a contract with the requirements of Section 12 of	Ciosea
	Reference 2	Agroomont with National
		SAFED Desponse Contor
		(NISPC) is in place

Potential NRC Evaluation Impacts

There are no potential impacts to the NRC Evaluation identified at this time.

References

The following references support the updates to the OIP described in this enclosure.

- Letter from TVA to NRC, "Tennessee Valley Authority (TVA) Overall Integrated Plan in Response to the March 12, 2012, Commission Order Modifying Licenses with Regard to Requirements for Mitigation Strategies for Beyond-Design-Basis External Events (Order Number EA-12-049) for Sequoyah Nuclear Plant," dated February 28, 2013 (ML13063A183)
- Letter from TVA to NRC, "First Six-Month Status Report in Response to March 12, 2012, Commission Order Modifying Licenses with Regard to Requirements for Mitigation Strategies for Beyond-Design-Basis External Events (Order Number EA-12-049) for Sequoyah Nuclear Plant," dated August 28, 2013 (ML13247A286)
- Letter from NRC to TVA, "Sequoyah Nuclear Plant, Units 1 and 2 Interim Staff Evaluation Relating to Overall Intergraded Plan in Response to Order EA-12-049 (Mitigation Strategies) (TAC Nos. MF0864 and MF0865)," dated February 19, 2014 (ML14002A109)
- Letter from TVA to NRC, "Second Six-Month Status Report and Revised Overall Integrated Plan in Response to the March 12, 2012, Commission Order Modifying Licenses with Regard to Requirements for Mitigation Strategies for Beyond-Design-Basis External Events (Order Number EA-12-049) for Sequoyah Nuclear Plant," dated February 28, 2014 (ML14064A181)
- Letter from TVA to NRC, "Third Six-Month Status Report in Response to the March 12, 2012, Commission Order Modifying Licenses with Regard to Requirements for Mitigation Strategies for Beyond-Design-Basis External Events (Order Number EA-12-049) for Sequoyah Nuclear Plant (TAC Nos. MF0864 and MF0865)," dated August 28, 2014 (ML14247A644)