Enclosure 1 to TXX-12061

2011 Radioactive Effluent Release Report



2011 RADIOACTIVE EFFLUENT RELEASE REPORT

January 1, 2011 - December 31, 2011

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ACRONYMS AND ABBREVIATIONS

CFR	Code of Federal Regulations
CPNPP	Comanche Peak Nuclear Power Plant
ECL	Effluent Concentration Limit
IFSFI	Independent Spent Fuel Storage Installation
LDCR	Licensing Document Change Request
LHMT	Laundry Holdup and Monitor Tanks
LVW	Low Volume Waste
ODCM	Offsite Dose Calculation Manual
PET	Primary Effluent Tanks
pCi	Picocurie
REC	Radiological Effluent Control
SORC	Station Operations Review Committee
uCi	Microcurie
WMT	Waste Monitor Tanks
WWHT	Waste Water Holdup Tanks

1.0 Introduction

This Radioactive Effluent Release Report, for Comanche Peak Nuclear Power Plant Unit 1 and Unit 2, is submitted as required by Technical Specification 5.6.3 and Offsite Dose Calculation Manual (ODCM) Administrative Control 6.9.1.4 for the period January 1, 2011, through December 31, 2011.

1.1 Executive Summary

The radioactive effluent monitoring program for the year 2011 was conducted as described in the following report. The results of the monitoring program indicate the continued effort to maintain the release of radioactive effluents to the environment as low as reasonably achievable (ALARA).

In June 2009, the NRC provided revised guidance in Regulatory Guide 1.21, *Measuring, Evaluating and Reporting Radioactive Material in Liquid and Gaseous Effluents and Solid Waste, Revision 2*, establishing an updated approach for identifying principal radionuclides. Because the overall quantity of radioactive releases has steadily decreased due to improvements in power plant operations, carbon 14 (C-14) now qualifies as a "principal radionuclide" (anything greater than one percent of overall radioactivity in effluents) under federal regulations at many plants. In other words, C-14 has not increased and C-14 is not a new nuclear plant emission. Rather, the improvements in the mitigation of other isotopes have made C-14 more prominent.

Attachment 10.2 on page 46 provides more detail about C-14.

Gaseous Effluents:

Gaseous Waste	2011	2010	Comments
Tritium	48.12 Ci	43.21 Ci	1
C-14 Activity	23.32 Ci	25.48 Ci	2
Total Fission and Activation Activity	3.73 Ci	3.89 Ci	
Total Particulate Activity	0 Ci	0 Ci	3
Gross Alpha Activity	0 Ci	0 Ci	3
Iodine Activity	0 Ci	0 Ci	3
Calculated Gamma Air Dose	6.21E-5 mRad	4.15E-4 mRad	
Calculated Beta Air Dose	7.54E-5 mRad	8.51E-4 mRad	
Total Whole body dose	0.11mRem	0.11 mRem	4

A summary of all the radioactive gaseous releases to the environment during 2011:

Comments:

1. The major contributor to gaseous tritium activity is evaporation from the spent fuel pools. Factors contributing to the tritium activity in the pools is related to the type of fuel used (i.e., 18month fuel) the core life and power output and number of core cycles.

2. Because the industry as a whole has minimized effluents, C-14 is now a principle isotope. The majority of the gaseous activity and dose increase is from the addition of C-14.

3. No alpha, iodine or particulate activity was released.

4. Despite the inclusion of C-14, total whole body dose is very low (0.44% of Technical Specification).

Overall the gaseous radioactivity releases from CPNPP are well controlled and maintained ALARA. CPNPP is well below all applicable limits for gaseous releases. Neither unit had significant fuel defects during 2011. There were 2 refueling outages and 3 other unplanned outages. Despite the higher than normal number of outages, most of the released activity and associated dose was significantly lower in 2011 versus 2010.

Liquid Effluents:

A summary of all the radioactive liquid releases to the environment during 2011:

Liquid Waste	2011	2010	Comments
Total Activity (excluding tritium)	6.56mCi	16.4 mCi	1
Tritium Activity	2790 Ci	1340 Ci	2
Total Whole Body Dose	0.149 mRem	0.155 mRem	1
Total Volume Released	1,155,673 Gal	28,005,284 Gal	3

Comments

- 1. The increased activity and dose in 2010 was due to issues with the liquid waste processing equipment from March to June. Changes in media and processing equipment repairs solved the problem and restored processing efficiency as can be seen by the decrease in 2011 totals.
- 2. The increased tritium activity for 2011 is attributed to waste water processing for 2 refueling outages.
- 3 Explanation for the larger than normal total volume released during 2010: CPNPP processes many millions of gallons of non-radioactive water each year from the secondary portion of the plant. Water plant waste, turbine building sumps and other sources all contribute to these totals. This waste water is processed through the low volume waste (LVW) system which then discharges to Squaw Creek Reservoir. A monthly composite sample from the LVW is analyzed quarterly for activity. Normally, this waste water does not contain any radionuclides.

However, in late 2009, the Unit 2 refueling water storage tank (RWST) overflowed into the turbine building sumps. Initially this water was diverted to the waste processing system. Once the cleanup was complete, the turbine building sump was directed back to the LVW system. A small amount of tritium remained in the turbine building sumps and was sent to the LVW. The first quarter LVW composite sample tested positive for tritium (a small amount above background). Consequently, the volume discharged from the LVW had to be accounted for as liquid waste.

Because the LVW sample is a quarterly composite, the volume processed as liquid waste for the first quarter (and for the year) is 20 times larger than normal. For 2011, the volume returned to normal values.

Meteorological Data

The CPNPP meteorological system achieved a greater than 90% recoverable data rate for the joint frequency parameters required by Regulatory Guide 1.23 for wind speed, wind direction and delta temperature. See section 7.1 for the actual recovery percentages.

Monitors OOS > 30 Days

During 2011 there were no Technical Specification/ODCM effluent radiation monitors out of service for >30 days.

ODCM Changes

There were 2 changes made to the ODCM. These changes are referenced in section 8.2

Solid Waste

Summary of the solid waste production

Total Waste	2011	2010	% Error
Shipped (m3)	667	476	25%
Shipped (Ci)	0.215	18.9	25%
Buried (m3)	70.7	65.6	25%
Buried (Ci)	0.149	18.9	25%

Comments

CPNPP disposed four containers of Class A resins in 2010 and a typical quantity of dry active waste. The impact of this resin disposal disproportionally increased the activity disposal with respect to 2011. In addition, during 2011, several containers of legacy dry activity waste were disposed of which resulted in greater volume during the year.

Over the past three years, CPNPP has only had access to a Class A radioactive waste disposal site. Class A waste represents the vast majority of volume but very little of the activity disposed of as radioactive waste resulting from operations and maintenance. Once the Texas waste disposal facility becomes available, the disposal activity is expected to spike high due to disposal of waste in storage and then return to normal.

Groundwater Tritium

CR-2011-003303, CR 2011-010251 and CR-2011-009873 documented some positive indications of tritium in the seepage sump near the water treatment plant and in the LVW Pond A Leachate. All of these samples were well below the state reportable criteria of 20,000 pCi/L. Because Squaw Creek Reservoir (SCR) water contains a low background concentration of tritium, SCR water used in the plant will contain similar concentrations.

See section 8.8 for details.

Conclusion

Overall, the radioactive effluent monitoring program has been conducted in an appropriate manner to ensure the activity released and associated dose to the public has been maintained as low as reasonably achievable (ALARA).

1.2 Historical Trend Graphs









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2.0 SUPPLEMENTAL INFORMATION

2.1 <u>Regulatory Limits</u>

The ODCM Radiological Effluent Control limits applicable to the release of radioactive material in liquid and gaseous effluents are described in the following sections.

2.1.1 Fission and Activation Gases (Noble Gases)

The dose rate due to radioactive materials released in gaseous effluents from the site to areas at and beyond the site boundary shall be limited to less than or equal to 500 mRem/yr to the whole body and less than or equal to 3000 mRem/yr to the skin.

The air dose due to noble gases released in gaseous effluents, from each unit, to areas at and beyond the site boundary shall be limited to the following:

- **a**. During any calendar quarter: Less than or equal to 5 mRad for gamma radiation and less than or equal to 10 mRad for beta radiation, and
- **b**. During any calendar year: Less than or equal to 10 mRad for gamma radiation and less than or equal to 20 mRad for beta radiation.

2.1.2 Iodine-131, Iodine-133, Tritium and Radioactive Material in Particulate Form

The dose rate due to iodine-131, iodine-133, tritium and all radionuclides in particulate form with half lives greater than 8 days, released in gaseous effluents from the site to areas at and beyond the site boundary, shall be limited to less than or equal to 1500 mRem/yr to any organ.

The dose to a MEMBER OF THE PUBLIC from iodine-131, iodine-133, tritium and all radionuclides in particulate form with half lives greater than 8 days, in gaseous effluents released, from each unit, to areas at and beyond the site boundary, shall be limited to the following:

a. During any calendar quarter: Less than or equal to 7.5 mRem to any organ, and

b. During any calendar year: Less than or equal to 15 mRem to any organ.

2.1.3 Liquid Effluents

The concentration of radioactive material released in liquid effluents to unrestricted areas shall be limited to 10 times the concentrations specified in 10 CFR Part 20, Appendix B, Table 2, Column 2 for radionuclides other than dissolved or entrained noble gases. For dissolved or entrained noble gases, the concentration shall be limited to $2.0E-4 \mu Ci/ml$ total activity.

The dose or dose commitment to a MEMBER OF THE PUBLIC from radioactive materials in liquid effluents released, from each unit, to unrestricted areas shall be limited:

a. During any calendar quarter to less than or equal to 1.5 mRem to the whole body and to less than or equal to 5 mRem to any organ, and

b. During any calendar year to less than or equal to 3 mRem to the whole body and to less than or equal to 10 mRem to any organ.

2.1.4 <u>LVW Pond Resin Inventory</u>

The quantity of radioactive material contained in resins transferred to the LVW pond shall be limited by the following expression:

$$(264/V) \bullet \Sigma_i A_i/C_i < 1.0$$

excluding tritium, dissolved or entrained noble gases and radionuclides with less than an 8 day half life, where:

 A_i = pond inventory limit for a single radionuclide j (Curies),

$$C_j$$
 = 10CFR20, Appendix B, Table 2 Column 2, concentration for a single radionuclide j (μ Ci/ml),

V = volume of resins in the pond (gallons), and

264 = conversion factor (μ Ci/Ci per ml/gal)

2.1.5 <u>Total Dose</u>

The annual (calendar year) dose or dose commitment to any MEMBER OF THE PUBLIC due to releases of radioactivity and to radiation from uranium fuel cycle sources shall be limited to less than or equal to 25 mRem to the whole body or any organ, except the thyroid, which shall be limited to less than or equal to 75 mRem.

2.2 <u>Effluent Concentration Limits</u>

2.2.1 Gaseous Effluents

For gaseous effluents, effluent concentration limits (ECL) values are not directly used in release rate calculations since the applicable limits are expressed in terms of dose rate at the site boundary.

2.2.2 Liquid Effluents

The values specified in 10 CFR Part 20, Appendix B, Table 2, Column 2 are used as the ECL for liquid radioactive effluents released to unrestricted areas. A value of $2.0E-04 \mu Ci/ml$ is used as the ECL for dissolved and entrained noble gases in liquid effluents.

2.3 <u>Measurements and Approximations of Total Radioactivity</u>

Measurements of total radioactivity in liquid and gaseous radioactive effluents were accomplished in accordance with the sampling and analysis requirements of Tables 4.11-1 and 4.11-2, respectively, of the CPNPP ODCM.

2.3.1 Liquid Radioactive Effluents

Each batch release was sampled and analyzed for gamma emitting radionuclides using gamma spectroscopy, prior to release. Composite samples were analyzed monthly and quarterly for the Primary Effluent Tanks (PET), Waste Monitor Tanks (WMT), Laundry Holdup and Monitor Tanks (LHMT) and Waste Water Holdup Tanks (WWHT). Composite samples were analyzed monthly for tritium and gross alpha radioactivity in the onsite laboratory using liquid scintillation and gas flow proportional counting techniques, respectively. Composite samples were analyzed quarterly for Sr-89, Sr-90 and Fe-55 by a contract laboratory. The results of the composite analyses from the previous month or quarter were used to estimate the quantities of these radionuclides in liquid effluents during the current month or quarter. The total radioactivity in liquid effluent releases was determined from the measured and estimated concentrations of each radionuclide present and the total volume of the effluent released during periods of discharge.

For batch releases of powdex resin to the LVW pond, samples were analyzed for gamma emitting radionuclides, using gamma spectroscopy techniques prior to release. Composite samples were analyzed quarterly for Sr-89 and Sr-90 by a contract laboratory.

For continuous releases to the Circulating Water Discharge from the LVW pond, daily grab samples were obtained over the period of pond discharge. These samples were composited and analyzed for gamma emitting radionuclides, using gamma spectroscopy techniques. Composite samples were also analyzed for tritium and gross alpha radioactivity using liquid scintillation and gas flow proportional counting techniques, respectively. Composite samples were analyzed quarterly for Sr-89, Sr-90 and Fe-55 by a contract laboratory.

2.3.2 **Gaseous Radioactive Effluents**

Each gaseous batch release was sampled and analyzed for radioactivity prior to release using gamma spectroscopy. Waste Gas Decay Tank samples were analyzed for gamma emitting radionuclides. Containment Building charcoal (iodine), particulate, noble gas, and tritium grab samples were also analyzed for gamma emitting radionuclides prior to each release. The results of the analyses and the total volume of effluent released were used to determine the total amount of radioactivity released in the batch mode.

For continuous effluent release pathways, noble gas and tritium grab samples were collected and analyzed weekly for gamma emitting radionuclides by gamma spectroscopy and liquid scintillation counting techniques, respectively. Continuous release pathways were continuously sampled using radioiodine adsorbers and particulate filters. The radioiodine adsorbers and particulate filters were analyzed weekly for I-131 and gamma emitting radionuclides using gamma spectroscopy. Results of the noble gas and tritium grab samples, radioiodine adsorber and particulate filter analyses from the current week and the average effluent flow rate for the previous week were used to determine the total amount of radioactivity released in the continuous mode. Monthly composites of particulate filters were analyzed for gross alpha activity, in the onsite laboratory using the gas flow proportional counting technique. Quarterly composites of particulate filters were analyzed for Sr-89 and Sr-90 by a contract laboratory.

C-14 was estimated in accordance with the methodology in the EPRI report *Estimation of Carbon-14 in Nuclear Power Plant Gaseous Effluents*. EPRI, Palo Alto, CA: 2010, 1021106. See attachment 10.2 on page 46 for more information on C-14.

2.4 Batch Releases

A summary of information for gaseous and liquid batch releases is included in Table 9.1.

2.5 Abnormal or Unplanned Releases

Abnormal releases are defined as the unintended discharge of a volume of liquid or airborne radioactivity to the environment. There was 1 normal (monitored) unplanned gaseous effluent releases during 2011 and is summarized in section 8.5.1. There were no abnormal unplanned liquid or gaseous releases during 2011. Table 9.2 summarizes the abnormal and unplanned releases.

3.0 GASEOUS EFFLUENTS

The quantities of radioactive material released in gaseous effluents are summarized in Tables 9.3 and 9.4. All releases of radioactive material in gaseous form are considered to be ground level releases.

4.0 LIQUID EFFLUENTS

The quantities of radioactive material released in liquid effluents are summarized in Tables 9.5 and 9.6.

5.0 <u>SOLID WASTES</u>

The quantities of radioactive material released as solid effluents are summarized in Table 9.10.

6.0 RADIOLOGICAL IMPACT ON MAN

6.1 Dose Due to Liquid Effluents

The dose to an adult from the fish and cow-meat consumption pathways from Squaw Creek Reservoir were calculated in accordance with the methodology and parameters in the ODCM. The results of the calculations are summarized on a quarterly and annual basis in table 9.7.

6.2 Dose Due to Gaseous Effluents

The air dose due to gamma emissions and the air dose due to beta emissions were calculated using the highest annual average atmospheric dispersion factor at the Site Boundary location, in accordance with the methodology and parameters in the ODCM. The results of the calculations are summarized on a quarterly and annual basis in Table 9.8.

6.3 Dose Due to Radioiodines, Tritium and Particulates

The dose to an adult, teen, child, and infant from radioiodines and particulates, for the pathways

listed in Part II, Table 2.4 of the ODCM, were calculated using the highest dispersion and deposition factors, as appropriate, in accordance with the methodology and parameters in the ODCM. The results of the calculations are summarized on a quarterly and annual basis in Table 9.9. Because of pathway similarity, C-14 dose is included in this table.

6.4 <u>40CFR190 Dose Evaluation</u>

ODCM Radiological Effluent Control 3.11.4 requires dose evaluations to demonstrate compliance with 40 CFR Part 190 only if the calculated quarterly or yearly dose exceed two times the applicable quarterly or annual dose limits. At no time during 2011 were any of these limits exceeded, therefore no evaluations are required.

6.5 Dose to a MEMBER OF THE PUBLIC From Activities Inside the Site Boundary

Three activities are considered in this evaluation: fishing on Squaw Creek Reservoir, recreation activities at the CPNPP employee recreational area and site tours through the CPNPP Visitors Center.

The highest dose occurred in the evaluation for fishing, resulting in a dose of 3.69E-03 mRem/yr. The dose to a MEMBER OF THE PUBLIC (fisherman) on Squaw Creek Reservoir was calculated based on fishing twice a week, five hours each day, six months per year. Pathways included in the calculation were gaseous inhalation and submersion. Liquid pathways are not considered since all dose is calculated at the point of circ water discharge into the reservoir.

The dose to a MEMBER OF THE PUBLIC engaged in recreational activities at the CPNPP employee recreational park was calculated based on one visit a week, five hours each day, six months per year. Pathways included in the calculation were gaseous inhalation, submersion and ground plane.

The dose to a MEMBER OF THE PUBLIC during site tours through the CPNPP Visitors Center was calculated based on two visits per year, thirty minutes each visit. Pathways included in the calculation were gaseous inhalation and submersion.

Due to increased security, routine fishing on Squaw Creek Reservoir and visitation by the public on-site has been significantly restricted. The calculations are still valid and included in the event security access restrictions are ever returned to previous conditions and controls.

All calculations were performed in accordance with the methodology and parameters in the ODCM.

7.0 METEROLOGICAL DATA

7.1 Meteorological Monitoring Program

In accordance with ODCM Administrative Control 6.9.1.4, a summary of hourly meteorological data, collected during 2011, is retained onsite. This data is available for review by the NRC upon request. Joint Frequency Tables are included in Attachment 10. During 2011, the goal of >90% joint data recovery was met. The individual percent recoveries are listed below:

Meteorological Data	Meteorological Data Recovery				
Channel	% Recovery				
Wind Speed	99.8				
Wind Direction	99.7				
Delta Temperature A	98.4				
Delta Temperature B	99.8				

8.0 <u>RELATED INFORMATION</u>

8.1 **Operability of Liquid and Gaseous Monitoring Instrumentation**

ODCM Radiological Effluent Controls 3.3.3.4 and 3.3.3.5 require an explanation of why designated inoperable liquid and gaseous monitoring instrumentation was not restored to operable status within thirty days.

During the period covered by this report, there were no instances where these instruments were inoperable for more than thirty days.

8.2 Changes to the Offsite Dose Calculation Manual

During the period covered by this report, there were 2 revisions to the ODCM:

REVISION 31

LDCR-OD-2011-1 (EV-CR-2010-010410-37) (SCD):

Dose limits at the site boundary require monitoring to demonstrate compliance with the limits of 10 CFR 72 as a result of direct radiation exposure emitted by the facility. The limits for the ISFSI are the same as the existing limits specified by 10 CFR 50 and therefore there are no changes to the monitoring criteria outlined in the ODCM.

REVISION 32

LDCR-OD-2010-1 (EV-CR-2010-011250-1) (SCD):

Revised the ODCM to incorporate the Carbon-14 isotope dose considerations into the CPNPP Radioactive Effluent Release Report in accordance with the new revision 2 of REGULATORY GUIDE 1.21, "Measuring, Evaluating, and Reporting Radioactive Material in Liquid and Gaseous Effluents and Solid Waste" This change is applicable to Units 1 and 2.

8.3 <u>New Locations for Dose Calculations or Environmental Monitoring</u>

ODCM Administrative Control 6.9.1.4 requires any new locations for dose calculations and/or environmental monitoring, identified by the Land Use Census, to be included in the Radioactive Effluent Release Report. Based on the 2011 Land Use Census, no new receptor locations were identified which resulted in changes requiring a revision in current environmental sample locations. Values for the current nearest resident, milk animal, garden, X/Q and D/Q values in all sectors surrounding CPNPP were included in the 2011 Land Use Census.

8.4 Liquid Holdup and Gas Storage Tanks

ODCM Administrative Control 6.9.1.4 requires a description of the events leading to liquid holdup or gas storage tanks exceeding the limits required to be established by Technical Specification 5.5.12. Technical Requirements Manual 13.10.33 limits the quantity of radioactive material contained in each unprotected outdoor tank to less than or equal to ten curies, excluding tritium and dissolved or entrained noble gases. Technical Requirements Manual 13.10.32 limits the quantity of radioactive material contained in each gas storage tank to less than or equal to 200,000 curies of noble gases (considered as Xe-133 equivalent). These limits were not exceeded during the period covered by this report.

8.5 Noncompliance with Radiological Effluent Control Requirements

This section provides a listing and description of Abnormal Releases, issues that did not comply with the applicable requirements of the Radiological Effluents Controls given in Part I of the CPNPP ODCM and/or issues that did not comply with associated Administrative Controls and that failed to meet CPNPP expectations regarding Station Radioactive Effluent Controls. Detailed documentation concerning evaluations of these events and corrective actions is maintained onsite.

8.5.1 Normal, Unplanned Gaseous Release

There was one normal, unplanned gaseous effluent releases occurred during 2011 documented by CR-2011-001885. RCS leakage from the Unit 1 charging system degassed and caused an increasing trend on PVG-384 plant stack A radiation monitor. A plant stack A gas sample indicated the presence of Xe-135 and Ar-41. A release permit was created to account for this monitored gas release.

8.5.2 Abnormal, Unplanned Gaseous Effluent Release

No abnormal, unplanned gaseous effluent releases occurred during 2011.

8.5.3 Abnormal, Unplanned Liquid Effluent Releases

No abnormal, unplanned liquid effluent releases occurred during 2011.

8.6 <u>Resin Releases to the LVW Pond</u>

A total of 927 ft^3 of powdex resin was transferred to the LVW pond during the period covered by this report. The cumulative activity deposited in the LVW pond since operations began through the end of 2011 is 2.14E-3 Curies, consisting of Co-58, Co-60, Cs-134, Cs-137, I-131, Sr-90 and Sb-125.

8.7 Changes to the Liquid, Gaseous, and Solid Waste Treatment Systems

In accordance with the CPNPP Process Control Program, Section 6.2.6.2, changes to the Radwaste Treatment Systems (liquid, gaseous and solid) should be summarized and reported to the Commission in the Radioactive Effluent Release Report if the changes implemented required a 10CFR50.59 safety evaluation.

For the reporting period of this report, no changes to the Radwaste Treatment Systems occurred that meet the reporting criteria of the Process Control Program.

8.8 **Groundwater Tritium monitoring Program**

Seepage Sump

Samples of perched groundwater are taken quarterly in accordance with the site groundwater tritium monitoring program. During 2011, samples from the Water Production Plant seepage sump (storm drain) had positive values for tritium as listed in the table below. The source of the tritium is from a small leak of Squaw Creek Reservoir (SQR) water that feeds the water plant. Because CPNPP discharges its effluents into Squaw Creek Reservoir, there is always a low level background concentration of tritium in the reservoir water. Essentially the seepage sump is discharging diluted Squaw Creek Reservoir water back to Squaw Creek Reservoir.

Basin A

Variations in the tritium values from the Pond A Leachate are thought to come from pockets of water trapped in between the inner and outer liner by many inches of lake sediment resting on top of the liner. These pockets of water are from basin water previously transferred to the space between the liners to test for liner repair work effectiveness. Work is in progress to remove this sediment from the pond. As the sediment is removed, pockets of residual basin water are reintroduced into the Leachate sample.

Basin A only receives Surface Water Treatment System wastewater; therefore, there is a high level of confidence the source of the tritium is Squaw Creek Reservoir water. Also, there is no indication of a release from the secondary liner, since tritium was not detected in down gradient monitoring wells.

In both of these cases, the tritium concentrations are well below the state reportable limit of 20,000 pCi/L. The source of the tritium is from SQR.

This has been documented in CR-2011-003303, CR 2011-010251 and CR-2011-009873.

I ritium Results in pCI/L						
		Pond A				
Date	Seepage Sump	Leachate				
Mar-11	9300	11300				
Jun-11	10400	2050				
Sep-11	Dry (no Sample)	12100				
Dec-11	5030	2260				

SECTION 9.0 EFFLUENT TABLES

<u>Table 9.1</u> <u>Site Liquid and Gaseous Batch Release Summary (2011)</u>

A. Liquid Releases	Units	Quarter 1	Quarter 2	Quarter 3	Quarter 4	Annual
1. Number of batch releases	•	15	15	19	12	61
2. Total time period for Batch releases	(Minutes)	4.69E+03	4.46E+03	5.93E+03	3.76E+03	1.88E+04
3. Maximum time period for a batch release	(Minutes)	3.45E+02	3.30E+02	3.40E+02	3.40E+02	3.45E+02
4. Average time period for a batch release	(Minutes)	3.13E+02	2.97E+02	3.12E+02	3.13E+02	3.09E+02
5. Minimum time period for a batch release	(Minutes)	2.90E+02	2.39E+02	2.60E+02	2.82E+02	2.39E+02
6. Average stream flow during periods of release of liquid Effluent into a flowing stream	(GPM)	6.35E+04	7.99E+04	4.91E+04	7.14E+04	2.64E+05
B. Gaseous Releases						
	Units	Ouarter 1	Quarter 2	Quarter 3	Ouarter 4	Annual

	Units	Quarter I	Quarter 2	Quarter 5	Quarter 4	Annuai
1. Number of batch releases		27	29	28	34	118
2. Total time period for batch releases	(Minutes)	9.73E+03	1.16E+04	9.57E+03	1.30E+04	4.39E+04
3. Maximum time period for a batch release	(Minutes)	4.46E+02	2.10E+03	4.11E+02	1.06E+03	2.10E+03
4. Average time period for a batch release	(Minutes)	3.60E+02	4.00E+02	3.42E+02	3.82E+02	3.72E+02
5. Minimum time period for a batch release	(Minutes)	2.31E+02	1.72E+02	1.82E+02	5.10E+01	5.10E+01

A. Liquid Releases 1. Number of Releases	Units	Quarter 1 0	Quarter 2 0	Quarter 3 0	Quarter 4 0	Totals 0
2. Total Time For All Releases	(Minutes)	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
3. Maximum Time For A Release	(Minutes)	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
4. Average Time For A Release	(Minutes)	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
5. Minimum Time For A Release	(Minutes)	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
6. Total activity for all releases	(Curies)	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00

<u>Table 9.2</u> <u>Site Abnormal Liquid and Gaseous Batch Release Summary (2011)</u>

B. Gaseous Releases

	Units	Quarter 1	Quarter 2	Quarter 3	Quarter 4	Totals
1. Number of Releases		1	0	0	0	1
2. Total Time For All Releases	(Minutes)	3.12E+02	0.00E+00	0.00E+00	0.00E+00	3.12E+02
3. Maximum Time For A Release	(Minutes)	3.12E+02	0.00E+00	0.00E+00	0.00E+00	3.12E+02
4. Average Time For A Release	(Minutes)	3.12E+02	0.00E+00	0.00E+00	0.00E+00	3.12E+02
5. Minimum Time For A Release	(Minutes)	3.12E+02	0.00E+00	0.00E+00	0.00E+00	3.12E+02
6. Total activity for all releases	(Curies)	2.05E-01	0.00E+00	0.00E+00	0.00E+00	2.05E-01

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Type of Effluent	Units	Quarter 1	Quarter 2	Quarter 3	Quarter 4	Total
A. Fission And Activation Gases						
1. Total Release	Curies	2.86E+00	3.35E+01	3.46E-01	5.99E-01	3.73E+01
2. Average Release rate for period	uCi/sec	3.63E-01	4.24E+00	4.39E-02	7.60E-02	4.72E+00
3. Percent of Applicable Limit	%	*	*	*	*	*
B. Radioiodines	-					
1. Total Iodine-131	Curies	2.89E-06	1.85E-06	0.00E+00	0.00E+00	4.74E-06
2. Average Release rate for period	uCi/sec	3.67E-07	2.35E-07	0.00E+00	0.00E+00	6.02E-07
3. Percent of Applicable Limit	%	*	*	*	*	*
C. Particulates			•			
1. Particulates (Half-Lives > 8 Days)	Curies	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
2. Average Release rate for period	uCi/sec	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
3. Percent of Applicable Limit	%	*	*	*	*	*
D. Tritium & C14						
1. Total Release	Curies	1.45E+01	2.08E+01	2.17E+01	1.45E+01	7.14E+01
2. Average Release rate for period	uCi/sec	1.83E+00	2.65E+00	2.74E+00	1.83E+00	9.04E+00
3. Percent of Applicable Limit	%	*	*	*	*	*
E. Gross Alpha						
1. Total Release	Curies	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
2. Average Release rate for period	uCi/sec	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00

<u>Table 9.3</u> <u>Site Gaseous Effluents - Summation of All Releases (2011)</u>

* Applicable limits are expressed in terms of dose.

Estimated Total Error For All Values Reported Is < 1.0%

<u>2011 Table 9.4</u> <u>Site Gaseous Effluents - Ground Level Releases (2011)</u>

Continuous Mode						
Nuclides Released	Unit	Quarter 1	Quarter 2	Quarter 3	Quarter 4	Total
Fission Gases						
Xe-133	Curies	2.12E+00	6.19E+00	0.00E+00	0.00E+00	8.32E+00
Xe-135	Curies	4.14E-01	0.00E+00	0.00E+00	0.00E+00	4.14E-01
Total For Period	Curies	2.54E+00	6.19E+00	0.00E+00	0.00E+00	8.73E+00
Iodines						
I-131	Curies	2.89E-06	1.85E-06	0.00E+00	0.00E+00	4.74E-06
Total For Period	Curies	2.89E-06	1.85E-06	0.00E+00	0.00E+00	4.74E-06
Particulates						
No Nuclides Found	Curies	N/A	N/A	N/A	N/A	N/A
Tritium & C-14						
Н-3	Curies	8.14E+00	1.55E+01	1.56E+01	8.76E+00	4.79E+01
C-14	Curies	1.87	1.60	1.82	1.70	7.00
Gross Alpha						
No Nuclides Found	Curies	N/A	N/A	N/A	N/A ·	N/A

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Batch Mode				•	•	•
Nuclides Released	Unit	Quarter 1	Quarter 2	Quarter 3	Quarter 4	Total
Fission Gases		•		• •	<i>,</i>	:
Ar-41	Curies	2.96E-01	7.65E-02	8.11E-02	1.62E-01	6.15E-01
Kr-85m	Curies	0.00E+00	1.99E-02	0.00E+00	7.42E-04	2.07E-02
Kr-85	Curies	0.00E+00	4.52E-01	2.65E-01	3.77E-01	1.09E+00
Kr-87	Curies	0.00E+00	0.00E+00	0.00E+00	3.82E-04	3.82E-04
Kr-88	Curies	0.00E+00	3.08E-03	0.00E+00	1.28E-03	4.36E-03
Xe-131m	Curies	0.00E+00	3.61E-01	0.00E+00	1.44E-05	3.61E-01
Xe-133m	Curies	0.00E+00	4.59E-01	0.00E+00	1.35E-03	4.61E-01
Xe-133	Curies	1.42E-02	2.47E+01	7.30E-04	4.21E-02	2.48E+01
Xe-135m	Curies	0.00E+00	7.66E-04	0.00E+00	2.06E-03	2.83E-03
Xe-135	Curies	1.15E-02	1.20E+00	0.00E+00	1.31E-02	1.23E+00
Total For Period	Curies	3.22E-01	2.73E+01	3.46E-01	5.99E-01	2.85E+01
Iodines					•	
No Nuclides Found	Curies	N/A	N/A	N/A	N/A	N/A ,
Particulates				• •	• •	• •
No Nuclides Found	Curies	N/A	N/A	N/A	N/A	N/A
Tritium	-	• • •				
H_3	Curies	8 16E-02	7 64E-02	2 39E-02	3 42E-02	2 16E-01
C-14	Curies	4.37	3.74	4.24	3.97	16.32
•		, .				
Gross Alpha						
No Nuclides Found	Curies	N/A	N/A	N/A	N/A	N/A

<u>2009 Table 9.4 (cont)</u> <u>Site Gaseous Effluents - Ground Level Releases (2011)</u>

* Zeroes in this table indicate that no radioactivity was present at detectable levels.

<u>Table 9.5</u> <u>Site Liquid Effluents - Summation Of All Releases (2011)</u>

	Units	Quarter 1	Quarter 2	Quarter 3	Quarter 4	Annual
A. Fission And Activation Products						
1. Total Release (not including tritium, gases, alpha)	Curies	2.64E-04	1.79E-03	2.66E-03	1.85E-03	6.56E-03
2. Average diluted concentration during period	uCi/ml	1.73E-11	1.06E-10	1.23E-10	1.40E-10	3.86E-10
3. Percent of Applicable Limit	%	*	*	*	*	*
B. Tritium						
1. Total Release	Curies	1.14E+03	6.42E+02	7.33E+02	2.72E+02	2.79E+03
2. Average diluted concentration during period	uCi/ml	7.47E-05	3.79E-05	3.39E-05	2.06E-05	1.67E-04
3. Percent of Applicable Limit	%	*	*	*	*	*
C. Dissolved and Entrained Gases						
1. Total Release	Curies	4.78E-03	2.45E-02	6.34E-04	4.61E-04	3.04E-02
2. Average diluted concentration during period	uCi/ml	3.15E-10	1.45E-09	2.93E-11	3.49E-11	1.83E-09
3. Percent of Applicable Limit	%	*	*	*	*	*
D: Gross Alpha Radioactivity						
1. Total Release	Curies	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
E: Waste Vol Release (Pre-Dilution)	Liters	1.10E+06	1.03E+06	1.36E+06	8.89E+05	4.38E+06
F. Volume of Dilution Water Used	Liters	1.52E+10	1.69E+10	2.16E+10	1.32E+10	6.69E+10
					,	

* Applicable limits are expressed in terms of dose. Estimated Total Error For All Values Reported Is < 1.0%

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<u>Table 9.6</u> Site Liquid Effluents (2011)

<u>Continuous Mode</u>							
Nuclides Released	Units	Quarter 1	Quarter 2	Quarter 3	Quarter	4 Annual	
Fission & Activation Products			,				
No Nuclides Found	Curies	N/A	N/A	N/A	N/A	N/A	
Tritium							
H-3	Curies	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	
Dissolved And Entrained Gases							
No Nuclides Found	Curies	N/A	N/A	N/A	N/A	N/A	
Gross Alpha Radioactivity	Curies	0	0	.0	0	0	
Batch Mode							
Nuclides Released		Quarter 1	Quarter 2	2 Quart	ter 3	Quarter 4	Annual
Fission & Activation Products							
Cr-51		0.00E+00	0.00E+00	0.00E	+00	1.30E-04	1.30E-04
Mn-54		0.00E+00	0.00E+00	0.00E	C+00	7.40E-06	7.40E-06
Co-57		0.00E+00	0.00E+00	4.07E	E-06	0.00E+00	4.07E-06
Co-58		2.09E-04	1.73E-03	2.56E	E-03	1.62E-03	6.12E-03
Co-60		5.50E-05	3.74E-05	9.13E	2-05	7.30E-05	2.57E-04
Nb-95		0.00E+00	0.00E+00	0.00E	+00	2.02E-05	2.02E-05
Ba-139		0.00E+00	2.58E-05	0.00E	+00	0.00E+00	2.58E-05
Total For Period		2.64E-04	1.79E-03	2.66E	2-03	1.85E-03	6.56E-03
Tritium							
H-3		1.14E+03	6.42E+02	7.33E	C+02	2.72E+02	2.79E+03
Dissolved And Entrained Gases							
Kr-85m		0.00E+00	4.31E-05	0.00E	+00	3.06E-06	4.62E-05
Kr-85		1.24E-03	9.35E-04	0.00E	C+00	0.00E+00	2.18E-03
Kr-88		0.00E+00	2.92E-06	0.00E	C+00	0.00E+00	2.92E-06
Xe-131m		0.00E+00	5.04E-04	0.00E	-00	0.00E+00	5.04E-04
Xe-133m		0.00E+00	1.89E-04	0.00E	2+00	0.00E+00	1.89E-04
Xe-133		3.54E-03	2.27E-02	6.34E	E-04	4.49E-04	2.73E-02
Xe-135		0.00E+00	1.12E-04	0.00E	2+00	8.19E-06	1.20E-04
Total For Period		4.78E-03	2.45E-02	6.34E	E-04	4.61E-04	3 04E-02

* Zeroes in this table indicate that no radioactivity was present at detectable levels

	Table 9.7	
		•

Dose to a member of the public due to Liquid Releases (2011)

				Cumulati	ve Doses pe	r Quarter	•			
Organ	Limit	Units	Quarter 1	% of Tech Spec Limit	Quarter 2	% of Tech Spec Limit	Quarter 3	% of Tech Spec Limit	Quarter 4	% of Tech Spec Limit
Total Body	1.50E+00	mRem	3.26E-02	2.17E+00	3.43E-02	2.29E+00	3.85E-02	2.57E+00	4.34E-02	2.89E+00
Thyroid	5.00E+00	mRem	3.26E-02	6.52E-01	3.43E-02	6.87E-01	3.85E-02	7.70E-01	4.34E-02	8.68E-01
Organ	5.00E+00	mRem	3.26E-02	6.52E-01	3.43E-02	6.87E-01	3.85E-02	7.70E-01	4.34E-02	8.68E-01
Bone	5.00E+00	mRem	0.00E+00	0.00E+00	1.07E-10	2.14E-09	0.00E+00	0.00E+00	4.00E-08	8.01E-07

Cumulative Doses per Year

Organ	Tech Spec Limit	Units	Year to Ending Date	% of Tech Spec Limit	Receptor	Limit
Total Body	2.50E+01	mRem	1.49E-01	5.95E-01	Liquid Receptor - Adult	40CFR190.10 (a) TB
Thyroid	7.50E+01	mRem	1.49E-01	1.98E-01	Liquid Receptor - Adult	40CFR190.10 (a) Thyroid
Organ	2.50E+01	mRem	1.49E-01	5.96E-01	Liquid Receptor - Adult	40CFR190.10 (a) Organ
Bone	2.50E+01	mRem	4.01E-08	1.61E-07	Liquid Receptor - Adult	40CFR190.10 (a) Organ
		·				
Total Body	3.00E+00 -	mRem	1.49E-01	4.96E+00	Liquid Receptor - Adult	Liq Annual TB Dose
Thyroid	1.00E+01	mRem .	1.49E-01	1.49E+00	Liquid Receptor - Adult	Liq Annual Organ Dose
Organ	1.00E+01	mRem	1.49E-01	1.49E+00	Liquid Receptor - Adult	Liq Annual Organ Dose
Bone	1.00E+01	mRem	4.01E-08	4.01E-07	Liquid Receptor - Adult	Liq Annual Organ Dose
Total Dilutio	n Volume for	Quarter 1		1.52E+10		

Total Blagion (blance for Quarter 1	1.0 2.0 1.0
Total Dilution Volume for Quarter 2	1.69E+10
Total Dilution Volume for Quarter 3	2.16E+10
Total Dilution Volume for Quarter 4	1.32E+10

Organ = GI-Lii, Kidney, Liver, or Lung

			<u>Air Do</u>	ose Due To	Gaseous	<u>Releases –</u>	Site (2011))		
			•	Cumu	lative Doses	Per Quarter				
Type of Radiation	Tech Spec Limit	Units	Quarter 1	% of Tech Spec Limit	Quarter 2	% of Tech Spec Limit	Quarter 3	% of Tech Spec Limit	Quarter 4	% of Tech Spec Limit
Gamma Air	5.00E+00	mRad	6.21E-05	1.24E-03	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Beta Air	1.00E+01	mRad	7.54E-05	7.54E-04	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00

<u>Table 9.8</u>

Cumulative Doses Per Year

Type of Radiation	Tech Spec Limit	Units	Year to Ending Date	% of Tech Spec Limit	Receptor	Limit
Gamma Air	1.00E+01	mRad	6.21E-05	6.21E-04	Gas Receptor SB - Adult	NG Annual Gamma Air Dose
Gamma Air	1.00E+01	mRad	6.21E-05	6.21E-04	Gas Receptor SB - Teen	NG Annual Gamma Air Dose
Gamma Air	1.00E+01	mRad	6.21E-05	6.21E-04	Gas Receptor SB - Child	NG Annual Gamma Air Dose
Gamma Air	1.00E+01	mRad	6.21E-05	6.21E-04	Gas Receptor SB - Infant	NG Annual Gamma Air Dose
Beta Air	2.00E+01	mRad	7.54E-05	3.77E-04	Gas Receptor SB - Adult	NG Annual Beta Air Dose
Beta Air	2.00E+01	mRad	7.54E-05	3.77E-04	Gas Receptor SB - Teen	NG Annual Beta Air Dose
Beta Air	2.00E+01	mRad	7.54E-05	3.77E-04	Gas Receptor SB - Child	NG Annual Beta Air Dose
Beta Air	2.00E+01	mRad	7.54E-05	3.77E-04	Gas Receptor SB - Infant	NG Annual Beta Air Dose

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<u>Table 9.9</u>	
Dose to A Member Of The Public Due To Radioiodines, Tritium, and Particulates in Gaseous Releases (2011)	

Cumulative Doses Per Quarter % of Tech % of Tech **Tech Spec** % of Tech % of Tech Units Organ Quarter 1 Quarter 2 Quarter 3 **Quarter 4** Limit Spec Limit **Spec Limit Spec Limit Spec Limit** Total Body 7.5 3.03E-01 4.19E-01 3.00E-01 mRem 2.27E-02 3.14E-02 3.27E-02 4.36E-01 2.25E-02 7.5 1.95E-01 Thyroid mRem 1.46E-02 2.38E-02 3.17E-01 2.18E-02 2.91E-01 1.64E-01 1.23E-02 7.5 2.27E-02 3.03E-01 Organ 3.14E-02 4.19E-01 3.27E-02 4.36E-01 3.00E-01 mRem 2.25E-02 Skin 7.5 6.40E-08 8.54E-07 4.10E-08 5.46E-07 0.00E+00 0.00E+00 0.00E+00 0.00E+00 mRem 7.5 mRem 5.81E-02 Bone 6.62E-01 5.64E-02 5.28E-02 7.04E-01 7.75E-01 4.97E-02 7.52E-01

Organ = GI-LIi, Kidney, Liver, or Lung

Quarterly limit values are the same for both 40CFR190.10 (a) and Gas Annual Organ Dose limits

Cumulative Doses per Year

Organ	Tech Spec Limit	Units	Year to Ending Date	% of Tech Spec Limit	Receptor	Limit
Total Body	25	mRem	1.09E-01	4.38E-01	Gas Receptor SB - Child	40CFR190.10 (a) TB
Thyroid	75	mRem	7.26E-02	9.67E-02	Gas Receptor SB - Child	40CFR190.10 (a) Thyroid
Organ	25	mRem	1.09E-01	4.38E-01	Gas Receptor SB - Child	40CFR190.10 (a) Organ
Skin	25	mRem	1.05E-07	4.20E-07	Gas Receptor SB - Child	40CFR190.10 (a) Organ
Bone	25	mRem	2.17E-01	8.68E-01	Gas Receptor SB - Child	40CFR190.10 (a) Organ
		4				
Total Body	15	mRem	1.09E-01	7.30E-01	Gas Receptor SB - Child	Gas Annual Organ Dose
Thyroid	15	mRem	7.26E-02	4.84E-01	Gas Receptor SB - Child	Gas Annual Organ Dose
Organ	15	mRem	1.09E-01	7.30E-01	Gas Receptor SB - Child	Gas Annual Organ Dose
Skin	15	mRem	1.05E-07	7.00E-07	Gas Receptor SB - Child	Gas Annual Organ Dose
Bone	15	mRem	2.17E-01	1.45E+00	Gas Receptor SB - Child	Gas Annual Organ Dose

Organ = GI-LIi, Kidney, Liver, or Lung

TABLE 9.10 SOLID RADWASTE AND IRRADIATED FUEL SHIPMENTS -2011

A. Solid Waste Shipped Offsite for Burial or Disposal (Not Irradiated Fuel)

1. Type of Waste	Shipped M ³	Shipped Ci	Buried m ³	Buried Ci	Percent Error
a. Spent resins/filters	0	0 .	0	0	N/A
b. Dry active waste	662	0.213	70.7	0.149	+/- 25%
c. Irradiated components	-0-	-0-	-0-	-0-	N/A
d. Other (oil/miscellaneous liquids sent to processor for volume reduction)	5.1	1.22E-03	-0- ·	-0-	N/A
TOTAL	667	0.215	70.7	0.149	

<u>Note</u>: Shipped volumes and curies are not always equal to the buried volumes and curies since some disposal occurs outside the twelve month time period in which shipments occurred.

Dry active waste also includes some low-level radioactive resins, tank sediments and filters that are handled and processed in a manner that is consistent with this waste stream.

2. Estimate of Major Nuclide Composition (by type of waste)	Nuclide	% Abund.	Activity (Ci)
a. Spent resins/filters	-0-	-0-	-0-

Nuclides representing <1% of total shipped activity: Mn-54,Co-57,Co-58,Fe-59,Zn-65,Sr-90,Nb-95,ZR-95,Ag-110m,Sn-113,Sb-125,Ce-144,Pu-238,Pu-239/240,Am-241,Pu-241,Cm-242,Cm-243/244.

TABLE 9.10 (cont.) SOLID RADWASTE AND IRRADIATED FUEL SHIPMENTS -2011

2. Estimate of Major Nuclide Composition (by type of waste)	Nuclide	% Abund.	Activity (Ci)
	Fe-55	29.86	6.37E-02
	Ni-63	22.25	4.75E-02
	Co-60	19.95	4.26E-02
	Co-58	19.82	4.23E-02
	Nb-95 ,	. 2.11	4.50E-03
	Cr-51	1.38	2.95E-03
b. Dry active waste	Zr-95	1.20	2.56E-03
	Mn-54	1.04	2.23E-03
	C-14	0.18	3.76E-04
	H-3	0.01	1.47E-05
	Tc-99	LLD	-0-
	I-129	LLD	- 0-
	Other*	<u>2.20</u>	<u>4.70E-03</u>
	Total	100.00	2.13E-01

A. Solid Waste Shipped Offsite for Burial or Disposal (Not Irradiated Fuel) cont

*Nuclides representing <1% of total shipped activity: Cr-51,Mn-54,Co-57,Nb-95,ZR-95,Sb-125,Cs-134,Cs-137,Ce-144,Pu-238,Pu-239/240,Am-241,Cm-242,Cm-243/244.

•	•		
2. Estimate of Major Nuclide Composition (by type of waste)	Nuclide	% Abund.	Activity (Ci)
· · · · · · · · · · · · · · · · · · ·	Ni-63	43.68	5.34E-04
	Fe-55	39.76	4.86E-04
	Co-60	15.53	1.90E-04
d. Other	H-3	LLD	-0-
(oil/miscellaneous liquids sent to processor for volume	C-14	LLD	-0-
reduction)	Tc-99	LLD	-0-
	I-129	LLD	. - 0-
	<u>Other</u>	<u>1.03</u>	<u>1.25E-05</u>
	Total	100.00	1.22E-03

*Nuclides representing <1% of total shipped activity: Cr-51,Mn-54,Co-57,Nb-95,ZR-95,Sb-125,Cs-134,Cs-137,Ce-144,Pu-238,Pu-239/240,Am-241,Cm-242,Cm-243/244.

3. Solid Waste	Dispositio	n (Mode of Transport	ation: Truck)	, ,	
Waste Type	Waste Class	Container Type	Solidification Agent(s)	Number of Shipments	Destination
a. Resin/filters	N/A	N/A	N/A	N/A	N/A
b. Dry active waste	A	General Design	N/A	10	Energy Solutions Oak Ridge, TN
d. Other	A	General Design	N/A	*	Energy Solutions Oak Ridge, TN

*Note: Solid Waste "Other" was included as part of a Dry Active Waste Shipment

B. Irradiated Fuel Shipments (Disposition)

Number of Shipments Mode of Transportation Destination

0	N/A	N/A

Attachment 10.1

2010 Meteorological Joint Frequency Table

T. U. ELECTRIC COMPANY

HOURS AT EACH WIND SPEED AND DIRECTION

PERIOD OF RECORD: 01-JAN-11 00:00 TO 31-DEC-11 23:59

STABILITY CLASS: A ELEVATION: 10 m.

Wind Direction	1-3	4-7	8-12	13-18	19-24	>24	τοτα

N	1	6	31	21	10	0	. 6
NNE	2	18	12	7	1	0	4
NE	2	23	7	2	0	0	3
ENE	6	12	0	0	0	0	1
E	2	13	4	0	0	0	1
ESE	0	20	39	3	0	0	ε
SE	2	25	58	19	3	0	10
SSE	0	27	86	63	7	1	18
S	0	13	18	26	3	0	E
SSW	0	1	2	0	0	0	
SW	0	0	0	0	0	0	
WSW	1	1	Ō	0	0	0	
W	0	0	0	0	0	0	
WNW	0	0	0	0	1	0	
NW	0	1	6	6	5	8	2
NNW	0	2	17	24	19	11	
ARIABLE	14	7	0	0	0	0	2
Total	30	169	280	 171	49	20	71 71

T. U. ELECTRIC COMPANY

HOURS AT EACH WIND SPEED AND DIRECTION

PERIOD OF RECORD: 01-JAN-11 00:00 TO 31-DEC-11 23:59

STABILITY CLASS: B ELEVATION: 10 m.

Wind	Wind	Speed	(mph) at	10 m. lev	vel		
Direction	1-3	4-7	8-12	13-18	19-24	>24	TOTAL
N	3	6	18	6	2	0	35
NNE	6	11	9	1	0	0	27
NE	10	11	3	. 0	0	0	24
ENE	2	15	1	1	0	0	19
E	1	8	0	0	0	0	9
ESE	0	10	8	3	0	0	21
SE	0	12	27	14	. 1	0	54
SSE	1	20	33	45	14	6	119
S	1	13	35	54	35	6	144
SSW	1	7	15	6	2	2	33
SW	2	4	7	1	0	0	14
WSW	0	2	4	2	0	0	8
W	0	1	3	1	1	0	6
WNW	0	2	0	1	1	0	4
N₩	1	1	4	7	3	8	24
NNW	1	5	15	. 8	10	8	47
VARIABLE	17	1	0	0	0	0	18
Total	46	129	182	150	69	30	606

Periods of calm(hours): Hours of missing data:

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0

T. U. ELECTRIC COMPANY

HOURS AT EACH WIND SPEED AND DIRECTION

PERIOD OF RECORD: 01-JAN-11 00:00 TO 31-DEC-11 23:59

STABILITY CLASS: C ELEVATION: 10 m.

Wind	W 1 110	i Specu (61		
Direction	1-3	4-7	8-12	13-18	19-24	>24	TOT.
N	4	6	16	13	. 4	0	
NNE	2	4	12	3	1	0	
NE	4	б	2	2	0	0	
ENE	1	8	0	1	0	0	
E	· 1	7	3	0	0	0	
ESE	4	19	7	0	0	0	
SE	2	8	16	9 `	0	0	
SSE	2	8	24	48	15	6	1
S	2	11	38	77	52	21	2
SSW	0	5	22	38	7	3	
SW	1	11	15	5	2	0	
WSW	2	7	6	2	0	0	
₩ .	1	0	4	1	0	0	
WNW	0	2	4	5	1	0	
NW	1	. 9	7	9	5	7	
NNW	3	5	8	21	17	11	
ARIABLE	16	1	0	0	0	0	
Total	46	117	184	234	104	48	-

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R.G. 1.21 JOINT CHECKEN TABLE

T. ... TLECTRIC COMPANY

HOURS AT TACE WORD SPEED AND DIRECTION

TELIOD OF RECORD: 01-JAN-11 00:00 TO 31-DEC-11 23:59

STABLLIN CLASS: D. ELEVATION: 10 m.

Wind	Wind	Speed	(mph) at	10 m. lev	el		
Direction	1-3	'-7 	8-12	13-18	19-24	>24	TOTAL
N	14	76	199	121	15	0	425
NNE	11	44	101	34	0	Ó	190
NE	Ş	36	44	11	0	0	97
ENE	9	26	25	0	0	0	60
E	12	38	13	0	0	0	63
ESE	14	89	43	10	0	0	156
SE	11	105	226	81	3	, O	428
SSE	13	62	301	432	116	17	941
S	2	43	281	342	132	19	819
SSW	7	41	93	70	9	1	221
SW	10	25	39	12	0	0	86
WSW	4	20	16	4	0	0	44
W	0	9	11	8	1	0	29
WNW	3	11	45	21	3	0	83
N₩	1	15	40	44	12	6	118
NNW	9	19	135	158	56	8	385
ARIABLE	56	22	2	1	0	0	t 8
- · · ·	192	681	1614	1349	317	51	122/

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T. U. ELECTRIC COMPANY

HOURS AT EACH WIND SPEED AND DIRECTION

PERIOD OF RECORD: 01-JAN-11 00:00 TO 31-DEC-11 23:59

STABILITY CLASS: E ELEVATION: 10 m.

Direction	1-3	4-7	8-12	13-18	19-24	>24	тот
N	3	25	38	7	0	0	
NNE	3	32	28	1	. 0	0	
NE	1	8	4	1	0.	0	
ENE	0	6	2	0	0	0	
E	10	20	0	. 0	0	0	
ESE	15	65	11	0	0	0	
SE	22	202	135	2	0	0	i
SSE	6	127	220	31	0	0	
S	11	56	83	7	0	0	:
SSW	16	46	55	28	0	0	:
SW	9	11	33	3	0	0	
WSW	3	13	6	1	0	0	
W	3	13	5	1	Ò	0	
WNW	8	28	19	2	0	0	
NW	9	48	27	2	0	0	
NNW	3	24	24	4	1	0	
ARIABLE	55	13	3	1	0	0	
Total	177	737	693	 91	 1	0	1

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T. U. ELECTRIC COMPANY

HOURS AT EACH WIND SPEED AND DIRECTION

PERIOD OF RECORD: 01-JAN-11 00:00 TO 31-DEC-11 23:59

STABILITY CLASS: F ELEVATION: 10 m.

Wind			0.40	·			
Direction	1-3	4-7	8-12	13-18	19-24	>24	101
N	3	0	3	0	0	0	
NNE	0	• 1	3	0	0	0	
NE	1	0	0	0	0	O	
ENE	1	1	0	0	0	0	
Ė	1	0	0	0	0	0	
ESE	3	4	. 0	0	0	0	
SE	12	46	13	0	0	0	
SSE	18	33	13	1	0	0	
S	17	28	10	0	0	0	
SSW	13	23	18	• 0	0	Ó	
SW	28	25	8	0	0	0	
WSW	27	27	6	0	0	0	
W	12	12	3	0	0	0	
WNW	11	15	1	0	0	0	
NW	11	37	4	Ó	0	0	
NNW	3	1	0	0	0	0	
ARIABLE	23	1	0	0	0	.0	نہ سہ سف منہ من
Total	184	254	82	Í	0	0	5

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T. U. ELECTRIC COMPANY

HOURS AT EACH WIND SPEED AND DIRECTION

PERIOD OF RECORD: 01-JAN-11 00:00 TO 31-DEC-11 23:59

STABILITY CLASS: G ELEVATION: 10 m.

Direction	1-3	4-7	8-12	13-18	19-24	>24	тс
N	2	0	0	0	0	0	
NNE	0	0	0	0	0	0	
NE	0	0	0	0	0	0	
ENE	0	0	0	0	0	0	
E	0	0	0	0	0	0	
ESE	0	0	0	0	0	0	
SE	0	4	1	. 0	· O	0	-
SSE	1	1	0	0	0	. 0	
S	8	3	1	0	0	0	
SSW	14	17	4	0	0	0	
SW	16	25	0	· 0	0	0	
WSW	13	23	4	0	0	0	
W	8	7	0	0	0	0	
WNW	13	9	0	0	0	0	
NW	4	20	0	0	• 0	0	
NNW	1	0	0	0	0	0	
VARIABLE	12	0	0	0	0	0	
Total	92	109	10	0	0	0	

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T. U. ELECTRIC COMPANY

HOURS AT EACH WIND SPEED AND DIRECTION

PERIOD OF RECORD: 01-JAN-11 00:00 TO 31-DEC-11 23:59

STABILITY CLASS: ALL ELEVATION: 10 m.

Wind	Win	d Speed (mph) at 1	0 m. 1ev	/el		
Direction	1-3	4-7	8-12	13-18	19-24	>24	TOTA
N	30	119	305	168	31	0	65:
NNE	24	110	165	46	2	0	343
NE	24	84	60	16	0	0	184
ENE	19	68	28	2	0	0	117
E	27	86	20	0	0	0	133
ESE	36	20 7	108	16	0	0	367
SE	49	402	476	125	7	0	1059
SSE	41	278	677	620	152	30	1798
S	41	167	466	506	222	46	1448
SSW	51	140	209	142	18	6	566
SW	66	101	102	21	2	0	292
WSW	50	93	42	9	0	0	194
W	24	42	26	11	2	0	105
WNW	35	67	69	29	6	0	208
NW	27	131	88	68	25	29	368
NNW	20	56	199	215	103	38	631
ARIABLE	193	45	5	2	0	. 0	245
Total	757	2196	3045	1996	570	149	8713

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Attachment 10.2

Carbon 14 Supplemental Information

Carbon-14 Supplemental Information

Carbon-14 (C-14) is a naturally occurring isotope of carbon produced by interactions with cosmic radiation in the atmosphere with a half-life of 5730 years. Nuclear weapons testing in the 1950s and 1960s significantly increased the amount of C-14 in the atmosphere. C-14 is also produced in commercial nuclear reactors, but the amounts are much less than the amounts produced from natural formation or from weapons testing.

In June 2009, the NRC provided revised guidance in Regulatory Guide 1.21, *Measuring, Evaluating and Reporting Radioactive Material in Liquid and Gaseous Effluents and Solid Waste, Revision 2,* establishing an updated approach for identifying principal radionuclides. Because the overall quantity of radioactive releases has steadily decreased due to improvements in power plant operations, C-14 now qualifies as a "principal radionuclide" (anything greater than one percent of overall radioactivity in effluents) under federal regulations at many plants. In other words, C-14 has not increased and C-14 is not a new nuclear plant emission. Rather, the improvements in the mitigation of other isotopes have made C-14 more prominent.

The dose contribution of C-14 from liquid radioactive waste is essentially insignificant compared to that contributed by gaseous radioactive waste. Therefore the evaluation of C-14 in liquid radioactive waste is not required by the new Reg. Guide 1.21, Rev. 2. The Reg. Guide 1.21, Rev. 2 also states that the quantity of gaseous C-14 released to the environment can be estimated by use of a C-14 source term production model.

A recent study produced by EPRI (*Estimation of Carbon-14 in Nuclear Power Plant Gaseous Effluents*, EPRI, Palo Alto, CA: 2010, 1021106) developed a model for estimation of C-14 source production. This model was used by CPNPP for the 2010 Radioactive Effluent Release Report. Also in the CPNPP report, the assumption that 70% of the C-14 gaseous effluent is estimated to be from batch releases (e.g. WGDTs), and 30% of C-14 gaseous effluent is estimated to be from continuous releases through the unit vents (Ref. IAEA Technical Reports Series no. 421, "Management of Waste Containing Tritium and Carbon-14", 2004).

The C-14 released from PWR's is primarily a mix of organic carbon and carbon dioxide released from the waste gas system. The C-14 species initially produced are primarily in the organic form, such as methane. The C-14 in the primary coolant can be converted to an inorganic chemical form of primarily carbon dioxide through a chemical transformation. Studies documented by the EPRI Report *Characterization of Carbon-14 Generated by the Nuclear Power Industry*, EPRI Palo Alto, CA: 1995, TR-105715, measured C-14 releases from PWRs indicating a range of 70% to 95% organic. The average value was indicated to be 80% organic with the remainder being carbon dioxide. As a result, a value of 80% organic C-14 is assumed by the CPNPP Radioactive Effluent Release Report methodology.

The public dose estimates from airborne C-14 in the CPNPP Effluent report are performed using dose models from NUREG-0133 and Regulatory Guide 1.109. The dose models and assumptions used for the dose estimates of C-14 are documented in the 2011 ODCM changes. The estimated C-14 dose impact on the maximum organ dose from airborne effluents released during 2011 is well below the 10CFR50, Appendix I, ALARA design objective of 15 mRem/yr per unit.

Putting Radiation Dose in Context

Humans are exposed to radiation every day. The majority comes from natural sources including the earth, food and water consumption, the air, the sun and outer space. A smaller fraction radiation comes from man-made source such as X-rays, nuclear medical treatments, building materials, nuclear power plants, smoke detectors and televisions.

Radiation is measured in units called millirem (mRem). One mRem is a very small amount of exposure. On average, Americans receive 620 mRem of radiation dose every year. Approximately one-half of the dose comes from natural sources and the other half comes from medical procedures such as CAT scans.

The table below can help to give some perspective to dose from various sources.

Source	Average Annual Dose
Smoke detector in the home	0.008 mRem
Live within 50 miles of a nuclear power plant	0.009 mRem
Live within 50 miles of a coal-fired power plant*	0.03 mRem
NRC guideline for keeping radiation dose from nuclear power plants as low as reasonably achievable (ALARA)	5 mRem
Round trip flight from New York City to Los Angeles	5 mRem
Medical X-ray	10 mRem
EPA limit for dose to the public from the commercial nuclear fuel cycle	25 mRem
Food and water consumed throughout the course of one year	30 mRem
NRC limit for dose to the public from nuclear power plants	100 mRem
Mammogram	100 mRem
Average annual exposure for a nuclear power plant worker	120 mRem
Average annual exposure from background radiation	300 mRem
CT scan	1,000 mRem
NRC's annual limit for occupational exposure	5,000 mRem
Cardiac catheterization or coronary angiogram	5,000 mRem

*Coal is naturally radioactive.

Sources: U.S. Environmental Protection Agency, Health Physics Society.