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JAFP-12-0044

April 30, 2012

U.S. Nuclear Regulatory Commission Document Control Desk Washington, D.C. 20555

SUBJECT: 2011 Annual Radioactive Effluent Release Report James A. FitzPatrick Nuclear Power Plant Docket No. 50-333 License No. DPR-59

Dear Sir or Madam:

This letter transmits the James A. FitzPatrick Nuclear Power Plant's (JAF) Annual Radioactive Effluent Release Report, for the period of January 1, 2011 through December 31, 2011. This document is submitted in accordance with the Reporting Requirements of the Technical Specifications, Section 5.6.3 and Appendix H of the Technical Requirements Manual, "Offsite Dose Calculation Manual (ODCM)," Part 1 Section 6.2, Annual Radioactive Effluent Release Report.

This report (Enclosure) includes, as an Addendum, an Assessment of the Radiation Doses to the Public due to the radioactive liquid and gaseous effluents released during the 2011 calendar year. The format used for the effluent data is outlined in Appendix B of Regulatory Guide 1.21, Revision 1. Distribution is in accordance with Regulatory Guide 10.1, Revision 4.

There are no commitments contained in this letter.

If you have any questions concerning the enclosed report, please contact Laurie Rayle, Chemistry Manager at (315) 349-6748.

Joseph Pecl

Licensing Manager

JP/LR/ed

Enclosure: Annual Radioactive Effluent Release Report, January 1, 2011-December 31, 2011

cc: next page

Mr. William Dean Regional Administrator, Region I U. S. Nuclear Regulatory Commission 475 Allendale Road King of Prussia, PA 19406-1415

Resident Inspector's Office U.S. Nuclear Regulatory Commission James A. FitzPatrick Nuclear Power Plant P.O. Box 136 Lycoming, NY 13093

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Document Contents: 001 Transmittal Letter, JAFP-12-0044 with enclosure

JAFP-12-0044

Enclosure

2011 Annual Radioactive Effluent Release Report (35 Pages)

JANUARY 1, 2011 - DECEMBER 31, 2011

DOCKET NO. 50-333

LICENSE NO. DPR-59

SUPPLEMENTAL INFORMATION

FACILITY: JAFNPP LICENSEE: ENTERGY NUCLEAR OPERATIONS, INC.

1. Offsite Dose Calculation Manual Part 1 Radiological Controls

- a. Fission and Activation Gases:
 - (1) The dose rate at or beyond the site boundary due to radioactive materials released from the plant in gaseous effluent shall be limited as follows:
 - (a) Less than or equal to 500 mrem/year to the whole body and less than or equal to 3000 mrem/year to the skin from noble gases.
 - (2) The air dose to areas at or beyond the site boundary from noble gases released from the plant in gaseous effluent shall be limited:
 - (a) During any calendar quarter, to less than or equal to 5 mrad from gamma radiation, and less than or equal to 10 mrad from beta radiation; and,
 - (b) During any calendar year, to less than or equal to 10 mrad from gamma radiation and less than or equal to 20 mrad from beta radiation.
- b. Tritium, Iodines and Particulates, Half Lives > 8 days:
 - (1) The dose to a member of the public at or beyond the site boundary from Iodine-131, Iodine-133, Tritium, and radionuclides in particulate form with half-lives greater than 8 days released from the plant in gaseous effluent shall be limited:
 - (a) During any calendar quarter to less than or equal to 7.5 mrem to any organ; and,
 - (b) During any calendar year to less than or equal to 15 mrem to any organ.
 - (c) Less than 0.1% of the limits of Specification 3.4.1.c.1.a and 3.4.1.c.1.b as a result of burning contaminated oil.
 - (2) The dose rate at or beyond the site boundary due to radioactive materials released from the plant in gaseous effluents shall be limited as follows:
 - (a) Less than or equal to 1500 mrem/year to any organ from Iodine-131, Iodine-133, Tritium and for radioactive materials in particulate form with half-lives greater than 8 days (inhalation pathway only).

SUPPLEMENTAL INFORMATION (continued)

c. Liquid Effluents:

- (1) The concentration of radioactive materials released to the unrestricted areas shall not exceed ten times the values specified in 10 CFR 20.1001-20.2402, Appendix B, Table 2, Column 2. For dissolved or entrained noble gases the concentration shall be limited to $2.00\text{E-}04 \,\mu\text{Ci/ml}$.
- (2) The dose to a member of the public from radioactive materials released from the plant in liquid effluents to unrestricted areas shall be limited as follows:
 - (a) During any calendar quarter, limited to less than or equal to 1.5 mrem to the whole body and to less than or equal to 5 mrem to any organ; and,
 - (b) During any calendar year, limited to less than or equal to 3 mrem to the whole body and to less than or equal to 10 mrem to any organ.

2. 10X Effluent Concentrations

a.	Fission and activation gases:	(None specified)			
b.	Iodines:	(None specified)			
c.	Particulates, half-lives >8 days:	(None specified)			
d.	Liquid effluents:	Quarter 1	Quarter 2	Quarter 3	Quarter 4
	(1) Fission and activation products (mixture EC) (μCi/ml)	None	None	None	None
	(2) Tritium (µCi/ml)	1.00E-02	1.00E-02	1.00E-02	1.00E-02
	(3) Dissolved and entrained gases (μCi/ml)	2.00E-04	2.00E-04	2.00E-04	2.00E-04

SUPPLEMENTAL INFORMATION (continued)

3. <u>Average Energy</u> (None specified)

4. Measurements and Approximations of Total Radioactivity

- a. Fission and Activation Gases: Continuous monitor on each release path calibrated to a marinelli grab sample analyzed by gamma spectroscopy; bubbler grab sample analyzed for Tritium.
- b. Iodines: Gamma spectral analysis of charcoal cartridge and particulate filter on each release path.
- c. Particulates: Gamma spectral analysis of each particulate filter and charcoal cartridge for each release path. A four week per quarter composite of particulate filters for each release path for Strontium-89 and Strontium-90. One week per month particulate filter for each release path for gross alpha.
- d. Liquid Effluents: Gamma spectral analysis of each batch discharged, except composite analysis for Strontium-89, Strontium-90, Iron-55, Tritium, and Alpha.
- e. Solid Waste: Gamma spectral analysis of a representative sample of each waste shipment. Scaling factors established from off-site composite sample analyses to estimate concentration of non-gamma emitters. Low activity trash shipments, curie content estimated by dose rate measurement and application of appropriate scaling factors.
- f. Error Estimation Method: Overall error for sampling and analysis estimated by combining individual errors using error propagation methods. This process is composed of determinate and undeterminate errors.

Determinate - Pump flowrates, volume measurements and analysis collection yields Undeterminate - Random counting error estimated using accepted statistical calculations

SUPPLEMENTAL INFORMATION (continued)

5. Batch Releases

a.	Liquid: Canal	Quarter 1	Quarter 2	Quarter 3	Quarter 4
	(1) Number of batch releases:	7.20E+01	7.20E+01	6.10E+01	8.00E+01
	(2) Total time period for batch release: (min)	4.03E+03	4.48E+03	3.26E+03	4.21E+03
	(3) Maximum time period for batch release: (min)	7.00E+01	5.35E+02	9.50E+01	8.70E+01
	(4) Average time period for batch release: (min)	5.60E+01	6.22E+01	5.35E+01	5.26E+01
	(5) Minimum time period for batch release: (min)	2.00E+00	1.00E+00	1.00E+00	1.00E+00
	(6) Total Activity Released (Ci)	2.25E-02	1.32E-02	9.85E-03	7.93E-03
	(7) Total Volume Released (liters)	4.39E+06	4.26E+06	3.57E+06	4.39E+06
b.	Liquid: Non-Canal	Quarter 1	Quarter 2	Quarter 3	Quarter 4
b.	Liquid: Non-Canal (1) Number of batch releases:	<u>Quarter 1</u>	<u>Quarter 2</u> 9.00E+00	<u>Quarter 3</u> 1.30E+01	<u>Quarter 4</u> 6.00E+00
b.	-	Quarter 1			
b.	(1) Number of batch releases:(2) Total time period for batch		9.00E+00	1.30E+01	6.00E+00
b.	 (1) Number of batch releases: (2) Total time period for batch release: (min) (3) Maximum time period for 		9.00E+00 7.81E+02	1.30E+01 1.63E+03	6.00E+00 7.01E+02
b.	 (1) Number of batch releases: (2) Total time period for batch release: (min) (3) Maximum time period for batch release: (min) (4) Average time period for 		9.00E+00 7.81E+02 3.00E+02	1.30E+01 1.63E+03 6.07E+02	6.00E+00 7.01E+02 3.76E+02
b.	 (1) Number of batch releases: (2) Total time period for batch release: (min) (3) Maximum time period for batch release: (min) (4) Average time period for batch release: (min) (5) Minimum time period for 		9.00E+00 7.81E+02 3.00E+02 8.68E+01	1.30E+01 1.63E+03 6.07E+02 1.26E+02	6.00E+00 7.01E+02 3.76E+02 1.17E+02

c. Gaseous

There were no gaseous batch releases for this report period.

TABLE 1AGASEOUS EFFLUENTS - SUMMATION OF ALL RELEASES

	GASEOUS EFFLUENTS - SUMMATION OF ALL RELEASES			SES	EST TOTAL		
		<u>UNIT</u>	<u>QTR 1</u>	<u>QTR 2</u>	<u>QTR 3</u>	<u>QTR 4</u>	EST TOTAL ERROR %
A.	FISSION AND ACTIVATION GASES						
	1. Total Release	Ci	8.20E+00	1.92E+01	8.45E+00	7.20E+00	≤2.50E+01
	2. Average release rate for period	µCi/sec	1.04E+00	2.45E+00	1.06E+00	9.05E-01	
	3. Applicable ODCM Limit	%	*	*	*	*	
B.	IODINE-131						
	1. Total Iodine-131	Ci	1.04E-05	5.03E-05	3.75E-05	6.76E-05	≤2.50E+01
	2. Average release rate for period	µCi/sec	1.32E-06	6.40E-06	4.72E-06	8.51E-06	
	3. Applicable ODCM Limit	%	*	*	*	*	
C.	PARTICULATES						
	1. Particulates with half-lives >8 days	Ci	5.08E-06	2.26E-06	2.19E-06	4.56E-05	≤3.60E+01
	2. Average release rate for period	µCi/sec	6.46E-07	2.87E-07	2.75E-07	5.74E-06	
	3. Applicable ODCM Limit	%	*	*	*	*	
	4. Gross alpha radioactivity	Ci	2.77E-07	3.14E-07	2.55E-07	3.95E-07	≤2.50E+01
D.	TRITIUM						
	1. Total Release	Ci	5.06E+00	7.26E+00	6.86E+00	5.98E+00	≤2.50E+01
	2. Average release rate for period	µCi/sec	6.43E-01	9.23E-01	8.63E-01	7.52E-01	
	3. Applicable ODCM Limit	%	*	*	*	*	
E.	CARBON (See attachment 8)						
*F.	PERCENT OF APPLICABLE ODCM L	IMITS					
	FISSION AND ACTIVATION GASES	<u>UNIT</u>	QTR 1	<u>QTR 2</u>	QTR 3	QTR 4	
	1. Quarterly gamma air dose limit	%	3.79E-03	9.86E-03	3.46E-03	3.33E-03	
	 Quarterly beta air dose limit 	%	2.43E-04	7.92E-04	1.89E-04	2.61E-04	
	3. Yearly gamma air dose limit	%	1.89E-03	4.93E-03	1.73E-03	1.67E-03	
	4. Yearly beta air dose limit	%	1.21E-04	3.96E-04	9.44E-05	1.31E-04	
	5. Whole body dose rate limit	%	4.02E-03	1.46E-02	1.48E-03	2.09E-03	
	6. Skin dose rate limit	%	8.62E-04	3.15E-03	4.36E-04	6.32E-04	
	HALOCENS TRITHIM AND BARTICI	III ATES WI		IVES >9 DAV	c		

HALOGENS, TRITIUM AND PARTICULATES WITH HALF-LIVES >8 DAYS

7.	Quarterly dose limit (organ)	%	3.48E-03	9.24E-03	9.84E-03	1.59E-02
8.	Yearly dose limit (organ)	%	1.74E-03	4.62E-03	4.92E-03	7.94E-03
9.	Organ dose rate limit	%	1.60E-05	1.81E-05	1.61E-05	1.78E-05

TABLE 1BGASEOUS EFFLUENTS - ELEVATED RELEASE

CONTINUOUS MODE

NUCLIDES RELEASED

1.	Fission Gases	<u>UNIT</u>	QUARTER 1	QUARTER 2	QUARTER 3	QUARTER 4
	Argon-41	Ci	3.67E+00	2.25E+00	3.03E+00	2.68E+00
	Krypton-85m	Ci	7.51E-01	1.25E+00	2.41E+00	1.05E+00
	Krypton-87	Ci	2.88E-01	1.27E+00		
	Krypton-88	Ci	1.93E-01	1.31E+00	1.02E+00	5.05E-01
	Xenon-133	Ci	1.43E+00	5.51E-01	1.96E+00	9.03E-01
	Xenon-133m	Ci		6.54E-02		
	Xenon-135	Ci	4.02E-01	1.24E+00		6.04E-02
	Xenon-135m	Ci	3.96E-01	2.36E+00		2.85E-01
	Xenon-137	Ci		1.11E+00		6.60E-01
	Xenon-138	Ci	1.03E+00	7.81E+00		1.01E+00
	TOTAL	Ci	8.16E+00	1.92E+01	8.42E+00	7.15E+00
2.	<u>Iodines</u>					
	Iodine-131	Ci	7.38E-06	3.88E-05	2.49E-07	2.67E-07
	Iodine-133	Ci	1.14E-05	2.76E-04	4.45E-06	2.22E-06
	Iodine-135	Ci		4.94E-04		
	TOTAL	Ci	1.88E-05	8.09E-04	4.70E-06	2.49E-06
3.	Particulates					
	Barium-140	Ci		3.64E-07		
	Strontium-89	Ci	8.10E-07	1.04E-06	8.96E-07	1.81E-07
	TOTAL	Ci	8.10E-07	1.40E-06	8.96E-07	1.81E-07
4.	<u>Tritium</u>					
	Hydrogen-3	Ci	4.38E-01	3.63E-01	8.95E-01	1.00E+00

Note: There were no batch releases for this report period.

TABLE 1CGASEOUS EFFLUENTS - GROUND LEVEL RELEASES

CONTINUOUS MODE

1.	Fission Gases	<u>UNIT</u>	<u>QUARTER 1</u>	QUARTER 2	QUARTER 3	QUARTER 4
	Xenon-133	Ci	2.52E-02	8.75E-03	9.48E-03	2.65E-02
	Xenon 135	Ci	1.96E-02	6.97E-03	8.33E-03	1.50E-02
	Xenon-135M	Ci	2.18E-03	6.55E-03	4.30E-03	3.59E-03
	TOTAL	Ci	4.70E-02	2.23E-02	2.21E-02	4.51E-02
2.	<u>Iodines</u>					
	Iodine-131	Ci	2.98E-06	1.15E-05	3.73E-05	6.73E-05
	Iodine-133	Ci	7.42E-06	1.31E-04	1.75E-04	4.92E-04
	Iodine-135	Ci				1.14E-05
	TOTAL	Ci	1.04E-05	1.43E-04	2.12E-04	5.71E-04
3.	<u>Particulates</u>					
	Cobalt-58	Ci				1.61E-05
	Cobalt-60	Ci	1.26E-06			6.96E-06
	Manganese-54	Ci	4.32E-07			1.61E-05
	Strontium-89	Ci	2.57E-06	8.52E-07	1.29E-06	6.87E-07
	Zinc-65	Ci				5.59E-06
	TOTAL	Ci	4.26E-06	8.52E-07	1.29E-06	4.54E-05
4.	<u>Tritium</u>					
	Hydrogen-3	Ci	4.62E+00	6.90E+00	5.96E+00	4.98E+00

Note: There were no batch releases for this report period.

NUCLIDES RELEASED

TABLE 2A LIQUID EFFLUENTS - SUMMATION OF ALL RELEASES

			<u>UNIT</u>	<u>QTR 1</u>	<u>QTR 2</u>	<u>QTR 3</u>	<u>QTR 4</u>	EST TOTAL ERROR %
A.	FIS	SSION AND ACTIVATION PRODUC	CTS					
	1.	Total Release (not including tritium, gases and alpha)	Ci	None	None	None	None	≤2.50E+01
	2.	Average diluted concentration during period	µCi/ml	None	None	None	None	
	3.	Applicable ODCM Limit	%					
B.	TR	AITIUM						
	1. 2.	Total Release Average diluted concentration	Ci	2.25E-02	1.40E-02	1.15E-02	8.86E-03	≤2.50E+01
	3.	during period (Note 1) Applicable ODCM Limit	µCi/ml %	3.72E-09 *	1.76E-07 *	3.94E-07 *	2.00E-07 *	
C.	DI	SSOLVED AND ENTRAINED GASE	2S					
	1. 2.	Total Release Average diluted concentration	Ci	None	None	None	None	≤2.50E+01
	3.	during period Applicable ODCM Limit	µCi/ml %	None *	None *	None *	None *	
D.	GR	ROSS ALPHA RADIOACTIVITY						
	1.	Total Release	Ci	None	None	None	None	≤4.20E+01
Е.		DLUME OF WASTE RELEASED RIOR TO DILUTION)	liters	4.39E+06	4.55E+06	4.21E+06	4.66E+06	
F.		DLUME OF DILUTION WATER ED DURING PERIOD	liters	6.03E+09	6.60E+09	5.09E+09	6.29E+09	
*G.	PE	RCENT OF APPLICABLE ODCM L	LIMITS					
	1.	Quarterly Whole Body Dose	%	1.54E-06	3.50E-04	3.39E-04	3.29E-04	
	2.	Quarterly Organ Dose	%	4.63E-07	1.05E-04	1.02E-04	9.86E-05	
	3.	Annual Whole Body Dose	%	7.71E-07	1.75E-04	1.70E-04	1.64E-04	
	4.	Annual Organ Dose	%	2.31E-07	5.25E-05	5.09E-05	4.93E-05	

Note: Concentration includes summation from diluted and undiluted values from Canal and Non-Canal releases (Table 2B).

TABLE 2BLIQUID EFFLUENTS CANAL

BATCH MODE

NU	JCLIDES RELEASED	<u>UNIT</u>	QUARTER 1	QUARTER 2	QUARTER 3	QUARTER 4
1.	Fission and Activation Produc	<u>ts</u>				
	None	Ci				
2.	<u>Tritium</u>					
	Hydrogen-3	Ci	2.25E-02	1.32E-02	9.85E-03	7.93E-03
3.	Dissolved and Entrained Gase	<u>s</u>				
	None	Ci				

Note: There were no continuous mode discharges during this report period.

TABLE 2B (SUPPLEMENT)LIQUID EFFLUENTS NON-CANAL

BATCH MODE

<u>NU</u>	UCLIDES RELEASED	<u>UNIT</u>	QUARTER 1	QUARTER 2	QUARTER 3	QUARTER 4
1.	Fission and Activation Product	<u>s</u>				
	None	Ci				
•	T • •					
2.	<u>Tritium</u>					
	Hydrogen-3	Ci		7.93E-04	1.66E-03	9.26E-04
3.	Dissolved and Entrained Gases					
	None	Ci				

Note: There were no continuous mode discharges during this report period.

TABLE 3ASOLID WASTE AND IRRADIATED FUEL SHIPMENTS

A. SOLID WASTE SHIPPED OFFSITE FOR BURIAL OR DISPOSAL (NOT IRRADIATED FUEL)

						EST. TOTAL
1.	<u>Type of Waste</u>	UNIT	CLASS A	CLASS B	CLASS C	ERROR %
	a. Spent resins, filter sludges,	m^3	0.00E+00	0.00E+00	0.00E+00	0.00E+00
	evaporator bottoms, etc.	Ci	0.00E+00	0.00E+00	0.00E+00	0.00E+00
	b. Dry compressible waste,	m^3	0.00E+00	0.00E+00	0.00E+00	0.00E+00
	contaminated equipment, etc.	Ci	0.00E+00	0.00E+00	0.00E+00	0.00E+00
	c. Irradiated componants,	m^3	0.00E+00	0.00E+00	0.00E+00	0.00E+00
	control rods, etc.	Ci	0.00E+00	0.00E+00	0.00E+00	0.00E+00
	d. Other: Dry compressible	m^3	9.66E+02	0.00E+00	0.00E+00	1.00E+01
	waste, contaminated equipment, spent resins for volume reduction.	Ci	4.74E+01	0.00E+00	0.00E+00	1.00E+01

2. Estimate of Major Nuclide Composition (by type of waste)

a. Spent resins, filter sludges, evaporator bottoms, etc.

None

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b. Dry compressible waste, contaminated equipment, etc.

None

c. Irradiated components, control rods, etc.

None

d. Other: Dry compressible waste, contaminated equipment, spent resins contaminated oil, glycol and water for volume reduction.

Isotope	Percent	Curies	Isotope	Percent	Curies
Cerium-144	1.73E-02	8.22E-03 E	Cobalt-58	1.83E-01	8.68E-02 E
Cobalt-60	1.20E+01	5.70E+00 E	Cesium-134	1.26E-01	5.97E-02 E
Cesium-137	9.08E-01	4.31E-01 E	Iron-55	6.01E+01	2.85E+01 E
Carbon-14	9.15E-01	4.31E-01 E	Manganese-54	6.43E+00	3.05E+00 E
Nickel-63	5.89E+00	2.80E+00 E	Strontium-90	2.03E-01	9.64E-02 E
Zinc-65	1.30E+01	6.14E+00 E	Hydrogen-3	2.63E-02	1.25E-02 E
Chromium-51	3.89E-02	1.85E-02 E	Iron-59	1.28E-01	6.07E-02 E
Antimony-124	3.64E-02	1.83E-02 E	Antimony-125	1.99E-02	9.42E-03 E

(E-Estimated M-Measured)

Percentage of nuclides and total activities are based on a combination of direct measurements and scaling for nongamma emitting nuclides.

TABLE 3A (continued) SOLID WASTE AND IRRADIATED FUEL SHIPMENTS

3. Solid Waste Disposition

No. of Shipments	Mode of Transportation	Destination				
12	Truck	*Energy Solutions Oak Ridge, TN				
27	Truck	*Energy Solutions Kingston, TN				
1	Truck	*Perma-Fix of Florida Gainsville, Fl				
* Volume Reduct	tion Facility					
IRRADIATED FUEL SHIPMENTS (Disposition)						

B. IRRADIATED FUEL SHIPMENTS (Disposition)

No. of Shipments	Mode of Transportation	Destination
None		

TABLE 3B SOLID WASTE AND IRRADIATED FUEL SHIPMENTS

A. NRC CLASS A

SOURCE OF <u>WASTE</u>	PROCESSING EMPLOYED	CONTAINER VOLUME	TYPE OF <u>CONTAINER</u>	NUMBER OF <u>CONTAINERS</u>
Dry compressible Waste (DAW), Contaminated Equipment, etc.	Non-compacted	1280 ft^3	STC	9
Dry compressible Waste (DAW), Contaminated Equipment, etc.	Non-compacted	96 ft^3	STC	28
Dry compressible Waste (DAW), Contaminated Equipment, etc.	Non-compacted	1033 ft^3	STC	16
Dry compressible Waste (DAW), Contaminated Equipment, etc.	Non-compacted	4.1 ft^3	STC	3
Dry compressible Waste (DAW), Contaminated Equipment, etc.	Non-compacted	7.5 ft^3	STC	2
Dry compressible Waste (DAW), Contaminated Equipment, etc.	Non-compacted	38.4 ft^3	STC	1
Dry compressible Waste (DAW), Contaminated Equipment, etc.	Non-compacted	159 ft^3	STC	1
Spent Resins, Filter Sludges, evaporator Bottoms, etc.	Air Drying Non-compacted	205.8 ft^3	STC	8
Spent Resins, Filter Sludges, evaporator Bottoms, etc.	Air Drying Non-compacted	120.3 ft^3	HIC	2

Contaminated Oil	Non-compacted	7.5 ft^3	STC	8
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TABLE 3BSOLID WASTE AND IRRADIATED FUEL SHIPMENTS

B. NRC CLASS B

SOURCE OF	PROCESSING	CONTAINER	TYPE OF	NUMBER OF
<u>WASTE</u>	EMPLOYED	VOLUME	<u>CONTAINER</u>	CONTAINERS

None

C. NRC CLASS C

SOURCE OF	PROCESSING	CONTAINER	TYPE OF	NUMBER OF
WASTE	<u>EMPLOYED</u>	<u>VOLUME</u>	<u>CONTAINER</u>	CONTAINERS

None

HIC- High Integrity Container STC- Strong Tight Container

ATTACHMENT NO. 1

CHANGES TO THE OFFSITE DOSE CALCULATION MANUAL (ODCM)

In accordance with the James A. FitzPatrick Nuclear Power Plant Offsite Dose Calculation Manual (ODCM), Part 1 Radiological Controls (REC) Section 6.2.3, changes made to the Offsite Dose Calculation Manual (ODCM) during the reporting period shall be included in the Annual Radioactive Effluent Release Report.

There were no changes to the Offsite Dose Calculation Manual (ODCM).

ATTACHMENT NO. 2

SUMMARY OF CHANGES TO THE PROCESS CONTROL PROGRAM

In accordance with the James A. FitzPatrick Nuclear Power Plant Offsite Dose Calculation Manual (ODCM), Part 1 Radiological Controls (REC) Section 6.2.3, changes made to the Process Control Program (PCP) during the reporting period shall be included in the Annual Radioactive Effluent Release Report.

There were no changes to the Process Control Program.

ATTACHMENT NO. 3

SUMMARY OF CHANGES TO THE ENVIRONMENTAL MONITORING AND DOSE CALCULATION LOCATIONS

In accordance with the James A. FitzPatrick Nuclear Power Plant Offsite Dose Calculation Manual (ODCM), Part 1, Section 6.2.3 a listing of new locations for dose calculation and/or environmental monitoring identified by the land use census shall be included in the Annual Radioactive Effluent Release Report.

During the reporting period, no changes in Dose Calculation Receptor Locations and/or the Environmental Monitoring were required based on the results of the land use census.

ATTACHMENT NO. 4

DEVIATIONS FROM THE REQUIRED ENVIRONMENTAL SAMPLING SCHEDULE

In accordance with the James A. FitzPatrick Nuclear Power Plant Offsite Dose Calculation Manual (ODCM), Part 1, Section 6.2.7, the cause for the unavailability of any environmental samples required during the report period shall be included in the Annual Radioactive Effluent Release Report.

The following is a report of samples that were a deviation from the requirements of ODCM Part 1, Table 5.1-1. ODCM Part 1, Section 5.1.1.c.1 allows for deviations from the program due to hazardous conditions, seasonal unavailability, theft, uncooperative residents, or to a malfunction of automatic sampling equipment.

A. ODCM Program Deviations

The following are deviations from the program specified by the ODCM:

- 1. Loss of power to environmental air sample stations R1 and R2 Offsite for 6.9 hrs each during the week of 2/1/11 to 2/8/11. The stations were found operating during normal weekly sample change outs. Sample volumes for the period are acceptable.
- 2. Loss of power to environmental air sample stations R3 and R4 Offsite for 4.5 hours each during the sample period 3/8/11 to 3/15/11. The stations were found operating during sample change outs. Sample volumes for the period are acceptable.
- 3. Loss of power to environmental air sample stations R3 and R4 for 2 hours each during the week of 5/10/11 to 5/17/11. The air sample stations were found operating during normal weekly sample change outs. Sample volumes for the period are acceptable.
- 4. Loss of power to environmental air sample station R5 for 7.4 hours during the week of 6/7/11 to 6/14/11. The station was found operating during normal weekly change outs. Sample volume for the period was acceptable
- 5. Loss of power to environmental air sample station R3 Offsite for 6 days during the week of 7/12/11 to 7/19/11. NMP technician found cabinet blower fan inoperable upon arrival. The cabinet blower fan and air sample pump were replaced.
- 6. Oswego Steam Station (OSS) Inlet Canal Sampler found not working on 7/22/2011. There was no power to the pump. Found that the GFI had been tripped. Building lost power on 7/21/2011 at 0810 hrs. Sampler was out of service for 26 hours.

B. Air Sampling Station Operability Assessment

The ODCM required air sampling program consists of 5 individual sampling locations. The collective operable time period for the air monitoring stations was 43,622 hours out of a possible 43,800 hours. The air sampling availability factor for the report period was 99.59%.

ATTACHMENT NO. 5

ANNUAL SUMMARY OF HOURLY METEOROLOGICAL DATA

The James A. FitzPatrick Nuclear Power Plant Offsite Dose Calculation Manual (ODCM), Part 1, Radiological Controls (REC) Section 6.2 and 6.2.2 states in part: The Annual Radioactive Effluent Release Report submitted prior to May 1 of each year may include an annual summary of meteorological data collected over the previous year. If the meteorological data is not included, the licensee shall retain it on file and provide it to the U.S. Nuclear Regulatory Commission upon request.

In accordance with the aforementioned ODCM requirement, meteorological data is not included in this report. It is retained on file and is available upon request.

ATTACHMENT NO. 6

MAJOR MODIFICATIONS TO RADIOACTIVE LIQUID, GASEOUS AND SOLID WASTE TREATMENT SYSTEMS

In accordance with the James A. FitzPatrick Nuclear Power Plant Offsite Dose Calculation Manual (ODCM), Part 1 Radiological Controls (REC) Section 7.0, Major Modifications to Radioactive Waste Treatment Systems (liquid, gaseous and solid) shall be reported in the Annual Radioactive Effluent Release Report for the period in which the modification is completed and made operational.

There were no major modifications to any liquid, gaseous, or solid radioactive waste treatment systems.

ATTACHMENT NO. 7

ONSITE GROUNDWATER MONITORING

In response to the Nuclear Energy Institute (NEI) Groundwater Protection Initiative, JAF instituted a groundwater monitoring program in 2007. Five monitoring wells were installed during the fourth quarter of 2007. The first samples were collected in November 2007. Starting in March 2010, 16 additional monitoring wells were drilled in the area of the reactor building and nearby SSCs.

All samples collected were analyzed for tritium, and gamma emitting radionuclides. The detection limits and results are reported in the following tables. Analysis results of tritium ranged from non-detectable at less than 408 pCi/l, up to a maximum concentration of 10,660 pCi/l for a MW CST-B sample from 4/6/11. Such levels are well below the voluntary communication reporting level of 20,000 pCi/l as established by the EPA drinking water standard. Although the EPA Standard provides a baseline for comparison, no drinking water sources are affected by this tritium. All of the effected wells are onsite. As such, there is no potential to influence any off-site drinking well. Even if worst case assumptions were made and the water from monitoring well CST-B was consumed as drinking water, the maximum dose would be 1.1 mrem/yr to the whole body. No drinking water pathway exists at the James A. Fitzpatrick site under normal operating conditions due to the direction and distance of the nearest water intake (Oswego, 8.5 miles west of the JAF discharge).

Starting in February 2011, the MW CST-B samples have been analyzed for Sr-90 on a monthly basis. The Sr-90 concentrations detected were in trace amounts. The range was 0.77 to 3.09 pCi/l. The Sr-90 could be from fallout, or from previous plant operations, or possibly a leak. The last two are highly unlikely since no plant-related gamma activity, especially Cs-137, or other hard-to-detects were identified. There was also high uncertainty in the results for Sr-90, since they were very close to the analytical laboratory's detection capability.

In conclusion, the only radionuclide detected in groundwater during the 2011 monitoring effort that is attributable to James A. FitzPatrick operations is tritium, and all concentrations were well below any reporting criteria.

ATTACHMENT NO. 7 (continued)

ONSITE GROUNDWATER MONITORING

A) Gamma Isotopic Monitoring

For 2011, the 21 monitoring wells were sampled and analyzed below the required lower limits of detection in accordance with the Offsite Dose Calculation Manual (ODCM) Part 1, Table 5.1-3. All 21 monitoring wells were analyzed at least quarterly, down to these values. These values are as follows:

Radionuclide	LLD Value (pCi/l)
Manganese-54	15
Cobalt-58	15
Iron-59	30
Cobalt-60	15
Zinc-65	30

Radionuclide	LLD Value (pCi/l)
Zirconium/Niobium-95	15
Iodine-131	15
Cesium-134	15
Cesium-137	18
Barium/Lanthanum-140	15

There were no plant related nuclides detected in the samples.

ATTACHMENT NO. 7 (continued)

ONSITE GROUNDWATER MONITORING

B) Tritium Summary

		# Positive	Minimum	Maximum
	# Samples	Samples in	Positive	Positive
Well Name	in 2011	2011	Concentration	Concentration
MW-5	17	8	396	536
MW-6	17	12	403	594
MW-7	17	1	458	458
MW-8	17	3	395	449
MW-9	17	2	404	431
MW-1A	14	0	0	0
MW-1B	14	0	0	0
MW-2A	14	13	407	667
MW-2B	14	0	0	0
MW-3A	14	12	390	689
MW-3B	14	2	410	518
MW-4A	14	4	392	479
MW-4B	13	0	0	0
MW-10A	14	0	0	0
MW-10B	14	0	0	0
MW-13	14	8	399	575
MW-14	11	2	464	625
MW-15	14	0	0	0
MW-16	14	8	391	603
MW-CSTA	13	13	438	719
MW-CSTB	13	13	1506	10660

Note 1: All results are in pCi/L.

Note 2: A total of 303 samples were analyzed for H-3 in 2011 with 101 positive results.

Note 3: The LLD was 500 pCi/L.

ATTACHMENT NO. 8

GASEOUS EFFLUENTS – CARBON-14

a)	Date:	January 01, 2011 – December 31, 2011
b)	Location:	Elevated Release – Main Stack
c)	Duration:	365 Days
d)	Flowrate:	N/A
e)	Volume Released:	N/A
f)	Nuclides Released:	Carbon-14
g)	Curies Released:	10.53 Ci
h)	Resultant Doses:	See Addendum 1—Assessment of Radiation Doses to the Public January-December 2011 Table 1D
i)	Dose Calculations:	Doses were calculated in accordance with the Offsite Dose Calculation Manual (ODCM) Section 4.4.1

ADDENDUM 1

ASSESSMENT OF RADIATION DOSES TO THE PUBLIC JANUARY - DECEMBER 2011

1. INTRODUCTION

The James A. FitzPatrick Nuclear Power Plant Offsite Dose Calculation Manual (ODCM), Part 1 Radiological Controls, requires an assessment of the radiation doses to the public due to radioactive liquid and gaseous effluents. This assessment of doses to the public is based on accepted methodologies found in the Offsite Dose Calculation Manual (ODCM).

2. DOSE LIMITS

A. DOSE FROM LIQUID EFFLUENTS (ODCM, Part 1, REC 2.3)

Applicability

Applies to doses from radioactive material in liquid effluents.

Objective

To ensure that the dose limitations of 10 CFR 50, Appendix I, are met.

Specifications

The dose to a member of the public from radioactive materials released from the plant in liquid effluents to unrestricted areas shall be limited as follows:

- 1. During any calendar quarter, limited to less than or equal to 1.5 mrem to the whole body and to less than or equal to 5 mrem to any organ.
- 2. During any calendar year, limited to less than or equal to 3 mrem to the whole body and to less than or equal to 10 mrem to any organ.

B. GASEOUS DOSE RATES (ODCM, Part 1, REC 3.2)

Applicability

Applies to the radiation dose from radioactive material in gaseous effluents.

Objective

To ensure that the dose rates at or beyond the site boundary from gaseous effluents do not exceed the annual dose limits of 10 CFR 20, for unrestricted areas.

ADDENDUM 1 (continued)

Specifications

The dose rate at or beyond the site boundary due to radioactive materials released from the plant in gaseous effluents shall be limited as follows:

- 1. Less than or equal to 500 mrem/year to the whole body and less than or equal to 3000 mrem/year to the skin from noble gases; and,
- 2. Less than or equal to 1500 mrem/year to any organ from Iodine-131, Iodine-133, Tritium and for radioactive materials in particulate form with half-lives greater than 8 days (inhalation pathway only).
- C. AIR DOSE, NOBLE GASES (ODCM, Part 1, REC 3.3)

Applicability

Applies to the air dose due to noble gases in gaseous effluents.

Objective

To ensure that the noble gas dose limitations of 10 CFR 50, Appendix I, are met.

Specifications

The air dose to areas at or beyond the site boundary from noble gases released from the plant in gaseous effluents shall be limited:

- 1. During any calendar quarter, to less than or equal to 5 mrad from gamma radiation, and less than or equal to 10 mrad from beta radiation; and,
- 2. During any calendar year, to less than or equal to 10 mrad from gamma radiation and less than or equal to 20 mrad from beta radiation.

ADDENDUM 1 (continued)

D. DOSE DUE TO IODINE-131, IODINE-133, TRITIUM AND RADIONUCLIDES IN PARTICULATE FORM (ODCM, Part 1, REC 3.4)

Applicability

Applies to the cumulative dose from Iodine-131, Iodine-133, Tritium, and radionuclides in particulate form with half-lives greater than 8 days in gaseous effluents.

Objective

To ensure that the dose limitations of 10 CFR 50, Appendix I, are met.

Specifications

The dose to a member of the public at or beyond the site boundary from Iodine-131, Iodine-133, Tritium, and radionuclides in particulate form with half-lives greater than 8 days released from the plant in gaseous effluents shall be limited:

- 1. During any calendar quarter to less than or equal to 7.5 mrem to any organ; and,
- 2. During any calendar year to less than or equal to 15 mrem to any organ.

E. TOTAL DOSE FROM URANIUM FUEL CYCLE (ODCM, Part 1, REC 4.0)

Applicability

Applies to radiation dose from releases of radioactivity and radiation from uranium fuel cycle sources.

Objective

To ensure that the requirements of 40 CFR 190 are met.

Specifications

The dose or dose commitment to any member of the public, due to releases of radioactivity and radiation, from uranium fuel cycle sources shall be limited as follows:

- 1. Less than or equal to 25 mrem/year to the whole body; and,
- 2. Less than or equal to 25 mrem/year to any organ except the thyroid which shall be limited to less than or equal to 75 mrem/year.

ADDENDUM 1 (continued)

3. DOSE ASSESSMENT

A. <u>METHODOLOGY</u>

The assessment of radiation doses to the public due to radioactive liquid and gaseous effluents is performed in accordance with the ODCM. The ODCM is based on methodologies and models suggested by the "Guidance Manual For Preparation of Radiological Effluent Technical Specifications for Nuclear Power Plants" (NUREG-0133) and "Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents for the purpose of Evaluating Compliance with 10CFR50, Appendix I" (Regulatory Guide 1.109).

B. ASSUMPTIONS

Dose calculations are performed using formulas and constants defined in the ODCM. Specific radioactive release activities used in the dose calculations are listed in the Annual Radioactive Effluent Release Report (1.21 Report) for the period of January 1, 2011 to December 31, 2011. Historical meteorological data was used to generate tables of average dispersion factors. Locations of interest were identified from the 2011 land use census. Dispersion factors and locations of interest used in performing the dose calculations are listed in Table 2.

C. ASSESSMENT RESULTS SUMMARY

The calculated doses to the public due to radioactive effluents are listed in Table 1. The calculated doses are small fractions of their respective dose limits.

4. 40 CFR 190 DOSE ASSESSMENT

A. <u>METHODOLOGY</u>

Evaluation to demonstrate compliance with the 40 CFR 190 dose limits must be performed when the doses calculated for 10 CFR 50 compliance exceed twice their respective limits. When additional dose assessment is required to demonstrate compliance with 40 CFR 190 it is performed in accordance with the ODCM.

B. <u>RESULTS SUMMARY</u>

The cumulative dose contribution from liquid and gaseous effluents for this report period were calculated and are listed in Table 1. The cumulative dose contribution from direct radiation from the reactor unit and from radwaste storage tanks is measured by environmental thermoluminescent dosimeters for the report period. This data is contained in the Annual Environmental Operating Report. The calculated doses from liquid and gaseous effluents are less than twice their respective 10 CFR 50 limits; therefore, additional calculations are not necessary to demonstrate compliance with 40 CFR 190 dose limits (ODCM, Part 1, REC 4.1.1.c)

ADDENDUM 1 (continued)

ANNUAL DOSE ASSESSMENT 2011					
A. LIQUIDS					
<u>QUARTER</u>	<u>1</u>	<u>2</u>	<u>3</u>	<u>4</u>	ANNUAL
	(a)	(a)	(a)	(a)	(a)
Organ (mrem)	2.31E-08	5.25E-06	5.09E-06	4.93E-06	1.53E-05
% of Limit	4.63E-07	1.05E-04	1.02E-04	9.86E-05	1.53E-04
	(b)	(b)	(b)	(b)	(b)
Whole Body (mrem)	2.31E-08	5.25E-06	5.09E-06	4.93E-06	1.53E-05
% of Limit	1.54E-06	3.50E-04	3.39E-04	3.29E-04	5.10E-04

TABLE 1 ANNUAL DOSE ASSESSMENT 2011

(a) Dose to the Child Liver primarily by the potable water pathway.

(b) Dose to the Child Whole Body primarily by the potable water pathway.

ADDENDUM 1 (continued)

TABLE 1 (cont) ANNUAL DOSE ASSESSMENT 2011

B. NOBLE GASES

<u>QUARTER</u>	<u>1</u>	<u>2</u>	<u>3</u>	<u>4</u>	<u>ANNUAL</u>
Total Body (mrem/yr)	2.01E-02	7.28E-02	7.38E-03	1.04E-02	7.28E-02
% of Limit	4.02E-03	1.46E-02	1.48E-03	2.09E-03	1.46E-02
Skin (mrem/yr)	2.59E-02	9.46E-02	1.31E-02	1.90E-02	9.46E-02
% of Limit	8.62E-04	3.15E-03	4.36E-04	6.32E-04	3.15E-03
Gamma (mrad)	1.89E-04	4.93E-04	1.73E-04	1.67E-04	1.02E-03
% of Limit	3.79E-03	9.86E-03	3.46E-03	3.33E-03	1.02E-02
Beta (mrad)	2.43E-05	7.92E-05	1.89E-05	2.61E-05	1.49E-04
% of Limit	2.43E-04	7.92E-04	1.89E-04	2.61E-04	7.43E-04

C. IODINES AND PARTICULATES

<u>QUARTER</u>	<u>1</u>	<u>2</u>	<u>3</u>	<u>4</u>	ANNUAL
	(a)	(a)	(a)	(a)	(a)
Organ (mrem)	2.61E-04	6.93E-04	7.38E-04	1.19E-03	2.88E-03
% of Limit	3.48E-03	9.24E-03	9.84E-03	1.59E-02	1.92E-02
	(a)	(a)	(a)	(a)	(a)
Organ Dose Rate	2.40E-04	2.72E-04	2.41E-04	2.67E-04	2.72E-04
(mrem/yr) % of Limit	1.60E-05	1.81E-05	1.61E-05	1.78E-05	1.81E-05

(a) Dose to the Child Thyroid primarily by the vegetation pathway.

ADDENDUM 1 (continued)

D. CARBON 14					
QUARTER	<u>1</u>	<u>2</u>	<u>3</u>	<u>4</u>	ANNUAL
	(a)	(a)	(a)	(a)	(a)
Organ (mrem)	8.27E-03	8.30E-03	8.59E-03	8.43E-03	3.36E-02
% of Limit	1.10E-01	1.11E-01	1.14E-01	1.12E-01	2.24E-01
					(a)
Organ Dose Rate (mrem/yr)					3.36E-02
% of Limit					2.24E-03

TABLE 1 (cont) ANNUAL DOSE ASSESSMENT 2011

(a) Dose to the Child Bone primarily by the vegetation pathway.

ADDENDUM 1 (continued)

TABLE 2

METEOROLOGICAL DATA AND LOCATIONS OF INTEREST

	RECEPTOR	GEOGRAPHIC LOCATION		ATMOS DISPERSION	PHERIC** N FACTOR
A.	IODINE & PARTICULATES	DISTANCE/ DIRECTION	RELEAS POINT	E X/Q (sec/m ³)	D/Q (1/m ²)
1.	Garden	0.90 mi @ 83°E	ST	2.83E-08*	1.75E-09
	Grazing Season	0.90 mi @ 83°E	RX	2.02E-07*	5.01E-09
	Cary	0.90 mi @ 83°E	TB	1.83E-07*	4.80E-09
	Location No. 78	0.90 mi @ 83°E	RF	2.02E-07*	5.01E-09
		0.90 mi @ 83°E	RW	3.21E-07*	5.76E-09
2.	Meat	1.18 mi @ 127°SE	ST	1.72E-08*	6.80E-10
	Grazing Season	1.18 mi @ 127°SE	RX	5.36E-08*	1.30E-09
	Parkhurst	1.18 mi @ 127°SE	TB	5.14E-08*	1.27E-09
	Location No. 26	1.18 mi @ 127°SE	RF	5.36E-08*	1.30E-09
		1.18 mi @ 127°SE	RW	9.12E-08*	1.46E-09
3.	Cow	2.50 mi @ 139°SE	ST	1.67E-08*	2.65E-10
	Grazing Season	2.50 mi @ 139°SE	RX	2.76E-08*	4.14E-10
	France	2.50 mi @ 139°SE	TB	2.71E-08*	4.07E-10
	Location No. 10	2.50 mi @ 139°SE	RF	2.76E-08*	4.14E-10
		2.50 mi @ 139°SE	RW	4.15E-08*	4.36E-10
4.	Goat (D/Q)	3.62 mi @ 113°ESE	ST		2.28E-10
	Grazing Season	3.62 mi @ 113°ESE	RX		3.40E-10
	Showers	3.62 mi @ 113°ESE	TB		3.33E-10
	Location No. 71	3.62 mi @ 113°ESE	RF		3.40E-10
		3.62 mi @ 113°ESE	RW		3.49E-10
5.	Goat (X/Q)	2.64 mi @ 152°SSE	ST	1.94E-08*	
	Grazing Season	2.64 mi @ 152°SSE	RX	2.58E-08*	
	Nickolas	2.64 mi @ 152°SSE	TB	2.57E-08*	
	Location No. 61	2.64 mi @ 152°SSE	RF	2.58E-08*	
		2.64 mi @ 152°SSE	RW	3.59E-08*	

ADDENDUM 1 (continued)

TABLE 2 METEOROLOGICAL DATA AND LOCATIONS OF INTEREST

RECEPTOR		GEOGRAPHIC LOCATION		ATMOSPHERIC** DISPERSION FACTOR		
A.	IODINE & PARTICULATES	DISTANCE/ DIRECTION	RELEASE POINT	E X/Q (sec/m3)	D/Q (1/m2)	
6.	Resident Annual Average					
	a. Inhalation ⁽²⁾	1.55 mi @ 90°E ⁽¹⁾	ST	2.99E-08		
	Cary	0.90 mi @ 83°E	RX	2.07E-07		
	Location No. 78	0.90 mi @ 83°E	ТВ	1.88E-07		
		0.90 mi @ 83°E	RF	2.07E-07		
		0.90 mi @ 83°E	RW	3.06E-07		
	b. Deposition ⁽³⁾	0.71 mi @ 118°ESE	ST		1.60E-09	
	Whaley	0.71 mi @ 118°ESE	RX		5.52E-09	
	Location No. 199	0.71 mi @ 118°ESE	TB		5.30E-09	
		0.71 mi @ 118°ESE	RF		5.52E-09	
		0.71 mi @ 118°ESE	RW		6.28E-09	
B.	NOBLE GASES					
1.	Air Dose	1.55 mi @ 90°E ⁽¹⁾	ST	2.99E-08		
	Annual Average	0.60 mi @ 90°E	ST(fc)	1.16E-07		
	Site Boundary	0.60 mi @ 90°E	RX	3.58E-07		
	2	0.60 mi @ 90°E	ТВ	3.19E-07		
		0.60 mi @ 90°E	RF	3.58E-07		
		0.60 mi @ 90°E	RW	5.39E-07		
	Total Body	0.60 mi @ 90°E	ST(fc)	1.16E-07		
	Annual Average	0.60 mi @ 90°E	RX	3.58E-07		
	Site Boundary	0.60 mi @ 90°E	TB	3.19E-07		
		0.60 mi @ 90°E	RF	3.58E-07		
		0.60 mi @ 90°E	RW	5.39E-07		

ADDENDUM 1 (continued)

TABLE 2METEOROLOGICAL DATA AND LOCATIONS OF INTEREST

RECEPTOR	GEOGRAPHIC LOCATION	D	ATMOSPHERIC** DISPERSION FACTOR		
B. NOBLE GASES (continued)	DISTANCE/ DIRECTION	RELEASE POINT	X/Q (sec/m3)	D/Q (1/m2)	
3. Skin	1.55 mi @ 90°E	ST	2.99E-08		
Annual Average Site Boundary	0.60 mi @ 90°E 0.60 mi @ 90°E 0.60 mi @ 90°E 0.60 mi @ 90°E 0.60 mi @ 90°E	ST(fc) RX TB RF RW	1.16E-07 3.58E-07 3.19E-07 3.58E-07 5.39E-07	 	

* Tritium Dose Calculation

** Based on ODCM X/Q, D/Q Values Rev. 11

- (1) Highest Sector Average X/Q in a populated area, not an identified residence.
- (2) Inhalation uses Annual Average X/Q values. All other receptors use grazing season meteorology.
- (3) Deposition uses Annual Average D/Q values. All other receptors use grazing season meteorology.
- ST = Main Stack
- RX = Reactor Building Vent
- TB = Turbine Building Vent
- RF = Refuel Floor Vent
- RW = Radwaste Building Vent
- fc = Finite Cloud