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NINE MILE POINT
NUCLEAR STATION

P.O. Box 63
Lycoming, NY 13093

May 1, 2011

U.S. Nuclear Regulatory Commission
Washington, DC 20555-0001

ATTENTION: Document Control Desk

SUBJECT: Nine Mile Point Nuclear Station
Unit No. 2; Docket No. 50-410

Radioactive Effluent Release Report, January – December 2010

In accordance with 10 CFR 50.36a and the Nine Mile Point Unit 2 (NMP2) Technical Specifications, enclosed is the Radioactive Effluent Release Report for the period January through December 2010.

Included in this report is a summary of gaseous and liquid effluents and solid waste released from the station during the reporting period (Attachments 1-6), a summary of revisions to the Offsite Dose Calculation Manual (ODCM) and the Radwaste Process Control Program during the reporting period (Attachments 7 and 8), and an explanation as to the cause and corrective actions regarding any station liquid and/or gaseous effluent monitoring instrumentation that was non-functional for greater than 30 days (Attachment 9). Attachments 10 and 11 provide a summary and assessment of radiation doses to members of the public within and outside the site boundary, respectively, from liquid and gaseous effluents, as well as direct radiation, in accordance with 40 CFR 190. Attachment 12 provides a summary of the tritium results from the groundwater protection program.

The format used for the effluent data is outlined in Appendix B of Regulatory Guide 1.21, Revision 1. Dose assessments were made in accordance with the NMP2 ODCM. During the reporting period from January through December 2010, NMP2 did not exceed any 10 CFR 20, 10 CFR 50, Technical Specification, or ODCM limits for gaseous or liquid effluents.

Should you have questions regarding the information in this submittal, please contact me at (315) 349-5219.

Very truly yours,

J. Dosa.

John J. Dosa
Director Licensing

JJD/KES

A009
IE48
NRR

Enclosure: Nine Mile Point Nuclear Station, Unit 2, Radioactive Effluent Release Report, January – December 2010

cc: Regional Administrator, Region I, NRC
Project Manager, NRC
Resident Inspector, NRC
J. Furia, NRC

ENCLOSURE

NINE MILE POINT NUCLEAR STATION, UNIT 2

RADIOACTIVE EFFLUENT RELEASE REPORT

January – December 2010

Nine Mile Point Nuclear Station, LLC
May 1, 2011

NINE MILE POINT NUCLEAR STATION - UNIT 2

RADIOACTIVE EFFLUENT RELEASE REPORT

January – December 2010

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NINE MILE POINT
NUCLEAR STATION

NINE MILE POINT NUCLEAR STATION - UNIT 2

RADIOACTIVE EFFLUENT RELEASE REPORT

JANUARY – DECEMBER 2010

SUPPLEMENTAL INFORMATION

Facility: Nine Mile Point Unit 2 **Licensee: Nine Mile Point Nuclear Station, LLC**

1. TECHNICAL SPECIFICATION/ODCM LIMITS

A) FISSION AND ACTIVATION GASES

1. The dose rate limit of noble gases released in gaseous effluents from the site to areas at or beyond the site boundary shall be less than or equal to 500 mrem/year to the whole body and less than or equal to 3000 mrem/year to the skin.

2. The air dose from noble gases released in gaseous effluents from Nine Mile Point Unit 2 to areas at or beyond the site boundary shall be limited during any calendar quarter to less than or equal to 5 mrad for gamma radiation and less than or equal to 10 mrad for beta radiation, and during any calendar year to less than or equal to 10 mrad for gamma radiation and less than or equal to 20 mrad for beta radiation.

B&C) TRITIUM, IODINES AND PARTICULATES, HALF LIVES > 8 DAYS

1. The dose rate limit of Iodine-131, Iodine-133, Tritium and all radionuclides in particulate form with half-lives greater than eight days, released in gaseous effluents from the site to areas at or beyond the site boundary shall be less than or equal to 1500 mrem/year to any organ.

2. The dose to a member of the public from Iodine-131, Iodine-133, Tritium and all radionuclides in particulate form with half-lives greater than eight days in gaseous effluents released from Nine Mile Point Unit 2 to areas at or beyond the site boundary shall be limited during any calendar quarter to less than or equal to 7.5 mrem to any organ, and during any calendar year to less than or equal to 15 mrem to any organ.

D) LIQUID EFFLUENTS

1. Improved Technical Specifications (ITS) limit the concentration of radioactive material released in the liquid effluents to unrestricted areas to ten times the concentrations specified in 10CFR20.1001-20.2402, Appendix B, Table 2, Column 2 for radionuclides other than dissolved or entrained noble gases. For dissolved or entrained noble gases, the concentration shall be limited to 2E-04 microcuries/ml total activity.

2. The dose or dose commitment to a member of the public from radioactive materials in liquid effluents released from Nine Mile Point Unit 2 to unrestricted areas shall be limited during any calendar quarter to less than or equal to 1.5 mrem to the whole body and to less than or equal to 5 mrem to any organ, and during any calendar year to less than or equal to 3 mrem to the whole body and to less than or equal to 10 mrem to any organ.

2. MEASUREMENTS AND APPROXIMATIONS OF TOTAL RADIOACTIVITY

Described below are the methods used to measure or approximate the total radioactivity and radionuclide composition in effluents.

A) FISSION AND ACTIVATION GASES

Noble gas effluent activity is determined by an on-line scintillation detector (calibrated against gamma isotopic analysis of a 4.0L Marinelli grab sample) of an isokinetic sample stream.

B) IODINES

Iodine effluent activity is determined by gamma spectroscopic analysis (at least weekly) of charcoal cartridges sampled from an isokinetic sample stream.

C) PARTICULATES

Activity released from the main stack and the combined Radwaste/Reactor Building vent is determined by gamma spectroscopic analysis (at least weekly) of particulate filters sampled from an isokinetic sample stream and composite analysis of the filters for non-gamma emitters.

D) TRITIUM

Tritium effluent activity is measured by liquid scintillation or gas proportional counting of monthly samples taken with an air sparging/water trap apparatus.

E) LIQUID EFFLUENTS

Isotopic contents of liquid effluents are determined by isotopic analysis of a representative sample of each batch and composite analysis of non-gamma emitters.

F) SOLID EFFLUENTS

Isotopic contents of waste shipments are determined by gamma spectroscopy analyses of a representative sample of each batch. Scaling factors established from primary composite sample analyses conducted off-site are applied, where appropriate, to find estimated concentration of non-gamma emitters. For low activity trash shipments, curie content is estimated by dose rate measurement and application of appropriate scaling factors.

G) C-14

The production of C-14 and the effluent dose consequences are estimates based on EPRI methodology provided in EPRI Report 1021106, *Estimation of Carbon-14 in Nuclear Power Plant Gaseous Effluents*, December 2010 and NUREG-0016, *Calculation of Releases of Radioactive Materials in Gaseous and Liquid Effluents for Boiling Water Reactors (BWR-GALE Code)*.

ATTACHMENT 1
SUMMARY DATA

Unit 1 _____ Unit 2 X Reporting Period January - December 2010

Liquid Effluents:

ODCM Required Maximum Effluent Concentration (MEC) = 10 x 10CFR20.1001 - 20.2402, Appendix B, Table 2, Column 2

Average MEC - µCi/ml (Qtr. 1) =	NO RELEASES	Average MEC - µCi/ml (Qtr. 3) =	9.83E-03
Average MEC - µCi/ml (Qtr. 2) =	9.96E-03	Average MEC - µCi/ml (Qtr. 4) =	NO RELEASES

Average Energy (Fission and Activation gases - MEV):

Qtr. 1:	$\bar{E}\gamma$	=	7.84E-01	$\bar{E}\beta$	=	2.87E-01
Qtr. 2:	$\bar{E}\gamma$	=	1.05E+00	$\bar{E}\beta$	=	3.62E-01
Qtr. 3:	$\bar{E}\gamma$	=	1.04E+00	$\bar{E}\beta$	=	3.10E-01
Qtr. 4:	$\bar{E}\gamma$	=	8.97E-01	$\bar{E}\beta$	=	2.77E-01

Liquid:

Number of Batch Releases	7
Total Time Period for Batch Releases (hrs)	22.5
Maximum Time Period for a Batch Release (hrs)	3.32
Average Time Period for a Batch Release (hrs)	3.21
Minimum Time Period for a Batch Release	3.05

Total volume of water used to dilute the liquid during the release period (L)	1st	2nd	3rd	4th
	N/A	2.26E+07	8.97E+07	N/A

Total volume of water available to dilute the liquid effluent during the report period (L)	1st	2nd	3rd	4th
	1.14E+10	1.13E+10	1.31E+10	1.26E+10

Gaseous (Emergency Condenser Vent) "Not applicable for Unit 2"

Number of Batch Releases	N/A
Total Time Period for Batch Releases (hrs)	N/A
Maximum Time Period for a Batch Release (hrs)	N/A
Average Time Period for a Batch Release (hrs)	N/A
Minimum Time Period for a Batch Release	N/A

Gaseous (Primary Containment Purge)

Number of Batch Releases	12
Total Time Period for Batch Releases (hrs)	364.2
Maximum Time Period for a Batch Release (hrs)	100.2
Average Time Period for a Batch Release (hrs)	30.3
Minimum Time Period for a Batch Release (hrs)	5.4

ATTACHMENT 1
SUMMARY DATA

Unit 1 _____ Unit 2 X Reporting Period January - December 2010

Abnormal Releases:

A. Liquids:

Number of Releases	0	
Total Activity Released	N/A	Ci

B. Gaseous:

Number of Releases	0	
Total Activity Released	N/A	Ci

ATTACHMENT 2

Unit 1 _____ Unit 2 X Reporting Period January - December 2010

GASEOUS EFFLUENTS - SUMMATION OF ALL RELEASES, ELEVATED AND GROUND LEVEL

		1st Quarter	2nd Quarter	3rd Quarter	4th Quarter	Est. Total Error, %
A. Fission & Activation Gases						
1. Total Release	Ci	1.42E+02	1.06E+02	1.00E+02	5.83E+01	5.00E+01
2. Average Release Rate	µCi/sec	1.83E+01	1.35E+01	1.26E+01	7.33E+00	
B. Iodines						
1. Total Iodine - 131	Ci	2.89E-03	1.17E-03	4.56E-04	4.61E-04	3.00E+01
2. Average Release Rate for Period	µCi/sec	3.98E-04	1.38E-04	5.81E-05	5.84E-05	
C. Particulates						
1. Particulates with Half-lives>8days	Ci	4.92E-03	1.51E-03	7.39E-04	2.25E-04	3.00E+01
2. Average Release Rate for Period	µCi/sec	6.78E-04	1.78E-04	9.41E-05	2.86E-05	
3. Gross Alpha Radioactivity	Ci	0.00E+00	0.00E+00	0.00E+00	0.00E+00	2.50E+01
D. Tritium						
1. Total Release	Ci	2.77E+01	2.62E+01	1.90E+01	1.50E+01	5.00E+01
2. Average Release Rate for Period	µCi/sec	3.82E+00	3.09E+00	2.41E+00	1.90E+00	
E. Percent of Tech. Spec. Limits Fission and Activation Gases						
Percent of Quarterly Gamma Air Dose Limit (5 mR)	%	2.56E-01	2.60E-01	2.42E-01	1.20E-01	
Percent of Quarterly Beta Air Dose Limit (10 mrad)	%	4.93E-03	4.27E-03	3.45E-03	2.54E-03	
Percent of Annual Gamma Air Dose Limit to Date (10 mR)	%	1.28E-01	2.58E-01	3.79E-01	4.41E-01	
Percent of Annual Beta Air Dose Limit to Date (20 mrad)	%	2.47E-03	4.60E-03	6.30E-03	7.60E-03	
Percent of Whole Body Dose Rate Limit (500 mrem/yr)	%	1.00E-02	1.01E-02	9.27E-03	4.72E-03	
Percent of Skin Dose Rate Limit (3000 mrem/yr)	%	1.97E-03	1.98E-03	1.81E-03	9.21E-04	
Tritium, Iodines, and Particulates (with half-lives greater than 8 days)						
Percent of Quarterly Dose Limit (7.5 mrem)	%	8.05E-01	2.98E-01	1.25E-01	1.28E-01	
Percent of Annual Dose Limit to Date (15 mrem)	%	4.05E-01	5.55E-01	6.18E-01	6.83E-01	
Percent of Organ Dose Limit (1500 mrem/yr	%	1.63E-02	5.98E-03	2.48E-03	2.60E-03	

ATTACHMENT 3

Unit 1 _____ Unit 2 X Reporting Period January - December 2010

GASEOUS EFFLUENTS - ELEVATED RELEASE

Continuous Mode (2)

Nuclides Released		1st Quarter	2nd Quarter	3rd Quarter	4th Quarter
Fission Gases (1)					
Argon-41	Ci	**	**	**	**
Krypton-85	Ci	4.87E-01	**	**	**
Krypton-85m	Ci	3.05E+01	2.81E+01	2.71E+01	1.68E+01
Krypton-87	Ci	2.54E+00	8.33E-01	1.87E+00	3.03E-01
Krypton-88	Ci	4.92E+01	5.29E+01	4.98E+01	2.40E+01
Xenon-127	Ci	**	**	**	**
Xenon-131m	Ci	**	**	**	**
Xenon-133	Ci	3.70E+01	1.56E+01	2.13E+01	1.16E+01
Xenon-133m	Ci	**	**	**	**
Xenon-135	Ci	1.63E+01	2.92E+00	1.56E-02	1.20E+00
Xenon-135m	Ci	5.28E+00	7.28E-01	**	3.62E+00
Xenon-137	Ci	**	4.07E+00	**	**
Xenon-138	Ci	3.56E-01	7.34E-01	1.37E-01	**
Iodines (1)					
Iodine-131	Ci	2.44E-03	1.10E-03	3.63E-04	3.60E-04
Iodine-133	Ci	2.43E-02	6.51E-03	4.82E-03	5.05E-03
Iodine-135	Ci	**	**	**	**
Particulates (1)					
Chromium-51	Ci	**	**	**	**
Manganese-54	Ci	9.30E-05	**	**	1.98E-05
Iron-55	Ci	3.71E-05	1.22E-04	1.51E-04	**
Iron-59	Ci	4.62E-05	**	**	**
Cobalt-58	Ci	**	**	**	**
Cobalt-60	Ci	**	9.72E-06	**	2.87E-05
Neodymium-147	Ci	1.77E-06	5.82E-06	2.61E-06	**
Zinc-65	Ci	**	**	**	**
Strontium-89	Ci	6.04E-05	5.43E-05	1.56E-05	2.73E-05
Strontium-90	Ci	**	**	**	**
Niobium-95	Ci	**	**	**	**
Zirconium-95	Ci	**	**	**	**
Molybdenum-99	Ci	7.12E-07	**	**	**
Cesium-134	Ci	**	**	**	**
Cesium-136	Ci	**	**	**	**
Cesium-137	Ci	**	**	**	**
Barium-140	Ci	7.98E-06	6.37E-06	**	**
Lanthanum-140	Ci	**	**	**	**
Cerium-141	Ci	**	**	**	**
Cerium-144	Ci	**	**	**	**
Tritium (1)	Ci	2.12E+01	1.83E+01	1.44E+01	1.14E+01

(1) Concentrations less than the lower limit of detection of the counting system used are indicated with a double asterisk. A lower limit of detection of 1.00E-04 µCi/ml for required noble gases, 1.00E-11 µCi/ml for required particulates and gross alpha, 1.00E-12 µCi/ml for required Iodines, 1.00E-11 µCi/ml for Sr-89/90 and 1.00E-06 µCi/ml for Tritium, as required by the ODCM, has been verified.

(2) Contributions from purges are included. There were no other batch releases during the reporting period.

ATTACHMENT 4

Unit 1 _____ Unit 2 X Reporting Period January - December 2010

GASEOUS EFFLUENTS - GROUND LEVEL RELEASES

Continuous Mode (2)

Nuclides Released		1st Quarter	2nd Quarter	3rd Quarter	4th Quarter
Fission Gases (1)					
Argon-41	Ci	**	**	**	**
Krypton-85	Ci	**	**	**	**
Krypton-85m	Ci	**	**	**	**
Krypton-87	Ci	**	**	**	**
Krypton-88	Ci	**	**	**	8.81E-02
Xenon-127	Ci	**	**	**	**
Xenon-131m	Ci	**	**	**	**
Xenon-133	Ci	**	6.05E-03	**	6.72E-01
Xenon-133m	Ci	**	**	**	**
Xenon-135	Ci	2.62E-01	9.51E-03	**	9.86E-02
Xenon-135m	Ci	2.40E-02	**	**	**
Xenon-137	Ci	**	**	**	**
Xenon-138	Ci	**	**	**	**
Iodines (1)					
Iodine-131	Ci	4.52E-04	6.82E-05	9.35E-05	1.01E-04
Iodine-133	Ci	1.31E-02	3.50E-04	1.44E-03	1.74E-03
Iodine-135	Ci	**	**	**	**
Particulates (1)					
Chromium-51	Ci	1.52E-04	5.32E-05	**	**
Manganese-54	Ci	5.08E-04	6.40E-05	**	4.06E-05
Iron-55	Ci	1.43E-03	9.23E-04	5.34E-04	3.82E-05
Iron-59	Ci	1.45E-05	1.51E-05	**	**
Cobalt-58	Ci	1.74E-04	4.80E-06	**	**
Cobalt-60	Ci	1.50E-03	2.04E-04	3.62E-05	6.52E-05
Neodymium-147	Ci	**	5.32E-06	**	5.15E-06
Zinc-65	Ci	8.16E-04	3.99E-05	**	**
Strontium-89	Ci	**	**	**	**
Strontium-90	Ci	**	**	**	**
Niobium-95	Ci	**	**	**	**
Zirconium-95	Ci	**	**	**	**
Molybdenum-99	Ci	5.11E-04	**	**	**
Cesium-134	Ci	**	**	**	**
Cesium-136	Ci	**	**	**	**
Cesium-137	Ci	**	**	**	**
Barium-140	Ci	8.03E-05	**	**	**
Lanthanum-140	Ci	**	**	**	**
Cerium-141	Ci	**	**	**	**
Cerium-144	Ci	**	**	**	**
Tritium (1)	Ci	6.55E+00	7.91E+00	4.51E+00	3.59E+00

(1) Concentrations less than the lower limit of detection of the counting system used are indicated with a double asterisk. A lower limit of detection of 1.00E-04 µCi/ml for required noble gases, 1.00E-11 µCi/ml for required particulates and gross alpha, 1.00E-12 µCi/ml for required iodines, 1.00E-11 µCi/ml for Sr-89/90 and 1.00E-06 µCi/ml for Tritium, as required by the ODCM, has been verified.

(2) There were no batch releases from this path during the reporting period.

ATTACHMENT 5

Unit 1 _____ Unit 2 X Reporting Period January - December 2010

LIQUID EFFLUENTS - SUMMATION OF ALL RELEASES (1)

		1st Quarter	2nd Quarter	3rd Quarter	4th Quarter	Est. Total Error, %
A. Fission & Activation Products						
1. Total Release (not including Tritium, gases, alpha)	Ci	No Releases	2.15E-05	2.54E-04	No Releases	5.00E+01
2. Average diluted concentration during reporting period	µCi/ml	No Releases	1.91E-12	1.94E-11	No Releases	
B. Tritium						
1.Total release	Ci	No Releases	1.72E+00	3.10E+00	No Releases	5.00E+01
2. Average diluted concentration during the reporting period	µCi/ml	No Releases	1.53E-07	2.36E-07	No Releases	
C. Dissolved and Entrained Gases						
1. Total release	Ci	No Releases	**	5.16E-04	No Releases	5.00E+01
2. Average diluted concentration during the reporting period	µCi/ml	No Releases	**	3.94E-11	No Releases	
D. Gross Alpha Radioactivity						
1. Total release	Ci	No Releases	**	**	No Releases	5.00E+01
E. Volumes						
1. Prior to Dilution	Liters	No Releases	1.78E+05	4.42E+05	No Releases	5.00E+01
2. Volume of dilution water used during release period	Liters	No Releases	2.26E+07	8.97E+07	No Releases	5.00E+01
3. Volume of dilution water available during reporting period	Liters	1.14E+10	1.13E+10	1.31E+10	1.26E+10	5.00E+01
F. Percent of Tech. Spec. Limits						
Percent of Quarterly Whole Body Dose Limit (1.5 mrem)	%	0.00E+00	2.14E-03	2.76E-03	0.00E+00	
Percent of Annual Whole Body Dose Limit to Date (3 mrem)	%	0.00E+00	1.07E-03	2.44E-03	2.44E-03	
Percent of Quarterly Organ Dose Limit (5 mrem)	%	0.00E+00	7.24E-04	1.94E-03	0.00E+00	
Percent of Annual Organ Dose Limit to Date (10 mrem)	%	0.00E+00	3.62E-04	1.33E-03	1.33E-03	
Percent of 10CFR20 Concentration Limit (2), (3)	%	0.00E+00	1.54E-03	2.41E-03	0.00E+00	
Percent of Dissolved or Entrained Noble Gas Limit (2.00E-04 µCi/ml)	%	0.00E+00	0.00E+00	1.97E-05	0.00E+00	

(1) Concentrations less than the lower limit of detection of the counting system used are indicated with a double asterisk. A lower limit of detection of 5.00E-07 µCi/ml for required gamma emitting nuclides, 1.00E-05 µCi/ml for required dissolved and entrained noble gases and tritium, 5.00E-08 µCi/ml for Sr-89/90, 1.00E-06 µCi/ml for I-131 and Fe-55, and 1.00E-07 µCi/ml for gross alpha radioactivity, as required by the Off-Site Dose Calculation Manual (ODCM), has been verified.

(2) The percent of 10CFR20 concentration limit is based on the average concentration during the quarter.

(3) Improved Technical Specifications limit the concentration of radioactive material released in the liquid effluents to unrestricted areas to ten times the concentrations specified in 10CFR20.1001 - 20.2402, Appendix B, Table 2, Column 2. Maximum Effluent Concentrations (MEC) numerically equal to ten times the 10CFR20.1001 - 20.2402 concentrations were adopted to evaluate liquid effluents.

ATTACHMENT 5

Unit 1 _____ Unit 2 X Reporting Period January - December 2010

LIQUID EFFLUENTS RELEASED

Nuclides Released		Batch Mode (1),(2) 1st Quarter	2nd Quarter	3rd Quarter	4th Quarter
Nuclides Released					
Strontium-89	Ci	No Releases	**	**	No Releases
Strontium-90	Ci	No Releases	**	**	No Releases
Cesium-134	Ci	No Releases	**	**	No Releases
Cesium-137	Ci	No Releases	**	**	No Releases
Iodine-131	Ci	No Releases	**	**	No Releases
Cobalt-58	Ci	No Releases	**	**	No Releases
Cobalt-60	Ci	No Releases	2.15E-05	1.49E-04	No Releases
Iron-59	Ci	No Releases	**	**	No Releases
Zinc-65	Ci	No Releases	**	**	No Releases
Manganese-54	Ci	No Releases	**	9.59E-05	No Releases
Chromium-51	Ci	No Releases	**	**	No Releases
Zirconium-95	Ci	No Releases	**	**	No Releases
Niobium-95	Ci	No Releases	**	**	No Releases
Molybdenum-99	Ci	No Releases	**	**	No Releases
Technetium-99m	Ci	No Releases	**	**	No Releases
Barium-140	Ci	No Releases	**	**	No Releases
Lanthanum-140	Ci	No Releases	**	6.44E-06	No Releases
Cerium-141	Ci	No Releases	**	**	No Releases
Tungsten-187	Ci	No Releases	**	**	No Releases
Arsenic-76	Ci	No Releases	**	3.09E-06	No Releases
Iodine-133	Ci	No Releases	**	**	No Releases
Iron-55	Ci	No Releases	**	**	No Releases
Neptunium-239	Ci	No Releases	**	**	No Releases
Silver-110m	Ci	No Releases	**	**	No Releases
Gold-199	Ci	No Releases	**	**	No Releases
Cerium-144	Ci	No Releases	**	**	No Releases
Cesium-136	Ci	No Releases	**	**	No Releases
Copper-64	Ci	No Releases	**	**	No Releases
Dissolved or Entrained Gases	Ci	No Releases	**	5.16E-04	No Releases
Tritium	Ci	No Releases	1.72E+00	3.10E+00	No Releases

(1) No continuous mode release occurred during the report period as indicated by effluent sampling.

(2) Concentrations less than the lower limit of detection of the counting system used are indicated with a double asterisk. A lower limit of detection of 5.00E-07 µCi/ml for required gamma emitting nuclides, 1.00E-05 µCi/ml for required dissolved and entrained noble gases and tritium, 5.00E-08 µCi/ml for Sr-89/90, 1.00E-06 µCi/ml for I-131 and Fe-55, and 1.00E-07 µCi/ml for gross alpha radioactivity, as identified in the ODCM, has been verified.

ATTACHMENT 6

Unit 1 _____ Unit 2 X Reporting Period January - December 2010

SOLID WASTE AND IRRADIATED FUEL SHIPMENTS

A1. TYPE	Volume (m^3) Class			Activity (1) (Ci) Class		
	A	B	C	A	B	C
a.1 Spent Resins (Dewatered)	3.67E+01	5.49E+00	0.00E+00	7.47E+01	3.15E+02	0.00E+00
a.2 Filter Sludge	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
a.3 Concentrated Waste	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Totals	3.67E+01	5.49E+00	0.00E+00	7.47E+01	3.15E+02	0.00E+00
b.1 Dry, compressible waste	5.01E+02	0.00E+00	0.00E+00	5.79E+00	0.00E+00	0.00E+00
b.2 Dry, non-compressible waste (contaminated equipment)	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Totals	5.01E+02	0.00E+00	0.00E+00	5.79E+00	0.00E+00	0.00E+00
c. Irradiated Components, Control Rods	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
d. Other (to vendor for processing)						
d.1 Scrap Metal, Liquid Drums, Grit/Rubble	1.48E+02	0.00E+00	0.00E+00	7.78E+00	0.00E+00	0.00E+00

(1) The estimated total error is 5.00E+01%.

ATTACHMENT 6

Unit 1 _____ Unit 2 X Reporting Period January - December 2010

SOLID WASTE AND IRRADIATED FUEL SHIPMENTS

A1. TYPE	Container	Package	Solidification Agent
a.1 Spent Resin (Dewatered)	Poly Liner	General Design Type A / Type B	None
a.2 Filter Sludge	N/A	N/A	N/A
a.3 Concentrated Waste	N/A	N/A	N/A
b.1 Dry, Compressible waste	Metal Box Steel Liner	General Design	None
b.2 Dry, non-compressible waste (Contaminated Equipment)	N/A	N/A	N/A
c. Irradiated Components, Control Rods	N/A	N/A	N/A
d. Other (to vendor for processing)			
d.1 Scrap Metal, Liquid Drums, Grit/Rubble	Metal Box	General Design	None

ATTACHMENT 6

Unit 1 _____ Unit 2 X Reporting Period January - December 2010

SOLID WASTE AND IRRADIATED FUEL SHIPMENTS

A2. ESTIMATE OF MAJOR NUCLIDE COMPOSITION (BY TYPE OF WASTE)

a. Spent Resins, Filter Sludges, Concentrated Waste

Nuclide	Percent
Fe-55	47.3
Co-60	37.2
Mn-54	9.9
Zn-65	2.8
Ni-63	2.2
H-3, Fe-59, Co-58, Sr-89, Sr-90, Ag-110m, Cs-134, Cs-137, Ce-144, Pu-238, Pu-239, Am-241, Cm-242, Cm-243	0.6

b. Dry, compressible waste, dry, non-compressible waste (contaminated equipment)

Nuclide	Percent
Fe-55	78.9
Co-60	18.5
Mn-54	1.9
H-3, Ni-63, Zn-65, Cs-134, Cs-137	0.7

c. Irradiated Components, Control Rods: There were no shipments.

Nuclide	Percent
N/A	N/A

d. Other (To Vendor for Processing)

1. Scrap Metal, Liquid Drums, Grit/Rubble

Nuclide	Percent
Fe-55	45.1
Co-60	38.4
Mn-54	11.1
Zn-65	2.6
Ni-63	2.5
H-3, Co-58, Sr-90, Cs-134, Cs-137, Ce-144 Pu-238, Pu-239, Am-241, Cm-242, Cm-243	0.3

ATTACHMENT 6

Unit 1 _____ Unit 2 X Reporting Period January - December 2010

SOLID WASTE AND IRRADIATED FUEL SHIPMENTS

A3. SOLID WASTE DISPOSITION

Number of Shipments	Mode of Transportation	Destination
16	R & R Trucking	Studsvik Processing Facility - Memphis
12	Hittman Transport	Duratek Services, Inc
10	Hittman Transport	Studsvik Processing Facility - Erwin
1	Hittman Transport	Studsvik Processing Facility - Memphis

B. IRRADIATED FUEL SHIPMENTS (Disposition): There were no shipments.

Number of Shipments	Mode of Transportation	Destination
0	N/A	N/A

D. SEWAGE WASTES SHIPPED TO A TREATMENT FACILITY FOR PROCESSING AND BURIAL

There were no shipments of sewage sludge in 2010 from Unit 2.

ATTACHMENT 7

Unit 1		Unit 2	X	Reporting Period	January - December 2010

SUMMARY OF CHANGES TO THE OFF-SITE DOSE CALCULATION MANUAL (ODCM)

The Unit 2 Off-Site Dose Calculation Manual (ODCM) was not revised during the reporting period.

ATTACHMENT 8

Unit 1 _____ Unit 2 X Reporting Period January - December 2010

SUMMARY OF CHANGES TO THE PROCESS CONTROL PROGRAM (PCP)

There were no changes to the NMP2 Process Control Program (PCP) during the reporting period.

ATTACHMENT 9

Unit 1 _____ Unit 2 X Reporting Period January - December 2010

SUMMARY OF NON-FUNCTIONAL MONITORS

Monitor	Dates Monitor was Non-Functional	Cause and Corrective Actions
2LWS-CAB206, Liquid Waste Discharge Monitor	January 1, 2010 to March 30, 2010 and August 7, 2010 to December 31, 2010	This monitor exceeded 30 days non-functional twice during 2010. The cause of the non-functionality was an intentional lapse of the required surveillances since no liquid discharges were planned or scheduled. While the monitor is non-functional, the manual discharge isolation valves are locked in the closed position. The current non-functionality is tracked under ESL 2010-0211.
2SWP*CAB146A, Service Water Discharge Radiation Monitor	April 23, 2010 to June 14, 2010	This monitor exceeded 30 days non-functional when a Low Voltage Power Supply (LVPS) failed. After replacing the power supply, several circuit cards were determined to be damaged by the LVPS failure. Due to obsolescence, limited spare parts are available, and additional time was required to restore the monitor to functional status.
2RMS-CAB170A, Stack Wide Range Gaseous Monitoring System (WRGMS)	January 7, 2010 to March 18, 2010	The cause of the non-functional event was a design error that allowed the total effluent flow to exceed the measurable design limits during standby gas treatment system operation in certain configurations. The design calculations were revised to exceed the actual maximum flow, and the flow transmitters were re-ranged to measure this flow.
2RMS-CAB180A, Combined Reactor Building/ Rad. Waste Building Effluent Wide Range Gaseous Monitoring System (WRGMS)	November 30, 2010 to December 31, 2010	The cause of the non-functional event was a solenoid valve in the mid/high range sampling train leaking past its seat during the 18 month calibration. A replacement valve was obtained, and the surveillance test was re-performed satisfactorily. During this test, however, the test methodology used to test the sample flow rate was found to be incorrect. A new test methodology was develped, procedures were corrected, and the system was tested satisfactorily to the new methodology. No adjustments were required when the new test methodology was used. This required change in test methodology and procedures caused the non-functional time to exceed 30 days.
2RMS-CAB170A and 2RMS-CAB180A, Stack and combined Vent WRGMS	See Cause and Corrective Actions	Both monitors were determined to have been tested with an invalid test methodology since installation. Procedures for both systems were corrected, and both systems were tested satisfactorily. No adjustments were required to either system when the corrected test methodology was used. 2RMS-CAB170A was tested satisfactorily and declared functional on December 30, 2010. 2RMS-CAB180A was tested satisfactorily and declared functional on December 31, 2010.

ATTACHMENT 10

Unit 1 ____ Unit 2 __X__ Reporting Period: **January - December 2010**

DOSES TO MEMBERS OF THE PUBLIC DUE TO THEIR ACTIVITIES INSIDE THE SITE BOUNDARY

Introduction

An assessment of the radiation dose potentially received by a Member of the Public due to their activities inside the site boundary from Nine Mile Point Unit 2 (NMP2) liquid and gaseous effluents has been conducted for the period January through December 2010.

This assessment considers the maximum exposed individual and the various exposure pathways resulting from liquid and gaseous effluents to identify the maximum dose received by a Member of the Public during their activities within the site boundary.

Prior to September 11, 2001, the public had access to the Energy Information Center for purposes of observing the educational displays or for picnicking and associated activities. Fishing also occurred near the shoreline adjacent to the Nine Mile Point (NMP) site. Fishing near the shoreline adjacent to the NMP site was the onsite activity that resulted in the potential maximum dose received by a Member of the Public. Following September 11, 2001 public access to the Energy Information Center has been restricted and fishing by Members of the Public at locations on site is also prohibited. Although fishing was not conducted during 2010 the annual dose to a hypothetical fisherman was still evaluated to provide continuity of data for the location.

Dose Pathways

Dose pathways considered for this evaluation included direct radiation, inhalation and external ground (shoreline sediment or soil doses). Other pathways, such as ingestion pathways, are not considered because they are either not applicable, insignificant, or are considered as part of the evaluation of the total dose to a member of the public located off-site. In addition, only releases from the NMP2 Stack and Radwaste/Reactor Building Vent were evaluated for the inhalation pathway. Dose due to aquatic pathways such as liquid effluents is not applicable since swimming is prohibited at the NMP site.

Dose to a hypothetical fisherman is received through the following pathways while standing on the shoreline fishing:

- External ground pathway; this dose is received from plant related radionuclides detected in the shoreline sediment.
- Inhalation pathway; this dose is received through inhalation of gaseous effluents released from the NMP2 Stack and Radwaste/Reactor Building Vent.
- Direct radiation pathway; dose resulting from the operation of Nine Mile Point Unit 1 (NMP1), NMP2 and the James A. Fitzpatrick Nuclear Power Plant (JAFNPP) Facilities.

Methodologies for Determining Dose for Applicable Pathways

External Ground (Shoreline Sediment) Pathway

Dose from the external ground (shoreline sediment) is based on the methodology in the NMP2 Offsite Dose Calculation Manual (ODCM) as adapted from Regulatory Guide 1.109. For this evaluation it is assumed that the hypothetical maximum exposed individual fished from the shoreline at all times.

Unit 1 _____ Unit 2 __X__ Reporting Period: **January - December 2010**

DOSES TO MEMBERS OF THE PUBLIC DUE TO THEIR ACTIVITIES INSIDE THE SITE BOUNDARY

The total dose received by the whole body and skin of the maximum exposed individual during 2010 was calculated using the following input parameters:

- Usage Factor = 312 hours (fishing 8 hours per week, 39 weeks per year)
- Density in grams per square meter = 40,000
- Shore width factor = 0.3
- Whole body and skin dose factor for each radionuclide = Regulatory Guide 1.109, Table E-6.
- Fractional portion of the year = 1 (used average radionuclide concentration over total time period)
- Average Cs-137 concentration = 1.89E-01 pCi/g

The total whole body and skin doses received by a hypothetical maximum exposed fisherman from the external ground pathway is presented in Table 1, Exposure Pathway Annual Dose.

Inhalation Pathway

The inhalation dose pathway is evaluated by utilizing the inhalation equation in the NMP2 ODCM, as adapted from Regulatory Guide 1.109. The total whole body dose and organ dose received by the hypothetical maximum exposed fisherman during 2010 calculated using the following input parameters for gaseous effluents released from both the NMP2 Stack and Radwaste/Reactor Building Vent for the time period exposure is received:

NMP2 Stack:

Variable	Fisherman [1]
X/Q (s/m^3)	9.6 E-07
Inhalation dose factor	Table E-7, Regulatory Guide 1.109
Annual air intake (m^3/year) (adult)	8000
Fractional portion of the year	0.0356
H-3 (pCi/sec)	1.82 E+06
C-14 (pCi/sec)[2]	4.87E+05
Mn-54 (pCi/sec)	8.16 E-01
Fe-55 (pCi/sec)	1.13 E+01
Co-60 (pCI/sec)	1.59 E+00
Sr-89 (pCI/sec)	4.01 E+00
I-131 (pCi/sec)	7.54 E+01
I-133 (pCi/sec)	6.76 E+02
Ba-140 (pCi/sec)	2.63 E-01
Nd-147 (pCi/sec)	3.48 E-01

ATTACHMENT 10

Unit 1 ____ Unit 2 __X__ Reporting Period: **January - December 2010**

DOSES TO MEMBERS OF THE PUBLIC DUE TO THEIR ACTIVITIES INSIDE THE SITE BOUNDARY

NMP2 Radwaste/Reactor Building Vent:

Variable	Fisherman [1]
X/Q (s/m^3)	2.8 E-06
Inhalation dose factor	Table E-7, Regulatory Guide 1.109
Annual air intake (m^3/year) (adult)	8000
Fractional portion of the year	0.0356
H-3 (pCi/sec)	6.62 E+05
Cr-51 (pCi/sec)	2.20 E+00
Mn-54 (pCi/sec)	4.33 E+00
Fe-55 (pCi/sec)	6.18 E+01
Fe-59 (pCi/sec)	6.24 E-01
Co-58 (pCi/sec)	1.99 E-01
Co-60 (pCi/sec)	1.26E+01
Zn-65 (pCi/sec)	1.65 E+00
I-131 (pCi/sec)	1.08 E+01
I-133 (pCi/sec)	1.46 E+02
Nd-147 (pCi/sec)	2.15E+01

1. The maximum exposed fisherman is assumed to be present on site during the period of April through December at a rate of 8 hours per week for 39 weeks per year equivalent to 312 hours for the year (fractional portion of the year = 0.0356). Therefore, the Average Stack and Radwaste/Reactor Building Vent flow rates and radionuclide concentrations used to determine the dose are represented by second, third and fourth quarter gaseous effluent flow and concentration values.
2. C-14 release rate determined from NUREG-0016, "Calculation of Releases of Radioactive Materials in Gaseous and Liquid Effluents for Boiling Water Reactors (BWR-GALE Code)," and EPRI Technical Report 1021106, "Estimation of Carbon-14 in Nuclear Power Plant Gaseous Effluents."

The total whole body dose and maximum organ dose received by the hypothetical maximum exposed fisherman is presented in Table 1, Exposure Pathway Annual Dose.

ATTACHMENT 10

Unit 1 ____ Unit 2 __X__ Reporting Period: **January - December 2010**

DOSES TO MEMBERS OF THE PUBLIC DUE TO THEIR ACTIVITIES INSIDE THE SITE BOUNDARY

Direct Radiation Pathway

The direct radiation pathway is evaluated in accordance with the methodology found in the NMP2 ODCM. This pathway considers four components: direct radiation from the generating facilities, direct radiation from any possible overhead plume, direct radiation from ground deposition and direct radiation from plume submersion. The direct radiation pathway is evaluated by the use of high sensitivity environmental Thermoluminescent Dosimeters (TLDs). Since fishing activities occur between April 1 and December 31, TLD data for the second, third, and fourth quarters of 2010 from TLDs placed in the general area where fishing once occurred were used to determine an average dose to the hypothetical maximum exposed fisherman from direct radiation. The following is a summary of the average dose rate and assumed time spent on site used to determine the total dose received:

Variable	Fisherman
Average Dose Rate (mRem/hr)	1.55 E-03
Exposure time (hours)	312

Total Doses received by the hypothetical maximum exposed fisherman from direct radiation is presented in Table 1, Exposure Pathway Annual Dose.

Dose Received By A Hypothetical Maximum Exposed Member of the Public Inside the Site Boundary During 2010

The following is a summary of the dose received by a hypothetical maximum exposed fisherman from liquid and gaseous effluents released from NMP2 during 2010:

TABLE 1
Exposure Pathway Annual Dose

Exposure Pathway	Dose Type	Fisherman (mrem)
External Ground	Whole Body	2.97 E-03
	Skin of Whole Body	3.46 E-03
Inhalation	Whole Body	1.62 E-04
	Whole Body (including C-14)	2.19 E-04
	Maximum Organ	Thyroid: 2.87 E-04
	Maximum Organ (including C-14)	Bone: 3.03 E-04
Direct Radiation	Whole Body	0.48

ATTACHMENT 10

Unit 1 _____ Unit 2 __X__ Reporting Period: **January - December 2010**

DOSES TO MEMBERS OF THE PUBLIC DUE TO THEIR ACTIVITIES INSIDE THE SITE BOUNDARY

Based on these values, the total annual dose received by a hypothetical maximum exposed Member of the Public inside the site boundary is as follows:

TABLE 2
Annual Dose Summary

Total Annual Dose for 2010	**Fisherman (mrem)**
Total Whole Body	4.87 E-01
Total Whole Body (including C-14)	4.87 E-01
Skin of Whole Body	3.46 E-03
Maximum Organ	Thyroid: 2.87 E-04
Maximum Organ (including C-14)	Bone: 3.03 E-04

ATTACHMENT 11

Unit 1 ____ Unit 2 __X__ Reporting Period: January - December 2010

DOSES TO MEMBERS OF THE PUBLIC DUE TO THEIR ACTIVITIES OUTSIDE THE SITE BOUNDARY

Introduction

An assessment of radiation doses potentially received by the likely most exposed Member of the Public located beyond the site boundary was conducted for the period January through December 2010 for comparison against the 40 CFR 190 annual dose limits.

The intent of 40 CFR 190 requires that the effluents of Nine Mile Point Unit 2 (NMP2), as well as other nearby uranium fuel cycle facilities, be considered. In this case, the effluents of Nine Mile Point Unit 1 (NMP1), NMP2 and the James A. FitzPatrick Nuclear Power Plant (JAFNPP) facilities must be considered.

40 CFR 190 requires the annual radiation dose received by Members of the Public in the general environment, as a result of plant operations, be limited to:

- < 25 mRem whole body
- < 25 mRem any organ (except thyroid)
- < 75 mRem thyroid

This evaluation compares doses resulting from liquid and gaseous effluents and direct radiation originating from the site as a result of the operation of the NMP1, NMP2 and JAFNPP nuclear facilities.

Dose Pathways

Dose pathways considered for this evaluation included doses resulting from liquid effluents, gaseous effluents and direct radiation from all nuclear operating facilities located on the Nine Mile Point Site.

Dose to the likely most exposed Member of the Public, outside the site boundary, is received through the following pathways:

- Fish consumption pathway; this dose is received from plant radionuclides that have concentrated in fish that is consumed by a Member of the Public.
- Vegetation consumption pathway; this dose is received from plant radionuclides that have concentrated in vegetation that is consumed by a Member of the Public.
- Shoreline Sediment; this dose is received as a result of an individual's exposure to plant radionuclides deposited in the shoreline sediment, which is used as a recreational area.
- Deposition, Inhalation and Ingestion pathways resulting from gaseous effluents; this dose is received through exposure to gaseous effluents released from NMP1, NMP2 and JAFNPP operating facilities.
- Direct Radiation pathway; radiation dose resulting from the operation of NMP1, NMP2 and JAFNPP facilities.

Methodologies for Determining Dose for Applicable Pathways

Fish Consumption

Dose received as a result of fish consumption is based on the methodology specified in the NMP2 Off-Site Dose Calculation Manual (ODCM) as adapted from Regulatory Guide 1.109. The dose for 2010 is calculated from actual analysis results of environmental fish samples taken near the site discharge points. For this evaluation it is assumed that the most likely exposed Member of the Public consumes fish taken near the site discharge points.

No radionuclides were detected in fish samples collected and analyzed during 2010; therefore, no dose was received by the whole body and organs of the likely most exposed Member of the Public during 2010.

Unit 1 _____ Unit 2 __X__ Reporting Period: January - December 2010

DOSES TO MEMBERS OF THE PUBLIC DUE TO THEIR ACTIVITIES OUTSIDE THE SITE BOUNDARY

Vegetation Consumption

Dose received as a result of vegetation consumption is based on the methodology specified in the NMP2 ODCM as adapted from Regulatory Guide 1.109. The dose for 2010 is calculated from actual analysis results of environnmental vegetation samples taken near the likely most exposed Member of the Public.

No radionuclides were detected in vegetation samples collected and analyzed during 2010; therefore, no dose was received by the whole body and organs of the likely most exposed Member of the Public during 2010.

For estimating C-14, dose received as a result of vegetation consumption is based on the methodology specified in the NMP2 ODCM as adapted from Regulatory Guide 1.109. The estimated concentration of C-14 in vegetation is based on the estimated concentration of C-14 in the plant gaseous effluents.

Shoreline Sediment

Dose received from shoreline sediment is based on the methodology in the NMP2 ODCM as adapted from Regulatory Guide 1.109. For this evaluation it is assumed that the most likely exposed Member of the Public spends 67 hours/year along the shoreline for recreational purposes.

No radionuclides were detected in shoreline sediment samples collected and analyzed during 2010; therefore, no dose was received by the whole body and organs of the likely most exposed Member of the Public during 2010.

Dose Pathways Resulting From Gaseous Effluents

Dose received by the likely most exposed Member of the Public due to gaseous effluents is calculated in accordance with the methodology provided in the NMP1 ODCM, NMP2 ODCM, and the JAFNPP ODCM. These calculations consider deposition, inhalation and ingestion pathways. The total sum of doses resulting from gaseous effluents from NMP1, NMP2 and JAFNPP during 2010 provides a total dose to the whole body and maximum organ dose for this pathway.

Carbon-14 Dose Pathways Resulting from Gaseous Effluents

The carbon-14 (C-14) effluent source terms are used to estimate radiological doses from C-14 in site gaseous waste effluents. These estimates were generated in order to meet the NRC requirement to incorporate C-14 in nuclear power plant 2010 Annual Radiological Effluent Release Reports (ARERRs). The C-14 production and effluent source term estimates were based on EPRI methodology provided in EPRI Report 1021106, *Estimation of Carbon-14 in Nuclear Power Plant Gaseous Effluents, December 2010.* The following methodology was used to estimate C-14 gaseous release activity and dose components for the 2010 ARERR.

Unit 1 ____ Unit 2 __X__ Reporting Period: January - December 2010

DOSES TO MEMBERS OF THE PUBLIC DUE TO THEIR ACTIVITIES OUTSIDE THE SITE BOUNDARY

EPRI Methodology for Estimating C-14 Production Rates in Boiling Water Reactors (BWRs):

For BWRs, EPRI Report 1021106 summarized the distribution of C-14 in release pathways as follows: gaseous 95% to 99%, liquid <0.5% and solid 1% to 5%. The report also states that ~95% of C-14 in BWR gaseous waste effluents exists in the carbon dioxide form, which contributes to population dose via photosynthesis uptake in the food consumption cycle.

For NMP1 and NMP2, C-14 gaseous dose calculations in the site ARERR are made using the following assumptions for each unit: (1) continuous release of the estimated C-14 generated during power operation based on the number of Effective Full Power Days (EFPDs) for the period, (2) maximum C-14 activity from literature values cited in EPRI Report 1021106, and (3) typical fraction as carbon dioxide for gaseous releases from literature values also cited in EPRI Report 1021106.

Equation 1 estimates the maximum annual production of C-14, PR_{MAX}, for each BWR unit.

$$PR_{MAX} = 5.1 \bullet MWT / 1000 \qquad [Eq\ 1]$$

Where:

5.1	=	BWR Normalized Production (Ci/GWt–yr)
MWT	=	MegaWatts Thermal (MWt)
1000	=	Conversion Factor (MWt to GWt)

Equation 2 estimates the C-14 activity released, A_{C-14}, into the gaseous pathway during the time period for each BWR unit.

$$A_{C-14} = PR_{MAX} \bullet 0.99 \bullet EFPD / 365, Ci\ (for\ time\ period) \qquad [Eq\ 2]$$

Where:

PR_{MAX}	=	maximum annual production rate of C-14
0.99	=	fraction C-14 in BWR gaseous pathway releases (maximum literature value in EPRI Report 1021106)
EFPD	=	number of effective full power days for the unit during the time period; e.g., quarterly or yearly (Table 1)
365	=	number of days in a typical year

Unit 1 ____ Unit 2 __X__ Reporting Period: January - December 2010

DOSES TO MEMBERS OF THE PUBLIC DUE TO THEIR ACTIVITIES OUTSIDE THE SITE BOUNDARY

Equation 3 estimates the C-14 activity released in carbon dioxide form, $A_{C\text{-}14,\ CO2}$, into the gaseous pathway during the time period for each BWR unit.

$$A_{C\text{-}14,\ CO2} = PR_{MAX} \bullet 0.99 \bullet 0.95 \bullet EFPD / 365,\ Ci\ (for\ time\ period) \qquad [Eq\ 3]$$

Where:

PR_{MAX}	=	maximum annual production rate of C-14
0.99	=	fraction of C-14 in BWR gaseous pathway releases (maximum literature value in EPRI Report 1021106)
0.95	=	fraction of C-14 as carbon dioxide in BWR gaseous pathway releases (typical literature value in EPRI Report 1021106
EFPD	=	number of effective full power days for the unit during the time period, e.g. quarterly or yearly (Table 1)
365	=	number of days in a typical year

For each BWR unit, the 2010 estimated C-14 activity releases (total and carbon dioxide chemical form) are summarized in Table 1.

Table 1
2010 BWR Estimated C-14 Gaseous Releases

BWR	Gaseous Release Fraction[a]	CO_2 Form Release Fraction[b]	EFPD Operation	Max. Annual Prod. Rate (Eq 1)	2010 Total Release (Eq 2)	2010 CO_2 Release (Eq 3)
NMP1	0.99	0.95	358 EFPD (98.08%)	9.44 Ci/yr	9.16 Ci	8.70 Ci
NMP2	0.99	0.95	328 EFPD (89.86%)	17.68 Ci/yr	15.73 Ci	14.9 Ci
JAFNPP	0.99	0.95	316 EFPD (86.6%)	10.82 Ci/yr	9.29 Ci	8.82 Ci

(a) Maximum literature values from EPRI Report 1021106.
(b) Typical value from EPRI Report 1021106.

As long as the core designs and power ratings are not significantly changed, the maximum annual production rates and annual total and carbon dioxide activity releases in Table 1 are considered acceptable for use in estimating C-14 gaseous release activity and dose components for the ARERR.

ATTACHMENT 11

Unit 1 ____ Unit 2 __X__ Reporting Period: January - December 2010

DOSES TO MEMBERS OF THE PUBLIC DUE TO THEIR ACTIVITIES OUTSIDE THE SITE BOUNDARY

Direct Radiation Pathway

Dose as a result of direct gamma radiation from the site encompasses doses from direct "shine" from the generating facilities, direct radiation from any overhead gaseous plumes, plume submersion, and ground deposition. This total dose is measured by environmental TLDs. The critical location is based on the closest year-round residence from the generating facilities as well as the closest residence in the critical downwind sector in order to evaluate both direct radiation from the generating facilities and gaseous plumes as determined by the local meteorology. During 2010, the closest residence and the critical downwind residence are at the same location.

Table 2
Dose Potentially Received by the Likely Most Exposed Member of the Public Outside the Site Boundary During 2010

Exposure Pathway	Dose Type	Dose (mrem)
Fish Consumption	Total Whole Body	No Dose
	Total Maximum Organ	No Dose
Shoreline Sediment	Total Whole Body	No Dose
	Total Skin of Whole Body	No Dose
Gaseous Effluents	Total Whole Body	1.06 E-03
	Thyroid	1.50 E-03
	Maximum Organ	Bone: 2.29 E -03
Ingestion (C-14)	Total Whole Body	4.32 E-02
	Thyroid	4.32 E-02
	Maximum Organ	Bone: 2.15 E-01
Direct Radiation	Total Whole Body	3.7

Based on these values the maximum total annual dose potentially received by the likely most exposed Member of the Public during 2010 is as follows:

- Total Whole Body: 3.79 E+00 mrem
- Total Thyroid: 4.47 E-02 mrem
- Maximum Organ: Bone: 2.17 E-01 mrem

40 CFR 190 Evaluation

The maximum total doses presented in this attachment are the result of operations at the NMP1, NMP2 and JAFNPP facilities. The maximum organ dose (Bone: 0.217 mrem), maximum thyroid dose (0.045 mrem) and the maximum whole body dose (3.79 mrem) are below the 40 CFR 190 criteria of 25 mrem per calendar year to the maximum exposed organ or the whole body, and below 75 mrem per calendar year to the thyroid.

ATTACHMENT 12

GROUNDWATER MONITORING DATA

Reporting Period January - December 2010

CONCENTRATION OF TRITIUM IN GROUNDWATER SAMPLES
(pCi/l ±1 sigma)

Control Location	Date	Tritium
GMX-MW1	4/7/2010	<410
GMX-MW2	4/7/2010	<410
GMX-MW1	9/27/2010	<409
GMX-MW2	9/27/2010	<172
GMX-MW1	10/11/2010	<409
GMX-MW2	10/12/2010	<409

Indicator Location	Date	Tritium
MW-1	4/7/2010	<410
MW-5	4/7/2010	<400
MW-6	4/7/2010	<410
MW-7	4/7/2010	<410
MW-8	4/7/2010	<410
MW-1	9/28/2010	<289
MW-5	9/28/2010	<292
MW-6	9/28/2010	<288
MW-7	9/28/2010	<287
MW-8	9/27/2010	<287
MW-1	10/12/2010	<409
MW-5	10/12/2010	<409
MW-6	10/11/2010	<409
MW-7	10/11/2010	<409
MW-8	10/11/2010	<409
MW-6	12/10/2010	<422
MW-7	12/10/2010	611[2]
MW-8	12/10/2010	<422

Indicator Location	Date	Tritium
NMP2 Depression Cone[1]	1/4/2010	<420
	4/5/2010	<405
	7/6/2010	<416
	10/4/2010	<289
	11/15/2010	446[3]
	11/22/2010	<420
	11/29/2010	<424
	12/6/2010	<415
	12/13/2010	<422
	12/20/2010	<416
	12/27/2010	<420

[1] Sample collected from storm drain, discharge point of Nine Mile Point 2 (NMP2) depression cone sumps.

[2] Monitoring well re-sampled and analyzed, tritium concentration was determined to be less than 268 pCi/l.

[3] Positive detection due to a monitored release in accordance with the Nine Mile Point Unit 1 Off-Site Dose Calculation Manual (Emergency Condenser Vent Releases).