## NUREG/IA-0251



# International Agreement Report

## Improvement of RELAP5/MOD3.3 Reflood Model Based on the Assessments against FLECHT-SEASET Tests

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Office of Nuclear Regulatory Research U.S. Nuclear Regulatory Commission Washington, DC 20555-0001

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## ABSTRACT

The reflood model of RELAP5/MOD3.3 was assessed against eight selected FLECHT-SEASET tests. Comparisons of predicted and measured peak cladding temperatures and quench times indicated that the code predicts peak cladding temperatures relatively well. However, rod quenches were predicted to occur too early. To improve the predictability for quench times, we carefully reviewed and modified the wall-to-fluid heat transfer models for the film boiling regime. After the modifications, the same set of eight FLECHT-SEASET tests was simulated again to show that the modifications made in this study improve the code's predictability not only for quench times but also for peak clad temperatures. The modifications reduced the RMS error in the prediction of peak clad temperatures and quench times from 48.3 to 36.7 K and 85.9 to 32.1 seconds, respectively.

#### FOREWORD

RELAP5 is one of the best-estimate safety codes to date. It was developed by United States Nuclear Regulatory Commission (USNRC) and RELAP5/MOD3.3 (patch 03), released in 2006, is its latest version. Though USNRC has been moving most of their developmental efforts from RELAP5 to TRACE, the code has been widely applied to analyze system transients of Pressurized Water Reactors (PWRs), including the postulated large break loss-of-coolant accident (LBLOCA).

A few years ago, three Korean organizations, Korea Atomic Energy Research Institute (KAERI), Korea Electric Power Research Institute (KEPRI), and Korea Nuclear Fuel (KNF), succeeded to co-develop a best-estimate LBLOCA evaluation model using a modified version of RELAP5/MOD3.1. And now, KNF is trying to extend the best-estimate evaluation model by adopting RELAP5/MOD3.3 because it has several new models or improvements to existing models, and user conveniences compared to (a modified version of) RELAP5/MOD3.1.

To develop a best estimate evaluation model for LBLOCA analyses using a code, we have to show how accurately the code predicts important phenomena occurring in that accident by comparing the code predictions to various separate effect test data and that is why we tried to assess the reflood model of RELAP5/MOD3.3 against FLECHT-SEASET tests.

This report was prepared basically to record the results that we obtained from the RELAP5/MOD3.3 assessment using the FLECHT-SEASET test data. By the way, we had to modify the code because the code was appraised to have some deficiencies in the reflood wall-to-fluid heat transfer models. So, this report explains also the code modifications we made and their effect on the code's predictive capability.

The code modifications suggested in this report makes the code predict much better the peak clad temperatures and quench times of FLECHT-SEASET tests. However, the reflood model of RELAP5/MOD3.3 should be further improved because the code does not show good predictive capability for some test conditions even with the modifications. Thus, this study needs to be followed by such improvements as the addition of a spacer grid heat transfer model.

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## EXECUTIVE SUMMARY

To assess the reflood model of RELAP5/MOD3.3, simulations of eight FLECHT-SEASET tests were conducted. The selected FLECHT-SEASET tests have various initial and boundary conditions such as the flooding rate varying from 2.1 to 15.5 cm/s, the upper plenum pressure varying from 0.13 to 0.41 MPa, and the coolant temperature varying from 306 to 408 K. A more detailed axial noding for the bundle test section was used in this assessment compared to other previous ones. Predictions for several significant parameters including peak clad temperatures, quench times, and heat transfer coefficients at various elevations were compared to the measurement data. From the comparison, the following findings were obtained.

- The code predicts the measured peak clad temperatures within ±10% deviation.
- The code predicts rod quenches to occur much earlier than the measurement data.
- The code has a tendency of under-predicting peak vapor temperatures at 2 different elevations, especially those of low flooding rate tests.
- The code has a tendency of over-predicting the overall pressure drop for the entire bundle (or collapsed level in the core).

Based on these findings, the wall-to-fluid heat transfer models which are activated during the reflood calculations were modified to improve the code's predictability mainly for quench times. The modifications can be summarized as follows:

- The film boiling heat transfer regime was divided into three different sub-regimes (inverted annular film boiling regime, inverted slug film boiling regime, and dispersed flow film boiling regime).
- Appropriate heat transfer correlations were selected to predict wall-to-vapor or wall-to-liquid heat transfers in each sub-regime.
- Minor coding errors in the subroutines computing the post-CHF droplet diameter and the transition boiling heat transfer coefficient, respectively were fixed.
- The radiation heat transfer model was slightly modified to assume more reasonable and consistent values for emissivity and droplet diameter.

All the modifications were embodied into a modified version and the same set of eight FLECHT-SEASET tests were simulated again with the modified version. From the comparisons of the predictions using the modified version to the predictions using the original version as well as the measurement data, the following conclusions were derived.

- The RMS error in overall PCT predictions can be reduced from 48.3 to 36.7 K by the modifications.
- The RMS error in overall quench time predictions can be reduced from 85.9 to 32.1 seconds by the modifications.
- The film boiling heat transfer coefficients predicted using the modified version shows better agreement with the measurement data compared to those predicted using the original version.
- Wall-to-vapor heat transfers are increased by the modifications so that the predicted vapor

temperatures are higher than those predicted by the original version.

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• Even with the modifications, the code still has a tendency of over-predicting the overall pressure drop for the entire core (or collapsed liquid level in the core). Thus, further efforts to improve the reflood model of RELAP5/MOD3.3 should be made to the wall or interfacial drag models.

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## ABBREVIATIONS

HTC	Heat Transfer Coefficient
KAERI	Korea Atomic Energy Research Institute
KEPRI	Korea Electric Power Research Institute
KNF	Korea Nuclear Fuel
LBLOCA	Large Break Loss-Of-Coolant-Accident
PCT	Peak Clad Temperature
PWR	Pressurized Water Reactor
RMS	Root-Mean-Square
USNRC	United States Nuclear Regulatory Commission

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## NOMENCLAUTURE

a <sub>f</sub>	liquid absorption coefficient
a <sub>gf</sub>	interfacial area per unit volume
CD	droplet drag coefficient
D <sub>h</sub>	hydraulic diameter
d	average droplet diameter
d <sub>max</sub>	maximum droplet diameter defined by Eq.(5)
d <sub>We=1.5</sub>	droplet diameter when Weber number = 1.5
d <sub>We=7.5</sub>	droplet diameter when Weber number = 7.5
F <sub>lt</sub>	Linear function depending on the gas Reynolds number
F <sub>wf</sub>	wall-to-liquid grey body factor
$f_w$	wall friction factor
$G_{f}$	liquid mass flux
g	gravitational acceleration
h <sub>Br</sub>	HTC calculated with the Bromley correlation
h <sub>ct</sub>	HTC calculated with the CATHARE correlation
h <sub>Ditt</sub>	HTC calculated with a call to the subroutine DITTUS
h <sub>fg</sub>	latent heat of vaporization
h <sub>FR</sub>	HTC calculated with the Forslund-Rohsenow correlation
h <sub>lam</sub>	laminar convective HTC
h <sub>max</sub>	HTC defined by Eq.(26)
h <sub>MBr</sub>	HTC calculated with the modified Bromley Correlation
h <sub>tur</sub>	turbulent convective HTC
h <sub>wf</sub>	wall-to-liquid HTC
h <sub>wf,DF</sub>	wall-to-vapor HTC for the DFFB regime
h <sub>wf,IA</sub>	wall-to-liquid HTC for the IAFB regime
h <sub>wg</sub>	wall-to-vapor HTC
h <sub>wg,DF</sub>	wall-to-vapor HTC for the DFFB regime
h <sub>wg,lA</sub>	wall-to-vapor HTC for the IAFB regime
K <sub>BY</sub>	Reynolds number dependent coefficient defined by Eq.(13)
k <sub>g</sub>	vapor conductivity
P/D <sub>R</sub>	pitch-to-diameter ratio
q <sub>CHF</sub>	critical heat flux
$R_1, R_2, R_3$	R terms in radiation heat transfer model used to calculate the grey body factor
Red	droplet Reynolds number

T_satsaturation temperatureT_wwall temperatureT_w,CHFwall temperature at the CHF pointvgvapor velocityr_rrelative velocityx, yweighting functions defined by Eq.(10)Z_Qmindistance from the point in question to the bottom quench front	Reg	vapor Reynolds number
Twwall temperatureTw,CHFwall temperature at the CHF pointvgvapor velocityvrrelative velocityx, yweighting functions defined by Eq.(10)Zqmindistance from the point in question to the bottom quench front	T <sub>sat</sub>	saturation temperature
Tw,CHFwall temperature at the CHF pointvgvapor velocityvrrelative velocityx, yweighting functions defined by Eq.(10)Zqmindistance from the point in question to the bottom quench front	T <sub>w</sub>	wall temperature
vgvapor velocityvrrelative velocityx, yweighting functions defined by Eq.(10)ZQmindistance from the point in question to the bottom quench front	T <sub>w,CHF</sub>	wall temperature at the CHF point
vrrelative velocityx, yweighting functions defined by Eq.(10)ZQmindistance from the point in question to the bottom quench front	v <sub>g</sub>	vapor velocity
x, yweighting functions defined by Eq.(10)Z <sub>Qmin</sub> distance from the point in question to the bottom quench front	v <sub>r</sub>	relative velocity
Z <sub>Qmin</sub> distance from the point in question to the bottom quench front	х, у	weighting functions defined by Eq.(10)
	Z <sub>Qmin</sub>	distance from the point in question to the bottom quench front

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### Greeks

$\alpha_{f}$	liquid volume fraction
α <sub>g</sub>	vapor volume fraction
۸T	difference between the wall temperature at the CHF point and the saturation
CHF	temperature
δ	vapor film thickness
ε <sub>f</sub>	liquid emissivity
ε <sub>w</sub>	wall emissivity
$\mu_{g}$	vapor viscosity
$ ho_{f}$	liquid density
$ ho_g$	vapor density
σ	surface tension
$\tau_{i}$	friction force per unit volume due to interfacial drag
τ <sub>w</sub>	fiction force per unit volume due to wall drag
$\Phi_{2p}$	two-phase enhancement factor
$\Phi_{tur}$	turbulent flow multiplier for a vertical bundle geometry

## <u>Subscripts</u>

IA	inverted annular film boiling
DF	dispersed flow film boiling
f	liquid
g	vapor
wf	wall-to-liquid
wg	wall-to-vapor
sat	saturation
lam	laminar
tur	turbulent

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## 1. INTRODUCTION

In the analyses of a Pressurized Water Reactor (PWR) Large Break Loss-Of-Coolant Accident (LBLOCA), one of the most important tasks is to predict accurately the thermal-hydraulic phenomena occurring in the core during the reflood phase. This phase typically starts with a nearly empty core under adiabatic conditions. Subcooled water is then injected from the bottom of core. The injected water interacts with hot fuel rods and produces large amounts of steam which binds the flooding rate via increasing upper plenum pressure. During the phase, various heat transfer regimes (or modes) such as single-phase vapor convection, nucleate boiling, transition boiling, film boiling, and single-phase liquid convection exist in the core, sometimes all at once.

To accurately predict complicated thermal-hydraulic phenomena in the core during reflood, RELAP5/MOD3.3 (patch 03) has a special heat transfer package that is activated during that phase only. Most of this package had been originally developed by Analytis [1] and was slightly modified in its official release.

Few activities, with the exception of developmental assessment by Analytis, have been made to assess the reflood model of RELAP5/MOD3.3. To the best of our knowledge, the most recent assessment of RELAP5 reflood model was conducted by Koszela [2] using an interim version (RELAP5/MOD3.2.2 Gamma). He simulated the ABB Atom 3×3-Rod Bundle Reflooding Tests using the default model options and concluded that the code still had several deficiencies in the reflood model. In his work, the cladding temperature peaks and final quenches were predicted to occur too early.

One of the purposes of this report is to assess the RELAP5/MOD3.3 reflood model using selected FLECHT-SEASET tests. Compared to previous similar assessments [3], [4], a more detailed axial noding for the bundle test section and more accurate initial and boundary conditions were used in this assessment.

Based on our assessment, we carefully reviewed and modified the wall-to-fluid heat transfer models. Note that these modifications introduced nearly no new heat transfer correlations. The correlations used in the original version were restructured to result in better predictions.

To investigate the effect of our modifications, assessment calculations were conducted again using the modified version against the same set of FLECHT-SEASET tests. Various significant parameters including peak clad temperatures (PCTs), quench times, and heat transfer coefficients (HTCs) were compared to both values predicted using the original version and measurements.

The assessment of the original RELAP5/MOD3.3 reflood model against eight FLECHT-SEASET tests is described in Chapter 2. A brief description for the FLECHT-SESET facility and the selected tests is also made in this chapter. Chapter 3 describes the reflood model of RELAP5/MOD3.3 briefly and explains how we modified the code to improve the wall-to-fluid heat transfer models. The effect of code modifications is also analyzed in this Chapter. Final conclusions of this study are presented in Chapter 4.

## 2. ASSESSMENT OF ORIGINAL REFLOOD MODEL

### 2.1 FLECHT-SEASET\_Tests

The FLECHT-SEASET tests have been regarded as representative reflood experiments because the test facility was large and relatively well instrumented. The test facility was constructed by modifying the FLECHT facility mainly to accept a new heater rod bundle of which dimensions were typical of the Westinghouse 17x17 fuel bundle. It consisted of a cylindrical test section, a coolant accumulator, an entrained liquid separation tank, an external pipe downcomer for the gravity reflood tests, a steam boiler for back-pressure regulation, and the required piping and valves. As the facility is described in detail in a FLECHT-SEASET data report [5], only the major components are briefed here. The overall facility layout is presented in Figure 1.

A cylindrical test section was composed of a lower plenum, a low mass housing, and an upper plenum. The low-mass housing contained the heater rod bundle consisting of 161 heater rods and 16 thimble rods which were placed with a square lattice array similar to the 17x17 Westinghouse fuel bundle design. It was a cylindrical vessel with a 19.37 cm (7.625 in) inside diameter by 0.477 cm (0.188 in) wall, constructed of SA-312, type 304 stainless steel. The minimum wall thickness was chosen so that the housing would absorb or release the minimum amount of heat to the rod bundle. The length of the heated part was 3.66 m (12 ft).

The cross-sectional view of the heater rod bundle is shown in Figure 2 with the corresponding instrumentation groups. The bundle comprised 161 heater rods (68 instrumented), 4 instrumented thimbles, 12 steam probes, 8 solid triangular fillers, and 8 spacer grids. A Kanthal heater coil imbedded in boron nitride was used to heat the rods which had stainless steel cladding. The outside diameter and the wall thickness of the heater rod were 9.5 mm (0.374 in) and 0.64 mm (0.025 in), respectively. Rod clad temperatures were measured by Type K thermocouples which were welded at the inner surface of the stainless steel cladding. The rod-to-rod pitch was 12.6 mm (0.496 in) and each rod had a cosine axial power profile shown in Figure 3. The triangular fillers were welded to the grids to maintain the proper grid locations. The fillers served also to reduce the amount of excess flow area.

The low mass housing was joined to the upper and lower plenums by flange connections. There were an upper and a lower extension in the upper and lower plenums. The entrained liquid collected in the upper plenum was prevented from falling back into the bundle by the upper extension. The lower extension had a role of distributing the injected coolant uniformly.

After passing through the upper plenum baffle pipe which helped improve liquid carryout separation, the exhaust vapor was directed to the entrainment separator through the exhaust line piping. The entrainment separator was a standard liquid-vapor separator having a large volume to minimize the housing pressure oscillations.

Before a test was conducted, auxiliary components such as the carryover vessel, entrainment separator, separator drain tank, test section upper plenum, and test section outlet piping were heated to slightly above the saturation temperature by clamp-on strip heaters. The test section, carryover vessel, and exhaust line components were pressurized to the desired pressure by a steam boiler. When the coolant supplied by the gas-pressurized accumulator reached the bottom of the heated part of the heater rod bundle, it was circulated to assure the water in the lower plenum were at the specified temperature. Power was then supplied to the test bundle

and the rods were heated up. When the bundle temperature from any two designated thermocouples reached the preset value, flooding and power decay were initiated automatically.

The instrumentation of the FLECHT-SEASET facility was quite extensive, including 205 heater rod thermocouples, 12 differential pressure cells positioned 0.3048 m (1 ft) apart along the axial direction of the heated section, 12 steam probes, and inlet and outlet flow meters.

Among the numerous tests performed at the FLECHT-SEASET test facility, 8 forced reflood tests were selected for RELAP5/MOD3.3 assessment. Their test conditions are presented in Table 1. These tests were also used for the assessment of TRACE [6]. Flooding rates varied from 2.1 to 15.5 cm/s (0.81 to 6.10 in/sec) with the upper plenum pressure varying from 0.13 to 0.41 MPa (19 to 60 psia) and the coolant temperature varying from 306 to 408 K (91 to 257 °F). All 8 tests had the same initial rod power of 2.3 kW/m (0.7 kW/ft) at the peak location and they had a uniform radial power distribution. Note that the test conditions listed in Table 1 are nominal values and the actual values varied to some extent with time.

#### 2.2 <u>RELAP5 Model</u>

For the RELAP5 simulations, only the bundle test section was modeled in a detailed manner. Other parts of the test facility, such as the coolant injection system and the bundle flow exhaust system, were greatly simplified. The facility was represented by four fluid components (Figure 4). Two time-dependent volume (TMDPVOL) components represent the coolant injection system and the flow exhaust system. One single-volume (SNGLVOL) component represents the inlet pipe and one pipe (PIPE) component does the heated section of the cylindrical housing. The heated section was modeled using 49 equal length (7.62 cm (3.0 in)) axial nodes. Note that the top and bottom elevations of the PIPE component were extended to 3.81 cm (1.5 in) above and below the heated section to make the center of the first and last nodes meet with the measurement locations. With this noding, it was possible to give the form losses at the spacer grids to the locations close to the real bottom elevations of spacer grids. It was also possible to obtain the heater rod clad temperatures at the exact locations of measurement. The form losses at the spacer grids were modeled by specifying the pressure loss coefficient to 1.20. The same loss coefficient was used also in the assessment of TRACE [6].

Five heat structures were located in the PIPE component to model the active heater rods, the housing wall, the thimbles, the filler rods, and the failed heater rods. The number of failed heater rods was 4 for Test 31302 and 2 for the others. The length of the first and last axial nodes is one half of other ones so that the center of all other axial nodes matched with the center of the hydraulic nodes in the PIPE component. Heater rods were divided into 7 radial nodes to model the Kanthal heater coil insulated with boron nitride that is encased with stainless steel cladding.

The electrical power was provided as a time-dependent table. The coolant inlet velocity as a function of time was given to a time-dependent junction (TMDPJUN) component. The inlet coolant temperature and the upper plenum pressure were given to the TMDPVOL components representing the coolant injection system and the flow exhaust system, respectively. The initial temperatures of heater rods and the housing wall at various axial locations were determined from a linear interpolation of measurement data. The initial temperature of the failed rods and fillers were assumed to be equal to that of the housing wall.

The calculations were initiated at the time of flooding. The user's guidelines in [7] were followed tightly to select appropriate inputs for the time step control, the volume-related options, and junction-related options. The maximum time step was set to 0.1 s in all calculations though the

time step was reduced automatically to satisfy the Courant limit or avoid water property failures.

We believe the modeling approach described above has several arguable features that should be discussed more.

First, compared to other previous FLECHT-SEASET calculations, a more detailed axial noding for the bundle test section was used in this study. At first, we tried to use 20 equal length nodes for the test section just like other previous assessments [3], [4], but the predicted bottom quench front in that case showed stepwise increases during the entire transient (Figure 5). In addition, with 20 equal length nodes, we can not compare predicted temperatures directly to measurements directly, as mentioned earlier.

Second, the entire test section was modeled as a single channel. Even though the tests we selected for this assessment had a uniform radial power distribution, the clad temperature measurements at an elevation have some distribution. Per Westinghouse [8], the distribution can be attributed to the individual heater rod resistance differences, manufacturing tolerances on materials, dimension variations, initial temperature differences, the radial location of a heater rod relative to the unheated guide thimbles, and sub-channel-to-sub-channel flow variations. It is nearly impossible to correctly take all these factors into account in a RELAP5 calculation because the code is basically a one-dimensional code. Certainly, some previous RELAP5 calculations used a two channel modeling for the test section, but we believe it may introduce another kind of uncertainties related to the use of cross-flow junctions. A single channel modeling for the test section has an effect of averaging the radial distribution of clad temperatures. Thus, the arithmetic average values of measured PCTs and quench times were used in the quantitative comparison to predictions in this study.

Third, we did not take into account the radiation heat transfer between the housing wall and peripheral rods. According to Westinghouse [8], the radiation heat transfer from the housing can be restricted to the outer-most rods. Thus, no readings from those thermocouples located in the outer-most row of rods were used in the quantitative comparison to predictions.

Appendix to this report presents the input deck for Test 31805 and only the inputs for test specific initial and boundary conditions can vary test by test.

#### 2.3 Assessment Results

FLECHT-SEASET test simulations using RELAP5/MOD3.3 (patch 03) were started at the time of reflood corresponding to the initial injection of cold water into the test section. To assess the predictive capability of the code, detailed comparisons between measurement data and RELAP5 calculations were made for each test. The compared parameters are as follows:

- Heater rod clad temperatures at 8 different elevations; 0.610 m (2 ft), 1.219 m (4 ft), 1.829 m (6 ft), 1.981 m (6.5 ft), 2.438 m (8 ft), 3.048 m (10 ft), 3.353 m (11 ft), and 3.505 m (11.5 ft)
- Vapor temperatures at 1.829 m (6 ft) and 3.048 m (10 ft) elevations
- Quench front elevation as a function of time
- Differential pressure for the entire 3.658 m (12 ft) core
- Differential pressure between 1.829 m (6 ft) and 2.134 m (7 ft) elevations
- Differential pressure between 3.048 m (10 ft) and 3.353 m (11 ft) elevations
- HTCs at 5 different elevations; 1.219 m (4 ft), 1.829 m (6 ft), 1.981 m (6.5 ft), 2.438 m (8 ft), and 3.048 m

FLECHT-SEASET had more than one rod temperature measurements at an elevation. For example, at 1.8 m (6 ft) elevation, there existed 21 thermocouples for the rod clad temperatures. In the comparison to the predicted one, all the valid measurement data were used.

Not only the measured but also the predicted quench front elevation was evaluated from the heater rod clad temperatures. If the rod clad temperature drop during a time step (or the measurement interval) at an elevation was lower than a specified value, the elevation was regarded as quenched. In other words, the rod clad temperature should satisfy the following relation at the quench time.

 $\frac{dT}{dt}\Big|_{t=\text{Quench Time}} = \frac{T_{t+3\cdot\Delta t} - T_{t+2\cdot\Delta t}}{t_{t+3\cdot\Delta t} - t_{t+2\cdot\Delta t}} \ \langle -X$ 

(1)

where T and t are the rod clad temperature and time, respectively and  $\Delta t$  is the time step or the measurement interval. For X, 27.8 K/s (50 °F/sec) was tried first, but in a few cases having a very gradual temperature decrease, 16.7 K/s (30 °F/sec) was used also. The measured quench time at an elevation was obtained by averaging the quench times which were determined using all the valid thermocouple readings at that elevation.

Both the measured and the predicted HTCs are defined as the heat flux divided by the difference between the clad and the saturation temperatures. The measured HTCs were already derived by the USNRC [6]. They used a program called IHCP1D with the 9 second average option to remove high oscillations. For consistency, the same time averaging process was also applied to get the predicted HTCs. For the measured HTCs, three HTCs (minimum, maximum, and average) derived from multiple rod clad temperature measurements are presented. The average one was computed as the arithmetic average of HTCs from all instrumented rods having valid clad temperature measurements at a given elevation.

Comparisons of predicted results to the measurement data are made in Figure 6 through Figure 157. The figures contain also the results using a modified version, but only the results with the original version are discussed in this chapter. The results with a modified version will be discussed in Chapter 3 emphasizing the effect of modifications.

#### 2.3.1 Test 31805

Test 31805 was a 2.1 cm/s (0.81 in/sec) reflood test at 0.28 MPa (40 psia). Inlet liquid temperature was 324 K (124 °F). Of all the tests simulated in this study, this test had the lowest flooding rate.

Heater rod clad temperatures predicted by RELAP5 at various elevations are compared to the measured data in Figure 6 through Figure 13. The maximum predicted PCT is 1359 K at 1.981 m (6.5 ft) elevation while the measured PCTs at the same elevation are in the range of 1353~1509 K. Though the prediction locates between the maximum and minimum measured values, the code is believed to under-predict the PCT because the prediction is 92 K lower than the arithmetic average of measurements. The tendency of under-predicting PCTs becomes stronger at higher elevations. The predicted PCTs at 2.438 m (8 ft), 3.048 m (10 ft), and 3.353 m (11 ft) elevations are even lower than the lowest measurements at 3.353 m (11 ft) and 3.505 m (11.5 ft) elevations are smaller than those at 2.438 m (8 ft) or 3.048 m (10 ft) elevations.

Vapor temperatures predicted are compared to the measured data in Figure 14 and Figure 15. The vapor temperature at 1.829 m (6 ft) was under-predicted much compared to the measurements. The rapid increase before ~30 seconds and the re-heating after ~60 seconds were not simulated well. The predicted vapor temperature at 3.048 m (10 ft) increased too fast as soon as the transient began and the peak temperature was predicted to occur too early.

The predicted and measured quench fronts are compared in Figure 16. The final quenches were predicted to occur much earlier than the measurements at all elevations except a bottom region (at and below 1.219 m (4 ft)). The difference between the predicted quench time and the arithmetic average of measured quench times (predicted – measured) varies from -177 to -7 seconds and the maximum absolute deviation occurred at 3.505 m (11.5 ft).

The predicted and measured differential pressures for the entire core region, a mid-level (1.829 to 2.134 m (6 to 7 ft)), and a top region (3.048 to 3.353 m (10 to 11 ft)) are compared in Figure 17, Figure 18, and Figure 19, respectively. The predicted value for the entire core is higher than the measurement data from the beginning of transient and the deviation is about 2~3 kPa (0.29~0.44 psi) after ~100 seconds. It means that the lower part of the test section was filled up with liquid too fast and the difference of collapsed liquid level was maintained until the end of transient as the differential pressure is essentially equivalent to the hydrostatic head. The predicted differential pressure for the mid-level becomes higher than the measurement at ~250 seconds. For the top region, the differential pressure was predicted relatively well until a quantitative comparison becomes doubtful due to large oscillations in the prediction.

The predicted and measured HTCs at five different elevations are compared in Figure 20 through Figure 24. Excluding the early steam cooling only period, the HTCs at all elevations were significantly over-predicted until the quench time. Generally speaking, the reflood HTCs increases abruptly as the transition boiling regime begins. Thus, the figures indicate that the code over-predicted much the HTCs in the film boiling regime.

#### 2.3.2 Test 31504

Test 31504 was a 2.4 cm/s (0.97 in/sec) reflood test at 0.28 MPa (40 psia). Inlet liquid temperature was 324 K (124 °F). The test had very similar initial and boundary conditions to those of Test 31805 but the flooding rate was slightly higher than that of Test 31805.

Heater rod clad temperatures predicted by RELAP5 at various elevations are compared to the measured data in Figure 25 through Figure 32. The maximum PCT predicted by the code is 1319 K at 1.981 m (6.5 ft) elevation while the measured PCTs at the same elevation are in the range of 1322-1423 K. The predicted maximum PCT is 42 K lower than the arithmetic average of measurements. The tendency of under-predicting PCTs becomes stronger at higher elevations. The predicted PCTs at 1.981 m (6.5 ft), 2.438 m (8 ft), and 3.048 m (10 ft) elevations are even lower than the lowest measured PCT at each elevation. The percent differences between the predicted PCTs and the measurements at 3.353 m (11 ft) and 3.505 m (11.5 ft) elevations are much smaller than at 2.438 m (8 ft) or 3.048 m (10 ft) elevations.

Vapor temperatures predicted are compared to the measured data in Figure 33 and Figure 34. The vapor temperature at 1.829 m (6 ft) was under-predicted much compared to the measurements. The rapid increase before ~20 seconds and the re-heating after ~50 seconds were not simulated well. The predicted vapor temperature at 3.048 m (10 ft) increased too fast as soon as the transient began and the peak temperature was predicted to occur too early.

The predicted and measured quench fronts are compared in Figure 35. The final quenches were predicted to occur much earlier than the measurements at all elevations except a bottom region (at and below 1.219 m (4 ft)). The difference between the predicted quench time and the arithmetic average of measured quench times (predicted – measured) varies from -149 to -5 seconds and the maximum absolute deviation occurred at 3.353 m (11 ft).

The predicted and measured differential pressures for the entire core region, a mid-level (1.829 to 2.134 m (6 to 7 ft)), and a top region (3.048 to 3.353 m (10 to 11 ft)) are compared in Figure 36, Figure 37, and Figure 38, respectively. The predicted value for the entire core is higher than the measurement data from the beginning of transient and the deviation is about 4~5 kPa (0.58~0.73 psi) after ~60 seconds. It means that the lower part of the test section was filled up with liquid too fast and the difference of collapsed liquid level was maintained until the end of transient. The predicted differential pressure for the mid-level becomes higher than the measurement after ~200 seconds. For the top region, the differential pressure was over-predicted early in the transient, but the deviation does not look large.

The predicted and measured HTCs at five different elevations are compared in Figure 39 through Figure 43. Excluding the early steam cooling only period, the HTCs at all elevations were significantly over-predicted until the quench time, which means that the code over-predicted much the HTCs in the film boiling regime.

#### 2.3.3 Test 31203

Test 31203 was a 3.8 cm/s (1.51 in/sec) reflood test at 0.28 MPa (40 psia). Inlet liquid temperature was 52 K (126 °F). The test has very similar initial and boundary conditions to those of Test 31805 or Test 31504 but the flooding rate was a little higher than that of Test 31504.

Heater rod clad temperatures predicted by RELAP5 at various elevations are compared to the measured data in Figure 44 through Figure 51. The maximum PCT predicted by the code is 1224 K at 1.829 m (6.0 ft) elevation while the measured PCTs at the same elevation are in the range of 1206-1297 K. Note that the highest average of the measured PCTs was obtained at 1.981 m (6.5 ft) elevation. The predicted maximum PCT is 45 K lower than the arithmetic average of measurements at that elevation. The predicted PCTs at 1.981 m (6.5 ft) and 2.438 m (8 ft) are even lower than the lowest measured PCT at each elevation. On the contrary, the predicted PCTs at 3.353 m (11 ft) and 3.505 m (11.5 ft) are higher than the arithmetic average of measurements at each elevation.

Vapor temperatures predicted are compared to the measured data in Figure 52 and Figure 53. The vapor temperature at 1.829 m (6 ft) was under-predicted much compared to the measurements. The rapid increase before ~10 seconds and the re-heating after ~30 seconds were not simulated well. The predicted vapor temperature at 3.048 m (10 ft) increased too fast as soon as the transient began and the peak temperature was predicted to occur too early.

The predicted and measured quench fronts are compared in Figure 54. The final quenches were predicted to occur much earlier than the measurements at all elevations except the lowest elevation (0.610 m (2 ft)). The difference between the predicted quench time and the arithmetic average of measured quench times (predicted – measured) varies from -148 to -6 seconds and the maximum absolute deviation occurred at 3.048 m (10 ft).

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The predicted and measured differential pressures for the entire core region, a mid-level (1.829 to 2.134 m (6 to 7 ft)), and a top region (3.048 to 3.353 m (10 to 11 ft)) are compared in Figure 55, Figure 56, and Figure 57, respectively. The predicted value for the entire core is about 3~4 kPa (0.44~0.58 psi) higher than the data after ~20 seconds. The predicted differential pressure for the mid-level becomes higher than the measurement after ~130 seconds. For the top region, the differential pressure was over-predicted early in the transient.

The predicted and measured HTCs at five different elevations are compared in Figure 58 through Figure 62. Excluding the early steam cooling only period, the HTCs at all elevations were significantly over-predicted until the quench time, which means that the code over-predicted much the HTCs in the film boiling regime.

#### 2.3.4 Test 31302

Test 31302 was a 7.7 cm/s (3.01 in/sec) reflood test at 0.28 MPa (40 psia). Inlet liquid temperature was 325 K (126 °F). The test has very similar initial and boundary conditions to those of Test 31203 but the flooding rate was about two times higher than that of Test 31203.

Heater rod clad temperatures predicted by RELAP5 at various elevations are compared to the measured data in Figure 63 through Figure 70. The maximum PCT predicted by the code is 1159 K at 1.829 m (6.0 ft) elevation while the measured PCTs at the same elevation are in the range of 1120~1191 K. The predicted maximum PCT is only 2 K lower than the arithmetic average of measurements at the same elevation. Only the PCTs at 3.353 m (11 ft) were definitely over-predicted.

Vapor temperatures predicted are compared to the measured data in Figure 71 and Figure 72. The vapor temperature at 1.829 m (6 ft) was predicted relatively well though the maximum temperature occurred a little earlier than the measurements. The vapor temperatures at 3.048 m (10 ft) were 100~200 K over-predicted.

The predicted and measured quench fronts are compared in Figure 73. The final quenches were predicted to occur much earlier than the measurements at all elevations. The difference between the predicted quench time and the arithmetic average of measured quench times (predicted – measured) varies from -120 to -7 seconds and the maximum absolute deviation occurred at 3.048 m (10 ft).

The predicted and measured differential pressures for the entire core region, a mid-level (1.829 to 2.134 m (6 to 7 ft)), and a top region (3.048 to 3.353 m (10 to 11 ft)) are compared in Figure 74, Figure 75, and Figure 76, respectively. The differential pressure for the entire core region was predicted relatively well before ~100 seconds, but it was over-predicted by 2~4 kPa (0.29~0.58 psi) after that time. The predicted differential pressure for the mid-level becomes higher than the measurement after ~70 seconds. For the top region, the predicted differential pressure looks higher than the measurement after ~120 seconds when the prediction starts to oscillate severely. The times when the predicted differential pressures for the mid-level and a top region become higher than the measurements are close to the times when the predicted rod clad temperatures at those elevations start to drop fast (Figure 65 and Figure 68). Thus, we believe the over-predicted differential pressure (or collapsed liquid level) is one of the reasons why the quench times of this test were predicted to occur much earlier than the experiment.

The predicted and measured HTCs at five different elevations are compared in Figure 77 through Figure 81. The HTCs in the film boiling regime were a little over-predicted at all

elevations, but the deviations are not as large as in the cases of lower flooding rate tests (Test 31805, Test 31504, and Test 31203).

#### 2.3.5 Test 31701

Test 31701 was a 15.5 cm/s (6.10 in/sec) reflood test at 0.28 MPa (40 psia). Inlet liquid temperature was 326 K (127 °F). Of all the tests simulated in this study, this test had the highest flooding rate.

Heater rod clad temperatures predicted by RELAP5 at various elevations are compared to the measured data in Figure 82 through Figure 89. The maximum PCT predicted by the code is 1135 K at 1.829 m (6.0 ft) elevation while the measured PCTs at the same elevation are in the range of 1114~1175 K. The predicted maximum PCT is 17 K lower than the arithmetic average of measurements at the same elevation. Just like Test 31302, the PCT predictions are relatively excellent compared to lower flooding rate tests (Test 31805, 31504, and 31203).

Vapor temperatures predicted are compared to the measured data in Figure 90 and Figure 91. The vapor temperature at 1.829 m (6 ft) was predicted relatively well though the heat up early in the transient was predicted to stop a few seconds earlier than the measurements. The vapor temperature at 3.048 m (10 ft) was over-predicted by 50~150 K.

The predicted and measured quench fronts are compared in Figure 92. The final quenches were predicted to occur a little earlier than the measurements at all elevations. The difference between the predicted quench time and the arithmetic average of measured quench times (predicted – measured) varies from -41 to -3 seconds and the maximum absolute deviation occurred at 3.048 m (10 ft). Note that the measured quench times show a wide distribution in a top region at and above 3.048 m (10 ft).

The predicted and measured differential pressures for the entire core region, a mid-level (1.829 to 2.134 m (6 to 7 ft)), and a top region (3.048 to 3.353 m (10 to 11 ft)) are compared in Figure 93, Figure 94, and Figure 95, respectively. The predicted value for the entire core is higher than the measurement data from the beginning of transient and the deviation is about 3~5 kPa (0.44~0.73 psi) after ~10 seconds. The predicted differential pressure for the mid-level becomes higher than the measurement after ~25 seconds. The top region was also predicted to be filled with liquid earlier than the measurement.

The predicted and measured HTCs at five different elevations are compared in Figure 96 through Figure 100. The code tends to over-predict the HTCs, especially in a lower part of the core. We believe that the rapid filling of the core with liquid, which is observed in Figure 93, Figure 94, or Figure 95, is one of the main reasons for the over-prediction of HTCs.

#### 2.3.6 Test 31108

Test 31108 was a 7.9 cm/s (3.11 in/sec) reflood test at 0.13 MPa (40 psia). Inlet liquid temperature was 306 K (91 °F). The test has similar initial and boundary conditions to those of Test 31302 but the upper plenum pressure is about one half of that of Test 31302.

Heater rod clad temperatures predicted by RELAP5 at various elevations are compared to the measured data in Figure 101 through Figure 108. The maximum PCT predicted by the code is 1145 K at 1.829 m (6.0 ft) elevation while the measured PCTs at the same elevation are in the range of 1126~1200 K. The predicted maximum PCT is 23 K lower than the arithmetic average

of measurements at the same elevation. The predicted PCTs are slightly lower than the arithmetic average of measurements at all elevations except 3.048 m (10 ft) and 3.353 m (11 ft) where PCTs were definitely over-predicted.

Vapor temperatures predicted are compared to the measured data in Figure 109 and Figure 110. The vapor temperature at 1.829 m (6 ft) was predicted relatively well though the maximum temperature occurred a little earlier than the measurements. The vapor temperatures at 3.048 m (10 ft) were 100~200 K over-predicted.

The predicted and measured quench fronts are compared in Figure 111. The final quenches were predicted to occur much earlier than the measurements at all elevations except 3.505 m (11.5 ft) elevation. The difference between the predicted quench time and the arithmetic average of measured quench times (predicted – measured) varies from -162 to +13 seconds and the maximum absolute deviation occurred at 3.048 m (10 ft).

The predicted and measured differential pressures for the entire core region, a mid-level (1.829 to 2.134 m (6 to 7 ft)), and a top region (3.048 to 3.353 m (10 to 11 ft)) are compared in Figure 112, Figure 113, and Figure 114, respectively. The differential pressure for the entire core region is higher than the measurement data from the beginning of transient and the deviation is  $3\sim7$  kPa (0.44 $\sim1.02$  psi) after  $\sim10$  seconds. The predicted differential pressure for the mid-level becomes higher than the measurement after  $\sim90$  seconds. For the top region, the predicted differential pressure is maintained a little higher than the measurement for  $\sim150$  seconds and oscillates highly after that time. The times when the predicted differential pressures for the mid-level and the top region become higher than the measurements are close to the times when the predicted rod clad temperatures at those elevations start to drop fast (Figure 103 and Figure 106). Thus, we believe the over-predicted differential pressure (or collapsed liquid level) is one of the reasons why the quench times of this test were predicted to occur much earlier than the experiment.

The predicted and measured HTCs at five different elevations are compared in Figure 115 through Figure 119. The film boiling HTCs were a little over-predicted in a lower part of the core while they were a little under-predicted in a top region.

#### 2.3.7 Test 32013

Test 32013 was a 2.6 cm/s (1.04 in/sec) reflood test at 0.41 MPa (60 psia). Inlet liquid temperature was 339 K (150 °F). The test has similar initial and boundary conditions to those of Test 31504 but the upper plenum pressure is 0.14 MPa (20 psi) higher that of Test 31504.

Heater rod clad temperatures predicted by RELAP5 at various elevations are compared to the measured data in Figure 120 through Figure 127. The maximum PCT predicted by the code is 1326 K at 1.981 m (6.5 ft) elevation while the measured PCTs at the same elevation are in the range of 1306~1431 K. The predicted maximum PCT is 55 K lower than the arithmetic average of measurements. The tendency of under-predicting PCTs becomes stronger at higher elevations. The predicted PCTs at 2.438 m (8 ft) and 3.048 m (10 ft) elevations are even lower than the lowest measured PCT at each elevation. The percent differences between the predicted PCTs and the measurements at 3.353 m (11 ft) and 3.505 m (11.5 ft) elevations are smaller than at 2.438 m (8 ft) and 3.048 m (10 ft) elevations.

Vapor temperatures predicted are compared to the measured data in Figure 128 and Figure 129. The vapor temperature at 1.829 m (6 ft) was under-predicted much compared to the

measurements. The rapid increase before ~20 seconds and the re-heating after ~50 seconds were not simulated well. The predicted vapor temperature at 3.048 m (10 ft) increased too fast as soon as the transient began and the peak temperature was predicted to occur too early.

The predicted and measured quench fronts are compared in Figure 130. The final quenches were predicted to occur much earlier than the measurements at all elevations except a bottom region (at and below 1.219 m (4 ft)). The difference between the predicted quench time and the arithmetic average of measured quench times (predicted – measured) varies from -101 to -2 seconds and the maximum absolute deviation occurred at 3.505 m (11.5 ft).

The predicted and measured differential pressures for the entire core region, a mid-level (1.829 to 2.134 m (6 to 7 ft)), and a top region (3.048 to 3.353 m (10 to 11 ft)) are compared in Figure 131, Figure 132, and Figure 133, respectively. The predicted value for the entire core was predicted quite well. However, the predicted differential pressure for the mid-level becomes higher than the measurement after ~180 seconds. For the top region, the differential pressure was over-predicted early in the transient, but the difference does not look large.

The predicted and measured HTCs at five different elevations are compared in Figure 134 through Figure 138. Excluding the early steam cooling only period, the HTCs at all elevations were over-predicted much until the quench time, which means that the code over-predicted the HTCs in the film boiling regime.

#### 2.3.8 Test 32114

Test 32114 was a 2.5~3.1 cm/s (1.00~1.22 in/sec) reflood test at 0.28 MPa (40 psia). Inlet liquid temperature was 408 K (257 °F). Of all the tests simulated in this study, this test had the smallest inlet subcooling of 5 K (10 °F). Other seven tests had similar inlet subcooling of 74~79 K (134~143 °F).

Heater rod clad temperatures predicted by RELAP5 at various elevations are compared to the measured data in Figure 139 through Figure 146. The maximum PCT predicted by the code is 1320 K at 1.981 m (6.5 ft) elevation while the measured PCTs at the same elevation are in the range of 1292~1443 K. The predicted maximum PCT is 67 K lower than the arithmetic average of measurements. Compared to the arithmetic average of measurement at each elevation, The PCTs at 0.610~3.048 m (2~10 ft) were under-predicted while those at higher elevations were over-predicted.

Vapor temperatures predicted are compared to the measured data in Figure 147 and Figure 148. The vapor temperature at 1.829 m (6 ft) was significantly under-predicted compared to the measurements. The predicted vapor temperature at 3.048 m (10 ft) increased too fast while the peak temperature predicted is not much different from the measured values.

The predicted and measured quench fronts are compared in Figure 149. The final quenches were predicted to occur much earlier than the measurements at all elevations, even at the elevations where PCTs were over-predicted. The difference between the predicted quench time and the arithmetic average of measured quench times (predicted – measured) varies from -113 to -15 seconds and the maximum absolute deviation occurred at 1.981 m (6.5 ft).

The predicted and measured differential pressures for the entire core region, a mid-level (1.829 to 2.134 m (6 to 7 ft)), and a top region (3.048 to 3.353 m (10 to 11 ft)) are compared in Figure 150, Figure 151, and Figure 152, respectively. The predicted value for the entire core is higher
than the measurement from the beginning of transient and the deviation is 3~4 kPa (0.44~0.58 psi) after ~40 seconds. The predicted differential pressures for the mid-level and top region were predicted relatively well until large oscillations were observed.

The predicted and measured HTCs at five different elevations are compared in Figure 153 through Figure 157. Excluding the early steam cooling only period, the film boiling HTCs at all elevations were over-predicted.

#### 2.3.9 All 8 Tests

The percent deviation between RELAP5/MOD3.3 predicted PCTs and measurements at 8 different elevations are compared for all 8 tests in Figure 158. As shown in the figure, RELAP5/MOD3.3 predicted the measured PCTs within  $\pm 10\%$  deviation. The code was well able to predict PCTs at lower elevations (up to 1.219 m (4 ft)) while it under-predicted those at middle elevations (1.829~2.438 m (6~8 ft)). The PCTs were less under-predicted at upper elevations at or above 3.048 m (10 ft), and even over-predicted for a few tests. This is understandable because RELAP5 does not have a spacer grid model. In other words, the under-prediction of PCTs at higher elevations was compensated to some extent by the lack of a spacer grid model. The under-prediction of PCTs at middle elevations was more severe for the tests having relatively lower flooding rates ( $\leq$  3.8 cm/s (1.51 in/sec)) and less severe for the tests having higher flooding rates ( $\geq$  7.7 cm/s (3.01 in/sec)). The predictive capability did not show any consistent or significant variance depending on other test conditions such as upper plenum pressure or coolant inlet subcooling. The RMS error in PCT predictions corresponding to Figure 158 is 48.3 K.

The code under-predicted the peak vapor temperatures at 1.829 m (6 ft) for most tests. The code over-predicted the peak vapor temperature at this elevation for only two tests (Test 31302 and Test 31108) which had flooding rates of 7.7 cm/s (3.01 in/sec) and 7.9 cm/s (3.11 in/sec), respectively. The largest deviation was 186 K under-prediction in Test 31504. The code had a tendency of under-predicting the peak vapor temperature at 3.048 m (10 ft) for the test having relatively lower flooding rates (Test 31805, 31504, 32013, and 32114), but the code over-predicted it for the tests having higher flooding rates (Test 31302, Test 31701, and Test 31108). The largest deviation was 165 K over-prediction for Test 31108. Note that this quantitative analysis for the vapor temperature predictions can not be regarded as sufficiently accurate since there only a few measurements are available at a specified elevation.

Though the code predicted PCTs relatively well, it could not do quench times. The differences between the predicted and measured quench times at 8 different elevations are compared for all 8 tests in Figure 159. As shown in the figure, rod quenches were predicted to occur too early for almost all elevations. In most tests, the differences between the predicted and measured quench times increased with elevations, reaching their maximum at around 3.048 m (10 ft) elevation. The differences were smaller in a test having a higher flooding rate, but we should also consider that the differences may be bigger when the real quench times are longer. Comparing Test 31108 to Test 31302 or Test 32013 to Test 31504, it can be seen that the code tends to predict better the quench times of a test having a higher upper plenum pressure. No clear conclusions can be made for the effect of inlet subcooling on the predictive capability for quench times. The RMS error in quench time predictions deduced from Figure 159 is 85.9 seconds.

The code over-predicted the overall pressure drop for the entire bundle in most tests. The differences between the predicted and measured pressure drops were well below the

measurement uncertainty of 2.66 kPa (0.39 psi) only for Test 32013. In other cases, the deviation varied from  $3 \sim 7$  kPa (0.44 $\sim$ 1.02 psi) though it was also a function of time. The pressure drop for a mid-level region (1.829 to 2.134 m (6 to 7 ft)) was also over-predicted for most tests after some time when the bottom quench front approached close to the region. The same tendency was observed for the prediction of the pressure drop for a top region (3.048 to 3.353 m (10 to 11 ft)).

The code tended to over-predict the HTCs at all elevation for all tests after a well predicted, short period of vapor only cooling. The over-prediction of HTCs in a top region is smaller than that in a middle region due to the lack of a spacer grid model.

In summary, RELAP5/MOD3.3 predicted the PCTs of the selected FLECHT-SEASET tests relatively well, within  $\pm 10\%$  deviation. However, the code predicted quench times to occur much earlier than the measurement data. As the code over-predicted significantly the film boiling HTCs in most cases, the wall-to-fluid heat transfer models in that heat transfer regime should be investigated intensively. The overall tendency of over-predicting differential pressure for the entire core (or collapsed liquid level) indicates that the interfacial or wall drag models of the code may also have some deficiencies. By the way, the predicted differential pressures can alter if the wall-to-fluid heat transfer models are changed. Thus, we focus on the wall-to-fluid heat transfer models first to improve the predictive capability for quench times. An intensive review and improvements of drag models is regarded as a future work.

# 3. MODIFICATION OF FILM BOILING WALL-TO-FLUID HEAT TRANSFER MODELS

To improve the predictive capability mainly for quench times, the film boiling wall-to-fluid heat transfer models of RELAP5/MOD3.3, which are activated when the reflood option is invoked, were modified. The current models of RELAP5/MOD3.3 are discussed in brief first, and the modified models are described thereafter.

### 3.1 Brief of Current Models

Differently from other best-estimate codes like TRACE [9], COBRA/TRAC [10], or COBRA-TF [8], RELAP5/MOD3.3 does not draw a line between the inverted annular film boiling (IAFB) heat transfer regime and the dispersed flow film boiling (DFFB) heat transfer regime. It uses the same wall-to-fluid heat transfer models for the entire film boiling regime.

The code takes the maximum of HTCs calculated by three different correlations as the film boiling HTC to liquid.

$$h_{wf} = max(h_{Br}, h_{CT}, h_{FR})$$

(2)

where  $h_{Br}$ ,  $h_{CT}$ , and  $h_{Fr}$  are the HTCs calculated using the Bromley correlation, an empirical CATHARE correlation, and the Forslund-Rohsenow correlation, respectively.

The film boiling wall-to-vapor HTC is returned from the DITTUS subroutine. Here, in the default model,  $h_{Ditt}$  is obtained by taking the maximum of a turbulent convective HTC calculated using the Dittus-Boelter correlation, a HTC corresponding to a constant laminar convective Nusselt number of 4.36, and a natural convective HTC calculated using the Churchill-Chu correlation. Note that the mass flux in the Dittus-Boelter correlation contains not only the vapor mass flux but also a portion of the liquid mass flux ( $G_f \cdot \rho_f / \rho_g$ ) in two-phase flow cases. Then,  $h_{Ditt}$  is void fraction ramped so that it goes to zero as the void fraction goes to zero.

 $h_{wg} = h_{Ditt} \cdot \alpha_g$ 

(3)

RELAP5/MOD3.3 takes into account the wall-to-liquid radiation heat transfer in the film boiling regime and adds it to the wall-to-liquid convective heat transfer. The radiation heat transfer model of the code is based on that developed by Sun [11]. In this model, the radiation heat flux is proportional to the wall-to-liquid grey body factor given by

1

F.	1	(4)
<sup>1</sup> wf		(+)
	$R_{2} + \frac{K_{3}}{1 + \frac{K_{3}}{2} + \frac{K_{3}}{2}}$	
	$\mathbf{R}_{1}$ $\mathbf{R}_{2}$	
	(	

where R terms are given as a function of the emissivity of wall, vapor, and liquid. Constant values of 0.02 and 0.9 are assumed for the vapor emissivity and the wall emissivity, respectively. The liquid emissivity depends on the liquid absorption coefficient,  $a_f$  which is defined as

$$a_f = \frac{1.11 \cdot \alpha_f}{d} \tag{5}$$

where  $\alpha_f$  is the liquid volume fraction and d is the liquid droplet diameter which is given as the minimum of the following two expressions.

$$d_{max} = \sqrt{1 - \alpha_g} \cdot D_h$$

$$d_{We=7.5} = \frac{7.5 \cdot \sigma}{\rho_g \cdot v_r^2}$$
(6)
(7)

More detailed information on the film boiling wall-to-fluid heat transfer models used in RELAP5/MOD3.3 can be found in the code manual [7].

#### 3.2 Modified Models

In the modified version, the film boiling heat transfer regime is divided into three separate regimes. The IAFB and DFFB regimes are assumed to exist if void fractions are under 0.6 and over 0.9, respectively. Appropriate wall-to-fluid heat transfer correlations, described later, are used for each regime, and the HTCs in the region having void fractions between 0.6 and 0.9 are obtained by spline interpolation of the IAFB and DFFB HTCs:

$$h_{wf} = y \cdot h_{wf,IA} + (1 - y) \cdot h_{wf,DF}$$

$$h_{wg} = y \cdot h_{wg,IA} + (1 - y) \cdot h_{wg,DF}$$
(8)
(9)

where the weighting function, y is calculated from

$$y = x \cdot (2 - x)$$
 with  $x = \frac{0.9 - \alpha_g}{0.9 - 0.6}$  (10)

Note that the void fraction limits defining each film boiling regime and the interpolation method are the same as used in TRACE [9].

For the IAFB wall-to-liquid HTC, the CATHARE correlation was selected among the three correlations in Eq.(2).

$$h_{wf,IA} = [1400 - 1880 \cdot \min(0.05, Z_{Qmin})] \cdot \min(0.999 - \alpha_g, 0.5) + h_{MBr} \cdot (1 - \alpha_g)^{0.5}$$
(11)

where  $h_{MBr}$  is calculated from a modified Bromley correlation.

$$h_{MBr} = 0.62 \cdot \left\{ \frac{g \cdot \rho_g \cdot \Delta \rho \cdot k_g^3 \left[ h_{fg} + 0.5 \cdot (T_w - T_{spt}) \right]}{\max(0.005, Z_{Qmin}) \cdot \mu_g \cdot (T_w - T_{spt})} \right\}^{0.25}$$
(12)

Here,  $Z_{Qmin}$  is the distance from the point in question to the bottom quench front. It replaces the wave length in the normal Bromley correlation.

For the DFFB wall-to-liquid HTC, a modified Forslund-Rohsenow correlation was applied.

$$h_{wf,DF} = K_{BY} \cdot 0.25\pi \cdot \left[\frac{6 \cdot (0.999 - \alpha_g)}{\pi}\right]^{2/3} \cdot \left\{\frac{g \cdot \rho_g \cdot \rho_f \cdot h_{fg} \cdot k^3}{\left[\left(T_w - T_{spt}\right) \cdot \mu_g \cdot d\left(\frac{\pi}{6}\right)^{1/3}\right]\right\}^{0.25}$$
(13)

where  $K_{BY}$  is a function of the gas Reynolds number given as

$$K_{BY} = \left(\frac{Re_g - 4000}{100000}\right)^{0.6}$$
(14)

Eq.(13) is identical to the Forslund-Rohsenow correlation used to find  $h_{FR}$  in Eq.(2) except that it has a Reynolds number dependent coefficient,  $K_{BY}$  proposed by Bajorek and Young [12].

The IAFB wall-to-vapor HTC is given as a value corresponding to the conduction-only heat transfer:

$$h_{wg,IA} = 2 \cdot k_g / \delta$$
 (15)

where  $k_g$  is the vapor conductivity and  $\delta$  is the vapor film thickness in the rod bundle geometry obtained from geometrical consideration:

$$\delta = 0.5 \cdot D_{h} \cdot \left\{ \left[ 1 + \alpha_{g} \cdot \left[ (4/\pi) \cdot (P/D_{R})^{2} - 1 \right] \right]^{0.5} - 1 \right\}$$
(16)

Note that we followed the same approach as used in TRACE for calculating the IAFB wall-tovapor HTC.

The DFFB wall-to-vapor HTC in the modified version is determined from

$$\mathbf{h}_{wg,DF} = \Phi_{2p} \cdot [\mathbf{F}_{lt} \cdot \mathbf{h}_{lam} + (1 - \mathbf{F}_{lt}) \cdot \Phi_{tur} \cdot \mathbf{h}_{tur}]$$
(17)

where  $h_{tur}$  is the single-phase turbulent convective HTC calculated using the Dittus-Boelter correlation. In this case only the vapor mass flux is used to find the Reynolds number.  $\Phi_{tur}$  is a turbulent flow multiplier applied to a vertical bundle geometry and it is equal to the rod pitch to diameter ratio [7]. This multiplier can be used as an option in a calculation using the original version but it is not a part of the default model.  $h_{lam}$  is the maximum of a single-phase laminar convective HTC corresponding to a constant Nusselt number of 10.0 and a single-phase natural convective HTC calculated using the Churchill-Chu correlation. Note that the laminar convective Nusselt number was increased from 4.36 to 10.0 because the former has been regarded as a value appropriate only for internal flows. The same laminar convective Nusselt number of 10.0

has been used in COBRA/TRAC [10].  $F_{lt}$  is a linear function of the gas Reynolds number. It has a value of 1.0 at  $Re_g = 3000$  and a value of 0.0 at  $Re_g = 10000$  as proposed by Bajorek and Young [12].  $\Phi_{2p}$  is a two-phase enhancement factor to account for increases in convective heat transfer due to the relative motion of vapor and droplets. A similar factor has been used not only in TRACE [9] but also in COBRA-TF [8]. The factor can be derived using a rough wall analogy [13]:

$$\Phi_{2p} = \sqrt{\frac{\tau_i + \tau_w}{\tau_w}}$$
(18)

where  $\tau_i$  and  $\tau_w$  are friction forces per unit volume due to interfacial and wall drags, respectively and they are given in RELAP5/MOD3.3 as

$$\tau_{\rm w} = \frac{1}{2} \cdot \rho_{\rm g} \cdot f_{\rm w} \cdot v_{\rm g}^2 / D_{\rm h}$$
(19)

$$\tau_{i} = \frac{1}{8} \cdot \rho_{g} \cdot a_{gf} \cdot C_{D} \cdot v_{r}^{2}$$
<sup>(20)</sup>

where the interfacial area per unit volume,  $a_{gf}$ , and the droplet drag coefficient,  $C_D$  are in turn given as

$$a_{gf} = \frac{3.6 \cdot \alpha_f}{d}$$
(21)

$$C_{\rm D} = \frac{24 \cdot (1 + 0.1 \cdot {\rm Re}_{\rm d}^{0.75})}{{\rm Re}_{\rm d}}$$
(22)

For the wall friction factor,  $f_w$  in Eq.(19), a constant value corresponding to its fully developed one for a smooth pipe, 0.02 is assumed as in TRACE [9].

Four minor modifications were made to the radiation heat transfer model. First, the vapor emissivity is now obtained from the EMISS subroutine instead of using a constant value. The same subroutine is also used in TRACE to solve for the emissivity of liquid and vapor. Second, the wall emissivity was reduced to 0.7 to make the code agree with its manual. Third, the droplet diameter is now determined using the same method in subroutines FIDISV and FIDIS2, where the interfacial area is determined for bubbles and droplets. Note that the droplet diameter calculation method in these subroutines had a minor error which was corrected as described in Section 3.3. Fourth, a simpler approximation, that overrides Eq.(4), was introduced to find the wall-to-liquid grey body factor for the IAFB regime.

$$F_{wf} = \frac{1}{\frac{1}{\varepsilon_{f} \cdot \sqrt{\alpha_{f}}} + \left(\frac{1}{\varepsilon_{w}} - 1\right)}$$
(23)

The radiation heat transfers using the grey body factors in Eq.(4) and Eq.(23) are lumped into the wall-to-liquid heat transfer components for the DFFB and IAFB regimes, respectively.

## 3.3 Coding Error Corrections

As mentioned in the previous section, the subroutines calculating the interfacial area for bubbles and droplets contain a coding error. In subroutines FIDISV and FIDIS2, the average droplet diameter during reflood is given as

$$d = \min(D_{h}, dcon2, diam)$$
(24)  
where dcon2 = 0.0015 m for the post-CHF droplets and diam is defined as  
diam = max(dmin, d<sub>We=1.5</sub>) (25)

where dmin = 0.0015 m if pressure is sufficiently low.

Equation (24) should be corrected because dcon2 is not a maximum but a minimum average droplet diameter as per Analytis [1]. The correct expression should read

$$d = min(D_h, diam)$$

(26)

Another coding error exists in the Weismann transition boiling wall-to-liquid HTC correlation. The correlation includes  $h_{max}$  which is defined as

$$h_{\text{max}} = \frac{0.5 \cdot q_{\text{CHF}}}{\Delta T_{\text{CHF}}}$$
(27)

where  $\Delta T_{CHF}$  is the difference between the wall temperature at the CHF point and the saturation temperature according to Analytis [1]. However, it is coded as if it were the difference between the current wall temperature and the saturation temperature:

$$\Delta T_{\text{CHF}} = \max[3,\min(40,T_w - T_{\text{sat}})]$$
(28)

The correct form of this parameter should be

$$\Delta T_{\text{CHF}} = \max[3,\min(40, T_{\text{w,CHF}} - T_{\text{sat}})]$$
<sup>(29)</sup>

From test calculations employing only these error corrections, it was revealed that they do not have a significant effect on the calculations for FLECHT-SEASET tests. Thus, the errors can be regarded as minor.

## 3.4 Effect of Modifications

To test the modifications, calculations for all the tests listed in Table 1 were conducted again using the modified version. The results are compared not only to those predicted by the original version but also to the measurement data in this section.

#### 3.4.1 Test 31805

The predicted heater rod clad temperatures using the modified version at various elevations are presented in Figure 6 through Figure 13. The maximum predicted PCT is 1383 K at 1.981 m (6.5 ft) elevation while it was 1359 K at the same elevation with the original version. The predicted PCTs at all elevations are higher than those predicted using the original version and they locate between the maximum and minimum measured values at all elevations.

The predicted vapor temperatures at 1.829 m (6 ft) and 3.048 m (10 ft) elevations are presented in Figure 14 and Figure 15, respectively. The predicted vapor temperatures at both elevations are higher than those predicted using the original version and closer to the measurements.

The predicted quench front is presented in Figure 16. The predictive capability for quench times was greatly improved by the modifications. The difference between the predicted quench time and the arithmetic average of measured quench times (predicted – measured) varies from -32 to +2 seconds and the maximum absolute deviation occurred at 1.981 m (6.5 ft). The difference varied from -177 to -7 seconds when the original version was used.

The predicted and measured differential pressures for the entire core region, a mid-level (1.829 to 2.134 m (6 to 7 ft)), and a top region (3.048 to 3.353 m (10 to 11 ft)) are presented in Figure 17, Figure 18, and Figure 19, respectively. The predicted value for the entire core is still higher than the measurement data, but the deviation was slightly reduced by the modifications. The time when the predicted differential pressure for the mid-level becomes higher than the measurement was ~30 seconds delayed compared to the prediction using the original version. The predicted differential pressure for the top region was not altered much by the modifications except that the time of large oscillations were ~150 seconds delayed compared to the predicted to the prediction using the original version. It means the liquid fraction in a top region was predicted to increase more slowly with the modified version than with the original version.

The predicted and measured HTCs at five different elevations are presented in Figure 20 through Figure 24. The predicted film boiling HTCs are still higher than the measured ones at most elevations, but the deviations were reduced compared to the predictions using the original version. The HTC at 3.048 m (10 ft) elevation agrees with the measured data very well.

#### 3.4.2 Test 31504

The predicted heater rod clad temperatures using the modified version at various elevations are presented in Figure 25 through Figure 32. The maximum PCT predicted by the code is 1370 K at 1.981 m (6.5 ft) elevation while it was 1319 K at the same elevation with the original version. The predicted PCTs are higher than those predicted using the original version and they locate between the maximum and minimum measured values at most elevations. Only the predicted PCT at 3.505 m (11.5 ft) is 41 K lower than that predicted using the original version.

The predicted vapor temperatures at 1.829 m (6 ft) and 3.048 m (10 ft) elevations are presented in Figure 33 and Figure 34, respectively. The predicted vapor temperatures at both elevations are higher than those predicted using the original version and closer to the measurements.

The predicted quench front is presented in Figure 35. The predictive capability for quench times was greatly improved by the modifications. Only the quench times at and below 1.219 m (4 ft) were slightly under-predicted and the difference between the predicted quench time and the arithmetic average of measured quench times (predicted – measured) varies from -15 to +14

seconds and the maximum absolute deviation occurred at 1.219 m (4 ft). The difference varied from -149 to -5 seconds when the original version was used.

The predicted and measured differential pressures for the entire core region, a mid-level (1.829 to 2.134 m (6 to 7 ft)), and a top region (3.048 to 3.353 m (10 to 11 ft)) are presented in Figure 36, Figure 37, and Figure 38, respectively. The predicted value for the entire core is still higher than the measurement data, but the deviation was slightly reduced by the modifications. The time when the predicted differential pressure for the mid-level becomes higher than the measurement was ~50 seconds delayed compared to the prediction using the original version. The predicted differential pressure for the top region was not altered much by the modifications except that the time of large oscillations were ~100 seconds delayed compared to the prediction using the original version.

The predicted and measured HTCs at five different elevations are presented in Figure 39 through Figure 43. The predicted film boiling HTCs are still higher than the measured ones at most elevations, but the deviations were reduced compared to the predictions using the original version. The HTCs at 2.438 m (8 ft) and 3.048 m (10 ft) elevations agree with the measured data fairly well.

#### 3.4.3 Test 31203

The predicted heater rod clad temperatures using the modified version at various elevations are presented in Figure 44 through Figure 51. The maximum predicted PCT is 1265 K at 1.981 m (6.5 ft) elevation while it was 1224 K at 1.829 m (6.0 ft) elevation with the original version. The predicted PCTs are higher than those predicted using the original version and they locate between the maximum and minimum measured values at most elevations. Only the predicted PCT at 3.505 m (11.5 ft) is 11 K lower than that predicted using the original version.

The predicted vapor temperatures at 1.829 m (6 ft) and 3.048 m (10 ft) elevations are presented in Figure 52 and Figure 53, respectively. The predicted vapor temperatures at both elevations are higher than those predicted using the original version and closer to the measurements.

The predicted quench front is presented in Figure 54. The predictive capability for quench times was greatly improved by the modifications. However, the final quenches were still predicted to occur a little earlier than the measurements at most elevations. The difference between the predicted quench time and the arithmetic average of measured quench times (predicted – measured) varies from -35 to +1 seconds and the maximum absolute deviation occurred at 3.048 m (10 ft). The difference varied from -148 to -6 seconds when the original version was used.

The predicted and measured differential pressures for the entire core region, a mid-level (1.829 to 2.134 m (6 to 7 ft)), and a top region (3.048 to 3.353 m (10 to 11 ft)) are presented in Figure 55, Figure 56, and Figure 57, respectively. The predicted value for the entire core is still higher than the measurement data, but the deviation was slightly reduced by the modifications. The time when the predicted differential pressure for the mid-level becomes higher than the measurement was ~30 seconds delayed compared to the prediction using the original version. The predicted differential pressure for the top region was not altered much by the modifications except the time of large oscillations were ~100 seconds delayed compared to the prediction using the original version.

The predicted and measured HTCs at five different elevations are presented in Figure 58 through Figure 62. Though the film boiling HTCs were predicted relatively well, the transition boiling was predicted to occur earlier than the measurements.

#### 3.4.4 Test 31302

The predicted heater rod clad temperatures using the modified version at various elevations are presented in Figure 63 through Figure 70. The maximum predicted PCT is 1166 K at 1.829 m (6.0 ft) elevation while it was 1159 K at the same elevation with the original version. The predicted PCTs are higher than those predicted using the original version and they locate between the maximum and minimum measured values at most elevations. The predicted PCT at 3.353 m (11 ft) is higher than the measurements just like that predicted using the original version.

The predicted vapor temperatures at 1.829 m (6 ft) and 3.048 m (10 ft) elevations are presented in Figure 71 and Figure 72, respectively. The predicted vapor temperatures at both elevations are slightly higher than those predicted using the original version. Time when the vapor loses superheating was delayed by the modifications at both elevations.

The predicted quench front is presented in Figure 73. The predictive capability for quench times was greatly improved by the modifications. However, the final quenches were still predicted to occur earlier than the measurements at most elevations. The difference between the predicted quench time and the arithmetic average of measured quench times (predicted – measured) varies from -57 to +38 seconds and the maximum absolute deviation occurred at 3.048 m (10 ft). The difference varied from -120 to -7 seconds when the original version was used. Though the difference was reduced much by the modifications, the effect of modifications is rather limited compared to that observed for lower flooding rate tests (Test 31805, Test 31504, and Test 31203).

The predicted and measured differential pressures for the entire core region, a mid-level (1.829 to 2.134 m (6 to 7 ft)), and a top region (3.048 to 3.353 m (10 to 11 ft)) are presented in Figure 74, Figure 75, and Figure 76, respectively. The modified version predicted better the differential pressure for the entire core than the original version, especially after ~100 seconds. The time when the predicted differential pressure for the mid-level becomes higher than the measurement was ~30 seconds delayed compared to the prediction using the original version. The predicted differential pressure for the top region was not altered much by the modifications except that the time of large oscillations were ~60 seconds delayed compared to the prediction using the original version.

The predicted and measured HTCs at five different elevations are presented in Figure 77 through Figure 81. Though the film boiling HTCs were predicted relatively well, the transition boiling was predicted to start earlier than the measurements.

#### 3.4.5 Test 31701

The predicted heater rod clad temperatures using the modified version at various elevations are presented in Figure 82 through Figure 89. The maximum predicted PCT is 1136 K at 1.829 m (6.0 ft) elevation while it was 1135 K at the same elevation with the original version. The predicted PCTs are slightly higher than those predicted using the original version at all elevations. They locate between the maximum and minimum measured values at all elevations except 3.353 m (11 ft) elevation.

The predicted vapor temperatures at 1.829 m (6 ft) and 3.048 m (10 ft) elevations are presented in Figure 90 and Figure 91, respectively. The peak vapor temperatures predicted at both elevations are not different much from those predicted using the original version. Only the time when the vapor loses superheating was delayed by the modifications.

The predicted quench front is presented in Figure 92. The predictive capability for quench times was a little improved by the modifications. However, the final quenches were still predicted to occur earlier than the measurements at most elevations. The difference between the predicted quench time and the arithmetic average of measured quench times (predicted – measured) varies from -30 to -1 seconds and the maximum absolute deviation occurred at 3.048 m (10 ft). The difference varied from -41 to -3 seconds when the original version was used. Though the difference was reduced a little by the modifications, the effect of modifications is rather limited compared to that observed for the tests having relatively low flooding rates (Test 31805, Test 31504, and Test 31203).

The predicted and measured differential pressures for the entire core region, a mid-level (1.829 to 2.134 m (6 to 7 ft)), and a top region (3.048 to 3.353 m (10 to 11 ft)) are presented in Figure 93, Figure 94, and Figure 95, respectively. The predicted value for the entire core is still higher than the measurement data, but the difference from the measurement was slightly reduced by the modifications. The time when the predicted differential pressure for the mid-level becomes higher than the measurement was ~15 seconds delayed compared to the prediction using the original version. The predicted differential pressure for the top region was not altered much by the modifications except the time of large oscillations were ~30 seconds delayed compared to the prediction using the prediction using the original version.

The predicted and measured HTCs at five different elevations are presented in Figure 96 through Figure 100. The predicted HTC at 1.219 m (4 ft) was not altered much by the modifications. The predictions for HTCs at other locations were improved a little, but the transition boiling was predicted to occur earlier than the measurements.

#### 3.4.6 Test 31108

The predicted heater rod clad temperatures using the modified version at various elevations are presented in Figure 101 through Figure 108. The maximum PCT predicted by the code is 1167 K at 1.981 m (6.5 ft) elevation while it was 1145 K at 1.829 m (6.0 ft) elevation with the original version. The predicted PCTs are higher than those predicted using the original version. The PCTs at and above 2.438 m (8 ft) were definitely over-predicted by the modified version. PCTs were predicted to occur later in time compared to not only the measurements but also the predictions using the original version.

The predicted vapor temperatures at 1.829 m (6 ft) and 3.048 m (10 ft) elevations are presented in Figure 109 and Figure 110. The predicted vapor temperatures at both elevations are nearly equal to or slightly higher than those predicted using the original version. The times when the vapor loses superheating were delayed by the modifications at both elevations.

The predicted quench front is presented in Figure 111. The predictive capability for quench times was much improved by the modifications. However, the final quenches were still predicted to occur earlier than the measurements at all elevations but 3.353 m (11 ft) and 3.505 m (11.5 ft) elevations. The difference between the predicted quench time and the arithmetic average of measured quench times (predicted – measured) varies from -77 to +130 seconds and the

maximum absolute deviation occurred at 3.505 m (11.5 ft). The difference varied from -162 to +13 seconds when the original version was used. Though the predicted quench front went closer to the measurement with the modifications, the top-down quenchings at a top region were not predicted well.

The predicted and measured differential pressures for the entire core region, a mid-level (1.829 to 2.134 m (6 to 7 ft)), and a top region (3.048 to 3.353 m (10 to 11 ft)) are presented in Figure 112, Figure 113, and Figure 114, respectively. The predicted value for the entire core is still higher than the measurement data, but the deviation was slightly reduced by the modifications. The time when the predicted differential pressure for the mid-level becomes higher than the measurement was ~40 seconds delayed compared to the prediction using the original version. The predicted differential pressure for the top region was not altered much by the modifications, but the large oscillations from ~150 seconds with the original version were replaced with a sudden increase at ~265 seconds.

The predicted and measured HTCs at five different elevations are presented in Figure 115 through Figure 119. Though the film boiling HTCs were reduced compared those predicted using the original version, the transition boiling was still predicted to start earlier than measurements.

#### 3.4.7 Test 32013

The predicted heater rod clad temperatures using the modified version at various elevations are presented in Figure 120 through Figure 127. The maximum predicted PCT is 1353 K at 1.981 m (6.5 ft) elevation while it was 1326 K at the same elevation with the original version. The predicted PCTs are higher than those predicted using the original version and they locate between the maximum and minimum measured values at all elevations.

The predicted vapor temperatures at 1.829 m (6 ft) and 3.048 m (10 ft) elevations are presented in Figure 128 and Figure 129, respectively. The vapor temperatures increased for a longer time compared to the predictions with the original version. The time when the vapor loses superheating was delayed by the modifications at both elevations.

The predicted quench front is presented in Figure 130. The predictive capability for quench times was greatly improved by the modifications. Only the quench times at 0.610 m (2 ft), 1.219 m (4 ft), and 3.048 m (10 ft) elevations were slightly under-predicted and the difference between the predicted quench time and the arithmetic average of measured quench times (predicted – measured) varies from -7 to +12 seconds and the maximum absolute deviation occurred at 3.353 m (11 ft). The difference varied from -101 to -2 seconds when the original version was used.

The predicted and measured differential pressures for the entire core region, a mid-level (1.829 to 2.134 m (6 to 7 ft)), and a top region (3.048 to 3.353 m (10 to 11 ft)) are presented in Figure 131, Figure 132, and Figure 133, respectively. The predicted value for the entire core agrees with the measurement excellently. Even the small over-prediction by the original version was removed by the modifications. The prediction of the differential pressure for the mid-level was significantly improved by the modifications. The over-prediction with the original version which was observed after ~180 seconds was reduced to an acceptable level. The predicted differential pressure for the top region was not altered much by the modifications except that the time of large oscillations was ~100 seconds delayed compared to the prediction using the original version.

The predicted and measured HTCs at five different elevations are presented in Figure 134 through Figure 138. The film boiling HTCs were predicted very well especially at or over 2.438 m (8 ft) elevation. HTCs at other elevations were slightly over-predicted compared to the measurement data.

#### 3.4.8 Test 32114

The predicted heater rod clad temperatures using the modified version at various elevations are presented in 139 through Figure 146. The maximum predicted PCT is 1356 K at 1.981 m (6.5 ft) elevation while it was 1320 K at the same elevation with the original version. The predicted PCTs are higher than those predicted using the original version and they locate between the maximum and minimum measured values at most elevations. The predicted PCT at 3.505 m (11.5 ft) is lower than that predicted with the original version while the predicted PCTs at 3.048 m (10 ft) and 3.353 m (11 ft) are higher than the measurements.

The predicted vapor temperatures at 1.829 m (6 ft) and 3.048 m (10 ft) elevations are presented in Figure 147 and Figure 148, respectively. The predicted vapor temperatures at both elevations are slightly higher than those predicted using the original version. The time when the vapor loses superheating was delayed by the modifications at both elevations.

The predicted quench front is presented in Figure 149. The predictive capability for quench times was greatly improved by the modifications. However, the final quenches of 1.981 m (6.5 ft) elevation and lower elevations were predicted to occur slightly earlier than the measurements while the quenches over 1.981 m (6.5 ft) elevation were predicted to occur later than the measurements. The difference between the predicted quench time and the arithmetic average of measured quench times (predicted – measured) varies from -26 to +64 seconds and the maximum absolute deviation occurred at 3.048 m (10 ft). The difference varied from -113 to -15 seconds when the original version was used.

The predicted and measured differential pressures for the entire core region, a mid-level (1.829 to 2.134 m (6 to 7 ft)), and a top region (3.048 to 3.353 m (10 to 11 ft)) are presented in Figure 150, Figure 151, and Figure 152, respectively. The predicted value for the entire core is still higher than the measurement data, but the deviation was slightly reduced by the modifications. For the predicted differential pressure for the mid-level, the time when large oscillations occur was ~100 seconds delayed by the modifications. A similar effect of code modifications can be observed in the prediction for the differential pressure for the top region.

The predicted and measured HTCs at five different elevations are presented in Figure 153 through Figure 157. Though the film boiling HTCs were reduced with the modifications, the HTCs at or below 1.981 m (6.5 ft) were predicted to increase to the magnitude of HTCs in the transition boiling earlier than the measurements.

#### 3.4.9 All 8 Tests

Similar to Figure 158, the deviations of the predicted PCTs from the measurements are presented in Figure 160. The predictability for PCTs was not improved significantly but the RMS error in overall PCT predictions dropped from 48.3 to 36.7 K. Comparisons of the maxima and minima curves from Figures 158 and 160 in Figure 161 show more clearly the effect of the modifications. The largest PCT under-prediction dropped from 9.4 to 5.9%. Though the largest PCT over-prediction increased from 10.4 to 14.5% after the modifications, we believe that the

results with the modified version is more reasonable because the code has no model to take into account the heat transfer enhancement by spacer grids, which is dominant at upper elevations. The improved predictability for PCTs can be assured by Figure 162 where all the predicted PCTs are plotted versus the measured ones. Least-square fitting curves before and after the modifications are also compared in this figure.

A similar but more significant effect of the modifications can be seen in terms of quench times. The differences between measurements and predictions with the modified version are presented in Figure 163, and the maxima and minima curves of Figure 163 are compared to those of Figure 159 in Figure 164. These figures indicate that quench times are better predicted with the modified version. The improved predictability for quench times can be seen more clearly in Figure 165 which presents the predicted quench times as a function of the measured ones. The RMS error in overall quench time predictions dropped from 85.9 to 32.1 seconds.

As shown in Eq.(2), the original code takes the maximum HTCs calculated by three different correlations as the film boiling HTC to liquid. That model may provide a good approximation for the IAFB HTC to liquid. However, it must over-predict the DFFB HTC to liquid because the HTC calculated with the CATHARE correlation is usually higher than that calculated with the Forslund-Rohsenow correlation. As we modified the code to use a modified Forslund-Rohsenow correlation for the DFFB HTC to liquid, the heat transfer to liquid must have been reduced.

On the contrary, the DFFB HTC to vapor was increased by the modifications. We introduced not only a turbulent flow multiplier but also a two-phase enhancement factor to the DFFB HTC to vapor, as shown in Eq.(17). These factors, as their names imply, have an effect of enhancing the wall-to-vapor heat transfer. As a result, the predicted vapor temperatures increased with the modifications. Though it is hard to say quantitatively since only two or three vapor temperature measurements are available at an elevation, the figures containing the vapor temperature predictions indicate that the DFFB HTC to vapor was modified to the right direction.

Note that even the modified version still over-predicts HTCs at lower or middle elevations. HTCs at higher elevations were predicted relatively well. However, if we had introduced a spacer grid model, it would have over-predicted also HTCs at higher elevations.

The code tends to over-predict the overall pressure drop for the entire bundle for most tests even with the modifications. Though predictions were improved a little by the modifications, the differences between the predicted and measured pressure drops are greater than the measurement uncertainty for all tests except Test 32013. In addition, the pressure drop was over-predicted from the beginning of the transient. Thus, we suspect that this over-predicted pressure drop for the entire bundle (or collapsed liquid level) is one of the reasons for the over-prediction of HTCs at lower and middle elevations.

# 4. CONCLUSIONS

The reflood model of RELAP5/MOD3.3 was assessed using eight FLECHT-SEASET tests. From the assessment, the following findings were obtained.

- The code predicts the measured PCTs within ±10% deviation.
- The code predicts rod quenches to occur significantly earlier than the measurement data.
- The code has a tendency of under-predicting peak vapor temperatures at 2 different elevations, especially those of low flooding rate tests.
- The code has a tendency of over-predicting the overall pressure drop for the entire bundle.

Based on these findings, the wall-to-fluid heat transfer models which are activated during the reflood calculations were modified to improve the code's predictability mainly for quench times. We divided the film boiling heat transfer regime into three different sub-regimes and selected appropriate heat transfer correlations for each sub-regime. In addition, we fixed minor coding errors in a few subroutines and modified slightly the radiation heat transfer model to assume more reasonable and consistent emissivity and droplet diameter.

Another set of calculations for the same FLECHT-SEASET tests were conducted with the modifications. From the comparisons of the predictions not only to the measurement data but also to the predictions using the original version, the following conclusions were derived.

- The RMS error in overall PCT predictions is reduced from 48.3 to 36.7 K by the modifications.
- The RMS error in overall quench time predictions is reduced from 85.9 to 32.1 s by the modifications.
- The film boiling HTCs predicted using the modified version shows better agreement with the measurement data compared to those predicted using the original version.
- Wall-to-vapor heat transfers were increased by the modifications so that the predicted vapor temperatures are higher than those predicted by the original version.
- Even with the modifications, the code still has a tendency of over-predicting the overall
  pressure drop for the entire core (or collapsed liquid level in the core). Thus, further efforts
  to improve the reflood model of RELAP5/MOD3.3 should be made to the wall or interfacial
  drag models.

## 5. **REFERENCES**

- Analytis, G. Th., Developmental Assessment of RELAP5/MOD3.1 with Separate Effect and Integral Test Experiments: Model Changes and Options. Nuclear Engineering and Design, 163, p.125-148, 1996.
- [2]. Koszela, Z., Assessment of RELAP5/MOD3.2.2 Gamma against ABB Atom 3×3-Rod Bundle Reflooding Tests. Nuclear Engineering and Design, 223, p.49-73, 2003.
- [3]. Chung, B. D. et al., Improvements to the RELAP5/MOD3 Reflood Model Assessment. Journal of Korean Nuclear Society, 26, p.265-276, 1994.
- [4]. Baek, J. S. et al., Assessment of FLECHT SEASET Unblocked Forced Reflood Tests Using RELAP5/MOD3. Journal of Korean Nuclear Society, 24, p.297-310, 1992.
- [5]. Loftus, M. J. et al., PWR FLECHT-SEASET Unblocked Bundle, Force and Gravity Reflood Task Data Report, NUREG/CR-1532, 1980.
- [6]. USNRC, TRACE V5.0 Assessment Manual, 2007.
- [7]. USNRC, RELAP5/MOD3.3 Code Manual. NUREG/CR-5535/Rev 1, 2001.
- [8]. Westinghouse, Analysis of FLECHT SEASET 163-Rod Blocked Bundle Data Using COBRA-TF. NUREG/CR-4166, 1985.
- [9]. USNRC, TRACE V5.0 Theory Manual, 2007.
- [10]. Thurgood, M. J. et al., COBRA/TRAC-A Thermal Hydraulic Code for Transient Analysis of Nuclear Reactor Vessels and Primary Coolant Systems. NUREG/CR-3046, 1983.
- [11]. Sun, K. H., Gonzales-Santalo, J. M., and Tien, C.L., Calculations of Combined Radiation and Convection Heat Transfer in Rod Bundles Under Emergency Cooling Conditions. Journal of Heat Transfer, 98:3, p.414-420, 1976.
- [12]. Bajorek, S. M. and Young, M. Y., Direct-Contact Heat Transfer Model for Dispersed-Flow Film Boiling. Nuclear Technology, 132, p.375-388, 2000.
- [13]. Kays, W., Convective Heat and Mass Transfer. McGraw-Hill, p.348-349, 1993.

No.	Test No.	Flooding Rate	Upper Plenum Pressure	Coolant Inlet Temperature	Inlet subcooling
		cm/s	MPa	K	K
		(in/sec)	(psia)	(°F)	(°F)
1	31805	2.10	0.28	324	79
		_(0.81)	(40)	(124)	(143)
2	31504	2.40	0.28	324	79
۲		(0.97)	(40)	(124)	(143)
3	31203	3.84	0.28	325	78
		(1.51)	(40)	(126)	(141)
Λ	31302	7.65	0.28	325	78
-4		(3.01)	(40)	(126)	(141)
5	31701	15.50	0.28	326	77
J		(6.10)	(40)	(127)	(140)
6	31108	7.90	0.13	306	74
0		(3.11)	(19)	(91)	(134)
7	32013	2.64	0.41	339	79
		(1.04)	(60)	(150)	(143)
0	32114	2.5~3.1	0.28	408	5
0		(1.0~1.22)	(40)	(257)	(10)

## Table 1. FLECHT-SEASET Tests for RELAP5 Assessment



Figure 1. Facility Flow Diagram for Forced Reflood Tests



Figure 2. Cross-sectional View of Bundle Heated Section



Figure 3. Axial Power Profile of Heater Rods



Figure 4. FLECHT-SEASET RELAP5 Noding Diagram



Figure 5. Effect of Axial Noding on Quench Front Elevation



Figure 6. Rod Clad Temperatures at 2 ft from Heated Bottom for Test 31805



Figure 7. Rod Clad Temperatures at 4 ft from Heated Bottom for Test 31805



Figure 8. Rod Clad Temperatures at 6 ft from Heated Bottom for Test 31805



Figure 9. Rod Clad Temperatures at 6.5 ft from Heated Bottom for Test 31805



Figure 10. Rod Clad Temperatures at 8 ft from Heated Bottom for Test 31805



Figure 11. Rod Clad Temperatures at 10 ft from Heated Bottom for Test 31805



Figure 12. Rod Clad Temperatures at 11 ft from Heated Bottom for Test 31805



Figure 13. Rod Clad Temperatures at 11.5 ft from Heated Bottom for Test 31805



Figure 14. Vapor Temperatures at 6 ft from Heated Bottom for Test 31805



Figure 15. Vapor Temperatures at 10 ft from Heated Bottom for Test 31805



Figure 16. Quench Profile as a Function of Time for Test 31805



Figure 17. Differential Pressure for the Entire 12 ft Core for Test 31805



Figure 18. Differential Pressure at 6~7 ft Elevation for Test 31805



Figure 19. Differential Pressure at 10~11 ft Elevation for Test 31805



Figure 20. Heat Transfer Coefficient at 4 ft from Heated Bottom for Test 31805



Figure 21. Heat Transfer Coefficient at 6 ft from Heated Bottom for Test 31805



Figure 22. Heat Transfer Coefficient at 6.5 ft from Heated Bottom for Test 31805



Figure 23. Heat Transfer Coefficient at 8 ft from Heated Bottom for Test 31805



Figure 24. Heat Transfer Coefficient at 10 ft from Heated Bottom for Test 31805



Figure 25. Rod Clad Temperatures at 2 ft from Heated Bottom for Test 31504



Figure 26. Rod Clad Temperatures at 4 ft from Heated Bottom for Test 31504



Figure 27. Rod Clad Temperatures at 6 ft from Heated Bottom for Test 31504



Figure 28. Rod Clad Temperatures at 6.5 ft from Heated Bottom for Test 31504



Figure 29. Rod Clad Temperatures at 8 ft from Heated Bottom for Test 31504



Figure 30. Rod Clad Temperatures at 10 ft from Heated Bottom for Test 31504



Figure 31. Rod Clad Temperatures at 11 ft from Heated Bottom for Test 31504



Figure 32. Rod Clad Temperatures at 11.5 ft from Heated Bottom for Test 31504



Figure 33. Vapor Temperatures at 6 ft from Heated Bottom for Test 31504



Figure 34. Vapor Temperatures at 10 ft from Heated Bottom for Test 31504


Figure 35. Quench Profile as a Function of Time for Test 31504



Figure 36. Differential Pressure for the Entire 12 ft Core for Test 31504



Figure 37. Differential Pressure at 6~7 ft Elevation for Test 31504



Figure 38. Differential Pressure at 10~11 ft Elevation for Test 31504



Figure 39. Heat Transfer Coefficient at 4 ft from Heated Bottom for Test 31504



Figure 40. Heat Transfer Coefficient at 6 ft from Heated Bottom for Test 31504



Figure 41. Heat Transfer Coefficient at 6.5 ft from Heated Bottom for Test 31504



Figure 42. Heat Transfer Coefficient at 8 ft from Heated Bottom for Test 31504



Figure 43. Heat Transfer Coefficient at 10 ft from Heated Bottom for Test 31504



Figure 44. Rod Clad Temperatures at 2 ft from Heated Bottom for Test 31203



Figure 45. Rod Clad Temperatures at 4 ft from Heated Bottom for Test 31203



Figure 46. Rod Clad Temperatures at 6 ft from Heated Bottom for Test 31203



Figure 47. Rod Clad Temperatures at 6.5 ft from Heated Bottom for Test 31203



Figure 48. Rod Clad Temperatures at 8 ft from Heated Bottom for Test 31203



Figure 49. Rod Clad Temperatures at 10 ft from Heated Bottom for Test 31203



Figure 50. Rod Clad Temperatures at 11 ft from Heated Bottom for Test 31203



Figure 51. Rod Clad Temperatures at 11.5 ft from Heated Bottom for Test 31203



Figure 52. Vapor Temperatures at 6 ft from Heated Bottom for Test 31203



Figure 53. Vapor Temperatures at 10 ft from Heated Bottom for Test 31203



Figure 54. Quench Profile as a Function of Time for Test 31203



Figure 55. Differential Pressure for the Entire 12 ft Core for Test 31203



Figure 56. Differential Pressure at 6~7 ft Elevation for Test 31203



Figure 57. Differential Pressure at 10~11 ft Elevation for Test 31203



Figure 58. Heat Transfer Coefficient at 4 ft from Heated Bottom for Test 31203



Figure 59. Heat Transfer Coefficient at 6 ft from Heated Bottom for Test 31203



Figure 60. Heat Transfer Coefficient at 6.5 ft from Heated Bottom for Test 31203



Figure 61. Heat Transfer Coefficient at 8 ft from Heated Bottom for Test 31203



Figure 62. Heat Transfer Coefficient at 10 ft from Heated Bottom for Test 31203



Figure 63. Rod Clad Temperatures at 2 ft from Heated Bottom for Test 31302



Figure 64. Rod Clad Temperatures at 4 ft from Heated Bottom for Test 31302



Figure 65. Rod Clad Temperatures at 6 ft from Heated Bottom for Test 31302



Figure 66. Rod Clad Temperatures at 6.5 ft from Heated Bottom for Test 31302



Figure 67. Rod Clad Temperatures at 8 ft from Heated Bottom for Test 31302



Figure 68. Rod Clad Temperatures at 10 ft from Heated Bottom for Test 31302



Figure 69. Rod Clad Temperatures at 11 ft from Heated Bottom for Test 31302



Figure 70. Rod Clad Temperatures at 11.5 ft from Heated Bottom for Test 31302



Figure 71. Vapor Temperatures at 6 ft from Heated Bottom for Test 31302



Figure 72. Vapor Temperatures at 10 ft from Heated Bottom for Test 31302



Figure 73. Quench Profile as a Function of Time for Test 31302



Figure 74. Differential Pressure for the Entire 12 ft Core for Test 31302



Figure 75. Differential Pressure at 6~7 ft Elevation for Test 31302



Figure 76. Differential Pressure at 10~11 ft Elevation for Test 31302



Figure 77. Heat Transfer Coefficient at 4 ft from Heated Bottom for Test 31302



Figure 78. Heat Transfer Coefficient at 6 ft from Heated Bottom for Test 31302



Figure 79. Heat Transfer Coefficient at 6.5 ft from Heated Bottom for Test 31302



Figure 80. Heat Transfer Coefficient at 8 ft from Heated Bottom for Test 31302



Figure 81. Heat Transfer Coefficient at 10 ft from Heated Bottom for Test 31302



Figure 82. Rod Clad Temperatures at 2 ft from Heated Bottom for Test 31701



Figure 83. Rod Clad Temperatures at 4 ft from Heated Bottom for Test 31701



Figure 84. Rod Clad Temperatures at 6 ft from Heated Bottom for Test 31701



Figure 85. Rod Clad Temperatures at 6.5 ft from Heated Bottom for Test 31701



Figure 86. Rod Clad Temperatures at 8 ft from Heated Bottom for Test 31701



Figure 87. Rod Clad Temperatures at 10 ft from Heated Bottom for Test 31701



Figure 88. Rod Clad Temperatures at 11 ft from Heated Bottom for Test 31701



Figure 89. Rod Clad Temperatures at 11.5 ft from Heated Bottom for Test 31701



Figure 90. Vapor Temperatures at 6 ft from Heated Bottom for Test 31701



Figure 91. Vapor Temperatures at 10 ft from Heated Bottom for Test 31701



Figure 92. Quench Profile as a Function of Time for Test 31701





Figure 95. Differential Pressure at 10~11 ft Elevation for Test 31701



Figure 96. Heat Transfer Coefficient at 4 ft from Heated Bottom for Test 31701



Figure 97. Heat Transfer Coefficient at 6 ft from Heated Bottom for Test 31701



Figure 98. Heat Transfer Coefficient at 6.5 ft from Heated Bottom for Test 31701



Figure 99. Heat Transfer Coefficient at 8 ft from Heated Bottom for Test 31701



Figure 100. Heat Transfer Coefficient at 10 ft from Heated Bottom for Test 31701



Figure 101. Rod Clad Temperatures at 2 ft from Heated Bottom for Test 31108



Figure 102. Rod Clad Temperatures at 4 ft from Heated Bottom for Test 31108



Figure 103. Rod Clad Temperatures at 6 ft from Heated Bottom for Test 31108







Figure 105. Rod Clad Temperatures at 8 ft from Heated Bottom for Test 31108



Figure 106. Rod Clad Temperatures at 10 ft from Heated Bottom for Test 31108


Figure 107. Rod Clad Temperatures at 11 ft from Heated Bottom for Test 31108



Figure 108. Rod Clad Temperatures at 11.5 ft from Heated Bottom for Test 31108



Figure 109. Vapor Temperatures at 6 ft from Heated Bottom for Test 31108



Figure 110. Vapor Temperatures at 10 ft from Heated Bottom for Test 31108



Figure 111. Quench Profile as a Function of Time for Test 31108



Figure 112. Differential Pressure for the Entire 12 ft Core for Test 31108



Figure 113. Differential Pressure at 6~7 ft Elevation for Test 31108



Figure 114. Differential Pressure at 10~11 ft Elevation for Test 31108



Figure 115. Heat Transfer Coefficient at 4 ft from Heated Bottom for Test 31108



Figure 116. Heat Transfer Coefficient at 6 ft from Heated Bottom for Test 31108



Figure 117. Heat Transfer Coefficient at 6.5 ft from Heated Bottom for Test 31108



Figure 118. Heat Transfer Coefficient at 8 ft from Heated Bottom for Test 31108



Figure 119. Heat Transfer Coefficient at 10 ft from Heated Bottom for Test 31108



Figure 120. Rod Clad Temperatures at 2 ft from Heated Bottom for Test 32013



Figure 121. Rod Clad Temperatures at 4 ft from Heated Bottom for Test 32013



Figure 122. Rod Clad Temperatures at 6 ft from Heated Bottom for Test 32013



Figure 123. Rod Clad Temperatures at 6.5 ft from Heated Bottom for Test 32013



Figure 124. Rod Clad Temperatures at 8 ft from Heated Bottom for Test 32013



Figure 125. Rod Clad Temperatures at 10 ft from Heated Bottom for Test 32013



Figure 126. Rod Clad Temperatures at 11 ft from Heated Bottom for Test 32013



Figure 127. Rod Clad Temperatures at 11.5 ft from Heated Bottom for Test 32013



Figure 128. Vapor Temperatures at 6 ft from Heated Bottom for Test 32013



Figure 129. Vapor Temperatures at 10 ft from Heated Bottom for Test 32013



Figure 130. Quench Profile as a Function of Time for Test 32013



Figure 131. Differential Pressure for the Entire 12 ft Core for Test 32013



Figure 132. Differential Pressure at 6~7 ft Elevation for Test 32013



Figure 133. Differential Pressure at 10~11 ft Elevation for Test 32013



Figure 134. Heat Transfer Coefficient at 4 ft from Heated Bottom for Test 32013



Figure 135. Heat Transfer Coefficient at 6 ft from Heated Bottom for Test 32013



Figure 136. Heat Transfer Coefficient at 6.5 ft from Heated Bottom for Test 32013



Figure 137. Heat Transfer Coefficient at 8 ft from Heated Bottom for Test 32013



Figure 138. Heat Transfer Coefficient at 10 ft from Heated Bottom for Test 32013



Figure 139. Rod Clad Temperatures at 2 ft from Heated Bottom for Test 32114



Figure 140. Rod Clad Temperatures at 4 ft from Heated Bottom for Test 32114



Figure 141. Rod Clad Temperatures at 6 ft from Heated Bottom for Test 32114



Figure 142. Rod Clad Temperatures at 6.5 ft from Heated Bottom for Test 32114



Figure 143. Rod Clad Temperatures at 8 ft from Heated Bottom for Test 32114



Figure 144. Rod Clad Temperatures at 10 ft from Heated Bottom for Test 32114



Figure 145. Rod Clad Temperatures at 11 ft from Heated Bottom for Test 32114



Figure 146. Rod Clad Temperatures at 11.5 ft from Heated Bottom for Test 32114



Figure 147. Vapor Temperatures at 6 ft from Heated Bottom for Test 32114



Figure 148. Vapor Temperatures at 10 ft from Heated Bottom for Test 32114



Figure 149. Quench Profile as a Function of Time for Test 32114



Figure 150. Differential Pressure for the Entire 12 ft Core for Test 32114



Figure 151. Differential Pressure at 6~7 ft Elevation for Test 32114



Figure 152. Differential Pressure at 10~11 ft Elevation for Test 32114



Figure 153. Heat Transfer Coefficient at 4 ft from Heated Bottom for Test 32114



Figure 154. Heat Transfer Coefficient at 6 ft from Heated Bottom for Test 32114



Figure 155. Heat Transfer Coefficient at 6.5 ft from Heated Bottom for Test 32114



Figure 156. Heat Transfer Coefficient at 8 ft from Heated Bottom for Test 32114



Figure 157. Heat Transfer Coefficient at 10 ft from Heated Bottom for Test 32114



Figure 158. Percent Deviations of Predicted PCTs with Original Version



Figure 159. Differences in Quench Times with Original Version



Figure 160. Percent Deviations of Predicted PCTs with Modified Version

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Figure 161. Effect of Modifications on PCT Predictions



Figure 162. Comparison of Predictability for PCTs



Figure 163. Differences in Quench Times with Modified Version



Figure 164. Effect of Modifications on Quench Time Predictions



Figure 165. Comparison of Predictability for Quench Times

## **APPENDIX. RELAP5 BASE DECK FOR FLECHT-SEASET TEST 31805**

```
=flecht seaset test no. 31805 using 49 nodes & 49 heat strs.
* option 50: original critical flow model
1
    50
100 new
           transnt
101 run
102 si
         si
105 10.0 20.0
*
   time step
201 810.0 1.e-8 0.1 3 10 5000 5000
******
÷
*
    mimor edits
361 cntrlvar 005 * core collapsed water level

        361
        Chtrivar
        005

        362
        chtrivar
        110

        363
        chtrivar
        200

        20800001
        fij
        200010000

        20800002
        fij
        200020000

        20800003
        fij
        200030000

                         * integrated water carry-over
                           * total power
20800004 fij 200040000
20800005 fij 200050000
20800006 fij 200060000
20800007 fij 200070000
20800008 fij 200080000
20800009 fii 200090000
20800010 fij 200100000
20800011 fij 200110000
20800012 fij
               200120000
20800013 fij
               200130000
20800013 fij
20800014 fij
20800015 fij
20800016 fij
20800017 fij
20800018 fij
20800019 fij
               200140000
               200150000
               200160000
               200170000
               200180000
               200190000
               200200000
20800020 fij
20800021 fij
               200210000
20800022 fij
               200220000
20800023 fij
               200230000
20800024 fij
               200240000
20800025 fii
               200250000
20800026 fij
               200260000
20800027 fij
               200270000
20800028 fij
                200280000
20800029 fij 200290000
20800030 fij 20030000
20800031 fij 200310000
20800032 fij 200320000
```

20800033 20800034 20800035 20800036 20800037 20800038 20800039 20800040 20800041 20800042 20800043 20800045	fij fij fij fij fij fij fij fij fij fij	20 20 20 20 20 20 20 20 20 20 20 20 20 2	1033 1034 1035 1036 1036 1036 1036 1036 1036 1036 1042 1042 1042 1043	100 100 500 500 700 100 100 100		
20800046 20800047 20800048 *	fij fij fij	20 20 20	046 047 048	500 700 800	00 00 00	
20800051 20800052 20800053 20800054 20800055 20800056 20800057 20800058 20800059 20800060 20800061 20800063 20800063	htmo htmo htmo htmo htmo htmo htmo htmo	de de de de de de de de de de de de de d	200 200 200 200 200 200 200 200 200 200	)10)10)10)10)10)10)10)10)10)10)10)10)10)	101 102 103 104 105 106 106 107 108 109 110 112 113 114	01 01 01 01 01 01 01 01 01 01 01 01
20800065 20800066 20800067 20800068 20800070 20800070 20800072 20800073 20800074 20800075 20800076 20800077 20800078	htmo htmo htmo htmo htmo htmo htmo htmo	de de de de de de de de de de de de de d	200 200 200 200 200 200 200 200 200 200	)10)10)10)10)10)10)10)10)10)10)10)10)10)	)16 )17 )17 )18 )20 )21 )22 )23 )24 )25 )26 )27 )27	01 01 01 01 01 01 01 01 01 01 01 01 01 0
20800079 2080080 2080081 2080082 2080083 2080084 2080085 2080085 2080086 2080087 2080088 2080088 2080089 2080090	htmo htmo htmo htmo htmo htmo htmo htmo	de de de de de de de de de de de de	200 200 200 200 200 200 200 200 200 200	010 010 010 010 010 010 010 010 010 010	)29 )30 )31 )32 )33 )34 )35 )36 )37 )38 )39 )39	01 01 01 01 01 01 01 01 01

```
20800091 htmode 200104101
20800092 htmode 200104201
20800093 htmode 200104301
20800093 htmode 200104301
20800094 htmode 200104401
20800095 htmode 200104501
20800096 htmode 200104601
20800097 htmode 200104701
20800098 htmode 200104801
20800099 htmode 200104901
20800101 dt
           0
20800102 dtcrnt 0
20800111 fines 2001
20800112 tchfqf 2001
20800113 trewet 2001
20800114 zgbot 2001
**********************
   TRIP LOGICS
0 gt null 0 0.0 1 -1.0
500 time
hydraulic components
*
   inlet boundary volume
1000000 lplenum tmdpvol
1000200 3
        500
* test specific pressure and temperature
1000201 0.0 275790.0 326.2
1000202 5.0 275790.0 326.8
1000203 40.0 275790.0 323.8
1000204 77.0 275790.0 323.8
1000205 208.0 275790.0 325.6
1000206 357.0 275790.0 325.6
1000207 400.0 275790.0 326.2
1000208 700.0 275790.0 326.2
1000209 820.0 275790.0 326.2
inlet boundary junction
1500000 inlet
           tmdpjun
1500101 100000000 160000000 0.0155565
1500200 0
           500
* test specific inlet velocity
1500201 0.0 0.01964 0.0 0.0
1500202 12.4 0.02087 0.0 0.0
1500203 160.0 0.01982 0.0 0.0
1500204 228.0
                 0.0
                     0.0
          0.01982
1500205 230.0
           0.02139
                 0.0
                     0.0
1500206 280.0
           0.02002
                 0.0
                     0.0
1500207 338.5
           0.02002
                 0.0
                     0.0
1500208 349.5
           0.02087
                 0.0
                     0 0
1500209 820.0
           0.02087
                 0.0
                     0.0
```

inlet volume 1600000 inletv snglvol 1600101 0.0155565 0.00381 0.0 0.0 90. 0.00381 1.e-6 0.0097 00 \* test specific pressure and temperature 1600200 3 275790.0 326.2 inlet junction 1650000 inletj sngljun 1650101 160010000 20000000 0.0 1.2 1.2 001000 1650110 0.0097 0. 1. 1. 0.0 1650201 1 0.0 0.0 core 2000000 core pipe 2000001 49 2000101 0.0155565 49 \* vol. flow area \* jun. flow area 2000201 0.0155565 48 \* vol. length 2000301 0.0762 49 

 2000401
 0.0
 49
 \* vol. volume

 2000601
 90.0
 49
 \* vol. vertical orient

 2000701
 0.0762
 49
 \* vol. elevation change

 2000801
 1.e-6
 0.0097
 49
 \* vol. friction data

 \* vol. volume \* vol. vertical orientation
\* ---' \* vol. elevation change \* bottom most grid k --> junction to core inlet 
 2000901
 0.0
 0.0
 6

 2000902
 1.20
 1.20
 7
 \* jun. loss coefficient 2000903 0.0 0.0 13 2000904 1.20 1.20 14 2000905 0.0 0.0 20 2000906 1.20 1.20 21 2000907 0.0 0.0 27 2000908 1.20 1.20 28 2000909 0.0 33 0.0 2000910 1.20 1.20 34 2000911 0.0 0.0 40 2000912 1.20 1.20 41 2000913 0.0 0.0 47 2000914 1.20 1.20 48 2001001 00100 49 \* vol. control flag 2001101 000000 48 \* jun. control flag \* test specific pressure and temperature 2001201 3 275790.0 403.8 0.0 0.0 0.0 1 2001202 3 275790.0 403.8 0.0 0.0 0.0 2 2001203 3 275790.0 415.5 0.0 0.0 0.0 3 2001204 3 275790.0 437.6 0.0 0.0 0.0 4 2001205 3 275790.0 459.6 0.0 0.0 0.0 5 2001206 3 275790.0 481.7 0.0 0.0 0.0 6 2001207 3 275790.0 503.8 2001208 3 275790.0 525.9 0.0 0.0 0.0 7 0.0 0.0 0.0 8 2001209 3 275790.0 548.0 0.0 0.0 0.0 9 2001210 3 275790.0 570.1 0.0 0.0 0.0 10 0.0 2001211 3 275790.0 592.1 0.0 0.0 11 2001212 3 275790.0 614.2 0.0 0.0 0.0 12 2001213 3 275790.0 636.3 0.0 0.0 0.0 13

2001214 3 2001215 3 2001216 3 2001217 3 2001218 3 2001219 3 2001220 3 2001221 3 2001222 3 2001222 3 2001223 3 2001224 3	275790.0 275790.0 275790.0 275790.0 275790.0 275790.0 275790.0 275790.0 275790.0 275790.0 275790.0 275790.0 275790.0	658.4 680.5 702.6 724.6 757.6 790.5 823.5 856.4 856.1 855.9 855.6	0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0	0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0	0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0	14 15 16 17 18 19 20 21 22 23 24		
2001225 3 2001226 3 2001227 3 2001228 3 2001229 3	275790.0 275790.0 275790.0 275790.0 275790.0	855.3 848.8 842.4 835.9 829.4	0.0 0.0 0.0 0.0 0.0	0.0 0.0 0.0 0.0 0.0	0.0 0.0 0.0 0.0 0.0	25 26 27 28 29		
2001230 3 2001231 3 2001232 3 2001233 3	275790.0 275790.0 275790.0 275790.0	800.7 772.0 743.3 714.6	0.0 0.0 0.0 0.0	0.0 0.0 0.0 0.0	0.0 0.0 0.0 0.0	30 31 32 33		
2001234 3 2001235 3 2001236 3 2001237 3 2001238 3	275790.0 275790.0 275790.0 275790.0 275790.0	697.3 680.1 662.9 645.7 625.3	0.0 0.0 0.0 0.0 0.0	0.0 0.0 0.0 0.0 0.0	0.0 0.0 0.0 0.0 0.0	34 35 36 37 38		
2001239 3 2001240 3 2001241 3 2001242 3 2001243 3	275790.0 275790.0 275790.0 275790.0 275790.0	604.9 584.5 564.1 553.9	0.0 0.0 0.0 0.0	0.0 0.0 0.0 0.0	0.0 0.0 0.0 0.0	39 40 41 42		
2001243 3 2001244 3 2001245 3 2001246 3 2001247 3	275790.0 275790.0 275790.0 275790.0 275790.0	543.8 533.7 523.6 513.5 503.4	0.0 0.0 0.0 0.0	0.0 0.0 0.0 0.0	0.0 0.0 0.0 0.0	43 44 45 46 47		
2001248 3 2001249 3 2001300 ( 2001301 ( 2001401 (	275790.0 275790.0 ) ).0 0.0 0	493.2 483.1 .0 48	0.0 0.0 * jui	0.0 0.0 n. ini	0.0 0.0 tial (	48 49 condition		
********* * outle *********	**************************************	********* ****************************	*****	*****	*****	*******	* * * * * * * * * * * * * * * * * * * *	*
* ccfl on * choking * reverse 2500101 2 2500102 2 2500201 (	(f = 1) off (c = flow is r 200010000 1.0 1.0 1 0 0.0 0.0	1) estricted 300000000 1.0 0.0 *********	l (kb 0 0.0	= 1.e6 155565 *****	5) 5 0.0	) 1.e6	101000	*
* upper ********** 3000000 1	plenum *************** plenum t	******** mdpvol	****	* * * * * *	****	* * * * * * * * * *	*****	*
3000101 : 3000200 : * test sp	L.O 1.O O 2 ecific pres	.0 0.0 (	0.0 0	.0 0.	0 0.0	00010		
2000201	0.0 276	1.0.0 T	. 0					

.

3000202	820.0	276550.	0 1.0					
*******	******	* * * * * * * *	******	* * * * * * * *	******	*****	******	*****
* • • •								
^ neat	structu	re						
******	******	******	******	******	******	******	******	*****
* fuol	rod							
********	*******	* * * * * * * *	******	* * * * * * * *	******	******	* * * * * * * *	*****
*								
12001000	49 8 3	2 0 0 0	) 1 1	32				
12001100	0 1	- 0 0.0	, , ,	52				
12001101	2 1.20	650e-3						
12001102	1 2.22	250e-3						
12001103	2 4.11	480e-3						
12001104	2 4.74	980e-3						
12001201	12		* bor	on nitr	ide			
12001202	23		* kan	thal				
12001203	15		* bor	on nitr	ide			
12001204	47		* SS	347				
12001301	0.0 2							
12001302	1.0 3							
12001303	0.0 7							
12001400	-1							
* test sp	ecific i	initial	wall ten	nperatur	e			
12001401	491.9	491.9	491.9	491.9	491.9	491.9	491.9	491.9
12001402	515.3	515.3	515.3	515.3	515.3	515.3	515.3	515.3
12001403	538.8	538.8	538.8	538.8	538.8	538.8	538.8	538.8
12001404	585 6	502.2	502.2 505 6	595 6	585 6	502.2	595 6	595 6
12001405	505.0 609 1	505.0 600 1	505.0 600 1	505.0	505.0	505.0	600 1	505.0
12001400	632 5	632 5	632 5	632 5	632 5	632 5	632 5	632 5
12001408	655.9	655.9	655.9	655.9	655.9	655.9	655.9	655.9
12001409	679.4	679.4	679.4	679.4	679.4	679.4	679.4	679.4
12001410	727.4	727.4	727.4	727.4	727.4	727.4	727.4	727.4
12001411	775.6	775.6	775.6	775.6	775.6	775.6	775.6	775.6
12001412	823.8	823.8	823.8	823.8	823.8	823.8	823.8	823.8
12001413	872.0	872.0	872.0	872.0	872.0	872.0	872.0	872.0
12001414	920.1	920.1	920.1	920.1	920.1	920.1	920.1	920.1
12001415	933.9	933.9	933.9	933.9	933.9	933.9	933.9	933.9
12001416	947.5	947.5	947.5	947.5	947.5	947.5	947.5	947.5
12001417	961.2	961.2	961.2	961.2	961.2	961.2	961.2	961.2
12001418	989.6	989.6	989.6	989.6	989.6	989.6	989.6	989.6
12001419	1017.9	1017.9	1017.9	1017.9	1017.9	1017.9	1017.9	1017.9
12001420	1046.3	1046.3	1046.3	1046.3	1046.3	1046.3	1046.3	1046.3
12001421	1074.7	1074.7	1074.7	1074.7	1074.7	1074.7	1074.7	1074.7
12001422	1104.2	1104.2	1104.2	1104.2	1104.2	1104.2	1104.2	1104.2
12001423	1133.6	1133.6	1133.6	1133.6	1133.6	1133.0	1133.0	1133.0
12001424	1143.8	1143.8	1143.8	1143.8	1143.8	1143.8	1143.8	1143.8
12001425	1120.3	1120.3	1120.3	1120.3	1120.3	1120.3	1120.0	1120.3
12001420	1115 2	1115 0	1115 2	1115 2	1115 2	1115 2	1115 2	1129.0
12001427	1106 1	1106 1	1106 1	1106 1	1106 1	1106 1	1106 1	1106 1
12001420	1097 0	1097 0	1097 0	100.1	1007 0	1007 0	1097 0	1097 0
12001430	1085 0	1085 0	1085 0	1085 0	1085 0	1085 0	1085 0	1085 0
12001431	1072.9	1072 9	1072 9	1072.9	1072.9	1072.9	1072.9	1072.9
12001432	1031.4	1031.4	1031.4	1031.4	1031.4	1031.4	1031.4	1031.4
12001433	989.8	989.8	989.8	989.8	989.8	989.8	989.8	989.8

,

e
12001434	933.5	933.5	933	.5	933.5	933.5	933.5	933.5	933.5
12001435	877.1	877.1	877	.1	877.1	877.1	877.1	877.1	877.1
12001436	862.2	862.2	862	.2	862.2	862.2	862.2	862.2	862.2
12001437	847.4	847.4	847	. 4	847.4	847.4	847.4	847.4	847.4
12001438	832.4	832.4	832	. 4	832.4	832.4	832.4	832 4	832.4
12001439	795 4	795 4	795	4	795 4	795 4	795 4	795 4	795 4
12001440	758 4	758 /	758		758 /	758 /	750 /	759 /	758 /
12001440	721 4	721 4	721		721 4	701 A	721 /	701 /	701 A
12001442	609 3	600 3	609	• • • •	600 2	600 2	600 2	/21.4 609 2	609 2
12001332	675 0	675 0	675	. J 2	675 2	675 2	090.3	090.3	090.3
12001443	652 1	652 1	650	• 2	652 1	675.2	675.2	675.2	675.2
12001444	630 0	652.1	632	.1	620 0	652.1	652.1	652.1	652.1
12001445	629.0	029.0	029	.0	029.0	629.0	629.0	629.0	629.0
12001446	614.8	614.8	614	.8	614.8	614.8	614.8	614.8	614.8
12001447	500.5	500.6	600	. 0	500.6	600.6	600.6	600.6	600.6
12001448	586.4	586.4	586	.4	586.4	586.4	586.4	586.4	586.4
12001449	5/2.2	5/2.2	5/2	.2	572.2	572.2	572.2	572.2	572.2
* test sp	ecific .	length	(159	rods	)				
12001501	0 0 0	1 6	.05790	) 1					
12001502	0 0 0	1 12	.11580	) 48					
12001503	0 0 0	1 6	.05790	) 49					
12001601	2000100	00 0	1	. 1	6.0579	0 1			
12001602	2000200	000 10	000	1 1	12.115	80 48			
12001603	2004900	0 00	1	. 1	6.0579	0 49			
* test sp	ecific p	power f	Eracti	on					
12001701	200 0.0	0447	0.0	0.0	1				
12001702	200 0.0	0895	0.0	0.0	2				
12001703	200 0.0	0895	0.0	0.0	3				
12001704	200 0.0	0895	0.0	0.0	4				
12001705	200 0.0	0895	0.0	0.0	5				
12001706	200 0.0	0895	0.0	0.0	6				
12001707	200 0.0	0895	0.0	0.0	7				
12001708	200 0.0	01051	0.0	0.0	8				
12001709	200 0.0	)1415	0.0	0.0	9				
12001710	200 0.0	1415	0.0	0.0	10				
12001711	200 0 0	1790	0 0	0 0	11				
12001712	200 0 0	1832	0 0	0 0	12				
12001713	200 0.0	12071	0.0	0.0	13				
12001714	200 0.0	12310	0.0	0.0	14				
12001715	200 0.0	12310	0.0	0.0	15				
12001715	200 0.0	12343	0.0	0.0	16				
12001717	200 0.0	2700	0.0	0.0	17				
12001719	200 0.0	12/00	0.0	0.0	10				
12001710	200 0.0	2981	0.0	0.0	18				
12001719	200 0.0	13101	0.0	0.0	19				
12001720	200 0.0	3172	0.0	0.0	20				
12001/21	200 0.0	13330	0.0	0.0	21				
12001722	200 0.0	13330	0.0	0.0	22				
12001723	200 0.0	3443	0.0	0.0	23				
12001724	200 0.0	3455	0.0	0.0	24				
12001/25	200 0.0	3455	0.0	0.0	25				
12001726	200 0.0	3455	0.0	0.0	26				
12001727	200 0.0	3442	0.0	0.0	27				
12001728	200 0.0	3330	0.0	0.0	28				
12001729	200 0.0	3330	0.0	0.0	29				
12001730	200 0.0	3168	0.0	0.0	30				
12001731	200 0.0	3101	0.0	0.0	31				
12001732	200 0.0	2984	0.0	0.0	32				
12001733	200 0.0	2706	0.0	0.0	33				

12001734	200 0.027	06 0.0	0.0	34					
12001735	200 0.023	350 0.0	0.0	35					
12001736	200 0.023	810 0.0	0.0	36					
12001737	200 0.020	0.0	0.0	37					
12001738	200 0.018	32 0.0	0.0	38					
12001739	200 0.017	90 0.0	0.0	39					
12001740	200 0.014	15 0.0	0.0	40					
12001741	200 0.014	15 0 0	0.0	41					
12001742	200 0 010	151 0.0	0.0	42					
12001742	200 0.010	95 0.0	0.0	43					
12001745		295 0.0	0.0	4.5					
12001745		295 0.0	0.0	45					
12001745		95 0.0	0.0	15					
12001740		0.0	0.0	40					
12001747		0.0	0.0	47					
12001740	200 0.008	0.0	0.0	40					
12001/49		14/ 0.0	0.0	49					
^ add1t10	onal left i	boundary			0 0 0 1	0 40			
12001801	0.0094996	, 20.0 4	20.0 0.0	0.0 0.	.0 0.0 1	.0 49			
* additio	onal right	boundary				1 00 1	20	0 04050	-
12001901	0.0094996	0.03810	3.65/60	0.00000	0.48890	1.20 1	.20	0.24250	T
12001902	0.0094996	0.07620	3.58140	0.07620	0.412/0	1.20 1	.20	0.29458	2
12001903	0.0094996	0.15240	3.50520	0.15240	0.33650	1.20 1	.20	0.3466/	3
12001904	0.0094996	0.22860	3.42900	0.22860	0.26030	1.20 1	.20	0.39875	4
12001905	0.0094996	0.30480	3.35280	0.30480	0.18410	1.20 1	.20	0.45083	5
12001906	0.0094996	0.38100	3.27660	0.38100	0.10790	1.20 1	.20	0.50292	6
12001907	0.0094996	0.45720	3.20040	0.45720	0.03170	1.20 1	.20	0.55500	7
12001908	0.0094996	0.53340	3.12420	0.00000	0.46360	1.20 1	.20	0.60708	8
12001909	0.0094996	0.60960	3.04800	0.07620	0.38740	1.20 1	.20	0.65917	9
12001910	0.0094996	0.68580	2.97180	0.15240	0.31120	1.20 1	.20	0.73000	10
12001911	0.0094996	0.76200	2.89560	0.22860	0.23500	1.20 1	.20	0.81333	11
12001912	0.0094996	0.83820	2.81940	0.30480	0.15880	1.20 1	.20	0.89917	12
12001913	0.0094996	0.91440	2.74320	0.38100	0.08260	1.20 1	.20	0.99500	13
12001914	0.0094996	0.99060	2.66700	0.45720	0.00640	1.20 1	.20	1.09083	14
12001915	0.0094996	1.06680	2.59080	0.02530	0.46360	1.20 1	.20	1.17333	15
12001916	0.0094996	1.14300	2.51460	0.10150	0.38740	1.20 1	.20	1.25249	16
12001917	0.0094996	1.21920	2.43840	0.17770	0.31120	1.20 1	.20	1.33166	17
12001918	0.0094996	1.29540	2.36220	0.25390	0.23500	1.20 1	.20	1.41082	18
12001919	0.0094996	1.37160	2.28600	0.33010	0.15880	1.20 1	.20	1.48998	19
12001920	0.0094996	1.44780	2.20980	0.40630	0.08260	1.20 1	.20	1.53583	20
12001921	0.0094996	1.52400	2.13360	0.48250	0.00640	1.20 1	.20	1.58168	21
12001922	0.0094996	1.60020	2.05740	0.02530	0.46360	1.20 1	.20	1.61000	22
12001923	0.0094996	1.67640	1.98120	0.10150	0.38740	1.20 1	.20	1.62667	23
12001924	0.0094996	1.75260	1.90500	0.17770	0.31120	1.20 1	.20	1.64334	24
12001925	0.0094996	1.82880	1.82880	0.25390	0.23500	1.20 1	.20	1.66000	25
12001926	0.0094996	1,90500	1.75260	0.33010	0.15880	1.20 1	.20	1.64333	26
12001927	0.0094996	1.98120	1 67640	0.40630	0.08260	1.20 1	.20	1.62667	27
12001928	0 0094996	2.05740	1 60020	0.48250	0.00640	1.20 1	.20	1.61000	28
12001920	0.0094996	2 13360	1 52400	0.02530	0.00010	1 20 1	20	1 58167	29
12001929	0.0094996	2 20980	1 44780	0.10150	0.36210	1 20 1	20	1 53584	30
12001931	0.0091996	2 28600	1 37160	0.10100	0 28590	1 20 1	20	1 49001	31
12001931	0 0004990	2 36220	1 29540	0 25390	0 20970	1 20 1	20	1 41084	32
12001932	0.0094990	2 12010	1 21020	0.20000	0.20970	1 20 1	20	1 33165	33
12001933	0.0094990	2 51/20	1 1/200	0.32010	0.10000	1 20 1	20	1 25240	34
12001334	0.0034330	2.01400	1 06600	0.40050	0.00/00	1 20 1	20	1 17220	35
12001030	0.0034336	2.09000	1.00000	0.40200	0.77420	1 20 1	.20	1 00000	32
12001027	0.0094996	2.00/00	0.99000	0.00090	0.43030	1 20 1	.20	1.03002	20
12001020	0.0094996	2.14320	0.91440	0.17.110	0.30200	1 20 1	.20	0.22439	20
TS001338	0.0094996	2.81940	0.03820	0.20330	0.20590	1.20 I	.20	0.0331/	30

12001939 0.0094996 2.89560 0.76200 0.27950 0.20970 1.20 1.20 0.81334 39 12001940 0.0094996 2.97180 0.68580 0.35570 0.13350 1.20 1.20 0.73001 40 12001941 0.0094996 3.04800 0.60960 0.43190 0.05730 1.20 1.20 0.65917 41 12001942 0.0094996 3.12420 0.53340 0.50810 0.48890 1.20 1.20 0.60709 42 12001943 0.0094996 3.20040 0.45720 0.05090 0.41270 1.20 1.20 0.55501 43 12001944 0.0094996 3.27660 0.38100 0.12710 0.33650 1.20 1.20 0.50293 44 12001945 0.0094996 3.35280 0.30480 0.20330 0.26030 1.20 1.20 0.45085 45 12001946 0.0094996 3.42900 0.22860 0.27950 0.18410 1.20 1.20 0.39877 46 12001947 0.0094996 3.50520 0.15240 0.35570 0.10790 1.20 1.20 0.34668 47 12001948 0.0094996 3.58140 0.07620 0.43190 0.03170 1.20 1.20 0.29460 48 12001949 0.0094996 3.65760 0.03810 0.00000 20.0000 1.20 1.20 0.24252 49 housing 12002000 49 3 2 0 0.097 0 0 12002100 0 1 12002101 2 0.10208 12002201 3 2 \* ss 304 12002301 0. 2 12002400 -1 \* test specific initial wall temperature 12002401368.2368.2368.212002402372.2372.2372.212002403376.2376.2376.2 12002404 380.2 380.2 380.2 12002405 384.2 384.2 384.2 12002406 388.2 388.2 388.2 12002407 392.2 392.2 392.2 12002408 396.2 396.2 396.2 12002409 400.2 400.2 400.2 12002410 404.2 404.2 404.2 12002411 408.2 408.2 408.2 12002412 412.2 412.2 412.2 12002413 416.2 416.2 416.2 12002414 420.2 420.2 420.2 12002415 424.2 424.2 424.2 12002416 428.2 428.2 428.2 12002417 432.2 432.2 432.2 12002418 435.8 435.8 435.8 12002419 439.4 439.4 439.4 12002420 443.0 443.0 443.0 12002421 446.7 446.7 446.7 12002422 450.3 450.3 450.3 12002423 453.9 453.9 453.9 12002424 457.5 457.5 457.5 12002425 461.2 461.2 461.2 12002426 458.4 458.4 458.4 12002427 455.7 455.7 455.7 12002428 452.9 452.9 452.9 450.2 12002429 450.2 450.2 12002430 447.4 447.4 447.4 12002431 444.7 444.7 444.7 12002432 441.9 441.9 441.9 12002433 439.2 439.2 439.2 12002434 436.9 436.9 436.9 12002435 434.7 434.7 434.7 12002436 432.4 432.4 432.4

```
12002437 430.2 430.2 430.2

        12002437
        430.2
        430.2

        12002438
        425.9
        425.9

        12002439
        421.7
        421.7

        12002440
        417.4
        417.4

                                  425.9
                                  421.7
                                  417.4
12002441 413.2 413.2
                                  413.2
12002442 411.9 411.9
                                  411.9
12002443 410.7 410.7
                                  410.7
12002444 409.4 409.4 409.4
12002445 408.2 408.2 408.2
12002446 406.9 406.9 406.9
12002447 405.7 405.7 405.7
12002448 404.4 404.4 404.4
12002449 403.2 403.2 403.2
12002501 200010000 0 1 1 0.0381 1
12002502 200020000 10000 1 1 0.0762 48
12002503 200490000 0 1 1 0.0381 49

      12002503
      200490000
      0
      1
      1
      0.0381
      49

      12002601
      0
      0
      1
      0.0381
      1

      12002602
      0
      0
      1
      0.0762
      48

      12002603
      0
      0
      1
      0.0381
      49

      12002701
      0
      0.0
      0.0
      49

      12002801
      0.0
      20.0
      20.0
      0.0
      0.0
      1.0
      49

12002901 0.0 20.0 20.0 0.0 0.0 0.0 0.0 1.0 49
thimbles
12003000 49 3 2 0 0.005461 0 1
12003100 0 1
12003101 2 0.0060198
12003201 3 2
                                         * ss 304
12003301 0.0 2
12003400 -1
* test specific initial wall temp.
12003401 368.2 368.2 368.2

        12003401
        368.2
        368.2
        368.2
        368.2

        12003402
        372.2
        372.2
        372.2

        12003403
        376.2
        376.2
        376.2

        12003404
        380.2
        380.2
        380.2

        12003405
        384.2
        384.2
        384.2

12003406 388.2 388.2 388.2
12003407 392.2 392.2 392.2
12003408 396.2 396.2 396.2
12003409 400.2 400.2 400.2
12003410 404.2 404.2 404.2
12003411 408.2 408.2 408.2
12003412 412.2 412.2 412.2
12003413 416.2 416.2 416.2
12003414 420.2 420.2 420.2
12003415 424.2 424.2 424.2
12003416 428.2 428.2 428.2
12003417 432.2 432.2 432.2
12003418 435.8 435.8 435.8
12003419 439.4 439.4 439.4
12003420 443.0 443.0
                                  443.0
                       446.7 446.7
12003421 446.7
12003422 450.3 450.3 450.3
12003423 453.9 453.9 453.9
12003424 457.5 457.5 457.5
12003425 461.2 461.2 461.2
```

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12003426 458.4 458.4 458.4
12003427 455.7 455.7 455.7
12003428 452.9 452.9 452.9
12003429 450.2 450.2 450.2
12003430 447.4 447.4
                        447.4
                444.7
12003431 444.7
                        444.7
12003432 441.9 441.9 441.9
12003433 439.2 439.2 439.2
12003434 436.9 436.9 436.9
12003435 434.7 434.7 434.7
12003436 432.4 432.4 432.4
12003437 430.2 430.2 430.2
12003438 425.9 425.9 425.9
12003439 421.7 421.7 421.7
12003440 417.4 417.4 417.4
12003441 413.2 413.2 413.2
12003442 411.9 411.9 411.9
12003443 410.7 410.7 410.7
12003444 409.4
                409.4 409.4
12003445 408.2
                408.2 408.2
                406.9 406.9
12003446 406.9
12003447 405.7
                405.7 405.7
12003448 404.4 404.4 404.4
12003449 403.2 403.2 403.2
* 16 thimbles
12003501 0 0 0 1 0.6096 1
12003502 0 0 0 1 1.2192 48
12003503 0 0 0 1 0.6096 49
12003601 200010000 0 1 1 0.6096 1
12003602 200020000 10000 1 1 1.2192 48
12003603 200490000 0 1 1 0.6096 49
12003701 0 0.0 0.0 0.0 49
12003801 0.0 20.0 20.0 0.0 0.0 0.0 0.0 1.0 49
12003901 0.0 20.0 20.0 0.0 0.0 0.0 0.0 1.0 49
fillers
*****
12004000 49 3 2 0 0.0 0 1
12004100 0 1
12004101 2 0.005022
12004201 3 2
12004301 0.0 2
12004400 -1
* test specific initial wall temp.
12004401 368.2 368.2 368.2
12004402 372.2 372.2 372.2
12004403 376.2 376.2 376.2
12004404 380.2 380.2 380.2

        12004404
        380.2
        380.2
        380.2
        380.2

        12004405
        384.2
        384.2
        384.2

        12004406
        388.2
        388.2
        388.2

        12004407
        392.2
        392.2
        392.2

        12004408
        396.2
        396.2
        396.2

12004409 400.2 400.2 400.2
12004410 404.2 404.2 404.2
12004411 408.2 408.2 408.2
12004412 412.2 412.2 412.2
12004413 416.2 416.2 416.2
```

```
12004414 420.2 420.2 420.2

        12004414
        420.2
        420.2

        12004415
        424.2
        424.2

        12004416
        428.2
        428.2

        12004417
        432.2
        432.2

        12004418
        435.8
        435.8

                       424.2
                       428.2
                       432.2
                       435.8
12004419 439.4 439.4 439.4
12004420 443.0 443.0 443.0
12004421 446.7 446.7 446.7
12004422 450.3 450.3 450.3
12004423 453.9 453.9 453.9
12004424 457.5 457.5 457.5
12004425 461.2 461.2 461.2
12004426 458.4 458.4 458.4
12004427 455.7 455.7 455.7
12004428 452.9 452.9 452.9
12004429 450.2 450.2 450.2
12004430 447.4 447.4
                       447.4
12004431 444.7 444.7
                       444.7
12004432 441.9 441.9
                       441.9
12004433 439.2 439.2
                       439.2
12004434 436.9 436.9
                       436.9
12004435 434.7 434.7
                       434.7
12004436 432.4 432.4 432.4
12004437 430.2 430.2 430.2
12004438 425.9 425.9 425.9
12004439 421.7 421.7 421.7
12004440 417.4 417.4 417.4
12004441 413.2 413.2 413.2
12004442 411.9 411.9 411.9
12004443 410.7 410.7 410.7
12004444 409.4 409.4 409.4
12004445 408.2 408.2
                       408.2
12004446 406.9 406.9 406.9
12004447 405.7 405.7 405.7
12004448 404.4 404.4
                       404.4
12004449 403.2 403.2
                       403.2
* 8 fillers
12004501 0 0 0 1 0.3048 1
12004502 0 0 0 1 0.6096 48
12004503 0 0 0 1 0.3048 49
12004601 200010000 0 1 1 0.3048 1
12004602 200020000 10000 1 1 0.6096 48
12004603 200490000 0
                       1 1 0.3048 49
12004701 0 0.0 0.0 0.0 49
12004801 0.0 20.0 20.0 0.0 0.0 0.0 0.0 1.0 49
12004901 0.0 20.0 20.0 0.0 0.0 0.0 0.0 1.0 49
failed rods
12005000 49 8 2 0 0.0 0 1
12005100 0 1
12005101 2 1.20650e-3
12005102 1 2.22250e-3
12005103 2 4.11480e-3
12005104 2 4.74980e-3
12005201 1 2
                          * boron nitride
12005202 2 3
                         * kanthal
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126
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12005203	1 5		* bor	on nitr	ide			
12005204	4 7		* SS	347				
12005301	0.0 2	2						
12005302	1.0	3						
12005303	0.0	7						
12005400	-1							
* test sp	pecific	initial	wall ter	nperatu	re			
12005401	368.2	368.2	368.2	368.2	368.2	368.2	368.2	368.2
12005402	372.2	372.2	372.2	372.2	372.2	372.2	372.2	372.2
12005403	376.2	376.2	376.2	376.2	376.2	376.2	376.2	376.2
12005404	380.2	380.2	380.2	380.2	380.2	380.2	380.2	380.2
12005405	384.2	384.2	384.2	384.2	384.2	384.2	384.2	384.2
12005406	388.2	388.2	388.2	388.2	388.2	388.2	388.2	388.2
12005407	392.2	392.2	392.2	392.2	392.2	392.2	392.2	392.2
12005408	396.2	396.2	396.2	396.2	396.2	396.2	396.2	396.2
12005409	400.2	400.2	400.2	400.2	400.2	400.2	400.2	400.2
12005410	404.2	404.2	404.2	404.2	404.2	404.2	404.2	404.2
12005411	408.2	408.2	408.2	408.2	408.2	408.2	408.2	408.2
12005412	412.2	412.2	412.2	412.2	412.2	412.2	412.2	412.2
12005413	416.2	416.2	416.2	416.2	416.2	416.2	416.2	416.2
12005414	420.2	420.2	420.2	420.2	420.2	420.2	420.2	420.2
12005415	424.2	424.2	424.2	424.2	424.2	424.2	424.2	424.2
12005416	428.2	428.2	428.2	428.2	428.2	428.2	428.2	428.2
12005417	432.2	432.2	432.2	432.2	432.2	432.2	432.2	432.2
12005418	435.8	435.8	435.8	435.8	435.8	435.8	435.8	435.8
12005419	439.4	439.4	439.4	439.4	439.4	439.4	439.4	439.4
12005420	443.0	443.0	443.0	443.0	443.0	443.0	443.0	443.0
12005421	446.7	446.7	446.7	446.7	446.7	446.7	446.7	446.7
12005422	450.3	450.3	450.3	450.3	450.3	450.3	450.3	450.3
12005423	453.9	453.9	453.9	453.9	453.9	453.9	453.9	453.9
12005424	457.5	457.5	457.5	457.5	457.5	457.5	457.5	457.5
12005425	461.2	461.2	461.2	461.2	461.2	461.2	461.2	461.2
12005426	458.4	458.4	458.4	458.4	458.4	458.4	458.4	458.4
12005427	455.7	455.7	455.7	455.7	455.7	455.7	455.7	455.7
12005428	452.9	452.9	452.9	452.9	452.9	452.9	452.9	452.9
12005429	450.2	450.2	450.2	450.2	450.2	450.2	450.2	450.2
12005430	447.4	447.4	447.4	447.4	447.4	447.4	447.4	447.4
12005431	444.7	444.7	444.7	444.7	444.7	444.7	444.7	444.7
12005432	441.9	441.9	441.9	441.9	441.9	441.9	441.9	441.9
12005433	439.2	439.2	439.2	439.2	439.2	439.2	439.2	439.2
12005434	436.9	436.9	436.9	436.9	436.9	436.9	436.9	436.9
12005435	434.7	434.7	434.7	434.7	434.7	434.7	434.7	434.7
12005436	432.4	432.4	432.4	432.4	432.4	432.4	432.4	432.4
12005437	430.2	430.2	430.2	430.2	430.2	430.2	430.2	430.2
12005438	425.9	425.9	425.9	425.9	425.9	425.9	425.9	425.9
12005439	421.7	421.7	421.7	421.7	421.7	421.7	421.7	421.7
12005440	417.4	417.4	417.4	417.4	417.4	417.4	417.4	417.4
12005441	413.2	413.2	413.2	413.2	413.2	413.2	413.2	413.2
12005442	411.9	411.9	411.9	411.9	411.9	411.9	411.9	411.9
12005443	410.7	410.7	410.7	410.7	410.7	410.7	410.7	410.7
12005444	409.4	409.4	409.4	409.4	409.4	409.4	409.4	409.4
12005445	408.2	408.2	408.2	408.2	408.2	408.2	408.2	408.2
12005446	406.9	406.9	406.9	406.9	406.9	406.9	406.9	406.9
12005447	405.7	405.7	405.7	405.7	405.7	405.7	405.7	405.7
12005448	404.4	404.4	404.4	404.4	404.4	404.4	404.4	404.4
12005449	403.2	403.2	403.2	403.2	403.2	403.2	403.2	403.2
- TAST ST			(/ rode)					

\* test specific length (2 rods)

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12005501 0 0 0 1 0.0762 1
12005502 0 0 0 1 0.1524 48
12005503 0 0 0 1 0.0762 49
12005601 200010000 0
                   1 1 0.0762 1
12005602 200020000 10000 1 1 0.1524 48
12005603 200490000 0 1 1 0.0762 49
12005701 0 0.0 0.0 0.0 49
12005801 0.0 20.0 20.0 0.0 0.0 0.0 0.0 1.0 49
12005901 0.0 20.0 20.0 0.0 0.0 0.0 0.0 1.0 49
*
   heat structure thermal properties
20100100 tbl/fctn 1 1 * boron nitride
20100200 tbl/fctn 1 1
                   * kanthal
20100300 tbl/fctn 1 1
                   * ss-304
20100400 tbl/fctn 1 1 * ss-347
conductivity
* boron nitride
20100101 300.0 25.16
20100102 400.0 24.91
20100103 500.0 24.67
20100104 600.0 24.42
20100105 700.0 24.18
20100106 800.0 23.93
20100107 900.0 23.69
20100108 1000.0 23.44
20100109 1100.0 23.19
20100110 1200.0 22.95
20100111 1300.0 22.70
20100112 1400.0 22.46
20100113 1500.0 22.21
20100114 1600.0 21.97
20100115 1700.0 21.72
* kanthal + boron nitride
20100201 300.0 29.13
20100202 400.0 29.61
            30.09
20100203 500.0
20100204 600.0 30.58
20100205 700.0 31.06
20100206 800.0 31.54
20100207 900.0 32.02
20100208 1000.0 32.51
20100209 1100.0 32.99
20100210 1200.0 33.47
20100211 1300.0 33.96
20100212 1400.0 34.44
20100213 1500.0 34.92
20100214 1600.0 35.41
20100215 1700.0 35.89
* ss-304
20100301 300.0 12.97
20100302 400.0 14.59
20100303 500.0 16.21
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20100304	600.0	17.82
20100305	700.0	19.44
20100306	800.0	21.06
20100307	900.0	22.68
20100308	1000.0	24.30
20100309	1100.0	25.91
20100310	1200.0	27.53
20100311	1300.0	29.15
20100312	1400.0	30.77
20100313	1500.0	32.39
20100314	1600.0	34.00
20100315	1700.0	35.62
*ss-347		
20100401	300.0	16.37
20100402	400.0	17.10
20100403	500.0	17.83
20100404	600.0	18.55
20100405	700.0	19.28
20100406	800.0	20.01
20100407	900.0	20.73
20100408	1000.0	21.46
20100409	1100.0	22.19
20100410	1200.0	22.92
20100411	1300.0	23.64
20100412	1400.0	24.37
20100413	1500.0	25.10
20100414	1600.0	25.82
20100415	1700.0	26.55
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	*****	***************************************
* volum	netric h	eat capacity
* volun ********	**************************************	eat capacity ************************************
* volum ********** *boron ni	tride	eat capacity
* volun ********* *boron ni 20100151	tride 300.0	eat capacity ************************************
* volum ********* *boron ni 20100151 20100152	tride 300.0 400.0	eat capacity ************************************
* volum ********* *boron ni 20100151 20100152 20100153	tride 300.0 400.0 500.0	eat capacity ************************************
* volum ******** *boron ni 20100151 20100152 20100153 20100154	tride 300.0 400.0 500.0 600.0	eat capacity ************************************
* volum ********* *boron ni 20100151 20100152 20100153 20100155 20100155	tride 300.0 400.0 500.0 600.0 700.0	eat capacity ************************************
* volum ********* *boron ni 20100151 20100152 20100153 20100154 20100155 20100156	tride 300.0 400.0 500.0 600.0 700.0 800.0	eat capacity ************************************
* volun ********* *boron ni 20100151 20100152 20100153 20100154 20100155 20100156 20100157 20100158	tride 300.0 400.0 500.0 600.0 700.0 800.0 900.0	eat capacity ************************************
* volun ******** *boron ni 20100151 20100152 20100153 20100154 20100155 20100156 20100157 20100158	tride 300.0 400.0 500.0 600.0 700.0 800.0 900.0 1000.0	eat capacity ************************************
* volum ******** *boron ni 20100151 20100152 20100153 20100154 20100155 20100156 20100157 20100158 20100159 20100160	tride 300.0 400.0 500.0 600.0 700.0 800.0 900.0 1000.0 1200 0	eat capacity ************************************
* volum ******** *boron ni 20100151 20100152 20100153 20100154 20100155 20100156 20100157 20100158 20100159 20100160 20100161	tride 300.0 400.0 500.0 600.0 700.0 800.0 900.0 1000.0 1100.0 1300.0	<pre>eat capacity ************************************</pre>
* volum ******** *boron ni 20100151 20100152 20100153 20100154 20100155 20100156 20100157 20100158 20100159 20100160 20100161 20100162	tride 300.0 400.0 500.0 600.0 700.0 800.0 900.0 1000.0 1100.0 1200.0 1400.0	<pre>eat capacity ************************************</pre>
* volum ********* *boron ni 20100151 20100152 20100153 20100154 20100155 20100156 20100157 20100158 20100159 20100160 20100161 20100162 20100163	tride 300.0 400.0 500.0 600.0 700.0 800.0 900.0 1000.0 1100.0 1200.0 1300.0 1400.0	<pre>eat capacity</pre>
* volum ******** *boron ni 20100151 20100152 20100153 20100154 20100155 20100156 20100157 20100158 20100159 20100160 20100161 20100162 20100163 20100164	tride 300.0 400.0 500.0 600.0 700.0 800.0 900.0 1000.0 1100.0 1200.0 1300.0 1400.0 1600.0	<pre>eat capacity</pre>
* volum ******** *boron ni 20100151 20100152 20100153 20100154 20100155 20100156 20100157 20100158 20100159 20100160 20100161 20100162 20100163 20100164 20100165	tride 300.0 400.0 500.0 600.0 700.0 800.0 900.0 1000.0 1100.0 1200.0 1300.0 1400.0 1500.0 1600.0	<pre>eat capacity ************************************</pre>
* volum ******** *boron ni 20100151 20100152 20100153 20100154 20100155 20100156 20100157 20100158 20100159 20100160 20100161 20100162 20100163 20100164 20100165 * kanthal	tride 300.0 400.0 500.0 600.0 700.0 800.0 900.0 1000.0 1100.0 1200.0 1300.0 1400.0 1500.0 1600.0 + borof	<pre>eat capacity 1.80040e6 2.34650e6 2.79610e6 3.16050e6 3.45090e6 3.67820e6 3.85370e6 3.98860e6 4.09380e6 4.18070e6 4.26020e6 4.34360e6 4.44200e6 4.56650e6 4.72820e6 n nitride</pre>
* volum ******** *boron ni 20100151 20100152 20100153 20100154 20100155 20100156 20100157 20100158 20100159 20100160 20100161 20100162 20100163 20100164 20100165 * kanthal 20100251	tride 300.0 400.0 500.0 600.0 700.0 800.0 900.0 1000.0 1100.0 1200.0 1300.0 1400.0 1500.0 1600.0 1700.0 + boror 300.0	<pre>eat capacity 1.80040e6 2.34650e6 2.79610e6 3.16050e6 3.45090e6 3.67820e6 3.85370e6 3.98860e6 4.09380e6 4.18070e6 4.26020e6 4.34360e6 4.34360e6 4.44200e6 4.56650e6 4.72820e6 n nitride 2.29400e6</pre>
<pre>* volun ******** *boron ni 20100151 20100152 20100153 20100154 20100155 20100156 20100157 20100158 20100159 20100160 20100161 20100162 20100163 20100164 20100165 * kanthal 20100251 20100252</pre>	tride 300.0 400.0 500.0 600.0 700.0 800.0 900.0 1000.0 1100.0 1200.0 1300.0 1400.0 1500.0 1600.0 1700.0 + boror 300.0	<pre>eat capacity 1.80040e6 2.34650e6 2.79610e6 3.16050e6 3.45090e6 3.67820e6 3.98860e6 4.09380e6 4.18070e6 4.26020e6 4.34360e6 4.34360e6 4.44200e6 4.56650e6 4.72820e6 n nitride 2.29400e6 2.92790e6</pre>
<pre>* volun ******** *boron ni 20100151 20100152 20100153 20100154 20100155 20100156 20100157 20100158 20100159 20100160 20100161 20100162 20100163 20100164 20100165 * kanthal 20100251 20100252 20100253</pre>	tride 300.0 400.0 500.0 600.0 700.0 800.0 900.0 1000.0 1100.0 1200.0 1300.0 1400.0 1500.0 1600.0 1700.0 + boror 300.0 400.0 500.0	<pre>eat capacity 1.80040e6 2.34650e6 2.79610e6 3.16050e6 3.45090e6 3.67820e6 3.85370e6 3.98860e6 4.09380e6 4.18070e6 4.26020e6 4.34360e6 4.44200e6 4.56650e6 4.72820e6 n nitride 2.29400e6 2.92790e6 3.26520e6</pre>
<pre>* volun ******** *boron ni 20100151 20100152 20100153 20100154 20100155 20100156 20100157 20100158 20100160 20100161 20100162 20100163 20100164 20100165 * kanthal 20100251 20100252 20100253 20100254</pre>	tride 300.0 400.0 500.0 600.0 700.0 800.0 900.0 1000.0 1100.0 1200.0 1400.0 1500.0 1600.0 1700.0 + boror 300.0 400.0 500.0 600.0	<pre>eat capacity 1.80040e6 2.34650e6 2.79610e6 3.16050e6 3.45090e6 3.67820e6 3.85370e6 3.98860e6 4.09380e6 4.18070e6 4.26020e6 4.34360e6 4.44200e6 4.56650e6 4.72820e6 n nitride 2.29400e6 2.92790e6 3.26520e6 3.50590e6</pre>
<pre>* volun ******** *boron ni 20100151 20100152 20100153 20100154 20100155 20100156 20100157 20100158 20100160 20100161 20100162 20100163 20100164 20100165 * kanthal 20100251 20100251 20100254 20100254</pre>	tride 300.0 400.0 500.0 600.0 700.0 800.0 900.0 1000.0 1100.0 1200.0 1300.0 1400.0 1500.0 1600.0 1700.0 + boror 300.0 400.0 500.0 600.0 700.0	<pre>eat capacity 1.80040e6 2.34650e6 2.79610e6 3.16050e6 3.45090e6 3.67820e6 3.98860e6 4.09380e6 4.18070e6 4.26020e6 4.34360e6 4.44200e6 4.56650e6 4.72820e6 n nitride 2.29400e6 2.92790e6 3.26520e6 3.50590e6 3.69620e6</pre>
<pre>* volun ******** *boron ni 20100151 20100152 20100153 20100154 20100155 20100156 20100157 20100158 20100160 20100161 20100162 20100163 * kanthal 20100251 20100252 20100253 20100254 20100255</pre>	tride 300.0 400.0 500.0 600.0 700.0 800.0 900.0 1000.0 1100.0 1200.0 1300.0 1400.0 1500.0 1600.0 1700.0 + boror 300.0 400.0 500.0 600.0 700.0 800.0	<pre>eat capacity 1.80040e6 2.34650e6 2.79610e6 3.16050e6 3.45090e6 3.67820e6 3.85370e6 3.98860e6 4.09380e6 4.18070e6 4.26020e6 4.34360e6 4.44200e6 4.56650e6 4.72820e6 n nitride 2.29400e6 2.92790e6 3.26520e6 3.50590e6 3.69620e6 3.85510e6</pre>
<pre>* volun ******** *boron ni 20100151 20100152 20100153 20100154 20100155 20100156 20100157 20100158 20100160 20100161 20100162 20100163 20100164 20100165 * kanthal 20100251 20100252 20100253 20100254 20100255 20100256 20100257</pre>	tride 300.0 400.0 500.0 600.0 700.0 800.0 900.0 1000.0 1100.0 1200.0 1300.0 1400.0 1500.0 1600.0 1700.0 + boror 300.0 400.0 500.0 600.0 700.0 800.0 900.0	<pre>eat capacity 1.80040e6 2.34650e6 2.79610e6 3.16050e6 3.45090e6 3.67820e6 3.85370e6 3.98860e6 4.09380e6 4.18070e6 4.26020e6 4.34360e6 4.44200e6 4.56650e6 4.72820e6 n nitride 2.29400e6 2.92790e6 3.26520e6 3.69620e6 3.85510e6 3.99240e6</pre>
<pre>* volun ******** *boron ni 20100151 20100152 20100153 20100154 20100155 20100156 20100157 20100158 20100160 20100161 20100162 20100163 20100164 20100165 * kanthal 20100251 20100252 20100253 20100254 20100255 20100256 20100257 20100258</pre>	tride 300.0 400.0 500.0 600.0 700.0 800.0 900.0 1000.0 1100.0 1200.0 1300.0 1400.0 1500.0 1400.0 1500.0 400.0 500.0 600.0 700.0 800.0 900.0 1000.0	<pre>eat capacity 1.80040e6 2.34650e6 2.79610e6 3.16050e6 3.45090e6 3.67820e6 3.85370e6 3.98860e6 4.09380e6 4.18070e6 4.26020e6 4.34360e6 4.44200e6 4.56650e6 4.72820e6 n nitride 2.29400e6 2.92790e6 3.26520e6 3.69620e6 3.69620e6 3.85510e6 3.99240e6 4.11360e6</pre>

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```
20100260 1200.0 4.32180e6

        20100261
        1200.0
        4.41300e6

        20100262
        1400.0
        4.49750e6

        20100263
        1500.0
        4.57640e6

20100264 1600.0 4.65030e6
20100265 1700.0 4.72010e6
* ss-304
20100351 300.0 3.61140e6
20100352 400.0 3.96460e6
20100353 500.0 4.14360e6
20100354 600.0 4.23160e6
20100355 700.0 4.28970e6
20100356 800.0 4.35670e6
20100357 900.0
                 4.45010e6
20100358 1000.0 4.56620e6
20100359 1100.0 4.68090e6
20100360 1200.0 4.75070e6
20100361 1300.0 4.71290e6
20100362 1400.0 4.48710e6
20100363 1500.0 3.97600e6
20100364 1600.0 3.06650e6
20100365 1700.0 1.63060e6
* ss-347
20100451 300.0
                 3.8513e6
20100452400.020100453500.0
                 3.9914e6
                 4.1316e6
20100454600.020100455700.020100456800.0
                 4.2718e6
                 4.4119e6
                 4.5521e6
20100457 900.0 4.6922e6
20100458 1000.0 4.8324e6
20100459 1100.0 4.9725e6
20100460 1200.0 5.1127e6
20100461 1300.0 5.2528e6
20100462 1400.0 5.3930e6
20100463 1500.0 5.5332e6
20100464 1600.0 5.6733e6
20100465 1700.0 5.8135e6
*
*
   power table
* test specific power and decay curve
20220000 power 500 1.0 8.04952e5
20220001 0.0 1.00000
20220002 3.6 0.98320
20220003 8.1 0.95981
20220004 15.8 0.92975
20220005 23.6 0.90135
20220006 41.4 0.85792
2022000755.80.827862022000876.90.791112022000998.10.76272
20220010 125.8 0.72764
20220011 162.5 0.69423
20220012 202.5 0.66417
```

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20220013 244.7 0.63911
20220014 303.6 0.61072
20220015 339.2 0.59735
20220016 391.4
             0.57898
20220017 451.4 0.56061
20220018 503.6
             0.54557
20220019 546.9
              0.53722
20220020 592.5
             0.52553
20220021 636.9
             0.51885
20220022 679.2 0.50883
20220023 729.2 0.50214
20220024 771.4 0.49379
20220025 798.1 0.49045
20220026 818.1 0.48711
20220027 819.2 0.35683
20220028 820.3 0.23823
20220029 821.4 0.12423
20220030 829.9 0.00000
*
   control variables
* cntrlvar-005 : core collapsed water level
20500100 wt-lvl1 sum 1.0 0.0 1
20500101 0.0 0.0381 voidf 200010000
          0.0762 voidf 200020000
0.0762 voidf 200030000
0.0762 voidf 200040000
0.0762 voidf 200050000
0.0762 voidf 200050000
20500102
20500103
20500104
20500105
           0.0762 voidf 200060000
20500106
          0.0762 voidf 200070000
20500107
20500108
          0.0762 voidf 200080000
20500109
          0.0762 voidf 200090000
          0.0762 voidf 200100000
20500110
20500111
          0.0762 voidf 200110000
          0.0762 voidf 200120000
20500112
          0.0762 voidf 200130000
20500113
20500114
          0.0762 voidf 200140000
20500115
          0.0762 voidf 200150000
20500116
          0.0762 voidf 200160000
           0.0762 voidf 200170000
20500117
           0.0762 voidf 200180000
20500118
           0.0762 voidf 200190000
20500119
           0.0762 voidf 200200000
20500120
20500200 wt-lv12 sum 1.0 0.0 1
20500201 0.0 0.0762 voidf 200210000
           0.0762 voidf 200220000
20500202
           0.0762 voidf 200230000
0.0762 voidf 200240000
20500203
20500204
20500205
           0.0762 voidf 200250000
          0.0762 voidf 200260000
20500206
20500207
          0.0762 voidf 200270000
20500208
          0.0762 voidf 200280000
```

```
20500209
           0.0762 voidf 200290000
20500210
           0.0762 voidf 200300000
20500211
           0.0762 voidf 200310000
           0.0762 voidf 200320000
20500212
           0.0762 voidf 200330000
20500213
           0.0762 voidf 200340000
20500214
           0.0762 voidf 200350000
20500215
           0.0762 voidf 200360000
20500216
           0.0762 voidf 200370000
0.0762 voidf 200380000
0.0762 voidf 200380000
0.0762 voidf 200390000
0.0762 voidf 200400000
20500217
20500218
20500219
20500220
20500300 wt-lvl3 sum 1.0 0.0 1
20500301 0.0 0.0762 voidf 200410000
            0.0762 voidf 200420000
20500302
           0.0762 voidf 200430000
20500303
20500304
           0.0762 voidf 200440000
20500305
           0.0762 voidf 200450000
           0.0762 voidf 200460000
20500306
           0.0762 voidf 200470000
20500307
20500308
           0.0762 voidf 200480000
20500309
           0.0381 voidf 200490000
20500500 wt-level sum 1.0 0.0 1
20500501 0.0 1.0000 cntrlvar 001
20500502 1.0000 cntrlvar 002
           1.0000 cntrlvar 003
20500503
* cntrlvar-011~022 : void fraction at every 12 inches
*------
* 0 ~ 1 ft
20501100 voidg1 sum 0.25 0.0 1
20501101 0.0 0.5 voidg 200010000
         1.0 voidg 200020000
20501102
           1.0 voidg 200030000
20501103
           1.0 voidg 200040000
20501104
20501105
            0.5 voidg 200050000
* 1 ~ 2 ft
20501200 voidg2 sum 0.25 0.0 1
20501201 0.0 0.5 voidg 200050000
20501202 1.0 voidg 200060000
20501203
           1.0 voidg 200070000
20501204
           1.0 voidg 200080000
20501205
            0.5 voidg 200090000
* 2 ~ 3 ft
20501300 voidg3 sum 0.25 0.0 1
20501301 0.0 0.5 voidg 200090000
20501302
           1.0 voidg 200100000
20501303
           1.0 voidg 200110000
20501304
           1.0 voidg 200120000
            0.5 voidg 200130000
20501305
* 3 ~ 4 ft
20501400 voidg4 sum 0.25 0.0 1
20501401 0.0 0.5 voidg 200130000
20501402 1.0 voidg 200140000
20501403 1.0 voidg 200150000
```

```
1.0 voidg 200160000
20501404
20501405
             0.5 voidg 200170000
* 4 ~ 5 ft
20501500 voidg5 sum 0.25 0.0 1
20501501 0.0 0.5 voidg 200170000
             1.0 voidg 200180000
20501502
             1.0 voidg 200190000
20501503
20501504
             1.0 voidg 200200000
             0.5 voidg 200210000
20501505
* 5 ~ 6 ft
20501600 voidg6 sum 0.25 0.0 1
20501601 0.0 0.5 voidg 200210000
             1.0 voidg 200220000
20501602
             1.0 voidg 200230000
20501603
             1.0 voidg 200240000
20501604
             0.5 voidg 200250000
20501605
* 6 ~ 7 ft
20501700 voidg7 sum 0.25 0.0 1
20501701 0.0 0.5 voidg 200250000
20501702 1.0 voidg 200260000
             1.0 voidg 200270000
20501703
             1.0 voidg 200280000
20501704
             0.5 voidg 200290000
20501705
* 7 ~ 8 ft
20501800 voidg8 sum 0.25 0.0 1
20501801 0.0 0.5 voidg 200290000
20501802
             1.0 voidg 200300000
             1.0 voidg 200310000
20501803
             1.0 voidg 200320000
20501804
20501805
             0.5 voidg 200330000
* 8 ~ 9 ft
20501900 voidg9 sum 0.25 0.0 1
20501901 0.0 0.5 voidg 200330000
            1.0 voidg 200340000
20501902
20501903
             1.0 voidg 200350000
             1.0 voidg 200360000
20501904
20501905
             0.5 voidg 200370000
* 9 ~ 10 ft
20502000 voidg10 sum 0.25 0.0 1
20502001 0.0 0.5 voidg 200370000
20502002
            1.0 voidg 200380000
20502003
             1.0 voidg 200390000
             1.0 voidg 200400000
20502004
             0.5 voidg 200410000
20502005
* 10 ~ 11 ft
20502100 voidg11 sum 0.25 0.0 1
20502101 0.0 0.5 voidg 200410000
             1.0 voidg 200420000
20502102
             1.0 voidg 200430000
20502103
20502104
             1.0 voidg 200440000
20502105
             0.5 voidg 200450000
* 11 ~ 12 ft
20502200 voidg12 sum 0.25 0.0 1
20502201 0.0 0.5 voidg 200450000
             1.0 voidg 200460000
20502202
20502203
             1.0 voidg 200470000
20502204
             1.0 voidg 200480000
```

20502205 0.5 voidg 200490000 \* cntrlvar-031~43 : wall temp - sat. temp at every measurement location \*-----\* 1 ft 20503100 dtemp1 sum 1.0 0.0 1 20503101 0.0 1.0 httemp 200100508 20503102 -1.0 sattemp 200050000 \* 2 ft 20503200 dtemp2 sum 1.0 0.0 1 20503201 0.0 1.0 httemp 200100908 20503202 -1.0 sattemp 200090000 \* 3 ft 20503300 dtemp3 sum 1.0 0.0 1 20503301 0.0 1.0 httemp 200101308 20503302 -1.0 sattemp 200130000 \* 4 ft 20503400 dtemp4 sum 1.0 0.0 1 20503401 0.0 1.0 httemp 200101708 20503402 -1.0 sattemp 200170000 \* 5 ft 20503500 dtemp5 sum 1.0 0.0 1 20503501 0.0 1.0 httemp 200102108 20503502 -1.0 sattemp 200210000 \* 6 ft sum 1.0 0.0 1 20503600 dtemp6 20503601 0.0 1.0 httemp 200102508 -1.0 sattemp 200250000 20503602 \* 7 ft 20503700 dtemp7 sum 1.0 0.0 1 20503701 0.0 1.0 httemp 200102908 20503702 -1.0 sattemp 200290000 \* 8 ft 20503800 dtemp8 sum 1.0 0.0 1 20503801 0.0 1.0 httemp 200103308 20503802 -1.0 sattemp 200330000 \* 9 ft 20503900 dtemp9 sum 1.0 0.0 1 20503901 0.0 1.0 httemp 200103708 20503902 -1.0 sattemp 200370000 \* 10 ft 20504000 dtemp10 sum 1.0 0.0 1 20504001 0.0 1.0 httemp 200104108 -1.0 sattemp 200410000 20504002 \* 11 ft 20504100 dtemp11 sum 1.0 0.0 1 20504101 0.0 1.0 httemp 200104508 20504102 -1.0 sattemp 200450000 \* 12 ft 20504200 dtemp12 sum 1.0 0.0 1 20504201 0.0 1.0 httemp 200104908 20504202 -1.0 sattemp 200490000 \* 6.5 ft 20504300 dtemp65 sum 1.0 0.0 1 20504301 0.0 1.0 httemp 200102708 20504302 -1.0 sattemp 200270000

cntrlvar-051~62 : heat transfer coefficient at every measurement location \*\_\_\_\_\_ \* 1 ft 20505100 htcl div 1.0 1.0 1 20505101 cntrlvar 031 htrnr 200100501 \* 2 ft 20505200 htc2 div 1.0 1.0 1 htrnr 200100901 20505201 cntrlvar 032 \* 3 ft 20505300 htc3 div 1.0 1.0 1 20505301 cntrlvar 033 htrnr 200101301 \* 4 ft 20505400 htc4 div 1.0 1.0 1 20505401 cntrlvar 034 htrnr 200101701 \* 5 ft 20505500 htc5 div 1.0 1.0 1 20505501 cntrlvar 035 htrnr 200102101 \* 6 ft 20505600 htc6 div 1.0 1.0 1 20505601 cntrlvar 036 htrnr 200102501 \* 7 ft 20505700 htc7 div 1.0 1.0 1 20505701 cntrlvar 037 htrnr 200102901 \* 8 ft 20505800 htc8 div 1.0 1.0 1 20505801 cntrlvar 038 htrnr 200103301 \* 9 ft 20505900 htc9 div 1.0 1.0 1 20505901 cntrlvar 039 htrnr 200103701 \* 10 ft 20506000 htc10 div 1.0 1.0 1 htrnr 200104101 20506001 cntrlvar 040 \* 11 ft 20506100 htc11 div 1.0 1.0 1 20506101 cntrlvar 041 htrnr 200104501 \* 12 ft 20506200 htc12 div 1.0 1.0 1 20506201 cntrlvar 042 htrnr 200104901 \* 6.5 ft 20506300 htc65 div 1.0 1.0 1 20506301 cntrlvar 043 htrnr 200102701 \* cntrlvar-071~82 : differential pressure between every measurement location \* 0 ~ 1 ft 20507100 dp01 sum 1.0 0.0 1 20507101 0.0 1.0 p 200010000 20507102 -1.0 p 200050000 \* 1 ~ 2 ft 20507200 dp12 sum 1.0 0.0 1 20507201 0.0 1.0 p 200050000 20507202 -1.0 p 200090000 \* 2 ~ 3 ft 20507300 dp23 sum 1.0 0.0 1

```
20507301 0.0 1.0 p 200090000
20507302
          -1.0 p 200130000
* 3 ~ 4 ft
20507400 dp34 sum 1.0 0.0 1
20507401 0.0 1.0 p 200130000
          -1.0 p 200170000
20507402
* 4 ~ 5 ft
20507500 dp45 sum 1.0 0.0 1
20507501 0.0 1.0 p 200170000
          -1.0 p 200210000
20507502
* 5 ~ 6 ft
20507600 dp56 sum 1.0 0.0 1
20507601 0.0 1.0 p 200210000
20507602
          -1.0 p 200250000
* 6 ~ 7 ft
20507700 dp67 sum 1.0 0.0 1
20507701 0.0 1.0 p 200250000
20507702
          -1.0 p 200290000
* 7 ~ 8 ft
20507800 dp78 sum 1.0 0.0 1
20507801 0.0 1.0 p 200290000
          -1.0 p 200330000
20507802
* 8 ~ 9 ft
20507900 dp89 sum 1.0 0.0 1
20507901 0.0 1.0 p 200330000
        -1.0 p 200370000
20507902
* 9 ~ 10 ft
20508000 dp910 sum 1.0 0.0 1
20508001 0.0 1.0 p 200370000
20508002
        -1.0 p 200410000
* 10 ~ 11 ft
20508100 dp1011 sum 1.0 0.0 1
20508101 0.0 1.0 p 200410000
20508102 -1.0 p 200450000
* 11 ~ 12 ft
20508200 dp1112 sum 1.0 0.0 1
20508201 0.0 1.0 p 200450000
        -1.0 p 200490000
20508202
* 0 ~ 12 ft
20508300 dp012 sum 1.0 0.0 1
20508301 0.0 1.0 p 200010000
20508302 -1.0 p 200490000
*_____
* cntrlvar-110 : liquid carry-over fraction
*_____
20510100 flowin mult 0.0155565 0.0 1
20510101 voidfj 165000000
20510102 rhofj 165000000
20510103 velfj 16500000
20510200 floin integral 1.0 0.0 1
20510201 cntrlvar 101
* add small no. to avoid dividing by zero
20510300 floina sum 1.0 0.0 1
20510301 1.e-20 1.0 cntrlvar 102
20510400 flowot mult 0.0155565 0.0 1
```

20510401 voidfj 25000000 20510402 rhofj 25000000 20510403 velfj 250000000 \* 20510500 floot integral 1.0 0.0 1 20510501 cntrlvar 104 \* 20511000 carryf div 1.0 1.0 1 20511001 cntrlvar 103 cntrlvar 105 \*\_\_\_\_\_\_\_ \* cntrlvar-200 : total power \*\_\_\_\_\_\_ 20520000 power function 1.0 0.0 1 20520001 time 0 200 \*

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