



In-Package Chemistry Abstraction

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1) ACN-001, DATED 12/01/2005

THIS DOCUMENT CONTAINS THE FOLLOWING, LOCATED AT THE BACK OF THE DOCUMENT:

1) ADDENDUM 001, DATED 08/10/2007

Prepared for:
U.S. Department of Energy
Office of Civilian Radioactive Waste Management
Office of Repository Development
1551 Hillshire Drive
Las Vegas, Nevada 89134-6321

Prepared by:
Bechtel SAIC Company, LLC
1180 Town Center Drive
Las Vegas, Nevada 89144

Under Contract Number
DE-AC28-01RW12101

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QA: QA

In-Package Chemistry Abstraction

ANL-EBS-MD-000037 REV 04

July 2005

Complete only applicable items.

2. Type of Mathematical Model

Process Model Abstraction Model System Model

Describe Intended Use of Model

3. Title

In-Package Chemistry Abstraction

4. DI (including Rev. No.):

ANL-EBS-MD-000037 REV 04

| | Printed Name | Signature | Date |
|-----------------------------------|-------------------------------|---------------|---------|
| 5. Originator | Emma Thomas | | 7/14/05 |
| 6. Independent Technical Reviewer | Laura Price | SEE NEXT PAGE | |
| 7. Checker | Patricia Bernot | | 7/14/05 |
| 8. QER | ^{for} Judith Gebhart | | 7/14/05 |
| 9. Responsible Manager/Lead | James Cunnane | | 7/14/05 |
| 10. Responsible Manager | Paul Dixon | | 7-14-05 |

11. Remarks

Appendix F – One (1) CD-ROM

Change History

| 12. Revision No. | 13. Description of Change |
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| 00 | Initial Issue. |
| 01 | The entire abstraction model was revised to reflect updated inputs/outputs from the feed process model AMR (ANL-EBS-MD-000050 REV 01). |
| 02 | Complete revision to incorporate new conceptual model. |
| 03 | Complete revision to support regulatory integration. Change bars were not used because the changes were too significant. This revision incorporates the suggestions identified in CR 2065. |
| 04 | Complete revision including updating from Cr(VI) to Cr(III), inclusion of LaBS glass analysis, and enhanced model validation. Change bars were not used because the changes were too significant. This revision incorporates the corrections identified in CRs 4961, 5001, and 5002. |

5
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ACKNOWLEDGEMENTS

This model report was developed through the contributions of the following individuals, listed in alphabetical order:

- Sara Arthur, AREVA. Contributor to all prior revisions of this report (REV00 through REV03). Responsible for various sensitivity analyses.
- Patrick Brady, Sandia National Laboratories. Co-author/major contributor to prior revisions of this report (REV00 through REV03). Developed the surface complexation model, and co-developed the batch reactor conceptual model (with Paul Domski).
- Paul Domski, AREVA. Originator, lead author, and major contributor to all prior revisions of this report (REV00 through REV03). Codeveloped the batch reactor conceptual model (with Pat Brady) and developed the model abstraction.
- Susan LeStrange, BSC. Contributor to REV03. Responsible for various sensitivity analyses.
- Clinton Lum, Sandia National Laboratories. Contributor to the REV04. Responsible for the vapor diffusion calculation.
- Paul Mariner, AREVA. Coauthored, and major contributor to previous and current revision of this report (REV03 through REV 04). Responsible for the development of the surface complexation model demonstration.
- Emma Thomas, BSC. Originator and lead author for REV03 and REV04 of this report. Responsible for incorporation of Regulatory Integration Team comments and items in the RIT Action Item Database.

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ACRONYMS

| | |
|---------|--|
| atm | atmospheres |
| B&W | Babcock and Wilcox |
| CDSP | codisposed |
| CPU | central processing unit |
| CSNF | commercial spent nuclear fuel |
| DOE | U.S. Department of Energy |
| DTN | data tracking number |
| EBS | engineered barrier system |
| FEPs | features, events, and processes |
| GPC | glass pour canisters |
| HFO | hydrous ferric oxide |
| HLW | high-level waste |
| HLWG | high-level waste glass |
| IED | information exchange drawing |
| IEP | isoelectric point |
| MCO | multi-canister overpack |
| PWR | pressurized water reactor |
| Q | water flux through the waste package |
| SNF | spent nuclear fuel |
| SRL | Savannah River Laboratory |
| TMI | Three Mile Island |
| TSPA-LA | Total Systems Performance Assessment–License Application |
| XRD | x-ray diffraction |
| ZPC | zero point of charge |

ELEMENTAL SYMBOLS

| | | | |
|----|------------|----|--------------|
| Ag | Silver | Na | Sodium |
| Al | Aluminium | Nb | Niobium |
| Am | Americium | Nd | Neodymium |
| Ba | Barium | Ni | Nickel |
| Be | Beryllium | Np | Neptunium |
| C | Carbon | P | Phosphorus |
| Ca | Calcium | Pa | Protactinium |
| Cl | Chlorine | Pb | Lead |
| Cm | Curium | Pd | Palladium |
| Co | Cobalt | Pm | Promethium |
| Cr | Chromium | Pr | Praseodymium |
| Cs | Cesium | Rh | Rhodium |
| Cu | Copper | Rn | Radon |
| Eu | Europium | Ru | Ruthenium |
| F | Fluorine | S | Sulfur |
| Fe | Iron | Si | Silicon |
| Gd | Gadolinium | Sm | Samarium |
| H | Hydrogen | Tc | Technetium |
| Ho | Holmium | Th | Thorium |
| In | Indium | Ti | Titanium |
| K | Potassium | U | Uranium |
| Kr | Krypton | V | Vanadium |
| La | Lanthanum | W | Tungsten |
| Li | Lithium | Xe | Xenon |
| Mg | Magnesium | Y | Yttrium |
| Mn | Manganese | Zn | Zinc |
| Mo | Molybdenum | Zr | Zirconium |
| N | Nitrogen | | |

1. PURPOSE

This report was developed in accordance with the requirements in *Technical Work Plan for Postclosure Waste Form Modeling* (BSC 2005 [DIRS 173246]). The purpose of the in-package chemistry model is to predict the bulk chemistry inside of a breached waste package and to provide simplified expressions of that chemistry as a function of time after breach to Total Systems Performance Assessment for the License Application (TSPA-LA).

The scope of this report is to describe the development and validation of the in-package chemistry model. The in-package model is a combination of two models, a batch reactor model, which uses the EQ3/6 geochemistry-modeling tool, and a surface complexation model, which is applied to the results of the batch reactor model. The batch reactor model considers chemical interactions of water with the waste package materials, and the waste form for commercial spent nuclear fuel (CSNF) waste packages and codisposed (CDSP) waste packages containing high-level waste glass (HLWG) and DOE spent fuel. The surface complexation model includes the impact of fluid-surface interactions (i.e., surface complexation) on the resulting fluid composition.

The model examines two types of water influx: (1) the condensation of water vapor diffusing into the waste package, and (2) seepage water entering the waste package as a liquid from the drift.

1. **Vapor-Influx Case:** The condensation of vapor onto the waste package internals is simulated as pure H₂O and enters at a rate determined by the water vapor pressure for representative temperature and relative humidity conditions.
2. **Liquid-Influx Case:** The water entering a waste package from the drift is simulated as typical groundwater and enters at a rate determined by the amount of seepage available to flow through openings in a breached waste package.

TSPA-LA uses the vapor-influx case for the nominal scenario for simulations where the waste package has been breached but the drip shield remains intact, so the seepage flow is diverted from the waste package. The chemistry from the vapor-influx case is used to determine the stability of colloids and the solubility of radionuclides available for transport by diffusion, and to determine the degradation rates for the waste forms.

TSPA-LA uses the liquid-influx case for the seismic scenario, where the waste package has been breached and the drip shield has been damaged such that seepage flow is actually directed into the waste package. The chemistry from the liquid-influx case, which is a function of the flow rate, is used to determine the stability of colloids and the solubility of radionuclides available for transport by diffusion and advection, and to determine the degradation rates for the CSNF and HLW glass. TSPA-LA does not use this model for the igneous scenario.

Outputs from the in-package chemistry model implemented inside TSPA-LA include pH, ionic strength, and total carbonate concentration. These inputs to TSPA-LA will be linked to the following processes: dissolution rates of the CSNF and HLWG, dissolved concentrations of radionuclides, and colloid generation.

Other outputs from the in-package chemistry model used in other models supporting TSPA-LA include Eh (redox potential), chloride, and fluoride concentration. Specific information available in the output files from the batch reactor model includes time series concentrations of the elements and their aqueous complexes constituting the waste package, waste form, seepage, gas composition, and mineralogical phase abundance.

The in-package chemistry model is activated within TSPA-LA when the waste package has been breached and the interior temperature of the waste package is below the boiling temperature of water. References to time within this report refer to the time after waste package breach and after the waste package cools to 100°C. Liquid water only persists in appreciable quantities when the drift environment has cooled to below approximately 100°C. While the boiling point of pure water at the repository elevation is approximately 96°C, the value of 100°C is used to reflect possible higher boiling points of liquids containing dissolved solids. The in-package chemistry model is applicable over the water volumetric flux (hereafter referred to as “flux”) range from 0.15 L/yr to 300 L/yr per waste package, a temperature range from 25°C to 100°C, a carbon dioxide partial pressure range of 10^{-5} to $10^{-1.5}$ atmospheres (atm), and an oxygen partial pressure up to 0.2 atm. Spatially, the applicability of the in-package chemistry model is limited to the waste package interior.

Limitations to this model include:

- Section 5, Assumption 5.1 discusses the limited availability of data on thin film chemistry.
- Section 5, Assumption 5.2 discusses the limited availability of long-term metal alloy corrosion information such as the evolution of the surface area of the metal waste package components as a function of time.
- Section 6.3.1.1 discusses the limited availability of thermodynamic data for certain elements that constitute the waste package and waste forms (transition metals and radionuclides) that prohibited the use of high-ionic strength solutions as initial starting compositions for the in-package chemistry model (concentrated brines). This data limitation requires the use of the B-dot activity coefficient equation in the EQ3/6 modeling, which is only valid up to ionic strengths of 4 molal (Section 6.3.1.1). This data limitation also influenced the boundary conditions used in the model where the ratio of water to reactants was selected for the base case.
- Section 6.9 addresses not accounting for feedback between the batch reactor and the surface complexation models during modeling since they are implemented in series.

Table 1-1 provides a list of upstream and downstream documents associated with this model.

Table 1-1. Upstream and Downstream Documents

| | |
|---|--|
| <p>Upstream Documents (Providing Directly Used Inputs to Develop this Model)</p> | <ul style="list-style-type: none"> • <i>CSNF Waste Form Degradation: Summary Abstraction</i> (BSC 2004 [DIRS 169987]) • <i>Defense HLW Glass Degradation Model</i> (BSC 2004 [DIRS 169988]) • <i>EBS Radionuclide Transport Abstraction</i> (BSC 2004 [DIRS 169868]) • <i>Abstraction of Drift Seepage</i> (BSC 2004 [DIRS 169131]) • <i>Qualification of Thermodynamic Data for Geochemical Modeling of Mineral-Water Interactions in Dilute Systems</i> (BSC 2004 [DIRS 171916]). |
| <p>Downstream Documents (Using Output of this Model for Development)</p> | <ul style="list-style-type: none"> • <i>Dissolved Concentration Limits of Radioactive Elements</i> (BSC 2005 [DIRS 173873]) • <i>Pitting Model for Zirconium-Alloyed Cladding</i> (BSC 2004 [DIRS 170043]) • <i>Waste-Form Features, Events, and Processes</i> (BSC 2004 [DIRS 170020]) • <i>Engineered Barrier System Features, Events, and Processes</i> (BSC 2004 [DIRS 169860]) • <i>Clad Degradation – FEPs Screening Arguments</i> (BSC 2004 [DIRS 170019]). |

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2. QUALITY ASSURANCE

2.1 QA PROGRAM APPLICABILITY

Development of this report has been determined to be subject to the Yucca Mountain Project's quality assurance (QA) program (BSC 2005 [DIRS 173246], Section 8) because it will be used to support performance assessments. Approved QA procedures identified in the technical work plan (BSC 2005 [DIRS 173246], Section 4) have been used to conduct and document the activities described. This report describes bulk chemistry inside of a breached waste package that affects the following items in *Q-List* (BSC 2005 [DIRS 171190]): the waste package (including internals), cladding, and waste.

2.2 ELECTRONIC MANAGEMENT OF INFORMATION

The technical work plan contains the process control evaluation used to evaluate the control of electronic management of information (BSC 2005 [DIRS 173246], Appendix A) during the modeling and documentation activities, and this evaluation determined that the methods in the implementing procedures are adequate; therefore, no deviations from these methods were necessary.

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3. USE OF SOFTWARE

A list of controlled and baselined software used in this report is provided in Table 3-1, and the operating environments are provided in Table 3-2. The range of use for each piece of software is within that for which it was qualified. Each of the software codes was selected because it is appropriate for use in geochemical modeling, uses the YMP-qualified thermodynamic database, and is subject to no limitations on the outputs due to the selected software (though there are limitations on the thermodynamic database used, which is discussed in more detail in Section 6.3.1.1). The use of the software was consistent with its intended use and within documented validation ranges. No software was used prior to qualification to develop any preliminary output. Microsoft Excel, a commercial off-the-shelf software program, is used in this report; however, the results are not dependent on the software program used, so this software is exempt from requirements in LP-SI.11Q-BSC, *Software Management*. Section 4 discusses formulas and inputs used in this model for all software. The outputs are discussed in Section 6. No other information is required for independent reproduction of the work.

Table 3-1. Computer Software Used

| Software Name | Version | Software Tracking Number (Qualification Status) | Description and Components Used | Input and Output Files ^a (Included in Appendix F) |
|-----------------|------------|--|---|--|
| EQ3/6 | 7.2b | UCRL-MA-110662 (LSCR198) [DIRS 153964] (Qualified on Windows 95 and HP-UX 10.20 B) | EQ3NR: a FORTRAN speciation-solubility code | input: *.3i output: *.3p, *.3o |
| | | | EQPT: a data file preprocessor in FORTRAN | input: data0.* output: data1.* |
| EQ6 | 7.2bLV | 10075-7.2bLV-02 [DIRS 159731] (Qualified on Windows 2000 and NT 4.0) | EQ6: a reaction path code that models water-rock interaction or fluid mixing in either a pure reaction progress mode or a time mode | input: *.6i pickup: *.6p output: *.6o *.elem_aqu.txt *.elem_min.txt *.elem_tot.txt *.min_info.txt *.bin |
| GetEQData | 1.0.1 | 10809-1.0.1-00 [DIRS 173680] (Qualified on Windows 2000, NT 4.0, 98, and 95) | A Microsoft Excel macro used to postprocess EQ3/6 output information | input: *.3o, *.6o output: *.xls |
| PHREEQC | 2.3 | 10068-2.3-01 [DIRS 157837] (Qualified on Windows 2000) | A code for geochemical speciation, reaction path modeling, reactive transport, and surface-complexation modeling | input: *. (no extension) output: *.out |
| transl | 2.0 | 10251-2.0-00 [DIRS 155029] (Qualified on Windows 98) | A code for translating a nonPitzer EQ3/6 database into PHREEQC format | input: data0.* output: *.dat |
| Mathcad | Version 11 | Commercial off-the-shelf software: Exempt | Used in this document for calculations | input: *.mcd output: *.mcd, *.xls |
| Microsoft Excel | 97 SR-2 | Commercial off-the-shelf software: Exempt | Used in this document for graphical representation and arithmetical manipulations | input: *.3o, *.6o output: *.xls |

NOTE: ^a Files are explained in more detail in Appendix G.

Table 3-2. Computers and Operating Systems Used

| Computer Make | CPU # | Operating System | Software Used |
|----------------------|--------------|-------------------------|---|
| Dell Optiplex GX300 | 117728 | Windows 95 | EQ3/6 V7.2b |
| Dell Latitude C610 | 632MT11 | Windows 2000 | EQ6 V7.2bLV, GetEQData, Microsoft Excel |
| Dell Optiplex GX260 | 4BPQW11 | Windows 2000 | PHREEQC, Microsoft Excel |
| Dell Optiplex GX110 | 3YS420B | Windows 98 | transl |
| Dell Optiplex GX260 | 152381 | Windows 2000 | EQ6 V7.2bLV, Microsoft Excel |
| Dell Optiplex GX260 | 501077 | Windows 2000 | Mathcad V 11 |
| Dell Latitude C640 | YMP000106 | Windows 2000 | EQ6 V7.2bLV, GetEQData, Microsoft Excel |
| Dell Optiplex GX400 | 151295 | Windows NT 4.0 | EQ6 V7.2bLV, GetEQData, Microsoft Excel |

NOTE: CPU = central processing unit.

4. INPUTS

This section lists the inputs used to develop this report. The information used to develop the model was not used to validate the model.

4.1 DIRECT INPUT

The information in the following sections is used as direct input to the in-package chemistry model. Table 4-1 lists the data used in the in-package chemistry model. All the inputs in Table 4-1 are qualified data.

Table 4-1. Summary of Direct Inputs

| Identifier | Input | Reference | Used in: |
|-------------------|-----------------------|---|--|
| data0.ymp.R4 | Thermodynamic Data | DTN: SN0410T0510404.002 [DIRS 172712] BSC 2004 [DIRS 171916] | For EQ6 runs and Sections 6.3.1, 6.10.5, 6.10.6 |
| J-13 Well Water | Water Composition | DTN: MO0006J13WTRCM.000 [DIRS 151029] | Section 6.3.1, Appendix F <i>CDSP.xls</i> , <i>CSNF.xls</i> , and in all EQ6 input files |
| Ca-porewater | Water Composition | DTN: GS020408312272.003 [DIRS 160899] | |
| Na-porewater | Water Composition | DTN: GS020408312272.003 [DIRS 160899] | |
| 51.3i | Water Composition | DTN: MO0303MWDSCMAB.000 [DIRS 162551], file: W6_THCabstraction3i3o3pfiles, path: ESP\Jolley\EQ36 for P&CE\THC Abstraction\W6\tf4 | |
| 10006.3i | Water Composition | DTN: MO0303MWDSCMAB.000 [DIRS 162551], file: W4_THCabstraction3i3o3pfiles, path: ESP\Jolley\EQ36 for PCE\THC Abstraction\W4\tf4 | |
| 20013.3i | Water Composition | DTN: MO0303MWDSCMAB.000 [DIRS 162551] file: W5_THCabstraction3i3o3pfiles, path: ESP\Jolley\EQ36 for P&CE\THC Abstraction\W5\tf4 | |
| Liquid Entry Rate | Flow Rates | BSC 2004 [DIRS 169131], Tables 6.8-1 and 6.8-2 | |
| Vapor Entry Rate | Flow Rates | BSC 2004 [DIRS 169868], Table 6.6-1 | |
| HLWG | Reaction Rate | BSC 2004 [DIRS 169988], Section 8.1, Equations 50 and 51, Table 8-1 | |
| | Density | Baxter 1988 [DIRS 106164], Table 10 | |
| | Composition | CRWMS M&O 2001 [DIRS 153263], Table 3 (for LaBS glass) Marra, J. and Ebert, W. 2003 [DIRS 172949] Table 2, column: SRTC Tests, Pu only | |
| | f_{exposure} | BSC 2004 [DIRS 169988], Table 8-1 | |
| N-Reactor Fuel | Reaction Rate | DOE 2000 [DIRS 152658], Equation 2-39 | |
| | Composition | DOE 2000 [DIRS 150095], Table 3-1 | |
| CSNF | Isotopic Inventory | BSC 2003 [DIRS 169110], Attachment III Disk 1 of 9, ATT III/ LPM1/ uniform_profile/ 3.5/ ft71-case10.N04 | |
| | Reaction Rate | BSC 2004 [DIRS 169987], Section 8.1, Tables 8-2 and 8-3 | |

Table 4-1. Summary of Direct Inputs (Continued)

| Identifier | Input | Reference | Used in: |
|---|--|--|---|
| 21-PWR CSNF Waste Package | Waste Package Component Dimensions | Punatar 2001 [DIRS 155635], p. 2-5 BSC 2005 [DIRS 173501], Tables 1 and 3 BSC 2004 [DIRS 172214], p. II-2 and II-3 BSC 2004 [DIRS 168199] BSC 2004 [DIRS 168253] | |
| 2-MCO / 2-DHLW Waste Package | Waste Package Component Dimensions | DOE 2000 [DIRS 150095], Section 4 DOE 1992 [DIRS 102812], Figure 3.4.1 and 3.4.2 Taylor 1997 [DIRS 126175] BSC 2005 [DIRS 174225], Table 3-1 BSC 2005 [DIRS 173501], Table 1 and 9 BSC 2004 [DIRS 172214], p. X-2 and X-3 | Section 6.3.1, Appendix F CDSP.xls, CSNF.xls, and in all EQ6 input files |
| Periodic Table | Atomic Weights of Elements and Isotopes | Audi and Wapstra 1995 [DIRS 149625] Parrington et al. 1996 [DIRS 103896] | |
| Stainless Steel Type 316 | Corrosion rate | DTN: MO0409SPAACRWP.000 [DIRS 172059] | |
| | Density | ASTM G 1-90 1999 [DIRS 103515], Table XI, p. 7 | |
| | Composition | ASTM A 240/A 240M-03b 2003 [DIRS 165003], Table 1, p. 4 | |
| Stainless Steel Type 304L | Corrosion rate | DTN: MO0409SPAACRWP.000 [DIRS 172059] | |
| | Density | ASTM G 1-90 1999 [DIRS 103515], Table XI, p. 7 | |
| | Composition | ASTM A 240/A 240M-03b 2003 [DIRS 165003], Table 1, p. 3 | |
| Carbon Steel Type A516 | Corrosion rate | DTN: MO0409SPAACRWP.000 [DIRS 172059] | |
| | Density | ASTM A 20/A 20M-95a 1995 [DIRS 104189], p. Section 14, p. 8 | |
| | Composition | ASTM A 516/A 516M-01 2001 [DIRS 162723], Table 1, grade 70, 1/2" to 2" thickness, product analysis | |
| Aluminum Alloy 6061 | Corrosion rate | DTN: MO0409SPAACRWP.000 [DIRS 172059] | |
| | Density | ASTM G 1-90 1999 [DIRS 103515], Table X1.1 | |
| | Composition | ASTM B 209-96 1996 [DIRS 144744], Table 1, p. 7 | |
| Aluminum Alloy 1100 | Corrosion rate | DTN: MO0409SPAACRWP.000 [DIRS 172059] | |
| | Density | ASTM G 1-90 1999 [DIRS 103515], Table XI, p. 7 | |
| | Composition | ASTM B 209-96 1996 [DIRS 144744], Table 1, p. 7 | |
| NiGd Alloy | Corrosion rate | DTN: MO0409SPAACRWP.000 [DIRS 172059] | |
| | Composition and density | ASTM B 932-04. 2004 [DIRS 168403] | |
| Goethite | Surface properties | Langmuir 1997 [DIRS 100051], Table 10.2, p. 345 | |
| Hydrous Ferric Oxide (HFO) Surface Complexation | Surface Complexation Reactions | Dzombak and Morel 1990 [DIRS 105483] | Sections 6.3.2, 6.7, and 6.8 |
| | | Appelo et al. 2002 [DIRS 168168] | |
| Cr(III) solids | LogK for Cr(OH) ₃ and eskolaite | Ball and Nordstrom 1998 [DIRS 163015] | Section 6.6.3 |
| Nitric Acid Production Rate | Reaction Rate | BSC 2004 [DIRS 172017], Table 21, p. 33 | Appendix B |
| G value of Nitric Acid Production | Used to calculate hydrogen peroxide production | BSC 2004 [DIRS 172017], p. 27 | |
| G value of Hydrogen Peroxide Production | | IAEA 1998 [DIRS 150560], Table 8.2, p. 214 | |

4.1.1 Thermodynamic Database

The in-package chemistry model uses the thermodynamic database, *data0.ymp.R4* (DTN: SN0410T0510404.002 [DIRS 172712]), as a necessary component in the execution of the EQ3/6 software. This database used in the in-package chemistry model is qualified for use for temperatures up to 200°C. The database is appropriate for the in-package chemistry model because it includes the elements that constitute the waste package, waste form, seepage, and gas compositions in the temperature range needed for the model.

The *data0.ymp.R4* file was modified to include the following material:

- The CSNF and HLWG (compositions given in Tables 6-6 and 4-5, respectively) were added to the database in order to take advantage of EQ3/6's ability to use a pH- and carbonate-dependent rate law, using the EQ6 transition state theory formalism to describe the degradation. A range of degradation rates based on pH can be specified only for reactants entered as “minerals” (solids contained in the database). “Special reactants” (reactants not contained in the database) must have a constant or fixed degradation rate.
- During the qualification of the *data0* file, several small errors related to log K values (still within the uncertainty range of the data) with their corrections were identified in *Qualification of Thermodynamic Data for Geochemical Modeling of Mineral-Water Interactions in Dilute Systems* (BSC 2004 [DIRS 171916], Table 6-37). These corrections were added to *data0.ymp.R4* in the creation of the *data0.cr3* and *data0.ert* files (Appendix F).
- This report models the oxidation state of chromium in repository conditions as Cr(III), as justified in Section 6.6.3, so the database was modified to allow the modeling of Cr(III) by removing all the Cr(VI) species. Additional information for two Cr(III) solids from a report by Ball and Nordstrom (1998 [DIRS 163015]) was also added to the database, for Cr(OH)₃ and eskolaite.

4.1.2 Input Water Compositions and Rates

Table 4-2 summarizes the input water compositions used in the in-package chemistry model as direct input.

Table 4-2. Input Water Compositions

| Parameter | Units | Ca-Porewater ^a | Na-Porewater ^b | J-13 Well Water ^c |
|-------------------------------|-------|---------------------------|---------------------------|------------------------------|
| Ca ²⁺ | mg/L | 94 | 81 | 13.0 |
| Mg ²⁺ | mg/L | 18.1 | 3.3 | 2.01 |
| Na ⁺ | mg/L | 39 | 120 | 45.8 |
| K ⁺ | mg/L | 7.6 | 6.1 | 5.04 |
| SiO ₂ | mg/L | 42 | 42 | 61.0 |
| NO ₃ ⁻ | mg/L | 2.6 | 0.41 | 8.78 |
| HCO ₃ ⁻ | mg/L | 397 | 362 | See Section 6.3.1.3.1 |
| Cl ⁻ | mg/L | 21 | 24 | 7.14 |

Table 4-2. Input Water Compositions (Continued)

| Parameter | Units | Ca-Porewater ^a | Na-Porewater ^b | J-13 Well Water ^c |
|-------------------------------|-------|---------------------------|---------------------------|------------------------------|
| F ⁻ | mg/L | 3.4 | 6 | 2.18 |
| SO ₄ ²⁻ | mg/L | 36 | 31 | 18.4 |
| pH | pH | 7.6 (8.1) ^d | 7.4 (8.3) ^d | 7.41 (7.8) ^d |

Sources: DTNs: GS020408312272.003 [DIRS 160899]^{a and b}; MO0006J13WTRCM.000 [DIRS 151029]^c.

NOTE: ^aSample: ECRB-SYS-CS1000/7.3-7.7/UC (calculated HCO₃⁻ value).

^bSample: ECRB-SYS-CS2000/16.3-16.5/UC (measured HCO₃⁻ value).

^cThe SiO₂ value is calculated from the 28.5 mg/L of Si given in DTN: MO0006J13WTRCM.000.

^dThe pH values in parenthesis are the pH values once the solution is equilibrated to log *f*CO₂ = -3.0.

For simplicity, and based on their calcium and sodium concentrations, samples ECRB-SYS-CS1000/7.3-7.7/UC and ECRB-SYS-CS2000/16.3-16.5/UC were termed “Ca-porewater” and “Na-porewater,” respectively.

Three water compositions were used as the initial condition(s) in the liquid-influx model of the in-package chemistry model for water entering a breached waste package. The decision to use these water compositions was based on several lines of reasoning. Although it is not expected to enter the repository, the J-13 well water composition was used for comparison purposes (i.e., to maintain continuity between the current work and past in-package chemistry analyses). The Ca- and Na-porewater compositions were used because they were obtained from core samples proximal to the repository. These waters are intended to represent seepage compositions that would enter a breached waste package, especially over the long term. The U.S. Geological Survey identified the location of boreholes from which the cores were removed by noting the following in its data spreadsheet (DTN: GS020408312272.003 [DIRS 160899]):

Chemical composition of pore water extracted from cores of 15 ECRB-SYS-CS Series boreholes, USW SD-9, and USW NRG-7/7a, 04/26/2001 to 02/12/2002.

High trace element values created due to reflux as a result of boiling events were not considered in the incoming water compositions, because these effects would be transitory compared to the regulatory timeframe (BSC 2004 [DIRS 169860], Section 6.5.4.2), and the chemistry would quickly revert to compositions similar to that listed in Table 4-2. The term “trace element” is defined in and information about their contents in potential in-drift seepage waters is reviewed in *Engineered Barrier System: Physical and Chemical Environment Model* (BSC 2004 [DIRS 169860] Section 6.5.4.2). The solute fluoride, often considered a trace element, is included in the input water compositions in Tables 4-2 and 4-3. Several elements generally considered trace elements in natural waters (e.g., Mo, Ni, and B) are found in high concentrations in in-package waters because they are constituents of degrading waste package materials (Figures 6-7, 6-8, 6-14, and 6-15). Other trace elements are considered in the in-package modeling but are present in only low concentrations in solutions because they form sparingly soluble solids under in-package conditions (Figures 6-5, 6-6, and 6-13). These include Fe, Cr, Al, Mn, Gd, and Ru.

Section 6.2.2.1 of *Drift-Scale THC Seepage Model* (BSC 2005 [DIRS 172862]) uses the Ca-porewater (labeled “W5”) and another composition (labeled “W4” [ECRB-SYS-CS2000/16.5-21/UC]) also from the U.S. Geological Survey data

(DTN: GS020408312272.003 [DIRS 160899]) that originates from the next interval in the same core as that from which the Na-porewater was extracted. The composition of the Na-porewater used in the in-package chemistry model differs only slightly from the “W4” composition used in *Drift-Scale THC Seepage Model* (BSC 2005 [DIRS 172862]). Thus, continuity has been established for key inputs for the in-package chemistry model and the drift scale processes models.

Compared to the J-13 well water composition, the porewater compositions are significantly more concentrated (Table 4-2). Two anions important with regard to steel corrosion (chloride and fluoride) are concentrated in the porewaters by factors of approximately 2 to 3 compared to J-13 well water. *Drift-Scale THC Seepage Model* (BSC 2005 [DIRS 172862]) states “W5” (Ca-porewater) has the highest $(Ca + Mg)/(Na + K)$ ratio of the ECRB samples, and “W4” (Na-porewater) has the lowest $(Ca + Mg)/(Na + K)$ ratio of the ECRB samples and higher fluoride than other samples. Thus, the two porewaters represent compositional end members and capture the spread of the porewater compositions. A Piper diagram (BSC 2005 [DIRS 172862], Figure 6.2-4) shows these two samples represent compositional end members.

As these samples cover the spectrum of observed porewater compositions sampled, and there is a possibility that waters of similar composition may enter a drift and contact a breached waste package, the use of these compositions is well justified. Furthermore, use of these data in the in-package chemistry model ensures feeds to the TSPA-LA model will reflect the compositional variation of the initial water composition.

Additional water compositions were also used as inputs for sensitivity analyses purposes. The waters from Section 6.6 of *Engineered Barrier System: Physical and Chemical Environment Model* (BSC 2004 [DIRS 169860]) represent seepage compositions modeled to enter the drift during the postclosure period and those that are thermally perturbed and in equilibrium with higher carbon dioxide fugacities. These include compositions that correspond to thermally perturbed equivalents of the Ca-porewater (W5) and the Na-porewater (W4). A third water composition (W6) from *Engineered Barrier System: Physical and Chemical Environment Model* (BSC 2004 [DIRS 169860], Section 6.6) was also used to represent a high-temperature composition that could enter a breached waste package. These compositions, along with the temperature and carbon dioxide fugacity, were taken from EQ3 files obtained from DTN: MO0303MWDSCMAB.000 [DIRS 162551] and are listed in Table 4.3.

The input waters selected for sensitivity tests of the in-package model have compositions corresponding to waters that could enter the drift as crown seepage. These compositions were taken from *Drift-Scale THC Seepage Model* (BSC 2005 [DIRS 172862]). The seepage water compositions used as input to the calculations of *Engineered Barrier System: Physical and Chemical Environment Model* (BSC 2004 [DIRS 169860]) are also based on the results of the THC model. For there to be seepage—as opposed to vapor diffusion—into canisters requires that the drip shields have been breached. In this case, water impinging on the canister will be crown seepage water rather than in-drift waters of the type discussed by *Engineered Barrier System: Physical and Chemical Environment Model* (BSC 2004 [DIRS 169860]). The in-drift waters can potentially have severe chemistries due to evaporation, and these waters are of concern due to their contact with the waste package, and their potential to enhance corrosion of the waste package. However, the waters that could potentially drip onto the waste package in significant quantities for significant lengths of time will be seeping from rock above the waste package due

to the forces of gravity, which will be the crown seepage waters and not the in-drift waters. While it is possible that some of the in-drift waters will make their way into the waste package as run off from those components residing above the waste package, the bulk of the waters seeping into the waste packages will be those that drip from above the drift.

Though the input waters do not include the evaporated in-drift waters, the thermally perturbed waters included in Table 4-3 have undergone evaporation; therefore, this model accounts for evaporation for the selected incoming waters.

Table 4-3. Thermally Perturbed Seepage Compositions

| Source | | Input File Names | | |
|-------------------------------|----------|---|--|--|
| Parameter | Units | 10006 | 20013 | 51 |
| | | Cross-drift water CS2000/16.5 <i>thc6_w4_r.xls</i> TF4 directory | Cross-drift water CS1000/7.3 <i>thc6_w5_r.xls</i> TF4 directory | Cross-drift water SD-9/990.4 <i>thc6_w6_r.xls</i> TF4 directory |
| Temperature | °C | 56.2 | 40.2 | 91.8 |
| logfCO ₂ | - | -1.950 | -2.198 | -2.532 |
| pH | - | 7.78 | 7.94 | 8.14 |
| HCO ₃ ⁻ | Molality | 6.58E-03 | 6.92E-03 | 2.04E-03 |
| Ca ²⁺ | Molality | 4.40E-04 | 5.73E-04 | 7.24E-05 |
| Cl ⁻ | Molality | 7.12E-04 | 5.61E-04 | 7.34E-04 |
| F ⁻ | Molality | 7.81E-04 | 6.43E-04 | 9.77E-04 |
| K ⁺ | Molality | 4.55E-04 | 2.76E-04 | 5.02E-04 |
| Mg ²⁺ | Molality | 5.19E-05 | 8.51E-05 | 2.54E-07 |
| Na ⁺ | Molality | 7.49E-03 | 7.31E-03 | 4.27E-03 |
| SO ₄ ²⁻ | Molality | 3.94E-04 | 3.55E-04 | 1.18E-04 |
| SiO ₂ (aq) | Molality | 2.43E-03 | 1.79E-03 | 4.15E-03 |
| NO ₃ ⁻ | Molality | 6.58E-05 | 3.97E-05 | 3.10E-04 |
| NH ₃ (aq) | Molality | 1.00E-12 | 1.00E-12 | 1.00E-12 |
| Al ³⁺ | Molality | 3.73E-09 | 1.50E-09 | 6.09E-08 |

Source: DTN: MO0303MWDSCMAB.000 [DIRS 162551].

NOTE: fCO₂ is carbon dioxide fugacity.

The incoming water composition for the vapor-influx case is pure water, which simulates water vapor entering and then condensing in the waste package.

The potential flow rate for water flux into the waste package is a function of seepage rate, drift-wall condensation, condition of the drip shield, and number and types of openings in a breached waste package. The potential flow rate for water vapor flux into the waste package is a function of relative humidity, temperature, and number and types of openings in a breached waste package. To cover the possible range of water fluxes, values from *Abstraction of Drift Seepage* (BSC 2004 [DIRS 169131]) and *EBS Radionuclide Transport Abstraction* (BSC 2004 [DIRS 169868]) used as bounding cases are presented in Table 4.4. The seepage values are used to ensure water flow rates considered in this model are sufficient to cover the potential seepage rates to be used by TSPA-LA. TSPA-LA starts with the seepage rates from *Abstraction of Drift Seepage* (BSC 2004 [DIRS 169131]) and adjusts the rate depending on the condition of the drip shield and the number and types of openings in the breached waste package. The maximum

seepage flow rate for the purposes of this calculation is the mean seepage rate for the upper-bound scenario with the glacial transition climate in the Tptpmn. The true maximum seepage flow is on the order of 10,000 kg/yr per waste package, but the drip shield and the remaining waste package divert most of the seepage flow, so it would not be reasonable to model this flow entering a waste package. Therefore, these rates represent the maximum range of rates that could enter a waste package, and are justified.

The water vapor rates from *EBS Radionuclide Transport Abstraction* (BSC 2004 [DIRS 169868]) were calculated to provide a range of vapor diffusion rates through stress corrosion cracks for representative temperature and relative humidity conditions, and ranged from 22 moles to 239 moles (which corresponds to 0.4 liters to 4.3 liters). These vapor rates are calculated in an alternative conceptual model (BSC 2004 [DIRS 169868]), and are not actually product output used by TSPA-LA. The use of this data as input is justified by an additional calculation in Appendix H, documented in Section 6.4.3, which corroborates these ranges.

Table 4-4. Flux Into Waste Package

| Phase | Minimum Flow Rate per Waste Package | Maximum Flow Rate per Waste Package | Source |
|------------------|-------------------------------------|-------------------------------------|---|
| Liquid (Seepage) | 0.0 kg/yr | 294.6 kg/yr | BSC 2004 [DIRS 169131], Table 6.8-1 and 6.8-2 |
| Vapor | 22.1 mol H ₂ O/yr | 239 mol H ₂ O/yr | BSC 2004 [DIRS 169868], Table 6.6-1 |

NOTE: Minimum and maximum water vapor flow rates were calculated in BSC 2004 [DIRS 169868] for 30°C and 70°C, respectively.

4.1.3 Waste Form Compositions and Degradation Rates

4.1.3.1 CSNF

The source for the composition of the PWR fuel used for the CSNF in the in-package chemistry model is *PWR Assembly End-Effect Reactivity Evaluation* (BSC 2003 [DIRS 169110]), which starts with fresh fuel (UO₂) and calculates the composition of the irradiated fuel when it is discharged from the reactor (specified by burnup) and at specified times after discharge. The calculations were performed in that report using the SAS2H sequence and the ORIGEN sequence of the SCALE computer code system. The calculation covers initial enrichment of 2 wt % to 5 wt % (fuels less than 2% enriched are bounded by this calculation) ²³⁵U, and burnup of 0 to 50 GWd/MTU. The quantity of fuel in one assembly is based on the Babcock and Wilcox (B&W) 15 × 15-assembly design with the mass of type MK-B2. The MK-B2 design contains the greatest mass of uranium per assembly (CRWMS M&O 1998 [DIRS 104440], p. 26). The B&W 15 × 15 fuel assembly is one of the most reactive 21-PWR designs (B&W Fuel Company 1991 [DIRS 104439], p. II 6-6). A representative assembly for CSNF with an initial enrichment of 3.5 wt % ²³⁵U and a burnup of 40 GWd/MTU was chosen by visual depiction of the central region of the CSNF assembly population illustrated in *21-PWR Waste Package with Absorber Plates Loading Curve Evaluation* (BSC 2004 [DIRS 172553], Figure 34).

The recommended CSNF degradation rate law (BSC 2004 [DIRS 169987], Tables 8-2 and 8-3) is pH, O₂ partial pressure, total carbonate, and temperature dependent. This rate, Log(F/A), is expressed in units of mg/m²/day, and is appropriate for temperatures less than 100°C, and total

carbonate concentrations greater than 2×10^{-4} molar, and an oxygen partial pressure from 0.002 atm to 0.2 atm. For low $f\text{CO}_2$ conditions (lower than a total carbonate molar concentration equal to 2×10^{-4} molar), the rate described in Equation 4-1 should be used with a total carbonate molar concentration equal to 2×10^{-4} molar (BSC 2004 [DIRS 169987], Section 8.2).

For $\text{pH} > 6.8$ (BSC 2004 [DIRS 169987], Section 8.1, using Table 8-2):

$$\text{Log}(F) = \text{Log}(A) + 4.705 + (-1,093.826) \times \text{IT} + (-0.102) \times p\text{CO}_3 + (-0.338) \times p\text{O}_2 \quad (\text{Eq. 4-1})$$

For $\text{pH} \leq 6.8$ (BSC 2004 [DIRS 169987], Section 8.1, using Table 8-3)

$$\text{Log}(F) = \text{Log}(A) + 6.60 + (-1,093.826) \times \text{IT} + (-0.338) \times p\text{O}_2 + (-0.340) \times \text{pH} \quad (\text{Eq. 4-2})$$

where

| | | |
|-----------------|---|--|
| $\text{Log}(F)$ | = | \log_{10} fractional dissolution rate of the fuel (per day) |
| $\text{Log}(A)$ | = | \log_{10} of the fuel effective specific surface area (m^2/mg) |
| IT | = | inverse temperature (Kelvin^{-1}) |
| $p\text{CO}_3$ | = | $-\log_{10}$ (total molar carbonate species concentration) |
| $p\text{O}_2$ | = | $-\log_{10}$ (oxygen partial pressure in atmospheres) |

This combined rate is appropriate for use for a pH range from 2 to 10.3 (BSC 2004 [DIRS 169987], Section 8.2).

Summary Report of Commercial Reactor Criticality Data for Crystal River Unit 3 (Punatar 2001 [DIRS 155635], Table 2-2, p. 2-5), *IED Waste Package Configuration [Sheet 1 of 1]* (BSC 2005 [DIRS 173501], Tables 1 and 3); BSC 2004 [DIRS 172214], p. II-2 and II-3; BSC 2004 [DIRS 168199]; BSC 2004 [DIRS 168253]) provide inputs required to calculate the quantity and surface areas of waste package materials comprising the 21-PWR waste package. These calculations are contained in Appendix F, *CSNF.xls*. The 21-PWR assembly volume, calculated in *CSNF.xls* is corroborated by a volume value of 0.081 m^3 from *Criticality Evaluation of Degraded Internal Configurations for the PWR AUCF WP Designs* (CRWMS M&O 1997 [DIRS 102824] Table 4.1-1).

4.1.3.2 N-Reactor Fuel

N Reactor (U-Metal) Fuel Characteristics for Disposal Criticality Analysis (DOE 2000 [DIRS 150095], Table 3-1), for Mark IV fuel, provides the N-Reactor fuel composition information, including multi-canister overpack (MCO) dimensions, used in the in-package chemistry model. This reference is an appropriate source for N-Reactor fuel composition and MCO dimensions because the purpose of the report was to document this information and it contains the most detailed information available on the specific MCO dimensions. Praga (1998 [DIRS 172869]) further corroborates the N-Reactor fuel composition and the MCO dimensions information. Due to the poor N-Reactor fuel cladding condition (DOE 2000 [DIRS 152658]), the fuel is always considered 100% exposed.

A linear degradation rate (i.e., constant reaction rate) of 1.4×10^4 mg/(m²·d) is used to describe the dissolution of the N-Reactor fuel. This rate is five times the constant U-metal rate reported in *Review of Oxidation Rates of DOE Spent Nuclear Fuel, Part 1: Metallic Fuel* (DOE 2000 [DIRS 152658], Equation 2-39). That report (DOE 2000 [DIRS 152658]) contains the N-Reactor fuel degradation rate, which is the property of interest. This reference is an appropriate source for the N-Reactor fuel rate, as it analyzed the linear rate for N-Reactor fuel over a suitable time under conditions relevant to the repository. Gray and Einziger (1998 [DIRS 109691], Section 4.3) document a rate of 1.3×10^4 mg/(m²·d), thereby corroborating the rate from the DOE (2000 [DIRS 152658]) reported rate. While the TSPA-LA models the dissolution of DSNF as instantaneous (i.e., occurring within one time step), the time steps in TSPA-LA are considerably larger than the time steps within EQ6. The typical TSPA-LA timestep is 100 years or more, and the minimum is 10 years. Using the above rate in EQ6 results in complete dissolution of the DSNF within 15 years, which is essentially the same as TSPA-LA instantaneously degrading the fuel. Because this rate equates to essentially instantaneous degradation of the DSNF, it is also appropriate for the higher temperature runs.

N Reactor (U-Metal) Fuel Characteristics for Disposal Criticality Analysis (DOE 2000 [DIRS 150095], Section 4, pp. 23 and 24) and *IED Waste Package Configuration [Sheet 1 of 1]* (BSC 2005 [DIRS 173501], Tables 1 and 9), BSC 2004 [DIRS 172214] (p. X-2 and X-3), BSC 2004 [DIRS 168199], and BSC 2004 [DIRS 168253] also provide the inputs required to calculate the quantity and surface areas of the waste package materials for the 2-MCO/2-DHLW waste package (as shown in Appendix F, *CDSP.xls*).

4.1.3.3 HLW Glass

The simplified HLW glass composition in Table 4-5 is from Table 3 of *EQ6 Calculations for Chemical Degradation of N Reactor (U-metal) Spent Nuclear Fuel Waste Packages* (CRWMS M&O 2001 [DIRS 153263]), which cites *Defense Waste Processing Facility Wasteform and Canister Description* (Baxter 1988 [DIRS 106164], Table 6-8). The Savannah River glass composition was used in the model to maintain continuity with other analyses (CRWMS M&O 2001 [DIRS 153263]). Furthermore, Savannah River glass has a higher concentration of potassium (about 23 times more than Hanford glass) (CRWMS M&O 2000 [DIRS 151947], p. 8, Table 5-1). Because of its higher alkali composition (i.e., potassium), the Savannah River glass will produce a broader range of pH conditions, thus encompassing the range of chemistries generated by more mild glasses such as Hanford glass.

Table 4-5. High-Level Waste Glass Composition

| Element | Mol/100g* HLWG |
|---------|----------------|
| O | 2.70E+00 |
| U | 7.82E-03 |
| Ba | 1.08E-03 |
| Al | 8.63E-02 |
| S | 4.01E-03 |
| Ca | 1.62E-02 |
| P | 4.89E-04 |
| Si | 7.76E-01 |
| B | 2.91E-01 |

Table 4-5. High-Level Waste Glass Composition (Continued)

| Element | Mol/100g* HLWG |
|---------|----------------|
| F | 1.66E-03 |
| Fe | 1.72E-01 |
| K | 7.51E-02 |
| Mg | 3.33E-02 |
| Na | 5.77E-01 |

Source: CRWMS M&O 2001 [DIRS 153263], Table 3.

NOTE: * Elemental mole % of the glass is based on 100 grams of reactant.

The HLWG degradation rate expression (BSC 2004 [DIRS 169988]) follows the transition-state rate law and is dependent on the pH of the solution. The rate law coefficients provided were deemed the “most probable” in *Defense HLW Glass Degradation Model* (BSC 2004 [DIRS 169988]). This rate is appropriate for temperatures less than 100°C. The exposure factor, f_{exposure} , is the value that is multiplied by the geometric surface area of the HLW glass to achieve an effective surface area, which includes an increase in surface area due to fractures in the glass. The most likely value for f_{exposure} is 4, and the maximum value is 17 (BSC 2004 [DIRS 169988], Table 8-1).

For acidic conditions (BSC 2004 [DIRS 169988], Section 8.1, Equation 50):

$$rate_G = k_{E_acidic} \times 10^{-0.49 \cdot \text{pH}} \times \exp(-31 \text{ kJ/mol / RT}) \quad (\text{Eq. 4-3})$$

For alkaline conditions (BSC 2004 [DIRS 169988], Section 8.1, Equation 51):

$$rate_G = k_{E_alkaline} \times 10^{0.49 \cdot \text{pH}} \times \exp(-69 \text{ kJ/mol / RT}) \quad (\text{Eq. 4-4})$$

where (BSC 2004 [DIRS 169988], Section 8.1):

- $rate_G$ = the glass rate law in units of g/m²/day
- k_{E_acidic} = the glass degradation rate coefficient for acidic solutions: the minimum and most probable value of k_{E_acidic} is 8.41×10^3 g/(m²·d), and the maximum value of k_{E_acidic} is 1.15×10^7 g/(m²·d)
- $k_{E_alkaline}$ = the glass degradation rate coefficient for alkaline solutions: the minimum and most probable value of $k_{E_alkaline}$ are 2.82×10^1 g/(m²·d), and the maximum value of $k_{E_alkaline}$ is 3.47×10^4 g/(m²·d).

IED Waste Package Configuration [Sheet 1 of 1] (BSC 2005 [DIRS 173501], Tables 1 and 9), BSC 2004 [DIRS 172214] (p. X-2 and X-3), BSC 2004 [DIRS 168199], and *Design and Engineering, Spread Ring Sub-Assembly* (BSC 2004 [DIRS 168253]) provide the inputs required to calculate the waste package material quantity and surface areas comprising the 2-MCO/2-DHLW waste package (as shown in Appendix F, *CDSP.xls*).

Taylor (1997 [DIRS 126175]) and *Characteristics of Potential Repository Wastes* (DOE 1992 [DIRS 102812], Figure 3.4.2) provide direct input dimensions (for increased length and thickness) of the glass pour canisters (GPC), which *DOE and Commercial Waste Package System Description Document* (BSC 2005 [DIRS 174225], Table 3-1) corroborates. The DOE

(1992 [DIRS 102812]) report contains the properties of interest, which are the GPC dimensions. This reference is an appropriate source for these dimensions because the purpose of the report was to document this information and it contains the most detailed information available on the specific dimensions of the MCO.

4.1.4 Waste Package Material Compositions and Degradation Rates

Table 4-6 contains the names and shorthand names used in this model and waste package type for the metal alloys used throughout this document.

Table 4-6. Materials Nomenclature and Waste Package Breakdown

| Material Nomenclature | Shorthand | Waste Package Type | |
|-------------------------------------|---------------------------|--------------------|------|
| | | CSNF | CDSP |
| Alloy 22 (UNS N06022) | Alloy 22 | X | X |
| SA-240 (UNS S31600) Stainless Steel | Stainless Steel Type 316 | X | X |
| SA-240 S30403 Stainless Steel | Stainless Steel Type 304L | | X |
| SA-516 Grade 70 Carbon Steel | Carbon Steel Type A516 | X | X |
| SB-209 6061 T4 | Aluminum Alloy Type 6061 | X | |
| Aluminum Alloy-1100 | Aluminum Alloy Type 1100 | | X |
| Alloy (UNS N06464) | NiGd Alloy | X | |
| Neutronit A978™ | Neutronit ^a | X | |

NOTE: ^aNeutronit, a borated stainless steel, was replaced by NiGd alloy in the latest CSNF design, and is only presented here for comparison.

The CSNF 21-PWR absorber plate waste package components use the following materials: Alloy 22 for the outer corrosion barrier; Stainless Steel Type 316 for the inner vessel; Carbon Steel Type A516 for the basket guides, stiffeners, and fuel basket tubes; and Aluminum Alloy Type 6061 and NiGd Alloy for the fuel basket plates (BSC 2004 [DIRS 172214], p. II-2 and II-3). The main CDSP waste package (N Reactor) components use the following materials: Alloy 22 for the outer corrosion barrier, Stainless Steel Type 316 for the inner vessel, Carbon Steel Type A516 for the divider plate fuel support plate assemblies (BSC 2004 [DIRS 172214], p. X-2 and X-3), Stainless Steel Type 304L for the multicaster overpack (MCO) (DOE 2000 [DIRS 150095], Section 4) and glass pour canisters (GPC) (BSC 2005 [DIRS 174225], Table 3-1), and Aluminum Alloy Type 1100 for the MCO spacer (DOE 2000 [DIRS 150095], Section 4).

Table 4-7 summarizes the composition of the steel and aluminum alloys present in the CSNF and CDSP waste packages. While there may be some variability in the composition of the alloys used in the construction of the waste packages, the values in Table 4-7 are ASTM standards, which vendors are required to meet with little variation. However, a sensitivity analysis is conducted in Section 6.6.9 that varies the composition of Carbon Steel Type A516 to show the effect on the model response. The alloy compositions represent the best available information on the materials that will be used in the construction of the waste package.

Table 4-7. Composition of Steel and Aluminum Alloys

| Element | Carbon Steel Type A516 (wt %) | Neutronit ^e (wt %) | Aluminum Alloy 6061 (wt %) | Stainless Steel Type 316 (wt %) | Aluminum Alloy 1100 (wt %) | Stainless Steel Type 304L (wt %) | NiGd Alloy (wt %) |
|--------------|----------------------------------|--|-----------------------------------|-----------------------------------|-----------------------------------|-----------------------------------|-----------------------------------|
| C | 0.28 ^a | 0.04 | — | 0.08 | — | 0.03 | 0.01 |
| Mn | 0.85 to 1.2 (1.025) ^b | — | 0.15 | 2.00 | 0.05 | 2.00 | 0.5 |
| P | 0.035 | — | — | 0.045 | — | 0.045 | 0.005 |
| S | 0.035 | — | — | 0.03 | — | 0.03 | 0.005 |
| Si | 0.13 to 0.45 (0.29) ^b | — | 0.40 to 0.8 (0.60) ^b | 0.75 | 0.95 (Si+Fe) 0.45 | 0.75 | 0.08 |
| Cr | — | 18.5 | 0.04 to 0.35 (0.195) ^b | 16.0 to 18.0 (17.00) ^b | — | 18.0 to 20.0 (19.00) ^b | 14.5- to 17.1 (15.8) ^b |
| Ni | — | 13 | — | 10.0 to 14.0 (12.00) ^b | — | 8.0 to 12.0 (10.00) ^b | 64.035 ^d |
| Co | — | 0.2 | — | — | — | — | 2 |
| Mo | — | 2.2 | — | 2.00 to 3.00 (2.50) ^b | — | — | 13.1 to 16.0 (14.55) ^b |
| N | — | — | — | 0.1 | — | 0.10 | 0.01 |
| Fe | 98.3 ^d | 64.815 | 0.7 | 65.495 ^d | 0.95 (Si+Fe) 0.50 | 68.045 ^d | 1 |
| B | — | 0.75 to 1.74 ^c (1.245) ^b | — | — | — | — | — |
| Zn | — | — | 0.25 | — | 0.10 | — | — |
| Cu | — | — | 0.15 to 0.40 (0.275) ^b | — | 0.05 to 0.20 (0.125) ^b | — | — |
| Mg | — | — | 0.8 to 1.2 (1.0) ^b | — | — | — | — |
| Ti | — | — | 0.15 | — | — | — | — |
| Al | — | — | 96.68 ^d | — | 98.78 ^d | — | — |
| Gd | — | — | — | — | — | — | 1.9 to 2.1 2.0 ^b |
| O | — | — | — | — | — | — | 0.005 |
| Total | 100.0 | 100.0 | 100.0 | 100.0 | 100.0 | 100.0 | 100.0 |

Source: Table 4-1.

NOTE: ^a Value is for Grade 70 thicknesses between ½ in to 2 in.

^b Represents the average of the range specified in source.

^c Ranges for types B3, B4, and B6.

^d Balance values calculated based on the averaged values.

^e Neutronit is presented in this table for comparison purposes only; its composition is not a direct input to this report, as this material has been changed to NiGd alloy. The reference for the Neutronit composition is Kügler 1996 [DIRS 107760]; for B content, ASTM A 887-89 2000 [DIRS 154062], Table 1, S30463, S30464, and S30466.

Table 4-8 provides the densities and corrosion rates for the waste package metal alloys described in Table 4-7.

Table 4-8. Steel and Alloy Densities and Corrosion Rates

| | Carbon Steel Type A516 | Neutronit ^b | Aluminum Alloy 6061 | Stainless Steel Type 316 | Aluminum Alloy 1100 | Stainless Steel Type 304L | NiGd Alloy |
|--------------------------------|------------------------|------------------------|---------------------|--------------------------|---------------------|---------------------------|---------------------|
| Density (g/cm ³) | 7.85 | 7.76 | 2.70 | 7.98 | 2.71 | 7.94 | 8.76 |
| Minimum Corrosion Rate (µm/yr) | 3.69 | 0.001 | 0.4 | 0.0014 | 0.4 | 0.001 | 0.0528 ^a |
| Maximum Corrosion Rate (µm/yr) | 130.7 | 29.2 | 110.9 | 14.8 | 110.9 | 39.1 | 0.0774 |

Source: Table 4-1.

NOTE: ^aAverage Rate from DTN: MO0409SPAACRWP.000 [DIRS 172059].

^bNeutronit (Kügler 1996 [DIRS 107760]) is presented in this table for comparison purposes only; its composition is not a direct input to this report, since this material has been changed to the NiGd Alloy.

The corrosion rates used in this model encompass the ranges provided in DTN: MO0409SPAACRWP.000 [DIRS 172059] for temperatures from 25°C to 90°C. The corrosion rates used are justified because they consider the full range intended to encompass the possible range of degradation rates for the materials that make up the waste package as documented by *Probability Analysis of Corrosion Rates for Waste Package Materials* (BSC 2004 [DIRS 169982], p. 1-1).

The base case reflects the use of the NiGd alloy as a neutron absorber within the CSNF waste package. Previously, the design had called for Neutronit as the neutron absorber (BSC 2004 [DIRS 170710]). Section 6.6.7 contains a sensitivity analysis comparing the base-case CSNF (containing NiGd Alloy) with the old design that included Neutronit. This analysis demonstrates that, although the change in design results in a minor change of overall chemistry, the resulting pH and ionic strength of the solutions are still within the abstracted range provided to TSPA-LA by the base analysis.

4.1.5 Atomic Weights

Atomic weights of the elements and radionuclide isotopes used were taken from *Atomic Mass Adjustment, Mass List for Analysis* (Audi and Wapstra 1995 [DIRS 149625]) and *Nuclides and Isotopes, Chart of the Nuclides* (Parrington et al. 1996 [DIRS 103896], p. 50). These documents have been used as a source for this information throughout the Yucca Mountain Project and are appropriate as a source for atomic weights in this analysis. The atomic weights of the elements are used to convert the weight percent of the elements in the metal alloys to moles of elements in the metal alloys used as input in the EQ6 files.

4.1.6 Surface Complexation Inputs

The surface complexation reactions and the associated equilibrium constants (log K) presented in Table 4-9 are used to estimate the pH-buffering effects in the analytical surface complexation model developed in Sections 6.3.2 and 6.7. These reactions and log K values are internally consistent (Appelo et al. 2002 [DIRS 168168], p. 3,097).

Table 4-9. Surface Complexation Reactions

| Reaction | log K |
|--|--------------------|
| $\text{HFO_sOH} + \text{H}^+ = \text{HFO_sOH}_2^+$ | 7.29 ^a |
| $\text{HFO_wOH} + \text{H}^+ = \text{HFO_wOH}_2^+$ | 7.29 ^a |
| $\text{HFO_sOH} = \text{HFO_sO}^- + \text{H}^+$ | -8.93 ^b |
| Reaction | log K |
| $\text{HFO_wOH} = \text{HFO_wO}^- + \text{H}^+$ | -8.93 ^b |
| $\text{HFO_wOH} + \text{CO}_3^{2-} + 2\text{H}^+ = \text{HFO_wHCO}_3 + \text{H}_2\text{O}$ | 20.37 ^c |
| $\text{HFO_wOH} + \text{CO}_3^{2-} + \text{H}^+ = \text{HFO_wCO}_3^- + \text{H}_2\text{O}$ | 12.78 ^c |

Sources: ^aDzombak and Morel 1990 [DIRS 105483], Table 5.7, Equation 5.2.

^bDzombak and Morel 1990 [DIRS 105483], Table 5.7, Equation 5.3.

^cAppelo et al. 2002 [DIRS 168168], p. 3,097.

NOTE: HFO = hydrous ferric oxide. HFO_s and HFO_w represent adsorption sites, strong (s) and weak (w), respectively.

The equilibrium constant (K) is defined as the product of the concentration of the product divided by the concentrations of the reactants.

Surface properties of iron corrosion products are presented in Table 4-10. The isoelectric point (pH_{IEP}) is the pH at which the surface has a net neutral charge. It is also called the pristine point of zero charge (pH_{PPZC}) (Langmuir 1997 [DIRS 100051], p. 350). Various formulae have been assigned to amorphous ferric hydroxide due to its variable hydration state. Dzombak and Morel (1990 [DIRS 105483]) treat this phase using a molecular weight that is nearly equivalent to that of goethite (FeOOH) when expressed in a per mol of Fe basis, but it has also been described as Fe(OH)₃.

Table 4-10. Properties of Major Iron Minerals

| Mineral | Surface Area (m ² /gm) | Site Density (sites/area or wt) | Sites (mol/mol Fe) | Isoelectric Point (pH _{IEP}) |
|---|-----------------------------------|--|---|--|
| Goethite (FeOOH) | 45 to 169 | 2.6 to 18 sites/nm ² (1.35 mmol/g) | 0.017 to 0.45 ^e 0.12 ^e | 5.9 to 6.7 7.28 ^{a,f} |
| Hematite (Fe ₂ O ₃) | 1.8 (natural) 3.1 (synthetic) | 5 to 22 sites/nm ² (0.114 to 0.015 mmol/g) | 0.0024 to 0.018 ^e | 4.2 to 6.9 8.67 ^{a,f} |
| Ferrihydrite (poorly crystalline hydrous ferric oxide, HFO) | 250, 306, 600 | 20 sites/nm ² (2.2 mmol/g) ^d | 0.1 to 0.9 (0.2) ^{c,f} | 8.5 to 8.8 7.9 to 8.2 ^b |
| Bulk composite geological material | 600 | 2.31 sites/nm ² (3.84E-06 mol/m ² = 2.3 mmol/g) | N/A | N/A |

Sources: Langmuir 1997 [DIRS 100051], pp. 345 and 351, Tables 10.2 and 10.3 (unless otherwise noted)

^aStumm (1992 [DIRS 141778], p. 54)

^bHFO (Dzombak and Morel 1990 [DIRS 105483], p. 95)

^cApproximate median value chosen by Dzombak and Morel (1990 [DIRS 105483], p. 94).

NOTES: ^dCalculated from 0.2 moles sites/mole Fe and a formula weight of 89g ferrihydrite/mol Fe

^eCalculated using the formula given in Langmuir 1997 [DIRS 100051], p. 346: Sites (mol/mol Fe) = Site Density × Surface Area × 10¹⁸ nm²/m² × 89g ferrihydrite/mol Fe/Avogadro's number (6.022E23 sites/mol of sites)

^fThese numbers are presented for comparison purposes only and are not direct inputs to this report.

The surface complexation inputs are justified for use because they are representative of the best available data on surface complexation.

4.2 CRITERIA

Work described in this document supports the following criteria from *Yucca Mountain Review Plan, Final Report* (NRC 2003 [DIRS 163274]) as described in *Technical Work Plan for Postclosure Waste Form Modeling* (BSC 2005 [DIRS 173246]). The full text of these criteria is quoted in Section 8.2 along with a detailed explanation of how this document addresses those criteria and the location where the appropriate information can be found.

- Quantity and Chemistry of Water Contacting Waste Packages and Waste Forms, Acceptance Criteria (NRC 2003 [DIRS 163274], Section 2.2.1.3.3)
 - AC1–System Description and Model Integration are Adequate
 - AC2–Data are Sufficient for Model Justification
 - AC3–Data Uncertainty is Characterized and Propagated Through the Model Abstraction
 - AC4–Model Uncertainty is Characterized and Propagated Through the Model Abstraction
 - AC5–Model Abstraction Output is Supported by Objective Comparisons

4.3 CODES, STANDARDS, AND REGULATIONS

Standard Practice for Prediction of the Long-Term Behavior of Materials, Including Waste Forms, Used in Engineered Barrier Systems (EBS) for Geological Disposal of High-Level Radioactive Waste (ASTM C 1174-97 [DIRS 105725]), is used to support the model development methodology, categorize the models developed with respect to their usage for long-term total system performance assessment, and relate the information or data used to develop the model to the requirements of the standard.

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5. ASSUMPTIONS

5.1 BULK WATER CHEMISTRY

Assumption: In the absence of a thin film model, it is assumed the bulk water chemistry calculated by this batch reactor model is applicable to thin films of water.

Rationale: For liquid films thicker than 8 to 20 monolayers, the behavior is similar to bulk water (Sposito 1984 [DIRS 127253]). This observation is supported by Franks 1975 [DIRS 173728], which indicated that for films with thicknesses greater than 10 nm (equivalent to approximately thirty monolayers) the film behaves as bulk water.

1. The OH bond distance in a water molecule is ~ 0.1 nm; the length of a hydrogen bond between water molecules is ~ 0.3 nm (Stumm and Morgan 1996 [DIRS 125332], p. 7).
2. The effect of a cation on the structure of bulk water is localized to a suite of no more than 6 to 20 water solvation molecules (Sposito 1984 [DIRS 127253], p. 57). About 10 molecular layers of water on clay minerals exist at humidities of 98% (Sposito 1984 [DIRS 127253], p. 61). The spatial extent of adsorbed water on a phyllosilicate surface is, conservatively, whatever is included in the region bounded by a plane about 1.0 nm from the basal plane of the clay mineral. The bounding plane at 1.0 nm is expected to include all but a few percent of the siloxane surface effects on water structure (Sposito 1984 [DIRS 127253], pp. 69 to 70). A 1-nm layer of water is roughly 4 to 10 water molecules, depending on how they are arranged, so an 8- to 20-water molecule layer is, therefore, around 2 nm. In addition, cation hydration is similar to hydration of surface complexation sites (Sposito 1984 [DIRS 127253], p. 64). The above clay observations and the magnitude of cation hydration shell radii both suggest that 1 nm to 2 nm of surface associated water is not structurally different than bulk water (Sposito 1984 [DIRS 127253], pp. 57, 61, 64, 69, and 70).
3. Experimental studies of a 20-nm-thick water layer adsorbed onto metal plates indicate that its physical characteristics are consistent with bulk water (Zang and Grischokowsky 2004 [DIRS 173729]).

Confirmation Status: This modeling assumption, when combined with the diffusion implementation of the EBS radionuclide transport model (BSC 2004 [DIRS 169868]) within TSPA-LA, is conservative in its estimation of radionuclide release and is, therefore, justified, and does not require confirmation.

Use in the Model: This assumption is used throughout Section 6 in the development of the conceptual model.

5.2 CONSTANT SURFACE AREA AND CORROSION RATES OF ALLOYS

Assumption: In the absence of a surface area function within a geochemistry-modeling tool, the surface areas of the reactants are fixed and assumed not to vary with time.

Rationale: As a material degrades, its surface area will certainly change, but how it varies with time and degree of degradation is extremely complex. Though the actual surface area will increase with time, the effective surface area will decrease due to the formation of a protective layer of corrosion products. The corrosion rate itself also decreases with time due to the formation of oxide layers (DTN: MO0409SPAACRWP.000 [DIRS 172059]). However, the constant surface area simplification can only result in faster degradation, which maximizes the impact of the degradation on the solution chemistry. Also, the EQ6 code does not have the ability to implement a surface area function.

Confirmation Status: The impact of this simplification is that reactants may not degrade as quickly as modeled, but this difference is accounted for by varying the range of degradation rates over three orders of magnitude (Section 6.6.10) and is, therefore, justified and requires no further confirmation.

Use in the Model: This assumption is used in Sections 6.3.

5.3 THERMODYNAMIC EQUILIBRIUM

Assumption: In the absence of data on the long-term behavior of man-made materials in the waste package for the regulatory period, it is assumed that all homogeneous solution and gas reactions, and all heterogeneous gas-solution reactions are reversible and at equilibrium. Heterogeneous reactions between solutions and precipitating solids are also assumed to be at equilibrium except for those by which waste and waste-package components degrade and dissolve in the in-package fluids. These reactions are irreversible and described by various expressions for reaction kinetics. These assumptions are implicit in the EQ3/EQ6 reaction and reaction path codes used to model the evolution of the in-package chemistry. Some exceptions to this assumption are the suppression of certain minerals known only to form at high temperatures (as discussed in Section 6.3.1.3.5), and that the oxidation state of chromium is limited to Cr(III).

Rationale: The justification for equilibrium between solution and precipitating solids is because some solids might require longer times to reach equilibrium (even longer than the times considered in the modeling). These solids, which include high-temperature minerals, are explicitly excluded from consideration as described in Section 6.3.1.1. In this model, kinetic factors do not control the reactions, even though it is unlikely all reactions will actually reach equilibrium, even over the regulatory period.

Confirmation Status: This assumption is justified because it does not lead to nonconservatism (such as moderate pH values that would lead to lower solubilities, or moderate ionic strengths (>0.05) that would destabilize colloids [BSC 2004 [DIRS 170025]]). The impact of kinetic control for several species is examined in sensitivity studies in Section 6.6 and is shown to result in chemistries within the predicted ranges provided to TSPA-LA; therefore, no further confirmation is required.

Use in the Model: This assumption is used in Sections 6.3.

6. MODEL DISCUSSION

6.1 OBJECTIVES

The objective of the in-package chemistry model is to predict the possible bulk water chemistry within a breached waste package and to provide simplified expressions of the chemistry as a function of time after breach to TSPA-LA. This chemistry is simplified in the form of abstractions for pH ($\text{pH} = -\log [\text{H}^+]$), ionic strength, Eh (redox potential), total carbonate (i.e., sum of aqueous carbonate species), chloride, and fluoride concentration. Ionic strength is defined as a measure of interionic effect resulting primarily from electrical attraction and repulsions between the various ions; $I = \frac{1}{2} \sum_i c_i Z_i^2$, where c_i is the ion concentration, and Z_i is the ion charge. The summation is carried out for all cations and anions in solution (Stumm and Morgan 1996 [DIRS 125332], p. 101).

The pH, ionic strength, and total carbonate abstractions are used by TSPA-LA in conjunction with the other models to determine the stability of colloids, the solubility of radionuclides, and the degradation rates of the waste forms. The fluoride abstraction is not directly used by TSPA-LA, but is used by *Dissolved Concentration Limits of Radioactive Elements* (BSC 2005 [DIRS 173873]) to ensure predicted solubilities are valid over the range of fluoride concentrations possible in waste packages.

6.2 FEATURES, EVENTS, AND PROCESSES

Table 6-1 provides a list of “included” FEPs determined to be related to the in-package chemistry model (DTN: MO0407SEPFELA.000 [DIRS 170760]). The table also indicates where within this report the indicated FEP is addressed. The discussion of these FEPs, in the indicated sections, provides a summary of how the respective FEP is treated within TSPA-LA.

Table 6-1. Included FEPs for the In-Package Chemistry Model Report

| FEP Number | FEP Name | Addressed In |
|--------------|--|--|
| 2.1.01.02.0B | Interactions between co-disposed waste | Sections 6.5.1.2, 6.5.3, 6.6.13, 6.10.9 |
| 2.1.02.09.0A | Chemical effects of void space in waste package | Sections 6.5.3, 6.6.1, 6.6.4, 6.10.9 |
| 2.1.09.01.0B | Chemical characteristics of water in waste package | Sections 6.5.3, 6.7.1, 6.10.9 |
| 2.1.09.06.0A | Reduction-oxidation potential in waste package | Sections 6.5.1.1.3, 6.5.3, 6.6.4, 6.10.9 |
| 2.1.09.07.0A | Reaction kinetics in waste package | Sections 6.5.1.1, 6.5.1.2, 6.5.3, 6.6.3, 6.6.4, 6.6.10, 6.10.9 |
| 2.1.09.02.0A | Chemical interaction with corrosion products | Sections 6.3.2, 6.5.1.1, 6.5.2.2, 6.5.3, 6.7.1, 6.10.9 |
| 2.1.11.08.0A | Thermal effects on chemistry and microbial activity in the EBS | Sections 6.5.1.1, 6.5.3, 6.6.6, 6.7.3, 6.10.9 |
| 2.2.08.12.0B | Chemistry of water flowing into the waste package | Sections 6.5.1.1, 6.5.3, 6.6.2, 6.10.9 |

6.3 BASE-CASE CONCEPTUAL MODEL

The base-case conceptual model is a combination of a batch reactor model with a surface complexation model. The batch reactor portion of the model combines water, oxygen, carbon

dioxide, waste forms, and metal alloys. The system is in equilibrium with atmospheric conditions, and the reactants degrade in the presence of water according to a rate determined by the physical properties and the exposed surface area of each reactant. For each reactant, a mass per time rate is selected based on experimental measurements, and the surface area available to react is calculated based on the dimensions of the waste package internals. During reaction progress, metal (hydr)oxide corrosion products and secondary mineral phases precipitate from solution, waste forms and metal alloys are consumed, and water changes in its initial composition and mass due to these reactions. The corrosion products are generally less dense than the starting metals and waste forms, so as the reactions progress, the volume of the system increases. The surface complexation portion of the model examines the effects of the contact of the solution with corrosion product surfaces. An overall model limitation is that these two portions of the model are implemented in series by applying the surface complexation effects to the output of the batch reactor model; when in fact, surface complexation occurs as the reactants are degrading. However, the impact of this simplification is considered in the uncertainty of the output abstractions and is discussed in Section 6.9.

The base case is divided into two water cases: (1) water enters the waste package as a vapor and condenses, and (2) water enters the waste package as a liquid.

1. **Vapor-Influx Case:** The condensation of vapor onto the waste package internals is simulated as pure H₂O and enters at a rate determined by the water vapor pressure for representative temperature and relative humidity conditions.
2. **Liquid-Influx Case:** The water entering a waste package from the drift is simulated as typical groundwater and enters at a rate determined by the amount of seepage available to flow through openings in a breached waste package.

TSPA-LA uses the vapor-influx case for simulations where the waste package has been breached and the drip shield remains intact, so all seepage flow is diverted from the waste package. In this case, the driving force for water vapor condensation onto surfaces inside the breached waste package is the lowering of the water activity as a result of sorption and dissolution of soluble materials released in the corrosion of the waste forms and other materials inside the waste package. These driving forces are counteracted by evaporation when the waste form and other surfaces are hotter than other locations in the drift environment. The combined effects of temperature and solutes on water film accumulation on surfaces inside the breached waste packages are not modeled explicitly. Instead, the amount of water potentially available for sorption and condensation is calculated from the upper-bound rate of diffusion of water vapor into the breached waste packages (Section 6.4.3). Uncertainties in the effects of the actual water accumulation rate are addressed by assessing the sensitivity of the in-package chemistry to the water flux into the breached waste packages (Section 6.6.5). The chemistry from the vapor-influx case is used to determine the stability of colloids and the solubility of radionuclides available for transport by diffusion, and to determine the degradation rates for the waste forms. TSPA-LA uses the liquid-influx case when simulating a seismic event in which the waste package has been breached and the drip shield has been damaged such that seepage flow is actually directed into the waste package. The chemistry from the liquid-influx case, which is a function of the flow rate, is used to determine the stability of colloids, the solubility of radionuclides available for transport by diffusion and advection, and the degradation rates for the

waste forms. For the vapor-influx and the liquid-influx cases, the batch reactor model considers bulk water, but varies the total quantity of water from a bathtub configuration all the way down the least amount of water, which can still enable the code to run within its range of validation (Section 6.6.1). TSPA-LA applies the chemistry from the batch reactor model to thin films of water. This issue is discussed in Section 5.1.

The chemical processes considered in the model include:

- Reaction-path tracing of the in-package chemical system
- Conservation of mass
- Kinetic dissolution of the waste package components and spent nuclear fuel (N-Reactor fuel) using linear rate laws
- Kinetic dissolution of waste forms (CSNF and HLWG) using transition-state theory rate laws (i.e., nonlinear rate laws, which are dependent on the fluid chemistry)
- Equilibrium precipitation and dissolution of metal corrosion products and complex mineral phases
- Thermal (i.e., temperature) effects on fluid chemistry
- Equilibrium oxidation and reduction reactions
- Chemical interaction of input solution with codisposed waste (DSNF with HLWG)
- Effect of variable input water compositions on the resulting fluid composition
- Chemical effects of void space (proximity of waste package components with atmospheric conditions)
- Influence of surface complexation on the fluid pH.

The physical processes considered in the model include:

- Water ingress and egress (egress for the liquid influx only) through the waste package
- Mixing of water in contact with waste package components.

6.3.1 Batch Reactor Model

6.3.1.1 Batch Reactor Modeling Concepts and Assumptions

Thermodynamic Equilibrium—A thermodynamic chemical equilibrium approach is used for aqueous species in this report. The main justification for the use of the thermodynamic chemical equilibrium approach is the extended period of concern mandated by the regulations. Although the use of kinetic factors would be appropriate, such an approach cannot be used with the limited

available data. The effects of kinetically controlled reactions would be a more-neutral pH and lower ionic strengths. The pH and ionic strength ranges projected by the thermodynamic equilibrium approach is wide enough to encompass the pH and ionic strength ranges that would be projected by a kinetic approach, as shown in the sensitivity analyses in Section 6.6.3. The uncertainty ranges for the ionic strength model passed to TSPA-LA favor lower ionic strengths that account for the effect of the surface complexation model on ionic strength, as well as the effect of kinetics.

A good example of the thermodynamic-versus-kinetic approach is the treatment of sulfur. The complete oxidation of sulfur to SO_4^{2-} is a conservative way to examine the effects of corrosion on pH. At thermodynamic equilibrium, the S, N, and P in the steels will be oxidized to the acid anions SO_4^{2-} , NO_3^- , and PO_4^{3-} . Even if there are kinetic barriers to the completion of these oxidation reactions, modeling their equilibria leads to more extreme chemistries; lower pH values, and higher ionic strengths. To balance this treatment, the surface complexation model adds more realism to the model by buffering pH (Section 6.3.2).

The exception to thermodynamic equilibrium is the treatment of chromium. Allowing Cr to oxidize to Cr(VI) leads to artificially high ionic strengths. Therefore, this model limits the oxidation state of Cr to Cr(III) rather than Cr(VI). Other examples of elements that oxidize due to the thermodynamic equilibrium model include C and B, but the amount of these elements in the composition of the waste package components is not significant enough to impact the results.

CO₂ and O₂ Partial Pressure—Oxygen and carbon dioxide maintain equilibrium with the waste package solution and the ambient atmosphere outside of the waste package. The partial pressure of CO₂ and O₂ of the ambient repository atmosphere are set to $10^{-3.0}$ and 0.2 atm, respectively ($\log f_{\text{CO}_2} = -3$ and $\log f_{\text{O}_2} = -0.7$). The oxygen partial pressure is set to equal atmospheric pressure (Weast 1977 [DIRS 106266], p. F-210). The carbon dioxide pressure is set higher than the atmospheric value because ambient fluids drawn from boreholes near the repository horizon appear to be in equilibrium with above-atmospheric carbon dioxide levels (Yang et al. 1996 [DIRS 100194], Table 8). Sensitivity analyses of the model response to the CO₂ and O₂ partial pressure are discussed in Section 6.6.4. Nitrogen and minor and trace gases in the atmosphere were not included in the model as gases because they are either inert or of such low concentration to be of no consequence on the model results.

Evaporation—The effects of evaporation inside of the waste package are not modeled explicitly for the liquid-influx case. In addition, the model does not explicitly account for the possibility of seepage concentrated via evaporative processes entering the waste package. Another assumption implicit in the “no-evaporation” assumption is that water will condense in the waste package when the waste package is breached; this assumption is required in order to have an in-package chemistry model.

There will be a period immediately after waste package breach (when water first enters and the relative humidity increases) where evaporation inside a waste package will be high, but this phenomenon was not included in the EQ6 modeling of the in-package chemistry. At high flux values, the effect of early-time evaporative processes is likely to have a negligible impact on the model results because the humidity inside of the breached waste package would increase quickly and the high water flux would diminish any evaporative effects on the fluid composition. At low

water flux values, the opposite would be true. Humidity would increase slowly, and low water flux would have less of a dilution effect.

Engineered Barrier System: Physical and Chemical Environment (BSC 2004 [DIRS 169860]) examines the effects of evaporation on crown seepage waters of the chemistries used as input to this model. For evaporated seepage waters, the results show that after minimal evaporation to RH values of no lower than 95% to 98%, the waters have reached ionic strengths exceeding 1 molal (BSC 2004 [DIRS 169860], Figures 6.13-2 to 6.13-12). Such water would be unsuitable for use as input water for the liquid-influx case because the EQ3/6 codes and database used for the in-package modeling are not intended for use with waters of ions strengths above 1 molal (Section 6.10.2). The version of EQ3/6 database for use at high ionic strengths (the Pitzer version) could not be used because at the time *Engineered Barrier System: Physical and Chemical Environment* (BSC 2004 [DIRS 169860]) and the in-package modeling were done, the available Pitzer database (DTN: SN0302T0510102.002 [DIRS 162572]) lacked parameters for such important constituents of in-package materials such as U, Ni, Mo, and Gd. However, these and additional elements have since been added to the database (DTN: SN0504T0502404.011 [DIRS 173493]). In addition, the principal solutes following evaporation will be those of soluble salts (such as Cl, Na, Mg, etc.). Figures 6-7 and 6-8, for CSNF (Section 6.5.1.1), and Figures 6-14 and 6-15 for CDSP (Section 6.5.1.2), illustrate that changes in the concentrations of solutes such as Na do not affect solutes derived from the waste package. As stated in Section 4.1.2, the crown seepage waters, rather than the in-drift waters, are the waters more likely to enter the waste package in any significant quantity; and the use of the thermally perturbed seepage waters (Table 4-3) do account for evaporation.

Therefore, the effects of evaporation in the EQ6 modeling are accounted for by the broad uncertainty range for the ionic-strength abstraction, and effects of the evaporation of the incoming waters are accounted for by use of the thermally perturbed seepage waters. Evaporation is examined in more detail in Sections 6.5.1.1.2 and 6.10.2.

Well-Mixed Batch Reactor—The reactants in the batch reactor are well mixed. Because the waste package components are evenly distributed and in proximity to one another inside of the waste package, and an important aspect of the model is to investigate the effects of the interactions of the various components, the well-mixed concept is justified. In addition, the waste package will be cooling slowly over time, so the temperature gradients within the waste package will contribute to convective flow that will further mix the contents of the waste package. Localized effects (small areas with reducing conditions and more extreme pH values and ionic strengths) will occur, but these will be transient and not significant enough to alter model results.

Approximately half of the waste package (CSNF or CDSP) volume is taken up with the waste form and structural components and the rest of the waste package is empty. This empty space is referred to as the void space and can fill with water. The waste package is tightly packed (there are no large gaps of space), but individual components are relatively thin sheets of metal or small pellets of fuel packed tightly together, so available space for water is spread throughout the waste package. This spatial configuration is further justification for a well-mixed model.

Scaling to One Liter—The waste package is represented in the EQ6 input file as amounts, compositions, and surface areas of the major materials, such as carbon and stainless steel, aluminum or other alloys, and different fuel types. Each material is also assigned a reaction rate that EQ6 combines with the surface area to determine the actual rate at which the material degrades. The water influx rate is calculated to provide a representative quantity of water to interact with the waste package components at a rate corresponding to the rate at which seepage water could enter a waste package, or the rate at which vapor could enter a waste package and condense.

EQ6 works on a per liter basis; therefore, all of the reactants and the water flow rate are scaled accordingly. To maintain the proper ratio of each of the materials to each other, the reactants are scaled based on the total surface area of all of the waste package materials. The ratio of the waste package components to the amount of water that they come in contact with is treated as a variable in this model. For one case, the ratio is such that the materials of the waste package are in contact with a volume of water equal to that of the void space. This case is referred to as the “bathtub” model and has the highest ratio of water to waste package materials. Other ratios are examined in which less water is considered. By lowering the ratio of water to reactants, more reactants are available to react that approximates the effects of capillary forces distributing the water to the tight spaces in the waste package, even if the waste package is not completely filled. The base case uses a ratio of water to reactants that is half of the “bathtub” model to examine a lower ratio. The impact of varying this ratio is examined in a sensitivity analysis in Section 6.6.1.

As the ratio of water to reactants is decreased, the ionic strength of the solution increases. As described in the B-dot discussion below, this model is appropriate for use up to about 4 molal. The lowest ratio of water to reactants used in this model approaches this limit, so that a lower ratio could not be examined without exceeding the 4-molal limit. Because ionic strength is an output of this model and this high ionic strength boundary condition affects radionuclide solubility, this 4-molal boundary condition is discussed in greater detail below. The base case uses a ratio of water to reactants that is half of the “bathtub” model to examine a lowest possible ratio.

The highest ionic strengths predicted by the in-package chemistry models are 3 to 4 molal. This upper limit captures the maximum concentrations that would be expected in the aqueous solution because as ionic strength gets above about 1 molal, a “salting out” effect starts to occur, in which cations get less soluble as the ionic strength increases. As an explanation, reactions that control solubility are written in terms of activity, where activity (a) is the molal concentration of the species (m) times the activity coefficient (γ), $a = m \times \gamma$ (Langmuir 1997 [DIRS 100051], Equation 4.1). Figure 4.2 of Langmuir (1997 [DIRS 100051]) shows that for cations, as the ionic strength increases above about 1 to 2 molal, the activity coefficient increases. For a set activity value, when the activity coefficient increases, the concentration of dissolved cations decreases.

Dissolved Concentration Limits of Radioactive Elements (BSC 2005 [DIRS 173873], Section 6.3.3.4) also notes, at high ionic strengths activity, coefficients calculated using the algorithm in EQ3/6 v. 7.2b diverge from those calculated from experimental data. In that report (BSC 2005 [DIRS 173873]), the influence of this divergence on solubilities is not expressed by decreasing the solubilities to reflect the salting out effect because the species dominating the solubilities in some cases are anions, which do not exhibit the salting out effect. Instead, the uncertainties of

solubilities modeled at ionic strengths above 1 molal are increased by an amount corresponding to the maximum divergence in the ionic strengths shown in Figures 6.3-4 and 6.3-5 of *Dissolved Concentration Limits of Radioactive Elements* (BSC 2005 [DIRS 173873]).

The surface area of the CSNF and N-Reactor fuel are treated as the geometric surface area because the rates reported in Section 4.1.3 are given in terms of geometric surface area. A sensitivity study in Section 6.6.10 examines the use of specific surface area for the CSNF. Similarly, the measured rates of the steels are based on a geometric surface area. The surface area of the HLWG is the geometric surface area multiplied by the exposure factor (BSC 2004 [DIRS 169988]).

Waste Package Materials not Included in the EQ6 Input File—The waste package outer barrier is constructed of Alloy 22, and the fuel cladding is made of Zircaloy, both of which degrade slowly compared to the other materials in the waste package. These two materials are not included in the EQ6 input file because they would react so slowly that the effect on the results would be negligible. The corrosion rate for Zircaloy is low (BSC 2004 [DIRS 169982], Section 6.2.5) and the probability of Alloy 22 corroding quickly is low as stated in of *General Corrosion and Localized Corrosion of Waste Package Outer Barrier* (BSC 2004 [DIRS 169984], Section 8.1).

In the EQ6 input file, only the inside surface area of the Stainless Steel Type 316 inner vessel is modeled because only the inside of the vessel is in contact with the solution. The gap between the inner vessel and the outer barrier is small and, even if water did manage to enter this gap, it would be a small quantity and would not contact the solution inside the inner vessel. Only the outside surface area of the glass pour canisters was used because they are filled with glass, so the majority of the surface area exposed to water will be the outside of the canister.

Codisposal Waste Packages—The N-Reactor spent nuclear fuel was chosen to represent all DOE spent nuclear fuel because it accounts for about 85 percent of the total metric tons of heavy metal of DOE spent nuclear fuel (DOE 2002 [DIRS 158405]). This selection was deemed appropriate by a sensitivity in Section 6.6.13 that examines the behavior of a waste package containing five HLWG canisters and one DOE canister containing one assembly of Three Mile Island (TMI) SNF (waste package type: 5-DHLW long), and the results are similar.

Corrosion—Long-term corrosion of the internal waste package components may proceed in a nonlinear fashion, where corrosion of fresh surfaces would be rapid and then decrease as corrosion product buildup limits corrosion by impeding oxygen and water diffusion. At some future time, if a mechanism to expose fresh metal surfaces were in action, such as a “sloughing off” mechanism, corrosion could again proceed unimpeded until a new layer of corrosion products formed. Under static conditions, with only gravity acting, the frequency of “sloughing off” could be envisioned to be partially a function of the orientation of a particular component (i.e., angle to vertical and the mass of corrosion product present on a surface). In a dynamic system, an outside force, such as a seismic event, acting on the waste package could also expose fresh metal surfaces. In essence, this model combines two events, sloughing off under gravity and sloughing off under an outside force; the timing of neither is predictable. It is a conservatism that the surface is continually reacting until all of the steels are fully corroded (Section 5.2). However, the range of corrosion rates examined account for this slowing down of corrosion.

The corrosion rates considered for this report are aqueous corrosion rates. Atmospheric rates can be higher or lower than aqueous rates (DTNs: MO0407SPAPCEML.005 [DIRS 172097] and MO0409SPAACRWP.000 [DIRS 172059]). The use of aqueous rates for this model is justified because the intent of this model is to examine the chemistry generated by water interacting inside a waste package. Additionally, sensitivity studies were performed (Section 6.6.10) to examine the effect of increasing and decreasing the corrosion rates.

Code Implementation—The EQ6 code simulates a titration in which reactants are added to the system according to their kinetic rate and surface area exposed. For the vapor-influx case, the water is simply entered as one of the reactants with a rate corresponding to the maximum diffusion rate of vapor through openings in a breached waste package. For the liquid-influx case, the water is entered as a “displacer” reactant type that takes advantage of the solid center flow through capability of EQ6, at a rate corresponding to seepage water entering a waste package.

The calculation that determines the vapor influx rate into a waste package (BSC 2004 [DIRS 169868], Table 6.6-1; Appendix H) is based on the assumption that the incoming vapor is immediately consumed by the corrosion processes, which maintains a 0-percent relative humidity in the waste package and is, thus, the driving force for more vapor to enter the waste package. This conceptualization is intended to maximize the amount of vapor that can diffuse into a waste package for the purposes of determining a bounding vapor-influx rate for a given set of temperature and relative humidity conditions. For the modeling case in EQ6 to actually mimic the 0-percent relative humidity conditions, the incoming vapor influx rate would have to exactly equal the rate at which the water is consumed, which would be based on the corrosion rates of all of the materials in the waste package. Different materials have different corrosion rates, so as some of the materials are depleted, the overall water consumption rate will decrease. However, the vapor-influx cases modeled in this report used constant vapor-influx rates, allowing for the potential of water to accumulate if the vapor-influx rate exceeds the rate at which the water is consumed. In addition, EQ6 works on the basis of one liter of water, which already exceeds the 0-percent relative humidity basis for the vapor-influx calculation. To assess the impacts of this excess water, a sensitivity study was conducted in Section 6.6.1, which shows that the excess water has minimal impact on pH, but directly affects the ionic strength. Therefore, additional discussion about the treatment of this issue is provided in ionic strength abstraction, in Section 6.10.2.

The physical system modeled in the batch reactor model differs slightly from the implementation in TSPA-LA. For the vapor-influx case, the rate at which the water enters is sometimes greater than the rate at which the water is consumed in corrosion reactions, depending on which reactants are still degrading in the system. For example, the Carbon Steel Type A516 degrades quickly and consumes water in the process, so during its lifespan, no water accumulates in the system. But once it has fully degraded, water can begin to accumulate in the system. TSPA-LA applies the chemistry of the vapor-influx case to a continuous water film that coats the waste package internals and transports radionuclides and colloids by diffusion across that film, so no water is accumulated. This difference is studied in a sensitivity analysis that looks at the effects of accumulating water in the system in Section 6.6.1.

If the waste package breaches, the quantity of water required by the EQ6 model (1 liter) is instantly present. In actuality, after the waste package breaches, water would enter the waste package and take a certain amount of time to accumulate. This amount of time for accumulation

is difficult to quantify, and would only result in a delay in radionuclide transport, so for the purposes of this model, the water is instantly available. The impact of this modeling simplification is explored in a sensitivity analysis in Section 6.6.1 and discussed in Section 6.10.2.

Applicability of the B-dot Equation—The thermodynamic database used in this report, SN0410T0510404.002 [DIRS 172712], is for dilute solutions and is used with the B-dot equation within the EQ3/6 software. The B-dot equation is defined and carefully examined in *Dissolved Concentration Limits of Radioactive Elements* (BSC 2005 [DIRS 173873], Section 6.3.3.4). For the purposes of the in-package chemistry model, EQ3/6 results generated using the B-dot activity coefficient equation for solutions with ionic strength greater than 1 molal and up to 4 molal are sufficiently accurate for the intent of this calculation (mainly to estimate pH). Experimental data (in sulfate, nitrate, and chloride solutions) show that EQ3/6 results using the B-dot activity coefficient equation can be used qualitatively up to an ionic strength of about 4 molal to indicate the general nature of the reactions that would actually occur (CRWMS M&O 1997 [DIRS 100222], Appendix D). *Dissolved Concentration Limits of Radioactive Elements* (BSC 2005 [DIRS 173873]) adds an uncertainty term to the solubilities if the ionic strength of the solution is between 1 and 3 molal, and does not provide solubilities if the ionic strength is greater than 3 molal, because of the applicability of the B-dot equation. The in-package chemistry model extends the range of applicability of the B-dot equation because the purpose of this model is to predict bulk chemistry properties such as pH and ionic strength, as opposed to *Dissolved Concentration Limits of Radioactive Elements* (BSC 2005 [DIRS 173873]), which predicts more precise solubility values.

6.3.1.2 Batch Reactor Base-Case Runs

For the vapor-influx case, several runs include only a single reactant, in addition to the complete runs that contain all of the components. The incoming water in the vapor-influx case is pure water, simulating water vapor that enters a waste package and condenses. These runs examine the chemistry and the contribution of each of the reactants and reactant combinations to the in-package chemistry.

For the liquid-influx case, a series of model runs was conducted in which key inputs were varied across their expected input range to generate a broad range of model response. The key parameters include water flux, fuel exposure (clad failure), initial water composition, temperature, fO_2 , fCO_2 , degradation rates of reactants, amount of water in the system, and the major iron mineral that forms. Many of these input parameters are also input variables in the total system performance assessment to the in-package chemistry model. Other model inputs were varied in sensitivity analyses (Section 6.6) to expand the applicability of the model abstraction for TSPA-LA implementation and to quantify model uncertainty.

Table 6-2 shows the input filenames and the varied associated parameters. Each case uses the base-case values for the other inputs: a flux of 1 L/yr, median rates for all of the alloys and fuels, J-13 well water, 25°C, $\log fCO_2 = -3$, and $\log fO_2 = -0.7$, 100% cladding exposure (for CSNF), and the most likely fracture factor for HLWG (for CDSP).

Table 6-2. Batch Reactor-Case Nomenclature

| CSNF | | CDSP | |
|----------|---|----------|---|
| Filename | Values Used | Filename | Values Used |
| 316_Max | Maximum 316 rate | 304L_Max | Maximum 316 rate |
| 316_Min | Minimum 316 rate | 304L_Min | Minimum 316 rate |
| 50C | 50°C | 316_Max | Maximum 316 rate |
| 90C | 90°C | 316_Min | Minimum 316 rate |
| A516_Min | Minimum A516 rate | 50C | 50°C |
| AI_Max | Maximum AI-6061 rate | 90C | 90°C |
| AI_Min | Minimum AI-6061 rate | A516_Min | Minimum 316 rate |
| BaseCase | Base Case (1 significant figure) | AI_Max | Maximum AI-1100 rate |
| Bathtub | Bathtub quantity of water | AI_Min | Minimum AI-1100 rate |
| Ca_Pore | Ca-porewater | BaseCase | Base case (1 significant figure) |
| Clad_1 | 1% Clad exposure | Bathtub | Bathtub quantity of water |
| Clad_10 | 10% Clad exposure | Ca_pore | Ca-porewater |
| CSNF_Max | Maximum 316 rate | fCO2_1_5 | Log $f_{CO_2} = -1.5$ |
| CSNF_Min | Minimum 316 rate | fCO2_5_0 | Log $f_{CO_2} = -5.0$ |
| fCO2_1_5 | Log $f_{CO_2} = -1.5$ | ff_Max | Maximum fracture factor |
| fCO2_5_0 | Log $f_{CO_2} = -5.0$ | fO2_-17 | Log $f_{O_2} = -17$ |
| fO2_-17 | Log $f_{O_2} = -17$ | fO2_-69 | Log $f_{O_2} = -69$ |
| fO2_-69 | Log $f_{O_2} = -69$ | fO2_Eh | Log $f_{O_2} = -8.7349$ (Adjusted Eh) |
| fO2_Eh | Log $f_{O_2} = -8.7349$ (Adjusted Eh) | Goethite | Hematite is suppressed |
| Goethite | Hematite is suppressed | HLWG_Max | Maximum HLWG rate |
| Na_Pore | Na-porewater | LaBS_GI | LaBS Glass composition |
| NiGd_Max | Maximum NiGd rate | LaBS_Max | LaBS Glass composition with max HLWG rate. |
| NiGd_Min | Minimum NiGd rate | Na_pore | Na-porewater |
| pw_10006 | Porewater 10006 | pw_10006 | Porewater 10006 |
| pw_20013 | Porewater 20013 | pw_20013 | Porewater 20013 |
| pw_51 | Porewater 51 | pw_51 | Porewater 51 |
| SigFig | Base case with 3 significant figures | SigFig | Base case with 3 significant figures |

NOTE: The values used for these cases can be found in Sections 6.3.1.3 and 6.6.

6.3.1.3 Batch Reactor Model Inputs

The inputs to the EQ3/6 input files are in terms of moles, centimeters, and seconds. The amounts and compositions of reactants are converted to moles, the surface areas are converted to cm^2 , and the degradation and flow rates are converted to moles per second. During the checking process, some minor inconsistencies were identified in the input files, and these inconsistencies are documented in Table A-2.

6.3.1.3.1 Water Compositions and Influx Rates

The water compositions listed in Tables 4-2 and 4-3 were used as input into EQ3NR. The output from EQ3NR, the “pickup” (*.3p) file, is used as an integral part of each EQ6 input file. EQ3NR recasts the reported concentrations from Table 4-2 into molal units that are the working units of EQ6. The modeling was conducted at the temperatures presented in Table 6-3.

For the J-13 well water composition, the measured pH of 7.41 from Table 4-2 was changed to 7.0 based on work reported by Harrar et al. (1990 ([DIRS 100814], p. 4-9) that contains measured values of pH: 6.9 to 7.1. The HCO_3^- was used to charge balance the solution in the EQ3 input file.

Table 6-3 provides the EQ3NR output molal concentrations of the elements used in the EQ6 input files. For code stability, each of the elements contained in the composition of any of the reactants must also be modeled as present in the fluid composition; therefore, certain elements are present in concentrations of 1.00E-16 molal. The pH of the J-13 well water is set at approximately the mean field measurement of 7 (Harrar et al. 1990 [DIRS 100814], p. 4-9). Many of the pH values included in that report are lab measurements typically higher than field measurements, likely due to loss of dissolved CO_2 (Harrar et al. 1990 [DIRS 100814], p. 4-9).

The diffusion flux of water vapor into a breached waste package used for the base case is a range from 0.1 to 10 liters per year per waste package; this range is based on the potential range of rates depending on the temperature and RH in the drift. The diffusion flux of water vapor into a waste package was calculated (BSC 2004 [DIRS 169868], Table 6.6-1) to determine ingress of water vapor through stress corrosion cracks ranged from 22 moles to 239 moles (which corresponds to 0.4 liters to 4.3 liters) per year per waste package. An alternative conceptual model is developed in Section 6.4.3 to examine the relationship between the vapor influx rate and the temperature and RH of the drift, and the possibility of implementing this relationship in TSPA-LA. This alternative conceptual model also provides a basis for the range of vapor influx rates. The results of this alternative conceptual model build confidence in the range of rates selected for the vapor-influx case.

The range of seepage rates into a breached waste package used for the base case in this report is 0.001 L/yr to 1,000 L/yr. These rates are based on the total potential seepage and drift wall condensation presented in Table 4-4, but only a fraction of these will actually enter a breached waste package, a fraction that is varied inside the TSPA-LA.

Table 6-3. EQ6-Input Fluid Compositions

| Constituent (molal) ^a | Ca-Porewater | Na-Porewater | J-13 Well Water ^b | Pure Water ^c | 51 ^d | 10006 ^d | 20013 ^d |
|----------------------------------|--------------|--------------|------------------------------|-------------------------|-----------------|--------------------|--------------------|
| Temperature | 25°C | 25°C | 25°C ^e | 25°C | 91.8°C | 56.2°C | 40.2°C |
| Ionic Strength | 0.011 | 0.012 | 0.0035 | N/A | 5.00E-03 | 1.00E-02 | 1.00E-02 |
| pH | 7.6 | 7.4 | 7.0 | 5.4 | 8.14 | 7.78 | 7.94 |
| O | 5.55E+01 | 5.55E+01 | 5.55E+01 | 5.55E+01 | 5.55E+01 | 5.55E+01 | 5.55E+01 |
| Al | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 6.09E-08 | 3.73E-09 | 1.50E-09 |
| B | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 |
| Ba | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 |

Table 6-3. EQ6-Input Fluid Compositions (Continued)

| Constituent (molal) ^a | Ca-Porewater | Na-Porewater | J-13 Well Water ^b | Pure Water ^c | 51 ^d | 10006 ^d | 20013 ^d |
|----------------------------------|--------------|--------------|------------------------------|-------------------------|-----------------|--------------------|--------------------|
| Ca | 2.35E-03 | 2.02E-03 | 3.24E-04 | 1.00E-16 | 7.24E-05 | 4.40E-04 | 5.73E-04 |
| Cl | 5.92E-04 | 6.77E-04 | 2.01E-04 | 1.00E-16 | 7.34E-04 | 7.12E-04 | 5.61E-04 |
| Co | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 |
| Cr | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 |
| Cs | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 |
| Cu | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 |
| F | 1.79E-04 | 3.16E-04 | 1.15E-04 | 1.00E-16 | 9.77E-04 | 7.81E-04 | 6.43E-04 |
| Fe | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 |
| Gd | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 |
| H | 1.11E+02 | 1.11E+02 | 1.11E+02 | 1.11E+02 | 1.11E+02 | 1.11E+02 | 1.11E+02 |
| C | 6.76E-03 | 8.62E-03 | 2.49E-03 | 3.79E-05 | 2.04E-03 | 6.58E-03 | 6.92E-03 |
| P | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 |
| I | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 |
| K | 1.94E-04 | 1.56E-04 | 1.29E-04 | 1.00E-16 | 5.02E-04 | 4.55E-04 | 2.76E-04 |
| Li | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 |
| Mg | 7.45E-04 | 1.36E-04 | 8.27E-05 | 1.00E-16 | 2.54E-07 | 5.19E-05 | 8.51E-05 |
| Mn | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 |
| Mo | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 |
| N | 4.19E-05 | 6.61E-06 | 1.42E-04 | 1.00E-16 | 3.10E-04 | 6.58E-05 | 3.97E-05 |
| Na | 1.70E-03 | 5.22E-03 | 1.99E-03 | 1.00E-16 | 4.29E-03 | 7.48E-03 | 7.31E-03 |
| Ni | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 |
| Np | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 |
| Pb | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 |
| Pu | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 |
| Ru | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 |
| S | 3.75E-04 | 3.23E-04 | 1.92E-04 | 1.00E-16 | 1.18E-04 | 3.94E-04 | 3.55E-04 |
| Si | 6.99E-04 | 6.99E-04 | 1.02E-03 | 1.00E-16 | 4.15E-03 | 2.43E-03 | 1.79E-03 |
| Tc | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 |
| Ti | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 |
| U | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 |
| Zn | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 |
| Zr | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 | 1.00E-16 |

Sources: ^a EQ3NR pickup file "Ca_trc.3p", "Na_trc.3p", "J13_trc.3p", "pure_trc.3p" (Appendix F).

^b C levels calculated from charge balance are in agreement with measured alkalinity.

^c EQ3NR pickup file (Appendix F). Pure water in contact with atmospheric carbon dioxide is mildly acidic.

^d DTN: MO0303MWDSCMAB.000 [DIRS 162551], 51.3p: Cross-drift water CS1000/7.3 thc6_w5_r.xls, 10006.3p: Cross-drift water CS1000/7.3 thc6_w5_r.xls, and 20013.3p: Cross-drift water SD-9/990.4 thc6_w6_r.xls (TF4 directory).

^e J-13 water temperature for the is 31°C in the EQ3 input file, but the EQ6 run is conducted at 25°C.

The representativeness of these incoming waters is described in more detail in Section 4.1.2 and the effects of evaporation are described in Section 6.3.1.1.

6.3.1.3.2 Metal Alloy Composition and Corrosion Rates

The in-package chemistry model represents the metal alloys as “special reactants,” in the EQ6 input files, with the compositions provided in Table 6-4 (converted to units appropriate for EQ6 in Appendix F, *CSNF.xls* and *CDSP.xls*). The steel corrosion rates given in Table 6-5 (from Table 4-8, converted to units appropriate to EQ6) represent values for the temperature range from 25°C to 90°C. The values are supported by the data in DTN: MO0409SPAACRWP.000 [DIRS 172059], which cover a range of temperatures and corroding water compositions.

Table 6-4. Metal Alloy Compositions

| Element | Carbon Steel Type A516 (moles) | Neutronit (moles) | Aluminum Alloy 6061 (moles) | Stainless Steel Type 316 (moles) | Aluminum Alloy 1100 (moles) | Stainless Steel Type 304L (moles) | NiGd Alloy (moles) |
|---------|--------------------------------|-------------------|-----------------------------|----------------------------------|-----------------------------|-----------------------------------|--------------------|
| C | 2.33E-02 | 3.33E-03 | | 6.66E-03 | | 2.50E-03 | 8.33E-04 |
| Mn | 1.87E-02 | | 2.73E-03 | 3.64E-02 | 9.10E-04 | 3.64E-02 | 9.10E-03 |
| P | 1.13E-03 | | | 1.45E-03 | | 1.45E-03 | 1.61E-04 |
| S | 1.09E-03 | | | 9.36E-04 | | 9.36E-04 | 1.56E-04 |
| Si | 1.03E-02 | | 2.14E-02 | 2.67E-02 | 1.60E-02 | 2.67E-02 | 2.85E-03 |
| Cr | | 3.56E-01 | 3.75E-03 | 3.27E-01 | | 3.65E-01 | 3.04E-01 |
| Ni | | 2.21E-01 | | 2.04E-01 | | 1.70E-01 | 1.09E+00 |
| Co | | 3.39E-03 | | | | | 3.39E-02 |
| Mo | | 2.29E-02 | | 2.61E-02 | | | 1.52E-01 |
| N | | | | 7.14E-03 | | 7.14E-03 | 7.14E-04 |
| Fe | 1.76E+00 | 1.16E+00 | 1.25E-02 | 1.17E+00 | 8.95E-03 | 1.22E+00 | 1.79E-02 |
| B | | 1.15E-01 | | | | | |
| Zn | | | 3.82E-03 | | 1.53E-03 | | |
| Cu | | | 4.33E-03 | | 1.97E-03 | | |
| Mg | | | 4.11E-02 | | | | |
| Ti | | | 3.13E-03 | | | | |
| Al | | | 3.58E+00 | | 3.66E+00 | | |
| Gd | | | | | | | 1.27E-02 |
| O | | | | | | | 3.13E-04 |

Source: Appendix F, *CSNF.xls* and *CDSP.xls*, “Materials” worksheet.

NOTE: Elemental mole % of the reactant is calculated based on 100 grams of reactant.

Table 6-5. Metal Alloy Corrosion Rates

| Corrosion Rate | Units | Carbon Steel Type A516 | Neutronit | Aluminum Alloy 6061 | Stainless Steel Type 316 | Aluminum Alloy 1100 | Stainless Steel Type 304L | NiGd Alloy |
|----------------|-------------------------|------------------------|-----------|---------------------|--------------------------|---------------------|---------------------------|------------|
| Minimum | μm/yr | 3.69 | 0.001 | 0.4 | 0.0014 | 0.4 | 0.001 | 0.0528 |
| | moles/s-cm ² | 9E-13 | 2.46E-16 | 3E-14 | 4E-16 | 3E-14 | 3E-16 | 1E-14 |
| Maximum | μm/yr | 130.7 | 29.2 | 110.9 | 14.8 | 110.9 | 39.1 | 0.0774 |
| | moles/s-cm ² | 4E-11 | 7.18E-12 | 9E-12 | 4E-12 | 9E-12 | 1E-11 | 2E-14 |
| Mean | μm/yr | 102.7 | 0.1 | 12.95 | 1.94 | 12.95 | 11.44 | 0.0528 |
| | moles/s-cm ² | 3E-11 | 2.54E-14 | 1E-12 | 5E-13 | 1E-12 | 3E-12 | 1E-14 |

Source: Appendix F, *CSNF.xls* and *CDSP.xls*, worksheet "Rates."NOTE: ^aNeutronit, a borated stainless steel, was replaced by NiGd alloy in the latest CSNF design, and is only presented here for comparison.

The corrosion rates used for the base case are those within the range provided in DTN: MO0409SPAACRWP.000 [DIRS 172059]. The results show that different rates do not affect the range of the model output (maximum or minimum pH or ionic strength). Changes in these rates only affect the timing of the pH or ionic strength responses that occur when a reactant is depleted. Since the output is not time-dependent, the timing of the shifts in pH or ionic strength does not impact the results. The rates in Table 6-5 were compared to the atmospheric rates given in DTN: MO0407SPAPCEML.005 [DIRS 172097], spreadsheet *atmospheric.xls*. For the Carbon Steel Type A516, some of the atmospheric rates were higher (1,057.18 μm/yr) than the 130.7 μm/yr value given in Table 6-5. For Aluminum Alloy Types 1100 and 6061, the atmospheric rates are lower than the minimum rate given in Table 6-5. For Stainless Steel Type 316, some of the atmospheric rates were higher (53 μm/yr) than the 14.8 μm/yr value provided in Table 6-5.

The amount of metal alloy that EQ6 "adds" to the reaction during a run (moles) is the product of the coefficients (moles/s-cm²) in Table 6-5, the duration of the EQ6 time step(s), and the surface area (cm²) of the reactant. Therefore, the amount of reactant added per time step = moles/s-cm² × s × cm². The duration of the EQ6 time step varies depending on how quickly the given reactions reach equilibrium.

6.3.1.3.3 SNF and HLWG Composition and Reaction Rates

For EQ6 runs, the CSNF was specified to have 3.5 wt % ²³⁵U enrichment, 40 GWd/MTU fuel burnup, and 10,000 years of decay time BSC 2003 [DIRS 169110]. The number of elements in the fuel composition was reduced to allow EQ6 to run more efficiently. EQ6 run time increases rapidly and nonlinearly as the number of elements is increased. Therefore, the moles of Ru in Table 6-6 include the moles of Ru, Rh, Pd, and Ag, and the moles of Gd in Table 6-6 include the moles of Gd, Y, Pr, Nd, Pm, Sm, Eu, and Ho in the CSNF composition (BSC 2003 [DIRS 169110]). The combined elements are expected to behave in a similar manner due to their position in the periodic table. The elements not included in the fuel composition used in the model are Th, Pa, Am, Cm, Li, Be, As, Kr, Nb, and Xe. The concentration of the elements excluded from the CSNF composition are estimated to be below levels which have influence over the calculated in-package chemistry; thus, their exclusion does not impact the results of the in-package chemistry model.

Table 6-6 summarizes the composition of the CSNF and N-Reactor fuels, respectively. The CSNF is an oxide fuel while the N-Reactor fuel is a uranium metal-type fuel.

Table 6-6. CSNF and N-Reactor Compositions

| Element | CSNF (Moles/100g) | N Reactor (Moles/100g) |
|---------|----------------------|---------------------------|
| U | 0.3617 | 0.420 |
| Np | 0.0009 | N/A |
| Pu | 0.0027 | N/A |
| Zr | 0.0005 | N/A |
| Mo | 0.0009 | N/A |
| Tc | 0.0008 | N/A |
| Ru | 0.0020 | N/A |
| Cs | 0.0013 | N/A |
| Ba | 0.0010 | N/A |
| Gd | 0.0035 | N/A |
| O | 0.7385 | N/A |

Source: Appendix F, *CSNF Fuel.xls*, and for N Reactor, moles of U = 100/238 g/mol.

NOTE: Elemental mole % of the reactant are calculated based on 100 grams of reactant.

For input into EQ6, the simplified HLWG formula is used based on 100 g from Table 4-5. As described in *EQ6 Calculations for Chemical Degradation of N Reactor (U-metal) Spent Nuclear Fuel Waste Packages* (CRWMS M&O 2001 [DIRS 153263]), to arrive at this simplified composition, several elements in the glass composition from *Defense Waste Processing Facility Wasteform and Canister Description* (Baxter 1988 [DIRS 106164], Table 10), were:

- Decayed to other elements (^{238}Pu to ^{234}U)
- Combined with other elements (Li added to Na) or
- Excluded to reduce the number of elements for ease in running EQ6.

The elements not included in the model of the glass composition were Ag, Cs, Cu, Mn, Cl, Ni, Pb, Th, and Ti. These elements occur in such low quantities (< 0.2 wt%) that their omission has no impact on the modeling results. These minor changes were made to the basic composition to increase the efficiency of the calculations, to decrease the EQ6 run time, and to allow the use of a pH dependent rate law. The simplified glass composition was included in the EQ6 database as a mineral named 'SRL_Bulk' (Stoichiometric fractions less than 0.0001 moles/100 g are not read by EQ3/6).

Table 6-7 provides the CSNF reaction rate constants at various temperatures for the equations provided in *CSNF Waste Form Degradation: Summary Abstraction* (BSC 2004 [DIRS 169987]). The two equations in Section 4.1.3 for the dissolution of CSNF were combined into a single closed-form expression for use in EQ6. The CSNF reaction rate is a transition state theory rate law that is dependent on temperature, pH, and bicarbonate concentration. The bicarbonate concentration is used rather than total carbonate because the CSNF rate law is insensitive to carbonate concentration in the output pH range (3 to 8) of the model. Figure A-1 of Appendix A

shows the CSNF total degradation constant as a function of pH and derived in terms of total carbonate and HCO_3^- . This figure shows that from pH 2 to about pH 10, the total degradation constant is not sensitive to carbonate, but above pH 10 the degradation constant is sensitive to carbonate. The geometric surface area (the Log(A) term) was varied from the minimum to the maximum values provided in *CSNF Waste Form Degradation: Summary Abstraction* (BSC 2004 [DIRS 169987]). The first rate mechanism (described with k_1) in Table 6-8 is dominant at pH values less than or equal to 6.8, while the second rate mechanism (described with k_2) is dominant at pH values greater 6.8. The rate constants in Table 6-7 were calculated using an oxygen partial pressure of 0.2 atmospheres.

Table 6-7. CSNF Transition State Theory Rate Law and Coefficients

| Total Degradation Rate Constant = $k_1[\text{H}^+]^{0.34} + k_2[\text{HCO}_3^-]^{0.10}$ (mol/cm ² ·s) | | | | | |
|--|----------------|---------|---------------------------------|---------------------------------|---------|
| Temperature (°C) | % Clad Failure | Log(A) | Rate Constant (k ₁) | Rate Constant (k ₂) | |
| 25 | 1 | Minimum | 2.9E-10 | 3.7E-12 | |
| | | Apex | 1.1E-09 | 1.5E-11 | |
| | | Maximum | 2.3E-08 | 2.9E-10 | |
| | 10 | Minimum | 2.9E-09 | 3.7E-11 | |
| | | Apex | 1.1E-08 | 1.5E-10 | |
| | | Maximum | 2.3E-07 | 2.9E-09 | |
| | 100 | Minimum | 1.4E-08 | 1.8E-10 | |
| | | Apex | 5.7E-08 | 7.3E-10 | |
| | | Maximum | 1.1E-06 | 1.5E-08 | |
| | 40 | 100 | Apex | 8.6E-08 | 1.1E-09 |
| | 50 | 100 | Apex | 1.1E-07 | 1.4E-09 |
| | 90 | 100 | Apex | 2.6E-07 | 3.3E-09 |

Source: Appendix F, CSNF.xls.

The glass dissolution rate law is the pH-dependent, transition state theory rate law provided in *Defense HLW Glass Degradation Model* (BSC 2004 [DIRS 169988], Section 8.1, Equations 40a and 40b). The first rate mechanism (described with k_1) in Table 6-8 is dominant at pH values greater than or equal to 7.1, while the second rate mechanism (described with k_2) is dominant at pH values below 7.1. The rate constants for various temperatures are given in Table 6-8.

Table 6-8. HLWG Transition State Theory Rate Law and N-Reactor Rate Constants

| HLWG | | |
|---|---------------------------------|---------------------------------|
| Total Degradation Rate Constant = $k_1[\text{H}^+]^{-0.49} + k_2[\text{H}^+]^{0.49}$ (mol/cm ² ·s) | | |
| Temperature (°C) | Rate Constant (k ₁) | Rate Constant (k ₂) |
| 25.0 | 2.7E-22 | 3.6E-13 |
| 40.2 | 1.0E-21 | 6.6E-13 |
| 50.0 | 2.3E-21 | 9.5E-13 |
| 56.2 | 3.7E-21 | 1.2E-12 |
| 90.0 | 3.9E-20 | 3.4E-12 |
| 91.8 | 4.3E-20 | 3.6E-12 |

Table 6-8. HLWG Transition State Theory Rate Law and N-Reactor Rate Constants (Continued)

| N Reactor ^a | |
|------------------------|---|
| Temperature (°C) | Total Degradation Constant K (mol/cm ² ·s) |
| 25.0 | 1.6E-10 |
| 50.0 | 1.3E-09 |
| 90.0 | 2.0E-08 |

Source: Appendix F, *CDSP.xls*, worksheet: "CDSP Rates."

NOTE: ^a The 25°C rate for CDSP was used for all EQ6 runs (the high-temperature rates were not used in any runs).

6.3.1.3.4 Waste Package Components Total Surface Areas and Moles

Table 6-9 presents the total surface areas and moles of reactants for the waste packages as calculated in *CSNF.xls* and *CDSP.xls*. The dimensions and masses of components from the IED (BSC 2005 [DIRS 173501]) were used to calculate surface areas and amounts that were then converted into cm² and moles for use by the EQ6 code. The methodology for these calculations is described in the associated spreadsheets (Appendix F, *CSNF.xls*, and *CDSP.xls*).

For the HLWG, the exposure factors given in Section 4.1.3.3 are: $f_{\text{exposure}} = 4$ (most likely) and 17 (maximum).

Table 6-9. Total Surface Areas and Moles for the CSNF and CDSP Waste Packages

| CSNF | | | CDSP | | |
|--------------------------|-------------------------|---|---|-------------------------|---|
| Material | Moles per Waste Package | Surface Area (cm ²) per Waste Package | Material | Moles per Waste Package | Surface Area (cm ²) per Waste Package |
| Carbon Steel Type A-516 | 55,996 | 2,390,930 | Carbon Steel Type A-516 | 16,510 | 331,210 |
| NiGd | 21,216 | 436,275 | Stainless Steel Type 304L | 57,532 | 631,626 |
| Aluminum Type 6061 | 4,384 | 255,701 | Aluminum Type 1100 | 1,245 | 92,038 |
| Stainless Steel Type 316 | 107,199 | 286,237 | Stainless Steel Type 316 | 116,065 | 310,960 |
| CSNF | 110,453 | 5,034,310 | N-Reactor fuel | 136,818 | 9,577,260 |
| | | | HLWG (most probable f_{exposure}) | 68,821 | 680,000 |

Source: Appendix F, *CSNF.xls*, and *CDSP.xls*.

6.3.1.3.4.1 Vapor-Influx Case: Model Inputs

The vapor-influx cases include single component runs that examine the interactions of water with a single waste package component and the base case, which includes all of the waste package components. The vapor-influx cases use a low ratio of water to reactants to simulate vapor condensing over a large surface area. For simplicity and comparison purposes, each of the single component runs considers the liter of water to be in contact with 4,000 cm² of available surface area (4,000 = 1 liter of water, spread over each reactant 0.25 cm thick). The single

component and the water flow rate are scaled to the surface area of 4,000 cm². The scaling calculations for the vapor-influx cases are provided in Appendix F, *CSNF.xls* and *CDSP.xls*.

6.3.1.3.4.2 Liquid-Influx Case: Model Inputs

The total moles and reactants for the liquid-influx models are scaled to one liter of water. The ratio of water to reactants for the liquid-influx cases is approximately half of what the ratio would be in a bathtub model. This ratio was selected to generate a model more representative than the bathtub model and more closely aligned to the conceptual implementation of the model within TSPA-LA. TSPA-LA applies the output of this model to a continuous film over the waste package interior available to transport radionuclides. The reactants are scaled to one liter of water and the total surface area. Different exposures of fuel result in different total surface areas, so each fuel exposure case is scaled differently. The water flow rates are also scaled to the total surface area. The scaling calculations for the liquid-influx cases are provided in Appendix F, *CSNF.xls* and *CDSP.xls*.

6.3.1.3.5 Mineral Controls

In the in-package chemistry model, the minerals that precipitate from solution during an EQ6 run are determined by the code in accordance with the thermodynamics of the chemical system. Based on the modeled conditions, only certain minerals will precipitate from solution. For EQ6 to choose “realistic” precipitation phases, the analyst must assume that the database includes data for all possible relevant phases. There is no test of completeness in the construction or qualification of the database. In using the EQ6 software the analyst must decide, on the basis of prior knowledge related to the phenomena, that mineral phases will not be allowed to form in a simulation; this is called mineral “suppression.” Table 6-10 lists the mineral phases suppressed in the EQ6 runs. By deciding which minerals to suppress, the analyst is implicitly deciding which minerals will form. Trial runs are often performed to allow the analyst to see first hand what minerals are forming and what minerals should be suppressed. By suppressing a phase, a decision is made based on knowledge of a phase’s mode of occurrence and the relative kinetics of formation. Table 6-11 provides a list of the major minerals that were not suppressed and were observed to precipitate in the EQ6 runs for the in-package chemistry model. Phases in abundance of greater than 10⁻⁴ moles over the 20,000-year model period are included in the list.

The corrosion product assemblage is likely to be made up of some mixture of hematite, goethite, and ferrihydrite (Section 6.3). From a modeling point of view, hematite was the primary ferric iron phase allowed to form in the batch reactor simulations because it is the most thermodynamically stable mineral. Because EQ6 is a thermodynamic equilibrium code, the use of hematite over goethite leads to the lowest dissolved iron concentrations. However, goethite also requires consideration because it is kinetically favored, so goethite formation was also allowed in some cases and its impact on the in-package chemistry is examined in Section 6.6.1.1. Ferrihydrite is not modeled in the batch reactor simulations because it has limited thermodynamic data. The fact that the pH predicted by the goethite and hematite simulations are similar to each other indicates ferrihydrite use would also be similar.

Table 6-10. Minerals Suppressed in In-Package Chemistry Model EQ6 Runs

| Mineral | Chemical Formula | Justification |
|--------------------------|--|---|
| Quartz | SiO ₂ | Quartz is formed at higher temperatures (around 573°C), but its extremely simple elemental makeup makes it resistant to corrosion, allowing it to exist widespread as a detrital fragment in many sedimentary rocks, which is why it has been noted in so many sedimentary deposits. However, other forms of SiO ₂ can form as primary sedimentary minerals at low temperatures (such as chalcedony) (Klein and Hurlbut 1985 [DIRS 105907], pp. 441 to 442). Therefore, the mineral quartz has been suppressed in the EQ6 runs, allowing the lower temperature varieties of SiO ₂ to form. |
| Goethite | α-FeOOH | Goethite (α-FeOOH) and hematite (α-Fe ₂ O ₃) are the two most thermodynamically stable Fe minerals under oxidizing conditions (Schwertmann and Taylor 1995 [DIRS 105959]). Goethite was suppressed in the EQ6 runs, however, suppressing goethite had no effect on the results of the model as hematite is more-thermodynamically stable than goethite, and EQ6 would not allow goethite to form unless hematite was suppressed. Sensitivity runs were performed where hematite was suppressed and goethite allowed to form; the results are included in Section 6.6.11 and the conclusion drawn, therein, was that the chemistry of the system was largely independent of which phase was allowed to form, either goethite or hematite. |
| Tridymite | SiO ₂ | Tridymite exists as both α and β types. The low-temperature α-tridymite forms only from preexisting β-tridymite that forms in the temperature range of 870°C to 1,470°C (Roberts et al. 1990 [DIRS 107105], pp. 881 to 882). |
| Muscovite | KAl ₂ (Si ₃ Al)O ₁₀ (OH,F) ₂ | Occurs in high temperature and/or pressure mineral assemblages and as a detrital mineral in sedimentary rocks (Roberts et al. 1990 [DIRS 107105], p. 586). |
| Celadonite | K(Mg,Fe ²⁺)(Fe ³⁺ ,Al)Si ₄ O ₁₀ (OH) ₂ | Celadonite is found in altered volcanic rocks (Roberts et al. 1990 [DIRS 107105], p. 149). Celadonite can form at low temperatures, but this is due to either diagenesis of preexisting material, low-grade metamorphic processes, or concentration of the liquid environment through evaporation (Li et al. 1997 [DIRS 159034]). |
| Annite | KFe ₃ ²⁺ AlSi ₃ O ₁₀ (OH,F) ₂ | Synthesized, natural occurrence uncertain (Roberts et al. 1990 [DIRS 107105], p. 32). |
| Dolomite | CaMg(CO ₃) ₂ | Dolomite is usually derived by secondary mineralization, from the replacement of calcium for magnesium in the calcite crystal structure in Mg-rich waters (Klein and Hurlbut 1985 [DIRS 105907], p. 340). Because it rarely occurs as a primary mineral, dolomite was also suppressed. |
| Andradite | Ca ₃ Fe ₂ (SiO ₄) ₃ | Andradite is a high pressure-temperature mineral found in metamorphic and igneous rocks (Deer et al. 1966 [DIRS 102773], p. 30). |
| Phlogopite | KMg ₃ AlSi ₃ O ₁₀ (OH,F) ₂ | Occurs chiefly in metamorphic limestones and ultrabasic rocks (Roberts et al. 1990 [DIRS 107105], p. 671). |
| Talc | Mg ₃ Si ₄ O ₁₀ (OH) ₂ | Talc is characteristically associated with low-grade metamorphic rock and hydrothermal alteration of ultrabasic rocks (Kerr 1977 [DIRS 161606], p. 450). |
| Ferrite-Ca Ferrite-Mg | CaFe ₂ O ₄ MgFe ₂ O ₄ | Magnesioferrite has been found in sintered magnesite of furnace linings and other refractories (Palache et al. 1944 [DIRS 163604], p. 705); not expected to form at low temperatures. |
| Zircon | ZrSiO ₄ | High pressure-temperature mineral. Found in sedimentary deposits as a detrital mineral (Roberts et al. 1990 [DIRS 107105], p. 975). |
| Soddyite | (UO ₂) ₂ SiO ₄ ·2H ₂ O | Suppression of soddyite had no effect on the results of this model report, since this mineral is fairly soluble under the conditions simulated by the model, and it would not have precipitated in any of the EQ6 runs even if its formation had not been suppressed. This suppression is discussed in more detail in Table A-1. |
| PuO ₂ | PuO ₂ | The measured solubilities of solid Pu(V) oxide/hydroxide scatter within several orders of magnitude because of the difficulties of establishing equilibrium of Pu(V), polymerization and disproportionation reactions and the strong sorption capacities of Pu ⁴⁺ (Runde 1999 [DIRS 144800]). Experimental plutonium solution concentrations during PuO ₂ or PWR SNF degradation have been shown to be between the solubility of PuO ₂ and that of a more soluble phase, Pu(OH) ₄ (or PuO ₂ ·2H ₂ O) (Rai and Ryan 1982 [DIRS 112060]; Wilson and Bruton 1989 [DIRS 137607], Section 3.1 and Table 3). |

Table 6-11. Major Mineral Phases Formed in EQ6 Runs

| Mineral | Chemical Formula | Justification |
|----------------------|---|--|
| Hematite | $\alpha\text{-Fe}_2\text{O}_3$ | Goethite ($\alpha\text{-FeOOH}$) and hematite ($\alpha\text{-Fe}_2\text{O}_3$) are the two most-thermodynamically stable and most-widespread Fe minerals under oxidizing condition. Hematite is usually found in tropical and subtropical regions where higher temperatures and lower water activities aid in its formation. Hematite needs a precursor such as ferrihydrite from which it forms through dehydration and rearrangement (Schwertmann and Taylor 1995 [DIRS 105959]). |
| Pyrolusite | MnO_2 | Secondary mineral forming from alteration of manganite or other manganese-bearing minerals. (Roberts et al. 1990 [DIRS 107105], p. 704). Manganite forms as a low-temperature hydrothermal vein mineral; in circulating meteoric water; or bog, lacustrine, or shallow marine deposits (Roberts et al. 1990 [DIRS 107105], p. 524). Found as a secondary mineral in the oxidation zone at the Morro de Ferro site in Brazil (Waber 1991 [DIRS 126924]). Formed in the system $\text{Mn-O}_2\text{-H}_2\text{O}$ at 25°C and one atmosphere (Bricker 1965 [DIRS 157873], Table 1). |
| Trevorite | NiFe_2O_4 | High pressure-temperature mineral (Roberts et al. 1990 [DIRS 107105], p. 881). Trevorite was allowed to form since nickel-substituted goethite, hematite, and NiFe_2O_4 can be synthesized at low pressure and temperature (Cornell et al. 1992 [DIRS 164025]), and Ni-substituted iron oxides are not in the EQ6 database. |
| Powellite | $\text{Ca}(\text{Mo,W})\text{O}_4$ | Occurs as a secondary mineral in the oxidation zones of ore deposits (Roberts et al. 1990 [DIRS 107105], p. 692). Secondary mineral, often by the alteration of molybdenite, in copper deposits; deposits encountering contact metamorphism (Palache et al. 1951 [DIRS 162280], p. 1,080). |
| Tenorite | CuO | Occurs chiefly in the oxidation zone of copper deposits, often associated with other secondary minerals. Also occurs as a sublimation product deposited on lavas in volcanic regions (Roberts et al. 1990 [DIRS 107105], p. 856; Palache et al. 1944 [DIRS 163604], p. 509). CuO is a component of patina formed in atmospheric and aqueous corrosion of copper alloys, though not specifically given the mineral name, tenorite (Sequeira 2000 [DIRS 162970]). |
| Schoepite | $\text{UO}_3 \cdot 2\text{H}_2\text{O}$ | Alteration product of uraninite (UO_2); associated with bequerelite, curite and other secondary minerals of uranium (Palache et al. 1944 [DIRS 163604], p. 628). The phase Na-boltwoodite was observed to form in some simulations where it was typically below 10^{-4} moles. It is a phase expected to be stable in the corroding waste package environment (BSC 2005 [DIRS 173873], Section 6.7.2). |
| Gibbsite | $\text{Al}(\text{OH})_3$ | Most common $\text{Al}(\text{OH})_3$ polymorph (Hsu 1995 [DIRS 105875]). AIOOH are rarer than hydroxides and are considered the product of weathering (Allen and Hajek 1995 [DIRS 159372]) so the $\text{Al}(\text{OH})_3$ polymorphs are the primary ones expected to form in the waste package. |
| Kaolinite | $\text{Al}_2\text{Si}_2\text{O}_5(\text{OH})_4$ | Most common kaolin, formation at 25°C is usually slow, however it can crystallize easily from the alteration of smectites (Dixon 1995 [DIRS 159374]). |
| Nontronite | $\text{Na}_{0.33}\text{Fe}_2(\text{Si,Al})_4\text{O}_{10}(\text{OH})_2 \cdot n\text{H}_2\text{O}$ | One of the three most common smectite minerals found in soils. Smectites are common in temperate and cold climates, but may form or be preserved in tropical environments where leaching is limited or drainage is restricted (Allen and Hajek 1995 [DIRS 159372]). |
| Gadolinium phosphate | $\text{GdPO}_4 \cdot 10 \text{H}_2\text{O}$ | Gadolinium phosphate hydrates are insoluble in near-neutral pH solutions (Firsching and Brune 1991 [DIRS 106357]). Monazite type minerals, of which GdPO_4 is one, is usually formed under hydrothermal conditions, but studies have indicated the formation of monazites at low temperatures (Spahiu and Bruno 1995 [DIRS 103804]), and the formation of $\text{GdPO}_4 \cdot 10 \text{H}_2\text{O}$ is consistent with the formation of $(\text{UO}_2)_3(\text{PO}_4)_2 \cdot x\text{H}_2\text{O}$. |
| Uranyl phosphates | $(\text{UO}_2)_3(\text{PO}_4)_2 \cdot x\text{H}_2\text{O}$ | NOTE: $x = 4$ or 6 . $(\text{UO}_2)_3(\text{PO}_4)_2 \cdot 4\text{H}_2\text{O}$ is a stable phase under oxidizing conditions, low temperature, and pH values relevant for natural water systems (Sandino 1991 [DIRS 113307], pp. 16 to 17). $(\text{UO}_2)_3(\text{PO}_4)_2 \cdot 6\text{H}_2\text{O}$ was also allowed to form since uranyl phosphates are associated with a wide range of weathered uranium deposits (Finch and Murakami 1999 [DIRS 145442]), but few uranyl phosphates are included in the EQ6 database. |
| Chalcedony | SiO_2 | Chalcedony readily precipitates from aqueous solutions and is frequently found lining or filling cavities in rocks (Klein and Hurlbut 1985 [DIRS 105907], pp. 441 to 442). |
| Baddeleyite | ZrO_2 | High-pressure-temperature mineral, found in sedimentary deposits as a detrital mineral (Roberts et al. 1990 [DIRS 107105], p. 62), also formed in metamict zircon (Deer et al. 1966 [DIRS 102773], p. 15). Baddeleyite was allowed to form in the waste package as hydrolysis of Zr salts leads to precipitation of poorly crystalline oxides at low temperatures (Mines and Fitzpatrick 1995 [DIRS 105911], pp. 1, 189 to 1,190), and soluble Zr may be incorporated in or sorb on clay mineral surfaces (Mines and Fitzpatrick 1995 [DIRS 105911], pp. 1,185 to 1,186). So, it is likely that some of the zirconium released by degradation of waste package components will precipitate or be sorbed. |

6.3.2 Surface Complexation Model

The surface complexation model examines the effects of the contact of the solution predicted by the batch reactor model (Section 6.3.1) with corrosion product surfaces. Waste form and waste package degradation will involve oxidation and hydration of fuel elements and steels. Carbon Steel Type A516 corrosion will be particularly rapid relative to fuel oxidation and will result in large accumulations of ferric (hydr)oxide corrosion products inside the waste package, where (hydr)oxide means any oxide, hydroxide, or oxyhydroxide that has ferric iron in it. Because there is substantially more iron in the waste packages than U – the most abundant radionuclide – it is reasonable to expect ferric (hydr)oxides to be the single most abundant phase in degraded waste packages.

Although the formation of large masses of ferric (hydr)oxide corrosion products is inevitable, it is difficult to predict which one will dominate the mineralogy of the whole. Goethite and hematite are expected to eventually be the most abundant iron oxides in corrosion products (Schwertmann and Cornell 1991 [DIRS 144629], Section 4.2.2). Poorly crystalline solids such as ferrihydrite ($\text{Fe}_5\text{HO}_8 \cdot 4\text{H}_2\text{O}$) and $\text{Fe}(\text{OH})_3$ that form during rapid oxidation of Fe(II) and hydrolysis of Fe(III), will probably be present throughout the period of steel degradation in the waste package (Schwertmann and Cornell 1991 [DIRS 144629], Sections 1.3 and 8.1). These poorly crystalline iron oxides are unstable with respect to hematite and goethite, but their transformation is significantly inhibited or retarded by their adsorption or structural substitution, or both, of silicate, phosphate, and Cr(III), which are common components of the waste package solution. To summarize, the corrosion product assemblage is likely to be made up of some mixture of hematite, goethite, and ferrihydrite. However, the batch reactor model uses hematite in the base case and goethite in a sensitivity study in Section 6.6.11, as described in Section 6.3.1.3.5, but the surface complexation uses ferric (hydr)oxides, as explained in Sections 6.7 and 6.8. Properties of these phases are presented in Table 4-10.

In natural systems, less crystalline ferric (hydr)oxides appear to control the surface chemistry of soils and aquifers by coating the surfaces of other more abundant minerals. The observation that “surface coatings on larger mineral grains in soils and sediments may exhibit site densities per unit mass within the range similar to ferrihydrite even though present as fine-grained crystalline material” (Davis et al. 2002 [DIRS 168855]) is routinely observed for natural systems. Specifically, surface coatings of natural minerals appear to possess the site densities per unit mass of amorphous materials (Davis et al. 1998 [DIRS 154436]; Waite et al. 2000 [DIRS 168856]). Consequently, the surface chemistry of natural sediments is routinely modeled by assuming that the surface area of the sediments is composed entirely of ferrihydrite (Waite et al. 2000 [DIRS 168856]). The present state of knowledge suggests that a reasonable approach to approximating the buffering of pH by ferric (hydr)oxide corrosion products inside of breached waste packages is to estimate quantitatively the properties and effects of the resulting ferric (hydr)oxide surfaces. Thus, similar to those described by Waite et al. (2000 [DIRS 168856]), surface complexation reactions in the waste packages are treated as if they are entirely due to ferrihydrite coatings.

Reported isoelectric points (pH_{IEP}) of ferric (hydr)oxides are listed in Table 4-10. The pH_{IEP} values from *Chemistry of the Solid-Water Interface, Processes at the Mineral-Water and Particle-Water Interface in Natural Systems* (Stumm 1992 [DIRS 141778]) and *Surface*

Complexation Modeling, Hydrous Ferric Oxide (Dzombak and Morel 1990 [DIRS 105483]) are considered more reliable measurements than those tabulated by Langmuir (1997 [DIRS 100051]) because Stumm (1992 [DIRS 141778]) and Dzombak and Morel (1990 [DIRS 105483]) works are devoted solely to the topic of adsorption. These pH_{IEP} values range from 7.3 to 8.7 (Table 4-10). Iron corrosion results in a mixture of amorphous and crystalline iron phases (Schwertmann and Cornell 1991 [DIRS 144629], Sections 4.2.1 and 4.2.2). However, the amorphous iron phases will likely dominate the surface reactions in the waste package due to their higher surface-site concentrations per mass of water. Ferrihydrite, therefore, is a good choice as an iron corrosion product to represent the mixture of iron corrosion products. This choice is consistent with the findings of Schwertmann and Cornell (1991 [DIRS 144629]) and Waite et al. (2000 [DIRS 168856]). Another reasonable choice is goethite because of its relatively high specific surface area and comparable surface site density (Table 4-10).

Based on work by Stumm (1992 [DIRS 141778]) and Dzombak and Morel (1990 [DIRS 105483]), the isoelectric points of ferrihydrite and goethite are between 7 to 8 (Table 4-10), indicating the predominant surfaces in the waste package tends to buffer the pH in the near-neutral range, adsorbing protons as pH values decrease and releasing protons as pH values increase. Considerable departures from the near-neutral pH range (i.e., outside the 5 to 9 pH value range) have the potential of increasing radionuclide solubilities. Therefore, steps must be taken to prevent overstating the potential effects of surface reactions. This concern is addressed in Sections 6.7 and 6.8.

A calculation shows that between 10 and 40 moles of ferric (hydr)oxides (ferrihydrite, goethite, or hematite) will form per kilogram of water in almost all runs due to the degradation of Carbon Steel Type A516 and Stainless Steel Type 316 (Appendix F, *CSNF.xls*, “Moles Corrosion Products” tab). Roughly 30% of the ferric (hydr)oxides appears in the first 100 years due to the degradation of Carbon Steel Type A516. The batch reactor model calculates the formation of ferric (hydr)oxides and accounts for their presence in, for example, the calculation of dissolved Fe levels. Nevertheless, the batch reactor model does not consider the ability of iron(III) mineral surfaces to control in-package pH through the acquisition or loss of protons at low and high pH values, respectively. The latter effect is likely to be large as there are from 0.02 to 0.9 moles of reactive sites per mole of Fe(III) precipitated as goethite or ferrihydrite (Table 4-10, 0.02 rounded up from 0.017). The total amount of reactive sites available for pH buffering is between 0.2 moles (10 moles \times 0.02) and 36 moles (40 moles \times 0.09) per liter of void space. Compare this to maximal estimated acid and base loads of the batch reactor model (respectively, $\sim 10^{-3}$ moles of protons per liter [corresponds to $pH \sim 3$] and 10^{-4} hydroxyls per liter [corresponds to $pH \sim 10$]).

The approach is then to consider the chemical reactivity of the fuel and steel components in isolation and in combination in order to bound the fluid compositions produced by oxidation, as modeled in the batch reactor model (Section 6.3.1) and to then consider the degree to which ferric (hydr)oxide surface will affect their chemistry.

The above discussion addresses the subject matter of “Included” FEP 2.1.09.02.0A (Chemical interaction with corrosion products) and indicates how that subject matter has been included and addressed within TSPA-LA. Corrosion products are generated during the degradation of the metal alloys that make up the waste package components. Therefore, the effect of the corrosion

products on the in-package chemistry is directly handled by the in-package surface complexation model that accounts for the effects of surface complexation of the in-package fluids with the corrosion products.

6.3.2.1 Development of the Surface Complexation Demonstration Database

The surface complexation demonstration database for hydrous ferric oxide (HFO) is developed in this section. This database is used in the surface complexation demonstration in Section 6.8, and is solely intended to build confidence in the surface complexation model discussed in Section 6.3.2, and is not an actual technical product of this model.

The data to be evaluated are the surface complexation reactions and log K values in Table 4-12 and include data by Dzombak and Morel (1990 [DIRS 105483]). *Surface Complexation Modeling, Hydrous Ferric Oxide* (Dzombak and Morel 1990 [DIRS 105483]) is a widely acclaimed treatise on surface complexation modeling for HFO.

A diffuse-layer surface complexation model for hydrous ferric oxide (HFO) is used in this report to demonstrate the adsorption effects predicted by the in-package chemistry model. Dzombak and Morel (1990 [DIRS 105483]) develop and document the model in detail in one of several adsorption models available within PHREEQC. The diffuse layer surface model was selected because it has the largest thermodynamic database for support.

The thermodynamic database used for the surface complexation demonstration is provided in Table 6-12 for ferrihydrite, in addition to the data provided in Table 4-9. The data in Tables 4-9 and 6-12 are internally consistent. These data are used as input to develop a surface complexation database. They are also used to develop a PHREEQC database for the adsorption demonstrations. In addition, several of these reactions are used to explain pH-buffering effects.

Table 6-12. Surface Complexation Reactions

| Reaction | log K | Location |
|--|-------|------------------------------------|
| $\text{HFO_sOH} + \text{Ca}^{2+} = \text{HFO_sOHCa}^{2+}$ | 4.97 | Table 10.1 |
| $\text{HFO_wOH} + \text{Ca}^{2+} = \text{HFO_wOCa}^{+} + \text{H}^{+}$ | -5.85 | Table 10.1 |
| $\text{HFO_sOH} + \text{Cd}^{2+} = \text{HFO_sOCd}^{+} + \text{H}^{+}$ | 0.47 | Table 10.2 |
| $\text{HFO_wOH} + \text{Cd}^{2+} = \text{HFO_wOCd}^{+} + \text{H}^{+}$ | -2.9 | Table 10.2 |
| $\text{HFO_sOH} + \text{Cr}^{3+} + \text{H}_2\text{O} = \text{HFO_sOCrOH}^{+} + 2\text{H}^{+}$ | 2.06 | Table 10.3 (and 6.43) ^a |
| $\text{HFO_sOH} + \text{Cu}^{2+} = \text{HFO_sOCu}^{+} + \text{H}^{+}$ | 2.89 | Table 10.2 |
| $\text{HFO_wOH} + \text{Cu}^{2+} = \text{HFO_wOCu}^{+} + \text{H}^{+}$ | 0.6 | Table 10.5 |
| $\text{HFO_sOH} + \text{Ni}^{2+} = \text{HFO_sONi}^{+} + \text{H}^{+}$ | 0.37 | Table 10.2 |
| $\text{HFO_wOH} + \text{Ni}^{2+} = \text{HFO_wONi}^{+} + \text{H}^{+}$ | -2.5 | Table 10.5 |
| $\text{HFO_sOH} + \text{Pb}^{2+} = \text{HFO_sOPb}^{+} + \text{H}^{+}$ | 4.65 | Table 10.2 |
| $\text{HFO_wOH} + \text{Pb}^{2+} = \text{HFO_wOPb}^{+} + \text{H}^{+}$ | 0.3 | Table 10.5 |
| $\text{HFO_sOH} + \text{Sr}^{2+} = \text{HFO_sOHSr}^{2+}$ | 5.01 | Table 10.1 |
| $\text{HFO_wOH} + \text{Sr}^{2+} = \text{HFO_wOSr}^{+} + \text{H}^{+}$ | -6.58 | Table 10.1 |
| $\text{HFO_wOH} + \text{Sr}^{2+} + \text{H}_2\text{O} = \text{HFO_wOSrOH} + 2\text{H}^{+}$ | -17.6 | Table 10.1 |
| $\text{HFO_sOH} + \text{Zn}^{2+} = \text{HFO_sOZn}^{+} + \text{H}^{+}$ | 0.99 | Table 10.2 |
| $\text{HFO_wOH} + \text{Zn}^{2+} = \text{HFO_wOZn}^{+} + \text{H}^{+}$ | -1.99 | Table 10.2 |
| $\text{HFO_wOH} + \text{B}(\text{OH})_3 = \text{HFO_wH}_2\text{BO}_3 + \text{H}_2\text{O}$ | 0.62 | Table 10.7 |

Table 6-12. Surface Complexation Reactions (Continued)

| Reaction | log K | Location |
|---|-------|-------------|
| $\text{HFO_wOH} + \text{CrO}_4^{2-} + \text{H}^+ = \text{HFO_wCrO}_4^- + \text{H}_2\text{O}$ | 10.85 | Table 10.8 |
| $\text{HFO_wOH} + \text{CrO}_4^{2-} = \text{HFO_wOHCrO}_4^{2-}$ | 3.9 | Table 10.10 |
| $\text{HFO_wOH} + \text{SeO}_3^{2-} + \text{H}^+ = \text{HFO_wSeO}_3^- + \text{H}_2\text{O}$ | 12.69 | Table 10.8 |
| $\text{HFO_wOH} + \text{SeO}_3^{2-} = \text{HFO_wOHSeO}_3^{2-}$ | 5.17 | Table 10.8 |
| $\text{HFO_wOH} + \text{SeO}_4^{2-} + \text{H}^+ = \text{HFO_wSeO}_4^- + \text{H}_2\text{O}$ | 7.73 | Table 10.8 |
| $\text{HFO_wOH} + \text{SeO}_4^{2-} = \text{HFO_wOHSeO}_4^{2-}$ | 0.8 | Table 10.8 |
| $\text{HFO_wOH} + \text{SO}_4^{2-} + \text{H}^+ = \text{HFO_wSO}_4^- + \text{H}_2\text{O}$ | 7.78 | Table 10.8 |
| $\text{HFO_wOH} + \text{SO}_4^{2-} = \text{HFO_wOHSO}_4^{2-}$ | 0.79 | Table 10.8 |
| $\text{HFO_sOH} + \text{Ba}^{2+} = \text{HFO_sOHBa}^{2+}$ | 5.46 | Table 10.1 |
| $\text{HFO_wOH} + \text{Ba}^{2+} = \text{HFO_wOBa}^+ + \text{H}^+$ | -7.2 | Table 10.5 |
| $\text{HFO_sOH} + \text{Co}^{2+} = \text{HFO_sOCo}^+ + \text{H}^+$ | -0.46 | Table 10.2 |
| $\text{HFO_wOH} + \text{Co}^{2+} = \text{HFO_wOCo}^+ + \text{H}^+$ | -3.01 | Table 10.2 |
| $\text{HFO_sOH} + \text{Hg}^{2+} = \text{HFO_sOHg}^+ + \text{H}^+$ | 7.76 | Table 10.2 |
| $\text{HFO_wOH} + \text{Hg}^{2+} = \text{HFO_wOHg}^+ + \text{H}^+$ | 6.45 | Table 10.2 |
| $\text{HFO_wOH} + \text{Mg}^{2+} = \text{HFO_wOMg}^+ + \text{H}^+$ | -4.6 | Table 10.5 |
| $\text{HFO_sOH} + \text{Mn}^{2+} = \text{HFO_sOMn}^+ + \text{H}^+$ | -0.4 | Table 10.5 |
| $\text{HFO_wOH} + \text{Mn}^{2+} = \text{HFO_wOMn}^+ + \text{H}^+$ | -3.5 | Table 10.5 |
| $\text{HFO_sOH} + \text{NpO}_2^{2+} = \text{HFO_sONpO}_2^+ + \text{H}^+$ | 5.9 | Table 10.5 |
| $\text{HFO_wOH} + \text{NpO}_2^{2+} = \text{HFO_wONpO}_2^+ + \text{H}^+$ | 3.6 | Table 10.5 |
| $\text{HFO_sOH} + \text{PuO}_2^{2+} = \text{HFO_sOPuO}_2^+ + \text{H}^+$ | 5.4 | Table 10.5 |
| $\text{HFO_wOH} + \text{PuO}_2^{2+} = \text{HFO_wOPuO}_2^+ + \text{H}^+$ | 3 | Table 10.5 |
| $\text{HFO_sOH} + \text{UO}_2^{2+} = \text{HFO_sOUO}_2^+ + \text{H}^+$ | 5.2 | Table 10.5 |
| $\text{HFO_wOH} + \text{UO}_2^{2+} = \text{HFO_wOUO}_2^+ + \text{H}^+$ | 2.8 | Table 10.5 |
| $\text{HFO_wOH} + \text{AsO}_4^{3-} = \text{HFO_wOHAsO}_4^{3-}$ | 10.58 | Table 10.6 |
| $\text{HFO_wOH} + \text{AsO}_4^{3-} + 2\text{H}^+ = \text{HFO_wHAsO}_4^- + \text{H}_2\text{O}$ | 23.51 | Table 10.6 |
| $\text{HFO_wOH} + \text{AsO}_4^{3-} + 3\text{H}^+ = \text{HFO_wH}_2\text{AsO}_4 + \text{H}_2\text{O}$ | 29.31 | Table 10.6 |
| $\text{HFO_wOH} + \text{F}^- + \text{H}^+ = \text{HFO_wF} + \text{H}_2\text{O}$ | 8.7 | Table 10.10 |
| $\text{HFO_wOH} + \text{F}^- = \text{HFO_wOHF}^-$ | 1.6 | Table 10.10 |
| $\text{HFO_wOH} + \text{H}_3\text{AsO}_3 = \text{HFO_wH}_2\text{AsO}_3 + \text{H}_2\text{O}$ | 5.41 | Table 10.7 |
| $\text{HFO_wOH} + \text{PO}_4^{3-} + 2\text{H}^+ = \text{HFO_wHPO}_4^- + \text{H}_2\text{O}$ | 25.39 | Table 10.6 |
| $\text{HFO_wOH} + \text{PO}_4^{3-} + 3\text{H}^+ = \text{HFO_wH}_2\text{PO}_4 + \text{H}_2\text{O}$ | 31.29 | Table 10.6 |
| $\text{HFO_wOH} + \text{PO}_4^{3-} + \text{H}^+ = \text{HFO_wPO}_4^{2-} + \text{H}_2\text{O}$ | 17.72 | Table 10.6 |
| $\text{HFO_wOH} + \text{SiO}_3^{2-} + \text{H}^+ = \text{HFO_wSiO}_3^- + \text{H}_2\text{O}$ | 15.9 | Table 10.10 |
| $\text{HFO_wOH} + \text{SiO}_3^{2-} = \text{HFO_wOHSiO}_3^{2-}$ | 8.3 | Table 10.10 |
| $\text{H}_2\text{SiO}_3^0 = \text{HSiO}_3^- + \text{H}^+$ | -9.86 | Table 10.9 |
| $\text{HSiO}_3^- = \text{SiO}_3^{2-} + \text{H}^+$ | -13.1 | Table 10.9 |

Source: Dzombak and Morel 1990 [DIRS 105483].

NOTE: ^aThere is a charge balance error in the reaction described by Dzombak and Morel (1990 [DIRS 105483], Table 10.3). Reaction is taken from Table 6.43 instead.

6.3.2.1.1 Evaluation of Experimental Data

Dzombak and Morel (1990 [DIRS 105483], Chapter 4) dedicate an entire chapter to the careful and thorough compilation of experimental data. Dzombak and Morel (1990 [DIRS 105483]) established a rigorous, systematic procedure to identify and collect all available adsorption data from refereed papers, books, technical reports, and dissertations. These data were then scrutinized for the critical information needed to determine whether to accept or reject the data

for the purpose of deriving surface complexation model reactions. The decision to accept or reject a given data set was based on reliability and/or usefulness of the data. Critical information included (Dzombak and Morel (1990 [DIRS 105483], p. 66):

- Method of preparation of the HFO
- Composition of the electrolyte solution
- Temperature
- Type of reaction vessel
- Methods employed to avoid carbonate contamination
- Sorbate/sorbent ratio
- Equilibration time
- Method of solid-liquid separation
- Analytical methods
- Method for controlling pH
- Other potentially important factors.

Experimental details for each data set, whether accepted or rejected, are included in *Surface Complexation Modeling, Hydrous Ferric Oxide* (Dzombak and Morel (1990 [DIRS 105483], Appendix A).

Dzombak and Morel (1990 [DIRS 105483]) did not include carbonate as one of the sorbates in their model. The carbonate adsorption reactions in Table 4-9 are obtained from work by Appelo et al. (2002 [DIRS 168168]). These reactions were derived from experimental data presented in a paper by Zachara et al. (1987 [DIRS 105963]), one work Dzombak and Morel (1990 [DIRS 105483]) accepted for its experimental chromate adsorption data. Based on the critical information necessary to accept or reject the data set (as outlined above), carbonate adsorption data from the report by Zachara et al. (1987 [DIRS 105963]) was found to satisfy the data criterion.

Several of the sorbates and their adsorption reactions in Tables 4-9 and 6-12 are estimated based on linear free energy relationships (Dzombak and Morel 1990 [DIRS 105483], Chapter 10). Their effects are reflected in this report's results (Section 6.8.4).

6.3.2.1.2 Evaluation of Accuracy of Derived Surface Complexation Reactions

The accuracy of the surface complexation database is evaluated by comparing model predictions to the experimental data (i.e., to those presented by Dzombak and Morel 1990 [DIRS 105483] and Appelo et al. 2002 [DIRS 168168]).

Surface complexation reactions were considered acceptable if:

- Predictions of percent adsorbed within 20% (absolute) of experimental data for at least 90% of the experimental data for a given sorbate
- pH predicted within 0.5 pH unit for at least 90% of the surface hydrolysis experimental data.

The percentage of experimental data within these limits was determined for each sorbate by examining the graphs presented by Dzombak and Morel (1990 [DIRS 105483]) and Appelo et al. (2002 [DIRS 168168]). On each graph by Dzombak and Morel (1990 [DIRS 105483]), the differences between predictions and measurements were determined by comparing the data points to the dashed lines. The dashed lines represent predictions based on the derived surface complexation database. The comparisons were performed vertically to determine the differences between predicted and measured adsorption for the known parameter on the x-axis, usually pH. For carbonate adsorption, experimental data were vertically compared to predictions represented by the solid lines in the graph presented by Appelo et al. (2002 [DIRS 168168], p. 3,097). The results are summarized in Table 6-13 for the sorbates with experimental adsorption data.

Table 6-13. Results of Accuracy Evaluation

| Sorbate | Accuracy Criterion | Location | Sorbate | Accuracy Criterion | Location |
|------------------|--------------------|----------------|---|----------------------|-----------------------|
| Pb ²⁺ | Met | pp. 108 to 109 | H ⁺ | Met | pp. 96 to 100 |
| Zn ²⁺ | Met | pp. 113 to 123 | PO ₄ ³⁻ | Not Met ^a | pp. 195 to 199 |
| Cd ²⁺ | Met | pp. 127 to 138 | AsO ₄ ²⁻ | Not Met | pp. 202 to 204 |
| Hg ²⁺ | Not Met | pp. 145 to 147 | VO ₄ ³⁻ | Met | pp. 207 to 212 |
| Cu ²⁺ | Met | pp. 150 to 154 | AsO ₃ ²⁻ | Not Met | pp. 214 to 216 |
| Ag ⁺ | Not Met | pp. 156 to 158 | B(OH) ₃ | Met | p. 218 |
| Ni ²⁺ | Met | p. 160 | SO ₄ ²⁻ | Met | p. 220 |
| Co ²⁺ | Not Met | pp. 166 to 169 | SeO ₄ ²⁻ | Met | pp. 222 to 225 |
| Cr ³⁺ | Met | pp. 171 to 172 | SeO ₃ ²⁻ | Met | p. 227 |
| Ca ²⁺ | Met | pp. 174 to 178 | S ₂ O ₃ ²⁻ | Met | p. 229 |
| Sr ²⁺ | Met | pp. 181 to 186 | CrO ₄ ²⁻ | Met | pp. 232 to 238 |
| Ba ²⁺ | Not Met | pp. 188 to 190 | CO ₃ ²⁻ | Met | p. 3,097 ^b |

Source: Dzombak and Morel 1990 [DIRS 105483].

NOTES: ^aNo graphs provide percent adsorbed versus pH for PO₄³⁻.

^bFrom Appelo et al. 2002 [DIRS 168168].

6.3.2.2 Thermodynamic Database for PHREEQC

YMP_R3.dat is the thermodynamic database developed for the PHREEQC calculations performed in Section 6.8. This database is developed in this section and documented in DTN: MO0403SPAYMPR3.000. This DTN is used in the surface complexation demonstration in Section 6.8, and is solely intended to build confidence in the surface complexation model discussed in Section 6.3.2, and is not an actual technical product of this model. It is nearly identical to the *data0.ymp.R2* database for EQ3/6, except for the following:

- It is in PHREEQC database format
- It only has equilibrium constants for 25°C and, therefore, is only valid for calculations at 25°C
- The error in the log K value and formula of solid PuO₂(OH)₂·H₂O was corrected

- Acetic acid, ethane, formate, formaldehyde, formic acid, methane, methanol, H₂O₂, and HO₂⁻ have been removed
- Nine CaO-Al₂O₃ minerals, 1.7Ca(OH)₂SiO₂·0.917H₂O, Be₁₃U, friedl salt, hydrogarnet, and U₃P₄ have been removed
- USe, USe₂(alpha), USe₂(beta), and USe₃ mineral phases have been renamed to SeU, Se₂U(alpha), Se₂U(beta), and Se₃U
- Surface complexation reactions documented in DTN: MO0403SPASURCR.000 have been added to the database.

Other minor additions and modifications (due to differences in how the EQ3/6 and PHREEQC codes work) are documented in the comment lines of the *YMP_R3.dat* file.

This database was initially translated from the *data0.ymp.R2* database in DTN: MO0210SPATHDYN.000 [DIRS 160229]. First, EQPT from EQ3/6, version 7.2b (STN: UCRL-MA-110662), was used on a personal computer using Windows 95 operating system to generate a *data1f.ymp* file from the *data0.ymp.R2* database. To translate this file to the PHREEQC format, the organic species and reactions listed above were removed and *transl* (transl V2.0, STN: 10251-2.0-00 [DIRS 155029]) was then used to perform the translation on a personal computer using Windows 98 operating system. The translation produced files named *phreeqc.ymp* (the database translation into PHREEQC) and *fix_redox.out* (a list of redox reactions requiring correction in the *phreeqc.ymp* file. As instructed in the *transl* software management report (BSC 2001 [DIRS 154880]), the first file was corrected based on the redox comments in the second file. These files can be found in Appendix F.

Two additional steps were involved in the final production of the *YMP_R3.dat* database. First, the *phreeqc.ymp* file described above (*YMP_R3.dat* database in this step) was updated to match the changes made in the final *data0.ymp.R2* EQ3/6 database (DTN: MO0302SPATHDYN.000 [DIRS 161756]) and to correct the error in the log K value and formula of solid PuO₂(OH)₂·H₂O. Second, the surface complexation reactions documented in DTN: MO0403SPASURCR.000 were added to the database.

The complete *YMP_R3.dat* database, which includes the surface complexation database for HFO, is documented in DTN: MO0403SPAYMPR3.000.

6.4 ALTERNATIVE CONCEPTUAL MODELS

6.4.1 Alternative Conceptual Model I

The interior of a waste package is spatially heterogeneous with respect to composition; therefore, a stream of water traveling vertically downward through the cross-sectional interior of a waste package would encounter different materials at different times. Along this flow path and at a given time, water molecules would react with the solid or solids to which they were immediately adjacent, resulting in a water stream with a variable composition along its flow path. This conceptual model could be viewed as a one-dimensional column composed of *n* cells in which the reactants in each cell represent the waste package components in vertical cross section of the

waste package—this model would eliminate the constraint of the EQ6 solid-centered flow-through mode, used in the in-package chemistry model, of a well-mixed batch reactor. This alternative conceptual model would also provide spatial information on the chemistry inside of a breached waste package, information not used by the total system performance assessment for the license application.

The vapor-influx model results showed (Section 6.5.1) the mixing model actually provides a good alternative to the more-complex Alternative Conceptual Model I, as the waste package materials tend to have an overwhelming effect on the in-package chemistry regardless of mixing.

6.4.2 Alternative Conceptual Model II

Seepage entering a breached waste package is likely to vary in composition as a function of time (i.e., as the prevailing conditions in the drift and engineered barrier system (EBS) vary over time). Therefore, a conceptual model that implemented a continuously varying input fluid composition would be more representative than the current in-package chemistry model, in which the input fluid compositions are constant over time. The liquid-influx model showed model response was largely insensitive to input water composition because the effect of the waste package materials on the in-package chemistry exerted much more control than the input water composition (Section 6.5.2). The alternative conceptual models are listed in Table 6-14.

Table 6-14. Alternative In-Package Chemistry Conceptual Models

| Alternative Conceptual Model | Key Features | Screening Assessment and Basis |
|---------------------------------|---|--|
| Alternative Conceptual Model I | Waste package is compositionally heterogeneous | Vapor-influx model showed resulting chemical effects of individual waste package components were comparable to those of their ensembles (Section 6.5.1) |
| Alternative Conceptual Model II | Composition of seepage entering a waste package is likely to vary as a function of changing conditions in the unsaturated zone and drift environments | The liquid-influx model showed wide compositional ranges in the seepage composition had little influence on the resulting in-package chemistry (Section 6.5.2) |

Although these alternative models have been considered, this report demonstrates the current base-case model is sufficient, robust, and adequate for its intended purpose.

6.4.3 Alternative Conceptual Model III – Diffusion of Vapor into the Waste Package

These calculations determine the amount of water available for sorption and precipitation in the waste package from diffusion of water vapor through stress corrosion cracks. The alternative conceptual model is developed here to lay the foundation for a future model in which the vapor influx ionic strength abstraction can be implemented within TSPA-LA as a function relative humidity and temperature. This calculation is also performed to show that the range of values for vapor influx (0.1 to 10 liter H₂O/yr per waste package) presented in Section 6.10.2 is reasonable and valid. To ensure this range is defensible, two methods of calculation were used. The rate of diffusion of water vapor through the stress corrosion cracks into the waste package was calculated using Fick's First Law (Bird et al. 1960 [DIRS 103524], p. 503) and Stefan Tube Equations (Cutlip and Shacham 2000 [DIRS 163339], pp. 259 to 263). For both methodologies,

the rate of diffusion is calculated over the range of relative humidity and temperature in which diffusion of water vapor into the package is expected to occur. A range of water fluxes can then be determined based on the anticipated waste package temperature and relative humidity provided by *Multiscale Thermohydrologic Model* (BSC 2005 [DIRS 173944], Figure 6.3-69).

For both calculations, the humidity is assumed to be zero inside the waste package. This maximizes the water vapor concentration gradient between the exterior and interior of the waste package, and considers all humidity entering the waste package is immediately consumed by corrosion reactions. Because this assumption maximizes the amount of water available within the waste package available for reactions and transport, it is, therefore, bounding. The assumption of zero relative humidity inside the waste package in this section is only for the purposes of maximizing the calculated water vapor fluxes; in the EQ6 modeling, ample water is available to react with the degrading materials, allowing in-package chemistry to be modeled. The amount of water available to react in the EQ6 modeling as it pertains to this vapor influx calculation is discussed in more detail in Section 6.3.1.1 (*Scaling to 1 Liter and Code Implementation*).

6.4.3.1 Fick's First Law Diffusion Calculation

Fick's First Law can be applied to calculate the diffusion of water vapor through stagnant air in the stress corrosion cracks. As shown by Equation 1, Fick's First Law states species A diffuses (moves relative to the mixture) in the direction of the decreasing mole fraction of A, just as heat flows by conduction in the direction of decreasing temperature (Bird et al. 1960 [DIRS 103524], pp. 502 to 503). The calculation of water vapor flux through the stress corrosion crack is described in detail in *EBS Radionuclide Transport Abstraction* (BSC 2004 [DIRS 169868], Section 6.6.2).

The rate of diffusion for Fick's First Law for a binary system with constant molar density is (BSC 2004 [DIRS 169868], Section 6.6.2):

$$q = D_{AB} A_{SCC} \frac{\partial C_{WV}}{\partial x}$$

(Eq. 6-1)

$$= D_{AB} A_{SCC} \frac{C_{WV}}{\Delta x}$$

where,

q = diffusive flux (mol s⁻¹) of water vapor into the waste package

A = water vapor

B = air

D_{AB} = diffusion coefficient (cm² s⁻¹) of water vapor in air

A_{SCC} = cross-sectional area (cm²) of stress corrosion cracks through which water vapor can diffuse

C_{WV} = concentration of water vapor in air outside the waste package (mol/cm³)

Δx = diffusion distance (2.54 cm) for the thickness of the waste package outer lid (BSC 2004 [DIRS 170710], Detail A).

As described in *EBS Radionuclide Transport Abstraction* (BSC 2004 [DIRS 169868], Section 6.6.2) and in Appendix H, the parameters D_{AB} and C_{WV} are dependent on temperature and relative humidity.

6.4.3.2 Stefan Tube Diffusion Calculations

In contrast to Fick's First Law that calculates the flux solely due to diffusion, a Stefan Tube calculation determines the flux based on diffusion and bulk flow (Bird et al. 1960 [DIRS 103524], pp. 519 - 526; Cutlip and Shacham 2000 [DIRS 163339], pp. 259 - 261). A Stefan Tube calculation is used to calculate flux for situations with fast evaporation and potentially concentrated solutions (Cussler, 1997 [DIRS 111468] p. 58). It is commonly referred to as "diffusion through a stagnant gas film," and has been commonly used for experimental determination of gas diffusivities (Bird et al. 1960 [DIRS 103524] Section 17.2).

$$\underbrace{N_A}_{\text{flux}} = \underbrace{-D_{AB}C \frac{dx_A}{dz}}_{\text{diffusion}} + \underbrace{\frac{C_A}{C}(N_A + N_B)}_{\text{bulk flow (convection)}}$$

$$\text{flux} = \text{diffusion} + \text{bulk flow (convection)} \quad (\text{Eq. 6-2})$$

where,

A = water vapor

B = air

N_A = flux of A normalized to 1 cm² (mol/cm² · s)

C = total concentration (mol/cm³)

x_A = mole fraction of species A

D_{AB} = diffusion coefficient (cm² s⁻¹) of water vapor in air

C_A = concentration of A (mol/cm³)

N_B = flux of B (mol/cm² · s).

The derivation of the analytical solution to this equation is documented by Bird et al. (1960 [DIRS 103524], pp. 522 to 526) and Cutlip and Shacham (2000 [DIRS 163339], pp. 260 to 261). The water vapor flux for the Stefan Tube based on the analytical solution is provided below and described in Appendix H.

The entire system is presumed to be at constant temperature and pressure. Gases A and B are assumed to be ideal; the total concentration, C , constant; and component B stagnant. The analytical solution for the fluxes (Bird et al. 1960 [DIRS 103524], Eq. 17.2-15):

$$N_{Az} \Big|_{z=z_1} = \frac{D_{AB}C}{(z_2 - z_1)(x_B)_{lm}} (x_{A1} - x_{A2}) \quad (\text{Eq. 6-3})$$

where,

$$(x_B)_{lm} = \frac{(x_{B2} - x_{B1})}{\ln(x_{B2}/x_{B1})} = \frac{(x_{A1} - x_{A2})}{\ln[(1 - x_{A2})/(1 - x_{A1})]} \quad (\text{Eq. 6-4})$$

where,

D_{AB} and C = as defined in Equation 6-2.

z_1 = distance just above the liquid or vapor reservoir (the external environment outside the package)

z_2 = distance of the end of the Stefan Tube (the interior of the waste package)

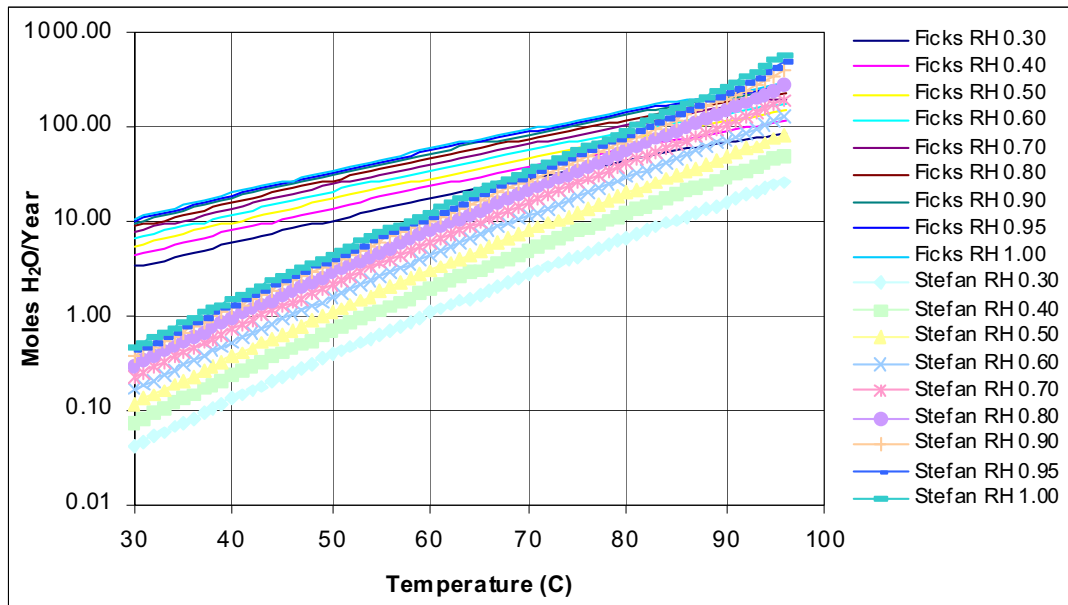
x_{A1} = concentration of gas A at location z_1

x_{A2} = concentration of gas A at location z_2 .

Cutlip and Shacham (2000 [DIRS 163339], Figure 7-1) illustrated the relationship between the location point and concentration of A.

6.4.3.3 Results

The results of the Fick's First Law and Stefan Tube calculations are shown in Figure 6-1. For both methodologies, the vapor flux was calculated for a temperature range of 30°C to 96°C and a range of relative humidity from 30 to 100 percent. Comparing the results of the two methodologies shows that low temperatures flux values calculated using the Stefan Tube are less than those calculated using Fick's First Law. However, as shown at high relative humidities 80, 90, 95, and 100 at high temperatures 94°C, 91°C, 89°C, and 88°C, respectively, the Stefan Tube calculated fluxes cross over and become higher than the values calculated based on Fick's First Law. This can be confirmed by comparing the tabulated results for the fluxes provided in Tables H.1-1 and H.2-1.

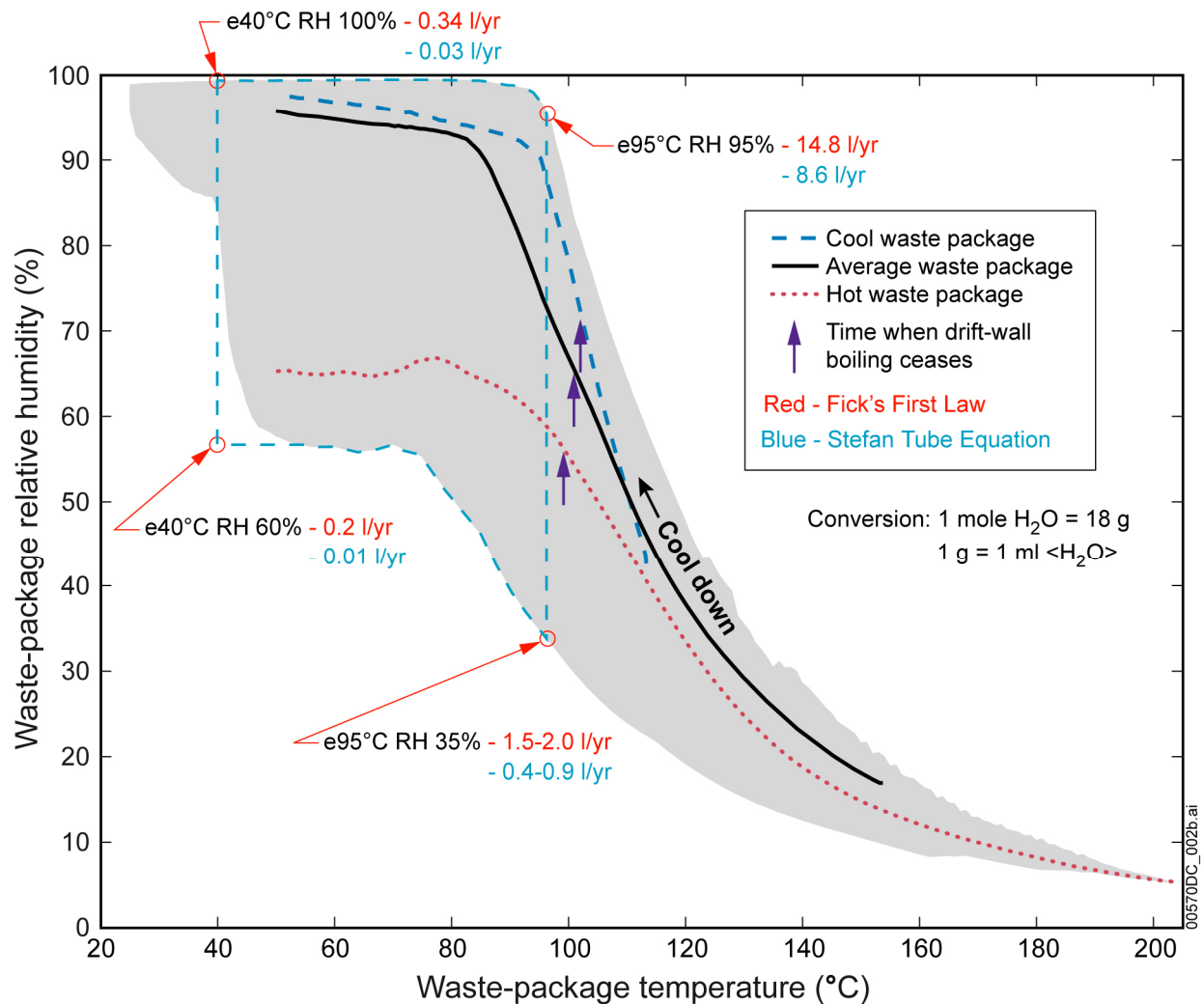


Source: *Fick's vs Stefan Tube - Flux vs Temp.xls.xls.xls* (Appendix F \Spreadsheets)

Figure 6-1. Vapor Influx Rates for Fick's First Law and Stefan Tube

Multiscale Thermohydrologic Model (BSC 2005 [DIRS 173944], Figure 6.3-69) illustrates the range of temperature and relative humidity anticipated outside a cooling waste package. As shown in Figure 6-2, four points that bounded the observed range in temperature and humidity were selected. These bounding points were then used to calculate the vapor diffusion flux. Because these bounding four points bracket the range of observed variation in temperature and humidity, they also bound the range of water vapor flux.

Figure 6-2 shows the calculated water vapor flux bounding the range of temperature versus relative humidity trajectories for all waste packages during cooldown, accounting for uncertainty in host-rock thermal conductivity and percolation flux. Higher vapor influx rates correspond to high temperatures and relative humidity (upper right of the bounded region of Figure 6-2), while lower influx rates are observed for low temperatures and relative humidity. At 95°C and 95% to 35% relative humidity, all calculated vapor fluxes, based on either Fick's First Law or Stefan Tube, fall above the minimum vapor influx of 0.1 liter/year (Section 6.10.2). However, at 40°C, the values calculated based on Fick's First Law exceed the minimum vapor influx, while the Stefan Tube-calculated values do not. At 40°C and 100% relative humidity, an influx rate of 0.03 L/yr is calculated, and at 60% relative humidity, a influx rate of 0.01 L/yr is calculated. These influx rates are below the modeled minimum vapor influx of 0.1 L/yr.



Source: BSC 2005 [DIRS 173944], Figure 6.3-69 updated to include the flux values calculated in this alternative conceptual model from *Fick's vs Stefan Tube - Flux vs Temp.xls.xls* (Appendix F \Spreadsheets).

Figure 6-2. Temperature versus Relative Humidity for Waste Packages

However, as indicated by the lines indicating the cool, average, and hot waste package with time, the waste package temperature will decrease along the lines in the direction of the “cooldown arrow.” Therefore, the overall time-integrated path of the waste package will experience higher temperatures and, thus, greater water vapor influxes during its cooldown history. Therefore, based on the higher influx rates calculated based on Fick's First Law and for the Stefan Tube higher-temperature history for the waste package, the range of water vapor influx (0.1 L/yr to 10 L/yr) presented in Section 6.10.2 is deemed to be valid and appropriate.

6.4.3.4 Thermal Diffusion

As discussed and illustrated by Bird et al. (1960 [DIRS 103524], pp. 564 to 565 and Table 18.4-1), there are two “driving forces” affecting diffusive mass flux. The first is due to concentration gradients and has been investigated by the application of Fick's First Law and Stefan Tube equations. The second (called the “Soret effect” or “thermal-diffusion effect”) is

due to temperature gradients. However, the effect of the “thermal-diffusion effect” was not calculated or determined, as its contribution to mass flux is secondary to concentration gradients (Bird et al. 1960 [DIRS 103524], p. 567; Housiadas et al. 2002 [DIRS 173263], p. 814).

6.5 BATCH-REACTOR BASE-CASE MODEL OUTPUT

This section provides the output from the batch reactor model runs. The pH value ranges from the surface complexation model are shown in some of these plots, but implementation of the surface complexation model is actually discussed in Section 6.9.

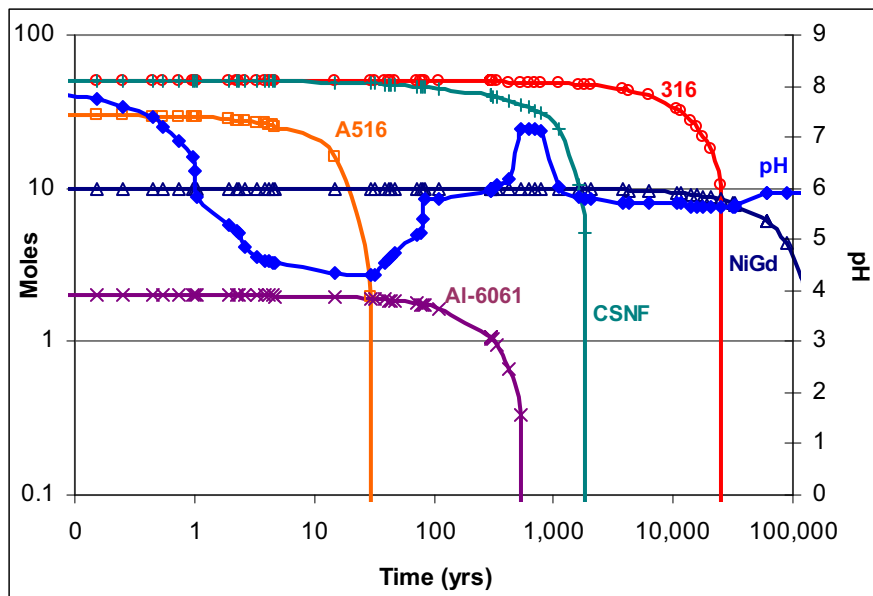
The “time” on the x-axis for all plots refers to the time after both the breach of the waste package has occurred and temperature in the repository has dropped below 100°C, at which time, liquid water can exist.

6.5.1 Liquid-Influx Case Model Results

6.5.1.1 CSNF Output

6.5.1.1.1 pH

The history of the reactants and pH values is plotted for the base case in Figure 6-3. The base case considers a reasonable representation of inputs by using median or most likely values: a flux of 1 L/yr, median rates for all of the alloys and fuels, and 100% cladding exposure at $\log f\text{CO}_2 = -3$, and $\log f\text{O}_2 = -0.7$. The incoming water composition is J-13, and the runs were conducted at 25°C.

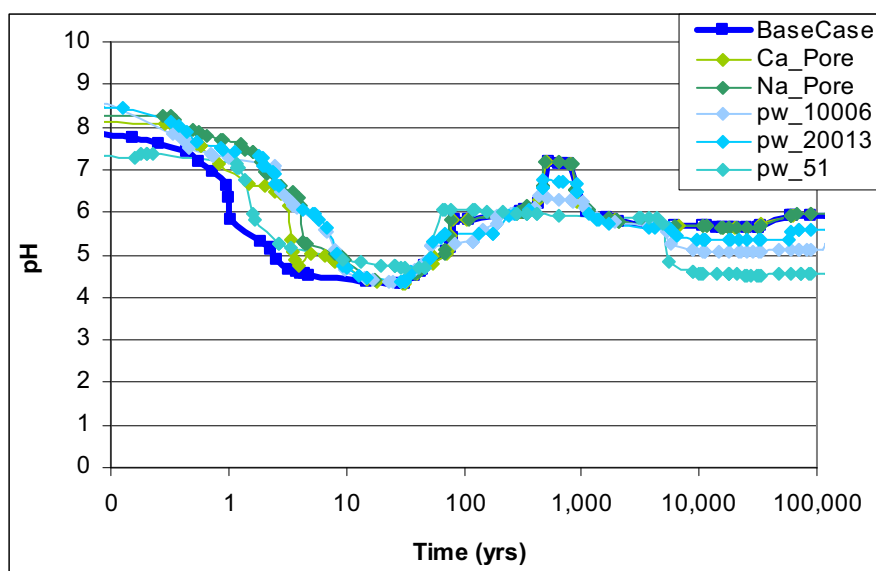


Source: CSNF Liquid Influx Model Output.xls (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-3. CSNF Liquid-Influx Base-Case Reactant Summary and pH Results

Figure 6-4 shows the pH history for different seepage compositions. The EQ6 results plotted in Figure 6-4 reflect only the physical chemistry of the system; the surface complexation model is not represented in this plot. The evolution of the pH in a CSNF waste package is complex and is controlled by the precipitation and dissolution of many phases. The following section provides a brief explanation for the trends in pH for this representative CSNF case, but a detailed discussion is provided in Appendix A. The pH reaches a minimum at around 30 years due to the rapid degradation of the A516, which releases sulfur and phosphorus that drive down the pH as they oxidize. With the A516 depleted, the pH increases sharply from ~30 to 100 years, and then more gradually until about 300 years. Shortly after 300 years, the pH increases again sharply until about 600 years. When the pH exceeds 7, saponite-tri solid solution $(Ca_{0.5},H,K,Mg_{0.5},Na)_{0.33}Mg_3Al_{0.33}Si_{3.67}O_{10}(OH)_2$ begins to precipitate up until the Al-6061 is depleted. Once the source for the Al has been depleted, the saponite-tri solid solution begins to dissolve. When the saponite-tri solid solution is exhausted at about 1000 years, the pH drops. At about 2300 years, the CSNF is exhausted and kaolinite begins to precipitate, after which the pH remains virtually constant at about 6.



Source: CSNF Liquid Influx Model Output.xls (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-4. CSNF Liquid-Influx Model Results for Various Incoming Water Compositions

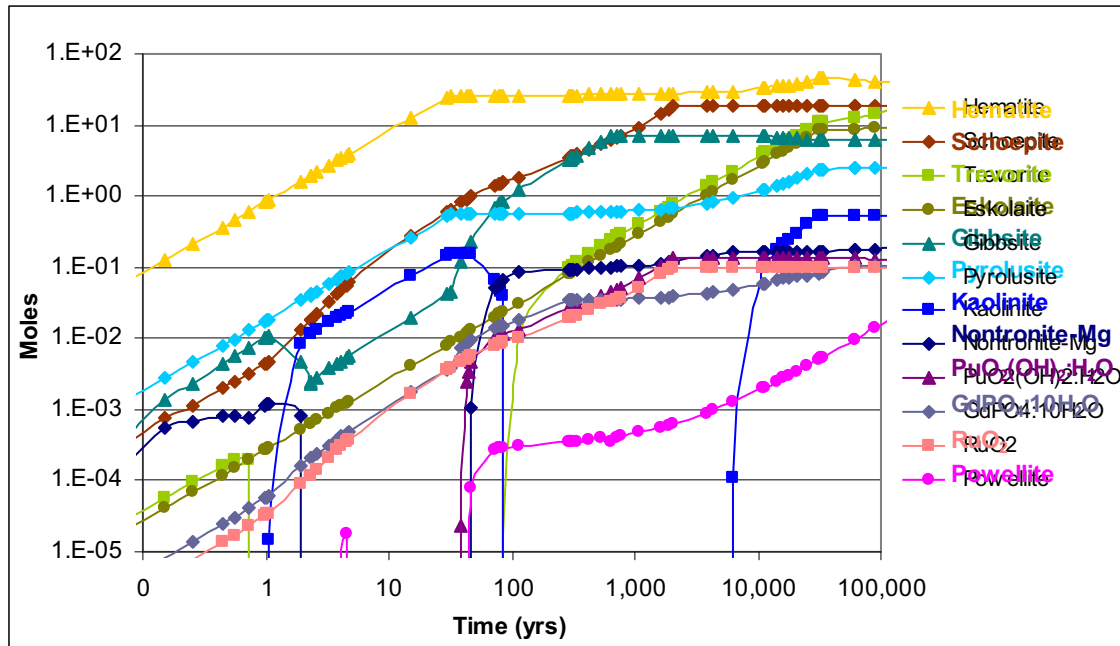
Figure 6-4 emphasizes composition of the seepage (Tables 4-2 and 4-3) has little effect on the pH profile. The most notable effect is for the long-term pH for the high-temperature waters (pw_51 is 91.8°C, pw_10006 is 56.2°C, and pw_20013 is 40.2°C). The separation in the early-time (less than approximately 0.5 years after waste package breach) pH profiles is a function of the seepage chemistry. However, this difference dissipates in approximately 10 years, and the pH profiles become nearly identical. The above discussion addresses the subject matter of “Included” FEP 2.2.08.12.0B (Chemistry of water flowing into the waste package) and indicates how that subject matter has been included and addressed within TSPA-LA. Thus, the composition of the water flowing into the EBS has a negligible impact on the in-package chemistry pH. The in-package chemistry model uses the results of cases examining the three seepage compositions, so the effect of the chemistry of the water flowing

into the EBS is included in the feed to TSPA-LA. (Even though the abstractions passed to TSPA-LA from this model are not defined in terms of the incoming water composition). These results show that the model is robust with respect to Alternative Conceptual Model II. If the liquid-influx model had been sensitive to the input water composition, then an alternative conceptual model with time-varying composition would be needed.

As described in Section 4.1.2, the model does not consider the possibility of seepage concentrated via evaporative processes entering the waste package. However, if small quantities of these in-drift waters did enter the waste package, as run off from some of the components located above the waste package, the additional incoming crown seepage water dripping from the drift ceiling and the waste package environment itself would tend to overwhelm the chemistry of concentrated solutions. Additionally, the principal solutes following evaporation will be those of soluble salts like Cl, Na, Mg, etc. Figures 6-6 and 6-8 illustrate that changes in the concentrations of solutes like Na do not affect solutes derived from the waste package.

Further examination of Figure 6-3 shows reactants dissolve as a function of time (i.e., they are kinetic reactants). Their effect on the in-package chemistry is explicitly modeled in the in-package chemistry model and, thus, is carried over to TSPA-LA. The above discussion addresses the subject matter of "Included" FEP 2.1.09.07.0A (Reaction kinetics in waste package) and indicates how that subject matter has been included and addressed within TSPA-LA.

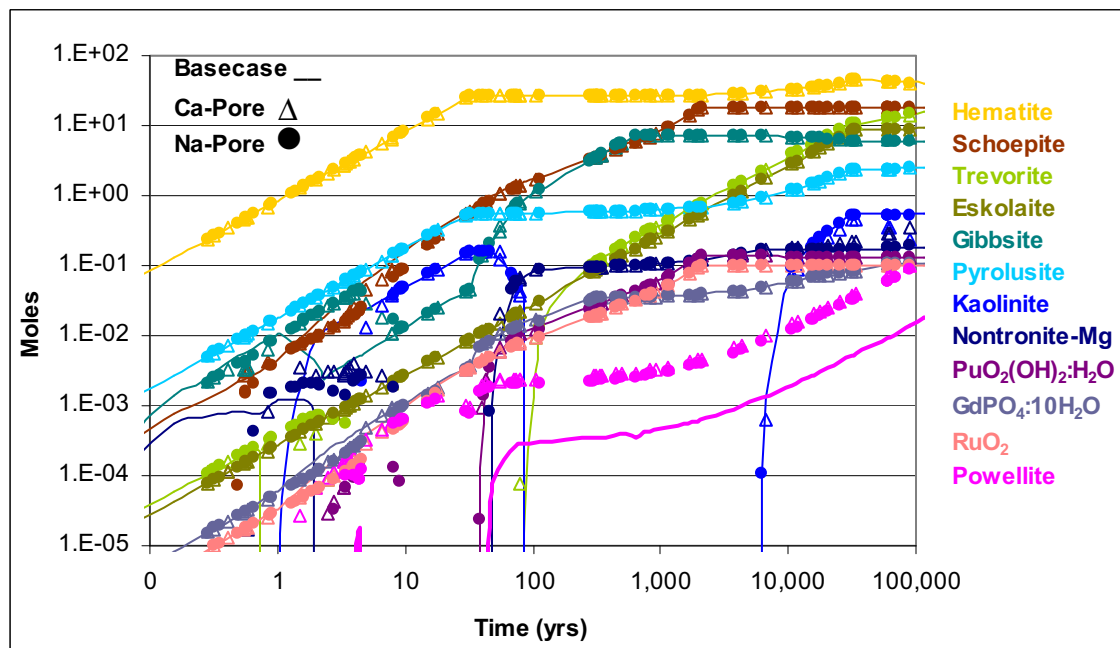
Figure 6-5 displays mineral phases that precipitated from solution for the base case. Hematite, gibbsite, schoepite, pyrolusite, and trevorite constitute the five most abundant phases, all of which are expected to form in the oxidizing waste package environment (Table 6-11). The large amount of gibbsite ($\text{Al}(\text{OH})_3$) that formed in these runs is due to the relatively low amount of available silica in the incoming waters and waste package components. Had more silica been available, greater quantities of aluminum-silicate phases would then have formed with proportionately less gibbsite formation within the near-neutral pH range. The silica phases that do form include kaolinite, nontronite-Mg, and nontronite-Ca, all of which are clay minerals. In addition to these most-abundant phases, some other phases of interest (actinides or neutron absorbers) that form in smaller quantities include: $\text{PuO}_2(\text{OH})_2 \cdot \text{H}_2\text{O}$, $\text{GdPO}_4 \cdot 10\text{H}_2\text{O}$, RuO_2 , $\text{Gd}_2(\text{CO}_3)_3$, NpO_2 , $(\text{UO}_2)_3(\text{PO}_4)_2 \cdot 4\text{H}_2\text{O}$, and uranophane(alpha). The mineral assemblage plots (Figures 6-5 and 6-6) support that the composition of the seepage has little influence over which minerals formed and their amounts. The J-13 well water minerals show some variability compared to the other water compositions; however, the major phases are present in nearly identical amounts. Differences in input water chemistry do not change the identities of minerals formed during waste-package degradation, but may lead to differences in the amounts of some minerals formed. For example, less powellite (CaMoO_4) forms from base case water than from either Ca- or Na-porewater. This is because the Ca contents of both these waters are higher than that of the base case water (Table 4-2).



Source: CSNF Liquid Influx Model Output.xls (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-5. CSNF Liquid-Influx Model Mineral Phases for the Base Case



Source: CSNF Liquid Influx Model Output.xls (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

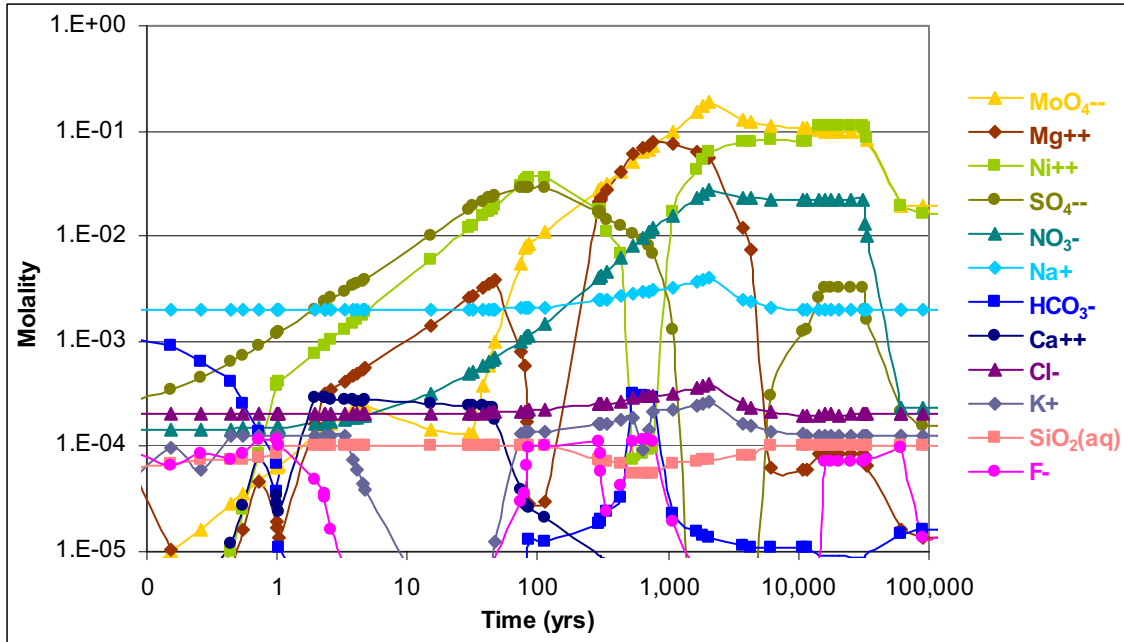
Figure 6-6. CSNF Liquid-Influx Model Mineral Phases for Three Seepage Compositions

Figures 6-7 and 6-8 display aqueous species for the runs with various input water compositions. Figure 6-8 shows the in-package chemical environment control elements not present in the initial

water compositions (Table 4-2). In other words, elements originating from the waste package and waste forms have aqueous species concentrations nearly identical and independent of the initial water composition for the duration of the simulations. Examples include Ni and Mo, which originate from oxidation of the metal waste package components. Elements present in the initial water composition and involved in reactions in the waste package also have similar aqueous species concentrations. For example, sulfur, which was present in all of the water compositions as sulfate (SO_4^{2-}), shows similar concentration trends in the simulations, independent of the starting water composition, due to the sulfur contained in the Carbon Steel Type A516 that dominates the sulfate concentration trends observed in Figures 6-7 and 6-8. Figures 6-7 and 6-8 show that, depending on the initial fluid composition, there is a small degree of variability in the minerals and aqueous species of the reacted fluid chemistry that may exit a breached waste package. However, this variability is slight and is captured in the overall uncertainty treatment in the pH and ionic strength of the reacted solutions.

Several of the porewaters are high-temperature seepage waters, and these cases were run using EQ6 at the corresponding higher temperatures using the temperature-appropriate CSNF degradation rates (Table 6-7). The metal alloy corrosion rates are the same as those used for the 25°C runs, because these rates are provided in DTN: MO0409SPAACRWP.000 [DIRS 172059] as appropriate over this range of temperatures and because many of the experimental rates were measured at higher temperatures. The thermodynamic database includes high temperature data for most mineral and aqueous species; therefore, the reacted solution chemistry and precipitated mineral phases are temperature appropriate.

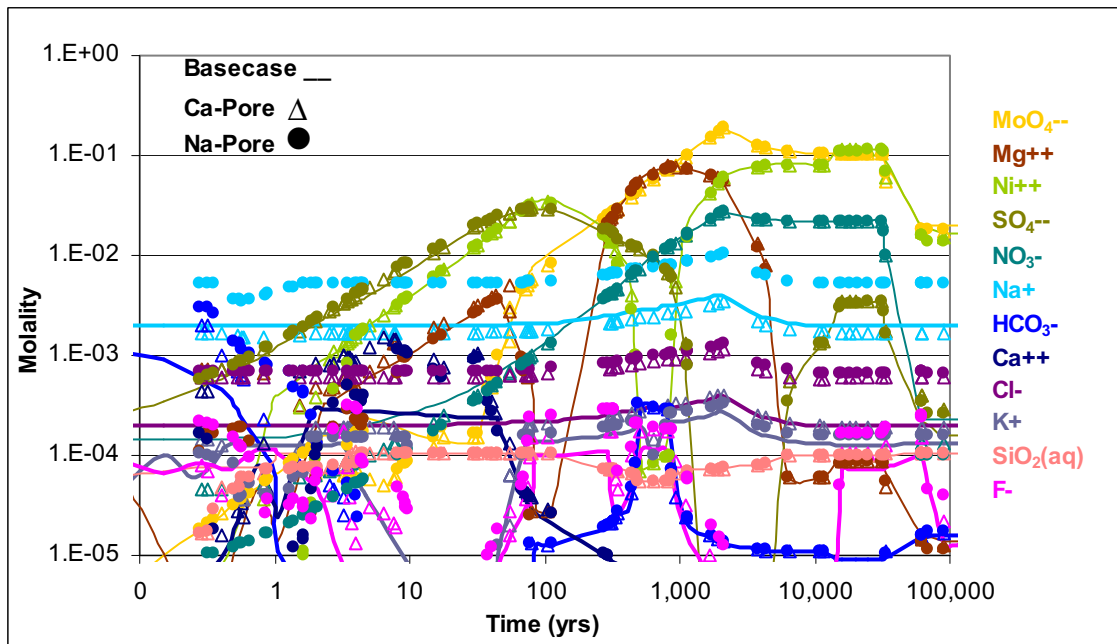
Higher temperature ($>25^\circ\text{C}$) runs achieve an overall lower pH profile at long time periods (as shown in Figure 6-4 and discussed further in Section 6.6.6) and slight variations in the stable mineral assemblages compared with the results at 25°C. Since several simulations are carried out at above ambient temperatures and the in-package chemistry model results reflect the increased thermal conditions, the TSPA-LA feed also reflects the increased thermal conditions. The above discussion addresses the thermal effects on chemistry portion of the subject matter of “Included” FEP 2.1.11.08.0A (Thermal effects on chemistry and microbial activity in the EBS) and indicates how that subject matter has been included and addressed within TSPA-LA. The microbial activity in the EBS subject matter is described in FEP 2.1.10.01.0A as documented in *Engineered Barrier System Features, Events, and Processes* (BSC 2004 [DIRS 169898]). The effect of temperature on the surface complexation model is discussed in Section 6.7.3.



Source: CSNF Liquid Influx Model Output.xls (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-7. CSNF Liquid-Influx Model Aqueous Species for the Base Case



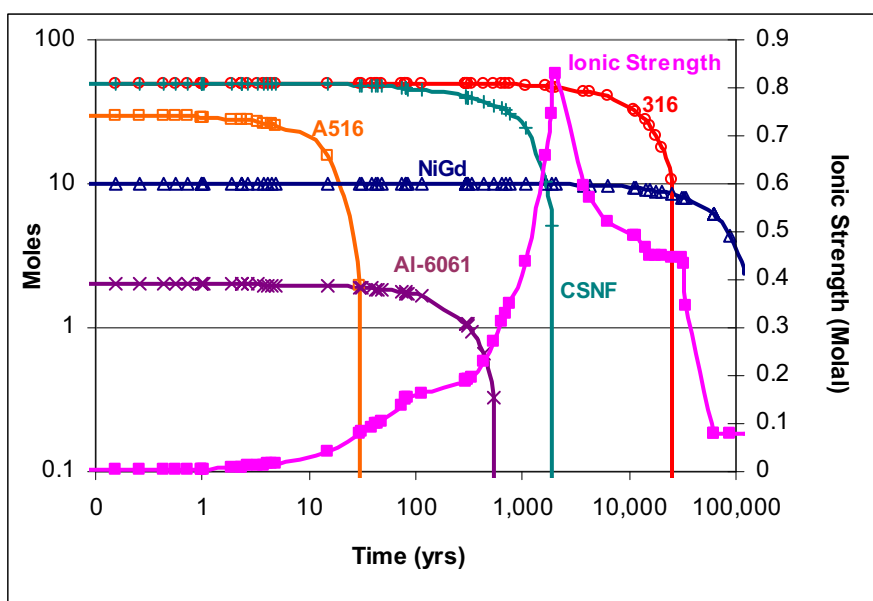
Source: CSNF Liquid Influx Model Output.xls (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-8. CSNF Liquid-Influx Model Aqueous Species for Three Seepage Compositions

6.5.1.1.2 Ionic Strength

Figure 6-9 displays the ionic strength profiles for the CSNF base case. The base case considers a reasonable representation of inputs: a flux of 1 L/yr, median rates for all of the alloys and fuels, J-13 well water, and 100% cladding exposure, at 25°C, $\log f_{\text{CO}_2} = -3$, and $\log f_{\text{O}_2} = -0.7$. The ionic strength abstraction is used by the *Colloids* and *Dissolved Concentration Limits of Radioactive Elements* submodels in the TSPA-LA. The colloidal suspensions are unstable at ionic strength values above 0.05 mol/L (BSC 2004 [DIRS 170025]), so for the colloid submodel, the significance of exceeding the limits of the B-dot activity coefficient equation is diminished. For the *Dissolved Concentration Limits of Radioactive Elements* model (BSC 2005 [DIRS 173873]), TSPA-LA applies an additional uncertainty term to the concentrations if the ionic strengths is greater than one molal, so the significance of exceeding the limits (~ 4 molal) of the B-dot activity coefficient equation is already captured.



Source: CSNF Liquid Influx Model Output.xls (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-9. CSNF Liquid Influx Base-Case Reactant Summary and Ionic Strength

Figure 6-9 shows ionic strength results with the reactant summary. The ionic strength curve shows cusps as a response to the depletion of each of the reactants; the most noticeable cusps occurring after the depletion of CSNF fuel and then Stainless Steel Type 316.

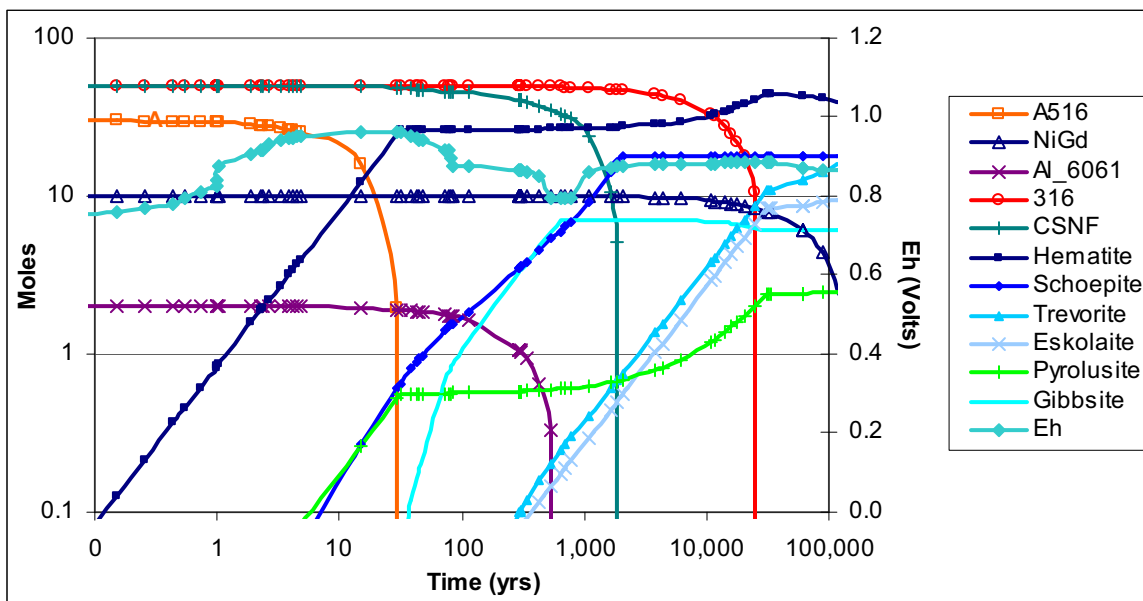
As described in Section 4.1.2, the model does not consider the possibility of seepage concentrated via evaporative processes entering the waste package. However, if small quantities of these in-drift waters did enter the waste package, as run off from some of the components located above the waste package, the additional incoming crown seepage water dripping from the drift ceiling and the waste package environment itself would tend to overwhelm the chemistry of concentrated solutions. The principal solutes following evaporation will be those of soluble salts like Cl, Na, Mg, which could increase the ionic strength of the in-package solution. This effect

is accounted for in the ionic strength abstraction, and is described in more detail in Section 6.10.2.

6.5.1.1.3 Eh

Figure 6-10 displays Eh (a measure of the redox potential) and oxidation products for the CSNF base-case run. The base case considers a reasonable representation of inputs: a flux of 1 L/yr, median rates for all of the alloys and fuels, J-13 well water, and 100% cladding exposure, at 25°C, $\log f_{CO_2} = -3$, and $\log f_{O_2} = -0.7$. The Eh, given in terms of the standard hydrogen electrode scale, varies between approximately 0.8 to 1.0 volts. The modeled conditions are uniformly oxidizing, so Eh changes only in response to pH changes, as shown in Figure 6-11. In reality, the presence of abundant reduced material and the ability of corrosion products to limit the access of water and oxygen are likely to cause substantial lowering of the in-package Eh. A lower Eh would lead toward more neutral pH values and more moderate ionic strengths, but because this effect cannot be quantified, uniformly oxidizing Eh value is used for the model, and sensitivity studies were conducted under less oxidizing conditions to determine the impact (Section 6.6.4).

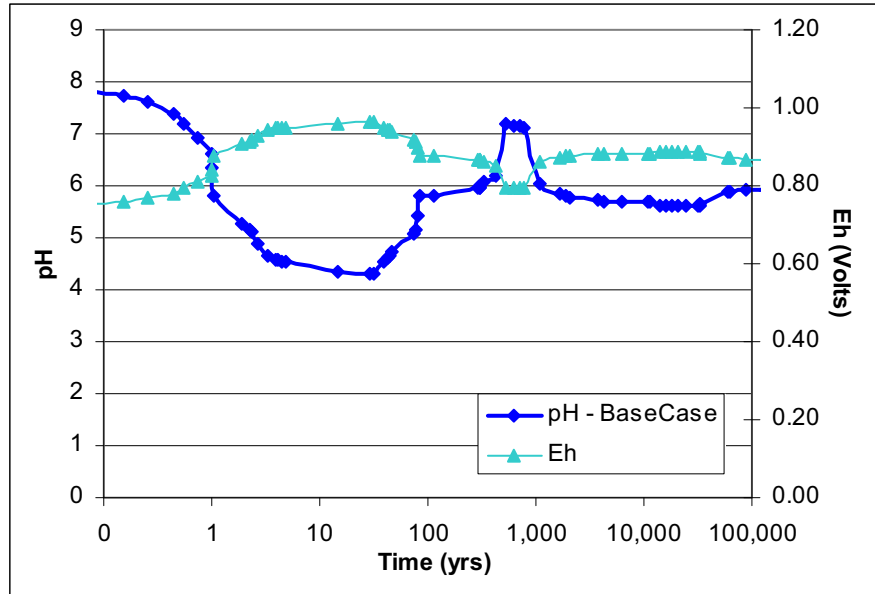
Since metal oxide corrosion products are precipitating from a solution in contact with the waste form and waste package components, the corrosion products have a first-order influence on the in-package chemistry. The above discussion addresses the subject matter of “Included” FEPs 2.1.09.02.0A (Chemical interaction with corrosion products) and 2.1.09.06.0A (Reduction-oxidation potential in waste package) and indicates how that subject matter has been included and addressed within TSPA-LA.



Source: CSNF Liquid Influx Model Output.xls (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-10. CSNF Liquid-Influx Model: Eh, Reactant Summary, and Corrosion Products



Source: CSNF Liquid Influx Model Output.xls (Appendix F \Spreadsheets).

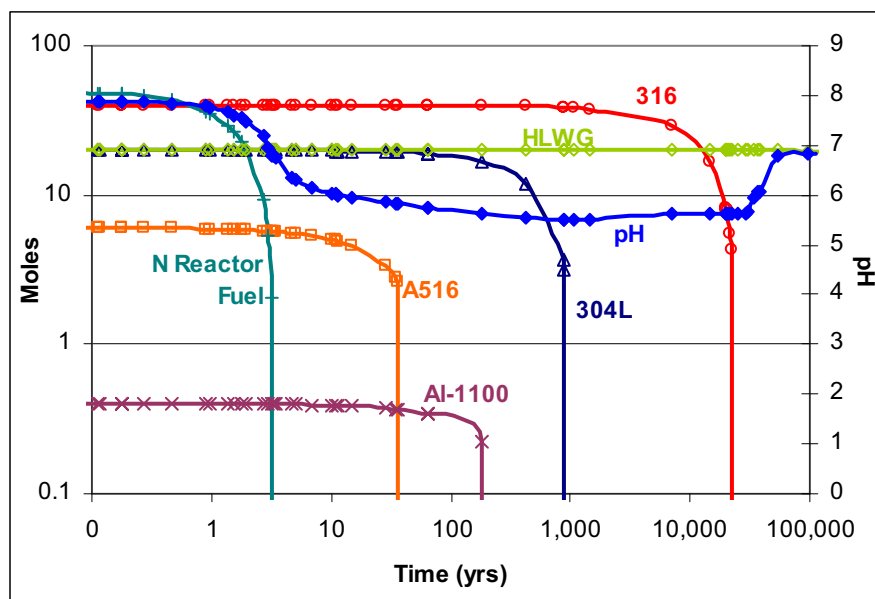
NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-11. CSNF Liquid-Influx Model: Eh and pH

6.5.1.2 CDSP Output

6.5.1.2.1 pH

The history of the reactants and the pH is plotted for the base case in Figure 6-12. The base case considers a reasonable representation of inputs by using median or most likely values: a flux of 1 L/yr, median rates for all of the alloys and fuels at $\log f_{\text{CO}_2} = -3$, and $\log f_{\text{O}_2} = -0.7$. The incoming water composition is J-13, and the runs were conducted at 25°C.



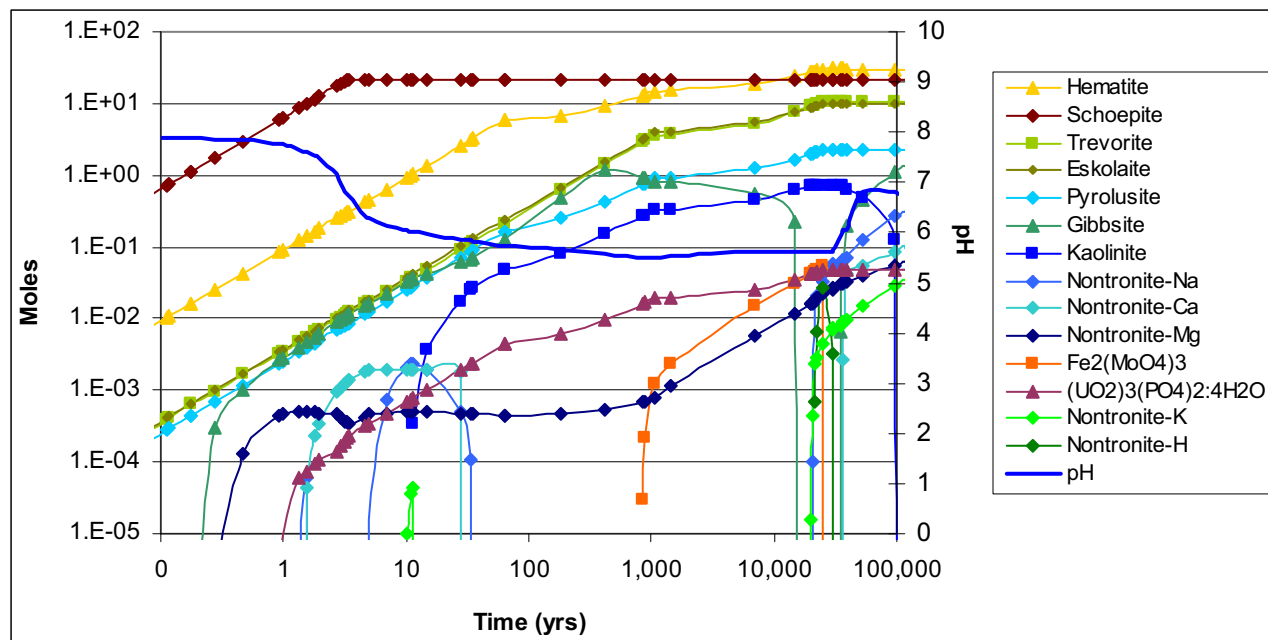
Source: *CDSP Liquid Influx Model Output.xls* (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-12. CDSP Liquid-Influx Base Case Reactant Summary and pH Results

The most outstanding feature in Figure 6-12 is the reactant summary for the N-Reactor fuel. The figure shows that the N-Reactor fuel dissolves over a short duration of just a few years. The rapid N-Reactor fuel dissolution diminishes the importance of the N-Reactor fuel to the overall in-package chemistry contribution. However, the large quantity of schoepite precipitated in the waste package influences the in-package chemistry for an extended duration by buffering pH in the near-neutral range.

Another important feature of the pH profiles in Figure 6-12 is the absence of a period of sustained high pH that might be expected from the dissolution of the HLWG, except at times greater than 100,000 years, after the steels have degraded, and the glass is the only remaining reactant in the system (as shown in the far right side of the plot). The single-component CDSP vapor-influx model predicted high pH conditions are possible in the absence of an acid producing reactant (i.e., steel alloy). However, in the liquid-influx model, seepage is allowed to “stream” through the waste package reacting with waste package components. It is unlikely seepage that has reacted with HLWG could exit a waste package without contacting a steel component, or schoepite from N-Reactor fuel degradation, along its flow path (both having the capacity to buffer the pH in the neutral range).

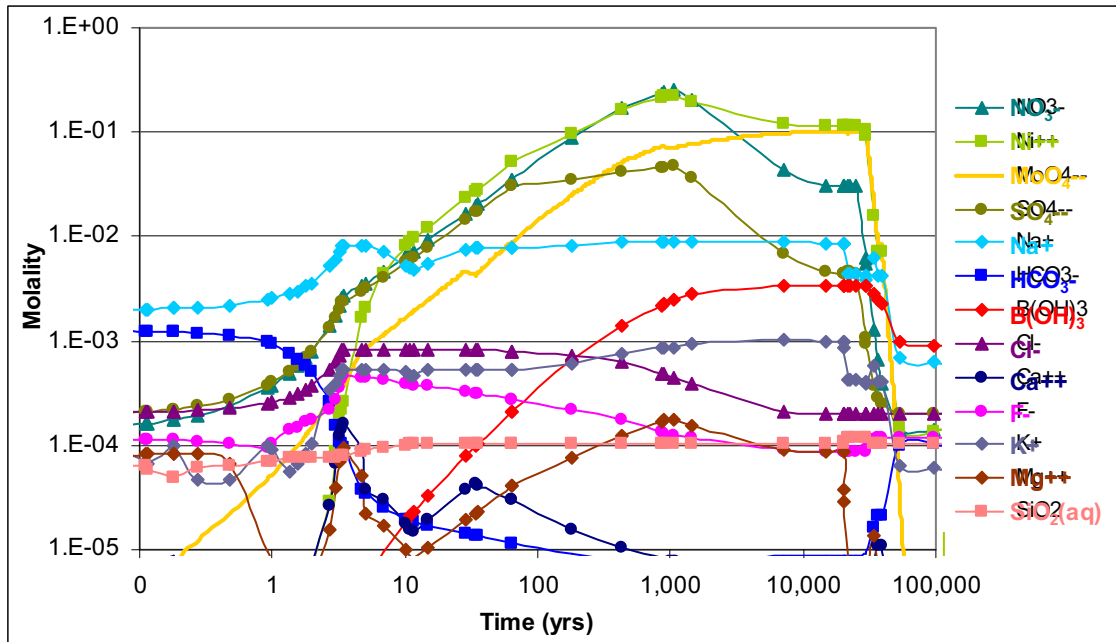


Source: CDSP Liquid Influx Model Output.xls (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-13. CDSP Liquid-Influx Model Mineral Phases for the Base Case

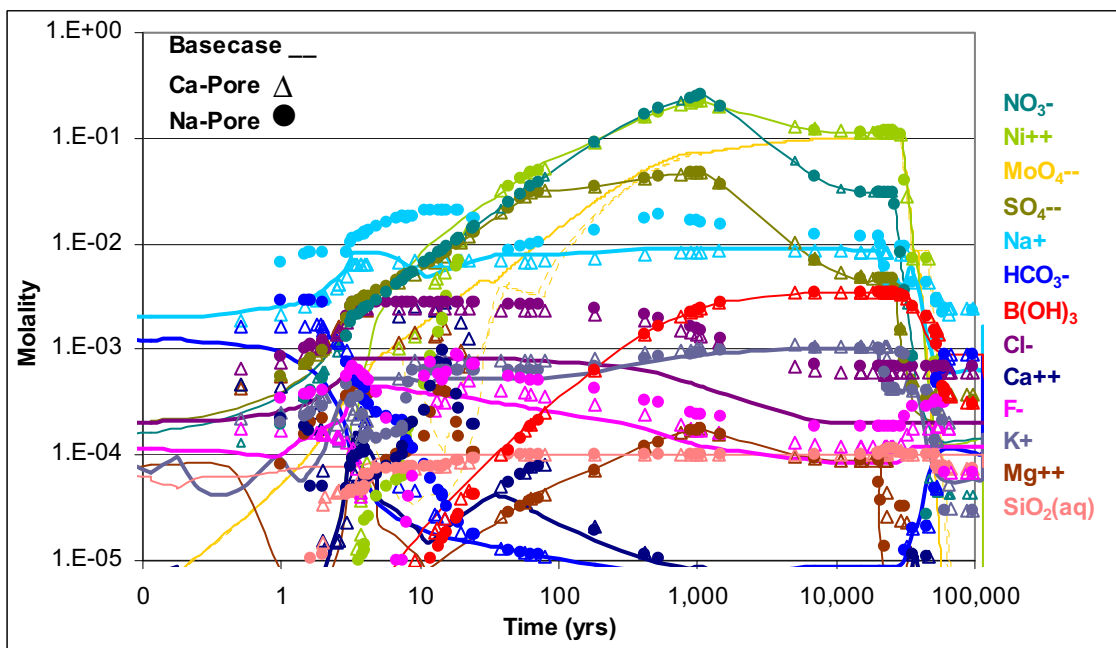
Figure 6-13 shows the minerals that formed for the CDSP liquid-influx model. This plot shows that hematite, schoepite, trevorite, eskolaite, pyrolusite, and gibbsite constitute the most abundant mineral phases that form from the degradation of a CDSP waste package. The abundance of schoepite ($\text{UO}_3 \cdot 2\text{H}_2\text{O}$) results from the complete reaction of the uranium metal N-Reactor fuel, while the hematite, trevorite and gibbsite are products of steel and aluminum alloys dissolution, respectively. Over the duration of the simulations, less gibbsite forms in CDSP runs compared to CSNF runs (Figure 6-5), while greater amounts of the nontronite clay minerals form in CDSP runs compared to CSNF runs. This occurs because silica is more available in CDSP compared to CSNF waste packages as a result of the presence and degradation of HLWG. This additional silica leads to greater amounts of nontronite formation in CDSP waste packages. In addition to these most abundant phases, some other phases of interest (actinides) that form in smaller quantities include: Boltwoodite-Na, $(\text{UO}_2)_3(\text{PO}_4)_2 \cdot 4\text{H}_2\text{O}$, and Uranophane(alpha).



Source: CDSP Liquid Influx Model Output.xls (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-14. CDSP Liquid-Influx Model Aqueous Species for the Base Case



Source: CDSP Liquid Influx Model Output.xls (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

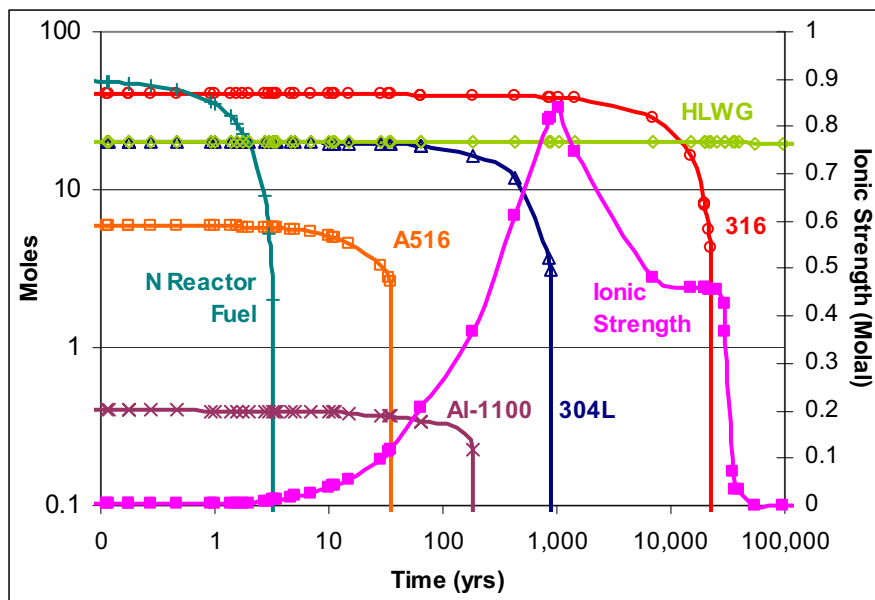
Figure 6-15. CDSP Liquid-Influx Model Aqueous Species for Three Seepage Compositions

Figures 6-14 and 6-15 display the aqueous species for the CDSP base case and for three seepage compositions respectively. Similar to CSNF, Figures 6-14 and 6-15 also show elements that were not present in the initial water compositions (Table 4-2) (i.e., those elements originating from the waste package and waste forms) have aqueous species concentrations nearly identical, and independent of the initial water composition, for the duration of the simulations. Nickel, sodium and boron have the highest concentrations, and either originated from HLWG dissolution or from the oxidation of the metal waste package components (in the case of nickel). Elements not originating from the waste form or package components and not participating in the formation of minerals will have aqueous species concentrations set by the incoming water composition (e.g., chloride).

Further examination of Figure 6-14 shows that the reactants dissolve as a function of time (i.e., kinetic reactants) and their effect on the in-package chemistry is explicitly modeled in the in-package chemistry model and, thus, is carried over to TSPA-LA. The above discussion addresses the subject matter of “Included” FEP 2.1.09.07.0A (Reaction kinetics in waste package) and indicates how that subject matter has been included and addressed within TSPA-LA.

6.5.1.2.2 Ionic Strength

Figure 6-16 displays the ionic strength profiles for the CDSP base case. The base case considers a reasonable representation of inputs: a flux of 1 L/yr, median rates for all of the alloys and fuels and J-13 well water, at 25°C, $\log f\text{CO}_2 = -3$, and $\log f\text{O}_2 = -0.7$. The ionic strength abstraction is used by the colloid and *Dissolved Concentration Limits of Radioactive Elements* submodels in the TSPA-LA. The colloidal suspensions are unstable at ionic strength values above 0.05 mol/L (BSC 2004 [DIRS 170025]). For the *Dissolved Concentration Limits of Radioactive Elements* model (BSC 2005 [DIRS 173873]), TSPA-LA applies an additional uncertainty term to the concentrations if the ionic strengths is greater than one molal.



Source: *CDSP Liquid Influx Model Output.xls* (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-16. CDSP Liquid-Influx Base Case Reactant Summary and Ionic Strength

Figure 6-16 shows ionic strength results with the reactant summary. The ionic strength curve shows cusps as a response to the depletion of each of the reactants; the most noticeable cusps occurring after the depletion of Stainless Steel Type 304L, and then Stainless Steel Type 316.

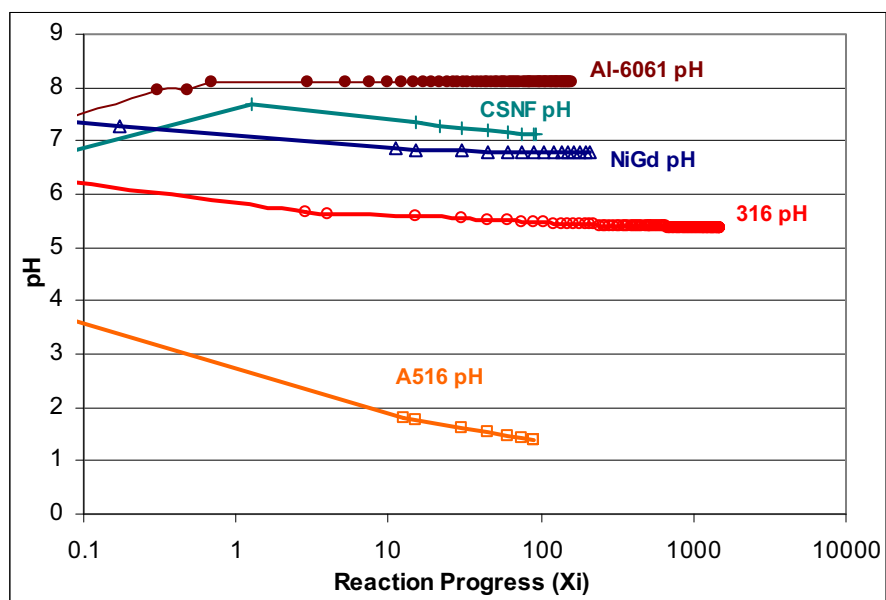
6.5.2 Vapor-Influx Model Results

For simplicity and comparison purposes, each of the single component runs considers the liter of water to be in contact with 4,000 cm² of available surface area (4,000 = 1 liter of water, spread over each reactant 0.25 cm thick). One exception to this surface area is for the aluminum alloy (Aluminum Alloy 6061 and Aluminum Alloy 1100) single component cases, which use a surface area of 2,000 cm². Modifying the surface area for this run does not impact the results because the single component runs were conducted to examine the steady state chemistry generated by each reactant. Therefore, the relative degradation rate is not important for this case. The main mineral that forms in the aluminum alloy single component runs is gibbsite, and the formation of gibbsite consumes more water than any of the other minerals, such as hematite and schoepite. When the runs were conducted with a surface area of 4,000 cm², the water was being consumed much faster than it was entering, so the surface area was divided in half to allow the runs to converge. However, the steady state pH and ionic strength for both runs are exactly the same (e.g., the steady state pH for Al-1100 is 6.70 and the ionic strength is 0.0001156 molal (see Appendix F/Spreadsheets/*CDSP Vapor Influx Model Output.xls* in)).

6.5.2.1 CSNF Output

Figure 6-17 displays the EQ6 results of the CSNF single component runs. The pH profiles in Figure 6-17 are plotted as a function of reaction progress (the EQ6 z_i (ξ) variable is moles reacted) rather than time elapsed. Plotting the results in this manner allows reactants' different

degradation rates to be compared with each other. This figure provides information on how each waste package component contributes to the in-package pH and shows the upper- and lower-pH bounds for the components in the CSNF package.



Source: CSNF Vapor Influx Model Output.xls in Appendix F.

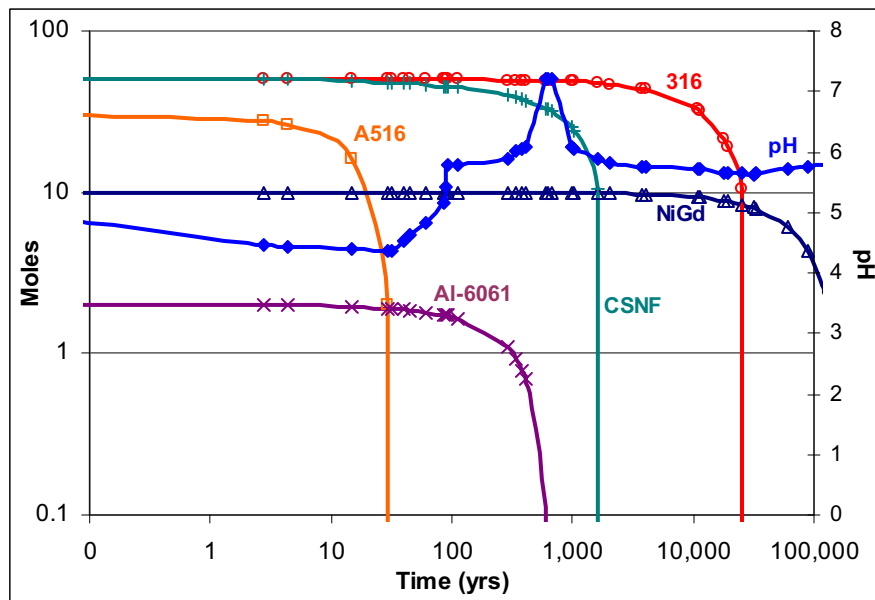
Figure 6-17. CSNF Single-Component Vapor Influx pH Outputs

The low pH resulting from Carbon Steel Type A516 corrosion is due to the combination of oxidation of sulfur, the large quantity of Carbon Steel Type A516, and its high corrosion rate relative to the other waste package components. When the surface complexation model is applied to the batch reactor results, pH values do not reduce below 4.5. In addition, the oxidation layer that would form on degrading steels would also slow degradation and further buffer the pH, though this effect is not modeled (Section 5.2). Aluminum Alloy 6061 and CSNF have neutral to slightly basic pH profiles, while Stainless Steel Type 316 and NiGd alloy have slightly acidic pH profiles. The trends observed in the pH are a function of the mineral phases becoming saturated and then precipitating as more and more reactant is added to solution.

The CSNF vapor-influx base case is shown in Figure 6-18. The base case considers a reasonable representation of inputs by using median or most likely values: a flux of 1 L/yr, median rates for the alloys and fuels, and 100% cladding exposure at $\log f_{\text{CO}_2} = -3$, and $\log f_{\text{O}_2} = -0.7$. The incoming water composition is deionized water and the runs were conducted at 25°C.

The most acid-producing profiles include those for the single component run Carbon Steel Type A516 profile shown in Figure 6-17 and in the base case, during the 0- to 600-year period in Figure 6-18, while the Carbon Steel Type A516 is degrading. The early-time pH (<100 years) is controlled by Carbon Steel Type A516 dissolution, the middle period (100 to 1,000 years) by Aluminum Alloy 6061 dissolution, and greater than 1,000 years by equilibrium with the corrosion products. The peak in pH at around 1,000 years also occurs in the liquid-influx case, and is explained in detail in Section 6.5.1.1.1 and in Appendix A.

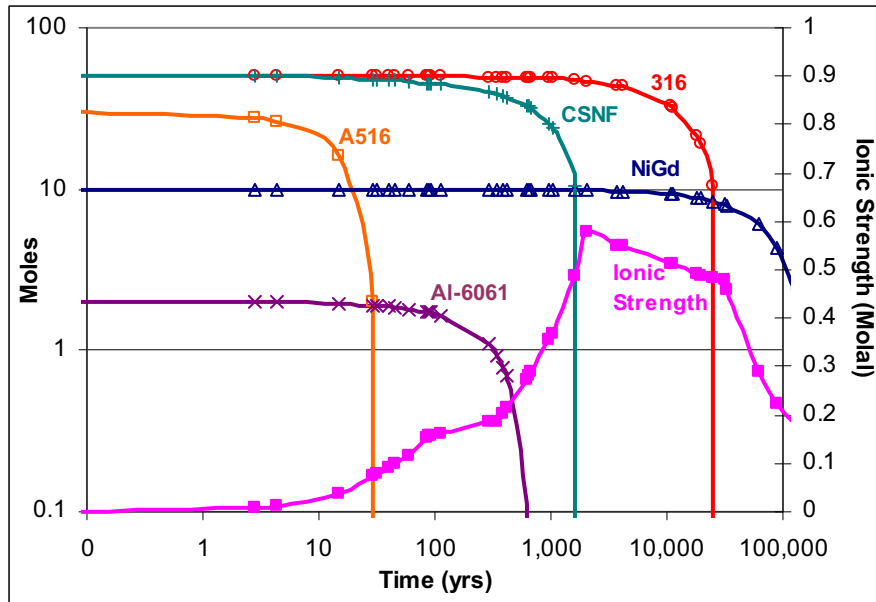
Figure 6-19 displays the ionic strength profiles for the CSNF vapor-influx base case. Under vapor influx conditions, the interior of the waste package is dry at the moment of waste package breach. With time, the humidity of the interior of the waste package will equilibrate to the ambient drift conditions and water will begin to adsorb or condense to the interior waste package components. The ratio of water to reactants is initially small and increases with time, so the ionic strength will initially be high and decrease with time as more water enters the system. This effect is not captured by the vapor influx EQ6 model because the batch reactor model considers the water needed for the model to be instantly available once the waste package breaches, rather than the water taking time to accumulate in the waste package (Section 6.3.1.1). However, this effect is accounted for in the ionic strength model abstraction (Section 6.10.2).



Source: CSNF Vapor Influx Model Output.xls in Appendix F.

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-18. CSNF Vapor-Influx Reactant Summary pH Profile

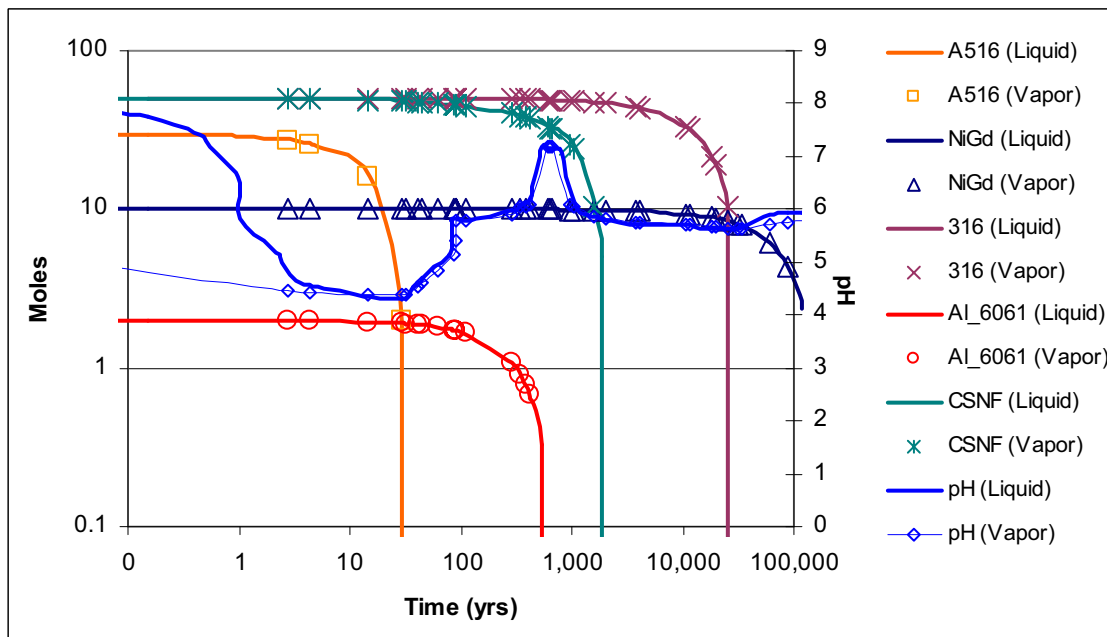


Source: CSNF Vapor Influx Model Output.xls (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-19. CSNF Vapor-Influx Reactant Summary and Ionic Strength

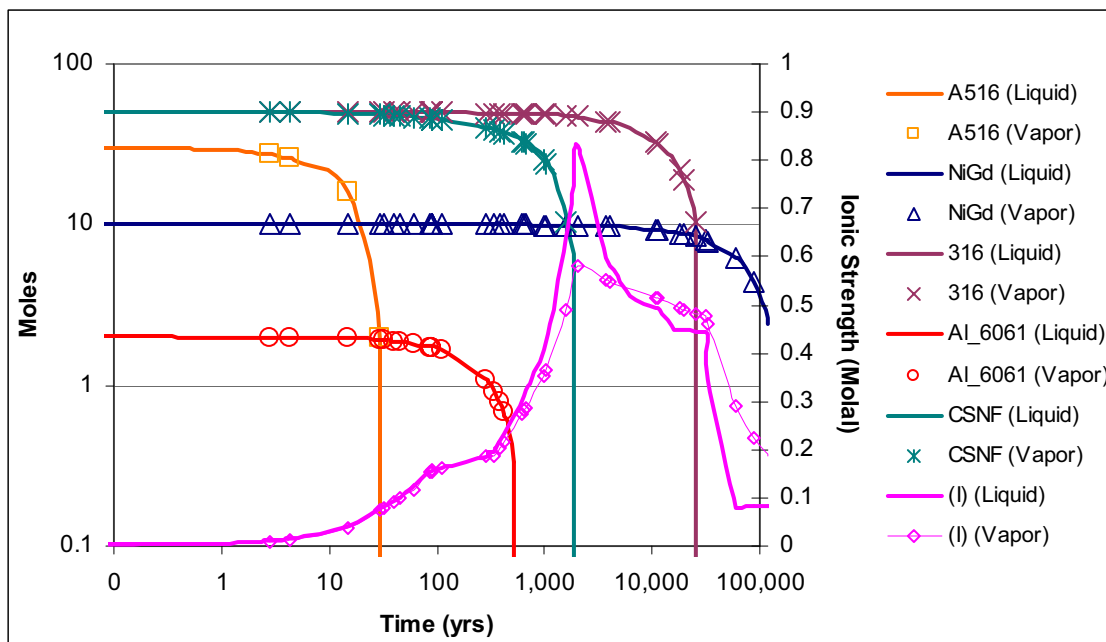
Figure 6-20 compares the vapor influx base-case model and the liquid-influx model pH values. This figure shows that the two cases are similar, except for the pH values at early times, due to the use of deionized water as opposed to J-13 well water in the vapor-influx case. Also, the pH is different at later times due to the accumulation of water described in Section 6.3.1.1.



Source: CSNF Water Influx Model Output.xls (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-20. Comparing the Vapor and Liquid-Influx Cases for CSNF pH



Source: CSNF Water Influx Model Output.xls (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

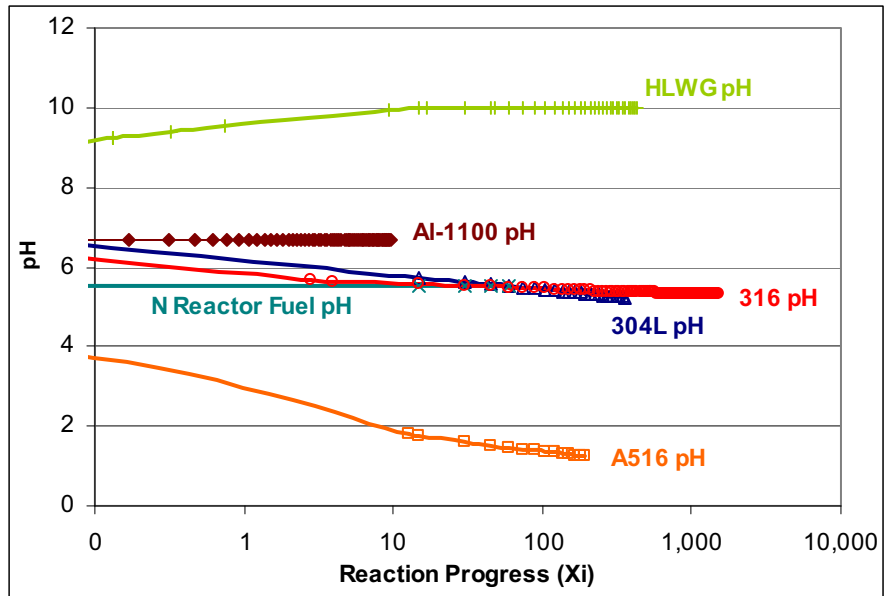
Figure 6-21. Comparing the Vapor and Liquid-Influx Cases for CSNF Ionic Strength

Figure 6-21 compares the ionic strength for the vapor-influx base-case model and the liquid-influx model. This figure shows that the two cases are similar, except that the ionic strength for the vapor-influx case is lower at later times due to the accumulation of water described in Section 6.3.1.1.

6.5.2.2 CDSP Output

Figure 6-22 displays the EQ6 output-pH profiles for the CDSP waste package components. The pH profiles in Figure 6-22 are plotted as a function of reaction progress (the EQ6 z_i (ξ) variable is moles reacted) rather than time elapsed. Plotting the results in this manner allows reactants' different degradation rates to be compared with each other. The CDSP waste package displays a much wider variation in pH compared to the CSNF waste package (Figure 6-17). HLWG generates high-pH conditions due the high concentrations of sodium and potassium. Aluminum Alloy 1100, Stainless Steel Types 316 and 304L, and N-Reactor fuel fall in the middle of the pH range at approximately 5.7 to 6.2. Carbon Steel Type A516 is a strong acid generator at pH values of 1.5 to 2.

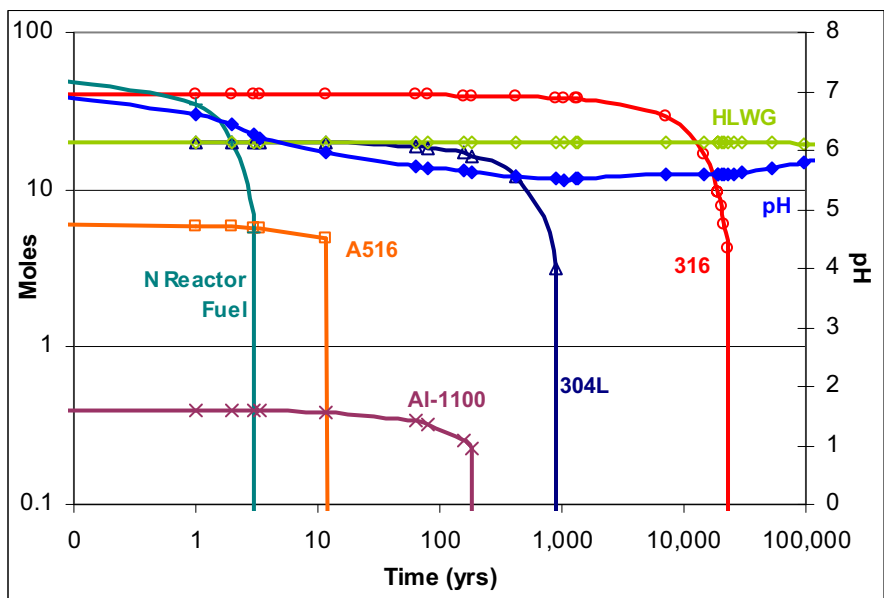
The CDSP base-case results, which include all of the waste package components, are displayed in Figure 6-23. The CDSP vapor-influx base case considers a reasonable representation of inputs by using median or most likely values: a flux of 1 L/yr, median rates for all of the alloys and fuels at $\log f\text{CO}_2 = -3$, and $\log f\text{O}_2 = -0.7$. The incoming water composition is deionized water and the runs were conducted at 25°C. The low end of the pH range in Figure 6-22 is not reproduced in the base-case results (Figure 6-23). The low pH generated by the Carbon Steel Type A516 (Figures 6-22 and 6-23) is mitigated due to the pH-buffering capacity that HLWG and N-Reactor fuel provide.



Source: CDSP Vapor Influx Model Output.xls (Appendix F).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-22. CDSP Single-Component Vapor Influx Results



Source: CDSP Vapor Influx Model Output.xls (Appendix F).

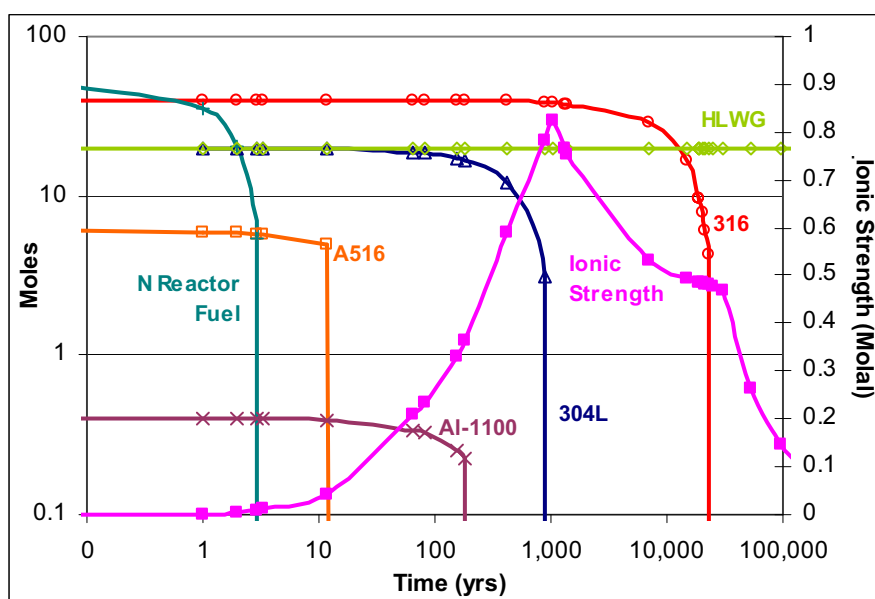
NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-23. CDSP Vapor-Influx Reactant Summary pH Profile

The base case includes HLWG and N-Reactor fuels as reactants; thus, the resulting chemistry of the reacted solutions ($\text{pH} \cong 6$) reflects interactions between the codisposed waste, the waste package components that are not spent nuclear fuel, the liquid phase water, and the atmosphere

in the voids in the waste package and drift. The above discussion addresses the subject matter of “Included” FEP 2.1.01.02.0B (Interactions between co-disposed waste) and indicates how that subject matter has been included and addressed within TSPA-LA. In addition, the fuel and metal alloys are kinetic reactants that undergo oxidation and precipitate corrosion products that are in contact with the liquid phase water, thereby having a first order influence on the chemistry. The above discussion addresses the subject matter of “Included” FEP 2.1.09.02.0A (Chemical interaction with corrosion products) and indicates how that subject matter has been included and addressed within TSPA-LA.

High-pH conditions are only realized in the single-component simulation for HLWG. In the base case, after the metal alloys have been exhausted, the pH starts to increase slowly due to the HLWG, but even at 100,000 years, pH is still below 6 because the glass dissolution rate is slow. HLWG is contained in Stainless Steel Type 304L canisters in which the steels will moderate the basic conditions generated by HLWG dissolution and maintain neutral-pH conditions inside the waste package, well past 20,000 years. The presence of schoepite, resulting from N-Reactor fuel degradation, and large quantity of steel in the waste package limits the potential of generating high-pH conditions in the CDSP waste package.



Source: *CDSP Vapor Influx Model Output.xls* (Appendix F).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-24. CDSP Vapor Influx Reactant Summary and Ionic Strength Profiles

Figure 6-24 displays the ionic strength profiles for the CDSP vapor-influx model. Under vapor influx conditions, the interior of the waste package is dry at the moment of waste package breach. With time, the humidity of the interior of the waste package will equilibrate to the ambient drift conditions and water will begin to adsorb or condense to the interior waste package components. The ratio of water to reactants is initially small and increases with time, so the ionic strength will initially be high and decrease with time as more water enters the system. This effect is not captured by the vapor influx EQ6 model, because the batch reactor model considers

the water needed for the model to be instantly available when the waste package breaches, rather than the water taking time to accumulate in the waste package (Section 6.3.1.1). However, this effect is accounted for in the ionic strength model abstraction (Section 6.10.2).

6.5.3 Conceptual Model Conclusions

The CSNF vapor-influx and liquid-influx models reacted solutions results reflect interactions between CSNF, waste package components, liquid-phase water, and the atmosphere in the voids in the waste package and drift. In addition, the reactants are kinetic reactants, the metal alloys undergo oxidation and precipitate corrosion products in contact with the liquid phase water (thereby, having a first order influence on the chemistry) and increased temperature was investigated. The above discussion addresses the subject matter of the following “Included FEPs,” and indicates how that subject matter has been included and addressed within TSPA-LA:

- 2.1.02.09.0A, Chemical effects of void space in waste package
- 2.1.09.01.0B, Chemical characteristics of water in the waste package
- 2.1.09.02.0A, Chemical interaction with corrosion products
- 2.1.09.06.0A, Reduction – oxidation potential in waste package
- 2.1.09.07.0A, Reaction kinetics in waste package
- 2.1.11.08.0A, Thermal effects on chemistry and microbial activity in the EBS
- 2.2.08.12.0B, Chemistry of water flowing into the waste package.

The CDSP vapor-influx and liquid-influx model runs include high-level radioactive waste and N-Reactor fuels as reactants; thus, the resulting chemistry of the reacted solutions reflects interactions between codisposed waste, the nonspent nuclear fuel waste package components, the liquid phase water, and the atmosphere in the voids in the waste package and drift. In addition, the reactants are kinetic (the fuel and metal alloys undergo oxidation and precipitate corrosion products in contact with the liquid phase water); thereby, having a first order influence on the chemistry. The above discussion addresses the subject matter of the following “Included FEPs,” and indicates how that subject matter has been included and addressed within TSPA-LA:

- 2.1.01.02.0B, Interactions between codisposed waste
- 2.1.02.09.0A, Chemical effects of void space in waste package
- 2.1.09.01.0B, Chemical characteristics of water in the waste package
- 2.1.09.02.0A, Chemical interaction with corrosion products
- 2.1.09.07.0A, Reaction kinetics in waste package
- 2.2.08.12.0B, Chemistry of water flowing into the waste package.

6.6 BATCH REACTOR SENSITIVITY ANALYSES

Sensitivity analyses were performed to:

1. Determine uncertainty ranges
2. Demonstrate how various inputs (waste package design configuration, water to reactant ratio) impact the model output.

The sensitivity analyses consider that variations in a model input will translate to the model output independent of the modeled scenario. For example, decreasing the corrosion rate of the steels used in the waste package for a specific fuel exposure–flux scenario has the effect of delaying the timing of a pH response. Therefore, it is expected that an equivalent model response will occur for a different fuel exposure–flux scenario.

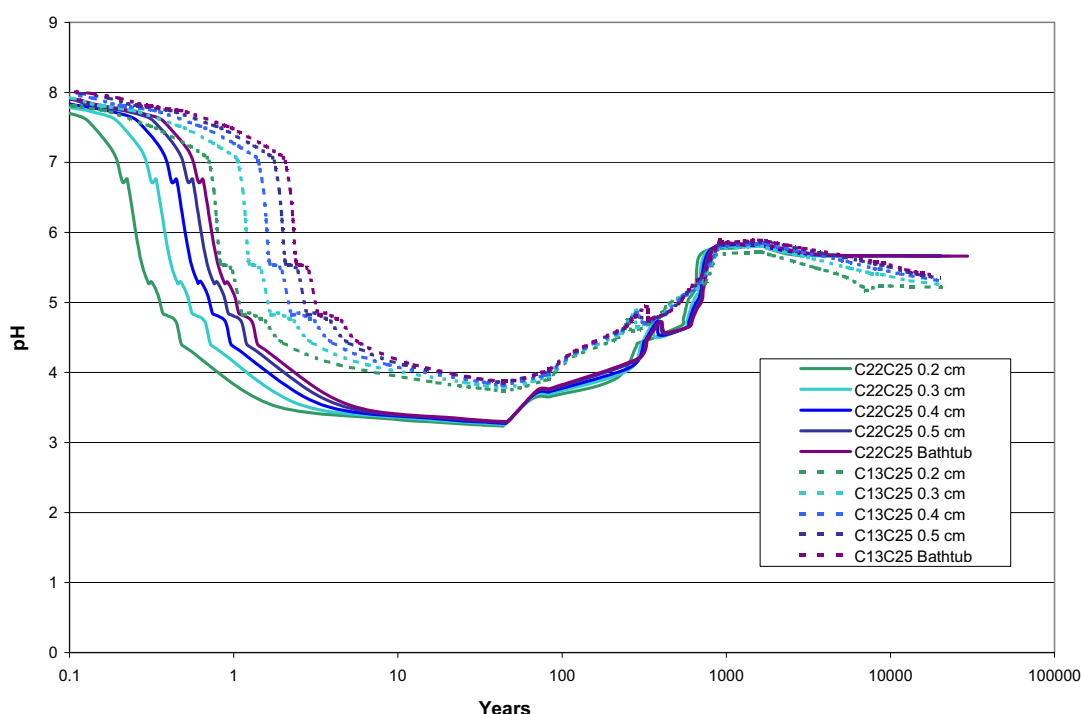
The sensitivities in this section are all compared to the liquid-influx base-case model described in Section 6.5. Based on Figures 6-20 and 6-21, which show similar results for the liquid-influx and vapor-influx models, the liquid-influx model base case can be considered representative of the vapor-influx model for the purposes of these sensitivities. The base case considers a reasonable representation of inputs by using median or most-likely values: a flux of 1 L/yr, median rates for all of the alloys and fuels, and 100% cladding exposure, at $\log f\text{CO}_2 = -3$, and $\log f\text{O}_2 = -0.7$. The incoming water composition is J-13, and the runs were conducted at 25°C.

Some of the sensitivities were not updated from Rev 03 of this report, so for these cases Cr(VI) was allowed to form instead of Cr(III). The source for the input to these sensitivities is documented in Rev 03 of this report, and the figures can be found in DTN: MO0411SPAMOICA.000 [DIRS 172244]. These sensitivities include Sections 6.6.1, 6.6.8, and 6.6.12. The rationale and impacts of using Cr(III) versus Cr(VI) are discussed in Section 6.6.3. The purpose of a sensitivity analysis is to observe the model response to varying certain inputs. For example, Section 6.6.1 examines different ratios of water to reactants and shows that increasing the ratio of water to reactants changes neither the minimum or maximum pH observed nor the shape of the pH profile, but increasing this ratio delays the timing of the minimum or maximum pH. This sort of trend is important in assessing which factors to include in the model abstraction. In this example, because the pH abstraction is not time dependent, and the ratio of water to reactants does not impact the range of pH, the ratio of water to reactants is not included in the model abstraction. If this sensitivity study were to be conducted using Cr(III) instead of Cr(VI), the series of Cr(III) results would show a more-moderate pH profile than the Cr(VI) results. However, the trends of interest—that increasing the ratio of water to reactants results in the same pH range and profile shape, but the timing of the pH response is delayed—would be the same for both the Cr(III) and the Cr(VI) sensitivities. Likewise, for the other two sensitivity analyses that use Cr(VI), the change in HLWG composition, and the 5-DHLW/DOE SNF waste package design, the trends of interest shown in the sensitivity analyses—that altering the HLWG composition has very little impact on pH, or that the 5-DHLW/DOE SNF waste package design does result in a lower pH, but this pH is still within the range of pH values predicted by the model abstraction—using Cr(VI) would also be supported by sensitivity analyses using Cr(III). This logic also applies to the trends observed in ionic strength results. For this reason, results using Cr(VI) are still credible for showing model response, and the use of the superceded DTN: MO0411SPAMOICA.000 [DIRS 172244] is justified. The sensitivity runs from Rev 03 were executed using the Ca-porewater, median seepage flux–fuel exposure case (filename: C22C25) at 25°C, with an equivalent set of runs executed for the codisposed case (filename: D23C25).

The sensitivity analyses in this section do not include the effects of the surface complexation model. The results of applying the surface complexation model to the batch reactor model are discussed in Section 6.9 with results shown in the plots.

6.6.1 Ratio of Water to Reactants

The following runs were conducted to see the impact of varying the ratio of water to reactants. The highest ratio is that of the “bathtub,” where all of the void space is filled with water. This ratio is decreased, and the impact on the pH and ionic strength are presented in Figures 6-25 and 6-26. The ratio of water to reactants is varied based on the surface area. For the “bathtub” case, if the liter of water were spread evenly over the scaled surface areas of the reactants, that layer of water would be approximately 0.58-cm thick for the CSNF waste package and 0.69-cm thick for the CDSP waste package. In this sensitivity analysis, this ratio of water to reactants is examined based on the thickness of the layer of water that would result from one liter of water spread over the scaled surface areas for each case. For more details on the scaling used in this sensitivity, see *CSNF.xls* or *CDSP.xls* in DTN: MO0411SPAMOICA.000 [DIRS 172244].



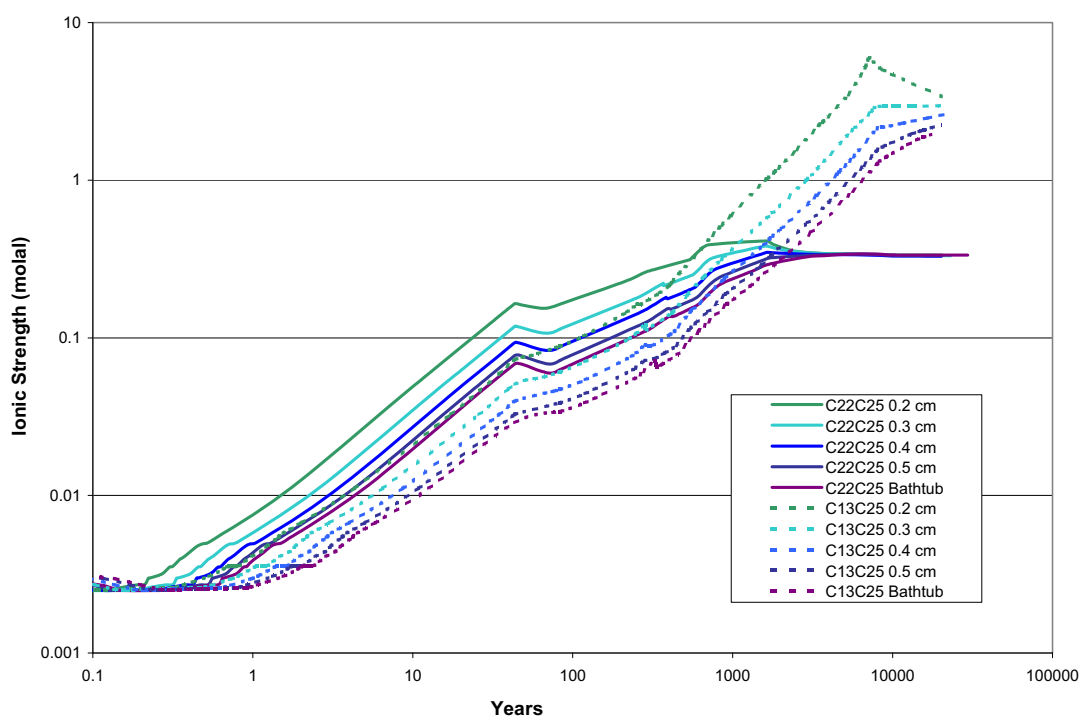
Source: *CSNF_Water Ratio Sensitivity.xls* (DTN: MO0411SPAMOICA.000 [DIRS 172244]).

Note: Time refers to time after waste package breach. This sensitivity uses Cr(VI) rather than Cr(III).

Figure 6-25. Ratio of Water to Reactants for CSNF: Impact on pH

Figures 6-25 and 6-27 show the ratio of water to reactants has little effect on the overall pH of the system for the CSNF and CDSP waste packages. The most notable effect is lower ratios of water-to-reactants see the pH values decrease earlier than the cases with a higher ratio, but the minimum and steady-state pH values are similar. This sensitivity uses Cr(VI) rather than Cr(III), but because all of these cases use the same oxidation state for chromium, Cr(VI), the important pH trends discussed above are a function of the quantity of water in the system and not the chromium oxidation state. The impacts of Cr(III) versus Cr(VI) are shown in Section 6.6.3.

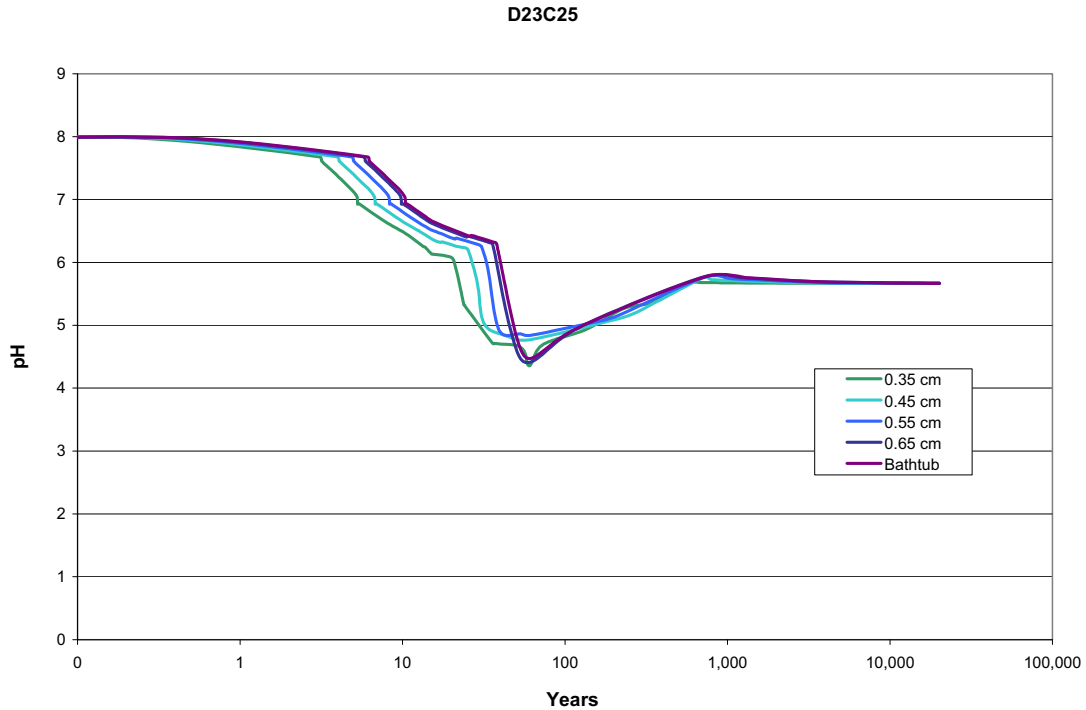
For the CSNF case, the water ratio sensitivity analysis also examined C13C25 that uses the lowest water flux with the highest fuel exposure (0.15 L/yr and 100% CSNF exposure) to maximize the impact to the chemistry. In addition, sensitivity also uses a case with the median seepage flux–fuel exposure case C22C25 at 25°C (the representative case used for the sensitivities from Rev 03). For all cases, the results also show the same trend in pH; the lower ratios of water to reactants see the pH values decrease earlier, but the minimum and steady-state pH values are similar. These results show that additional water in the system has minimal impact on pH, which addresses two points dealing with the quantity of water that were raised while discussing code implementation in Section 6.3.1.1. In the vapor-influx case, where water accumulates in the EQ6 model (as opposed to the implementation in TSPA-LA with no accumulation of water), the impacts of this accumulating water on pH are minimal. In addition, the immediate availability of water in the described batch reactor model also has minimal impact on the pH value of the system. Therefore, the difference between the batch reactor modeling, and the conceptualization of the vapor influx within TSPA-LA has minimal impact.



Source: CSNF_Water Ratio Sensitivity.xls (DTN: MO0411SPAMOICA.000 [DIRS 172244]).

Note: Time refers to time after waste package breach. This sensitivity uses Cr(VI) rather than Cr(III).

Figure 6-26. Ratio of Water to Reactants for CSNF: Impact on Ionic Strength

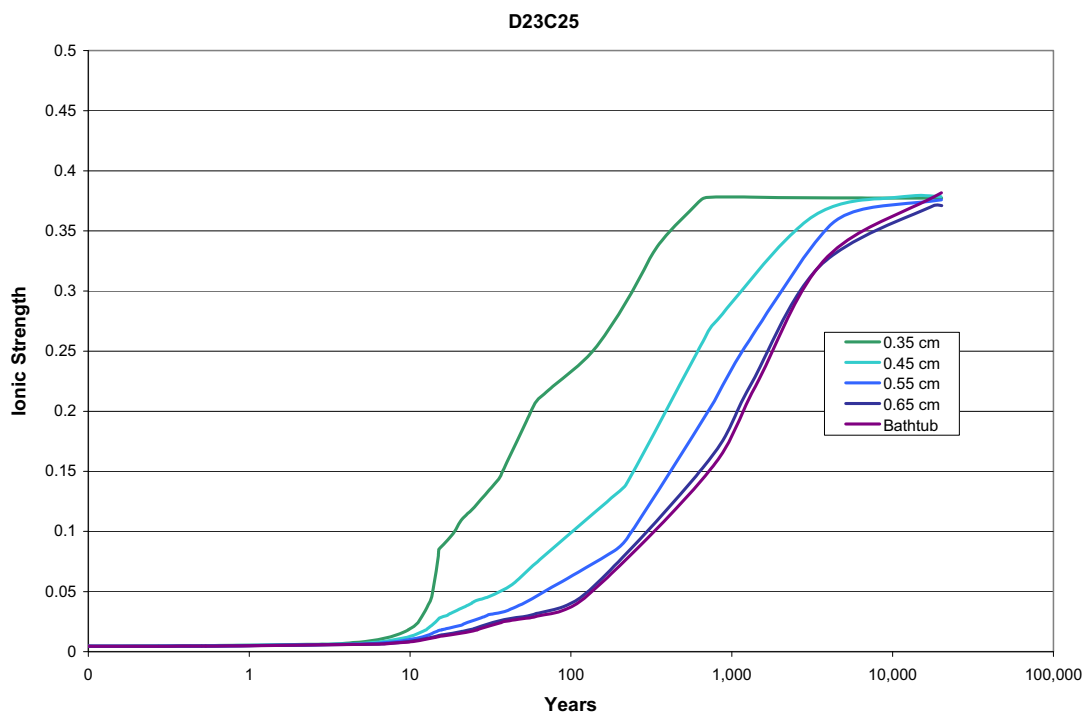


Source: *CDSP_Water Ratio Sensitivity.xls* (DTN: MO0411SPAMOICA.000 [DIRS 172244]).

Note: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

This sensitivity uses Cr(VI) rather than Cr(III).

Figure 6-27. Ratio of Water to Reactants for CDSP: Impact on pH



Source: *CDSP_Water Ratio Sensitivity.xls* (DTN: MO0411SPAMOICA.000 [DIRS 172244]).

Note: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

This sensitivity uses Cr(VI) rather than Cr(III).

Figure 6-28. Ratio of Water to Reactants for CDSP: Impact on Ionic Strength

The important trend shown in Figures 6-26 and 6-28 is the ratio of water to reactants affects the timing of the ionic strength response; lower ratios of water to reactants see the ionic strength increase earlier than the cases with a higher ratio, but the ionic strength maximum are similar. This sensitivity uses Cr(VI) rather than Cr(III); however, because all of these cases use the same oxidation state for chromium, Cr(VI), the ionic strength trends discussed above are due to the ratio of water to reactants, not the oxidation state of chromium, and are, therefore, still valuable.

The water ratios examined in this section use the base-case quantity of water as the minimum, and increase the ratio of water from there. If the quantity of water were decreased much lower than the base case, the ionic strength increases dramatically, well beyond the valid range of the model. For this reason, the ionic strength model includes additional uncertainty to account for the potential variation in water-to-reactants ratio. For the two points dealing with the quantity of water (raised in the code implementation discussion in Section 6.3.1.1), the effects of water accumulation on ionic strength must be addressed. The difference between the EQ6 batch reactor model and the conceptualization of the vapor-influx case within TSPA-LA is more significant for the ionic strength than for pH. This difference leads to lower modeled ionic strengths at later times (due to accumulating water), and the immediate availability of water in the batch reactor model leads to lower modeled ionic strengths at early times.

Low ionic strengths (<0.05 mol/L) are conservative for the TSPA-LA relative to colloid stability because colloids are stable and can transport radionuclides when the ionic strength is less than

0.05 mol/L (BSC 2004 [DIRS 170025]). High ionic strengths (>1 molal) are conservative for the TSPA-LA relative to the dissolved concentrations of radionuclides because the TSPA-LA increases the uncertainty applied to the *Dissolved Concentration Limits of Radioactive Elements* (BSC 2005 [DIRS 173873]), increasing the concentrations of radionuclides in solution. Ionic strengths between 0.05 and 1 molal are the least conservative to the TSPA-LA relative to the transport of radionuclides. Therefore, the uncertainty applied to the ionic strength should be broad enough to encompass extreme ionic strengths resulting in an overall representative output to the TSPA-LA that does not minimize risk.

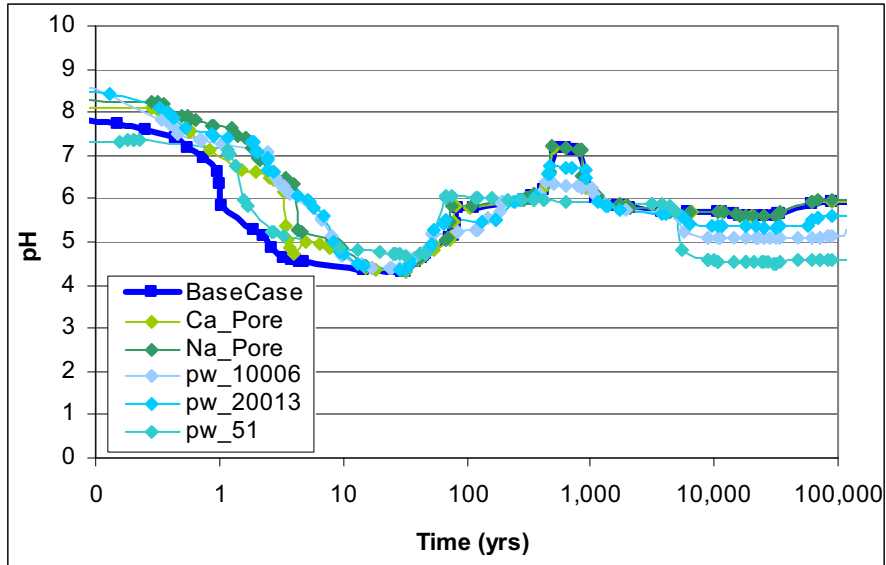
The above discussion addresses the subject matter of “Included” FEP 2.1.02.09.0A (Chemical effects of void space in waste package) and indicates how that subject matter has been included and addressed within TSPA-LA.

6.6.2 Thermally Perturbed Fluids

This section uses three thermally perturbed fluid compositions abstracted in *Engineered Barrier System: Physical and Chemical Environment Model* (BSC 2004 [DIRS 169860], Section 6.6) as input for the in-package chemistry model.

The three waters have different carbon dioxide fugacity ($f\text{CO}_2$) and temperatures (Table 4-3: (pw_51 is 91.8°C, pw_10006 is 56.2°C, and pw_20013 is 40.2°C). The EQ6 runs were set up to reflect increased temperature and $f\text{CO}_2$, and CSNF (Table 6-7) and HLWG (Table 6-8) kinetic rate constants were recalculated at the new temperatures. EQ6 runs were executed at these conditions using the thermally perturbed seepage compositions; Na-porewater and Ca-porewater were plotted with the results for comparative purposes.

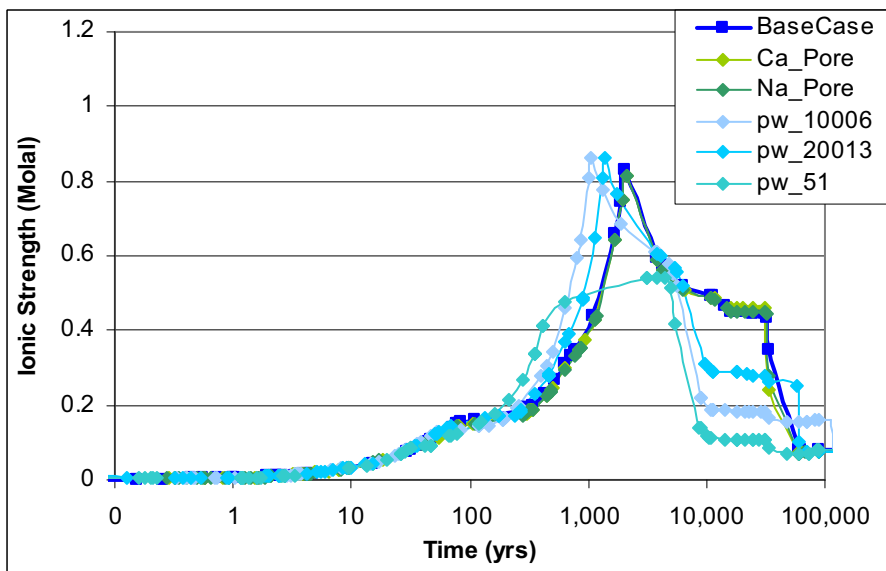
Figures 6-29 and 6-30 are sample outputs and show the results for CSNF, and Figures 6-31 and 6-32 show the results for CDSP. The simulation results for the various water compositions are in close agreement for each case, even though their initial compositions were different, which implies that incoming water composition has little influence over the in-package pH and ionic strength.



Source: CSNF Liquid Influx Model Output.xls (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

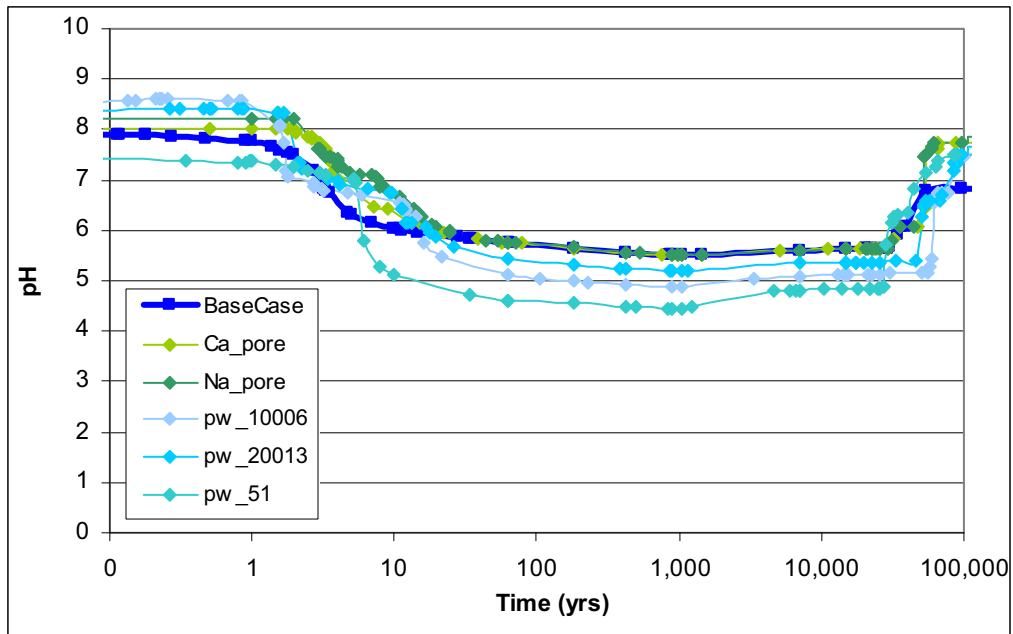
Figure 6-29. pH Profiles for Thermally Perturbed Compositions for CSNF



Source: CSNF Liquid Influx Model Output.xls (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

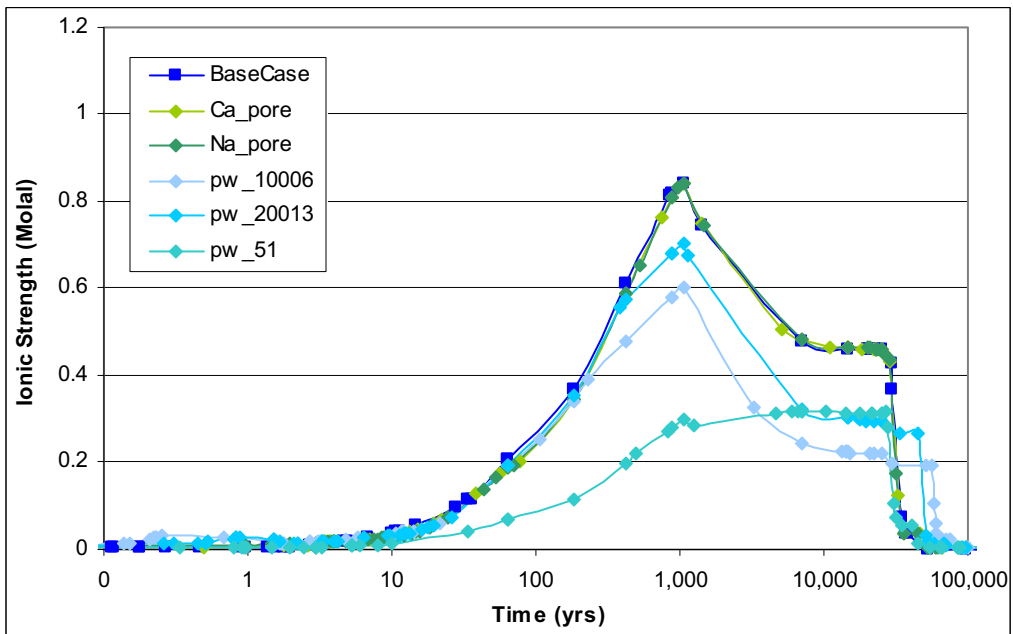
Figure 6-30. Ionic Strength Profiles for Thermally Perturbed Composition for CSNF



Source: *CDSP Liquid Influx Model Output.xls* (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-31. pH Profiles for Thermally Perturbed Compositions for CDSP



Source: *CDSP Liquid Influx Model Output.xls* (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-32. Ionic Strength Profiles for Thermally Perturbed Composition for CDSP

It is expected that surface complexation, which will occur during the first 600 years (Section 6.9), will have little impact on the conclusions drawn regarding thermally perturbed waters. This is because variations in the bulk fluid composition would be negligible for elements originating in the waste package, and only slight variations are expected for elements peculiar to the water composition; therefore, these compositional variations are anticipated to have no impact on the results of the model.

The above discussion addresses the subject matter of "Included" FEP 2.2.08.12.0B (Chemistry of water flowing into the waste package) and indicates how that subject matter has been included and addressed within the TSPA-LA.

6.6.3 Oxidation State of Chromium

The element chromium (Cr) is a major constituent of several of the stainless steels and alloys used in the construction of the CSNF and CDSP waste packages (Table 4-7). A sensitivity analysis was performed to determine the influence of the Cr oxidation state (kinetic control), on the in-package pH.

Revision 03 of the in-package chemistry model report allowed complete oxidation of chromium to Cr(VI), but *Engineered Barrier System: Physical and Chemical Environment [Model]* (BSC 2004 [DIRS 169860], Section 6.8.2), and the current version of the in-package chemistry model report limit the oxidation of chromium to Cr(III). Justification for the selection of the Cr(III) oxidation state from *Engineered Barrier System: Physical and Chemical Environment [Model]* (BSC 2004 [DIRS 169860]), which is also applicable to this model, is provided below:

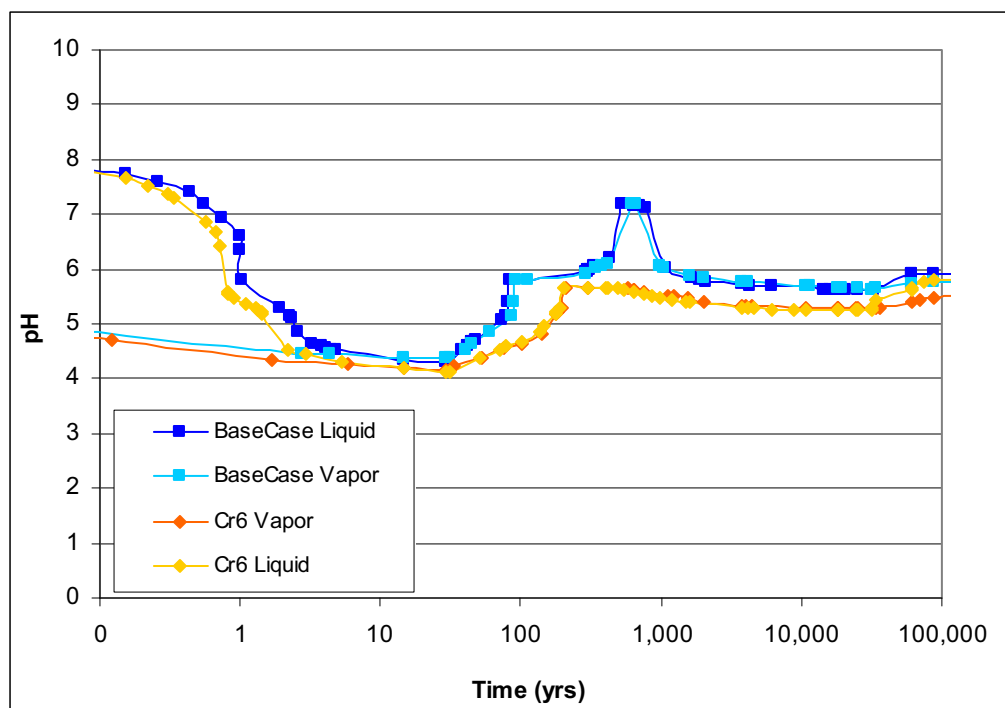
Selection of Cr(III) over that of the more soluble Cr(VI) species is based on experimentally observed corrosion products, and on the kinetics and conditions required to obtain the fully oxidized Cr(VI) state (Smith and Purdy 1995 [DIRS 162976]). Smith and Purdy's examination of the actual chromium speciation as a result of corrosion of 316L stainless steel demonstrated a predominance of the less soluble Cr(III) species, except under the conditions of hot concentrated nitric acid (111°C and >7 molal HNO₃, see Smith and Purdy 1995, Figure 6).

The choice of Cr(VI) in Rev 03 had been based on thermodynamic equilibrium, as described in Section 6.3.1.1, for the purpose of modeling the most extreme chemistries (Cr(VI) lead to higher ionic strengths and lower pH values). The choice of Cr(VI) was also based on the predominance of manganese as a component of the steels (Table 4-7), which when oxidized to either MnO₂ (pyrolusite), or to MnOOH (manganite) (Eary and Rai 1987 [DIRS 105780]; Johnson and Xyla 1991 [DIRS 105878]), tends to favor the Cr(VI) oxidation state.

Results are shown for pH and ionic strength for Cr(III) and Cr(VI) for CSNF in Figures 6-33 and 6-34. The pH and ionic strength results for CDSP are shown in Figures 6-35 and 6-36. The plots showing pH values indicate the Cr(III) oxidation state has more moderate pH values. The figures show the Cr(III) system significantly reduces the ionic strength. The reason for the decreased ionic strength is the presence of several Cr(III) solids in the database that precipitate and remove chromium from solution.

As shown in Figures 6-33 and 6-35, the oxidation state of chromium has a significant impact on pH, with the Cr(VI) case resulting in a much lower long term pH due to the formation of chromic acid from the Cr(VI) released from the degradation of the stainless steels and the NiGd. When using Cr(VI), there is a strong effect on the pH for the Stainless Steel Type 304L single-component run; the differences between the pH produced by the two stainless steel types are due to the chromium-to-nickel ratio in the alloys, as shown in *CDSP_Stainless Steel pH Cr_Ni ratio.xls* (Appendix F \Spreadsheets). For molar ratios of chromium to nickel greater than or equal to two, as in the case of Stainless Steel Type 304L, the pH of the solution after 15 moles have reacted is less than two. For chromium-to-nickel ratios below two, as in the case of the Stainless Steel Type 316, the pH is above five. For the Cr(III) system, the ratio of chromium-to-nickel is lower than for the Cr(VI) system, because the Cr(III) in solution precipitates as a solid, so the extreme low pH is never realized (Figure 6-22).

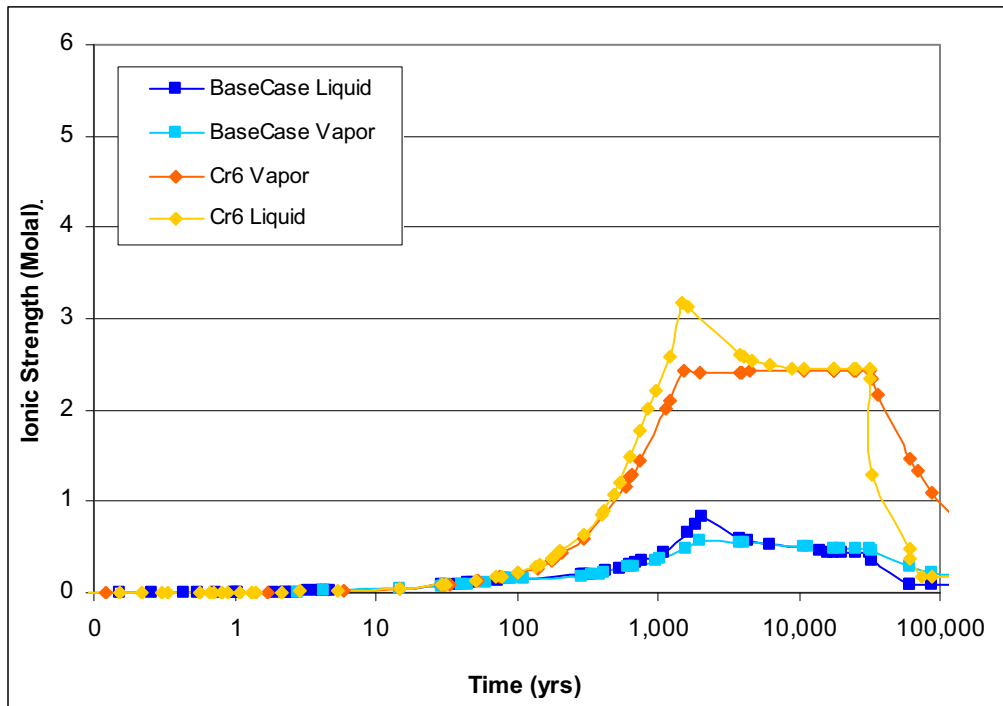
The above discussion addresses the kinetics of the Cr species that is the subject matter of “Included” FEP 2.1.09.07.0A (Reaction kinetics in waste package) and indicates how that subject matter has been included and addressed within TSPA-LA.



Source: *CSNF Liquid Influx Model Output.xls* (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

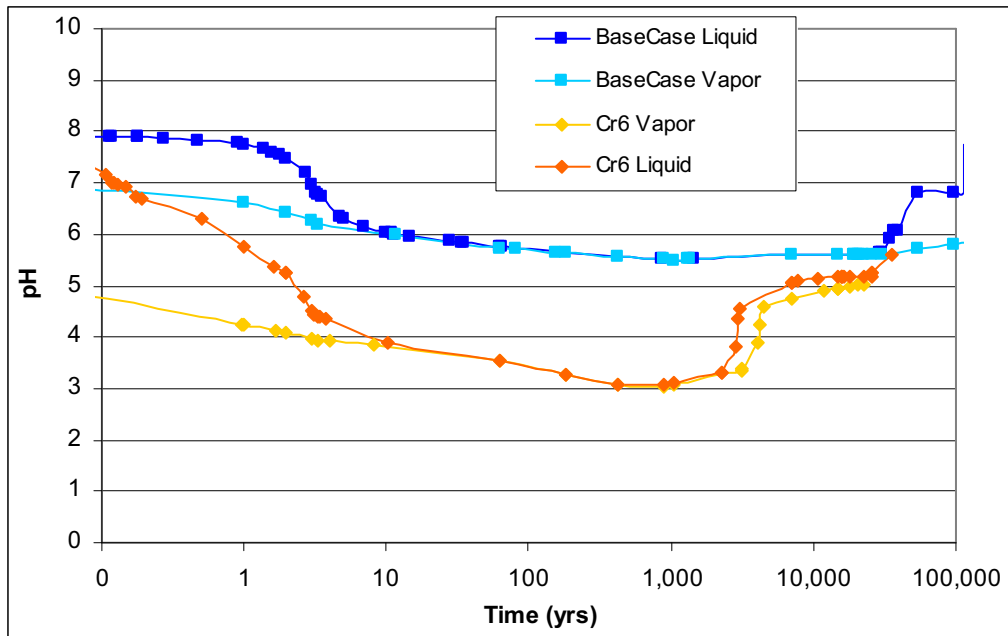
Figure 6-33. Cr(III) versus Cr(VI) for CSNF: Impact on pH



Source: CSNF Liquid Influx Model Output.xls (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

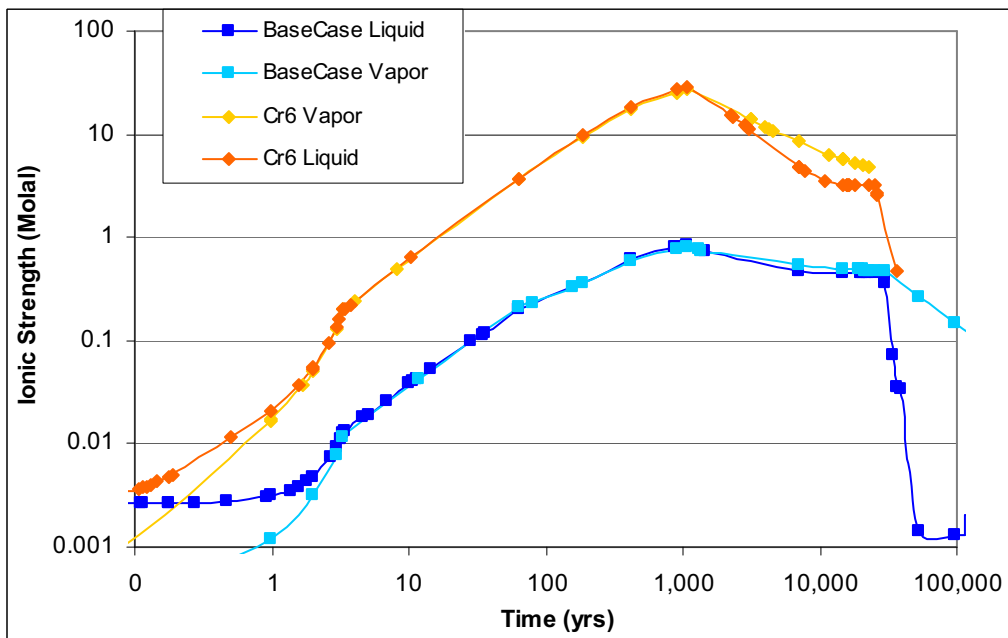
Figure 6-34. Cr(III) versus Cr(VI) for CSNF: Impact on Ionic Strength



Source: *CDSP Liquid Influx Model Output.xls* (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-35. Cr(III) versus Cr(VI) for CDSP: Impact on pH



Source: *CDSP Liquid Influx Model Output.xls* (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

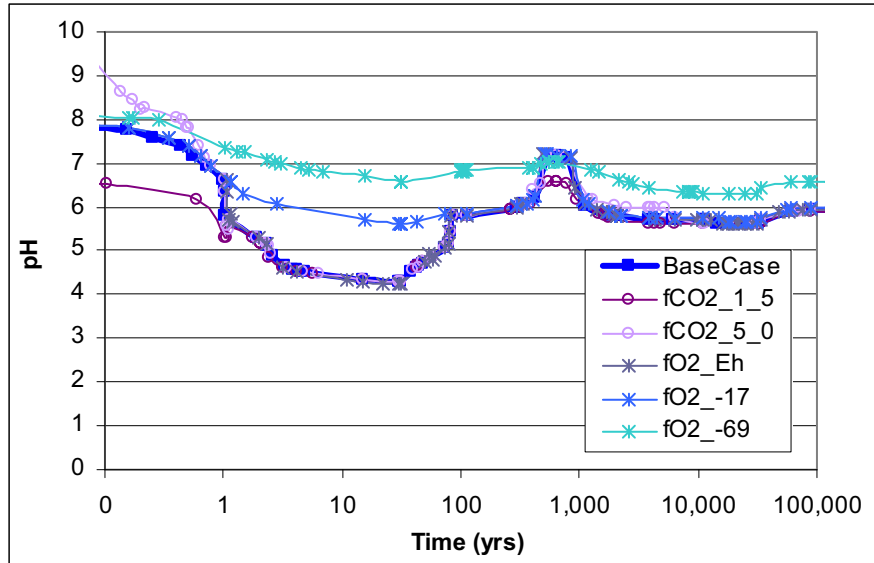
Figure 6-36. Cr(III) versus Cr(VI) for CDSP: Impact on Ionic Strength

The ionic strength for the Cr(VI) CDSP case is high, higher than the range of validation for the thermodynamic database; therefore the results of the Cr(VI) cases should not be used quantitatively, but are only presented here for comparison purposes. The reason for the high ionic strength is that there are no stable Cr(VI) solid species in the thermodynamic database, so all of the chromium that comes out of the stainless steels remains in solution, driving up the ionic strength.

6.6.4 Carbon Dioxide and Oxygen Fugacity

The sensitivity studies in this section vary the $\log f\text{CO}_2$ from -5.0 to -1.5 and the $\log f\text{O}_2$ from -6.9 to -0.7. Because EQ3/6 is a thermodynamic code, to actually model conditions other than fully oxidizing, the $\log f\text{O}_2$ must be set to a very low value. The CSNF rate constants used in this sensitivity are from Table 6-7, which were calculated using an oxygen partial pressure of 0.2 atmospheres, even though calculating the rates at lower $f\text{O}_2$ values would have resulted in lower rates or even non-existent rates (meaning that no degradation occurs). However, the purpose of this sensitivity is to examine the effects of lower $f\text{O}_2$ on the system, and the degradation of the CSNF has a strong influence on the chemistry of the system, so the faster rates were retained. For total carbonate concentrations less than 2×10^{-4} molar, the recommended CSNF degradation rate (BSC 2004 [DIRS 169987]) is to use Equation 4-1 with the total carbonate concentration set to 2×10^{-4} molar. This rate was compared to the rate that would be calculated by EQ6 using Equation 4-1 with lower total carbonate concentrations, and the two rates were similar, with the EQ6 rate slightly higher (as shown in Appendix F, *Total Carbonate and Eh Abstractions.xls* in Appendix F). The code calculates the rate of the CSNF based on the pH and total carbonate of the system for each time step. Since the total carbonate is above the 2×10^{-4} molar limit for the majority of the run, this sensitivity uses the same transition state theory formalism (described in Section 4.1.1) as the base case (rather than trying to decouple the CSNF rate from the total carbonate for the brief period when the total carbonate drops below 2×10^{-4} molar).

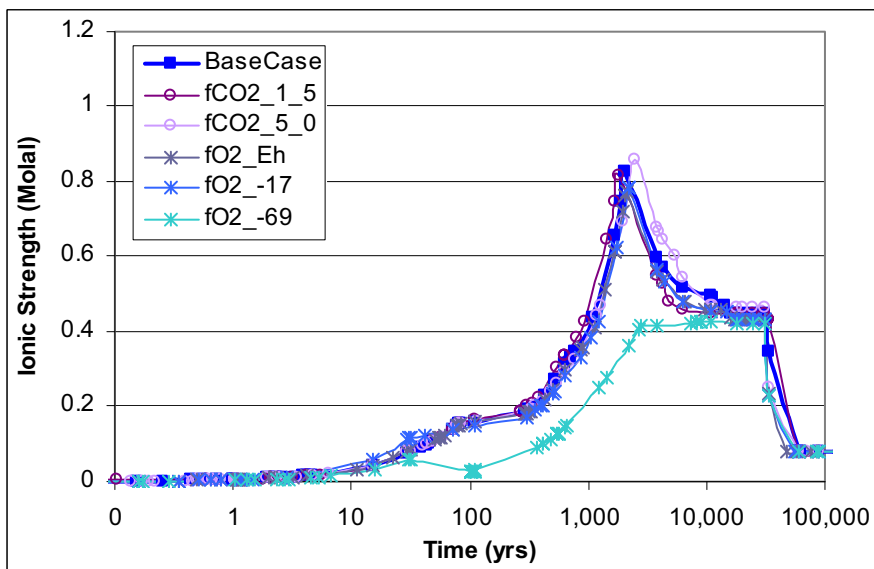
The following plots (Figures 6-37 through 6-40) show the results of varying carbon dioxide fugacity on the pH and ionic strength.



Source: CSNF Liquid Influx Model Output.xls (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

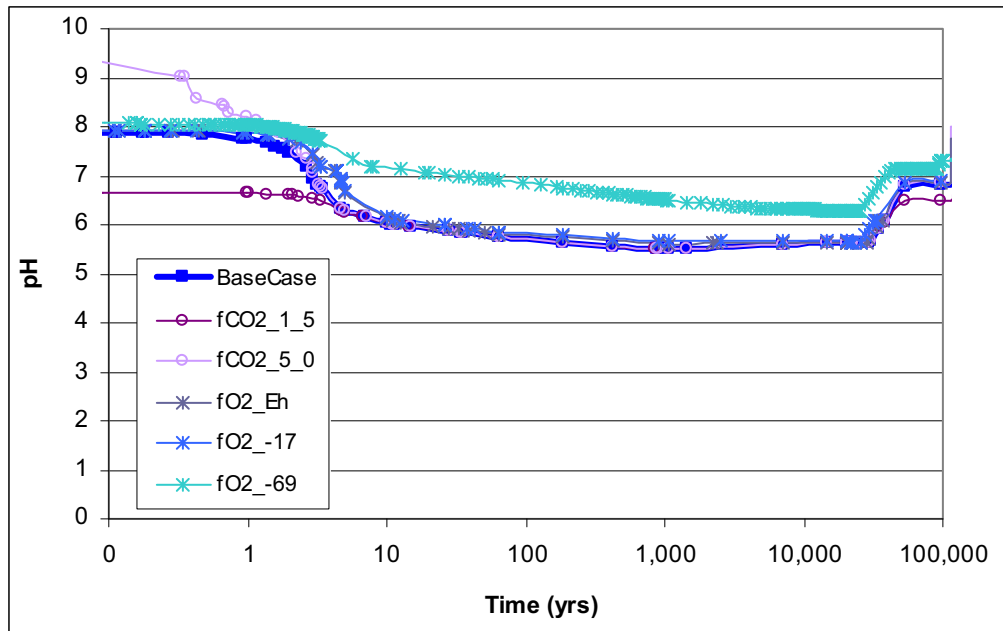
Figure 6-37. Variation of $f\text{CO}_2$ and $f\text{O}_2$ for CSNF: Impact on pH



Source: CSNF Liquid Influx Model Output.xls (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

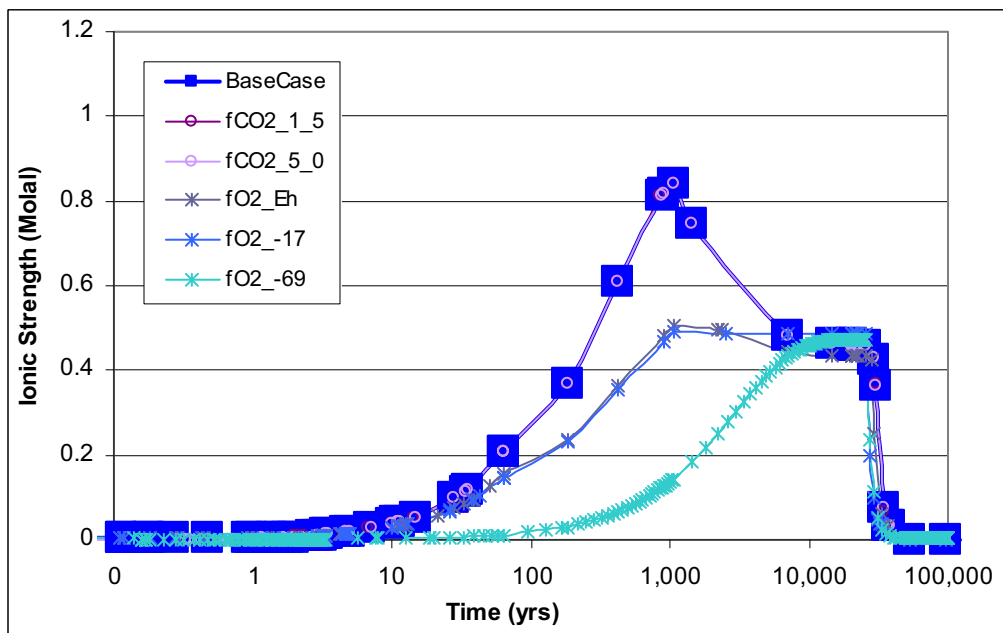
Figure 6-38. Variation of $f\text{CO}_2$ and $f\text{O}_2$ for CSNF: Impact on Ionic Strength



Source: *CDSP Liquid Influx Model Output.xls* (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-39. Variation of $f\text{CO}_2$ and $f\text{O}_2$ for CDSP: Impact on pH



Source: *CDSP Liquid Influx Model Output.xls* (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-40. Variation of $f\text{CO}_2$ and $f\text{O}_2$ for CDSP: Impact on Ionic Strength

The results show for CSNF and CDSP, neither pH nor ionic strength is affected by increasing $\log f\text{CO}_2$ to a value of -1.5 . However, decreasing $\log f\text{CO}_2$ to -5.0 results in a high initial pH

The high pH for the $\log f\text{CO}_2 = -5.0$ case is not considered, because the initial high pH is only a function of the solution being equilibrated with that low $f\text{CO}_2$, and is short lived (one year) and quickly overwhelmed by the corrosion of the waste package internals.

The lack of variation in system pH as the carbon dioxide fugacity was increased (up to $\log f\text{CO}_2 = -1.5$) renders it possible to use pH as a means of calculating total carbonate (Section 6.10.5) over a wide range of $f\text{CO}_2$ values. In the surface complexation model (Section 6.8), the CSNF pH distribution is cast as a function of $f\text{CO}_2$; thus, allowing the formation of carbonate surface complexes. As the carbon dioxide is fixed at a given value in the batch reactor modeling runs, the carbonate species in the bulk in-package fluid will reequilibrate in accordance with the relationships provided in Section 6.10.5.

The results shown in Figures 6-37 and 6-39 show that using the adjusted Eh ($Eh = 1.1 - 0.0592 \times \text{pH}$) from *Dissolved Concentration Limits of Radioactive Elements* (BSC 2005 [DIRS 173873]) provides results similar to the base case. Eh was calculated at the pH value of the incoming water, applied to the entire run, and did not vary with pH. The results of this sensitivity show similar pH and ionic strength results. More-significant decreases in the $f\text{O}_2$ (i.e., lower than the adjusted-Eh value) tend to raise pH and lower ionic strength; however, all of the results shown in Figures 6-37 through 6-40 are within the range of values provided to TSPA-LA.

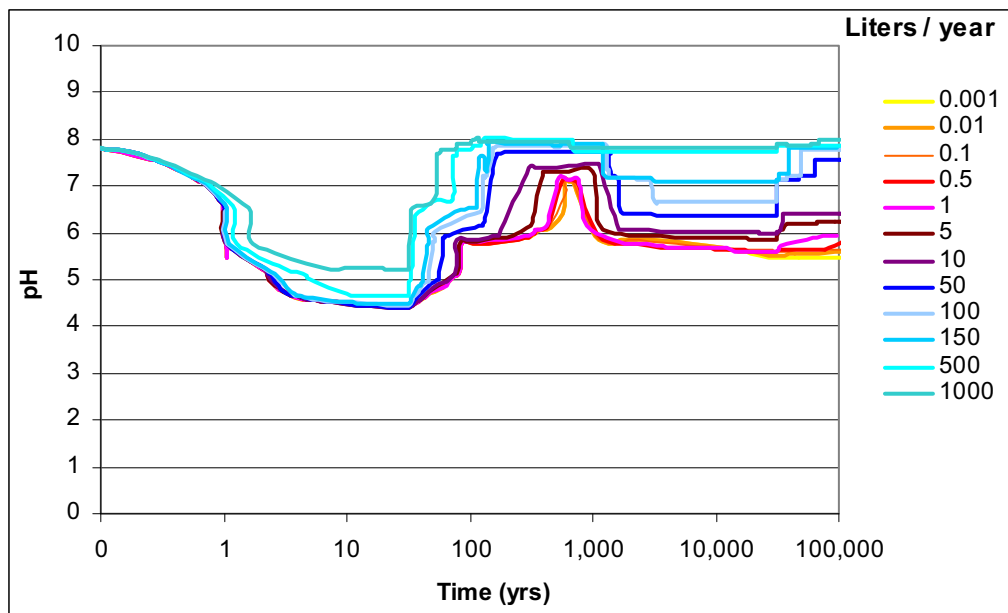
These sensitivities vary the composition of air in contact with the waste package internals. In addition, this sensitivity examines various states of kinetic equilibrium with the atmosphere by varying the $f\text{CO}_2$ and $f\text{O}_2$. The results of these sensitivities are used to broaden the range of applicability of this model and used in the model abstraction.

The above discussion addresses the subject matter of “Included” FEPs 2.1.02.09.0A (Chemical effects of void space in waste package), 2.1.09.06.0A (Reduction-oxidation potential in waste package), and 2.1.09.07.0A (Reaction kinetics in waste package) and indicates how that subject matter has been included and addressed within TSPA-LA.

6.6.5 Water Flux

The flux of water through a waste package has direct impacts on the chemistry of the water exiting the package. The lower the influx rate the longer the residence time of the water in the package and greater the extent of the reaction between the water and the waste package components. There must exist an upper limit to flux at which there is virtually no reaction between the water and the waste package components.

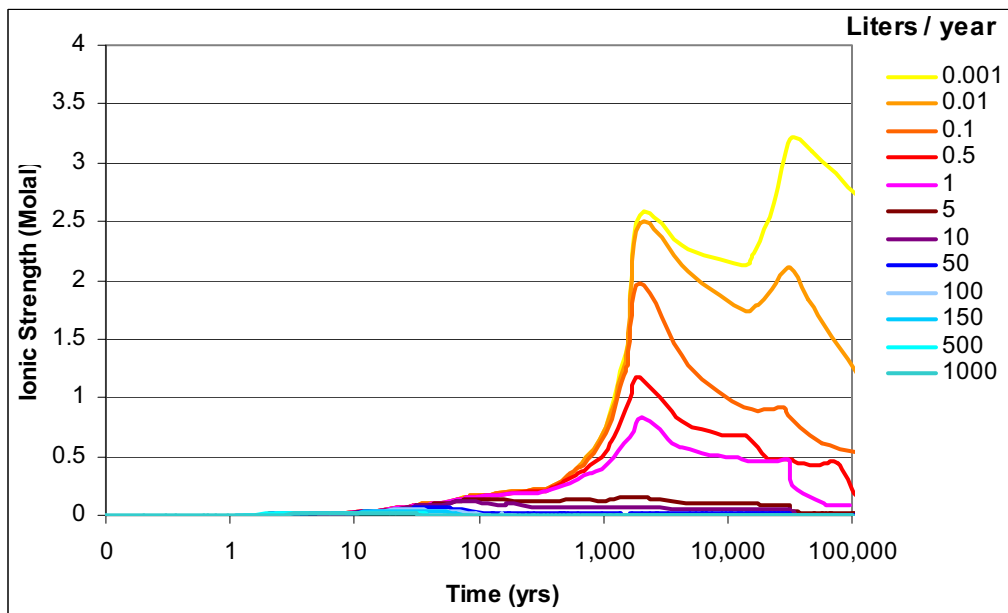
This was confirmed as displayed in the following plots (Figures 6-41 through 6-44) where the pH and ionic strength approach that of the incoming fluid as the flux was increased. As the flux increases, the chemistry stabilizes in a shorter time frame and the run terminates.



Source: (I) *Abstraction CSNF Liquid Influx.xls* (Appendix F /Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

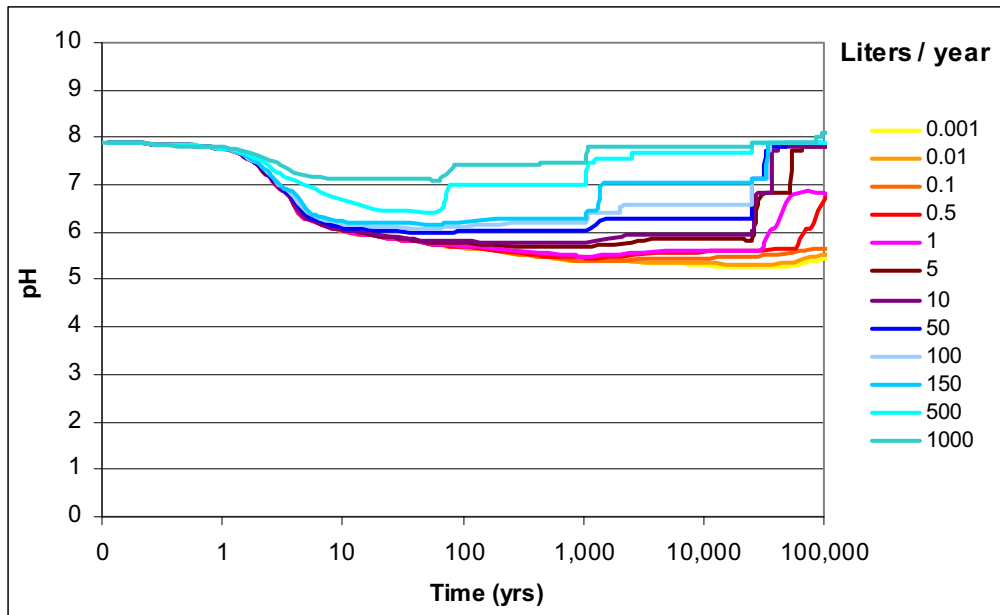
Figure 6-41. Effect of Water Flux on pH for CSNF



Source: (I) *Abstraction CSNF Liquid Influx.xls* (Appendix F /Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

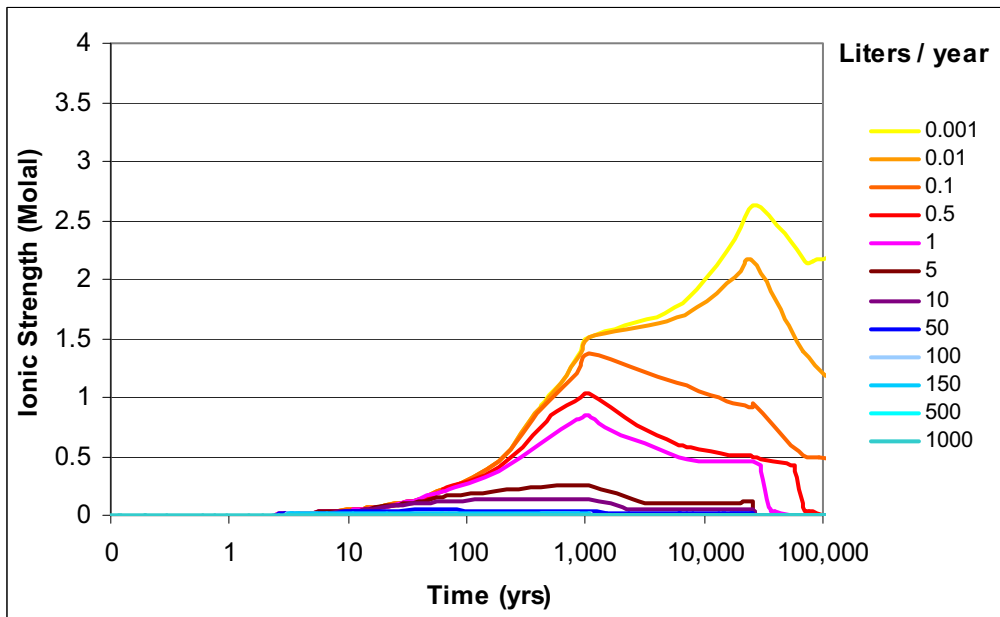
Figure 6-42. Effect of Water Flux on Ionic Strength for CSNF



Source: (I) Abstraction CDSP Liquid Influx.xls (Appendix F /Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-43. Effect of Water Flux on pH for CDSP

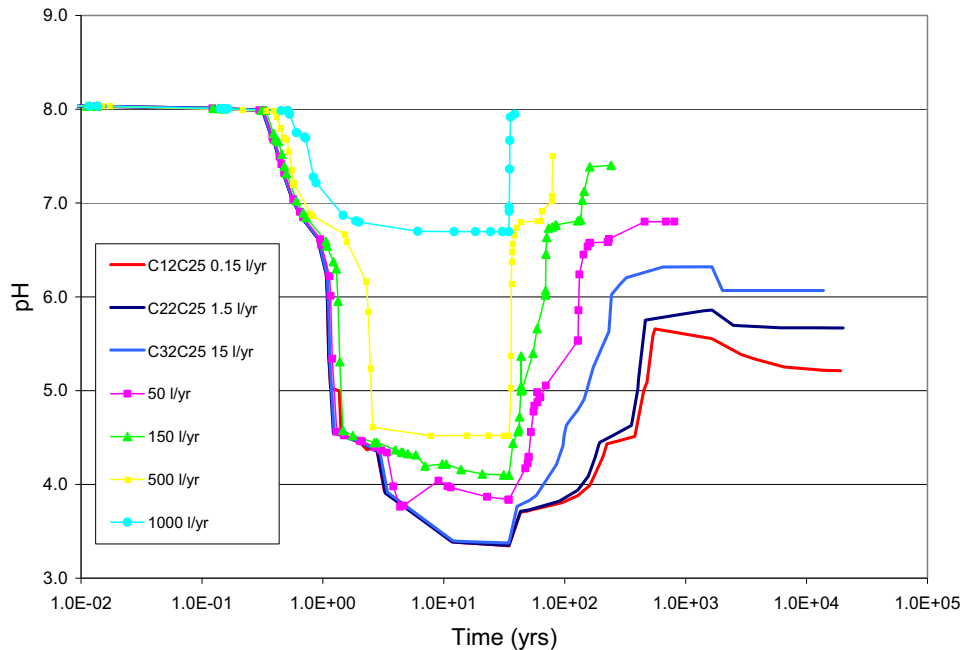


Source: (I) Abstraction CDSP Liquid Influx.xls (Appendix F /Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-44. Effect of Water Flux on Ionic Strength for CDSP

Because influx has such an impact on the in-package chemistry, the ionic strength abstraction is determined as a function of flux. Section 6.10.2 develops the ionic strength abstraction varying the water flux over the full range for the liquid influx and vapor-influx cases. For comparison and historical purposes, Figure 6-45 shows the Cr(VI) pH history from Rev 03 of this document, for various fluxes for the CSNF waste package, and the results show the same trend seen in the Cr(III).



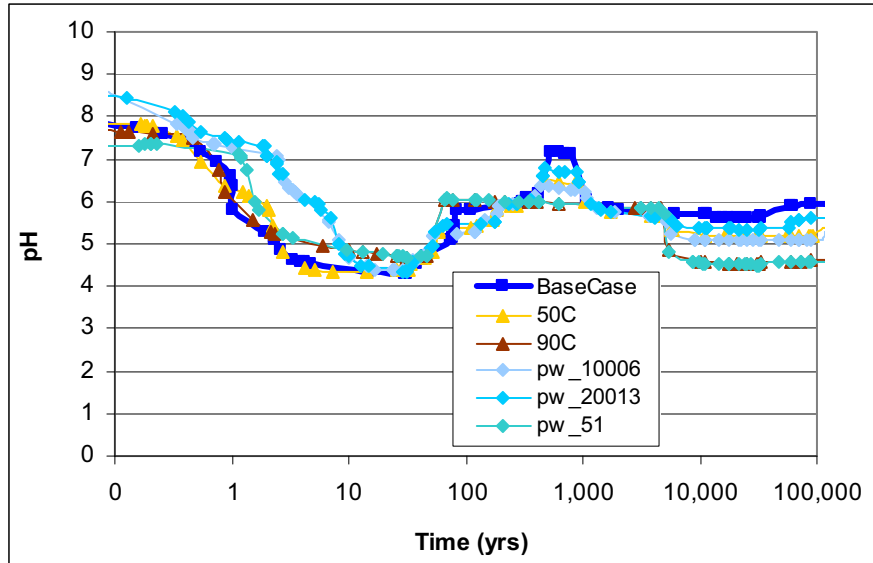
Source: *CSNF_Sensitivity.xls* (DTN: MO0411SPAMOICA.000 [DIRS 172244]).

Note: Time refers to time after waste package breach. This sensitivity uses Cr(VI) rather than Cr(III).

Figure 6-45. Effect of Water Flux on pH for CSNF (using Cr(VI))

6.6.6 Temperature

The temperature of the waste package receiving seepage could vary significantly depending on the timing of waste package breach and seepage ingress. In addition to the sensitivity analysis that examines the type of fluids expected at high temperatures (the thermally perturbed fluid compositions), a set of simulations was performed at 50°C and 90°C with the base-case waters. Also included in this sensitivity are the thermally perturbed waters, since they are high-temperature runs (pw_51 is 91.8°C, pw_10006 is 56.2°C, and pw_20013 is 40.2°C). The coefficients for the CSNF- and HLWG-rate expressions were recalculated at the temperature of the run (Tables 6-7 and 6-8).



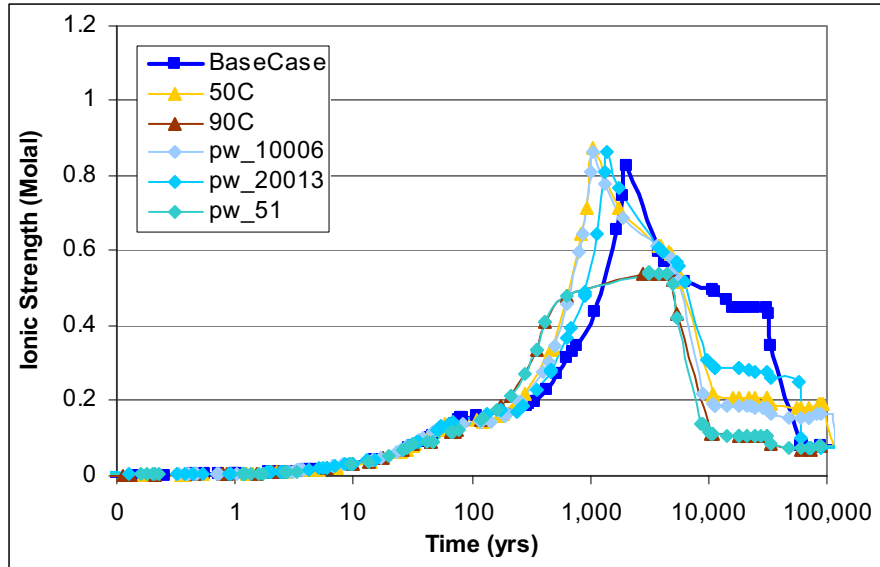
Source: CSNF Liquid Influx Model Output.xls (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-46. Effect of Increasing Temperature on the pH for CSNF

Figure 6-46 shows increasing the temperature lowers late-time pH for the CSNF runs. One reason for the decreasing pH values at high temperature is the decrease in the dissociation constant of water with increased temperature.

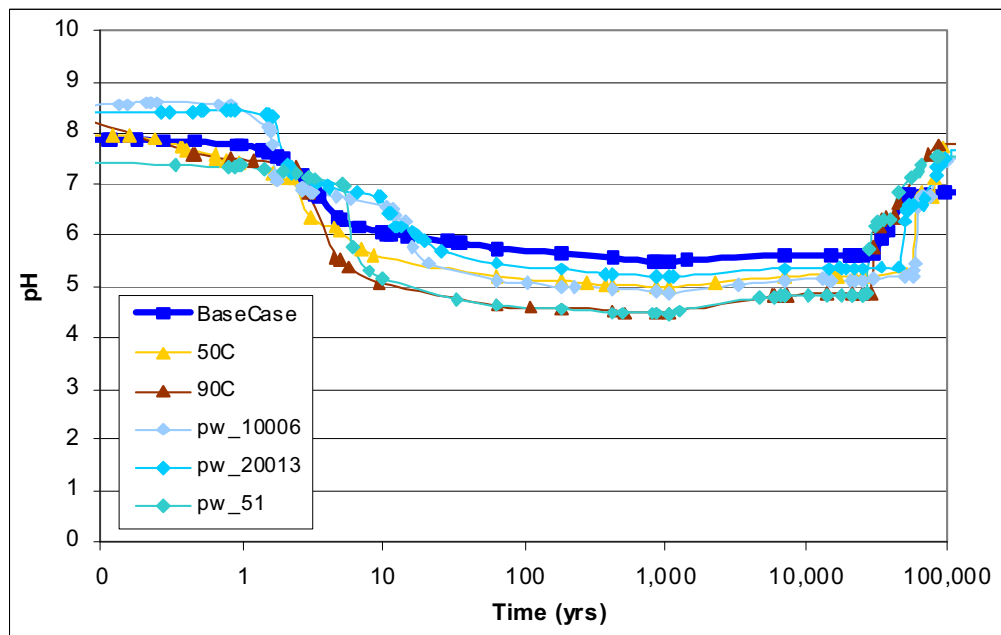
Figure 6-47 shows the ionic strength results of EQ6 simulations at higher temperatures for CSNF waste packages. The results plotted in Figure 6-47 show temperature had a minor influence on ionic strength, and this effect is even more pronounced for high temperatures. Although evaporation from the interior of the waste package is not expected to be great, due to high humidity in the waste package interior, evaporation could still influence the ionic strength as temperature increases.



Source: CSNF Liquid Influx Model Output.xls (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-47. Effect of Increasing Temperature on Ionic Strength for CSNF



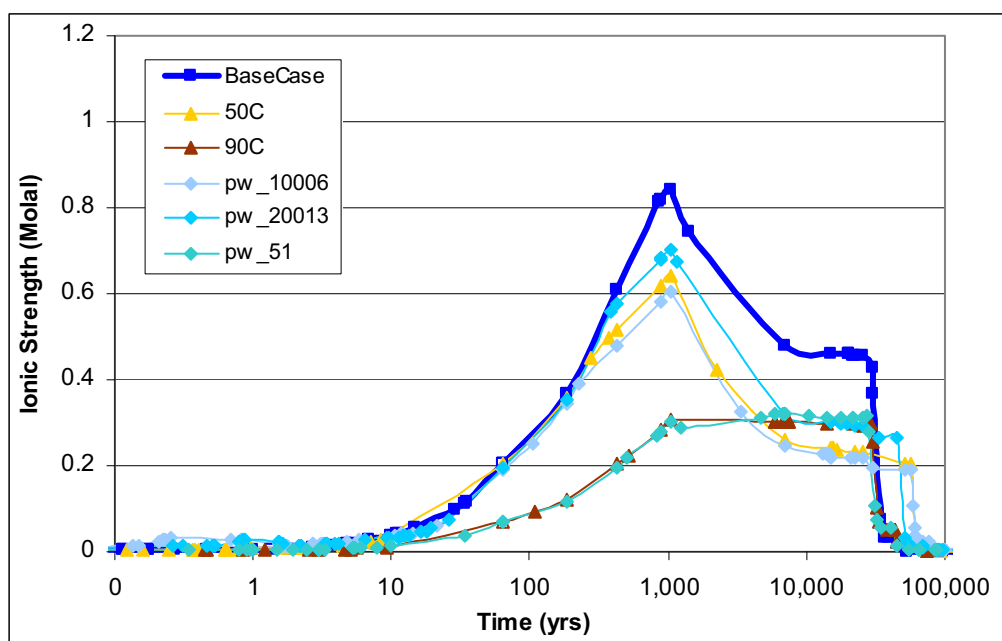
Source: CDSP Liquid Influx Model Output.xls (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-48. Effect of Increasing Temperature on the pH for CDSP

Figure 6-48 shows increasing the temperature lowers the late-time pH for the CDSP runs. One reason for decreasing pH at high temperature is the decrease in the dissociation constant of water with increased temperature. The HLWG is more-quickly depleted at higher temperatures because of the higher rate constants (Table 6-8) at elevated temperature. The batch reactor runs

at higher temperatures use a temperature-appropriate HLWG-degradation rate (Table 6-8). The metal alloy corrosion rates for the high temperature runs are the same as those used for the 25°C runs; however, these rates are also considered appropriate for higher temperatures because many of the experiments that generated the data for these rates were conducted at higher temperatures. As shown in the corrosion rate DTN (MO0409SPAACRWP.000 [DIRS 172059]), the degradation rates at 25°C are similar to those at 90°C. In addition, sensitivity studies were conducted that examined the impact of varying the corrosion rates over several orders of magnitude (Section 6.6.10). The thermodynamic database includes high temperature data for most mineral and aqueous species; therefore, the reacted solution chemistry and precipitated mineral phases are temperature appropriate.



Source: *CDSP Liquid Influx Model Output.xls* (Appendix F /Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-49. Effect of Increasing Temperature on Ionic Strength for CDSP

The results plotted in Figure 6-49 show temperature had a minor influence on steady-state ionic strength. The effects of evaporation inside of the waste package are not modeled explicitly, but the effects are accounted for by the broad uncertainty range for the ionic strength abstraction.

Although the simulations depicted in Figures 6-46 through 6-49 were run at a maximum of 90°C, the results of these high-temperature simulations have been used to extend the temperature limit of the pH and ionic strength abstractions (Sections 6.10.1 and 6.10.2) up to 100°C. The temperature effect on pH from 50°C to 90°C is small enough to conclude that the additional ten degrees will have little impact on lowering the late-time pH for either CSNF or CDSP waste packages, because TSPA-LA, and this model consider the transition from water to steam occurs at 100°C.

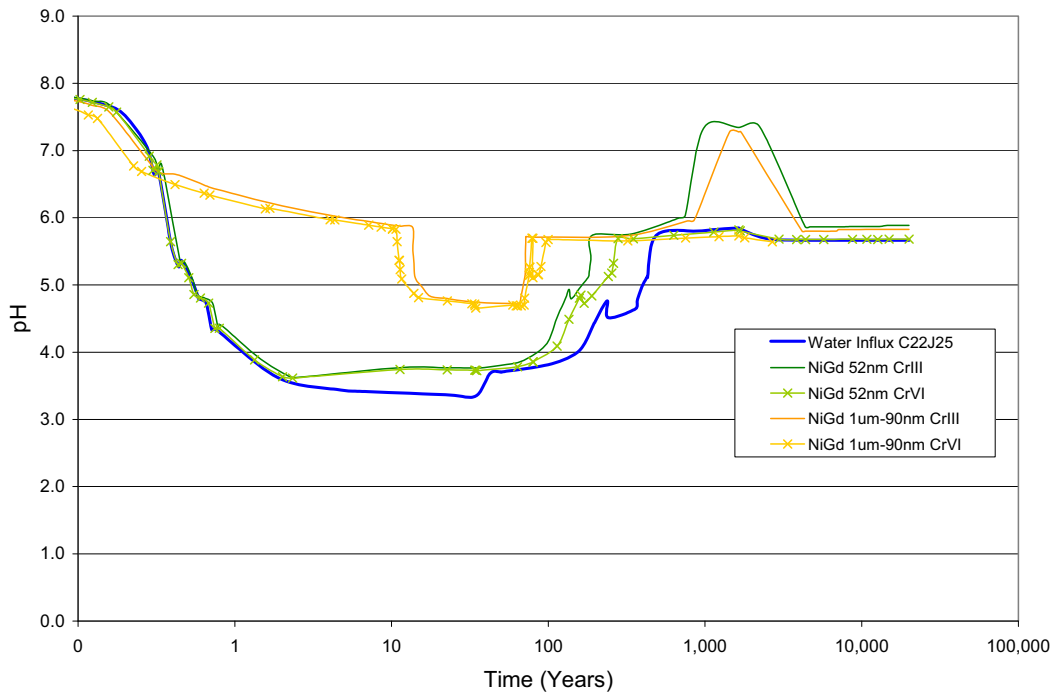
The high-temperature runs achieve an overall higher pH profile at long time periods and slight variations in the stable mineral assemblages compared with the results at 25°C. Since several of

the runs are carried out at above ambient temperatures and the in-package chemistry model results reflect the increased thermal conditions, the TSPA-LA feed also reflects the increased thermal conditions. The effect of temperature on the surface complexation model is discussed in Section 6.7.3. The above discussion addresses the thermal effects on chemistry portion of the subject matter of “Included” FEP 2.1.11.08.0A (Thermal effects on chemistry and microbial activity in the EBS) and indicates how that subject matter has been included and addressed within TSPA-LA. The microbial activity in the EBS subject matter is described in FEP 2.1.10.01.0A and documented in *Engineered Barrier System Features, Events, and Processes* (BSC 2004 [DIRS 169898]).

6.6.7 Design Change from Neutronit to NiGd Alloy

The impact of replacing the Neutronit with NiGd alloy in the CSNF waste package (Section 4.1.4) was examined in Rev 03 of this report, and is presented here for historical purposes. The files associated with this sensitivity can be found in DTN: MO0411SPAMOICA.000 [DIRS 172244]. The quantity (moles) and surface area for the NiGd alloy are modeled as the same as the Neutronit values. Two corrosion rates for the NiGd alloy are used: one run considers 0.052 $\mu\text{m}/\text{yr}$ (52 nm/yr) from Table 4-8, and the second case uses 1 $\mu\text{m}/\text{yr}$ for the first ten years, followed by 90 nm/yr (BSC 2004 [DIRS 168434]) for the remainder of the run. The second rate case was examined to factor in the potential that the initial rate could be as high as 1 $\mu\text{m}/\text{yr}$, but could slow down due to the passive layer that forms.

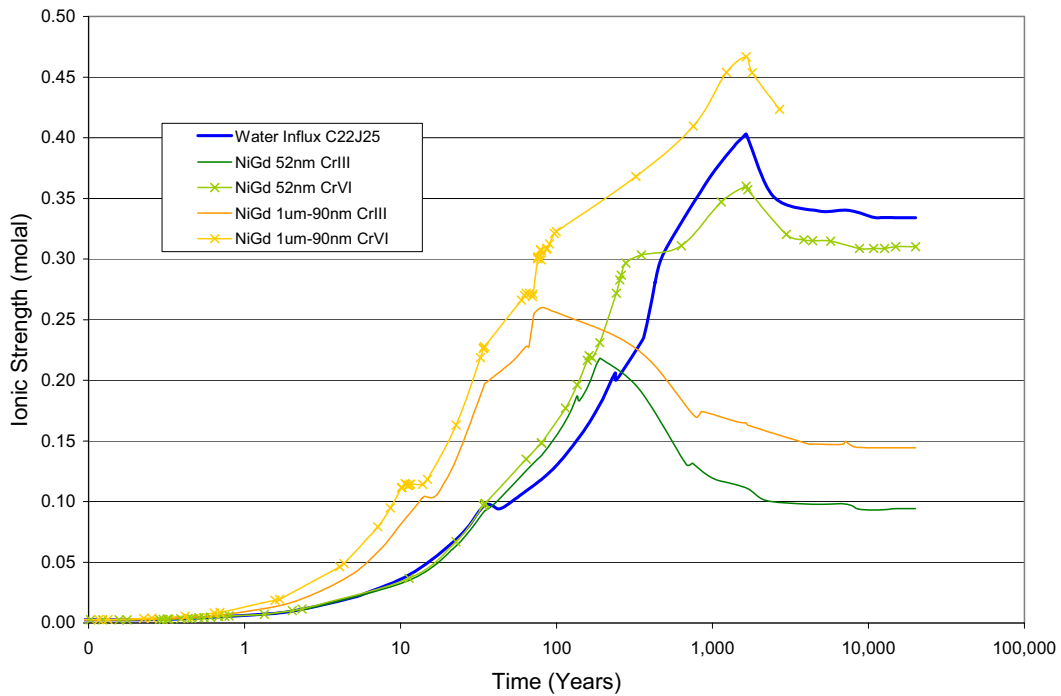
A CSNF run containing Neutronit with a mean flux value and mean fuel exposure ($Q = 1.5 \text{ L}/\text{yr}$ and 10% exposed) was used as the base case for this sensitivity. Because chromium is a major component of the NiGd alloy, and also has a great impact on the chemistry of the system, this sensitivity also examines the Cr(III) and Cr(VI) oxidation states for each run. The results are presented in Figures 6-50 through 6-53.



Source: CSNF_NiGd Sensitivity.xls (DTN: MO0411SPAMOICA.000 [DIRS 172244]).

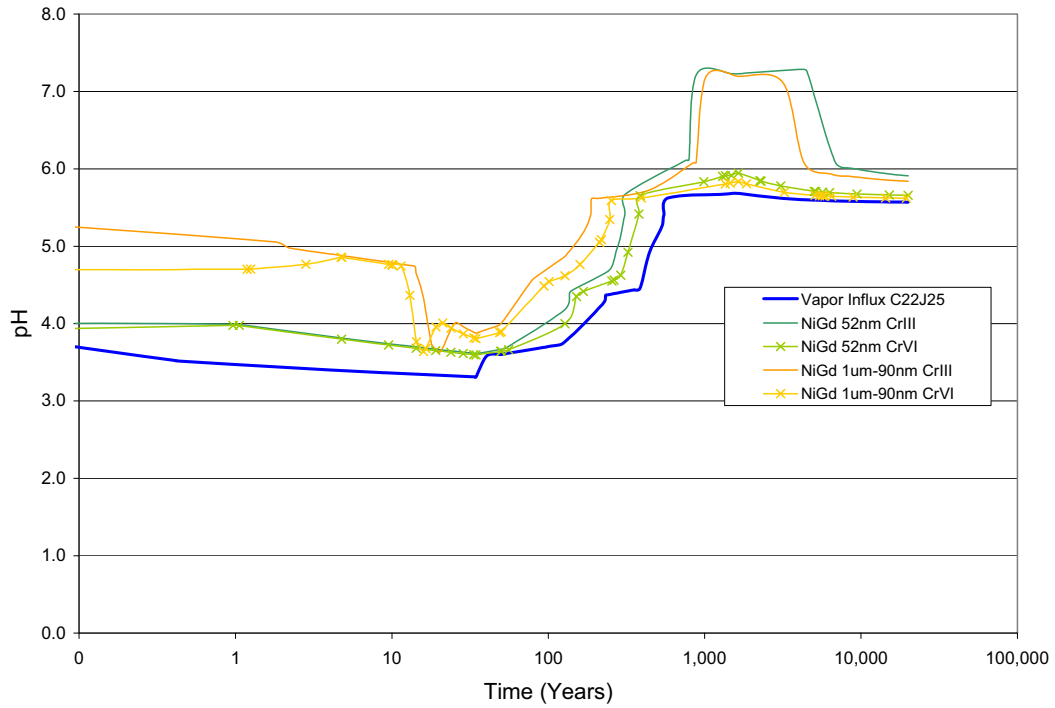
NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-50. NiGd Alloy Design Change for Liquid-Influx Case: Impact on pH



Source: CSNF_NiGd Sensitivity.xls (DTN: MO0411SPAMOICA.000 [DIRS 172244]).

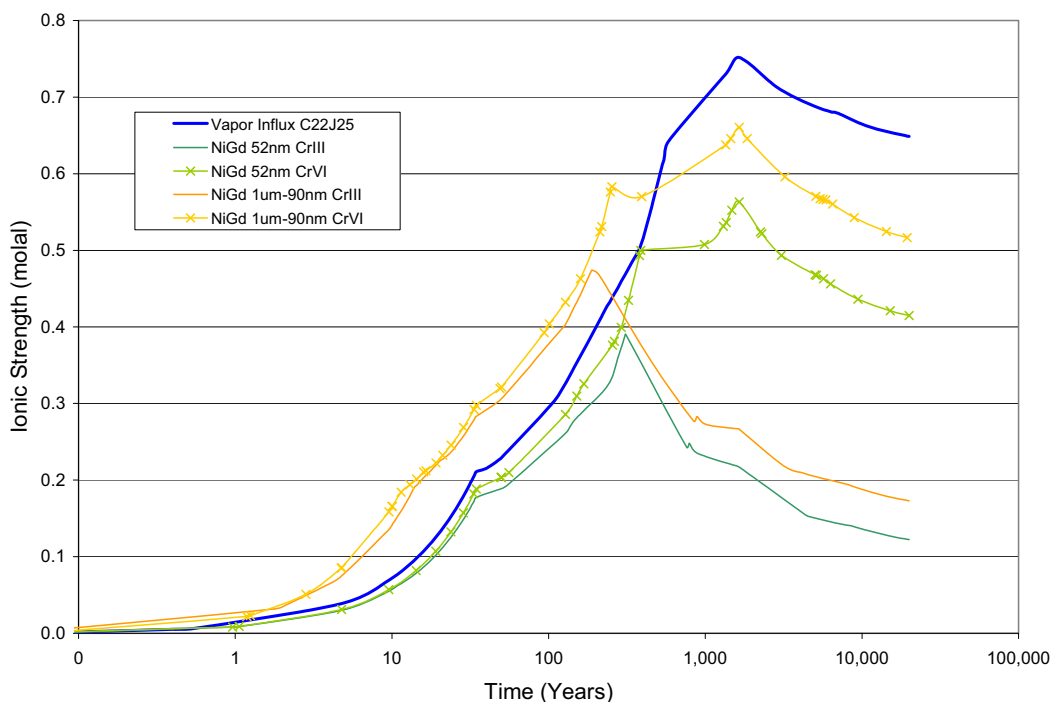
Figure 6-51. NiGd Alloy Design Change for Liquid-Influx Case: Impact on Ionic Strength



Source: CSNF_NiGd Sensitivity.xls (DTN: MO0411SPAMOICA.000 [DIRS 172244]).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-52. NiGd Alloy Design Change for Vapor-Influx Case: Impact on pH



Source: *CSNF_NiGd Sensitivity.xls* (DTN: MO0411SPAMOICA.000 [DIRS 172244]).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-53. NiGd Alloy Design Change for Vapor-Influx Case: Impact on Ionic Strength

The NiGd alloy runs have a slightly higher pH than the base case for early times that is more pronounced for the higher NiGd alloy rates, but the stabilized, long-term pH is similar for all the cases. The Cr(VI) runs exhibit a high pH between 1,000 and 10,000 years as shown in the Cr sensitivity.

The ionic strength profiles for the NiGd alloy runs are similar to the base case, but are slightly higher than the base case for the liquid-influx case, and slightly lower than the base case for the vapor-influx case, as shown by comparing Figures 6-51 and 6-53. The ionic strength effects are more impacted by the Cr oxidation state than the presence of NiGd alloy instead of Neutronit.

The results of this sensitivity show that the change in design to NiGd alloy does impact the chemistry, but that the resulting pH and ionic strength are still within the abstracted range provided to TSPA-LA (Section 8.1.2).

6.6.8 HLWG Composition Change

The impact of the HLWG composition on the in-package chemistry was evaluated in Rev 03 of this report by replacing the simplified glass composition described in Table 4-5 with two alternate glass compositions and running the CDSP liquid-influx case. The D23C25 case was chosen because it has the highest glass exposure (100%) and since this run was used to assess several other sensitivities. The inputs associated with this sensitivity are documented in Rev 03 of this report, and the files can be found in DTN: MO0411SPAMOICA.000 [DIRS 172244].

The sources for the “raw” HLWG chemical composition used for these sensitivity runs are “Waste Treatment and Immobilization Plant (WTP) High-Level Waste (HLW) Canister Production Estimates to Support Analyses by the Yucca Mountain Project” (Hamel 2003 [DIRS 164947], Table 1) and *Projected Glass Composition and Curie Content of Canisters from the Savannah River Site (U)* (Allison 2004 [DIRS 168734], Attachment I). The specific glass compositions chosen from these sources contain high concentrations of alkali metals (Na and K), which increase solution pH during glass dissolution (which was the Waste Acceptance Product Specifications Low Viscosity Purex Only Glass Composition for the SRL_new glass composition, and the WTP Technology Case for the Hanford glass composition).

For input into EQ6, several elements in the glass composition from Hamel (2003 [DIRS 164947], Table 1) and Allison (2004 [DIRS 168734], Attachment I) were simplified in a manner similar to the base-case glass presented in Table 4-5, and described in CRMWS M&O 2001 [DIRS 153263]. The simplification is presented in detail in *CDSP_HLWG_Sensitivity.xls* in DTN: MO0411SPAMOICA.000 [DIRS 172244]. These minor changes were made to the basic composition to increase the efficiency of the calculations, to decrease the EQ6 run time, and to allow the use of a pH-dependent rate law. These simplified glass compositions were calculated in *CDSP_HLWG_Sensitivity.xls* (DTN: MO0411SPAMOICA.000 [DIRS 172244]). These simplified glass compositions were also included in the EQ6 database as the minerals ‘SRL_New’ and ‘HanfordTC’ (Stoichiometric fractions less than 0.0001 moles/100 g are not read by EQ3/6). The resulting simplified HLWG formula based on 100 g is in Table 6-15. The base-case composition is also presented for comparison purposes.

Table 6-15. HLWG Sensitivity: High-Level Waste Glass Composition

| Element | Base Case Moles/100g HLWG | SRL_New Moles/100g HLWG | HanfordTC Moles/100g HLWG |
|---------|---------------------------|-------------------------|---------------------------|
| O | 2.7038 | 2.6338 | 2.5781 |
| U | 0.0078 | 0.0102 | 0.0107 |
| Ba | 0.0011 | 0.0017 | 0.0018 |
| Al | 0.0863 | 0.0578 | 0.2537 |
| S | 0.0040 | 0.0028 | 0.0048 |
| Ca | 0.0162 | 0.0182 | 0.0217 |
| P | 0.0005 | 0.0003 | 0.0186 |
| Si | 0.7765 | 0.7400 | 0.6068 |
| B | 0.2912 | 0.2895 | 0.1845 |
| Fe | 0.1722 | 0.2064 | 0.1364 |
| K | 0.0752 | 0.0751 | 0.0059 |
| Mg | 0.0333 | 0.0327 | 0.0060 |
| Na | 0.5767 | 0.5931 | 0.8907 |

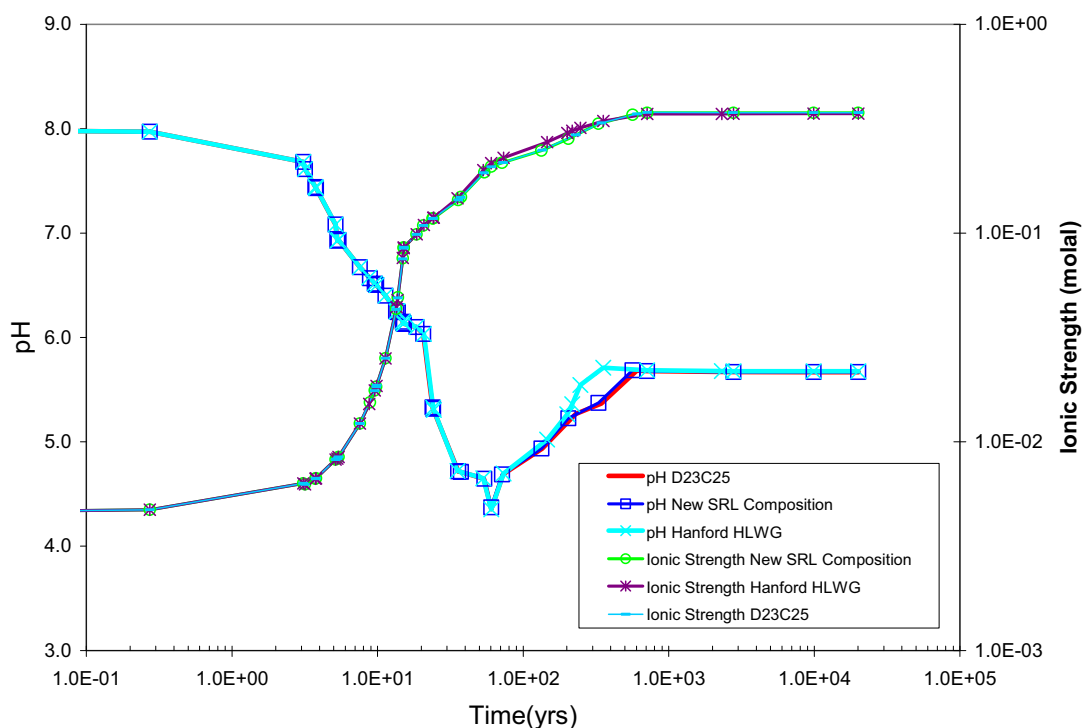
Source: *CDSP_HLWG_Sensitivity.xls* (DTN: MO0411SPAMOICA.000 [DIRS 172244]).

NOTE: Elemental mole % of the reactant are calculated based on 100 grams of reactant.

SRL = Savannah River Laboratory

The results of the sensitivity runs with the alternate glass compositions, Figure 6-54, show only small differences in pH or ionic strength. This sensitivity uses Cr(VI) rather than Cr(III), but

because these cases use the same oxidation state for chromium, Cr(VI), the important pH and ionic strength trends shown in this sensitivity are impacts caused by the differences in the base-case glass composition and the new SRL and Hanford glass compositions. Because this sensitivity used Cr(VI) instead of Cr(III), the ionic strengths shown in Figure 6-54 are higher than they would be if Cr(III) had been used. The impacts of Cr(VI) versus Cr(III) are described in more detail in Section 6.6.3.



Source: *CDSP_HLWG_Composition Sensitivity.xls* (DTN: MO0411SPAMOICA.000 [DIRS 172244]).

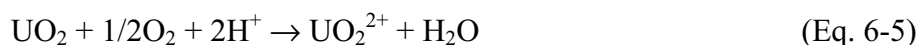
Note: Time refers to time after waste package breach. This sensitivity uses Cr(VI) rather than Cr(III).

Figure 6-54. Comparison of HLW Glass Compositions

6.6.9 Quantity of Carbon Steel Type A516 in Waste Package Design

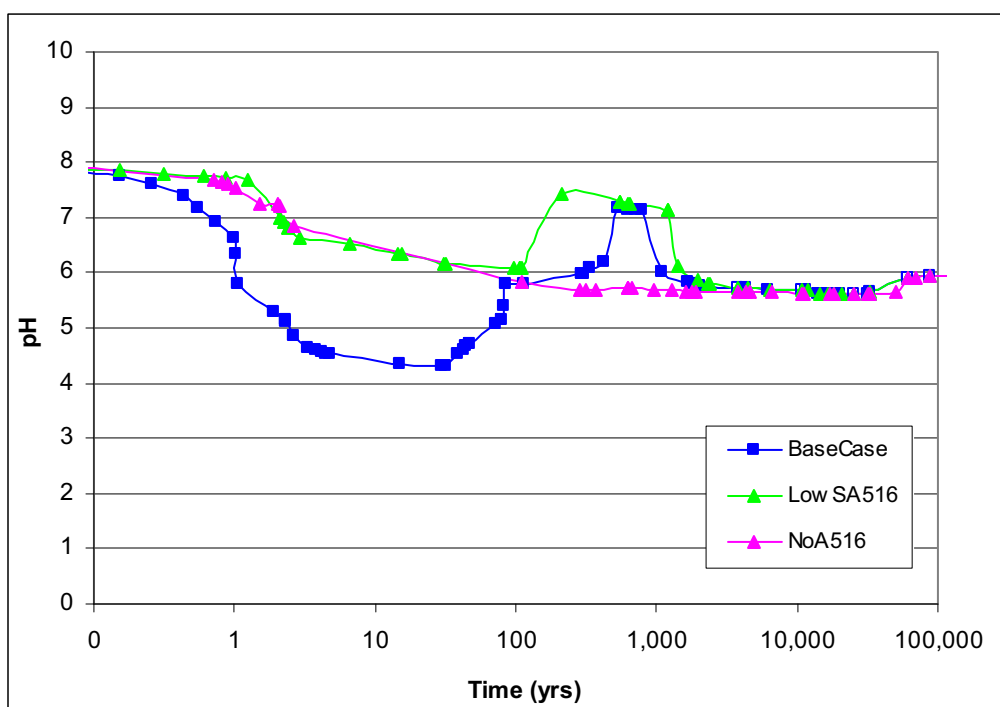
The impact of the Carbon Steel Type A516 on the in-package chemistry was evaluated by replacing the Carbon Steel Type A516 with an equivalent amount of Stainless Steel Type 316 (filename: NoA516), decreasing the amount of sulfur in the A516 by a factor of ten (filename: LowA516), and running the CSNF liquid-influx model.

The effect of the fuel exposure on the pH is direct, in that increasing the fuel exposure buffers the pH due to the acid neutralizing capacity of the CSNF via the reaction:



where UO_2 represents the CSNF. Thus, dissolution of UO_2 consumes hydrogen ion (H^+) and moderates the pH towards a more neutral value.

The results of the run in which Carbon Steel Type A516 was replaced by Stainless Steel Type 316 are displayed in Figure 6-55. As would be expected, the low-pH conditions observed in the base-case run in the first 100 years are not observed in the run in which Carbon Steel Type A516 is replaced with Stainless Steel Type 316. The reason for the moderation in the pH between the two runs is the lower corrosion rate, nearly three orders of magnitude, of Stainless Steel Type 316 compared to the Carbon Steel Type A516 steel. The slower corrosion of Stainless Steel Type 316 translates to a decrease in the amount of oxidized sulfur released into the aqueous phase as a function of time and, thus, the pH remains closer to neutral. At times greater than about 600 years (Figure 6-55), the pH is less for the Stainless Steel Type 316 case than for the base case due to the excess chromium in solution from the additional stainless steel.



Source: CSNF Liquid Influx Model Output.xls (Appendix F \Spreadsheets).

Note: Time refers to time after waste package breach.

Figure 6-55. Effect of Replacing Carbon Steel Type A516 with Stainless Steel Type 316

The implication of replacing the Carbon Steel Type A516 with stainless steel in the CSNF waste package is a more moderate pH condition during the period when Carbon Steel Type A516 would be corroding. Thus, with less-extreme pH conditions, the CSNF dissolution that increases at low pH would also be lower. Additionally, it would be expected for waste packages that do not contain Carbon Steel Type A516, the low-pH conditions would not develop. For example, for the naval waste packages, McKenzie (2002 [DIRS 165219], p. 9 of 47) states :

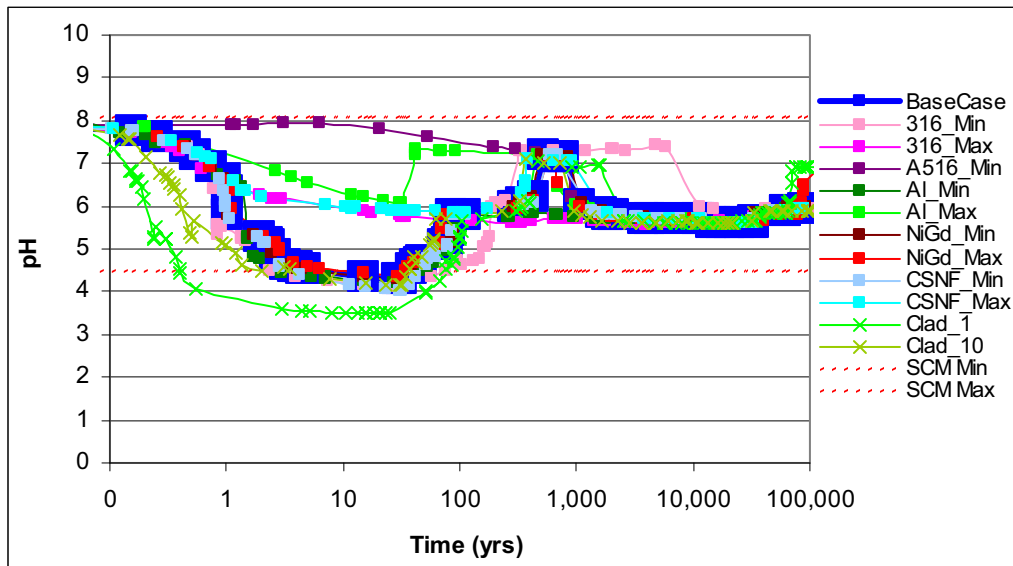
Spacers are used for some basket designs to fill the internal length of the canister. Canister, basket, and spacer materials are stainless steels (including material dual certified as Type 316 and 316L stainless, Nitronic 50 and Nitronic 60) and Zircaloy. Hastelloy alloy C-22 may be used in future designs.

For the waste packages that do not contain Carbon Steel Type A516 in their design, the expected pH profiles would be near those displayed in Figure 6-55.

6.6.10 Degradation Rates

To account for the full possible range of corrosion rates for all the materials in the waste package, a series of sensitivity runs was conducted to examine the minimum and maximum rates for the alloys and fuels from Tables 6-5, 6-7, and 6-8. In addition, the CSNF cladding exposure and the HLWG fracture factor were varied.

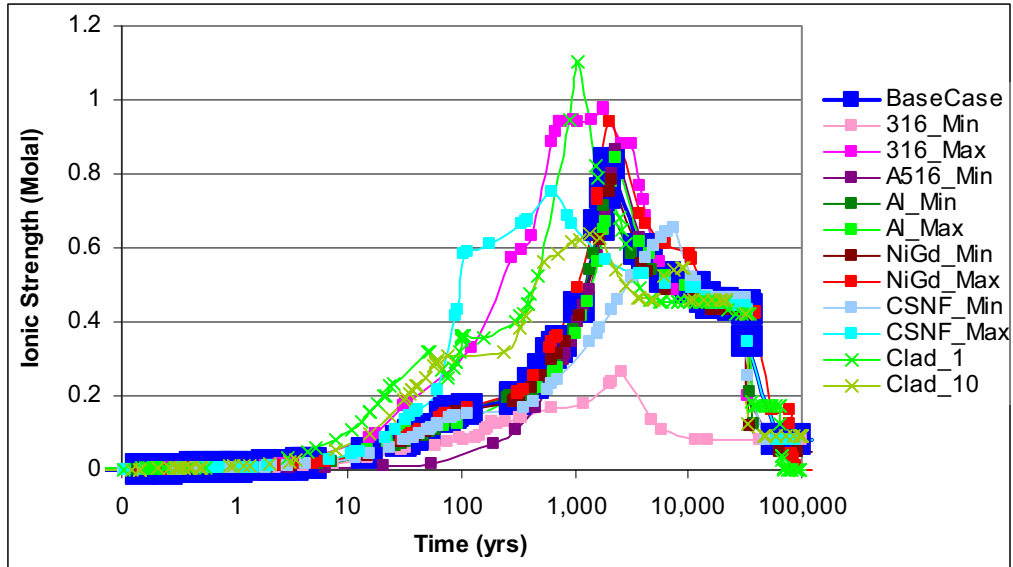
The results of these runs are presented in Figures 6-56 through 6-59.



Source: *CSNF Liquid Influx Model Output.xls* (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years). The dotted red lines indicate the minimum and maximum pH values after the surface complexation model has been applied.

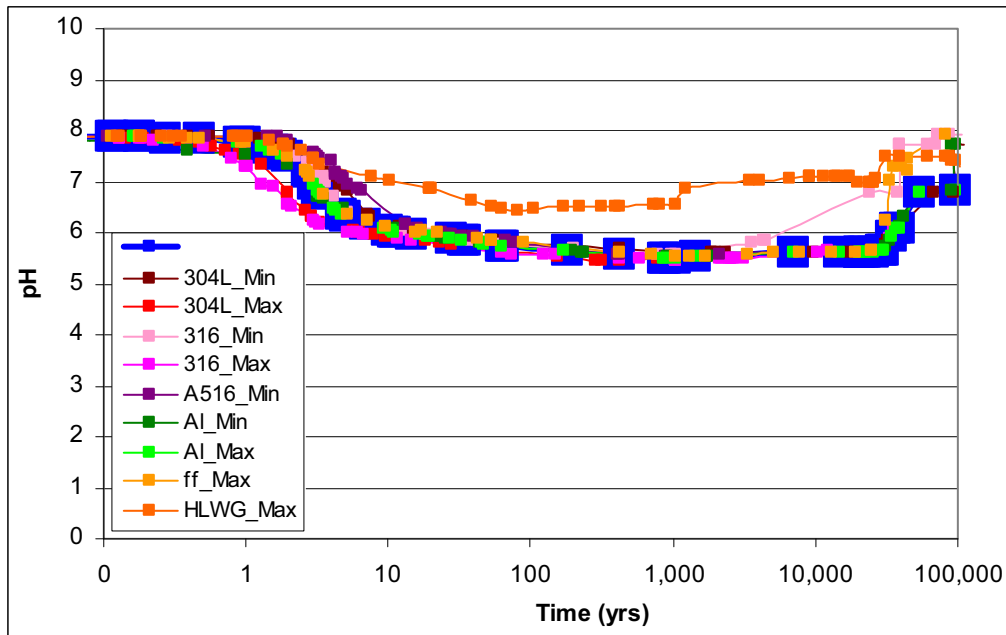
Figure 6-56. Corrosion Rate Comparison for CSNF: pH Profiles



Source: CSNF Liquid Influx Model Output.xls (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

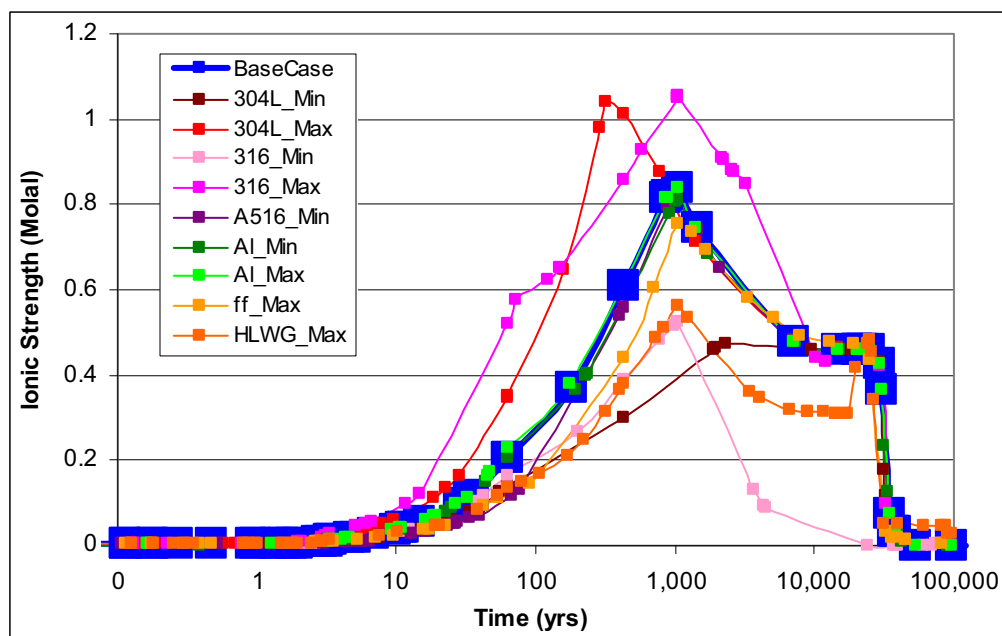
Figure 6-57. Corrosion Rate Comparison for CSNF: Ionic Strength Profiles



Source: CDSP Liquid Influx Model Output.xls (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-58. Corrosion Rate Comparison for CDSP: pH Profiles



Source: *CDSP Liquid Influx Model Output.xls* (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-59. Corrosion Rate Comparison for CDSP: Ionic Strength Profiles

The results in Figures 6-56 and 6-58 show that pH and ionic strength are sensitive to the corrosion rates used. The pH responses occur as a reactant is depleted and when that reactant no longer impacts the chemistry. The CSNF pH curves are slightly different, and the faster rates correspond to a lower pH minimum. A comparison of the CSNF runs shows pH values (by up to 4 pH units) and timing of the pH response can vary by up to 1,000 years. However, these variations are accounted for in the pH abstraction that samples the pH between 4.5 and 7 or 8, depending on the waste package and the $f\text{CO}_2$ (Section 6.10.1). The effects of the surface complexation model are plotted in Figure 6-56 to indicate that even though the results of the EQ6 model show a pH minimum of less than 4, the output of this model gives a pH minimum of 4.5 due to the effects of the surface complexation model. The CDSP pH curves show little change in pH; the most notable changes are the increase in pH with an increase in the HLWG rate. For CDSP runs, over the full range of rates, pH values drop only by one pH unit (the minimum pH drops from ~ 5.5 to ~ 4.0).

The results in Figures 6-57 and 6-59 indicate that cases using higher corrosion rates result in higher ionic strengths for both CSNF and CDSP waste packages. The higher ionic strength is a result of the steel corrosion; the faster the elements are released from the steel, the higher the resulting ionic strength. Sensitivity cases using the highest rates result in ionic strengths approaching 1 molal for both the CSNF and CDSP cases.

The above discussion addresses the subject matter of “Included” FEP 2.1.09.07.0A (Reaction kinetics in waste package) and indicates how that subject matter has been included and addressed within TSPA-LA.

The CSNF rate equation (BSC 2004 [DIRS 169987], Tables 8-2 and 8-3) includes a term, $\text{Log}(A)$, that factors in geometric surface area and surface area of cracks in the fuel. The $\text{Log}(A)$ term was calculated using the mass of CSNF fuel for the most likely or apex distribution ($\text{Log}(A) = -6.7$) and the maximum distribution ($\text{Log}(A) = -5.4$).

The results for the CSNF rates used show the maximum rate results in a higher pH for the first hundred years, but shows the same steady-state pH value. The maximum rate also results in a higher initial ionic strength, but similar long-term ionic strengths.

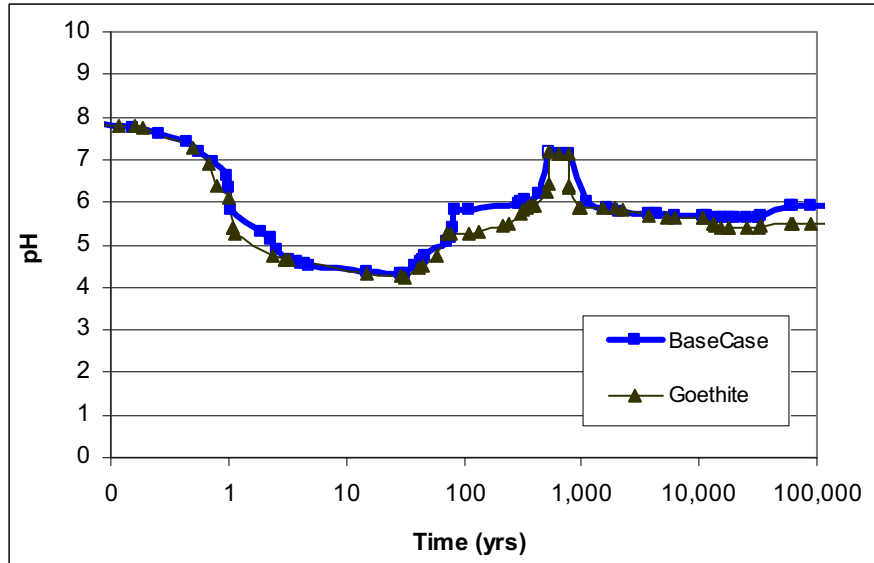
The HLWG rate has a high uncertainty applied to the glass degradation rate coefficient (k_E term) of the rate equation (Section 4.1.3). The base case uses the most probable values for k_E : k_{E_acidic} ($8.41 \times 10^3 \text{ g}/(\text{m}^2 \cdot \text{d})$), and $k_{E_alkaline}$ ($2.82 \times 10^1 \text{ g}/(\text{m}^2 \cdot \text{d})$). The sensitivity uses the maximum values for k_E ($k_{E_acidic} = 1.15 \times 10^7 \text{ g}/(\text{m}^2 \cdot \text{d})$ and $k_{E_alkaline} = 3.47 \times 10^4 \text{ g}/(\text{m}^2 \cdot \text{d})$). These results show a considerably higher pH and ionic strength profile for the higher HLWG rate.

The exposure factor, f_{exposure} , is the value multiplied by the geometric surface area of the HLWG to achieve an effective surface area, which includes an increase in surface area due to fractures in the glass. *Defense HLW Glass Degradation Model* (BSC 2004 [DIRS 169988, Table 8-1) provides a minimum and most-likely value of the exposure factor equal to 4 and a maximum of 17. Figure 6-58 shows the pH is about $\frac{1}{2}$ pH unit lower for the lower-exposure factor case. These results do not reflect the surface complexation effects expected to buffer the pH in the first few hundred years. As shown in Section 6.8.4, solutions with pH values from 3 to 4 are buffered to pH values from 7 to 8 when contacted with iron oxides.

6.6.11 Hematite versus Goethite

Which ferric (hydr)oxide phase will form in a degrading waste package is uncertain. Section 6.3.2 provides information that suggests there is a time dependency where poorly crystalline ferrihydrite may initially dominate with a slow transformation, depending on the system chemistry to goethite and hematite. The likely outcome is a mixture of these three phases in varying proportions. In the EQ6 modeling included in this report, the runs were executed such that hematite was the ferric phase allowed to precipitate from solution, while the surface complexation model (Sections 6.7 and 6.8) uses goethite as the ferric phase active in the complexation of aqueous species.

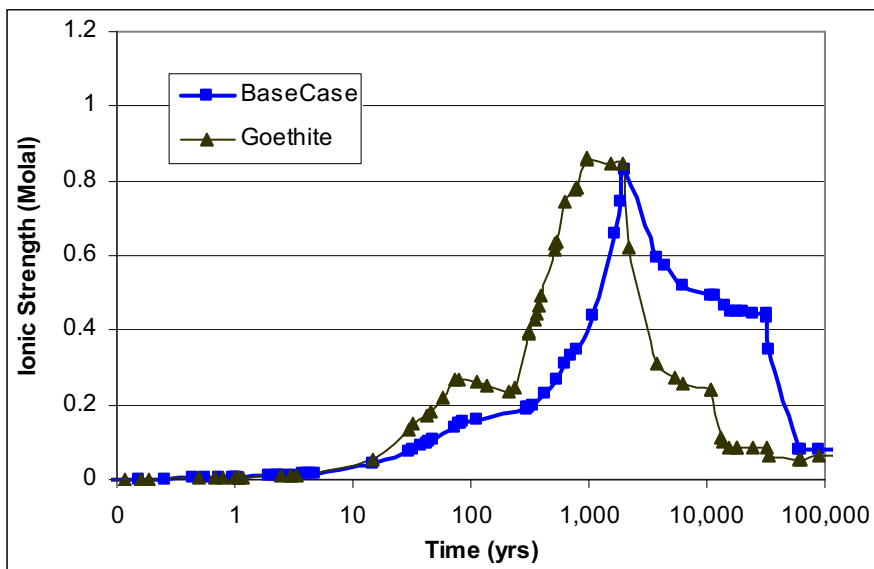
To examine the influence of the ferric (hydr)oxide on the CSNF batch reactor model pH and ionic strength predictions, runs were completed where goethite was allowed to form in place of hematite, and the results are shown in Figures 6-60 through 6-63. It was not possible to test the effect of ferrihydrite formation because it is not included in the thermodynamic database. However, based on its similarities to goethite, the conclusion drawn from this sensitivity can also be applied to ferrihydrite formation.



Source: *CDSP Liquid Influx Model Output.xls* (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

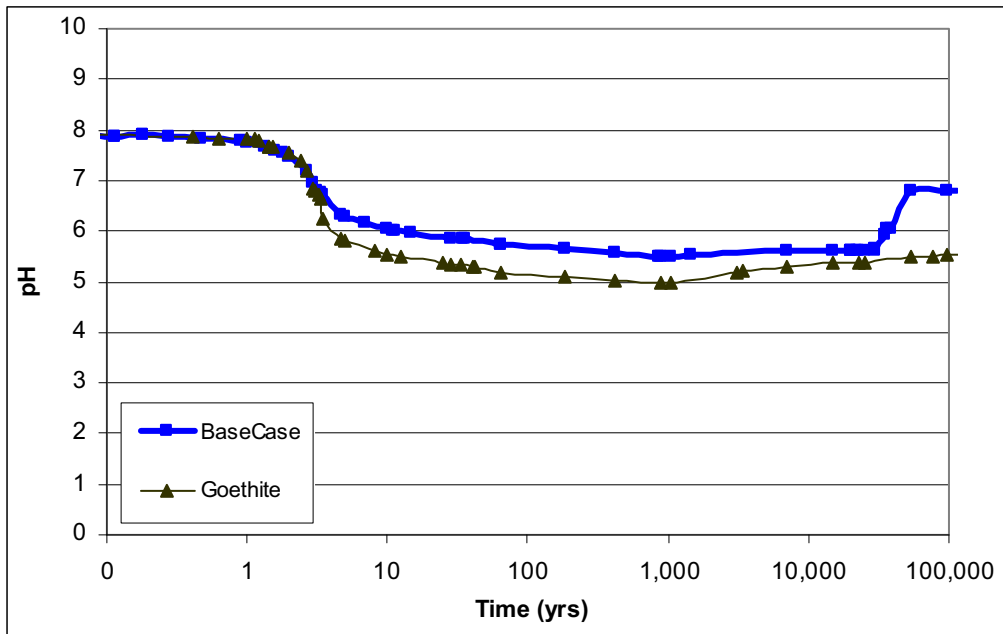
Figure 6-60. Hematite and Goethite Comparison for CSNF: pH Profiles



Source: *CDSP Liquid Influx Model Output.xls* (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

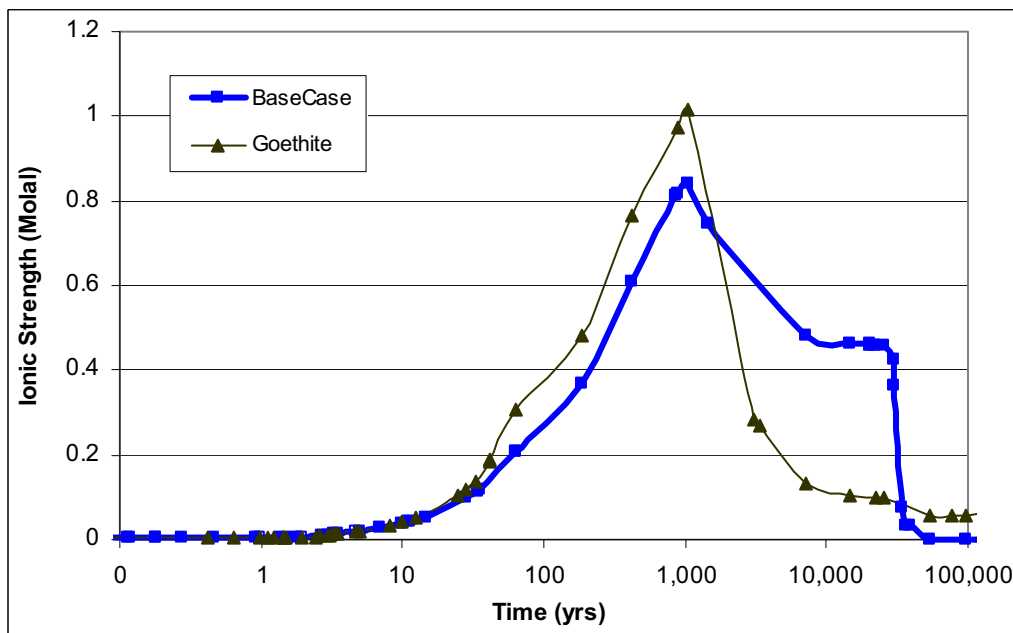
Figure 6-61. Hematite and Goethite Comparison for CSNF: Ionic Strength Profiles



Source: *CDSP Liquid Influx Model Output.xls* (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-62. Hematite and Goethite Comparison for CDSP: pH Profiles



Source: *CDSP Liquid Influx Model Output.xls* (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-63. Hematite and Goethite Comparison for CDSP: Ionic Strength Profiles

Additional results are plotted in Appendix A (Figure A-13 for pH profiles and Figure A-14 for ionic strength profiles). As shown in Figures 6-60 and 6-62, the pH results for the hematite and

goethite cases are similar for the CSNF and CDSP cases, with the goethite runs about 0.5 pH units lower than those of the hematite runs. This 0.5-pH difference is insignificant and within the uncertainty of the model.

There is a slight difference in the ionic strength between 10 and 1,000 years (for both the CSNF and CDSP cases) where the goethite runs have higher ionic strength because the formation of goethite consumes more water than does hematite. This difference is insignificant in terms of performance because the ionic strength for goethite and hematite are above the 0.05-mol/L colloid threshold (BSC 2004 [DIRS 170025]) for this entire period.

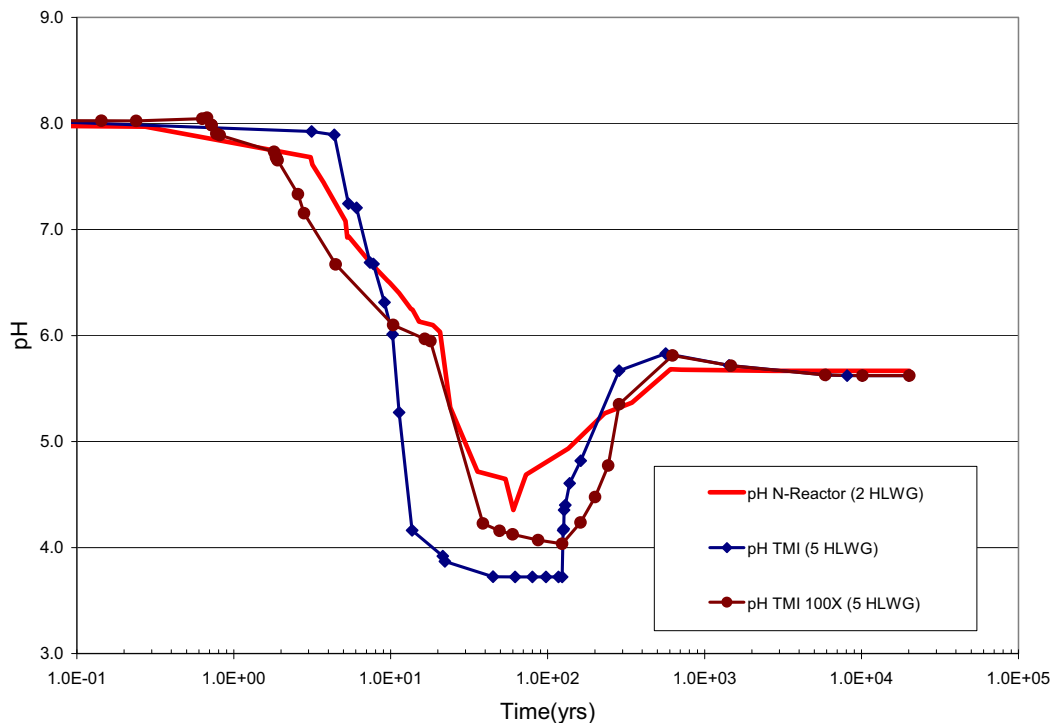
This sensitivity analysis has shown the results of the batch reactor model are largely independent of which ferric (hydr)oxide mineral is allowed to form in the simulations. Therefore, the use of ferrihydrite, which is similar to goethite, in the surface complexation model is acceptable because it would not cause unusual excursions in the chemistry of the batch reactor model that used hematite as the primary phase. In addition to the similarities, there are also some differences between ferrihydrite and goethite, such as their surface areas. To account for this difference, the treatment of the ferric (hydr)oxide site density is varied by an order of magnitude, as described in Section 6.7.1 (Table 6-18).

6.6.12 Codisposal Waste Package Containing 5-DHLW Canisters

The CDSP base case considers codisposal waste packages contain N-Reactor fuel (2-MCO/2-DHLW). In addition to this type of waste package, the repository is expected to contain other waste packages containing the 5-DHLW/DOE SNF waste package design (BSC 2004 [DIRS 173501], Table 12). The 5-DHLW/DOE SNF waste package design contains five HLWG canisters (5-DHLW waste package) and one DOE canister containing DOE SNF. A sensitivity analysis was conducted in Rev 03 of this report to determine the impact on the in-package chemistry of the degradation of the 5-DHLW/DOE SNF waste package under the same conditions as the base-case N-Reactor D23C25 EQ6 run. The inputs to this sensitivity are documented in Rev 03 of this report, and files associated with this sensitivity can be found in DTN: MO0411SPAMOICA.000 [DIRS 172244].

The 5-DHLW/DOE SNF waste package design has a short waste package and a long waste package. This sensitivity examines the long waste package design because that is the design that will be used with the Three Mile Island (TMI) fuel. The EQ6 batch reactor model, liquid-influx case, was run for a codisposal waste package containing five HLWG canisters (5-DHLW long waste package) and one DOE canister containing one assembly of TMI SNF. TMI SNF was chosen because it is chemically similar to the N-Reactor fuel, and the purpose of this sensitivity is to examine the impact of the 5-DHLW/DOE SNF design. Using a drastically different fuel composition would have masked any differences due to the waste package design. The TMI SNF also represents a significant number of DOE canisters expected at the repository (DOE 2004 [DIRS 170071], Table A-2, "LEU oxide"). Two EQ6 calculations were run using the same drip rate, degradation rates (except fuel), temperature, and water composition as case D23C25. Two fuel degradation rates were used for these runs. Since the TMI fuel is a uranium oxide, the rates used in the EQ6 simulations were set equal to the CSNF rate and 100× the CSNF surface area, as recommended for DOE UO₂-type SNF in (DOE 2002 [DIRS 158405], Section 6.8).

The results of the sensitivity runs with the TMI codisposal waste package are shown in Figure 6-64. The pH is lower in the first few hundred years. However, these results do not reflect the surface complexation effects expected to buffer the pH in the first few hundred years. As shown in Section 6.8.4, solutions with a pH value of 3 to 4 are buffered to a pH value of 7 to 8 when contacted with iron oxides. Therefore, due to the influence of the surface complexation, no significant pH change would be expected with water contacting a breached TMI codisposal waste package as opposed to an N-Reactor codisposal waste package.

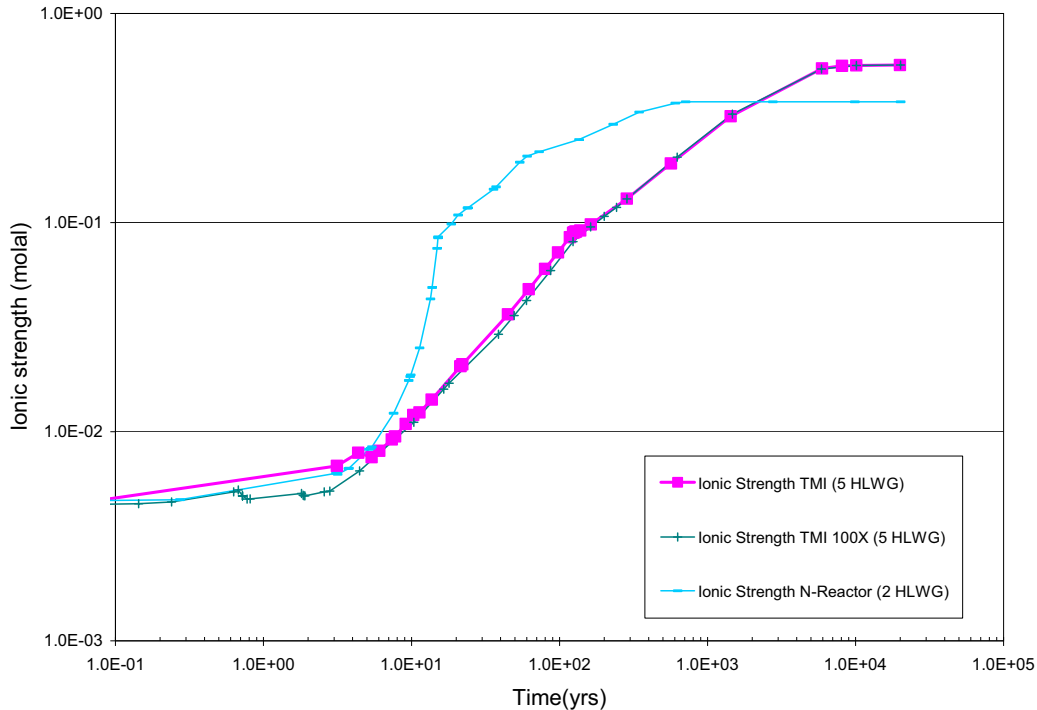


Source: *CDSP_5_DHLWG WP Sensitivity.xls* (DTN: MO0411SPAMOICA.000 [DIRS 172244]).

Note: Time refers to time after waste package breach. This sensitivity uses Cr(VI) rather than Cr(III).

Figure 6-64. MCO/2-DHLW Design versus 5-DHLW/DOE SNF: Impact on pH

Figure 6-65 shows the range of ionic strength values for the TMI codisposal waste package is similar to the N-Reactor codisposal waste package, except for slightly higher values after 4,000 years. The maximum increase in ionic strength, from 0.38 to 0.57 molal at 20,000 years, is small compared to the variation in ionic strength due to the influence of other variables in the system, such as the drip rate (Section 6.5.2.2.2). Therefore, the degradation of the TMI codisposal waste package is not expected to have a significant impact on the ionic strength of the solution compared to the N-Reactor codisposal waste package. Therefore, the different waste package configurations expected in the repository are sufficiently characterized by this model.



Source: *CDSP_5_DHLWG WP Sensitivity.xls* (DTN: MO0411SPAMOICA.000 [DIRS 172244]).

Note: Time refers to time after waste package breach. This sensitivity uses Cr(VI) rather than Cr(III).

Figure 6-65. MCO/2-DHLW Design versus 5-DHLW/DOE SNF: Impact on Ionic Strength

This sensitivity uses Cr(VI) rather than Cr(III), but because all of these cases use the same oxidation state for chromium, Cr(VI), the important pH and ionic strength trends shown in this sensitivity are impacts caused by differences in the waste package design, not the oxidation state of the chromium. Using Cr(VI) instead of Cr(III) means that the ionic strengths are higher than they would be if Cr(III) had been used. The impacts of Cr(VI) versus Cr(III) are described in more detail in Section 6.6.3.

The above discussion addresses the subject matter of “Included” FEP 2.1.01.02.0B (Interactions between co-disposed waste) and indicates how that subject matter has been included and addressed within TSPA-LA.

6.6.13 LaBS Glass Composition

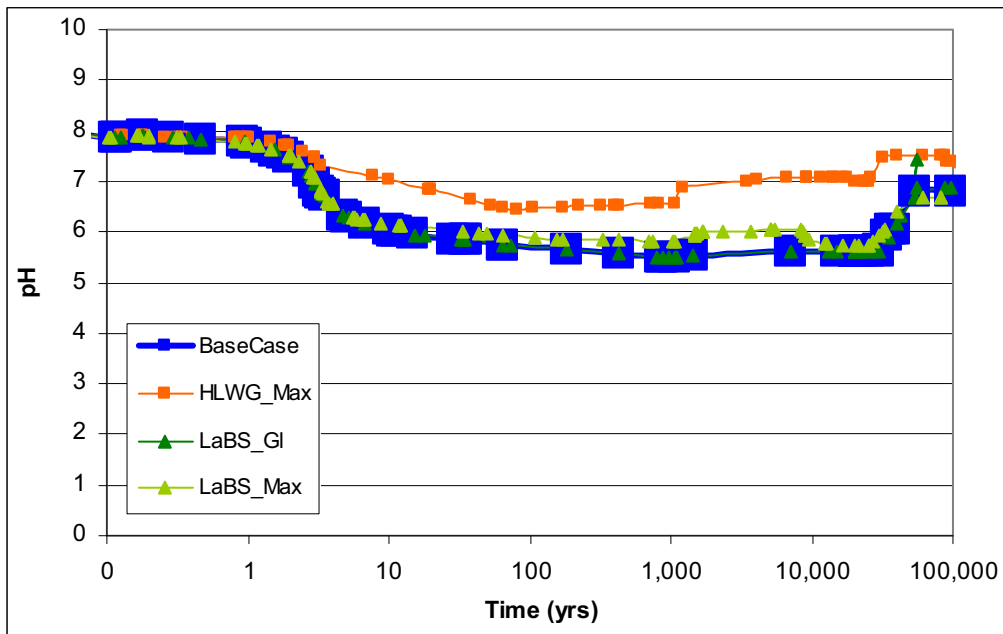
This sensitivity study examines the impact of using LaBS glass in a CDSP waste package. The composition of the LaBS glass is described by Marra and Ebert (2003 [DIRS 172949], Table 2, column: SRTC Tests, Pu only), and is provided in the Table 6-16. This sensitivity uses the most probably HLWG rate and f_{exposure} from Section 4.1.3.3, (Equations 4-3 and 4-4) for the LaBS glass.

Table 6-16. Simplified LaBS Glass Composition

| Element | LaBS Glass Moles/100g HLWG |
|---------|----------------------------|
| Al | 0.3657 |
| B | 0.2921 |
| Gd | 0.0408 |
| La | 0.0693 |
| Nd | 0.0610 |
| Pu | 0.0333 |
| Si | 0.4945 |
| Sr | 0.0210 |
| Zr | 0.0087 |
| O | 2.3540 |

Source: *CDSP_LaBS_Sensitivity.xls*
(Appendix F).

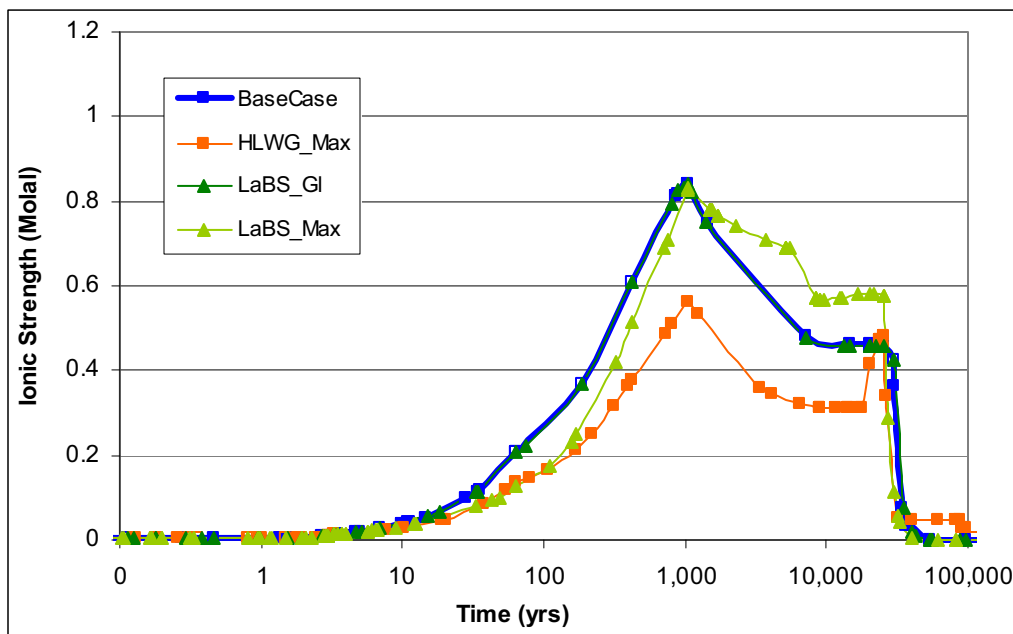
NOTE: Elemental mole % of the reactant are calculated based on 100 grams of reactant.



Source: *CDSP Liquid Influx Model Output.xls* (Appendix F).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-66. LaBS Glass pH Results



Source: *CDSP Liquid Influx Model Output.xls* (Appendix F).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-67. LaBS Glass Ionic Strength Results

As shown in Figures 6-66 and 6-67, the results of this sensitivity indicate that the LaBS glass has less of an impact on chemistry than the existing HLWG. The two cases that use the most probably rates (“BaseCase” and “LaBS_GI”) are identical for the HLWG and the LaBS glass for pH and ionic strength. For the two cases that use the high glass degradation rate (“HLWG_Max” and “LaBS_Max”), the case with HLWG has a higher pH than the case with the LaBS glass, as shown in Figure 6-66, but the LaBS glass case has a higher ionic strength than the HLWG case, as shown in Figure 6-67.

6.7 SURFACE COMPLEXATION MODEL OUTPUT

An analytical model is developed in this section using inputs from Section 4.1.6 to account for the pH buffering effects of iron corrosion product surfaces. The results of this analytical model are applied to the results of the batch reactor model in Section 6.9. As discussed in Section 6.3.2, ferric hydr(oxide) corrosion products such as ferrihydrite, goethite, and hematite will be present in abundance in the waste package in the short and long term (Schwertmann and Cornell 1991 [DIRS 144629]). Justification is made in Section 6.3.2 for using either ferrihydrite or goethite as the representative iron corrosion product as long as the concentration of surface sites is not overestimated. Therefore, for the analytical model developed here, a compromise is made to use a thermodynamic database for hydrous ferric oxide (HFO) while using the surface properties of goethite (i.e., specific surface area and specific site density). While there are differences in complexation reactions between HFO and goethite (Appelo et al. 2002 [DIRS 168168]), these differences are small compared to those in specific surface areas of HFO and goethite, and especially between these minerals and hematite (Section 6.3.2). Choosing the lower surface site concentrations of goethite helps to decrease the modeled concentration of adsorption sites per liter water and ensure the effects of complexation with iron corrosion

product surfaces are not overestimated in the model. The modeled adsorption site concentration is determined in Section 6.7.1.

Studies have shown that the surface chemistry of natural sediments can be successfully modeled using the surface complexation approach by assuming that ferrihydrite is the dominant adsorbent in natural sediments or that the surface area of the sediments is composed entirely of ferrihydrite (Payne and Waite 1991 [DIRS 168860]; Waite et al. 2000 [DIRS 168856]). Ferrihydrite is equated with HFO in Dzombak and Morel (1990 [DIRS 105483]). The general formula for HFO is $\text{Fe}_2\text{O}_3 \cdot n\text{H}_2\text{O}$ (Dzombak and Morel 1990 [DIRS 105483], p. 90); however, the bulk structure is uncertain and gradually changes from highly amorphous to more crystalline structures with age (Dzombak and Morel 1990 [DIRS 105483], p. 90). Ferrihydrite is predicted to occur in abundance in the waste package at early times and persist over the long term (Schwertmann and Cornell 1991 [DIRS 144629]), and it will continue to form as a product of steel corrosion. This knowledge and the studies cited above justify the use of the HFO surface complexation approach for the analytical model developed in Section 6.7.1.

6.7.1 pH Buffering During Dissolution of Carbon Steel

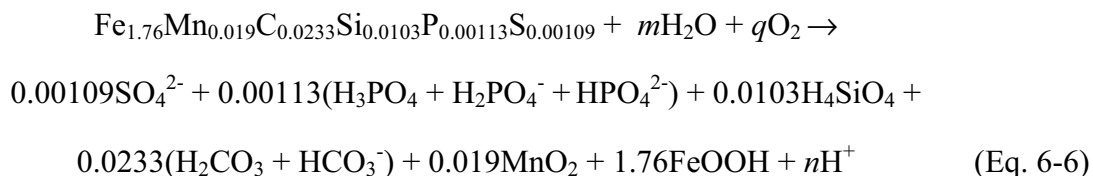
In a multireactant system, such as the waste package interior, the evolution of the chemistry is most influenced by reactants that react rapidly, contain elements with the potential to perturb the ambient chemical conditions, and are present in large enough quantities to exert a chemical change on the system. For these reasons, Carbon Steel Type A516 is considered to be of primary importance because it has a high corrosion rate (Table 4-8) relative to the other reactants, its composition (which includes reduced sulfur which lowers system pH when oxidized), and its abundance in the waste package. During the period of rapid corrosion of Carbon Steel Type A516, a large quantity of ferric (hydr)oxides will accumulate in the waste packages in a relatively short period of about 200 years, and the oxidation of sulfur will tend to lower the system pH. Occurring simultaneously with Carbon Steel Type A516 corrosion are corrosion and degradation of the other waste package metal alloys and waste forms, albeit at much slower rates than Carbon Steel Type A516. While the system chemistry would be dominated by Carbon Steel Type A516 corrosion, the complexity of the system chemistry would be increased by the corrosion or degradation of the other waste package components. The stainless steel components would form ferric (hydr)oxides and other metal corrosion products (e.g., pyrolusite), the aluminum alloys (6061 and 1100) would form gibbsite and clay minerals, and while the spent nuclear fuel would form primarily schoepite, the HLWG would form clays (as shown in Figure 6-5 for CSNF and Figure 6-13 for CDSP). Nearly all of the degradation products of the waste package component materials other than Carbon Steel Type A516 are phases with surface complexation properties (e.g., Langmuir 1997 [DIRS 100051] p. 351). Considering only the reaction of Carbon Steel Type A516 would tend to underestimate the total number of surface sites available for the surface complexation reactions, as well as the nature of the sites (e.g., site density) because the surface sites of other materials and corrosion products (e.g., gibbsite, clays) would be ignored.

It is during the 0- to 600-year period after waste package breach that the analytic surface complexation model developed in this section was used to predict the in-package pH. The reasons for limiting the application of the surface complexation model to this period are: 1) increasing uncertainty in the ferric (hydr)oxide mineral assemblage as a function of time; and

2) this relatively brief period of low pH, as predicted by the EQ6 batch reactor model, would be most influenced by the presence of surface complexation reactions.

The model is restricted to 25°C simply because the surface complexation database is restricted to this temperature. After waste package breach, and water is present in the waste package, the temperature can still range up to the boiling point of water. Temperatures above 25°C tend to decrease the pH_{zpc} , the pH of zero surface charge, for oxides and hydroxides (Brady 1992 [DIRS 154467], p. 2,941 and Table 1). This relationship implies that an increase in temperature increases the deprotonation of surface sites, resulting in a reduction in pH. The magnitude of this effect can only be determined by additional modeling because there are numerous side effects associated with an increase in temperature. For example, the log K of the dissociation of water increases from -14 to -13 as temperature increases from 25°C to 60°C (DTN: SN0410T0510404.002 [DIRS 172712]). Cation adsorption also tends to increase as temperature increases (Brady 1992 [DIRS 154467], p. 2,941). Temperature also affects aqueous and water-air equilibria of numerous potential adsorbates (e.g., carbonate, radionuclides, etc.) in various ways. Without carefully modeling all of the effects of temperature, the only assertion that can be made is that higher temperatures may moderate (or augment) the pH-buffering effects of the surface sites in the waste package. However, regardless of temperature effects, surface complexation will markedly buffer pH in the waste package, which is the primary point of this exercise.

A mole of Carbon Steel Type A516 dissolves into neutral and subneutral solutions as (Table 6-4):



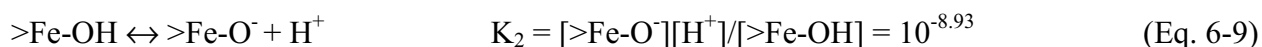
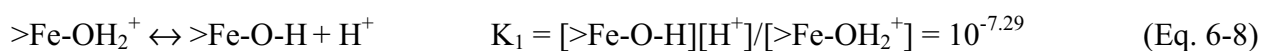
where m , n , and q are positive numbers that depend ultimately on the pH of the solution (at 25°C, HCO_3^- is the predominant carbonate species above pH 6.3; H_2PO_4^- is the dominant P species between pH 2.4 and 7; H_3PO_4 is the dominant species below pH 2.4). The choice of goethite (FeOOH) as the iron corrosion product is irrelevant to the value of n . Goethite is used here and in Section 6.8.2 simply because the surface site concentrations of goethite are used. Charge balance requires that n , the amount of acid produced in the reaction, be equal to $2(\text{SO}_4^{2-}) + (\text{H}_2\text{PO}_4^-) + 2(\text{HPO}_4^{2-}) + (\text{HCO}_3^-)$ where the bracketed terms denote amounts added from Carbon Steel Type A516 dissolution (moles). The acid input and the resulting pH can be estimated for when a liter of water reacts with a mole of Carbon Steel Type A516 as:

$$n = 0.00218 + 0.00113 + 0 + 0 = 0.00331 \text{ mol/L H}^+ \quad (\text{Eq. 6-7})$$

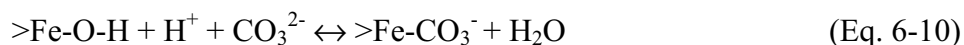
which, when used in a solution charge balance equation, corresponds to a pH of ~ 2.5 (assuming that all of C reacts to carbonic acid). 0.1 moles of Carbon Steel Type A516 dissolution would result in $n = 0.000331$ and a solution with a pH of ~ 3.5. The pH derived analytically for Carbon Steel Type A516 dissolution is close to the output pH calculated by EQ6. For example, Figure 6-4 shows the pH profiles for the three water compositions, and the stabilized pH when

Carbon Steel Type A516 dominates the system chemistry at about ten years is ~ 3.5 . The high initial pH values of around pH = 8 (Figure 6-56), 0.1 to 10 years, reflects the chemistry of the starting solutions and a finite amount of reaction needs to occur to overcome the buffer capacity of the water before the low pH (3.5) of Carbon Steel Type A516 is realized.

The analytic equation (Eq. 6-6) and the EQ6 calculations neglect the tendency for iron hydroxide corrosion product surfaces to buffer pH in the near-neutral range through the following surface reactions (Table 4-9):



$>\text{Fe}^-$ is a general representation of an adsorption site at the iron (hydr)oxide-solution interface. The values of the equilibrium constants in this section are for 25°C. For the HFO reactions in Table 4-9, HFO_s (strong site) and HFO_w (weak site) represent the adsorption sites. Sorbed bicarbonate also appears through (Table 4-9):

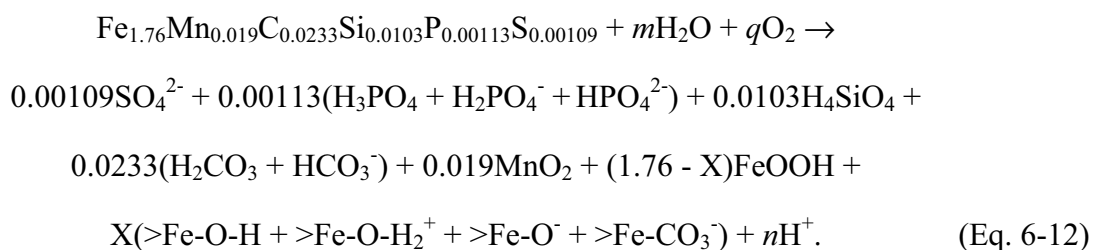


where

$$K_{\text{CO}_3} = \frac{[>\text{Fe-CO}_3^-]a_{\text{H}_2\text{O}}}{[>\text{Fe-OH}][\text{H}^+][\text{CO}_3^{2-}]} = 10^{12.78} \quad (\text{Eq. 6-11})$$

Brackets in the above equations indicate activities. For surface species, activities are treated as equivalent to concentrations. The activity of an aqueous species is equivalent to the species concentration multiplied by the species activity coefficient, which is a function of ionic strength.

To account for the surface effect, the Carbon Steel Type A516 dissolution reaction must be expanded to:



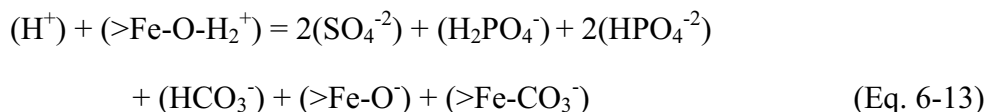
where X is the amount of Fe in FeOOH (moles) exposed to solution.

The approximations applied to solving the charge balance equation for the expanded reaction above are listed and justified in Table 6-17. Specifically, since the output pH is calculated to be between pH 2.4 (no surface effect) and \sim pH 8 (surfaces considered), the magnitude of the charged species are calculated at these two end members. The sulfate contribution to the charge balance is ~ 0.002 mol/L; species whose concentrations are less than a factor of 10 of this will tend to have a minor effect on the charge-balance calculation.

Table 6-17. Approximation for Charge Balance

| Approximation | Justification | Notes |
|--|--|--|
| Dissolved Mn can be neglected | MnO ₂ growth limits dissolved Mn to low levels | EQ3/6 runs show pyrolusite to limit dissolved Mn to low levels over the pH range of interest |
| Deprotonated Si species do not contribute to charge balance | Deprotonated Si species only form at pH > 9 | At pH 7 [H ₃ SiO ₄ ⁻] ~ 10 ⁻⁴ mol/L At pH 2.4 [H ₃ SiO ₄ ⁻] = 3.2 × 10 ⁻¹⁰ mol/L |
| OH ⁻ can be neglected in the charge balance | OH ⁻ levels will always be much less than sulfate levels below pH 9 | At pH 8 [OH ⁻] = 10 ⁻⁶ mol/L At pH 2.4 [OH ⁻] = 2.5 × 10 ⁻¹² mol/L |
| HSO ₄ ⁻ can be neglected in the charge balance | HSO ₄ ⁻ only exceeds SO ₄ ²⁻ levels below pH 2 | At pH 8 [HSO ₄ ⁻] = 10 ⁻⁹ mol/L At pH 2.4 [HSO ₄ ⁻] = 4 × 10 ⁻⁴ mol/L |

Given a value of $X = 0.176$ (10% of reacted Fe become surface sites – a lower-end value for goethite, Table 4-10) – for dissolution of 1 mole of Carbon Steel Type A516 into a near-neutral solution: $n = 2(\text{SO}_4^{2-}) + (\text{H}_2\text{PO}_4^-) + 2(\text{HPO}_4^{2-}) + (\text{HCO}_3^-) + (>\text{Fe-O}^-) + (>\text{Fe-CO}_3^-) - (>\text{Fe-O-H}_2^+)$. Similarly, charge balance for a solution in contact with carbon dioxide gas at 0.001 atm and into which 1 mole of Carbon Steel Type A516 was dissolved would be:



The two phosphate terms can be manipulated with the equilibrium constant for their conversion to give: $(\text{H}_2\text{PO}_4^-) = 0.00113/(1 + 10^{-7.2}\gamma_{\text{H}_2\text{PO}_4^-}/[\text{H}^+])$ and $(\text{HPO}_4^{2-}) = 0.00113(1 - 1/(1 + 10^{-7.2}\gamma_{\text{H}_2\text{PO}_4^-}/[\text{H}^+]))$. Similarly, mass action expressions for $(\text{OH}^-) + (\text{HCO}_3^-)$ can be manipulated to show they are equal to $10^{-14}/[\text{H}^+]/\gamma_{\text{OH}^-}$ and $10^{-7.82} \text{PCO}_2/[\text{H}^+]/\gamma_{\text{HCO}_3^-}$, respectively (DTN: MO0302SPATHDYN.000 [DIRS 161756]). Substituting these expressions, carbonate equilibrium constants, the surface equilibria above, and rearranging to solve for pH gives:

$$\begin{aligned} (\text{H}^+) &= 0.00218 + 0.00113/(1 + 10^{-7.2}\gamma_{\text{H}_2\text{PO}_4^-}/[\text{H}^+]) + 0.00226(1 - 1/(1 + 10^{-7.2}\gamma_{\text{H}_2\text{PO}_4^-}/[\text{H}^+])) + \\ &10^{-14}/[\text{H}^+]/\gamma_{\text{OH}^-} + 10^{-7.82}\text{PCO}_2/[\text{H}^+]/\gamma_{\text{HCO}_3^-} + [>\text{Fe-O-H}](10^{-8.93}/[\text{H}^+] + \\ &\text{PCO}_2 10^{-5.36}/[\text{H}^+] - 10^{7.29}[\text{H}^+]) \end{aligned} \quad (\text{Eq. 6-14})$$

Note: PCO_2 is the partial pressure of CO_2 .

where, because of surface site mole balance, the site master-species concentration $[>\text{Fe-O-H}]$ at 25°C can be approximated by:

$$[>\text{Fe-O-H}] = 0.176/(1 + 10^{-8.93}/[\text{H}^+] + \text{PCO}_2 10^{-5.36}/[\text{H}^+] + 10^{7.29}[\text{H}^+]) \quad (\text{Eq. 6-15})$$

Equations 6-14 and 6-15 can be solved for pH as a function of fixed partial pressure of CO_2 (Table 6-18). These pH calculations are insensitive to the number of moles of Carbon Steel

Type A516 that corrodes per kg of water as long as corrosion exceeds 0.1 moles per kg of water. This is the case for each Carbon Steel Type A516 corrosion calculation in this report. The pH is calculated using a site density of 0.1 moles of sites per mole of iron. The most significant input to the surface complexation model is the site density of the iron mineral. Due to the potential uncertainty on the site density, an additional calculation shows the resulting pH if the site density were a 0.01 moles of sites per mole of iron, as shown in Table 6-18. Using one tenth the site density only results in slightly less pH buffering, 0.4 pH units at the higher pH values, and 0.2 pH units at the lower pH values. These results show significant pH buffering, even when using a low site density.

The results in Table 6-18 are for an ionic strength of 0.1 molal. At this ionic strength, activity coefficients in equation 6-14 are in the range of 0.76 to 0.83 (Stumm and Morgan 1996 [DIRS 125332] p. 104). Concentrated solutions with high ionic strengths could result in a relatively negligible change in pH (negligible pH buffering). Moreover, the pHs were calculated for each fixed $f\text{CO}_2$ conditions. Thus, ionic strength has a negligible effect on the calculated pH values relative to the site density and partial pressure of carbon dioxide.

Table 6-18. Calculated pH Values as a Function of $f\text{CO}_2$

| Log $f\text{CO}_2$ (atm) | 0.1 moles of sites/mole of Fe (pH) | 0.01 moles of sites/mole of Fe (pH) |
|-----------------------------|--|---|
| -5.0 | 8.1 | 7.7 |
| -4.5 | 8.1 | 7.7 |
| -4.0 | 8.0 | 7.6 |
| -3.5 | 7.9 | 7.6 |
| -3.0 | 7.7 | 7.5 |
| -2.5 | 7.5 | 7.3 |
| -2.0 | 7.3 | 7.1 |
| -1.5 | 7.0 | 6.8 |

Source: *SCM analytical.xls* in Appendix F.

To simplify the calculation, adsorption of phosphate, uranium, nickel, chromate, sulfate, and other sorbates was neglected. Under favorable conditions, these sorbates, if abundant, could have a noticeable impact on the final pH values calculated. As a check on the potential effects of these sorbates, surface complexation reactions involving the entire database in Table 4-9 are performed in Section 6.8.

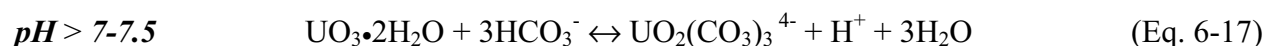
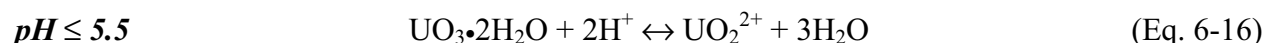
The greater the number of surface sites, the greater is the tendency for surface reactions to resist acid production from the steel and instead to maintain the pH near 7.6 (for dissolution of 1 mole of Carbon Steel Type A516 to produce a pH below 7 at $\text{PCO}_2 = 0.001$ atm requires fewer than 1.5% of the Fe sites to be exposed). The calculations above were done without activity corrections for aqueous or surface species – so they are somewhat approximate. Nevertheless, they suggest that degradation of Carbon Steel Type A516 will result in solution pH values close to 7 to 8 because of corrosion product surface chemistry. The output pH values shown in Table 6-18 depend simply on interaction of produced acidity with near-neighbor Fe sites and not transport and mixing of fluids.

The effect of corrosion products on the pH of the in-package environment is a first-order process that the surface complexation model provides as input to TSPA-LA. The above discussion addresses the subject matter of “Included” FEPs 2.1.09.02.0A (Chemical interaction with corrosion products) and 2.1.09.01.0B (Chemical characteristics of water in waste package) and indicates how that subject matter has been included and addressed within TSPA-LA.

6.7.2 pH Buffering During Dissolution of Stainless Steel, Fuel, and HLWG

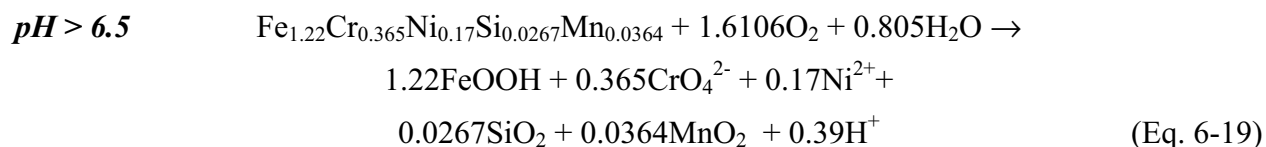
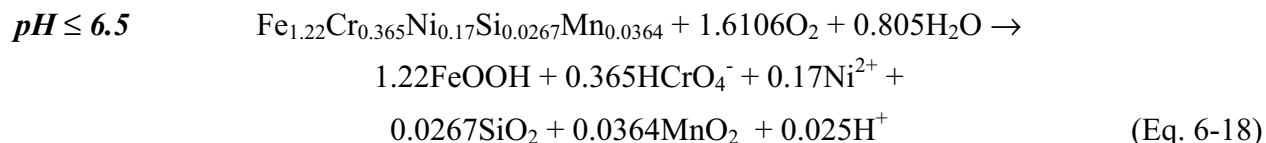
Over time periods greater than a thousand years, the primary reactions affecting pH are dissolution of Stainless Steel Types 316 and 304L, spent fuel and alteration products (schoepite) and, in codisposal packages, HLWG. Simplified dissolution reactions for each are written below. Minor components have been left out of the respective compositions. The approximated compositions provide a sufficiently clear picture of the degree to which each reaction affects proton balances in solution. SiO₂ refers either to aqueous silica or silica solid (e.g., chalcedony).

Schoepite

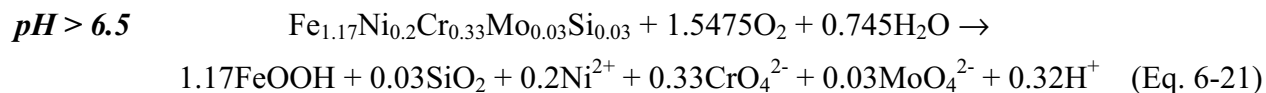
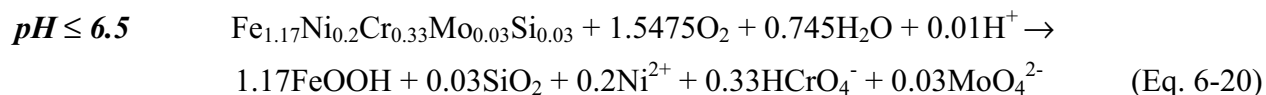


NOTE: Schoepite stability for $5.5 \leq \text{pH} \leq 7-7.5$ is dependent on $f\text{CO}_2$.

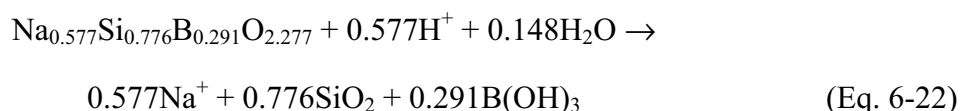
Stainless Steel Type 304L



Stainless Steel Type 316



HLWG



Schoepite and iron corrosion products dominate the composition of the alteration products from time periods of 500 years and beyond, and the schoepite dissolves at acid pH to consume protons and at above neutral pH to produce protons. These two reactions, thus, constitute a negative feedback that resists pH excursions to high or low pH, and ultimately anchors pH values near the schoepite solubility minimum – pH 6.5 to 7, depending on the ambient carbon dioxide pressure. The EQ6 output supports the buffering of the pH near neutral by schoepite (see, for example Figure 6-23).

Considering the steels and glass in isolation from the schoepite points to another mechanism by which alkalinity increases from HLWG dissolution are resisted – namely above pH 6.5 Cr oxidizes to CrO_4^{2-} , whereas below it forms HCrO_4^- . This means ambient pH has an important effect on proton production by steel degradation. Specifically, at $\text{pH} \leq 6.5$, steel degradation either consumes protons or produces relatively small amounts of them; whereas, at $\text{pH} > 6.5$, degradation of Stainless Steel Types 304L and 316 produces relatively large amounts of protons. This has the effect of anchoring pH values close to 6.5 and consuming proton consumption by glass degradation. Equilibrium surface reactions will also buffer pH (e.g., $\text{>Fe-OH} = \text{>Fe-O}^- + \text{H}^+$ and $\text{>Fe-HCO}_3 = \text{>Fe-CO}_3^- + \text{H}^+$).

As proton consumption prevails at $\text{pH} \leq 6.5$, but proton production amplifies when $\text{pH} > 6.5$, indicates long-term pH values will converge towards 6.5 as long as sufficient steel and HLWG exists in the package. There is over twice as much Stainless Steel Type 316 as HLWG and roughly 20% more Stainless Steel Type 304L than HLWG, so the presence of the degrading stainless steel will tend to keep the pH from going too high due to the effects of the degrading HLWG. As shown in Figure 6-58, if the highest HLWG rate is used, the resulting pH is about a unit higher than the other cases. A potential case examining the highest HLWG combined with low degradation rates for the stainless steels could result in an even higher pH, but that resulting pH would also be in contact with significant quantities of schoepite, according to the well-mixed batch reactor (Section 6.3.1.1), and the buffering effects of the schoepite would prevent the pH from going much higher than neutral. The most reasonable high-end pH for packages containing HLWG is, therefore, 6.5 to 7. FeOOH corrosion products will also be abundant in codisposal packages, from steel degradation and from the earlier dissolution of the Carbon Steel Type A516.

To summarize, over the first hundred years when the degraded waste form consists primarily of Carbon Steel Type A516 corrosion products and intact fuel, pH values will be held to near neutral by the surface properties of the corrosion products (the most abundant proton donor or acceptor in the package). Over longer time periods, when the degraded waste form will consist of schoepite, dissolving steels, and possibly HLWG, the pH will be anchored near neutral by dissolution reactions of schoepite and steel which consume protons at acidic pH and release them at alkaline pH values. Extremely unlikely scenarios involving degradation of individual components are required to overcome the intrinsic capacity of the degraded waste form to control its ambient pH to near neutral.

6.7.3 Temperature

Temperature affects surface equilibria, aqueous speciation, and dissolution rates and for this reason might be expected to affect pH excursion limits (maximum and minimum pH). In the absence of a schoepite solubility model, it is unreasonable to speculate about the location of the

high temperature schoepite solubility minimum. The pH_{zpc} of hematite changes from roughly 9.1 to 7.7 (with a temperature shift of 5°C to 75°C) (Stumm 1992 [DIRS 141778], p. 76), which suggests a downward pH shift of possibly 1.5 units in goethite surface equilibria with an increase in temperature from 25°C to 90°C . Site densities might decrease if higher temperatures favor formation of lower surface area materials. There are not enough high-temperature surface chemistry data to advance more than a qualitative model for corrosion product surface behavior at higher temperatures.

The pH below which steel dissolves to HCrO_4^- , and above which it dissolves to CrO_4^{2-} , is 6.5 at 25°C , 6.7 at 50°C , and 6.9 at 100°C . Higher temperatures might, therefore, be expected to raise the upper-limit basic-pH range due to HLWG dissolution to ~ 7 .

The above discussion addresses the thermal effects on chemistry portion of the subject matter of "Included" FEP 2.1.11.08.0A (Thermal effects on chemistry and microbial activity in the EBS) and indicates how that subject matter has been included and addressed within TSPA-LA. The microbial activity in the EBS subject matter is described in FEP 2.1.10.01.0A and documented in *Engineered Barrier System Features, Events, and Processes* (BSC 2004 [DIRS 169898]).

6.8 SURFACE COMPLEXATION DEMONSTRATION

The effects of ferric (hydr)oxide surfaces on pH, as defined in Section 6.7, are demonstrated in this section using PHREEQC version 2.3. PHREEQC is a USGS geochemical code capable of performing surface complexation reactions in aqueous systems. The DTNs generated for the surface complexation demonstration are solely intended to build confidence in the surface complexation model discussed in Section 6.3.2, and are not actual technical products of this model. These DTNs include: MO0403SPASURCR.000, MO0403SPAYMPR3.000, and MO0403MWDSRFCC.000.

The purpose of the PHREEQC surface complexation calculations in this section is to build confidence in the analytical model developed in Section 6.7. Additional complexity in surface complexation calculations is afforded by PHREEQC due to its ability to include effects of surface charge, ionic strength, and a much larger assortment of surface complexation reactions. However, as the calculations demonstrate, this increased complexity does not result in pH predictions largely different from those made by the simplified analytical model in Section 6.7. Unlike the results of the analytical model, the PHREEQC calculations are not passed on to TSPA-LA.

Surface complexation calculations are calculations of equilibrium adsorption reactions. Similar to aqueous reactions, each adsorption reaction is represented by a specific chemical reaction and an associated equilibrium constant. The primary difference between aqueous reactions in bulk water and adsorption reactions is that surface sites have fixed locations and densities relative to one another. As a result, adsorption and desorption of protons and other ions from these sites cause the surface to take on an electrostatic charge, which in turn, affects the behavior of nonadsorbed ions in the vicinity of the surface. Therefore, equilibrium surface complexation calculations incorporate equations that simulate the effects of surface charge.

The particular surface complexation model chosen for these calculations is from Dzombak and Morel's (1990 [DIRS 105483], Chapter 2) generalized two-layer model, which is based on the Gouy-Chapman theory of the electrical double layer. In this model, there is one layer associated with the surface charge (affected by site-specific adsorption) and a diffuse layer between the surface layer and the bulk solution that contains counter-charged ions. Details of the mathematical representation of surface effects are provided by Dzombak and Morel (1990 [DIRS 105483], Chapter 2) and the PHREEQC 2.3 user manual (BSC 2001 [DIRS 154877]).

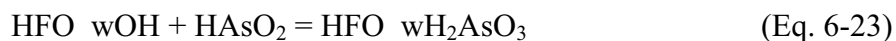
The surface complexation calculations presented below require several inputs. These inputs include a thermodynamic database (Section 6.8.1), surface properties (Section 6.8.2), and initial water and gas compositions (Section 6.8.3). Results of this demonstration are presented in Section 6.8.4.

6.8.1 Surface Complexation Reaction Databases

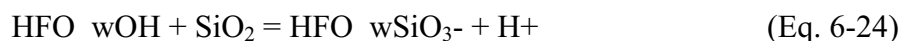
The surface complexation database was taken primarily from *Surface Complexation Modeling, Hydrous Ferric Oxide* (Dzombak and Morel 1990 [DIRS 105483]), which develops and documents a broad, self-consistent database of surface complexation reactions for hydrous ferric oxide (HFO) based on the generalized two-layer model. Two additional reactions for carbonate adsorption are included in work by Appelo et al. (2002 [DIRS 168168]). These reactions were derived by Appelo et al. (2002 [DIRS 168168], p. 3097) to be consistent with the Dzombak and Morel (1990 [DIRS 105483]) database.

In this report, two variations of this database are used for the surface complexation calculations. The first database, *YMP_R3.dat*, is developed in Section 6.3.2.1 and documented in DTN: MO0403SPAYMPR3.000. The other, *YMP_R3mr.dat*, is identical to the first, but includes all reactions in Table 6-12, which are estimated using linear free-energy relationships. These estimated reactions are of sufficient accuracy for the intended use of the surface complexation calculations in this report.

For *YMP_R3mr.dat*, all reactions from Table 6-12 were entered exactly as displayed, with the exception of arsenite and silica. To conform to the PHREEQC representation of the uncharged aqueous arsenite species (HAsO_2), the arsenite adsorption reaction was changed to:



The log K for this reaction was not changed because addition of H_2O to both sides of the equation has a negligible effect on the log K at ionic strengths below 1 molal. In addition, because the PHREEQC database does not include a SiO_3^{2-} aqueous species, the silica adsorption reactions were changed to:



and



with log K values of -7.06 and -14.66 , respectively. These reactions and log K values were calculated by combining the silica reactions from Table 6-12.

6.8.2 Surface Properties

As presented in Section 6.7, goethite is the representative iron product of each steel corrosion reaction. Because goethite is a major product of these reactions and it has a relatively large surface area and adsorption site density, goethite is the surface used in the surface complexation calculations. The rationale for using goethite surface areas in combination with the HFO thermodynamic database is discussed in detail in Sections 6.3.2, 6.6.11, and 6.7.

The surface properties of goethite are provided in Table 4-10. Multiplying the site concentration of 0.00135 mol/g by the molecular weight of goethite (88.85 g/mol) gives a site concentration of 0.120 mol/mol of goethite. This site concentration is equivalent to a specific surface area of 49 m²/g with a specific site density of 16.6 sites/nm², as well as equivalent to a specific surface area of 160 m²/g with a specific site density of 5 sites/nm². These values largely span the general ranges listed in Table 4-10 for these parameters. For the calculations below, the lower value of 49 m²/g specific surface area is considered so as to not over estimate the potential buffering effects.

Between 10 and 40 moles of ferric (hydr)oxides are calculated to form per kilogram of water (Section 6.3.2). For the calculations presented below, the molality of goethite is set at 10 moles per kg water, though thin film conditions could lead to much higher molalities. Multiplying this molality by the molecular weight of goethite implies approximately 900 grams of goethite per kg water, a value used in the PHREEQC input files to quantify the abundance of the adsorbent. Multiplying this value by the site concentration from Table 4-10 in Section 6.3.2 (0.00135 mol/g) results in a surface site molality of 1.2. Because the database (Dzombak and Morel 1990 [DIRS 105483]) divides the total number of sites into strong and weak sites, whereby the concentration of strong sites is 2.5% of the weak site concentration, the molalities of weak and strong sites were set at 1.2 and 0.03, respectively.

The electric double layer thickness is the only other surface parameter necessary for the PHREEQC runs. This was estimated by solving Equation 2.4 in *Surface Complexation Modeling, Hydrous Ferric Oxide* (Dzombak and Morel 1990 [DIRS 105483], p. 11 and 12) for $1/\kappa$, which is the double-layer thickness (in meters):

$$1/\kappa = \sqrt{\frac{\epsilon\epsilon_o RT}{2F^2 I \cdot 10^3}} \quad (\text{Eq. 6-26})$$

where,

ϵ = dielectric constant of water (78.5),

ϵ_o = permittivity of free space ($8.853\text{E-}12$ C/V-m),

R = universal gas constant (8.314 J/mol-K),

T = temperature (298.13 K),

F = Faraday constant (96,485 C/mol), and

I = ionic strength in molality.

Ionic strength in these calculations is on the order of 0.01 mol/L. Thus, the double-layer thickness is on the order of 3×10^{-9} m. This value was held constant for all runs because its effect on the calculated final pH is small compared to uncertainties in other parameters, such as the total site concentration.

6.8.3 Water and Gas Compositions

A total of four water compositions were equilibrated with the goethite surface, each at three different fugacities of carbon dioxide: $10^{-1.5}$, $10^{-3.0}$, and $10^{-5.0}$ bars. The four water compositions came from DTN: MO0403SPAIPCHM.004 and include:

- Seepage water reacted with CSNF at 35 years (C31J25)
- Distilled water reacted with CSNF at 35 years (CS_comp3)
- Seepage water reacted with codisposal package at 60 years (D23N25)
- Distilled water reacted with codisposal package at 54 years (CD_comp7).

These compositions are summarized in *SCM modeling summary.xls*, Appendix F. The initial pH values of these waters, listed in Table 6-19, are low because they represent the effects of corrosion of Carbon Steel Type A516 and ignore the effects of surface complexation. Only those components whose concentrations exceeded 1.00E-08 molal were included in the simulations, with the exception of Fe, which was always included regardless of its concentration. Components with total concentrations below 1.00E-08 molal have negligible effects on pH in these waters.

Table 6-19. Initial pH of Waters Prior to Surface Complexation Calculations

| Water | Model | Year | pH |
|----------|---------------|------|------|
| C31J25 | Liquid influx | 35 | 3.29 |
| CS_comp3 | Vapor Influx | 35 | 3.29 |
| D23N25 | Liquid Influx | 60 | 4.70 |
| CD_comp7 | Vapor Influx | 54 | 3.34 |

Source: Appendix F, *Surface_Complexation_initial_water_comps3.xls*.

6.8.4 Surface Complexation Calculations

The surface-complexation calculations were performed using the inputs in Sections 6.8.1 through 6.8.3 and several mineral constraints. Consistent with Table 6-11, gibbsite, hematite, and pyrolusite were allowed to precipitate to saturation. The surface complexation model uses the site concentrations for HFO, which is justified in Section 6.7, but the model demonstration precipitates hematite in order to be consistent with the EQ6 model. Section 6.6.11 examines the impact of forming hematite versus goethite. In addition, calcite and PuO₂(hydr,aged) were

allowed to precipitate to saturation. Calcite only precipitated in the D23N25 seepage run at 60 years and at $\log f\text{CO}_2$ of -1.5. The only mineral allowed to dissolve was schoepite that was allowed to dissolve to saturation due to its abundance in the waste packages. Because the molalities of the various phases forced to saturation were low compared to the surface site molalities, the predicted pH values were insensitive to the minerals allowed to precipitate (e.g., PuO_2 , $\text{PuO}_2(\text{hydr,aged})$, boltwoodite-Na).

The pH values resulting from the surface complexation calculations are presented in Table 6-20. The non-parenthetical pH values were calculated using the *YMP_R3mr.dat* database, which contains all surface complexation reactions in Tables 4-9 and 6-12. Those in parentheses were calculated using the *YMP_R3.dat* database developed in Section 6.3.2 and documented in DTN: MO0403SPAYMPR3.000. These runs and the *YMP_R3mr.dat* database are documented in Appendix F (PHREEQC files).

The pH values of the initial waters in Table 6-19 are for waters in equilibrium with a fugacity of carbon dioxide of 10^{-3} bars. As a result, the effects of surface complexation are best evaluated by comparing these pH values with the pH values in the “ $10^{-3.0}$ bars CO_2 ” column in Table 6-20. The 3.3 to 4.7 range of initial pH values (Table 6-19) is buffered by surface complexation reactions to a pH range of approximately 7.1 to 8.3 (Table 6-20). There are two reasons for this effect. First, the surface acts as a pH buffer (Section 6.7), and second, the surface sites have a high concentration per mass of water (Section 6.8.2). The low pH values of the initial waters in Table 6-17 should never exist because corrosion of the steel is accompanied by the generation of iron (hydr)oxide corrosion products that counteract the acidity generated by corrosion.

pH values increase with decreasing CO_2 fugacity and decrease with increasing CO_2 fugacity, as predicted in Table 6-18. Based on the results using the expanded *YMP_R3mr.dat* database, at $10^{-1.5}$ bars the pH is calculated to fall as low as about 6.5, while at $10^{-5.0}$ bars, it increases to a maximum around 9.0, depending on the waste package type and initial water composition. The abridged *YMP_R3.dat* database calculations give a slightly different pH range of 6.4 to 8.2. The similarity in the pH values predicted by each database suggests that the abridged database contains the generally dominant surface complexation reactions.

The differences between *YMP_R3.dat* and *YMP_R3mr.dat* pH calculations are due to the expanded surface complexation database in *YMP_R3mr.dat*. The additional adsorbates in *YMP_R3mr.dat* are Ba^{2+} , Co^{2+} , Hg^{2+} , Mg^{2+} , Mn^{2+} , NpO_2^{2+} , PuO_2^{2+} , UO_2^{2+} , AsO_4^{3-} , AsO_3^{3-} , F^- , PO_4^{3-} , and SiO_3^{2-} . While the expanded database allows consideration of additional surface complexation reactions, the representations of these reactions in the database are not verified for accuracy, as are the surface complexation reactions in the abridged database (*YMP_R3.dat*).

The pH ranges predicted in these calculations and presented in Table 6-20 are slightly larger than the range predicted by the analytical model in Table 6-18. There are several reasons for this. First, the range in Table 6-20 covers four different corrosion fluids. Table 6-18 covers corrosion of Carbon Steel Type A516 only. Second, the analytic model neglected ionic strength corrections for the primary surface equilibria and ignored exchange of other sorbing ions for protons or hydroxyls, except for carbonate. However, the first-order agreement between the analytic and the full surface complexation model suggests that the primary mechanisms controlling pH are adequately captured in Table 6-18.

According to the results of the *YMP_R3mr.dat* calculations, the following adsorbates account for 1% or more of all surface species in at least one of the simulations: H^+ , CO_3^{2-} , UO_2^{2+} , Ni^{2+} , CrO_4^{2-} , Ca^{2+} , PO_4^{3-} , and SO_4^{2-} . Concentrations of adsorbed UO_2^{2+} are always around 15% to 25% of the concentration of all surface species. Percentages of surface sites adsorbing CO_3^{2-} are a strong function of CO_2 fugacity. At $10^{-1.5}$ bars, around 50% to 60% of all surface sites involve CO_3^{2-} adsorption. This percentage range drops to approximately 15% to 25% at $10^{-3.0}$ bars and to less than 1% at $10^{-5.0}$ bars.

UO_2^{2+} is the only adsorbate not included in *YMP_R3.dat* calculated to occupy at least 5% or more of the surface sites in the *YMP_R3mr.dat* calculations. Thus, UO_2^{2+} adsorption may be largely responsible for the differences between the pH calculations within each cell in Table 6-20. This relationship can be assessed by suppressing only the UO_2^{2+} adsorption reactions from the *YMP_R3mr.dat* database and rerunning the simulations. Suppression of UO_2^{2+} adsorption has the important repercussion of reducing schoepite dissolution, which also affects pH (Section 6.7).

Table 6-20. Equilibrium pH Predicted in Surface Complexation Calculations

| Water | Model | Year | Log fCO_2 = -1.5 | Log fCO_2 = -3.0 | Log fCO_2 = -5.0 | Input Filenames | Output Filenames |
|----------|------------------|------|-----------------------------|-----------------------|-----------------------|----------------------------------|--|
| C31J25 | Liquid influx | 35 | 6.60 (6.59) ¹ | 7.54 (7.56) | 7.98 (7.93) | cs35-15 cs35-30 cs35-50 | cs35-15.out cs35-30.out cs35-50.out |
| CS_comp3 | Vapor Influx | 35 | 6.49 (6.45) | 7.16 (7.37) | 7.53 (7.69) | cd35-15 cd35-30 cd35-50 | cd35-15.out cd35-30.out cd35-50.out |
| D23N25 | Liquid influx | 60 | 7.05 (6.96) | 8.06 (7.95) | 8.31 (8.17) | co60-15 co60-30 co60-50 | co60-15.out co60-30.out co60-50.out |
| CD_comp7 | Vapor Influx | 54 | 7.11 (6.96) | 8.24 (7.85) | 9.05 (8.16) | cod54-15 cod54-30 cod54-50 | cod54-15.out cod54-30.out cod54-50.out |

NOTE: ¹Values in parentheses are calculated using the abridged *YMP_R3.dat*. Preceding values are calculated using the expanded *YMP_R3mr.dat*.

6.9 APPLICATION OF SURFACE COMPLEXATION MODEL TO THE BATCH REACTOR MODEL RESULTS

The surface complexation model is applied only to the CSNF model. Corrosion product surfaces represent an important pH buffer that tends to anchor pH values near neutral. Its effect is most important to consider when pH values are otherwise predicted to depart substantially from neutral. For this reason it was applied for CSNF calculations. The pH values of CDSF effluents are not predicted to be as extreme as those for the CSNF waste packages, so the explicit surface complexation calculation was not included in the model, although buffering of pH will certainly occur.

The low-pH trend observed during the initial 600 years was caused by the dissolution of Carbon Steel Type A516 where sulfur contained in the steel was oxidized and formed sulfuric acid. This period is also characterized by the formation of ferric (hydr)oxide corrosion products; hematite was allowed to form in the simulations because of its long term stability. Runs in which goethite

was allowed to form are documented in Section 6.7 and show much the same behavior in the pH profile. It is during this period that the surface complexation model (Section 6.7) is applied in the pH abstraction.

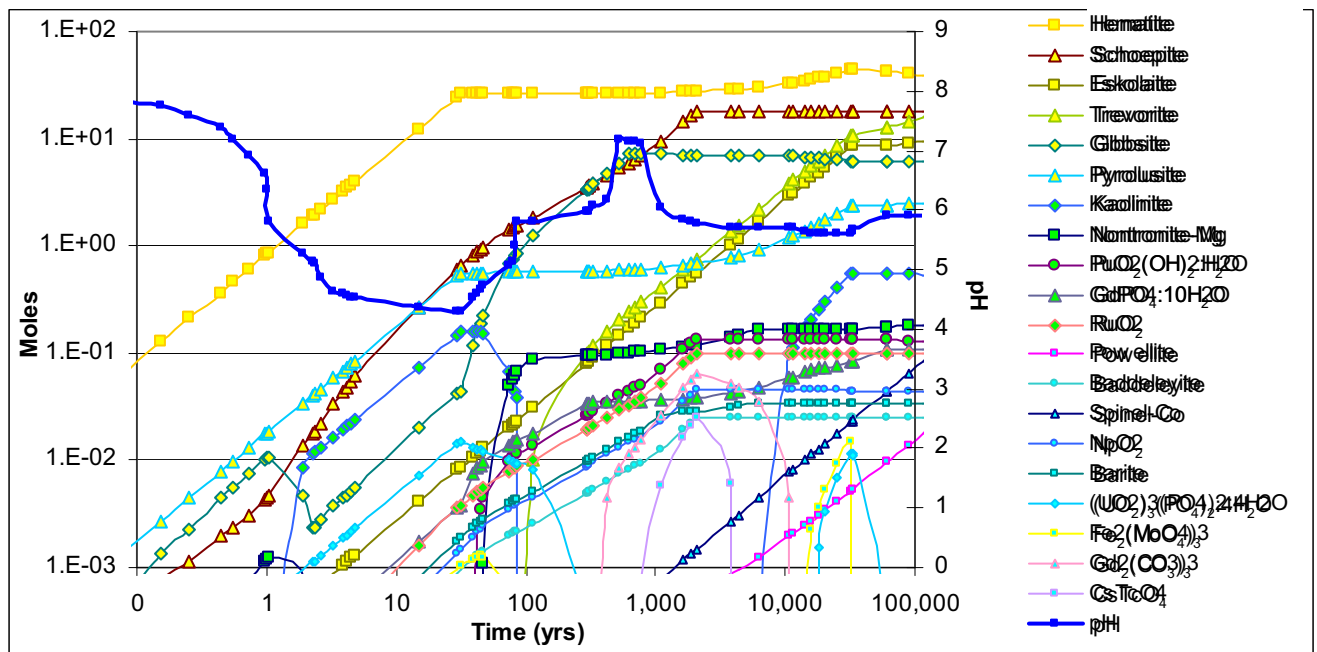
Because the surface complexation effects are accounted for by modifying the pH after the batch reactor results are presented, there is no feedback from the surface complexation model to the batch reactor model. Therefore, the impact of the modified pH on the batch reactor runs must be examined. All of the metal alloys use linear corrosion rates, which are unaffected by pH. The CSNF rate law is a function of pH and tends toward lower values in the neutral pH range. During the first six hundred years, the period when the surface complexation model is employed, the CSNF total degradation rate, from Figure A-1, decreases from approximately $2\text{E-}13$ mol/sec-cm² at pH 4 to $7\text{E-}14$ mol/sec-cm² at pH 7, a decrease of less than a factor of three. Thus, a factor of three less CSNF would dissolve during the first six hundred years, and the total duration that CSNF would be present in the waste package would be extended. The influence on the chemistry of the system would be minimal because schoepite, the primary CSNF degradation product, would still form just in smaller quantities.

During the period when the surface complexation is implemented, the redox potential of the system will vary in accordance with the pH predicted by the surface complexation model. Section 6.10.6 derives Eh as a function of temperature, oxygen partial pressure, and pH; thus, the pH predicted from the surface complexation model will be used to calculate the redox potential for the initial 600 years, and will likely be different from that displayed in the 0- to 600-year period in Figures 6-10 and 6-11. This raises the question whether the low pH and Eh for the first six hundred years are different than displayed in Figures 6-10 and 6-11 and will result in the same mineral assemblage under the new conditions. This question was investigated by using EQ6 and reducing the sulfur of the Carbon Steel Type A516 composition. Sulfur was identified as the cause of the observed pH low in the model output, and its removal, or use of more realistic sulfur oxidation states, would create a pH similar to that predicted by the surface complexation model.

The results of this exercise showed that during the 0- to 200-year period when the Carbon Steel Type A516 was corroding, the system pH values varied between 5 and 8. This is the same pH as was predicted by the surface complexation model (Section 6.7), and the same set of minerals form as did when the pH was controlled by the sulfur contained in the Carbon Steel Type A516. A plot of the minerals and the pH is provided in Appendix A, Figure A-10. This exercise lends confidence to the argument that although the batch reactor model and the surface complexation model are decoupled, the surface complexation model's effect on pH would not influence the corrosion product mineral assemblage. Furthermore, since the Eh is a function of pH, and the pH shown in Figure A-10 is in the same range as that for the surface complexation model (Section 6.7), it follows that there is no impact on the Eh from the decoupling of the models.

These same concepts apply to the period when the surface complexation model is active. During this time, dissolved ions will complex with the surfaces of the ferric (hydr)oxides minerals and deviations in ionic concentrations in the bulk fluid for various input water compositions will be slight for elements that do not originate from in-package materials. Therefore, it is expected that the input water composition will have the same effect on the in-package chemistry with surface complexation as it does when surface complexation is not active.

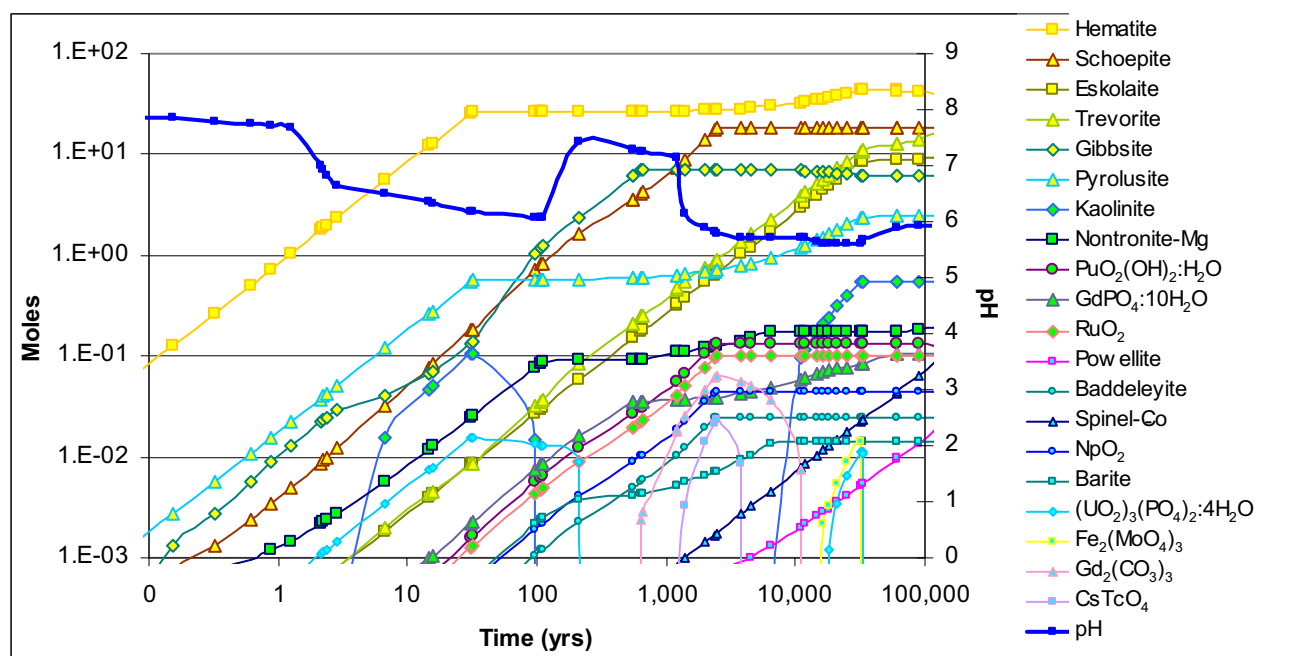
The mineral assemblages for the base case and for the same base case with a pH moderated to a minimum of 4.5 (by using 10× less sulfur in the Carbon Steel Type A516) are shown in Figures 6-68 and 6-69. All of the major minerals are the same. The only differences are slight and include the timing of the formation of certain minerals as well as small changes in the quantities of kaolinite, baddeleyite, barite, and $\text{Fe}_2(\text{MoO}_4)_3$. This comparison shows that the lack of feedback from the surface complexation model to the batch reactor model has minimal effect on the minerals that form.



Source: CSNF Liquid Influx Model Output.xls (Appendix F \Spreadsheets)

Note: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-68. Mineral Assemblage for Base Case



Source: CSNF Liquid Influx Model Output.xls (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-69. Mineral Assemblage for Moderated pH Case

6.10 MODEL ABSTRACTION

The in-package chemistry model is activated within TSPA-LA once the waste package has been breached and the interior temperature of the waste package is below the boiling temperature of water. All references to time within this report refer to the time after waste package breach and after the waste package cools to temperatures lower than 100°C where liquid water is stable. The in-package chemistry model is applicable over the water volumetric flux (hereafter referred to as “flux”) range from 0.001 L/yr to 1000 L/yr per waste package (Section 6.6.5), a temperature range from 25°C to 100°C (Section 6.6.6), a carbon dioxide partial pressure range of 10^{-5} to $10^{-1.5}$ atmospheres (Section 6.6.4), and an oxygen partial pressure up to 0.2 atmospheres (Section 6.6.4). Spatially, the applicability of the in-package chemistry model is limited to the waste package interior.

In the CSNF vapor-influx model, there are no fluoride or chloride abstractions because these elements are not a constituent of the water vapor or in any of the waste package components. Therefore, only the pH and ionic strength are abstracted from the CSNF vapor-influx model; the total carbonate and Eh are calculated as functions of pH and $f\text{CO}_2$. The mapping of batch reactor cases to the abstractions is presented in Table 6-21.

Table 6-21. Mapping of Batch Reactor Cases to Abstractions

| Batch Reactor Cases | Developed Abstractions |
|-------------------------|--|
| CSNF Vapor-Influx Case | pH and ionic strength |
| CSNF Liquid-Influx Case | pH, ionic strength, fluoride and chloride concentrations |
| CDSP Vapor-Influx Case | pH, ionic strength, and fluoride concentrations |
| CDSP Liquid-Influx Case | pH, ionic strength, fluoride and chloride concentrations |

NOTE: The CSNF waste package does not contain any F^- or Cl^- , so for the vapor-influx case that condenses pure water onto the waste package materials, there are no F^- or Cl^- abstractions. The CDSP waste package contains F^- , but no Cl^- .

6.10.1 pH Abstractions

The pH abstractions for the CSNF and CDSP waste packages are described in the following sections. The results of the single component runs were not used in the development of either abstraction, since the purpose of this section is to describe the bulk chemistry of the waste package. Because of the similarities between the vapor influx runs and liquid influx runs for both the CSNF and the CDSP waste packages (as shown in a single plot, Figure 6-20 for the CSNF waste package, and by comparing Figures 6-12 and 6-23 for the CDSP waste package), the CSNF and CDSP pH abstractions are applicable to both the vapor-influx case and the liquid-influx case. Additionally, due to these similarities, the full suite of runs varying all of the key inputs are conducted for the liquid-influx case, and are not repeated for the vapor-influx case. The trends observed by the liquid-influx case are applicable to the trends that would be observed in the vapor-influx case.

6.10.1.1 CSNF pH Abstraction

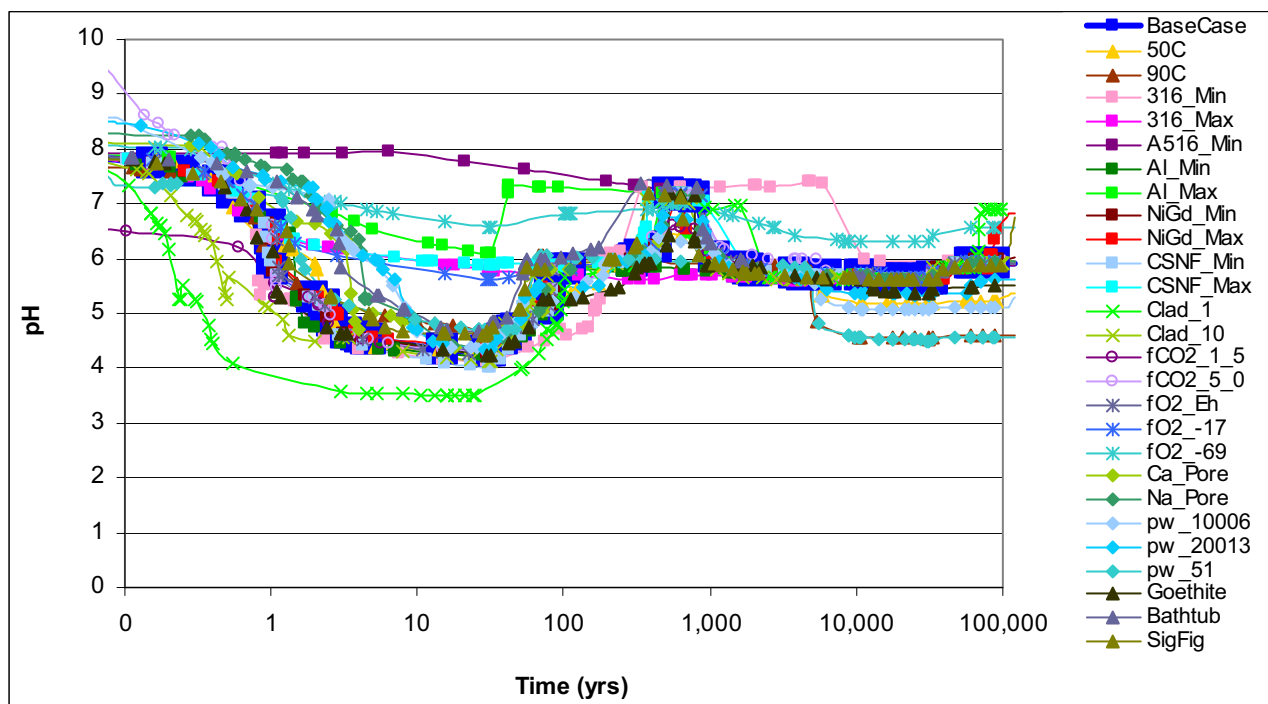
The CSNF pH abstraction is grouped into four abstractions: two time frames: 0 yrs to 600 yrs and 600 yrs to 20,000 yrs, and two flux conditions: flux less than or equal to 150 L/yr and flux greater than 150 L/yr.

The pH history for the full suite of CSNF liquid-influx cases is presented in Figure 6-70 (excluding the cases which vary in the water influx rates, which are shown in Figure 6-41).

As described in Sections 5.2, 6.3.2, and 6.7.1, the long-term condition of the surfaces of the waste package components is uncertain enough that the most realistic approach to anticipating in-package pH values is to simply allow it to vary between the limits set by the surface complexation model and batch reactor model, independent of time. However, Section 6.7.1 states that it is only appropriate to apply the surface complexation model for the first 600 years, thus making the model dependent on time. This time dependence is only a function of when the surface complexation model is applied, and is not a function of the time series pH values calculated by EQ3/6.

For the CSNF waste package, during the period when the surface complexation model is implemented (0 to 600 years), the pH ranges are defined by the surface complexation model as a function of the fugacity of carbon dioxide gas in Table 6-18 (values using 0.1 moles of sites per mole Fe). The maximum pH values for the 0- to 600-year period are the same as the values calculated in Section 6.7.1 (Table 6-18), with no additional uncertainty added. Likewise, the

minimum pH value should also correspond to the values in Table 6-18, however, to account for any uncertainty in when the surface complexation model might really be in effect, the pH minimum was set by the long-term pH minimum of 4.5. Applying a pH minimum of 4.5 for both the 0- to-600-year time frame and the long term also accounts for the uncertainty associated with the steel and alloy corrosion rates. As shown in Figure 6-56, depending on the corrosion rates used, the pH minimum can occur anywhere between 10 and 200 years. This pH is also based on the 90°C simulation displayed in Figure 6-46, where for times greater than 600 years, pH stabilizes around 4.5. Allowing the pH to be sampled down to the long-term pH minimum, which is lower than the pH minimum predicted by the surface complexation model, provides TSPA-LA with a broader pH range intended to capture uncertainty in corrosion rates and the surface areas available for corrosion. In addition, sampling the long-term pH minimum also removes any time dependence on the EQ6 results. As was discussed in Section 6.7.3, there is little surface chemistry data at high temperatures; however, the data that are available support the trend of decreasing pH with increased temperature observed in Figure 6-46. Each distribution should be uniformly sampled, pH values for intermediate values of $\log f\text{CO}_2$ may be linearly interpolated between nearest neighbor values, and no uncertainty term should be added to the sampled value.



Source: CSNF Liquid Influx Model Output.xls (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years). The surface complexation model has not yet been applied to these results.

The maximum rate for the A516 is the same as the mean rate.

Figure 6-70. CSNF Liquid-Influx Model pH Results

For the CSNF waste packages for the period from 600 to 20,000 years, the pH should be uniformly sampled between pH 4.5 and 7. The pH minimum is based on the Cr(VI) sensitivity that shows a lower pH at late times (Section 6.6.3, Figure 6-33) due to the degrading stainless

steel which is the major source of Cr. Though the model uses Cr(III), there is still the potential for some of the Cr to oxidize to Cr(VI), so the pH minimum of 4.5 accounts for that potential. This pH minimum is supported by the long-term pH observed in the high temperature runs (Figure 6-46). Though the minimum long-term pH is more likely to be closer to 5.5 or 6, based on all of the 25°C runs shown in Figure 6-70, and considering the fact that the repository will have cooled significantly by late times, the pH minimum of 4.5 was retained so as to not minimize risk. A lower pH is more conservative in most respects; it results in higher dissolved concentrations as well as high CSNF dissolution rates. The pH maximum of 7 is based on long-term pH observed for the higher liquid-influx cases in Figure 6-41 and the low- fO_2 case in Figure 6-37, with no additional uncertainty term added.

If the flux into a failed waste package exceeds 150 L/year, the maximum pH could reach that of the incoming water, as shown in Section 6.6.5. Therefore, for fluxes greater than 150 L/year, the maximum pH is set at 8.5 for 0 to 600 years, and for 600 to 20,000 years to coincide with the maximum reasonable water that could enter a waste package for a significant duration, based on Figure 6-70. (The pH for the $\log fCO_2 = -5.0$ case is not considered, because the initial high pH is only a function of the solution being equilibrated with that low fCO_2). The reason for the flux dependence is that at high flux values, there is less opportunity for reactions to occur and the pH of the water more closely reflects the pH of the incoming water. The pH minimum is set to 4.5 to coincide with the minimum pH observed in Figure 6-41 or with a reasonable water that could enter a waste package for a significant duration, based on Figures 6.13-2 and 6.13-15 of the *Physical and Chemical Environment* (BSC 2004 [DIRS 169860]). There is no additional uncertainty term added. Figure 6-45 shows that pH increases with increasing liquid influx; therefore, the high influx and the low influx pH abstractions should be sampled in a similar manner, meaning if a high pH is sampled for a low liquid influx, a corresponding high pH should be sampled for a high liquid influx.

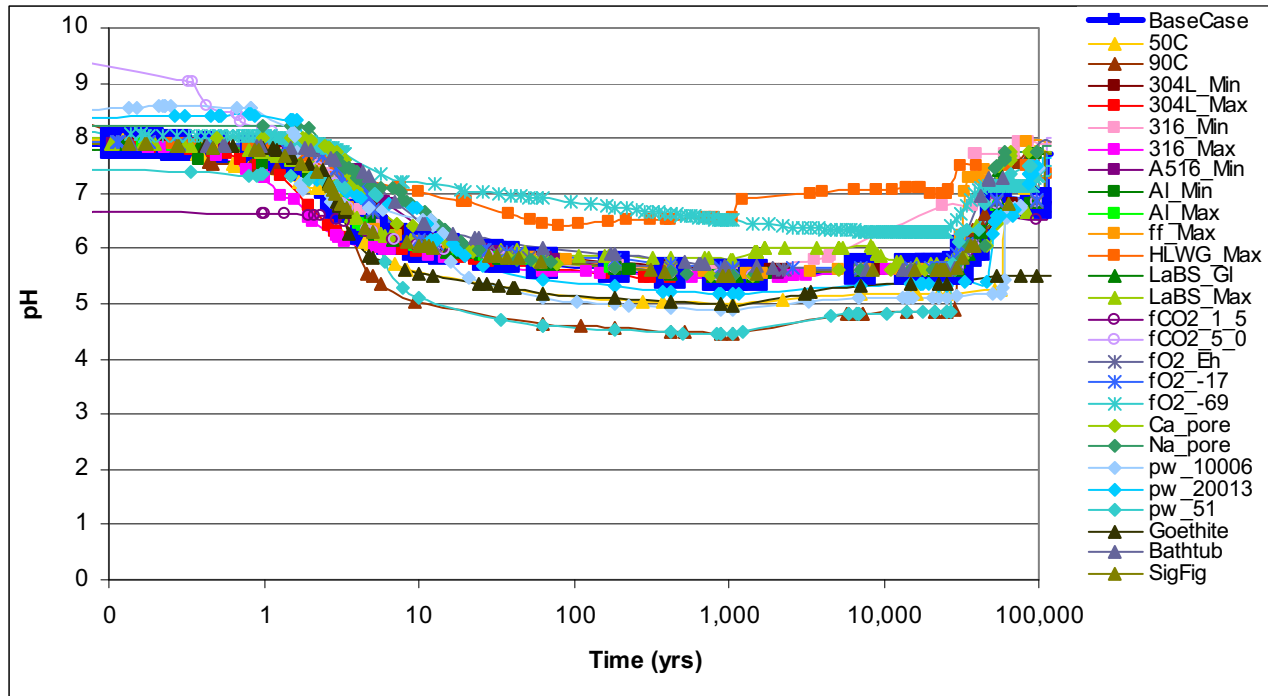
For late times (> 40,000 years), the pH would be less affected by the processes inside the waste package and would be more a function of the incoming water pH. By 40,000 years, most of the reactants that strongly impact the chemistry have degraded, and only the NiGd remains, which has a slow degradation rate. Figure 6-70 shows the long term pH to be between 4.5 and 7, but the runs showing a low pH are the high temperature runs that are modeled at 90°C or above (cases 90C and pw_51). According to Figure 6.3-67 of BSC 2005 [DIRS 173944] the repository has cooled to below 60°C by 20,000 years, so these high temperature cases are not relevant at these late times. Therefore, the expected pH minimum for times greater than 40,000 years is 6, and the expected pH maximum would be set by that of the incoming water.

For naval waste packages that do not contain Carbon Steel Type A516 in their design, the expected pH profiles (pH between 5 and 8) would be near those displayed in Figure 6-55. The **naval nuclear propulsion program** provides a release rate from naval spent nuclear fuel based on this pH range. The use of the CSNF pH abstraction for the transport of the radionuclides from naval spent nuclear fuel is justified because the CSNF pH abstraction encompasses the range of expected in-package pH conditions for naval fuel. In addition, the naval waste packages comprise 300 waste packages compared to 7,772 CSNF packages (BSC 2004 [170022], Table 4-2), any consequence of modeling naval fuel as CSNF, resulting in a pH range extending 0.5 pH units is not likely to be significant.

6.10.1.2 CDSP pH Abstraction

The CDSP pH abstraction is grouped into two flux conditions, flux less than or equal to 150 L/yr and flux greater than 150 L/yr.

The pH history for the full suite of CDSP liquid-influx cases is presented in Figure 6-71 (excluding the cases which vary in the water influx rates, which are shown in Figure 6-43).



Source: *CDSP Liquid Influx Model Output.xls* (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

The maximum rate for the A516 is the same as the mean rate.

Figure 6-71. CDSP Liquid-Influx Model pH Results

For the CDSP waste package, the surface complexation model is not implemented, and the pH ranges for the entire regulatory timeframe should be uniformly sampled between a minimum and maximum pH. The minimum pH is 4.5 is based on the Cr(VI) sensitivity that shows a lower pH at late times (Section 6.6.3, Figure 6-35) due to the degrading stainless steels which is the major source of Cr. Though the model uses Cr(III), there is still the potential for some of the Cr to oxidize to Cr(VI), so the pH minimum of 4.5 accounts for that potential. Though the minimum long-term pH is more likely to be closer to 5.5 or 6, based on the lower temperature runs shown in Figure 6-71, and considering the fact that the repository will have cooled significantly by late times, the pH minimum of 4.5 was retained so as to not minimize risk. A lower pH is more conservative in most respects; it results in higher dissolved concentrations as well as high HLWG dissolution rates. The maximum pH is dependent on the liquid influx rate. As shown in Figure 6-43, at flow rates higher than 150 L/yr, the pH increases back towards that of the incoming water. Therefore, for low flow rates ($Q \leq 150$ L/yr), the maximum pH is 7.0 (Figure 6-39 for the low fO_2 case).

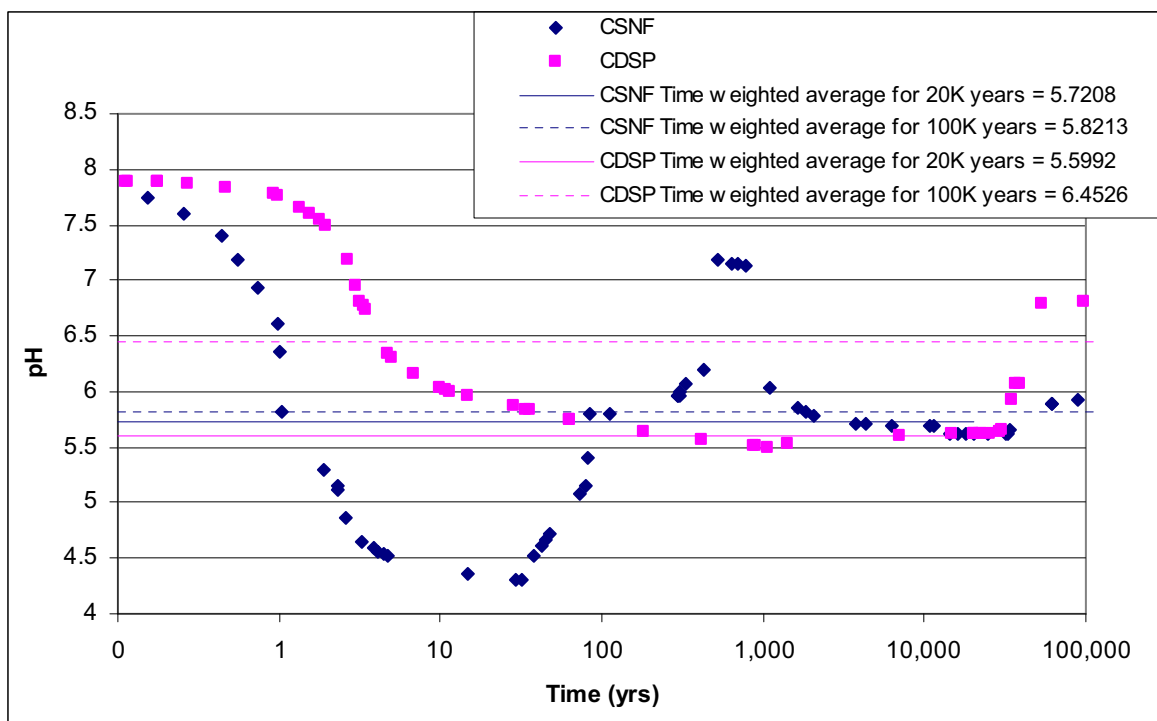
If the flux into a failed waste package exceeds 150 L/year, the maximum pH could reach that of the incoming water, as shown in Section 6.6.5. Therefore, the maximum pH is set at 8.5 to coincide with the maximum reasonable water that could enter a waste package for a significant duration, based on Figure 6-71. (The pH for the $\log f\text{CO}_2 = -5.0$ case is not considered, because the initial high pH is only a function of the solution being equilibrated with that low $f\text{CO}_2$). The pH minimum is set to 4.5 to coincide with a reasonable water that could enter a waste package for a significant duration, based on Figures 6.13-2 and 6.13-15 of the *Physical and Chemical Environment* (BSC 2004 [DIRS 169860]). The reason for the flux dependence is that at high flux values, there is less opportunity for reactions to occur and the pH of the water more closely reflects the pH of the incoming water. There is no additional uncertainty term added. Figure 6-43 shows that pH increases with increasing liquid influx; therefore, the high influx and the low influx pH abstractions should be sampled in a similar manner, meaning if a high pH is sampled for a low liquid influx, a corresponding high pH should be sampled for a high liquid influx.

For late times ($> 40,000$ years), the pH would be less affected by the processes inside the waste package and would be more a function of the incoming water pH. By 40,000 years, most of the reactants that strongly impact the chemistry have degraded, and only the HLWG remains, which has a slow degradation rate. Figure 6-71 shows the long term pH to be between 4.5 and 7, but the runs showing a low pH are the high temperature runs that are modeled at 90°C or above (cases 90C and pw_51) and the run that allows goethite to form. According to Figure 6.3-67 of BSC 2005 [DIRS 173944] the repository has cooled to below 60°C by 20,000 years, so these high temperature cases are not relevant at these late times. In addition, at these late times, the consideration of goethite becomes more questionable, since the more thermodynamically stable hematite will likely dominate. Therefore, the expected pH minimum for times greater than 40,000 years is 6.5, and the expected pH maximum would be set by that of the incoming water.

6.10.1.3 Comparison of CSNF and CDSP pH Abstractions

The in-package chemistry is highly dependent on the waste package type; therefore, there is no correlation between CSNF and CDSP pH abstractions.

The pH profiles for the CSNF and CDSP abstractions are different from one another because of the different materials used in two waste package types, as shown in Table 4-6. Figure 6-72 shows the pH profiles for liquid-influx base case for both CSNF and CDSP waste packages.



Source: *pH Comparison for CSNF and CDSP.xls* (Appendix F).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-72. Comparison of CSNF and CDSP Liquid-Influx pH Profiles

A counter-intuitive observation on the pH profiles is that the CDSP 20,000-year time-weighted pH is lower than that for the CSNF. The dissolution of HLWG elevates pH, as shown in Figure 6-22, but when all the CDSP waste package components are combined in a single run, the resulting pH for the first 20,000 years is actually lower than the CSNF pH for the same duration, due to the low degradation rate of the HLWG and the presence of more stainless steel (higher quantities of 316 than the CSNF waste package and the additional 304L stainless steel not present in the CSNF waste package). For later times, the pH trends are reversed, and the 100,000 year time weighted average pH for the CDSP waste package is higher than that for the CSNF waste package, due to the effects of the HLWG dissolution becoming more of a factor as the other reactants are depleted.

6.10.2 Ionic Strength Abstractions

Based on previous versions of this model, it is known that the most significant factor affecting ionic strength is water flux. Therefore, ionic strength abstractions are calculated as a function of flux. The ionic strength abstraction is a triangular distribution between a minimum, a mode, and a maximum function of ionic strength versus flux. Based on the full suite of EQ6 runs, three cases are selected to represent the minimum, the mode, and the maximum, and these three cases are then run with the entire range of liquid or vapor fluxes. The resulting time-weighted ionic strength for each case is calculated and then abstracted as a function of the base ten logarithm of either the seepage flux or vapor flux, using linear regression in Microsoft Excel using “Add Trendline” from the “Chart” submenu to fit the exponential function to the data (the resulting trendline is shown in Figures 6-74, 6-76, 6-79, and 6-80 as the “Power” function).

The ionic strength results predicted using the EQ6 model were based on water ratios that were adjusted to avoid exceeding the ionic strength limit of the aqueous model; thus, the ionic strength boundary condition was set by the model itself. Compounding this issue are three points. First is the fact that at early times, the ratio of water to reactants is initially small and increases with time, so the ionic strength will initially be high and decrease with time as more water enters the system. Second is the fact that the corrosion rates used in the modeling consume water faster than the liquid or vapor influx rate, resulting in higher ionic strengths. This water consumption (shown in Figure 6-75 for liquid influx and Figure 6-77 for vapor influx) indicates that there is not sufficient water to sustain the maximum corrosion rates examined, or even the average rates in some cases. When there is insufficient water to support corrosion, the corrosion process slows down. Third is that this model does not account for evaporative processes. The principal solutes following evaporation will be those of soluble salts like Cl, Na, Mg, which could increase the ionic strength of the in-package solution.

These three effects are not necessarily captured in the EQ6 model, but are accounted for in the broad uncertainty applied to the ionic strength abstraction. In contrast to the factors above, the ionic strengths predicted by EQ6 are likely to be high as the underlying calculations do not account for removal of dissolved salts from solution by corrosion product sorption or the lower oxidation state the system, which impacts important materials in the waste package. However, the complexities involved with predicting the exact identities and abundances (hence site densities) of the corrosion products over long periods of time, prevents estimation of the magnitude of the effect. For example, complete conversion of corrosion products to low site density hematite over time would lead to ionic strengths closer to the EQ6 prediction.

The possibility of ionic strength being higher than was possible to model with the batch reactor model, or being lower due to sorption reactions or lower oxidation states, is dealt with because the ionic strength abstractions capture extreme ionic strengths by including high water influx rates and high corrosion rates, as described in Sections 6.10.2.1 and 6.10.2.2. The results of the ionic strength abstractions are shown in Figure 6-74, Figure 6-76, Figure 6-79, Figure 6-80.

Unlike the batch reactor conceptual model (Section 6.3.1.1), under vapor influx conditions, the interior of the waste package is dry at the moment of waste package breach. With time, the humidity of the interior of the waste package will equilibrate to the ambient drift conditions and water will begin to adsorb or condense to the interior waste package components. The ratio of water to reactants is initially small and increases with time, so the ionic strength will initially be high and decrease with time as more water enters the system. This is an effect that is not captured by the vapor influx EQ6 model. Therefore, the ionic strength data that show a linear increase as a function of time, <30 years (Figure 6-73 for example), is appropriately weighted by using a time-weighted average for the ionic strength values. The potentially high-ionic strength solutions that could occur based on the low ratio of water to reactants at early times, but not captured in the EQ6 model, are captured by the broad ionic strength abstraction that encompasses high ionic strengths, as shown in the abstractions developed in the following sections.

The ionic strength abstraction for late times is also based on a time-weighted average ionic strength. This approach is justified for the liquid-influx cases because a single TSPA timestep can be much longer than the short-lived (I) peaks shown in Figures 6-73 and 6-78. Also the

uncertainty associated with the timing and duration of the ionic strength peak is difficult to quantify, and is a factor of the stainless steel rates and the surface area of the stainless steels; therefore, it is appropriate to make the abstraction time independent. The use of a time-weighted average ionic strength for the vapor-influx case is justified for these same reasons, and also because the vapor-influx rate is only sampled once per realization, so the abstracted ionic strength is only calculated once per realization, and it would be inappropriate to apply a short-lived ionic strength peak to the entire realization. The time-weighted average ionic strength is further justified in Section 6.10.2.4.

Further complicating the effect of increasing water content is the role of corrosion products and surface complexation on ionic strength. Increasing water content and surface complexation have the same potential impact of decreasing the ionic strength. The applied broad uncertainty range covers this effect. It was demonstrated that the waste package becomes filled with water and mineral phases by about 5,000 to 7,000 years (*WP_volume.xls*, Appendix F). This has no impact on the ionic strength abstraction because the colloid threshold ionic strength, 0.05 mol/L (BSC 2004 [DIRS 170025]), occurs well before this time around 10 to 30 years. The significance of the quantity of water in the model is discussed in more detail in Section 6.10.2.4.

Low ionic strengths (<0.05 mol/L) are conservative for TSPA-LA relative to colloid stability because colloids are stable and can transport radionuclides when the ionic strength is less than 0.05 mol/L (BSC 2004 [DIRS 170025]). High ionic strengths (>1 molal) are conservative for TSPA-LA relative to the dissolved concentrations of radionuclides because TSPA-LA increases the uncertainty applied to the *Dissolved Concentration Limits of Radioactive Elements* (BSC 2005 [DIRS 173873]), increasing the concentrations of radionuclides in solution. Ionic strengths between 0.05 and 1 molal are the least conservative to TSPA-LA relative to the transport of radionuclides. Therefore, the uncertainty applied to the ionic strength should be broad enough to encompass extreme ionic strengths resulting in an overall representative output to TSPA-LA that does not minimize risk.

One of the limitations in the model relevant to the ionic strength abstraction is the B-dot limitation, described in Sections 1 and 6.3.1.1. Because the colloidal suspensions are only stable at ionic strength values below 0.05 mol/L (BSC 2004 [DIRS 170025]), there is no significance in exceeding the 4 molal limits of the B-dot activity coefficient equation. For the *Dissolved Concentration Limits of Radioactive Elements* model (BSC 2005 [DIRS 173873]), TSPA-LA applies an additional uncertainty term to the concentrations if the ionic strengths is greater than one molal, so the significance of exceeding the 4 molal limits of the B-dot activity coefficient equation is already captured.

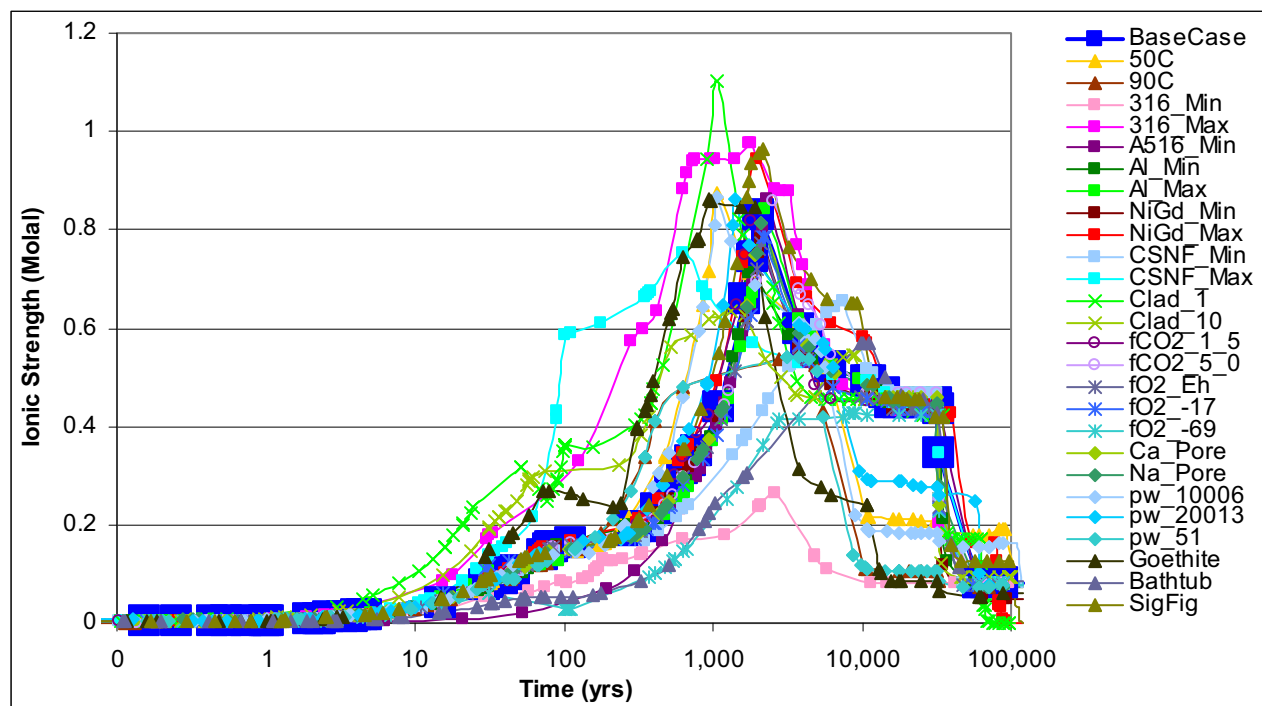
Because of the similarities between the vapor influx runs and liquid influx runs for both the CSNF and CDSP waste packages, (as shown in Figure 6-21 for CSNF waste packages), the full suite of runs varying all of the possible inputs conducted for the liquid-influx case is sufficient to show the model response, and is not repeated again for the vapor-influx case. This similarity also shows that the ionic strength abstraction for the vapor-influx case should take the same form as the liquid-influx case.

The ionic strength abstractions developed in Sections 6.10.2.1 through 6.10.2.2 are summarized and tabulated for use by the TSPA-LA in Section 6.10.2.3.

6.10.2.1 CSNF Ionic Strength Abstractions

6.10.2.1.1 Liquid Influx Abstraction

The ionic strength histories for the full suite of CSNF liquid-influx cases are presented in Figure 6-73 (excluding the cases which vary in the water influx rates, which are shown in Figure 6-42). Because there are so many inputs, each with several values associated with them, each input is varied individually, while keeping the base-case value for the other inputs. The CSNF liquid-influx base case is defined in Section 6.5.1.1. For example, the cases that examine different incoming water compositions, all use the base-case values for flux, corrosion rates for all of the alloys and fuels, cladding exposure, $\log f\text{CO}_2 = -3$, and $\log f\text{O}_2 = -0.7$.



Source: CSNF Liquid Influx Model Output.xls (Appendix F).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

The maximum rate for the A516 is the same as the mean rate.

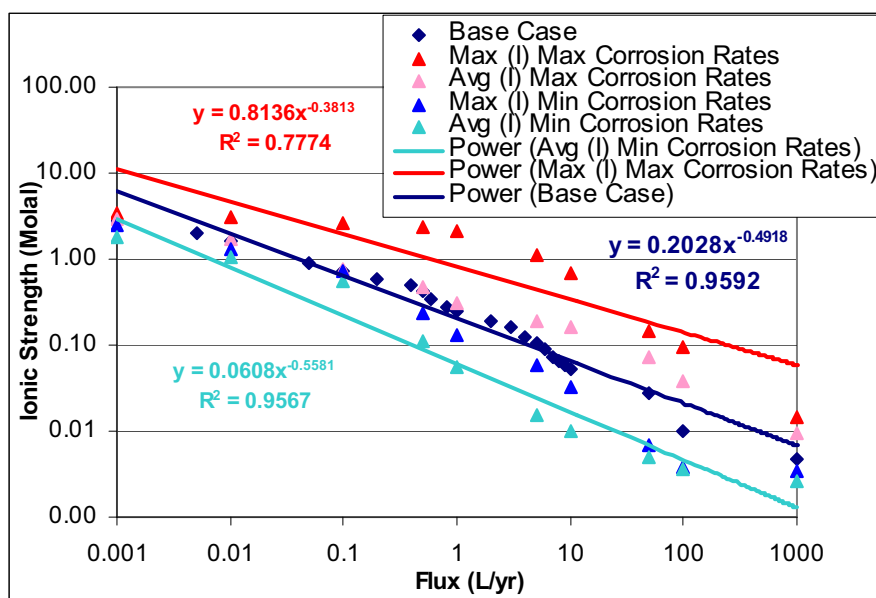
Figure 6-73. CSNF Liquid-Influx Model Ionic Strength Profiles

The runs shown in Figure 6-73 were all conducted at a flux of 1 L/year. The point of this figure is to determine which other factors are important in selecting the minimum, maximum and mode cases, and this figure clearly shows that the next most significant factor in ionic strength is the corrosion rates used for the A516 and 316 steels (from Table 6-5). The cases that examine lower fuel exposures (1% and 10% cladding failure) also have higher ionic strengths, but this is an artifact of the scaling used. The quantity of water is scaled to the reactants based on the surface area, so for the lower fuel exposures, more of the remaining reactants are exposed, such as the A516 and 316. Since the degradation rate is the product of the kinetic rate, and the surface area exposed (Section 6.3.1.3.2), the lower fuel exposure cases model essentially the same as the increased degradation rate cases.

The ionic strength for each of the three cases is calculated as a function of the liquid influx by linear regression, and the results are shown in Figure 6-74, which shows the base-case results as the time-weighted average ionic strength, and the two extreme cases show both the maximum ionic strength for the entire run as well as a time-weighted ionic strength.

These abstractions, were all generated by plotting in Microsoft Excel the time-weighted average ionic strength versus the base ten logarithm of seepage flux and then using “Add Trendline” from the “Chart” submenu to fit the exponential function to the data (the resulting trendline is shown in Figure 6-74, Figure 6-76, Figure 6-79, and Figure 6-80 as the “Power” function).

For the liquid-influx cases, the flux is varied from 0.001 to 1,000 L/yr.



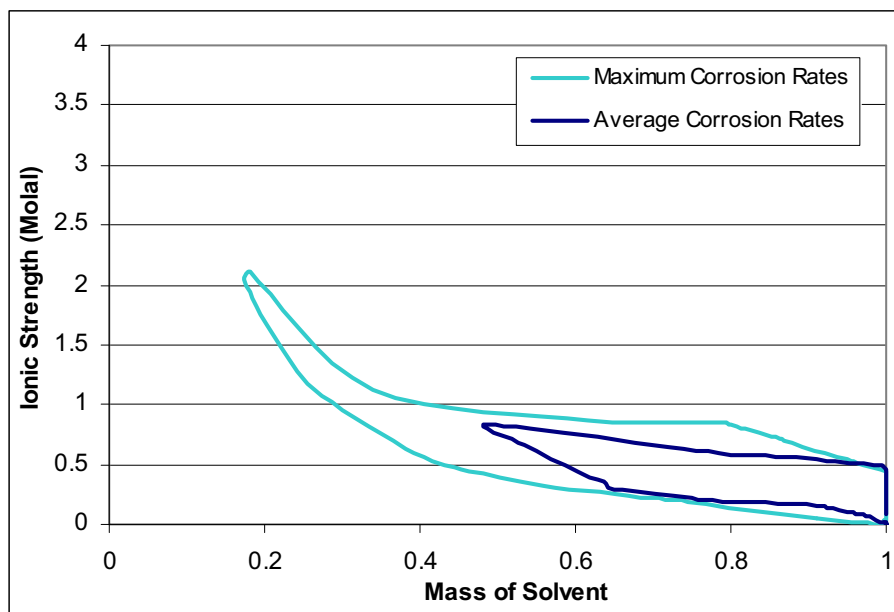
Source: (I) CSNF_Liquid Influx Model Output.xls (Appendix F).

Figure 6-74. CSNF Liquid-Influx Model Ionic Strengths as a Function of Flux

There is some question about the appropriateness of using the maximum corrosion rate case, because as Figure 6-75 shows, there is not sufficient water in the system to sustain the high corrosion rates (or even the average corrosion rates). The corrosion process consumes water, and if there is no water available, the corrosion process slows down. The EQ6 model is scaled to 1 liter of water (1 kg), but as the water is consumed faster than the influx rate, the mass of the solvent decreases below 1 kg. Therefore, the use of the maximum corrosion rate case over estimates the ionic strength. Section 6.10.2.4 discusses the quantity of water in the EQ6 runs as it pertains to the ionic strength abstractions.

The relationship between ionic strength and flux becomes less linear with very high corrosion rates. The reason for this trend is that even with very high corrosion rates, the ionic strength at high fluxes is overwhelmed by the low ionic strength of the incoming water. If the runs had extended out to greater than 1000 liters per year, the ionic strength for all of the cases would converge on the ionic strength of the incoming water. At lower fluxes, the ionic strength for the

high corrosion rate case is not as high as would be expected because the higher concentrations in solution tend to cause minerals to precipitate.



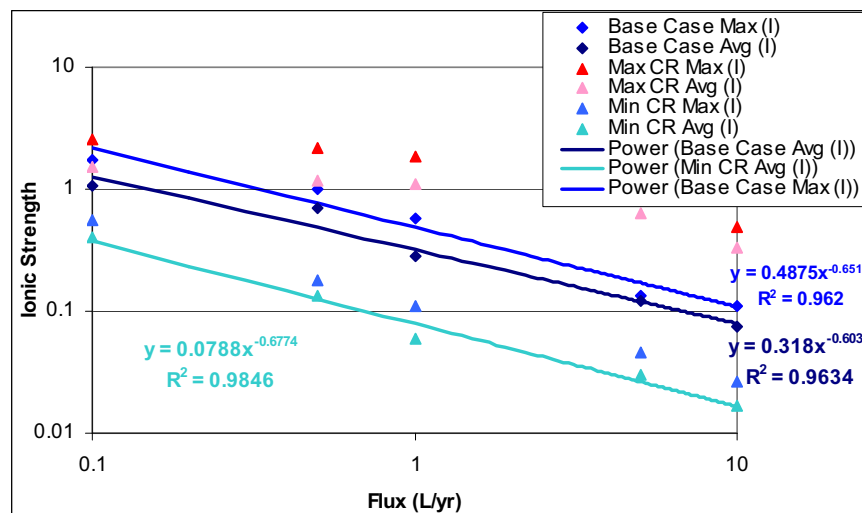
Source: (l) *Abstraction CSNF Liquid Influx.xls* (Appendix F).

Figure 6-75. CSNF Liquid-Influx Model Ionic Strengths as a Function of Mass of Solvent

6.10.2.1.2 Vapor Influx Abstraction

Because of the similarities between the vapor-influx and liquid-influx runs for the CSNF waste packages (as shown in Figure 6-21), the ionic strength abstraction for the vapor-influx case should take the same form (base case = mode; maximum corrosion rates = maximum; minimum corrosion rates = minimum) as the liquid-influx case. However, the EQ6 model is based on 1 liter of water (as described in Sections 6.3.1.1 and 6.10.2.4); this water is available to react with the various components being modeled and can be consumed by corrosion processes. If too much of the water is consumed, due to very high corrosion rates, the remaining quantity of water becomes so low, the system is no longer modeling a dilute aqueous environment, and the range of validity of the model has been exceeded (i.e., when ionic strength exceeds 4 molal). In a real system, if there is not sufficient water to sustain an aqueous environment, the corrosion processes slow down. To model this slowing down of the corrosion rates, the abstraction is based on the average corrosion rates rather than the high corrosion rates that cannot be sustained by the quantity of water available to react. Figures 6-75 and 6-77 show that the maximum corrosion rate case consumes far more water than the average corrosion rate case, artificially driving up the ionic strength. Therefore, the average corrosion rate case is a more reasonable case to use for the ionic strength abstraction.

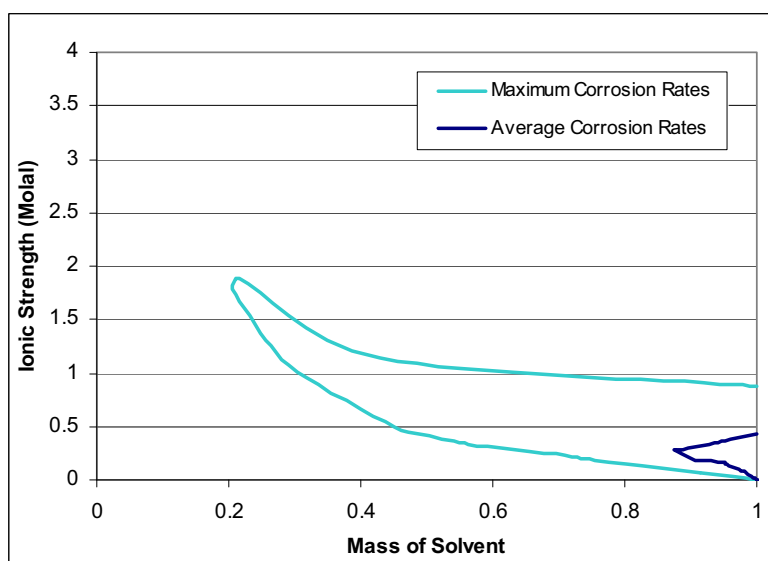
For the vapor-influx case, the flux is varied from 0.1 to 10 L/yr.



Source: (I) CSNF Vapor Influx Model Output.xls (Appendix F).

Figure 6-76. CSNF Vapor-Influx Model Ionic Strengths as a Function of Flux

For the vapor-influx case, the maximum ionic strength was determined from the maximum ionic strength value for the base case, unlike the liquid-influx model, that uses the maximum corrosion rate case. As shown for the liquid-influx case, the maximum corrosion rates consume more water than is available to react, and this effect is shown in Figure 6-77. For the vapor case with high corrosion rates, the high ionic strengths occur when the mass of solvent is well below 1 kg (1 liter), while for the average corrosion rate case, the maximum ionic strength value occurs when the mass is at 1 kg. For the liquid case, both the ionic strength maximums for the maximum corrosion rate case and the average corrosion rate case, occur when the mass of solvent is well below 1 kg (1 liter), as shown in Figure 6-75.



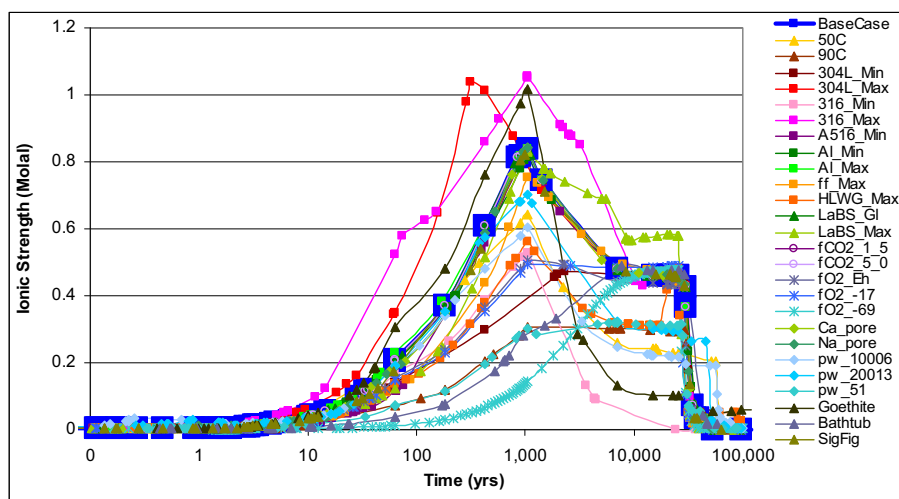
Source: (I) Abstraction CSNF Vapor Influx.xls (Appendix F).

Figure 6-77. CSNF Vapor-Influx Model Ionic Strengths as a Function of Mass of Solvent

6.10.2.2 CDSP Ionic Strength Abstractions

6.10.2.2.1 Liquid Influx Abstraction

The ionic strength histories for the full suite of CDSP liquid-influx cases are presented in Figure 6-78 (excluding the cases which vary in the water influx rates, which are shown in Figure 6-44). Because there are so many inputs, each with several values associated with them, each input is varied individually, while keeping the base-case value for the other inputs. The CDSP liquid-influx base case is defined in Section 6.5.1.2. For example, the cases that examine different incoming water compositions, all use the base-case values for flux, corrosion rates for all of the alloys and fuels, cladding exposure, $\log f\text{CO}_2 = -3$, and $\log f\text{O}_2 = -0.7$.



Source: (I) *CDSP Liquid Influx Model Output.xls* (Appendix F).

Note: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years). The maximum rate for the A516 is the same as the mean rate.

Figure 6-78. CDSP Liquid-Influx Model Ionic Strength Profiles

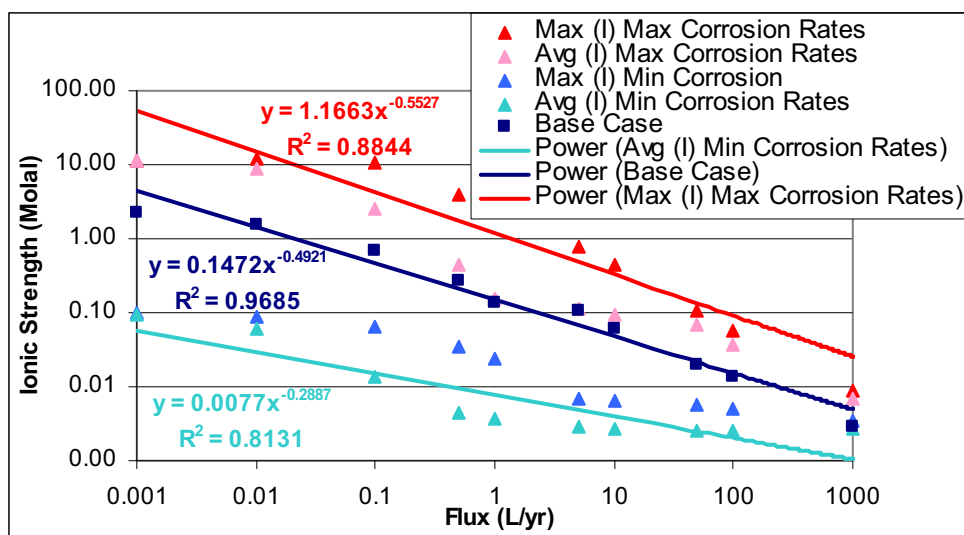
As described in Section 6.10.2.1, the most significant factor affecting ionic strength is flux. The runs shown in Figure 6-78 were all conducted at a flux of 1 L/year. The point of this figure is to determine which other factors are important for selecting which case to represent the minimum, the mode and the maximum, and this figure clearly shows that the next most significant factor in ionic strength is the corrosion rates used for the A516 and 316 steels (from Table 6-5).

Therefore, the liquid influx and vapor influx ionic strength abstractions take the base-case run, the run with the minimum corrosion rates, and the case with the maximum corrosion rates, and simulates these three cases over the full range of influx rates. The ionic strength for each of the three cases is then calculated as a function of the liquid influx by linear regression. These results are shown in Figure 6-79; for the base-case results, the ionic strength is the time-weighted average ionic strength, and the two extreme cases show both the maximum ionic strength for the entire run as well as a time-weighted ionic strength. The abstracted ionic strength is given as a function of the liquid influx for the base case. The maximum abstracted ionic strength is given as a function of the liquid influx for the case using the maximum corrosion rates, and the

minimum abstracted ionic strength is given as a function of the liquid influx for the case using the minimum corrosion rates.

These abstractions, were all generated by plotting in Microsoft Excel the time-weighted average ionic strength versus the base ten logarithm of seepage flux and then using “Add Trendline” from the “Chart” submenu to fit the exponential function to the data.

For the liquid-influx cases, the flux is varied from 0.001 L/yr to 1,000 L/yr.



Source: (I) CDSP Liquid Influx Model Output.xls (Appendix F).

Figure 6-79. CDSP Liquid-Influx Model Ionic Strengths as a Function of Flux

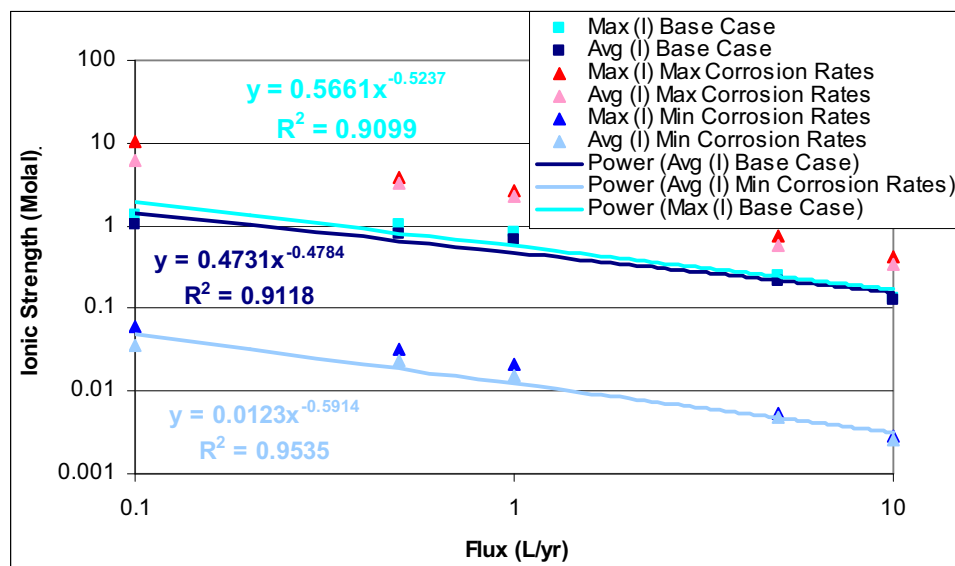
Like the CSNF liquid-influx case, there is some question about the appropriateness of using the maximum corrosion rate case, because as Figure 6-75 shows, there is not sufficient water in the system to sustain the high corrosion rates. The corrosion process consumes water, and if there is no water available, the corrosion process slows down. The EQ6 model is scaled to 1 liter of water (1 kg), but as the water is consumed faster than the influx rate, the mass of the solvent decreases below 1 kg. Therefore, the use of the maximum corrosion rate case over estimates the ionic strength. For the stability of colloids (transport of radionuclides), a low ionic strength is conservative, but since the maximum corrosion rates were used to develop the maximum ionic strength, the colloid threshold of 0.05 mol/L has already been exceeded.

The relationship between ionic strength and flux becomes less linear with very high corrosion rates. The reason for this trend is that even with very high corrosion rates, the ionic strength at high fluxes is overwhelmed by the low ionic strength of the incoming water. If the runs had extended out to greater than 1,000 liters per year, the ionic strength for all of the cases would converge on the ionic strength of the incoming water. At lower fluxes, the ionic strength for the high corrosion rate case is not as high as would be expected because the higher concentrations in solution tend to cause minerals to precipitate.

6.10.2.2 Vapor Influx Abstraction

Because of the similarities between the vapor influx runs and liquid influx runs (as shown in Figure 6-21), the ionic strength abstraction for the vapor-influx case should take the same form (base case = mode; maximum corrosion rates = maximum; minimum corrosion rates = minimum) as the liquid-influx case. However, as described in Section 6.10.2.1.2, the maximum corrosion rate case consumes far more water than the average corrosion rate case, artificially driving up the ionic strength. Therefore, the average corrosion rate case is a more reasonable case to use for the ionic strength abstraction.

For the vapor-influx case, the flux is varied from 0.1 to 10 L/yr.



Source: (I) *CDSP Vapor Influx Model Output.xls* (Appendix F).

Figure 6-80. CDSP Vapor-Influx Model Ionic Strengths as a Function of Flux

As with the CSNF vapor-influx case, the maximum ionic strength was determined from the maximum ionic strength value for the base case, unlike the liquid-influx model, that uses the maximum corrosion rate case. As shown for the CSNF liquid-influx case, the maximum corrosion rates consume more water than is available to react, and this effect is even more prominent for the vapor-influx case, as shown in (I) *CDSP Vapor Influx Model Output.xls* (Appendix F).

6.10.2.3 Form of the Ionic Strength Abstractions

The ionic strength abstractions are a function of the liquid or vapor influx, and take the following form:

$$y = e \cdot x^f \quad (\text{Eq. 6-27})$$

where:

$$y = \text{ionic strength (moles/kg)}$$

e, f = constants (specific to waste package type)
 x = liquid or vapor flux (L/yr)

The abstraction uses a Log Triangular Distribution:

$$y \sim \log \text{tri}(a_{\min}, c_{\text{mode}}, b_{\max}) \quad (\text{Eq. 6-28})$$

and:

$$a_{\min} = e_{\min} \cdot x^{f_{\min}} \quad (\text{Eq. 6-28a})$$

$$c_{\text{mode}} = e_{\text{mode}} \cdot x^{f_{\text{mode}}} \quad (\text{Eq. 6-28b})$$

$$b_{\max} = e_{\max} \cdot x^{f_{\max}} \quad (\text{Eq. 6-28c})$$

The e and f constants for each influx scenario and waste package type are provided in Figures 6-73, 6-76, 6-79, and 6-80.

If the liquid influx rate drops below the sampled vapor influx rate, the vapor-influx model should be used for that case.

For flow rates greater than 1,000 L/yr, the model should be used with a flow rate of 1,000 L/yr.

For the vapor-influx case, the vapor mass flow rate is similar for both the CSNF and CDSP waste packages because the conditions that control the vapor mass flow rate, temperature and RH, are similar throughout the drift where the different waste packages are located. Differences in the CSNF waste package and CDSP waste package temperatures and relative humidities could cause a potential difference in the vapor flux rate because of the temperature dependence of the diffusion coefficient. This difference is expected to be insignificant considering that the sampled flux rate is applied for the entire realization in the TSPA-LA model and, therefore, does not account for reductions in the vapor diffusion rate as the repository cools and differences between CSNF and CDSP temperatures and relative humidities are much smaller. For this reason the same vapor flux rate applied in the CSNF waste package ionic strength abstraction can be applied to the CDSP waste package ionic strength abstraction. For the vapor-influx case, the ionic strength uncertainty is calculated once per realization, since the flux is only sampled once per realization.

For the liquid-influx case, the liquid mass flow rate is output from *Abstraction of Drift Seepage* (BSC 2004 [DIRS 169131]), and is different for each waste package type, because it is dependent on location within the drift, and is calculated with time-dependent variables. For the liquid-influx case, the ionic strength uncertainty is recalculated with each timestep, as a function of the flux, which is calculated with time-dependent variables. However, the random number sampled between 0 and 1 that TSPA-LA uses to sample between the minimum and maximum uncertainty is only sampled once per realization. So, if a given realization applies a high uncertainty (the random number is close to 1), the ionic strength uncertainty will remain high for that entire realization.

The uncertainty treatment of the ionic strength abstraction is broad in order to address the two intended uses of the ionic strength abstractions within TSPA-LA, determining colloids stability and the solubility of radionuclides. Low ionic strengths (<0.05 mol/L) are conservative for

TSPA-LA relative to colloid stability because colloids are stable and can transport radionuclides when the ionic strength is less than 0.05 mol/L (BSC 2004 [DIRS 170025]). High ionic strengths (>1 molal) are conservative for TSPA-LA relative to the dissolved concentrations of radionuclides because TSPA-LA increases the uncertainty applied to the *Dissolved Concentration Limits of Radioactive Elements* (BSC 2005 [DIRS 173873]), increasing the concentrations of radionuclides in solution.

6.10.2.4 Significance of Quantity of water

Using a bulk chemistry model to simulate thin films of water is appropriate, as justified in Section 5.1, but there are ramifications to doing so, particularly relating to modeling ionic strength (as shown in Section 6.6.1, the quantity of water in the EQ6 model has minimal impact on pH). The vapor-influx cases are modeled such that water enters the waste package but does not flow out (to simulate the scenario where water vapor condenses and forms a film of water over the reactants with no advective flow), which is reasonable if the water is consumed by the corrosion processes at the same rate it enters, so there is no accumulation of water. The vapor diffusion into a waste package is calculated (in Section 6.4.3) based on the assumption that once the water enters the waste package, it is immediately consumed by corrosion products, thereby maintaining a strong driving force for more water vapor to diffuse into the waste package. In the EQ6 modeling however, due to the various materials with different degradation rates, and the difficulty in varying the incoming water (or vapor) flux rate within a given run, sometimes water does accumulate (referred to here as “code-induced water accumulation”, where EQ6 calculates that the water influx rate is higher than the rate of consumption of water by corrosion products) or the opposite occurs, and the quantity of water decreases below the liter of water used as the basis for the EQ6 model (EQ6 is an aqueous model scaled to 1 liter of water).

When the ionic strength output from the vapor-influx cases is assessed, the total quantity of water is considered, and for cases where there is “code-induced water accumulation”, the ionic strength values calculated after that point are discounted. These values are discounted for two reasons: first, the values will be artificially low because of the “code-induced water accumulation”, and second, the model has started to model water vapor continuing to diffuse into the waste package and accumulate faster than it is consumed by corrosion, which is a non-physical process for which there is no driving force. For the CSNF vapor-influx base case, the “code-induced water accumulation” begins at about 1,300 years (Appendix F, (I) Abstraction *CSNF Vapor Influx.xls*). For the corresponding CSNF liquid-influx case, the maximum ionic strength is not realized until about 2,000 years (Figure 6-9), which means that the ionic strength for the vapor base case does not reach the same maximum value as the corresponding liquid base case. However, the ionic strength abstraction for the vapor-influx case is higher than the abstraction for the liquid-influx case, because the vapor-influx case is based only on the ionic strength values between 0 yrs and 1,300 yrs, and does not include the lower ionic strengths that are modeled in the later years with “code-induced water accumulation.”

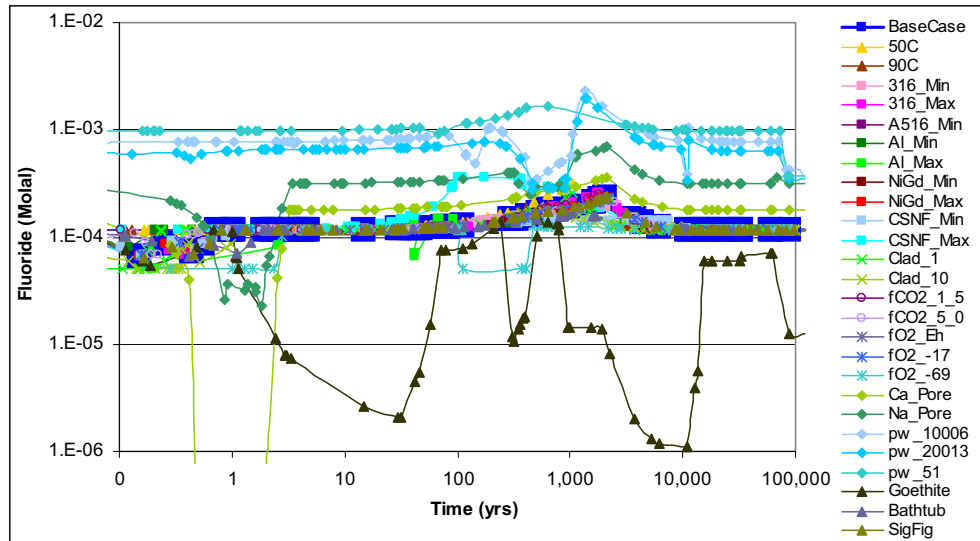
The potential impact of cutting the results short is that the abstracted ionic strength for the vapor-influx case for a given flux is higher than for the corresponding liquid-influx case. However, this is justified because the lower ionic strengths that might occur at later times, due to the depletion of reactants that generate high ionic strengths, are not justified because the run can not sustain those high water fluxes (the water is not being consumed, therefore, there is no driving force for

water to continue in at that rate). So, basing the ionic strength abstraction for a high vapor flux on early times is appropriate because those are the only times during which high water fluxes are possible (while the A516 and other components are degrading fast enough to consume the entire entering flux). For the ionic strength abstraction for lower flux cases, the runs continue out until 18,000 years for the 0.1 L/yr flux, which shows that this flux is reasonable for the entire regulatory timeframe and the time-weighted averaged ionic strength includes that entire time frame.

For the CDSP vapor-influx base case (1 liter per year), “code-induced water accumulation” does not occur until 2,300 years (Appendix F, (I) Abstraction *CDSP Vapor Influx.xls*). Unlike the CSNF case, the maximum ionic strength for the liquid-influx case occurs around 1,000 years, well before the “code-induced water accumulation” in the vapor-influx case, so the abstracted ionic strength for the CDSP vapor-influx case captures the maximum ionic strength.

6.10.3 Fluoride Abstractions

The fluoride profiles for the CSNF liquid-influx model simulations are plotted in Figure 6-81. Other than the input seepage compositions, there are no other sources of fluorine in the CSNF waste package. The dip in fluoride concentration at about one year (Figure 6-81) is the result of fluorapatite precipitation with subsequent dissolution as the concentration recovers to approximately 1E-04 molal. This early fluoride minimum was not considered for the abstraction because the abstraction only applies a maximum value to the fluoride concentration. The maximum fluoride concentrations correspond to the cases with different porewater composition, which contain more fluoride than J-13. It is possible that surface complexation reactions could decrease the effective fluoride concentration of the waste package solutions; however, the surface complexation model was not used to generate the fluoride abstraction. Not using the surface complexation model to moderate the fluoride concentrations is justified because higher fluoride concentrations are conservative to the downstream model (BSC 2005 [DIRS 173873]) that uses this output.



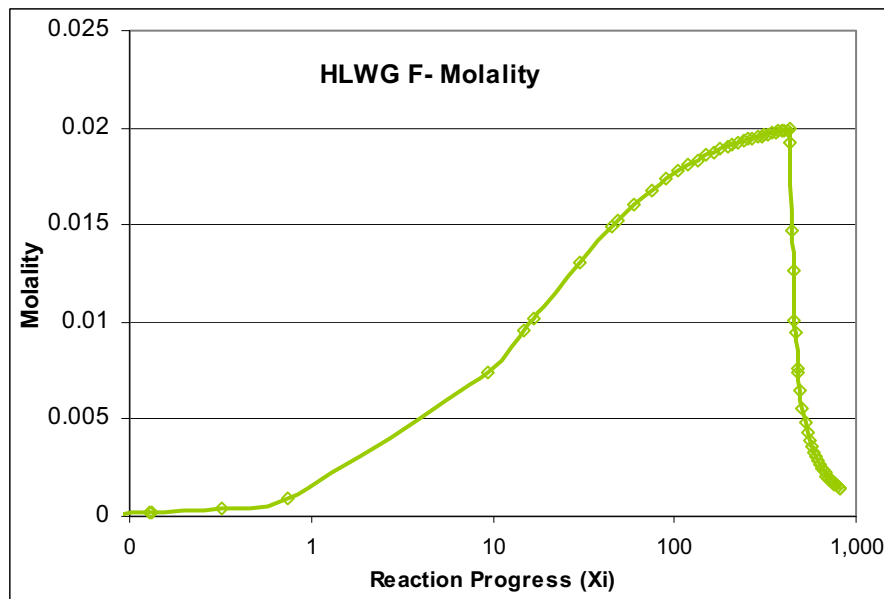
Source: CSNF Liquid Influx Model Output.xls (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-81. CSNF Liquid-Influx Model Fluoride Profiles

To cover the maximum fluoride concentrations in the CSNF waste package, based on the values in Figure 6-81, the maximum fluoride concentration is set at 3E-03 molal. This value is greater than any of the values in Figure 6-81, so it is not necessary to apply an uncertainty value to the fluoride value.

Although there is no dissolved fluoride in the water vapor, the HLWG contains fluorine (Table 4-5); therefore, a fluoride abstraction is provided for the CDSP vapor-influx model.



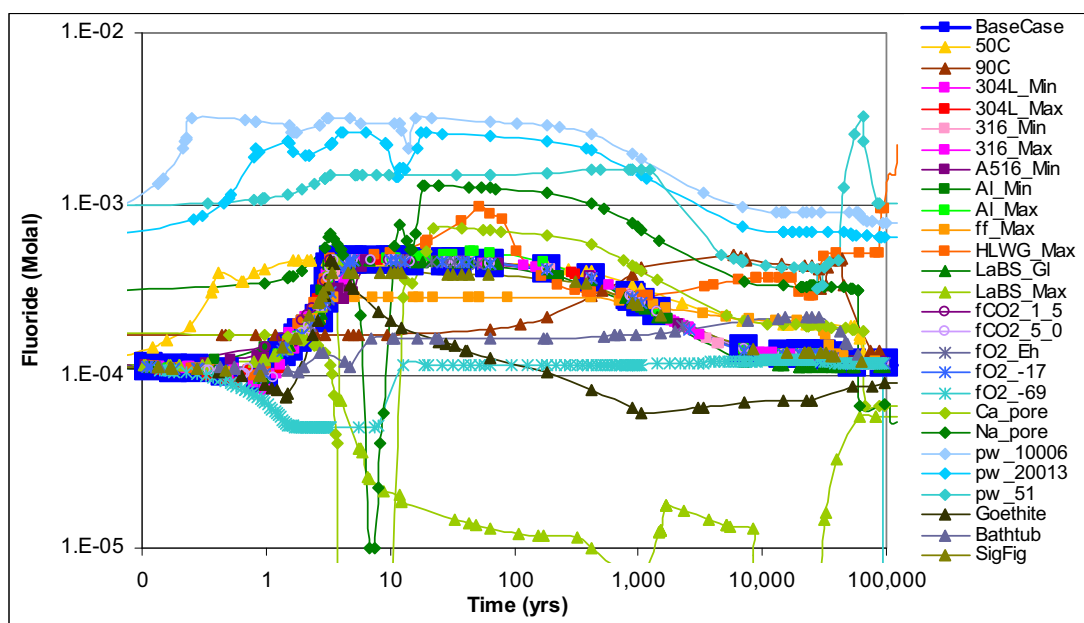
Source: CDSP Vapor Influx Model Output.xls (Appendix F \Spreadsheets).

NOTE: 0 on the x-axis refers to 0.1 Xi.

Figure 6-82. CDSP Vapor-Influx Model Fluoride Profiles

The fluoride profile for the CDSP vapor-influx model simulation is plotted in Figure 6-82. In the vapor-influx model, the only source of the fluoride is in the glass, so the HLWG single component run is used for this abstraction. To cover the maximum fluoride concentrations in the CDSP waste package, based on the values in Figure 6-82, the maximum fluoride concentration is set at 2E-02 molal.

The CDSP liquid-influx model simulations are plotted in Figure 6-83. The variation in the fluoride concentration (Figure 6-83) is due to precipitation/dissolution reactions of minor fluoride bearing phases (fluorite and fluorapatite) at early times (note the dip in fluoride concentrations around 1 year), and the depletion of the HLWG at later times. It is possible that surface complexation reactions could decrease the effective fluoride concentration of the waste package solutions; therefore, the values provided should be considered maximums. The minimum fluoride spike at about 5 years was not included in the abstraction because of its short duration. Compared to CSNF fluoride profiles (Figure 6-81), CDSP fluoride profiles are much more complex because the pH-dependent HLWG dissolution, of which fluoride is a component, controls aqueous fluoride concentration (Table 6-8). It is possible surface complexation reactions could decrease the effective fluoride concentration of the waste package solutions; therefore, the values provided should be considered maximums.



Source: *CDSP Liquid Influx Model Output.xls* (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

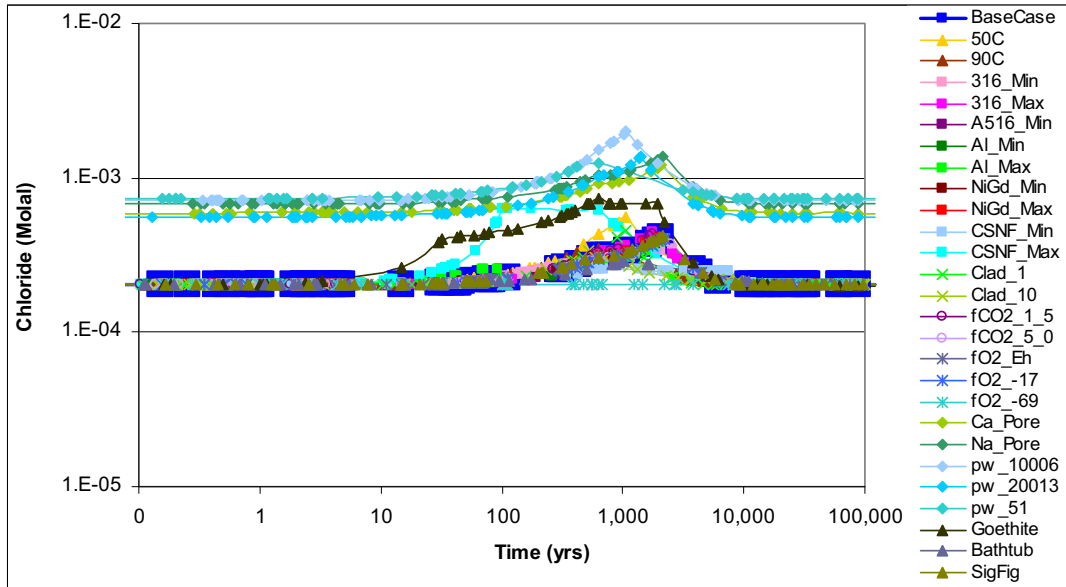
Figure 6-83. CDSP Liquid-Influx Model Fluoride Profiles

To cover the maximum fluoride concentrations in the CDSP waste package, based on the values in Figure 6-83, the maximum fluoride concentration is set at 4E-03 molal. This value is greater than any value in Figure 6-83 therefore; it is not necessary to apply an uncertainty value to the fluoride value.

6.10.4 Chloride Abstractions

Figure 6-84 shows the CSNF liquid-influx model chloride profiles. It is possible that surface complexation reactions could decrease the effective chloride concentration of the waste package solutions; therefore, the values provided should be considered maximums.

CDSP liquid-influx model chloride profiles are plotted in Figure 6-85. It is possible that surface complexation reactions could decrease the effective chloride concentration of the waste package solutions; therefore, the values provided should be considered maximums.

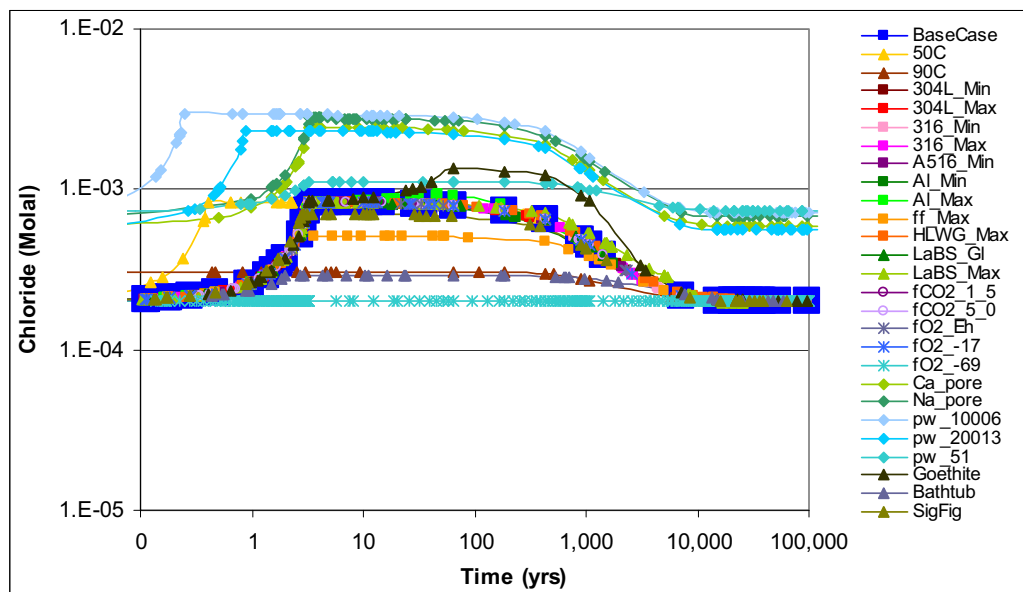


Source: CSNF Liquid Influx Model Output.xls (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-84. CSNF Liquid-Influx Model Chloride Profiles

To cover the maximum chloride concentrations in the CSNF waste package, based on the values in Figure 6-84, the maximum chloride concentration is set at 3E-03 molal. This value is greater than any value in Figure 6-84 therefore; it is not necessary to apply an uncertainty value to the chloride value.



Source: CDSP Liquid Influx Model Output.xls (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 6-85. CDSP Liquid-Influx Model Chloride Profiles

To cover the maximum chloride concentrations in the CDSP waste package, based on the values in Figure 6-85, the maximum chloride concentration is set at 4E-03 molal. This value is greater than any value in Figure 6-85 therefore; it is not necessary to apply an uncertainty value to the chloride value.

6.10.5 Total Carbonate Abstraction

Total carbonate is used in the kinetic rate expression for the dissolution of CSNF; therefore, abstracted values are needed for the TSPA-LA. The bulk solution total carbonate concentration should not be effected by carbonate surface complex reactions because of the large reservoir of atmospheric carbon dioxide available will maintain equilibrium with the bulk solution. In a system where the partial pressure of carbon dioxide (CO₂) is constant over the modeled period and the pH and temperature known, the total carbonate can be calculated using the equilibrium mass action expressions.

Where the total carbonate (0 < pH < 14) is equal to:

$$\Sigma C = [CO_2(aq)] + [HCO_3^-] + [CO_3^{2-}] \quad (\text{Eq. 6-29})$$

$$CO_2(g) \Leftrightarrow CO_2(aq) \quad \log K = k_1 \quad [CO_2(aq)] = PCO_2(g) 10^{k_1} \quad (\text{Eq. 6-29a})$$

$$CO_2(aq) \Leftrightarrow H^+ + HCO_3^- \quad \log K = k_2 \quad [HCO_3^-] = (PCO_2(g) 10^{k_1} 10^{k_2}) / 10^{-pH} \quad (\text{Eq. 6-29b})$$

$$HCO_3^- \Leftrightarrow H^+ + CO_3^{2-} \quad \log K = k_3 \quad [CO_3^{2-}] = [(PCO_2(g) 10^{k_1} 10^{k_2}) / 10^{-pH}] 10^{k_3} / 10^{-pH} \quad (\text{Eq. 6-29c})$$

Note : PCO_2 is the partial pressure of CO_2 (which is equivalent to fCO_2 for the purposes of this report).

Combining terms and substituting back into expression (Eq. 6-29), the total carbonate for the system is equal to the expression in Table 6-22.

Table 6-22. Total Carbonate as a Function of pH

| Equation of Total Carbonate (mol/kg) |
|--|
| $Total\ C = PCO_2 (10^{k_1} + 10^{(pH + k_1 + k_2)} + 10^{(2pH + k_1 + k_2 + k_3)})$ |

Source: *Total Carbonate and Eh Abstractions.xls* (Appendix F \Spreadsheets).

NOTE: k is defined as $\log K$, and PCO_2 is the partial pressure of CO_2 (in atmospheres). For the purposes of this report and ranges of this model (25°C to 100°C and up to 4 molal solutions), mol/L is considered equivalent to mol/kg, and PCO_2 is equivalent to fCO_2 .

Since the equilibrium constants for the carbonate species vary as a function of temperature, it is possible to derive expressions for K_1 , K_2 , and K_3 that include this functionality. The thermodynamic data from 0°C to 100°C for the carbonate species was extracted from *data0.ymp.R4* (DTN: SN0410T0510404.002 [DIRS 172712]) and is compiled in Table 6-23. K_1 (Table 6-23) is the result of combined expressions.

Table 6-23. Log K Values for the Carbonate Species at Temperature

| K | Reaction | Log K at Temperature | | | |
|----------------|-----------------------------------|----------------------|----------|----------|----------|
| | | 0°C | 25°C | 60°C | 100°C |
| K ₁ | $CO_2(g) + H_2O = H^+ + HCO_3^-$ | -7.6765 | -7.8136 | -8.0527 | -8.3574 |
| | $HCO_3^- + H^+ = CO_2(aq) + H_2O$ | + 6.5804 | + 6.3447 | + 6.2684 | + 6.3882 |
| | $CO_2(g) = CO_2(aq)$ | -1.0961 | -1.4689 | -1.7843 | -1.9692 |
| K ₂ | $CO_2(aq) + H_2O = HCO_3^- + H^+$ | -6.5804 | -6.3447 | -6.2684 | -6.3882 |
| K ₃ | $HCO_3^- = CO_3^{2-} + H^+$ | -10.6241 | -10.3288 | -10.1304 | -10.0836 |

Source: DTN: SN0410T0510404.002 [DIRS 172712]

Log K temperature expressions were derived by plotting the log K values from 0° to 100°C (Table 6-24) versus their respective temperatures and fitting a polynomial to the data in Microsoft Excel. This is the same method used by the EQPT code when it compiles the log K grid of an EQ3/6 data0 file (Daveler and Wolery 1992 [DIRS 103803], p. 12). Therefore, consistency is maintained between the approach used in the model report and that used in EQ3/6.

Plots of the log K temperature interpolations are included in *Total Carbonate and Eh Abstraction.xls* (Appendix F \Spreadsheets). The expressions in Table 6-23 are valid for temperatures in the range of 0°C to 100°C. This range is larger than that for the pH abstractions (25°C to 100°C) in Section 6.10.1.

Table 6-24. Log K Temperature Interpolation Functions for Total Carbonate Abstraction

| Log K | Log K expression | R ² |
|-------|--|----------------|
| k_1 | $\text{Log } K_1 = 7\text{E-}05T^2 - 0.0159T - 1.1023$ | 0.9992 |
| k_2 | $\text{Log } K_2 = 5\text{E-}07T^3 - 0.0002T^2 + 0.0132T - 6.5804$ | 1.0 |
| k_3 | $\text{Log } K_3 = -8\text{E-}05T^2 + 0.0128T - 10.618$ | 0.9977 |

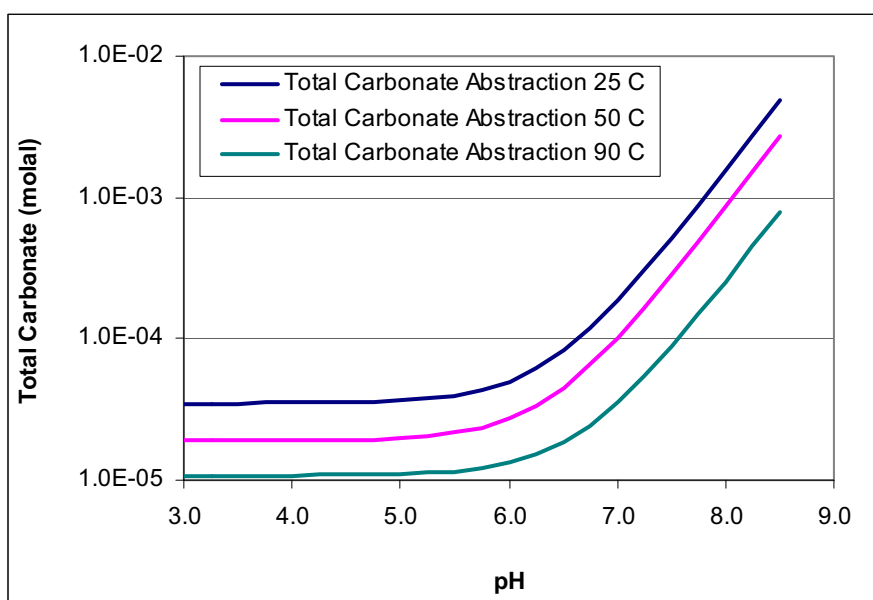
Source: *Total Carbonate and Eh Abstraction.xls* (Appendix F \Spreadsheets).

NOTE: k is defined as $\log K$.
T = temperature in °C.

Three feeds are required for the total carbonate abstraction, as it should be implemented in TSPA-LA. The first feed is the temperature required to calculate the appropriate log K values. The second feed is the in-package pH estimated using the relationships and conditions specified in Section 6.10.1. The third input is PCO_2 . Once the log K values and in-package pH are calculated, they can be used to calculate the total carbonate via the expression in Table 6-22.

Since the pH abstractions account for uncertainty, there is no need to introduce further uncertainty at the level of the total carbonate abstraction. Therefore, the carbonate abstraction can be used as is with no uncertainty term added to it.

Figure 6-86 shows the carbonate abstraction calculated at three temperatures



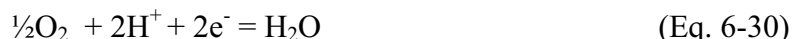
Source: *Total Carbonate and Eh Abstraction.xls* (Appendix F \Spreadsheets).

Note: $\text{Log } f_{\text{CO}_2} = -3.0$

Figure 6-86. Total Carbonate Abstraction at Three Temperatures

6.10.6 Eh Abstraction

In aqueous systems in equilibrium with a constant partial pressure of oxygen, the Eh may be calculated directly from the pH. For the formation of water, the pE° may be calculated from the following expression, where ΔG° is the Gibbs energy of formation, F is the Faraday constant (23.06 kcal/volt-gram equivalent), Eh is the electron activity expressed in units of volts, and pE is the negative log base 10 of electron activity.



$\Delta G^\circ = -56.688$ kcal/mol (DTN: SN0410T0510404.002 [DIRS 172712])

$$\Delta G^\circ = -nFEh^\circ \quad (\text{Eq. 6-31})$$

(Drever 1982 [DIRS 100725], p. 254, Equation 11-6)

$$pE = F/(2.303RT)Eh \quad (\text{Eq. 6-32})$$

(Drever 1982 [DIRS 100725], p. 254)

Solving Equation 6-32 for Eh and substituting into Equation 6-31 results in Equation 6-33, establishing a relationship between ΔG° and pE° :

$$\Delta G^\circ = -2.303nRT(pE^\circ) \quad (\text{Eq. 6-33})$$

Where, n = number of electrons [2], R = Gas Constant (1.987E-03 kcal/mol·K), PO_2 is the partial pressure of oxygen (in atmospheres), and T = absolute temperature (Kelvin).

Solving for pE° with temperature dependency results in:

$$pE^\circ = 6193.973 / T \quad (\text{Eq. 6-34})$$

Thus, for the formation of water in terms of one electron mole:



$$pE = pE^\circ + \log(PO_2^{1/4} [H^+]) \quad (\text{Eq. 6-36})$$

$$pE = 6,193.973 / T - pH + 0.25 \log(PO_2) \quad (\text{Eq. 6-37})$$

Converting to Eh gives the expression in Table 6-25. The expression for Eh as a function of pH, oxygen partial pressure, and temperature (Table 6-25) is applicable for both CSNF and CDSP waste packages. The temperature should be input in degrees Kelvin, and the output is in volts.

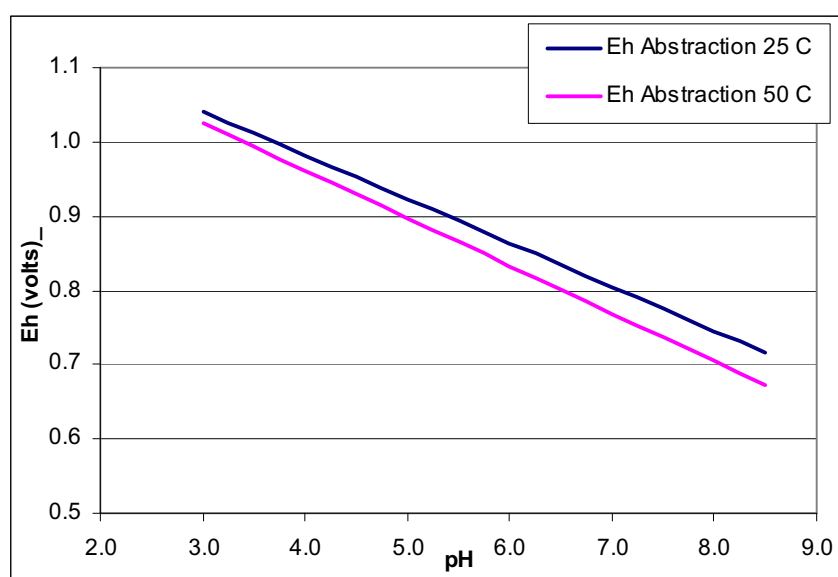
Table 6-25. Eh as a Function of pH, PO₂, and Absolute Temperature

| Equation of Eh (volts) |
|--|
| $Eh = 2.303 R T / F(6193.973 / T - pH + 0.25 \log PO_2)$ |

Source: *Total Carbonate and Eh Abstractions.xls*
(Appendix F \Spreadsheets).

PO₂ is the partial pressure of O₂ (in atmospheres)

Figure 6-87 shows the Eh abstraction function plotted. Again, fluids are expected to be uniformly oxidizing. Variations in Eh come from pH shifts. Like the carbonate abstraction, uncertainty is built into the Eh abstraction by virtue of the input pH, which already has associated uncertainty.



Source: *Total Carbonate and Eh Abstraction.xls* (Appendix F \Spreadsheets).

Figure 6-87. Eh Abstraction at Two Temperatures

6.10.7 Linking the Vapor-Influx and Liquid-Influx Models for TSPA-LA

In a potential repository situation, one could imagine a scenario where a waste package could be breached under vapor influx conditions, and then at some future time, it is possible that seepage might also enter this same waste package and react with the remaining materials. The opposite sequence might also occur, where seepage enters a waste package and at some future time, the seepage might cease and vapor influx conditions would prevail. This would be a scenario where the vapor-influx and the liquid-influx models would need to be linked. In Section 6.5.2, it was demonstrated that the initial composition of the seepage entering a breached waste package did not exert a significant influence on the resulting in-package chemistry. As Figures 6-20 and 6-21 demonstrate, the results of the liquid-influx model follow similar behavior as the ensemble outputs from the vapor-influx model.

Based on the comparison of the results in Figure 6-20 and Figure 6-21, it is recommended that for the purposes of TSPA-LA, should it be necessary to link the vapor-influx model and the liquid-influx model, the change from one model to the other should occur synchronously. If a package is initially receiving water vapor and at some later time, seepage begins to enter the package, then the vapor influx abstraction should cease and the seepage abstraction should begin at that point in time.

6.10.8 Output Uncertainty

The in-package chemistry model combines two approaches to model uncertainty: (1) application of a factorial design approach to account for known large potential variations in model input (reactant combinations, water flux, fuel exposure, temperature, and seepage composition); and (2) sensitivity analysis of lesser known, not as well-defined input variations (carbon dioxide partial pressure, Carbon Steel Type A516 sulfur content, corrosion rates, and extreme temperatures and flux values). Where (1) provided the functional basis of the in-package chemistry model abstraction, (2) provided the uncertainty ranges of the abstracted parameters. Thus, the information to be used in TSPA-LA directly incorporates uncertainty exterior to the in-package environment (1) and interior to the in-package environment in a form that can be readily implemented in TSPA-LA (i.e., model uncertainty is propagated through the abstractions). Thus, the only restrictions on the subsequent use of the in-package chemistry model abstraction in TSPA-LA are that all of the abstractions presented in Section 6.10 must be applied within the stated limits (flux, temperature, $f\text{CO}_2$, and fuel exposure) as specified in this report.

6.10.9 Model Abstraction Conclusions

The inputs to the batch reactor model are scaled according to their surface area, and the base case considers the waste package void space to be approximately half filled with water. The other half of the void space is filled with air in equilibrium with the drift atmosphere (Section 6.3.1). Sensitivity studies were conducted that varied the ratio of water to air (Section 6.6.1) and the composition of the air in contact with the water (Section 6.6.4). Therefore, the chemical effects of the void space (composition and quantity) have been investigated and incorporated into the model.

The batch reactor model generally considers the redox potential to be fully oxidizing, and this modeling assumption is discussed in Section 6.3.1.1. The redox potential is varied in a sensitivity analysis in Section 6.6.4 by using lower fixed $f\text{O}_2$ values, including the adjusted Eh model used in the *Dissolved Concentration Limits of Radioactive Element* model (BSC 2005 [DIRS 173873]), all the way down to $\log f\text{O}_2 = -69$. The sensitivity shows that the pH and ionic strength results are sensitive to the $f\text{O}_2$ values, so the results of this sensitivity were used to develop the pH and ionic strength limits.

The batch reactor model generally allows reactions to reach thermodynamic equilibrium, and this modeling assumption is discussed in Section 6.3.1.1 and the most important species, chromium species, are examined in more detail in Section 6.6.3. This sensitivity examines the impact of allowing kinetics to control the oxidation state, allowing only Cr(III) species to form. The sensitivity shows that the ionic strength results are sensitive to the oxidation state of chromium.

The sensitivity studies involving changes in design (Sections 6.6.7, 6.6.8, 6.6.9, and Appendix A) show that minor changes to the waste package or waste form design will not impact the results of this model. This conclusion applies to the design changes that call for the replacement of Neutronit with a NiGd alloy, or the addition of a purge port.

The above discussion addresses the subject matter of “Included” FEPs 2.1.01.02.0B (Interactions between co-disposed waste), 2.1.02.09.0A (Chemical effects of void space in waste package), 2.1.09.01.0B (Chemical characteristics of water in waste package), 2.1.09.06.0A (Reduction-oxidation potential in waste package), 2.1.09.07.0A (Reaction kinetics in waste package), 2.1.09.02.0A (Chemical interaction with corrosion products), 2.1.11.08.0A (Thermal effects on chemistry and microbial activity in the EBS), and 2.2.08.12.0B (Chemistry of water flowing into the waste package) and indicates how that subject matter has been included and addressed within TSPA-LA.

7. VALIDATION

The purpose of the in-package chemistry model and the in-package chemistry model abstraction is to predict the bulk chemistry inside of a breached waste package and to provide simplified expressions of that chemistry for use in TSPA-LA. The in-package chemistry model is made up of two main models: the batch reactor model, and the surface complexation model. The batch reactor model is made up of two water-influx models, the liquid-influx and the vapor-influx models, and several other component submodels, including the fuel-degradation and the steel-degradation submodels. Based on these models, several output model abstractions were generated in Section 6.10, including: the total carbonate, fluoride, chloride, and Eh models. The technical work plan (BSC 2005 [DIRS 173246], Table 2-3) states that the postdevelopment validation of the model will focus on the three submodels. The fuel-degradation submodel will be validated by comparison with results from laboratory experiments published in refereed journals or literature. The types of minerals formed and the aqueous concentrations predicted by the model will be compared to experimental results to quantify the uncertainty range of the model. The steel-degradation submodel will be validated based on results published from laboratory experiments on man-made analog materials. The surface complexation model will be validated by technical review. The technical work plan (BSC 2005 [DIRS 173246]) does not contain validation criteria for three additional model outputs, so this model deviates from the technical work plan by providing these criteria in Table 7-1.

Typical validation efforts outlined in the technical work plan and cited above were subsequently considered to provide insufficient confidence for predicting in-package chemistry over the much longer time spans to which the model is to be applied. Consequently, in addition to the validation methods outlined in the technical work plan, model validation efforts focused on natural analogue observation that might provide confidence in model predictions over near geologic time spans.

The primary outputs of the in-package chemistry model abstraction to total system performance assessment are ranges of pH and ionic strength. Adequate validation of the in-package chemistry model, therefore, entails showing on the basis of natural analogue evidence that the pH and ionic strength ranges are comparable.

The salient critical features of the in-package chemistry model are:

1. Alkaline waters (pH > 7, HLWG from Figure 6-22) formed by interaction of dilute solutions with waste form glass dissolution
2. Acidic waters (pH < 7, Figure 6-17 and Figure 22) formed by interaction of incoming solutions with certain waste package components
3. Production of high ionic strength solutions (greater than 1 M) and low ionic strength solutions (less than 0.05 M) by reaction with waste form and waste package components (Section 6.10.2)
4. Buffering to intermediate pH values via interaction of fluids with corrosion products, (i.e., surface complexation) (Table 6-18).

7.1 DOCUMENTED DECISIONS AND ACTIVITIES IMPLEMENTED DURING MODEL DEVELOPMENT PROCESS

The technical work plan (BSC 2005 [DIRS 173246], Table 2-3) indicates that the in-package chemistry model requires a medium level of confidence (Level II) because of the impact of pH, one of the main outputs of the in-package chemistry model, on plutonium and neptunium solubility.

Achieving Level I criteria requires a single postdevelopment model validation method. Achieving Level II criteria requires satisfying Level I criteria and providing rationale for the selection of that postdevelopment model validation method.

Satisfying Level I criteria entails demonstrating that the model development efforts include:

- (a) *Evaluating and selecting input parameters and data:* Section 4.1 provides the inputs used in the in-package chemistry model, which include waste package design information, material compositions, atomic weights, densities, water compositions, waste form compositions, corrosion or reaction rates, and thermodynamic data. These inputs were chosen to best represent all of the components of the in-package chemistry model. Therefore, this criterion has been met for all of the salient features.
- (b) *Formulating defensible assumptions and simplifications:* Sections 5, 6.3.1, and 6.3.2 provide the assumptions (Section 5) and simplifications (Sections 6.3.1 and 6.3.2) used in the in-package chemistry model. All model assumptions and simplifications have been shown to be either conservative or captured within the uncertainty of the model. Therefore, this criterion has been met for all of the salient features of the model.
- (c) *Ensuring consistency with physical principles, such as conservation of mass, energy, and momentum:* Sections 6.3, 6.3.1, and 6.3.2 provide brief discussions of the EQ6 geochemical modeling software used for the in-package chemistry model simulations. EQ6 conserves mass, both of solids and water; therefore, the in-package chemistry model also conserves these properties. This criterion has been met for all of the salient features.
- (d) *Representing important future state (aleatoric), parameter, and alternative model uncertainties:* Unexpected random (aleatoric) events are outside of the realm of the in-package chemistry model. However, the in-package chemistry model was designed to be robust enough to handle large changes in temperature, flux, and seepage composition. Alternative conceptual models were discussed in Section 6.4 and the current in-package chemistry model robustness eliminates the need for these alternative conceptual models. Therefore, this criterion has been met for all of the salient features.
- (e) *Ensuring simulation conditions have been set up to span the range of intended use and the avoidance of inconsistent outputs:* Sections 6.3.1 and 6.3.2 outline the wide range of inputs (flux, fuel exposure, temperature, reactant combinations) that were used in the in-package chemistry model. These conditions span the range of intended use in TSPA-LA for which the in-package chemistry model will be used. Therefore, this criterion has been met for all of the salient features.

- (f) *Ensuring that model predictions (performance parameters) adequately represent the range of possible outcomes, consistent with important uncertainties:* Sections 6.3.1 and 6.3.2 provide the input conditions that span the range of the intended use of the model; thus, logic dictates that the range of outputs (performance parameters) also represents the range of possible outcomes. Additionally, sensitivity analyses have been performed to expand the performance parameters to values consistent with important conditional uncertainties. Therefore, this criterion has been met for all of the salient features.

7.2 POST-MODEL DEVELOPMENT VALIDATION METHODS

Level II validation includes one method of postmodel development validation consistent with a model of moderate importance to mean annual dose and documentation of the rationale for selection of postmodel development activities.

7.3 RATIONALE FOR SELECTION OF VALIDATION METHODS

The technical work plan (BSC 2005 [DIRS 173246], Table 2-3) names three methods for validation, different methods for different aspects of the model report, and these methods are listed in Table 7-1. This model deviated from the TWP (BSC 2005 [DIRS 173246]) for the validation of the fluoride, chloride, Eh and total carbonate model. The TWP does not list any validation criteria for these models, so new validation criteria are provided for them in Table 7-1.

Table 7-1. Validation Criteria, Activities, and Metrics

| Validation Criteria / Activities | Metric / Criteria |
|---|---|
| Is the fuel-degradation model consistent with results from laboratory experiments published in refereed journals or literature? (VA 1, 3) | The criteria for meeting all of the corroborating validation activities will be qualitative |
| Are the types of minerals formed and the aqueous concentrations predicted by the model consistent with experimental results? (VA 1) | |
| Is the steel-degradation model consistent with results from laboratory experiments on man-made analog materials? (VA 1) | |
| Because the output will be applied over geologic time spans, are the specific outputs of this model (pH, ionic strength) consistent with natural analogue observation (rather than experimental)? (VA 3) | |
| The surface-complexation portion of the model will be validated by independent technical review, and this review will address the following two points: (VA 5) <ul style="list-style-type: none"> Is the surface complexation modeling approach technically correct? Does it adequately capture the likely processes in the degraded waste form and in the waste package internal components? | Independent technical review: assessment of the validation activities will be qualitative, and considered successful if deemed defensible by the reviewer |
| Is the fluoride model output consistent with experimental data? (VA 1) | The criteria for meeting all of the corroborating validation activities will be qualitative, and considered successful if the model is consistent with and / or bounds the corroborating data |
| Is the total carbonate model output consistent with published literature? (VA 3) | |
| Is the Eh model output consistent with experimental data? (VA 1) | |

NOTE: VA is validation activity as defined in LP-SIII.2Q-BSC, Section 5.3.

For the fuel-degradation and the steel-degradation portion of the model, the results were compared to experimental data from laboratories because the laboratory experiments were

conducted in various environments intended to capture the possible range of conditions anticipated to exist in the repository.

Comparison of the existing data with information about either natural or man-made analogs was determined to be the most appropriate method to validate the model results regarding the types of minerals that are anticipated to form in the waste package prior to and following any postulated breaches. This selection was based on the conclusion that data regarding the long-term environmental performance (i.e., 10,000 or more years) addressed by the model would be difficult to duplicate in the laboratory and the use of analogs presented the best opportunity to provide a meaningful comparison with the model outputs of interest (i.e., pH, ionic strength data) that are developed and discussed in Section 6.10.

The validation of the surface complexation model introduces additional challenges due to the relatively new analytical approach to buffering pH that is used in the model. The surface complexation model is based in large part on the most readily available experimental data regarding this subject, and the limited, if any, availability of information regarding the surface complexation of natural or man-made analogs. As a result, the use of an independent technical review by an individual with appropriate expertise was deemed to be the most appropriate method of validation for the surface complexation model. Dr. Yifeng Wang, Principal Member of Technical Staff, Sandia National Laboratories, a recognized expert in the field of geochemistry, with a Ph.D. from the Dept. of Geological Sciences, Indiana University, was selected to review and report on the model developed herein. Dr. Wang's independent review was conducted in accordance with the requirements of *Technical Work Plan for Postclosure Waste Form Modeling* (BSC 2005 [DIRS 173246]) and the results are reported in Appendix E.

For the fluoride, chloride, Eh and total carbonate portions of the model, the results were compared to experimental data from laboratories because the laboratory experiments were conducted in various environments intended to capture the possible range of conditions anticipated to exist in the repository.

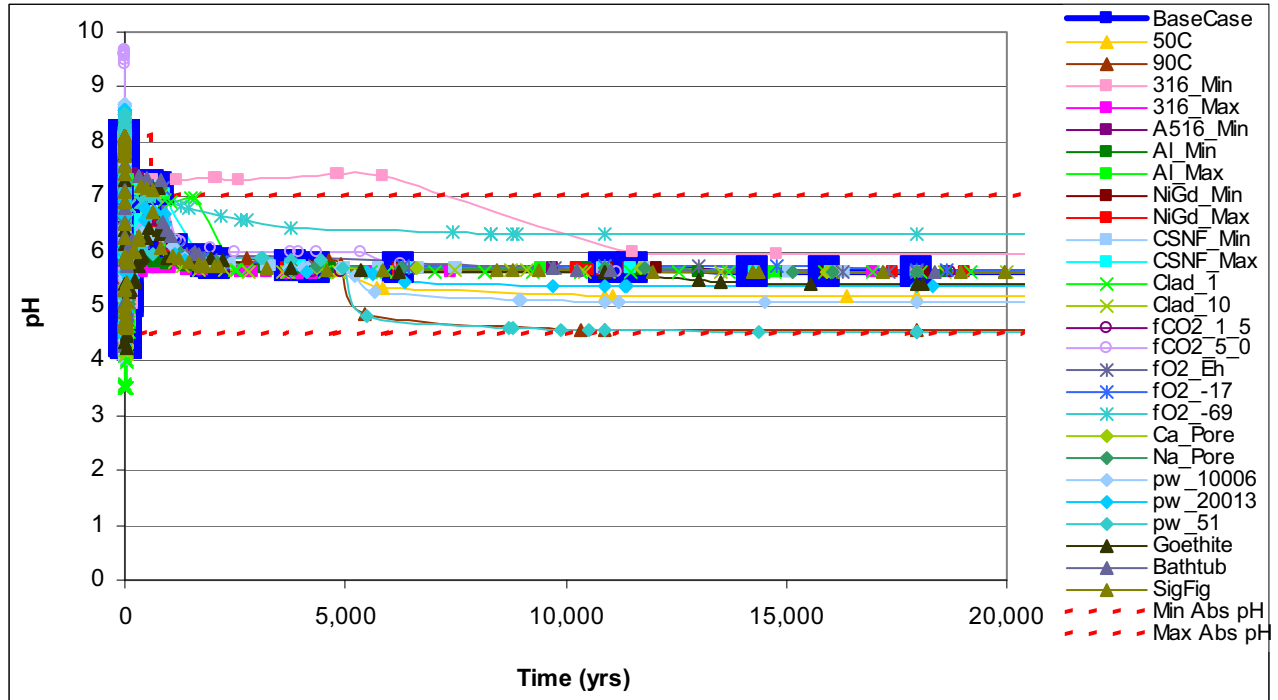
7.4 pH AND IONIC STRENGTH VALIDATION

7.4.1 Abstraction Validation

This abstraction model uses mathematical relationships and statistical distributions to fit product output (i.e., product output is used as input for the abstraction). Thus, product output was used to generate the abstraction model (i.e., to generate the mathematical relationships and statistical distributions that will be used in TSPA-LA). Therefore, the acceptance criterion, for the abstraction relationships, is that they reflect the product output used in their generation. In other words, validation of the abstraction model requires comparison of abstracted parameter values to their corollary product output parameter values. It must be emphasized that this does not constitute validation of the product output, but only that the abstraction can reproduce a subset of the process model output through the use of mathematical relationships.

The abstracted pH is plotted with the model output for the CSNF waste package in Figure 7-1, which shows that the model abstraction accurately captures the model behavior. Note that the pH extends outside the abstracted range between 0-600 years, because the effects of surface complexation are not included in the batch reactor results.

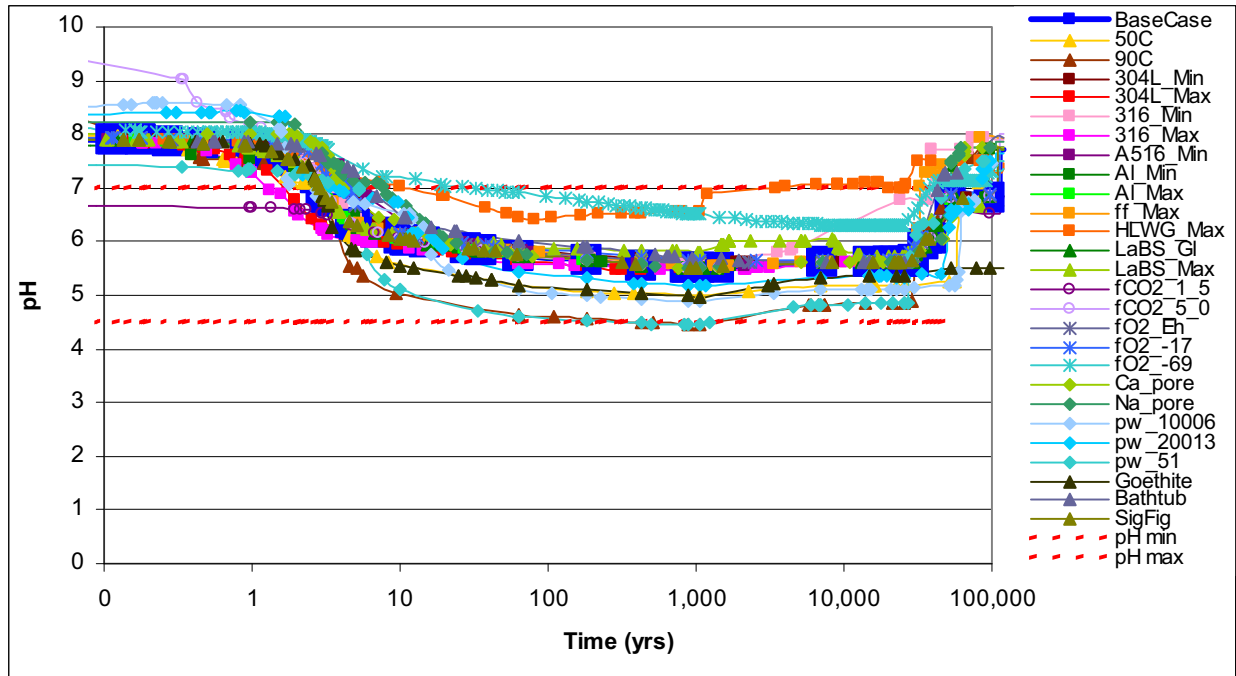
The abstracted pH is plotted with the model output for the CDSP waste package in Figure 7-2. Figure 7-2 shows that the model abstraction accurately captures the model behavior. Note that the pH extends above the abstracted range between 0-10 years, because of the incoming water composition, but this effect is quickly overwhelmed by the corrosion of the waste package internals. Because TSPA-LA samples a pH value once per simulation, it would not be appropriate to allow a transient pH value (the high pH values lasting less than 1 year) to be sampled. The pH also extends above the abstracted range, but this occurs well past the 10,000-year regulatory period. Also, for high flux cases, the abstracted pH maximum extends up to 8.5.



Source: CSNF Liquid Influx Model Output.xls (Appendix F \Spreadsheets).

Note: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years). The maximum rate for the A516 is the same as the mean rate.

Figure 7-1. CSNF pH Abstraction Compared to Model Output

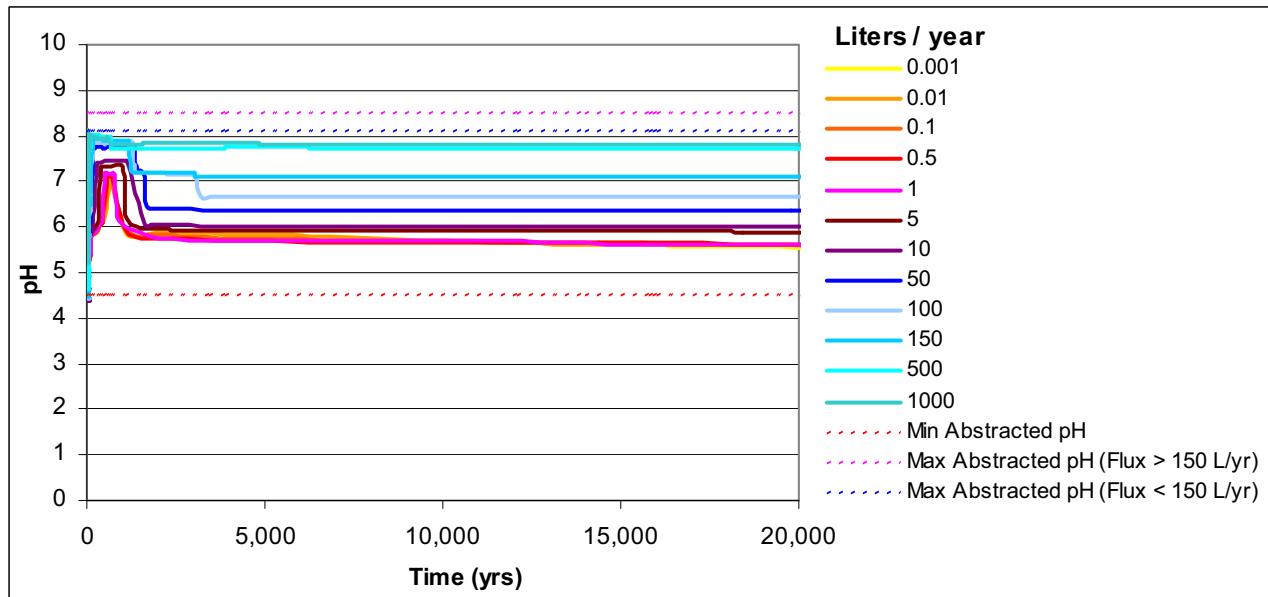


Source: *CDSP Liquid Influx Model Output.xls* (Appendix F \Spreadsheets).

Note: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years). The maximum rate for the A516 is the same as the mean rate.

Figure 7-2. CDSP pH Abstraction Compared to Model Output

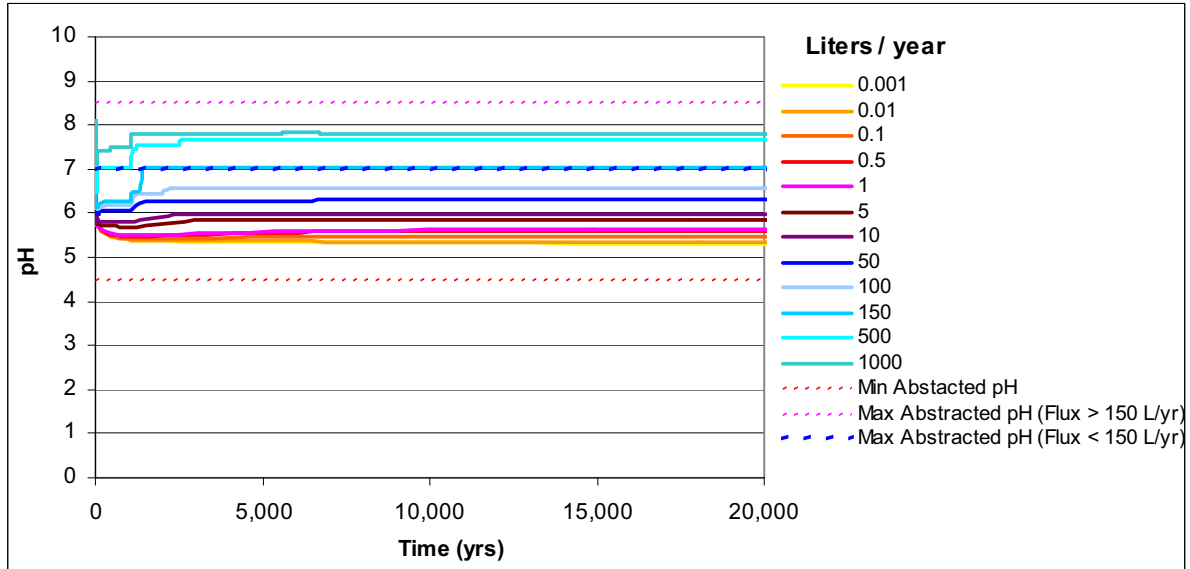
Comparisons of the model abstraction to the model output for the high flux pH abstractions are shown in the following figures.



Source: (I) *Abstraction CSNF Liquid Influx.xls* (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 7-3. CSNF pH Abstraction for High Flux Compared to Model Output

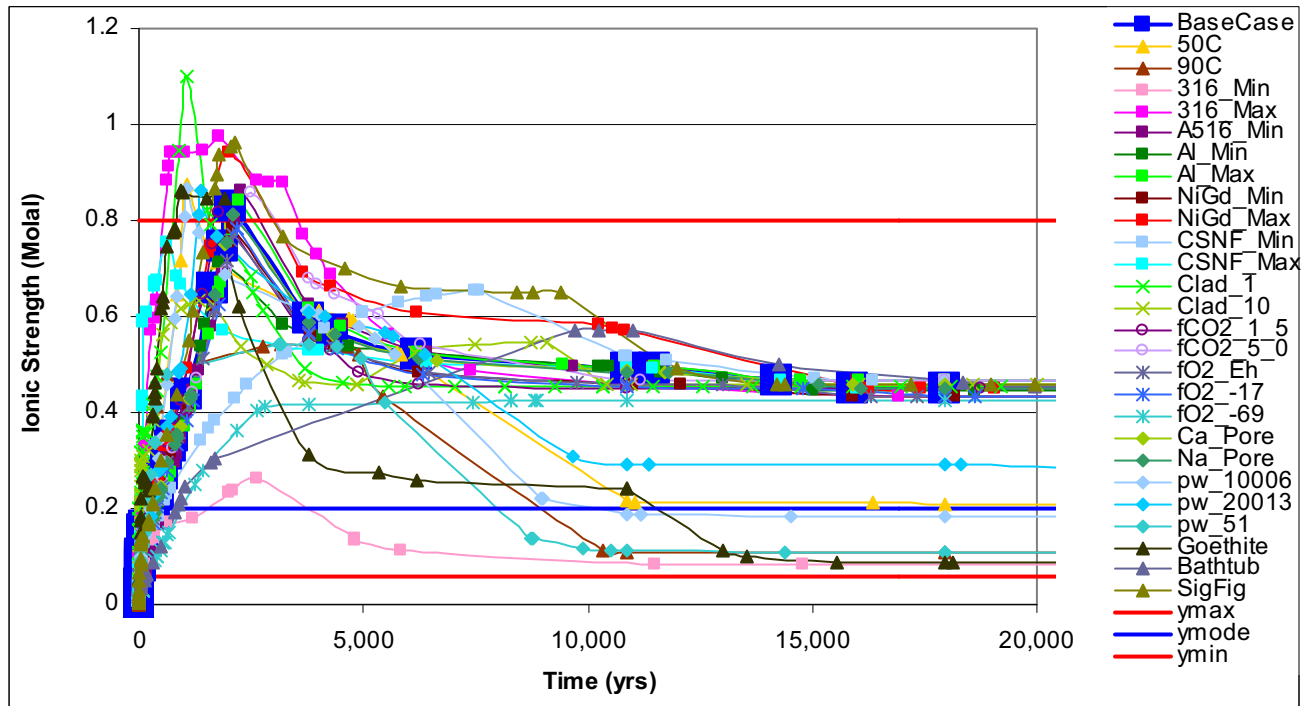


Source: (I) *Abstraction CDSP Liquid Influx.xls* (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 7-4. CDSP pH Abstraction for High Flux Compared to Model Output

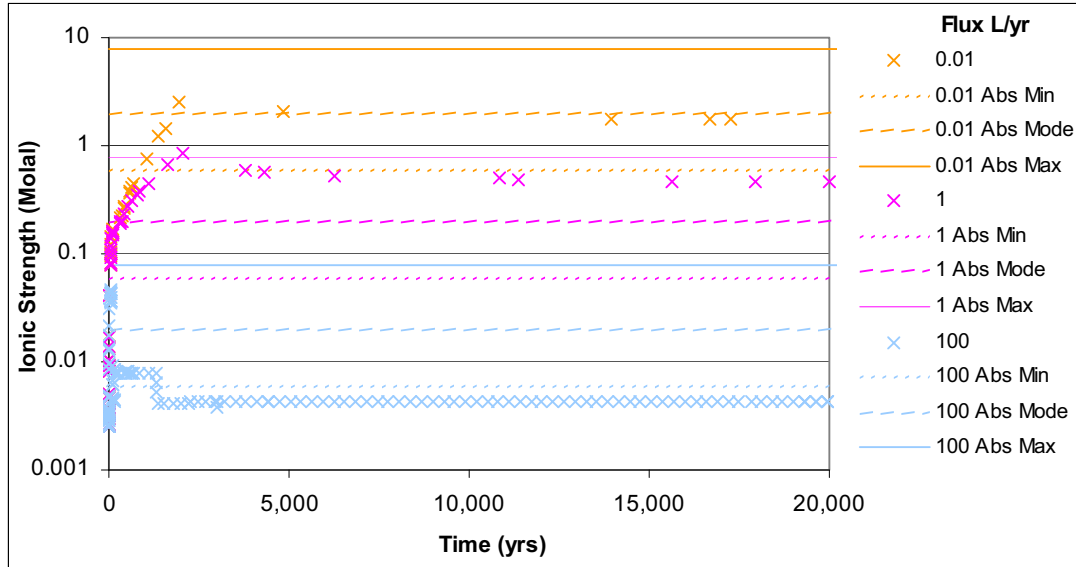
Comparisons of the model abstraction to the model output for the ionic strength abstractions are shown in the following figures.



Source: *CSNF Liquid Influx Model Output.xls* (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 7-5. CSNF Ionic Strength Abstraction Compared to Model Output (1 L/yr)

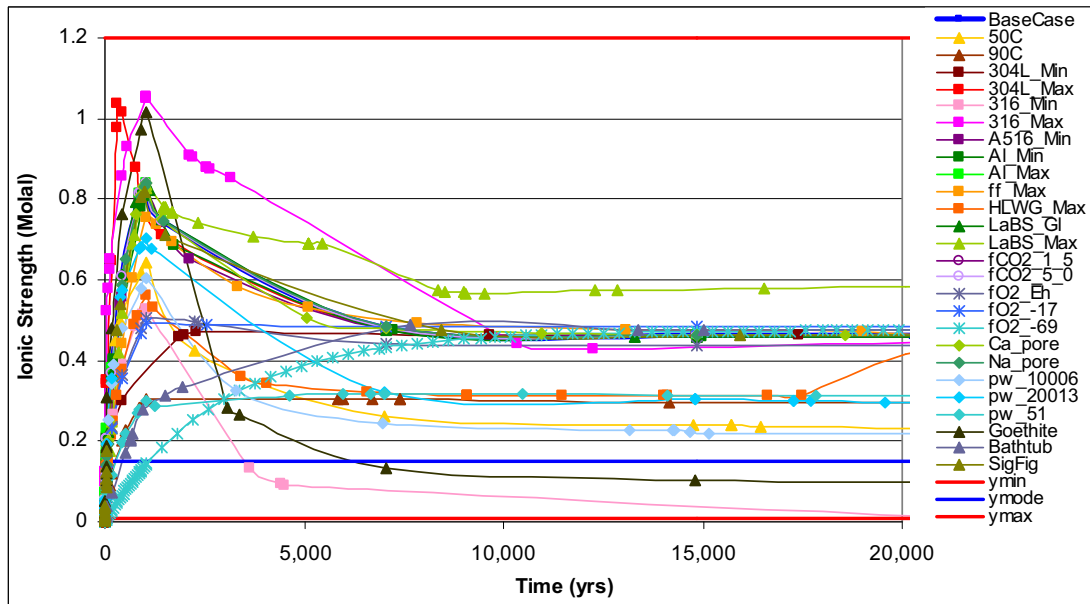


Source: (I) CSNF_Liquid Influx Model Output.xls (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 7-6. CSNF Ionic Strength Abstraction Compared to Model Output for Various Fluxes

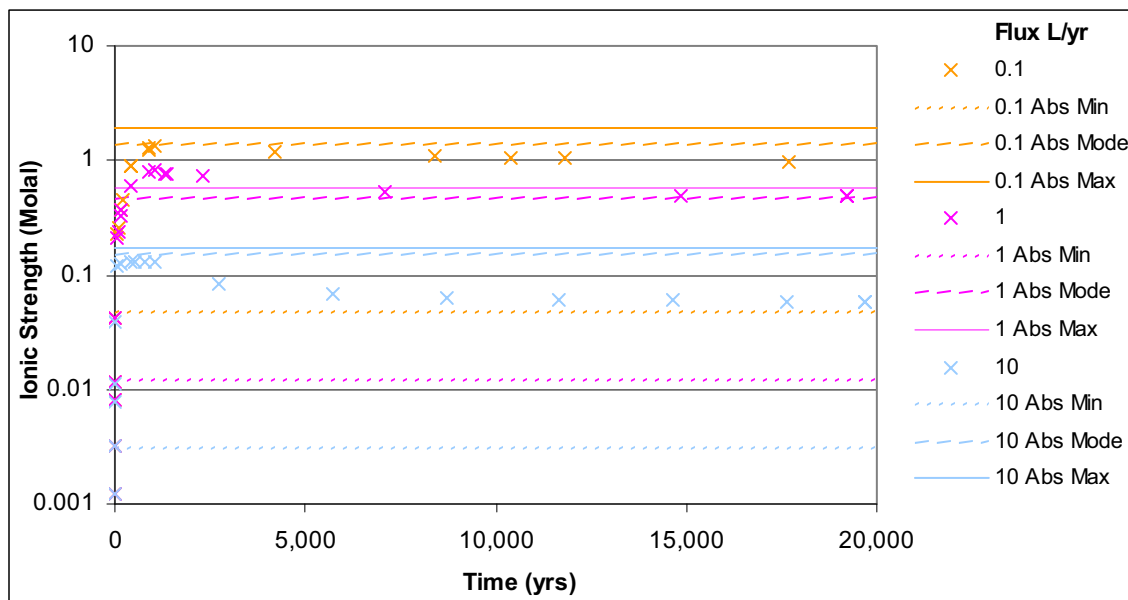
As shown in Figure 7-5, the abstracted ionic strength captures the majority of the model output. The abstraction is based on the time-weighted averages of the ionic strength values, so for this reason, some of the maximum ionic strength values fall outside of the abstraction. Because TSPA-LA time steps can be quite large, it would not be appropriate to allow the sampled ionic strength to go above the time-weighted average ionic strength for the most extreme case.



Source: CDSP Liquid Influx Model Output.xls (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 7-7. CDSP Ionic Strength Abstraction Compared to Model Output (1 L/yr)



Source: (I) *CDSP_Liquid Influx Model Output.xls* (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 7-8. CDSP Ionic Strength Abstraction Compared to Model Output for Various Fluxes

Figure 7-7 shows that the CDSP ionic strength abstraction captures the model output. For the CDSP model, the time-weighted average of the most extreme case does bound the maximum ionic strengths for all of the runs. The reason that the time-weighted average of the most extreme case for CDSP is higher than all of the runs is because unlike the CSNF waste package that contains Stainless Steel Type316 only, the CDSP waste package contains Stainless Steel Types 316 and 304L. It is the degradation of the stainless steels that generates the high ionic strength; therefore, it is appropriate that the maximum ionic strength for the CDSP waste package is higher than for the CSNF waste package.

As shown in Figures 6-20 and 6-21, the liquid influx and the vapor influx results are similar; therefore, the liquid influx results are indicative of the vapor influx results also capturing the model results.

The validation process used for the pH abstractions was a comparison of the abstracted pH for each time period to the product output value for a given set of input parameters. The response surfaces are mathematical constructs whose limits were set by the process models inputs and outputs. The ionic strength abstractions represent a best fit to the process model data, and are, therefore, able to reproduce the process model output for a given set of input conditions within a range of uncertainty; thus, rendering them self validating.

The pH is used by TSPA-LA to determine the dissolved concentration of radionuclides, and the degradation rate of the CSNF; therefore, the pH abstraction should reasonably represent the in-package pH. Likewise, the ionic strength is used by TSPA-LA to determine the stability of colloids and the dissolved concentrations of radionuclides (for ionic strengths >1molal, additional uncertainty is added); therefore, the ionic strength abstractions should reasonably represent the in-package ionic strength.

7.4.2 Fuel-Degradation Model Consistent with Results from Laboratory Experiments

The fuel degradation model was derived from laboratory experiments done by DOE contractors, which are themselves consistent with results published in refereed journals and literature (BSC 2004 [DIRS 169987]). In particular, the CSNF degradation rate law is broadly consistent with that proposed by Grambow (1989 [DIRS 113233]). The HLW glass degradation model is consistent with experimental and literature data (BSC 2004 [DIRS 169988], Table 7-1). The DSNF degradation model is consistent with experimental data (BSC 2004 [DIRS 172453], Section 6.1.7).

7.4.3 Types of Minerals Formed Consistent with Experimental Results

The primary phases predicted to form in the model are ferric oxyhydroxides (hematite, goethite, etc.) from steel corrosion, and schoepite from the degradation of spent fuel. Steel is observed in nature and in experiments to corrode to metal oxides whose specific identities depend upon the particular conditions of corrosion and the composition of the steel. Hematite and goethite are routinely observed as the most common iron oxides in soil (Schwertmann and Taylor 1995 [DIRS 105959]). Schoepite is observed as an alteration product in experiments (BSC 2004 [DIRS 169987]) and at the Pena Blanca natural analogue (Langmuir 1997 [DIRS 100051], p. 512). The aqueous concentrations of, respectively, Fe and U in fluids in contact with ferric hydroxides and schoepite tend to be fixed by the presence of these solids (Langmuir 1997 [DIRS 100051]; Stumm and Morgan 1996 [DIRS 125332]).

7.4.4 Steel-Degradation Model Consistent with Results from Laboratory Experiment

The steel degradation rates were derived from a compilation of laboratory experiments and are therefore implicitly consistent. Moreover, this range of rates, 4 to 131 $\mu\text{m}/\text{yr}$ for low carbon steel, is somewhat higher but otherwise broadly consistent with the rates determined from iron archeological artifacts, 0.1 to 10 $\mu\text{m}/\text{yr}$ (Miller et al. 2000 [DIRS 156684], p. 101). The latter probably saw lower temperatures than the data from which the rate model was derived. The stainless steel degradation rates cover a broad range of rates and are consistent with many experimental results documented in *Aqueous Corrosion Rates for Waste Package Materials* (BSC 2004 [DIRS 169982]).

Recent experimental work was conducted on “corrosion of miniature waste packages” (Zarrabi et al. 2003 [DIRS 171238]) in bathtub and flow-through configurations. The miniature waste packages were fabricated from carbon steel with similar interior geometry as Codisposal waste packages. The x-ray diffraction (XRD) analysis of the corrosion products revealed a variety of iron oxyhydroxide minerals formed, regardless of the chemical conditions of inflow water to the miniature waste packages. The corrosion products were mainly goethite ($\text{FeO}(\text{OH})$), lepidocrocite ($\text{FeO}(\text{OH})$), magnetite (FeFe_2O_4), and maghemite-C (Fe_2O_3). Hematite was not observed, likely due to the short duration of the experiment. But, these minerals are consistent with other experimental work that did see the formation of hematite (Pednekar 1987 [DIRS 159329]).

In dynamic systems (bathtub and flow-through configurations), formation of goethite is favored over hematite (Zarrabi et al. 2003 [DIRS 171238]). However, the thermodynamic stability of

these two minerals, in the presence of water, is similar. The mineral that actually begins to form typically depends on the nature of the substrates upon which the mineral begins to grow. Once started, that mineral will generally persist, even though it may not be the most stable. These results are consistent with many available “natural analogs” that point in the same direction. For example, Angus et al. 1962 [DIRS 167071] used electron diffraction to determine that corrosion products, taken from piles of buried Roman nails found in England were mainly Fe_2O_3 containing some Mn (bixbyite), which is similar to the composition of hematite.

In the study of “miniature waste packages” (Zarrabi et al. 2003 [DIRS 171238]), changes in pH and conductivity of several inflow solutions with different compositions were monitored. In one case, the pH of the inflow was adjusted to 2.1, and the outflow pH increased for the first week to 4.9 and then stabilized at a pH of 4.2. In the same experiment, conductivity of the effluent decreased from its original value of about 4,030 (μS) to about 1,950 (μS) and remained low for the duration of the experiment. This indicates the steady-state buffering and sorption capacity of carbon steel corrosion products.

In summary, this experiment validates the types of corrosion products that form from steel corrosion and also supports the pH trends used in the surface complexation model.

7.4.5 pH and Ionic Strength Consistent with Natural Analogue Observation

Because the output will be applied over geologic time spans, the specific outputs of this model (pH, ionic strength) are compared to natural analogue observation (rather than experimental data).

7.4.5.1 Production of Alkaline Waters by Glass Dissolution

Glass dissolution causes pH values greater than 7 when protons from solution exchange with alkalis, typically Na^+ , K^+ , or Ca^{2+} , present at the dissolving glass-solution interface, e.g.:



The general link between dissolution of alkali-bearing silicate minerals and glasses with acidity destruction (alkalinity production) is amply documented in the peer-reviewed scientific literature. Typically, large-scale weathering of alkali-bearing silicates ultimately leads to high pH values in alkali lakes (Berner and Berner 1987 [DIRS 161746], pp. 280 to 281). Experiments seeking to mimic interaction of seawater with basaltic glass on the ocean floor occasionally observe alkaline pH values (note that basaltic glass is generally considered to be an appropriate natural analogue for nuclear waste glass (Ewing and Haaker 1979 [DIRS 161749])).

Because basaltic glass is an appropriate analog for waste glass, the alkaline, long-term pH values calculated for glass-bearing packages should be similar to those of ground waters from basalt formation and other rock types of basaltic composition. In his authoritative review of the chemistry of natural waters, Hem (1995 [DIRS 115670]) gives the analyses of two basalt ground waters and one water from a tuff containing the basaltic mineral olivine. The basalt waters have pH values of 7.8 and 7.9 (Hem 1995 [DIRS 115670], Table 12, Analyses 4 and 5) and water from an olivine tuff has a pH of 8.2 (Hem 1995 [DIRS 115670], Table 16, Analysis 3).

Besides saturated environments, such as lakes and deep groundwater mentioned above, increases in pH due to degradation of glass have also been observed in unsaturated environments that are more analogous to repository conditions. A study of the natural alteration of ancient vitreous slags (the glassy refuse that remains after the smelting of metallic ore) found that all samples (100 to 4,000 years old) had a first stage of weathering in which the solution pH increased due to the reaction of the protons of the solution in the acid environment with the vitreous slags. This was found for different chemical compositions of glass and for different weathering environments, from open-air stacks to underground conditions. If sufficient oxygen exists, such as in the case of the open-air samples, the second stage involves further increases in pH toward a neutral to moderately basic environment, as the dissolution of the glass skeleton proceeds (Mahé-Le Carlier et al. 2000 [DIRS 171509]).

In short, the high-end pH values predicted by the single component HLWG runs in the in-package chemistry model are corroborated by natural observations documented in the peer-reviewed literature. This, in tandem with the satisfaction of Level I criteria (a) through (f), therefore, provides sufficient justification for a Level II validation for this particular feature of the in-package chemistry model. On the basis of meeting the criteria for this feature, the level of confidence has been met.

7.4.5.2 Lower Limits to pH Inside the Package

The pH values inside the degrading waste form are predicted to be between 4.5 and 8.5. The chemical makeup of the waste form corrosion products will be dominated by metal oxides and hydroxides similar to natural soils. The pH values in natural waters typically only become more acidic than 4.5 when high levels of organic acids are present, or when reduced sulfur is oxidized. Hem (1995 [DIRS 115670], Table 13, Analyses 1 and 4; Table 14, Sample 7) gives analyses of natural waters with pH values of 4.0, 3.8, and 3.0, respectively. These waters all contain relatively high proportions of dissolved sulfate, indicating that the source of their acidity is the oxidation of reduced sulfur. Oxidation of reduced sulfur is also a major source of acidity in waters corroding waste package materials, particularly the low carbon steel. The levels of reduced sulfur in the waste package are too low to exceed the pH-buffering capacity of the corrosion products themselves (Section 6.7.1). Given a relative absence of high levels of organic acids and reduced sulfur, the in-package chemistry model predicts low-end pH values of 4.5. This low-end pH value is similar to the pH values of natural waters that contain little dissolved organic acids, and which are not influenced by sulfide oxidation, such as those described in Hem (1995 [DIRS 115670]), validating the model predictions of low-end pH values.

7.4.5.3 Neutralization of Alkaline and Acidic Waters

This section describes the neutralization of the alkaline waters produced by the dissolution of the HLWG (Section 7.2.5.1) and the acidic waters produced by the oxidation of the sulfur in the low carbon steels (Section 7.2.5.2). This neutralization is shown by comparing the pH values generated by the single components of the CDSP waste package in Figure 6-22 with the combined reactants shown in Figure 6-23. The concept of acids and bases neutralizing one another is a generally accepted fact and does not require validation.

7.4.5.4 Production of Extreme Ionic Strength Solutions

High ionic strength solutions (>1 molal) are predicted to form when low volumes of fluid remain in contact with soluble waste form and waste package components for long periods of time. This observation is validated by the observation that older geologic fluids tend to be saltier than younger ones that have had less time to react with solids (Drever 1982 [DIRS 100725]).

Low ionic strength solutions (<0.05 mol/L) are predicted to form when high volumes of water flow past the waste form and waste package components or when water only contacts the HLWG and aluminum alloy without contacting any other waste package materials. These low ionic strengths are conservative for colloid stability and thus result in the greatest radionuclide release in advective flow conditions, and therefore do not require validation.

7.4.6 Relationship Between Ionic Strength and Flow Rate

Freeze and Cherry (1979 [DIRS 101173], Figure 7.17(b)) show the relationship between groundwater flow rate and concentration of solution for a constant reaction rate. The plot shows that water flowing at a higher flow rate, but same degradation rate and same flow distance will have a lower concentration than water flowing at a lower flow rate. For the in-package system, this is equivalent to saying that if water is flowing through the waste package, the water with the highest flow rate will exit the waste package with the lowest ionic strength. This behavior is exhibited in this model as shown in Figures 6-74 and 6-76 for CSNF, and Figures 6-79 and 6-80 for CDSP.

7.4.7 Surface Complexation Model

A technical review of the surface complexation model was conducted and is documented in Appendix E. The reviewer, Yifeng Wang, who was independent of the development and checking of the document, concluded:

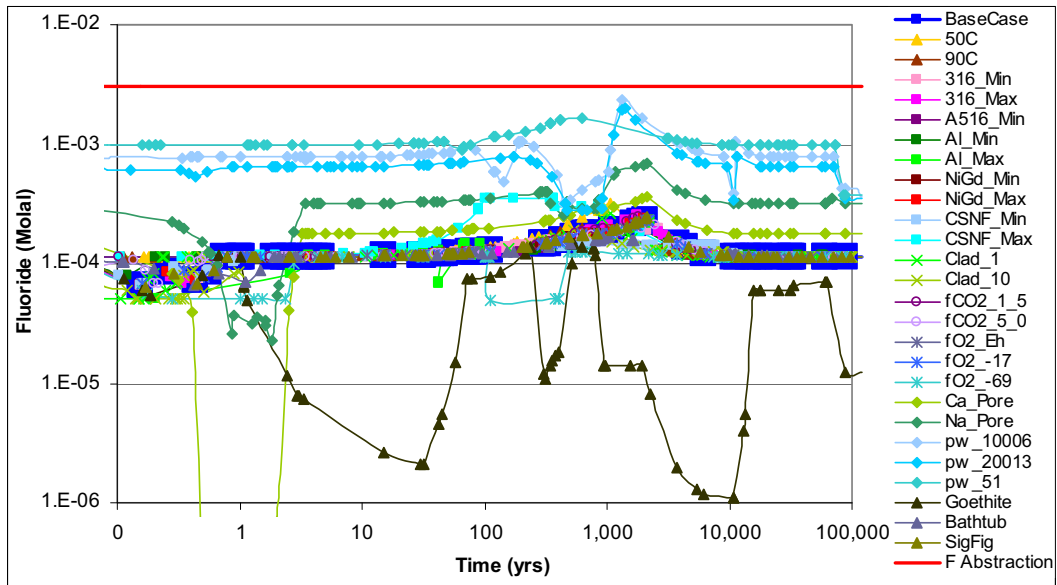
A simplified version of surface complexation model, described by an analytic equation, is used in the package chemistry. I have checked the derivation of the equation, and no error has been found. The prediction of the equation has been confirmed by PHREEQC calculations.

In general, the surface complexation model developed for in-package chemistry is adequate for its intended use, i.e., to provide a more realistic prediction of in-package pH.

Therefore, the surface complexation model has been validated by means of a technical review.

7.5 FLUORIDE VALIDATION

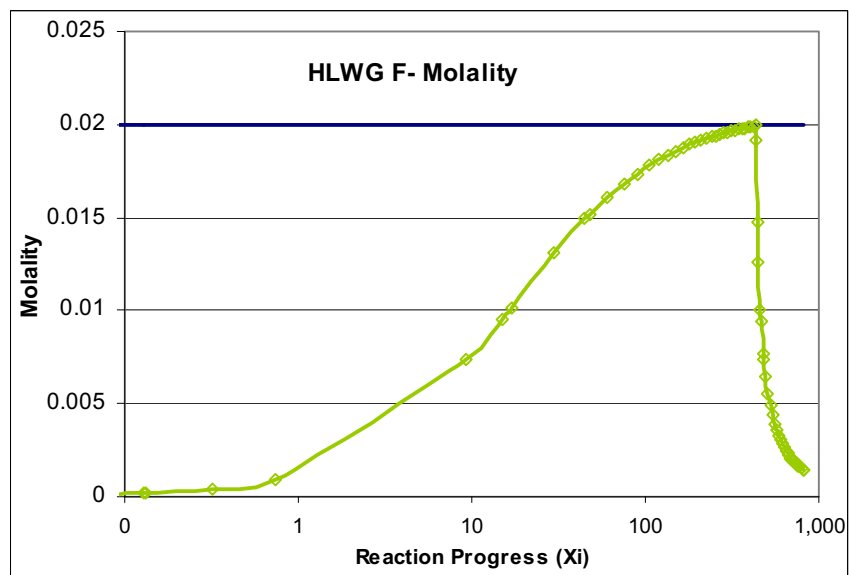
The fluoride abstractions are plotted in Figures 7-9 through 7-11, along with the model output.



Source: CSNF Liquid Influx Model Output.xls (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 7-9. CSNF Fluoride Abstraction Compared to Model Output

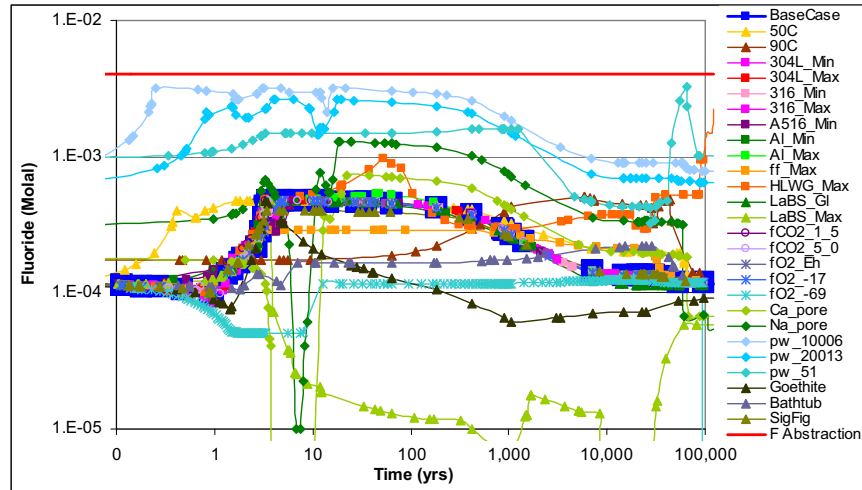


Source: CDSP Vapor Influx Model Output.xls (Appendix F \Spreadsheets).

NOTE: 0 on the x-axis refers to 0.1 Xi.

Figure 7-10. CDSP Vapor-Influx Fluoride Abstraction Compared to Model Output

These figures illustrate that the abstraction adequately bounds the model output.

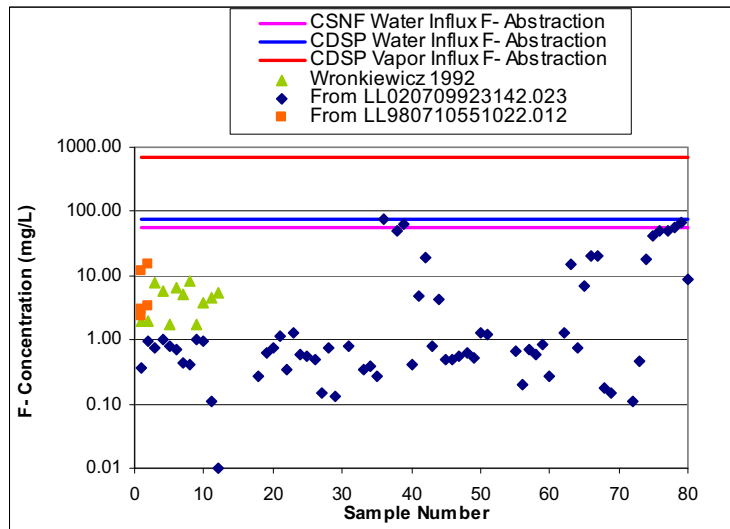


Source: *CDSP Liquid Influx Model Output.xls* (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 7-11. CDSP Liquid Influx Fluoride Abstraction Compared to Model Output

The experimental data used to validate these models is shown in Figure 7-12 and is from three sets of experiments: Wronkiewicz et al. 1992 [DIRS 100493] and LL020709923142.023 [DIRS 161677], and LL980710551022.012. The Wronkiewicz et al. 1992 [DIRS 100493] data comes from unsaturated testing of UO₂, and the LL020709923142.023 [DIRS 161677] data comes from the testing of hydrological boreholes. Coupons of candidate waste package materials were placed at strategic locations such as hot/dry locations near heaters and warm/wet regions in the condensation zones in hydrological boreholes and in the heated drift. These coupons consisted of one of three materials (Alloy 22, carbon steel, and Monel-400). LL980710551022.012 [DIRS 173475] contains data from the unsaturated drip condition testing of spent fuel and unsaturated dissolution tests of glass.



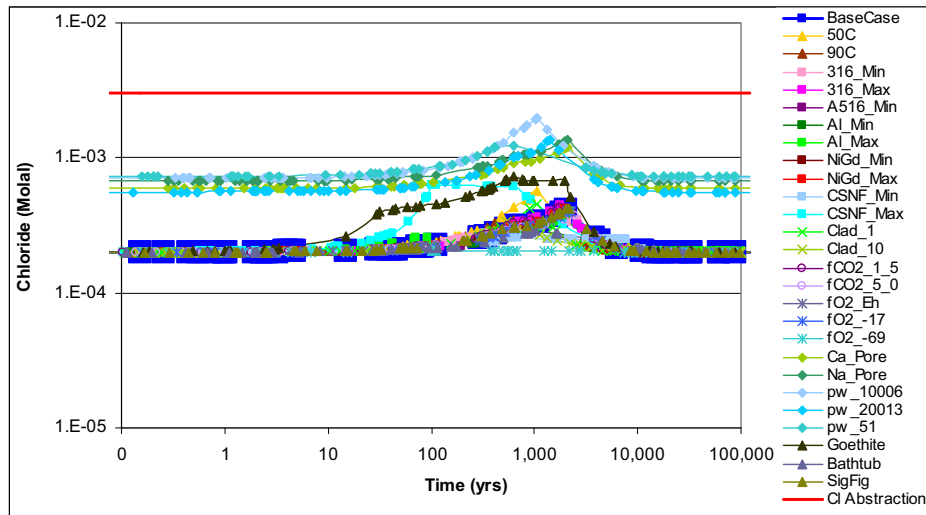
Source: *F and CI Abstractions.xls* (Appendix F \Spreadsheets).

Figure 7-12. CSNF and CDSP Fluoride Abstractions Compared to Experimental Data

The fluoride model is used by the *Dissolved Concentration Limits of Radioactive Elements Model* (BSC 2005 [DIRS 173873]). High fluoride concentrations increase dissolved concentrations of radionuclides; therefore, higher fluoride values are conservative. The fact that the model is consistent with the experimental data, and bounds the vast majority of the experimental data shows that this model is valid for its intended use.

7.6 CHLORIDE VALIDATION

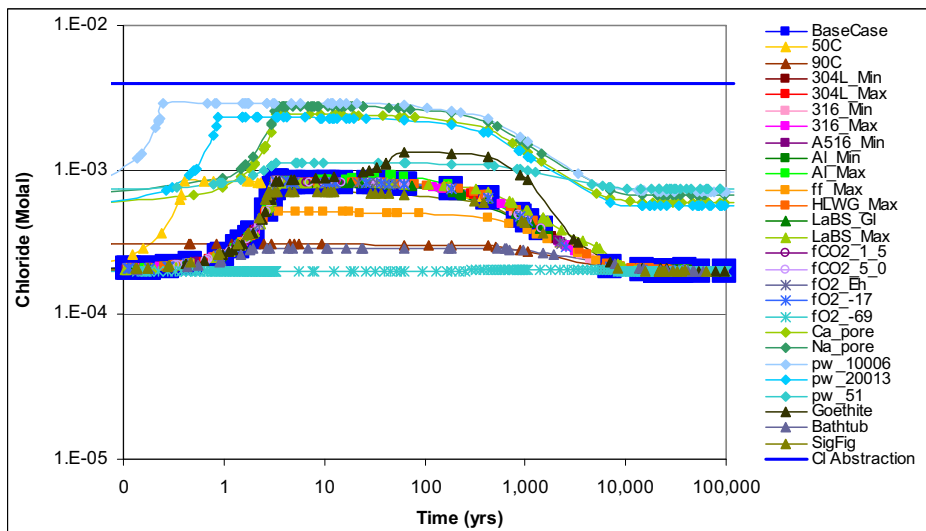
The chloride abstraction is plotted in Figures 7-13 and 7-14, along with the model output.



Source: *CSNF Liquid Influx Model Output.xls* (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 7-13. CSNF Chloride Abstraction Compared to Model Output



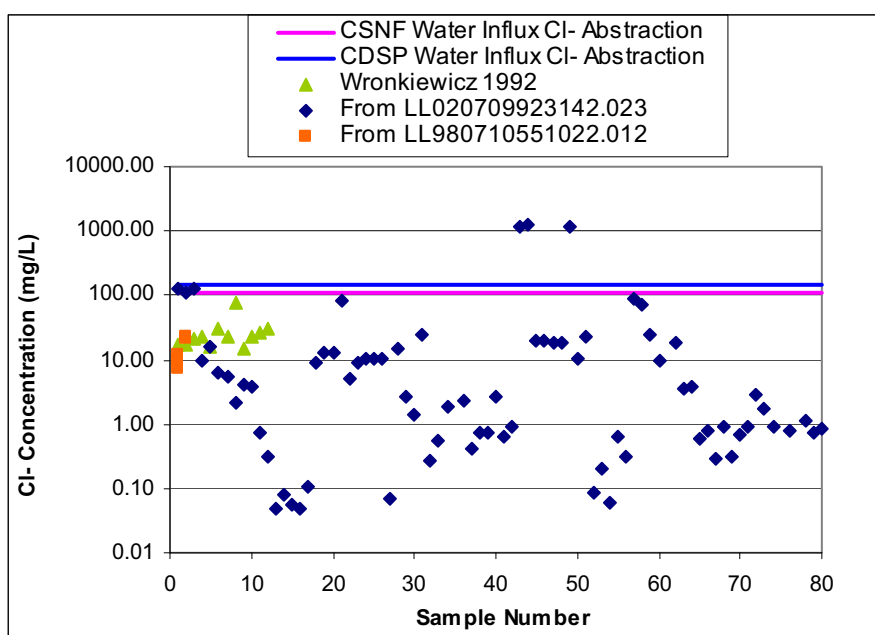
Source: *CDSP Liquid Influx Model Output.xls* (Appendix F \Spreadsheets).

NOTE: Time refers to time after waste package breach (0 on the x-axis refers to 0.1 years).

Figure 7-14. CDSP Chloride Abstraction Compared to Model Output

These figures illustrate that the abstraction adequately bounds the model output.

The experimental data used to validate these models is shown in Figure 7-15 and is from three sets of experiments: Wronkiewicz et al. 1992 [DIRS 100493] and LL020709923142.023 [DIRS 161677], and LL980710551022.012. The Wronkiewicz et al. 1992 [DIRS 100493] data comes from unsaturated testing of UO_2 , and the LL020709923142.023 [DIRS 161677] data comes from the testing of hydrological boreholes. Coupons of candidate waste package materials were placed at strategic locations such as hot/dry locations near heaters and warm/wet regions in the condensation zones in hydrological boreholes and in the heated drift. These coupons consisted of one of three materials (Alloy 22, carbon steel, and Monel-400). LL980710551022.012 [DIRS 173475] contains data from the unsaturated drip condition testing of spent fuel and unsaturated dissolution tests of glass.



Source: *F and Cl Abstractions.xls* (Appendix F \Spreadsheets).

Figure 7-15. CSNF and CDSP Chloride Abstractions Compared to Experimental Data

The intended use of the chloride model had been by the *Pitting Model for Zirconium-Alloyed Cladding*, BSC 2004 [DIRS 170043] for the determination of whether the conditions for Zirconium pitting exist in the waste package. High chloride concentrations are more likely to cause pitting; therefore, higher chloride values are conservative. However, the pitting model no longer uses this model, so the level of validation required for this model is low. The fact that the model is consistent with the experimental data, and bounds the vast majority of the experimental data shows that this model is valid for its intended use. Even though the chloride model is no longer used, it is retained as validated output of this model to maintain consistency with previous versions and to be available for future potential use.

7.7 TOTAL CARBONATE VALIDATION

The TSPA model calculates total carbonate (Total C) using the equation given in Table 6-22. The variables and parameters in this equation are the PCO_2 and pH, and the temperature-dependent values of three chemical equilibrium constants. The PCO_2 and pH values are established by the TSPA model. The values of the equilibrium constants are calculated by the TSPA model using the equations given in Table 6-24 and the temperature corresponding to the TSPA realization.

The equation for Total C in Table 6-22 is based on the laws of equilibrium thermodynamics. The equations for the temperature-dependent equilibrium constants are polynomial fits to data from the thermodynamic database (DTN: SN0410T0510404.002 [DIRS 172712]) used in the EQ3/6 calculations supporting this report.

An equation for Total C and temperature-dependent values of parameters required for its solution can be taken from standard textbooks of aqueous geochemistry and used to validate the Total C calculations of this report. One such textbook, Stumm and Morgan (1996 [DIRS 125332]) discusses carbonate chemistry in Chapter 4. Substituting equations 2 and 3 into equation 5 of Table 4.2 of that chapter leads to the following expression for Total C:

$$\text{Total C} = \text{PCO}_2 (k_H + k_H k_1 / [\text{H}^+] + k_H k_1 k_2 / [\text{H}^+]^2) \quad (\text{Eq. 7-2})$$

In this equation, $[\text{H}^+]$ is the H^+ ion concentration, k_1 and k_2 are temperature-dependent constants of equations 2 and 3, and $k_H = [\text{H}_2\text{CO}_3^*] / \text{PCO}_2$ where $[\text{H}_2\text{CO}_3^*] = [\text{CO}_2(\text{aq})] + [\text{H}_2\text{CO}_3]$. Stumm and Morgan (1996 [DIRS 125332]) give values for the negative logs of these constants for several temperatures in their Table 4.3.

Table 7-2 shows values of Total C calculated at various temperatures, PCO_2 and pH values using the abstraction equation of Table 6-22, and the equations for the temperature dependent parameters from Table 6-24, and compares them with values calculated using Eq. 7-2 and the parameter values from Stumm and Morgan (1996 [DIRS 125332] Table 4.3).

Table 7-2. Comparison of Total Carbonate Model

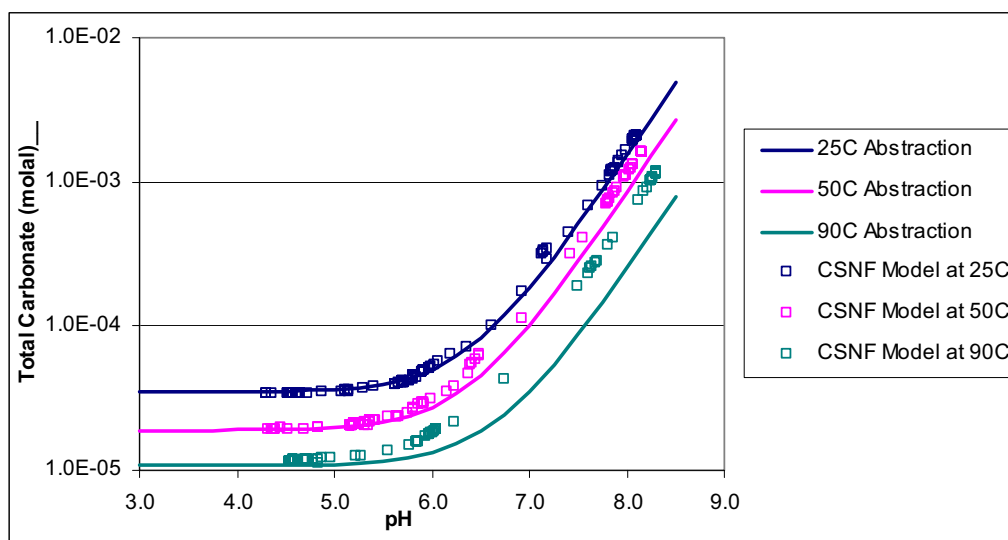
| Temperature (°C) | Partial Pressure of CO_2 (PCO_2) | pH | Total Carbonate from Equation in Table 6-22 | Total Carbonate from Eq. 7-2 | Difference / Mean (%) |
|------------------|--|-----|---|------------------------------|-----------------------|
| 25 | 0.010 | 8.0 | 1.54E-02 | 1.55E-02 | -0.8 |
| 25 | 0.010 | 6.0 | 5.00E-04 | 4.90E-04 | 2.0 |
| 25 | 0.001 | 4.0 | 3.51E-05 | 3.40E-05 | 3.2 |
| 25 | 0.001 | 6.0 | 5.00E-05 | 4.90E-05 | 2.0 |
| 25 | 0.001 | 8.0 | 1.54E-03 | 1.55E-03 | -0.8 |
| 40 | 0.010 | 8.0 | 1.11E-02 | 1.05E-02 | 5.3 |
| 40 | 0.010 | 6.0 | 3.44E-04 | 3.31E-04 | 3.8 |
| 40 | 0.001 | 4.0 | 2.38E-05 | 2.30E-05 | 3.2 |
| 40 | 0.001 | 6.0 | 3.44E-05 | 3.31E-05 | 3.8 |
| 40 | 0.001 | 8.0 | 1.11E-03 | 1.05E-03 | 5.3 |
| 100 | 0.010 | 8.0 | 1.88E-03 | 3.76E-03 | -66.5 |

Table 7-2. Comparison of Total Carbonate Model (Continued)

| Temperature (°C) | Partial Pressure of CO ₂ (PCO ₂) | pH | Total Carbonate from Equation in Table 6-22 | Total Carbonate from Eq. 7-2 | Difference / Mean (%) |
|------------------|---|-----|---|------------------------------|-----------------------|
| 100 | 0.010 | 6.0 | 1.19E-04 | 1.39E-04 | -14.9 |
| 100 | 0.001 | 4.0 | 1.02E-05 | 1.03E-05 | -0.7 |
| 100 | 0.001 | 6.0 | 1.19E-05 | 1.39E-05 | -14.9 |
| 100 | 0.001 | 8.0 | 1.88E-04 | 3.76E-04 | -66.5 |

Source: *Total Carbonate and Eh Abstractions.xls* (Appendix F \Spreadsheets)

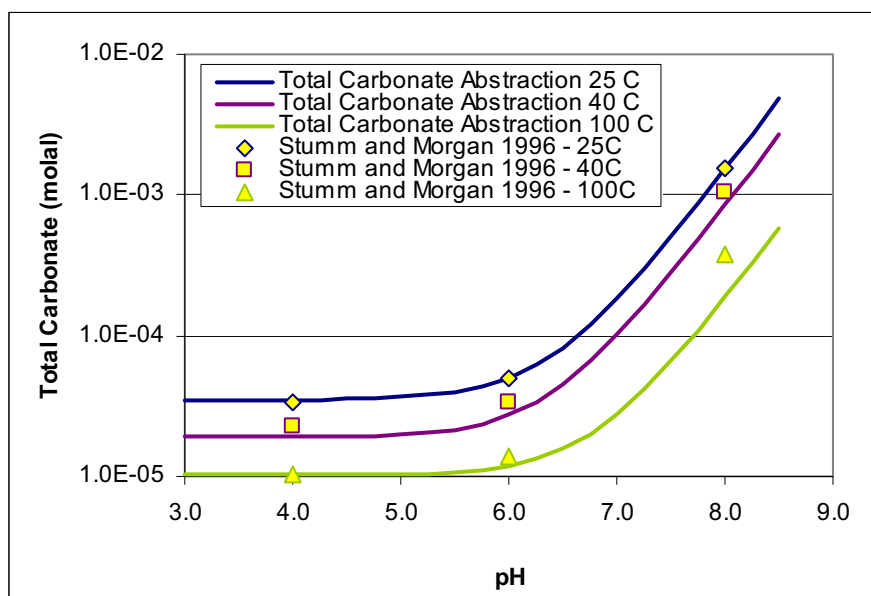
This table illustrates that at 25°C the differences between the two sets of calculations are not more than 2%, at 40°C they are less than 6%, but at 100°C, the differences can be as much as 66%. It is likely the precision could be improved if more significant figures were displayed in the parameters in Table 6-24 for the temperature dependence of the K values. The total carbonate abstraction (along with the pH abstraction) is used in the TSPA-LA as an input to the CSNF degradation rate. The CSNF degradation rate is fairly insensitive to total carbonate; therefore, the abstraction equation for the TSPA and the supporting table of temperature dependencies are adequate for their purposes.



Source: *Total Carbonate and Eh Abstraction.xls* (Appendix F \Spreadsheets).

Note: Log PCO₂ = -3.0

Figure 7-16. Total Carbonate Abstraction Compared to Model Output



Source: *Total Carbonate and Eh Abstraction.xls* (Appendix F \Spreadsheets).

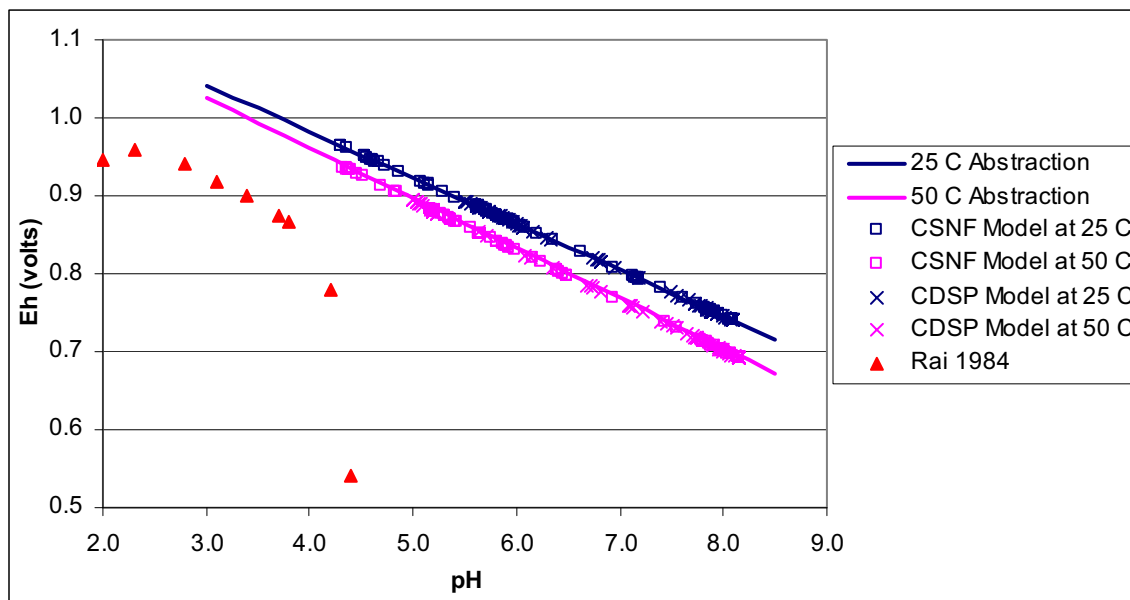
Note: $\text{Log PCO}_2 = -3.0$

Figure 7-17. Total Carbonate Abstraction Compared to Validation Data

Figure 7-16 shows the agreement of the model abstraction and the model output, which illustrates that the abstraction adequately reproduces the model output. Figure 7-17 shows the model abstraction with validating data from the report by Stumm and Morgan (1996 [DIRS 125332]), which also shows agreement. The total carbonate model is used in the kinetic rate expression for the dissolution of CSNF within TSPA-LA. The fact that the model is consistent with the literature data shows that this model is valid for its intended use.

7.8 EH VALIDATION

The Eh abstraction is plotted in Figure 7-18, along with the model output and experimental data. The experimental data used to validate this model is from the report by Rai (1984 [DIRS 122768]).



Source: *Total Carbonate and Eh Abstraction.xls* (Appendix F \Spreadsheets).

Figure 7-18. Eh Abstraction Compared to Model Output and Experimental Data

The close agreement of the abstraction and the model output illustrates that the abstraction adequately reproduces the model output. The intended use of the Eh model had been by *Pitting Model for Zirconium-Alloyed Cladding*, BSC 2004 [DIRS 170043] for the determination of the corrosion potential, E_{corr} . However, the pitting model no longer uses this Eh model, so the level of validation required for this model is low. The fact that the model is consistent with the experimental data, and bounds the experimental data shows that this model is valid for its intended use. Even though the Eh model is no longer used, it is retained as validated output of this model to maintain consistency with previous versions and to be available for future potential use.

7.9 SUMMARY

To summarize, the in-package chemistry model is a numerical titration of water with acid-producing solids (steel components), alkali-producing solids (glass), and pH buffering via iron (hydr)oxide surface complexation. The outputs are ionic strength trajectories and pH distributions. Documenting that these are within the bounds of pH values and ionic strengths observed in natural situations, as done in the peer-reviewed literature, where similar processes prevail is critical to model validation of the in-package chemistry model. Additionally, the surface complexation model was validated via a technical review to determine if the approach and application were technically sound. Yifeng Wang (Appendix E) agrees that the surface complexation conceptual model and its formulation are adequate for intended use.

Achieving a sufficient level of confidence required documenting:

1. Fuel and steel degradation models
2. Formation of corrosion products
3. Production of low-pH waters by steel degradation

4. Production of high-pH waters by glass degradation
5. Production of ionic strength extremes
6. pH buffering via surface complexation.

This model requires Level II validation (BSC 2005 [DIRS 173246]). In addition to satisfying Level I criteria (a) through (f), the evidence above collectively provides broad justification for assignment of a Level II validation for the in-package chemistry model. In addition, the requirement for Level II validation is to provide rationale for the selection of the validation methods used. This justification is provided in Section 7.3. The validation of the in-package chemistry model meets the requirements for implementation in the TSPA-LA and does not require any further validation.

8. CONCLUSIONS

The in-package chemistry process models are developed and documented in Section 6 and Appendix B. The in-package chemistry model is made up of two main models: the batch reactor model, and the surface complexation model. The batch reactor model is made up of two water-influx models, the liquid-influx and the vapor-influx models, and several other component submodels, including the fuel-degradation and the steel-degradation submodels. Based on these models, several output model abstractions were generated in Section 6.10, including: the total carbonate, fluoride, chloride, and Eh models. Based on the results of the in-package chemistry process models, it may be concluded that the following inputs or processes have only minor impact on the model response such that they were not explicitly carried forward into the abstractions.

- Initial water composition
- Partial pressure of carbon dioxide and oxygen
- Radiolysis (Appendix B)
- Modifications to waste package design configuration (including the replacement of Neutronit with NiGd alloy, or the addition of a purge port)
- Sulfur content of Carbon Steel Type A516 waste package components
- Composition and fracturing of the HLW glass log
- Oxidation state of chromium
- Waste package component degradation rates
- Controlling iron mineral (hematite versus goethite)
- CDSP waste package configuration (2-MCO/2-DHLW versus 5-DHLW/DOE SNF).

The effect of the above factors on the model response is finite and accounted for in the abstractions via applied uncertainty terms. The following variables have a large influence on the in-package chemistry:

- Waste package type (CSNF versus codisposed)
- Water flux
- Temperature.

These variables are used as input to the in-package chemistry abstractions (i.e., the abstractions are functions of these variables). Therefore, in TSPA-LA the in-package chemistry is implemented as a function of external variables that directly links the in-package chemistry to the repository environment.

8.1 MODEL OUTPUT

The output of this model includes abstracted values for pH, ionic strength, total carbonate, Eh, fluoride, and chloride concentrations. The uncertainties for these abstracted values are also presented.

The output of this model starts once the waste package has been breached and once the temperature of the waste package has dropped to 100°C. The range of applicability (restrictions for subsequent use) for the output of this model is provided in Table 8-1. The output of the model is provided below and in the output DTN: MO0502SPAINPCA.000 [DIRS 172893].

Table 8-1. Range of Applicability

| | Value | Units | Source |
|-------------------------|----------------------------|-------|---------------------------|
| Oxygen Fugacity | up to $10^{-0.7}$ | atm | Section 6.6.4 |
| Carbon Dioxide Fugacity | $10^{-5.0}$ to $10^{-1.5}$ | atm | Section 6.6.4 |
| Temperature | 25 to 100 | °C | Section 6.6.6 |
| Water Flux | 0.001 to 1,000 | L/yr | Sections 6.6.5 and 6.10.2 |

8.1.1 pH Abstraction

Table 8-2 shows the TSPA-LA pH abstraction.

Table 8-2. TSPA-LA pH Abstraction

| Water Entry Case | Waste Package Type | Period (yrs) | Temp (°C) | Flux (L/yr per Waste Package) | log fCO ₂ | Min pH | Max pH | Distribution | Uncertainty |
|--------------------------------|--------------------|--------------|-----------|-------------------------------|----------------------|--------|--------|--------------|---------------------------|
| Liquid Influx and Vapor Influx | CSNF | 0 to 600 | 25 to 100 | Q ≤ 150 | -5.0 | 4.5 | 8.1 | Uniform | No Additional Uncertainty |
| | | | | | -4.5 | 4.5 | 8.1 | | |
| | | | | | -4.0 | 4.5 | 8.0 | | |
| | | | | | -3.5 | 4.5 | 7.9 | | |
| | | | | | -3.0 | 4.5 | 7.7 | | |
| | | | | | -2.5 | 4.5 | 7.5 | | |
| | | | | | -2.0 | 4.5 | 7.3 | | |
| | | | | | -1.5 | 4.5 | 7.0 | | |
| | Q > 150 | N/A | 4.5 | 8.5 | | | | | |
| | CDSP | 0 to 20,000 | 25 to 100 | Q ≤ 150 | N/A | 4.5 | 7.0 | | |
| Q > 150 | | | | N/A | 4.5 | 8.5 | | | |

Source: Section 6.10.1.

NOTE: The CDSP pH abstraction is not dependent on log fCO₂.

As shown in Figure 6-41 for CSNF, as flux rate increases, pH values increase. Therefore, in the CSNF pH abstraction, the in-package pH value was developed with a flux dependence. At early times (less than or equal to 600 years) at lower-flux conditions, less than or equal to 150 L/yr, the pH inside a failed CSNF waste package is sampled uniformly from the range, 4.5 to 8.1, depending $f\text{CO}_2$. At late times, the pH is sampled from a uniform distribution between 4.5 and 7.0. For higher flux conditions, greater than 150 L/yr, the sampled uncertainty range is expanded from the same minimum, 4.5, to a maximum value of 8.5. To enforce the trend in higher pH values at higher flow rates observed in Figure 6-41, sampling the low- and high-flux distributions should not be independent. Sampling the distributions independently could yield a sampled value for the high-flux condition pH that is lower than the sampled value of the low-flux condition pH. This is contrary to the results plotted in Figure 6-41. Therefore, it is recommended that correlation be applied when sampling the two flux-dependent distributions. Because the low- and high-flux distribution types are the same and use the same minimum value, and the high-flux condition maximum pH value is always greater than the low-flux condition maximum pH, regardless of $f\text{CO}_2$, a higher high-flux condition pH is always obtained if these two distributions are sampled with perfect correlation (i.e., a correlation factor of 1). The same flux correlation applies to the CSNF late-time pH abstraction (based on Figure 6-41) and the CDSP pH abstraction (based on Figure 6-43).

There is no correlation between the CSNF and CDSP pH abstraction. For the CSNF abstraction, there is no correlation between the early and late times.

The expected pH minimum value for times greater than 40,000 years is 6 for CSNF and 6.5 for CDSP, and the expected pH maximum value would be set by that of the incoming water.

8.1.2 Ionic Strength Abstraction

The ionic strength abstraction is a function of the liquid or vapor influx, and takes the following form:

$$y = e \cdot x^f \quad (\text{Eq. 8-1})$$

where:

- y = ionic strength (moles/kg)
- e, f = constants (specific to waste package type)
- x = liquid or vapor flux (L/yr)

For the vapor case, the flux, x, is a uniform distribution from 0.1 to 10 L/yr. For the liquid case, the flux is calculated by TSPA.

The abstraction uses a Log Triangular Distribution:

$$y \sim \log \text{tri}(a_{\min}, c_{\text{mode}}, b_{\max}) \quad (\text{Eq. 8-2})$$

and:

$$a_{\min} = e_{\min} \cdot X^{f_{\min}} \quad (\text{Eq. 8-2a})$$

$$c_{\text{mode}} = e_{\text{mode}} \cdot X^{f_{\text{mode}}} \quad (\text{Eq. 8-2b})$$

$$b_{\max} = e_{\max} \cdot X^{f_{\max}} \quad (\text{Eq. 8-2c})$$

The e and f constants for each influx scenario and waste package type are provided in the table below.

Table 8-3. Constants for TSPA-LA Ionic Strength Abstraction

| | Liquid Influx | | | | Vapor Influx | | | |
|----------------|---------------|-------|-------|-------|--------------|-------|-------|-------|
| | CSNF | | CDSP | | CSNF | | CDSP | |
| | e | f | e | f | e | f | e | f |
| Minimum | 0.061 | -0.55 | 0.008 | -0.29 | 0.079 | -0.68 | 0.012 | -0.59 |
| Mode | 0.20 | -0.49 | 0.15 | -0.49 | 0.32 | -0.60 | 0.47 | -0.48 |
| Maximum | 0.81 | -0.38 | 1.17 | -0.55 | 0.49 | -0.65 | 0.57 | -0.52 |

Source: Section 6.10.2.

If the liquid influx rate drops below the sampled vapor influx rate, the vapor-influx model should be used for that case.

For flow rates greater than 1,000 L/yr, the model should be used with a flow rate of 1,000 L/yr.

For the vapor-influx case, the vapor mass-flow rate is similar for the CSNF and CDSP waste packages because the conditions that control the vapor mass-flow rate, temperature, and relative humidity are similar throughout the drift where the different waste packages are located. Differences in the CSNF and CDSP waste package temperatures and relative humidities could cause a potential difference in the vapor-flux rate because of the temperature dependence of the diffusion coefficient. This difference is expected to be insignificant considering that the sampled flux rate is applied for the entire realization in the TSPA-LA model and, therefore, does not account for reductions in the vapor diffusion rate as the repository cools and differences between CSNF and CDSP temperatures and relative humidities are much smaller. For this reason the same vapor-flux rate applied in the CSNF waste package ionic strength abstraction can be applied to the CDSP waste package ionic strength abstraction. For the vapor-influx case, the ionic strength uncertainty is calculated once per realization, since the flux is only sampled once per realization.

For the liquid influx case, the liquid mass flow rate is output from *Abstraction of Drift Seepage* (BSC 2004 [DIRS 169131]), and is different for each waste package type, because it is dependent on location within the drift, and is calculated with time-dependent variables. For the liquid influx case, the ionic strength uncertainty is recalculated with each timestep, as a function of the flux, which is calculated with time-dependent variables. However, the random number sampled between 0 and 1 that TSPA-LA uses to sample between the minimum and maximum uncertainty is only sampled once per realization. So if a given realization applies a high uncertainty (the random number is close to 1), the ionic strength uncertainty will remain high for that entire realization.

8.1.3 Total Carbonate Abstraction

Table 8-4. TSPA-LA Total Carbonate Abstraction

| Equation of Total Carbonate (mol/kg) | | |
|---|--|----------------|
| Total C = $PCO_2 (10^{k_1} + 10^{(pH+k_1+k_2)} + 10^{(2pH+k_1+k_2+k_3)})$ | | |
| Log K | log K expression | R ² |
| k_1 | $\log K_1 = 7E-05T^2 - 0.0159T - 1.1023$ | 0.9992 |
| k_2 | $\log K_2 = 5E-07T^3 - 0.0002T^2 + 0.0132T - 6.5804$ | 1.0 |
| k_3 | $\log K_3 = -8E-05T^2 + 0.0128T - 10.618$ | 0.9977 |

Source: Section 6.10.5.

NOTE: Temperature is in °C.

k is defined as log K, and PCO_2 is the partial pressure of CO_2 (in atmospheres). For the purposes of this report and ranges of this model (25°C to 100°C and up to 4 molal solutions), mol/L is considered equivalent to mol/kg, and PCO_2 is equivalent to fCO_2 .

8.1.4 Eh Abstraction

Table 8-5. TSPA-LA Eh Abstraction

| Equation of Eh (volts) |
|--|
| $Eh = 2.303RT / F(6193.973 / T - pH + 0.25 \log PO_2)$ |

Source: Section 6.10.6.

NOTE: Temperature is in K.

 PO_2 is the partial pressure of O_2 (in atmospheres)

8.1.5 Fluoride and Chloride Abstraction

Table 8-6. TSPA-LA Fluoride and Chloride Abstraction

| Water Entry Case | Waste Package Type | Period (yrs) | Max F ⁻ (Mol/kg) | Max Cl ⁻ (Mol/kg) |
|------------------|--------------------|--------------|-----------------------------|------------------------------|
| Vapor Influx | CSNF | 0 to 20,000 | N/A | N/A |
| | CDSP | 0 to 20,000 | 2E-02 | N/A |
| Liquid Influx | CSNF | 0 to 20,000 | 3E-03 | 3E-03 |
| | CDSP | 0 to 20,000 | 4E-03 | 4E-03 |

Source: Sections 6.10.3 and 6.10.4.

NOTE: The CSNF waste package does not contain any F⁻ or Cl⁻, so for the vapor-influx case that condenses pure water onto the waste package materials, there are no F⁻ or Cl⁻ abstractions. The CDSP waste package contains F⁻, but no Cl⁻.

The mapping of batch reactor cases to the abstractions is presented in Table 6-21.

8.2 YMRP CRITERIA

The following acceptance criteria are from Section 2.2.1.3.3 of *Yucca Mountain Review Plan, Final Report* (NRC 2003 [DIRS 163274]) and are based on the requirements of 10 CFR 63.114 (a)-(c) and (e)-(g) [DIRS 173273].

Acceptance Criterion 1 – System Description and Model Integration Are Adequate

(1) TSPA adequately incorporates important design features, physical phenomena and couplings and uses consistent and appropriate assumptions throughout the quantity and chemistry of water contacting waste packages and waste forms abstraction process;

This model accurately reflects the design features of the repository, including the design of the waste package and waste form, as discussed in Sections 4.1.3 and 4.1.4, and the design of the drift as it impacts the incoming quantity and chemistry of water, as discussed in Section 4.1.2. The physical phenomena and couplings important to the process of possible degradation of the waste package and waste form, are discussed in Section 5 and in the conceptual models in Sections 6.3.1 and 6.3.2. The use of assumptions in this document is consistent throughout this document and with assumptions used in other reports, as discussed in Sections 5, 6.3.1, and 6.3.2. An example of one of these assumptions includes the oxidation state considered in this report, which is fully oxidizing for the base case, but a sensitivity study was included in Section 6.6.4, which examines the impact of using a slightly lower oxidation state used in *Dissolved Concentration Limits of Radioactive Elements* (BSC 2005 [DIRS 173873]). A second example is the incoming waters used in this report and is discussed in Section 4.1.2. These waters are considered in a sensitivity study in Section 6.6.2. These compositions were taken from *Drift-Scale THC Seepage Model* (BSC 2005 DIRS 172862)]. The seepage water compositions used as input to the calculations of *Engineered Barrier System: Physical and Chemical Environment Model* (BSC 2004 [DIRS 169860]) are also based on the results of the THC model. A third example is the use of surface properties for iron minerals, described in Section 6.3.2 and 6.7, which are potentially different than the sorption portion of *EBS Radionuclide Transport* (BSC 2004 [DIRS 169868]). To address this potential difference, a sensitivity study was conducted in this report using various site densities in Section 6.7.

(2) The abstraction of the quantity and chemistry of water contacting the waste packages and waste forms uses assumptions, technical bases, data and models that are appropriate and consistent with other related DOE abstractions.

The abstraction of the in-package water is described in Section 6.10, and uses assumptions, technical bases, data and models listed in Sections 5, 6.3.1, 6.3.2, and 4.1, respectively, which are consistent with other model abstractions.

The seepage rates and compositions selected as input for this model (Section 4.1.2) are consistent with *Abstraction of Drift Seepage* (BSC 2004 [DIRS 169131])., *Physical and Chemical Environment* (BSC 2004 [DIRS 169860]), and *Drift-Scale THC Seepage Model* (BSC 2005 [DIRS 172862]). Other models use the chemistry of the water determined in this report within the TSPA-LA and are, therefore, consistent; these models include: *Waste Form and In-Drift*

Colloids (BSC 2004 [DIRS 170025]), *Dissolved Concentration Limits of Radioactive Elements* (BSC 2005 [DIRS 173027173873]), *CSNF Waste Form Degradation* (BSC 2004 [DIRS 169987]), and *Defense HLW Glass Degradation* (BSC 2004 [DIRS 169988]).

The significance of the quantity of water modeled as contacting the waste packages in this model is described in Section 6.10.2.4. Some assumptions in this model related to the quantity of water contacting the waste package, described in Section 6.3.1.1, could potentially differ from other TSPA-LA models, such as the quantity of water associated with the thin films described in *EBS Radionuclide Transport* (BSC 2004 [DIRS 169868]), or the quantity of water required to fill the rind volumes associated with degraded CSNF fuel and degraded HLWG, described in *CSNF Waste Form Degradation* (BSC 2004 [DIRS 169987]) and *Defense HLW Glass Degradation* (BSC 2004 [DIRS 169988]), so the impact of these assumptions is analyzed in Section 6.6.1 and accounted for in the abstracted model in Section 6.10.2.

(3) Important design features, such as waste package design and material selection, backfill, drip shield, ground support, thermal loading strategy, and degradation processes are adequate to determine the initial and boundary conditions for calculations of the quantity and chemistry of water contacting waste packages and waste forms.

Throughout this model, information is presented related to the design of the waste package (Section 4.1.4) and waste form (Section 4.1.3), degradation processes (Sections 4.1.4 and 6.3.1.1), physical phenomena (Section 6.3.1.1), and couplings that may affect the degradation of the engineered barriers (Section 6.3.1.1). These descriptions are consistent with the information delineated elsewhere in other reports and are adequate for their intended purposes (i.e., to enable the modelers to develop and implement appropriate models and to enable the reviewer to understand the bases for the analytical activities performed by the applicant and those results)

(4) Spatial and temporal abstractions appropriately address physical couplings (thermal-hydrologic-mechanical-chemical).

The spatial abstraction for the batch reactor model is described in Section 6.3.1.1 and appropriately addresses physical couplings. The temporal abstractions appropriately address physical couplings by using degradation rates that account for coupled processes for the waste package materials and the waste forms (Sections 4.1.3 and 4.1.4). Additionally, the incoming water compositions (Section 4.1.2) are consistent with *Drift-Scale THC Seepage Model* (BSC 2005 [DIRS 172862]), which accounts for coupled processes.

(5) Sufficient technical bases and justification are provided for TSPA assumptions and approximations for modeling coupled T-H-M-C effects on seepage and flow, the waste package chemical environment, and the chemical environment for radionuclide release. The effects of the distribution of flow on the amount of water contacting the waste package and waste form are consistently addressed in all relevant abstractions.

The incoming water compositions (Section 4.1.2) are consistent with *Drift-Scale THC Seepage Model* (BSC 2005 [DIRS 172862]), which accounts for coupled processes. The amount of water contacting the waste package and waste form is an input consistent with *Abstraction of Drift Seepage* (BSC 2004 [DIRS 169131]), which provides the incoming seepage rates for this report. This model does not explicitly model evaporation, which is consistent with *Abstraction of Drift Seepage* (BSC 2004 [DIRS 169131]), but is potentially different from other models, specifically *Engineered Barrier System: Physical and Chemical Environment Model* (BSC 2004 [DIRS 169860]). The justification for this difference is described in Sections 4.1.2 and 6.3.1.1, and the impacts of this potential difference are addressed in the model abstraction, as described in Section 6.10. The vapor-influx rates used in this report (Section 4.1.2), could potentially be different than other TSPA-LA models, as described in Section 6.10.2.4, including *EBS Radionuclide Transport* (BSC 2004 [DIRS 169868]); therefore, the impact of the rate of flow is examined in Section 6.6.5, and the results are incorporated into the abstracted results in Section 6.10. The vapor-influx rates used in this model are independent of drift conditions, as currently implemented in the TSPA-LA, which is different than *Engineered Barrier System: Physical and Chemical Environment Model* (BSC 2004 [DIRS 169860]), which uses in-drift environmental conditions to determine the chemistry of the water for the purposes of colloid stability. Calculation of the vapor influx rates as a function of the drift conditions is examined in an alternative conceptual model in *EBS Radionuclide Transport* (BSC 2004 [DIRS 169868]) as well as in this model (Section 6.4.3), and both calculations support the range of vapor influx rates selected as input to this report. The impact of varying these rates is incorporated into the ionic strength abstraction (Section 6.10.2), where the vapor influx rate is varied from 0.1 to 10 liters per year. The alternative conceptual model (Section 6.4.3) shows an alternative implementation for the TSPA-LA to calculate the vapor-influx rate based on the in-drift temperature and relative humidity.

(6) The expected ranges of environmental conditions within the waste package emplacement drifts, inside of breached waste packages, and contacting the waste forms and their evolution with time are identified. These ranges may be developed to include: (i) the effects of drip shield and backfill on the quantity and chemistry of water; (ii) conditions that promote corrosion of engineered barriers and degradation of waste forms; (iii) irregular wet and dry cycles; (iv) gamma-radiolysis; and (v) size and distribution of penetrations of waste packages.

This report uses the expected ranges of environmental conditions of the drift as inputs (Sections 4.1.2 and 6.3.1.1) and then determines the resulting ranges of environmental conditions inside the waste package (Section 6.10). The quantity of water entering the waste package is dependent upon the condition of the drip shield and waste package and the size and distribution of penetrations of waste packages, and this model varies the quantity to encompass a broad range of conditions, as described in Section 6.3.1.3. The liquid-influx rates used in this model come from *Abstraction of Drift Seepage* (BSC 2004 [DIRS 169131]), and vapor-influx rates come from an alternative conceptual model in *EBS Radionuclide Transport* (BSC 2004 [DIRS 169868]), which is enhanced in Section 6.3.4. These influx rates are calculated based on the expected environmental conditions and the conditions of the waste package and drip shield. Other environmental conditions considered in this model include $f\text{CO}_2$, $f\text{O}_2$, and temperature, which are varied across the possible range in the emplacement drifts in Sections 6.6.4 ($f\text{CO}_2$ and

fO_2) and 6.6.6 (temperature), and these results are incorporated into the abstracted results in Section 6.10. This model specifically studies the effects of gamma radiolysis on water composition, as presented in Appendix B.

(7) The model abstraction for quantity and chemistry of water contacting the waste packages and waste forms is consistent with the detailed design information on waste package design and other engineered features.

This model uses the design of the waste package and the waste package materials as input, as described in Section 4.1.4, and the input files for the model are based on this design information.

(8) Adequate technical bases are provided, including activities such as independent modeling, laboratory or field data, or sensitivity studies, for inclusion of any T-H-M-C couplings and features, events and processes;

This model is validated by laboratory data that studies coupled processes, as described in Section 7.2.2. The corrosion rates used by this model were determined by laboratory and field data, which include couplings, as described in Sections 4.1.3 and 4.1.4. This report addresses eight FEPs considered appropriate, as is discussed in Section 6.2. Table 6-1 includes a detailed listing of where within this report the specific FEPs are discussed and addressed. The technical information contained in Sections 4, 5, 6, and 7 is sufficient to provide the basis for the inclusion or exclusion decisions related to these FEPs.

(9) Performance affecting processes that have been observed in thermal-hydrologic tests are included in the TSPA.

The effects observed in thermal-hydrologic tests and experiments (from the drift scale test) are used for validation purposes in *Drift-Scale THC Seepage Model* (BSC 2005 [DIRS 172862], Section 7.1), and the incoming water compositions (Section 4.1.2) are consistent with that model.

(10) Likely modes for container corrosion are identified and considered in determining the quantity and chemistry of water entering the waste packages and contacting the waste forms.

The quantity of water entering the waste package is dependent upon the condition of the drip shield and waste package and the size and distribution of penetrations of waste packages, and this model varies the quantity to encompass a broad range of conditions, as described in Section 6.3.1.3. Section 6.10 determines the chemistry of the water within the waste package is a function of the degradation of the waste package materials and waste forms. These degradation processes are described in Sections 6.3.1 and 6.3.2.

(11) The abstraction of the in-package criticality or external-to-package criticality within the emplacement drifts provides an adequate technical basis for screening these events. If either event is included in the TSPA, then the DOE uses acceptable technical bases for selecting the design criteria that mitigate the potential impact of in-package criticality on the repository performance; identifies the FEPs that may increase the reactivity of the system inside the waste package, identifies the configuration classes and configurations that have the

potential for nuclear criticality; and includes changes in thermal conditions and degradation of engineered barriers in the abstraction of the quantity and chemistry of water contacting waste packages and waste forms.

The subject of potential criticality is discussed in other reports. This criterion is considered not to be applicable to this report.

(12) Guidance in NUREG 1297 and NUREG 1298 (Altman et al. 1988 a, b) or other acceptable approaches is followed.

The information contained in this report is based on established physical models, justified assumptions and previously qualified input data. As a result, there was no need to use the guidance of NUREG 1297 and/or NUREG 1298 in the development of this report.

Acceptance Criterion 2 – Data Are Sufficient For Model Justification

(1) Geological, hydrological and geochemical values used in the License Application are adequately justified. Adequate description of how the data were used, interpreted and appropriately synthesized into the parameters is provided.

The sources for the geological, hydrological, and geochemical data are justified in Sections 4.1.1 through 4.1.6 and 6.3.1.3 describes how the data were used and converted into values appropriate for the model input files. The chemical thermodynamic database used in the modeling was developed specifically for the project and is consistent with the thermodynamic data being used by other reports.

(2) Sufficient data were collected on the characteristics of the natural system and engineered materials to establish initial and boundary conditions for conceptual models of T-H-M-C coupled processes that affect seepage and flow and the waste package environment;

The sources for the geological, hydrological, and geochemical data (Sections 4.1.1 through 4.1.6) come from a broad body of data collected on the characteristics of the natural system and engineered materials. Section 6.3.1.3 describes how the data were used to establish initial and boundary conditions.

(3) Thermo-hydrologic tests were designed and conducted with the explicit objective of observing T-H processes for the temperature ranges anticipated in the repository and making measurements for mathematical models. Data are sufficient to verify that T-H conceptual models address important T-H phenomena;

Natural system data are used to develop *Drift-Scale THC Seepage Model* (BSC 2005 [DIRS 172862]), which is consistent with the water composition input to this model (Section 6.6). The natural system data are used to establish initial and boundary conditions that affect seepage and flow and are discussed in Section 6.3.1.1.

(4) Sufficient information to formulate the conceptual approaches for analyzing water contact with the drip shield, waste packages and waste forms is provided;

The conceptual model for water contacting the waste package materials and waste forms is provided in Section 6.3.1.1.

(5) Sufficient data are provided to complete a nutrient and energy inventory calculation, if it has been used to justify the inclusion of the potential for microbial activity affecting the waste package chemical environment and chemical environment for radionuclide release. As necessary, data are adequate to support determination of the probability for microbially induced corrosion and microbial effects, such as the production of organic by-products and microbially enhanced dissolution of the high-level radioactive waste glass form.

Microbially influenced corrosion is not addressed in this report.

Acceptance Criterion 3 - Data Uncertainty is Characterized and Propagated Through the Model Abstraction

Most of the input values were treated as variables and were varied over a wide range of possible values for the base case (Sections 6.3.1 and 6.3.2). In addition, many sensitivity analyses (Section 6.6) were run, which varied additional parameters, as well as expanded range of values that were used in the base case. The results of these analyses were used in the abstraction of the model to ensure that the model output captured the full possible range of variability (Section 6.10). In this way, the results of this model capture the expected behavior and the most extreme possibilities, so do not underrepresent the estimate of risk.

(1) Models use parameter values, assumed ranges, probability distributions and/or bounding assumptions that are technically defensible, reasonably account for uncertainties and variabilities, and do not result in under-representation of the risk estimate.

Each of the models developed in this report use parameter values, assumed ranges, probability distributions or bounding assumptions, or both, that are technically defensible, reasonably account for uncertainties and variabilities, and do not result in underrepresentation of the risk estimate. In each situation, discussion and consideration of the uncertainties associated with specific data are addressed in detail. The treatment of uncertainty is explicitly addressed in Section 6.10.8 and specific sections throughout the document including Sections 6.6 and 6.10.2..

(2) Parameter values, assumed ranges, probability distributions and bounding assumptions used in the TSPA calculations of quantity and chemistry of water contacting waste packages and waste forms are technically defensible and reasonable based on data from the Yucca Mountain region (e.g., results from large block and drift-scale heater and niche tests) and a combination of techniques that may include laboratory experiments, field measurements, natural analog research and process modeling studies.

The models developed and discussed in this AMR use three different input water compositions for the base case, two of which are based on waters obtained from core samples proximal to the repository. The model uses three additional water compositions from *Drift-Scale THC Seepage Model* (BSC 2005 [DIRS 172862]) in a sensitivity analysis in Section 6.6.2. This subject is discussed in detail in Sections 4.1.2 and 6.3.1.3.1.

(3) Input values used in the TSPA calculations of quantity and chemistry of water contacting engineered barriers (e.g., drip shield and waste package) are consistent with the initial and boundary conditions and the assumptions of the conceptual models and design concepts for the Yucca Mountain site. Correlations between input values are appropriately established in the DOE TSPA. Parameters used to define initial conditions, boundary conditions and computational domain in sensitivity analyses involving coupled T-H-M-C effects on seepage and flow, the waste package chemical environment, and the chemical environment for radionuclide release, are consistent with available data. Reasonable or conservative ranges of parameters or functional relations are established.

The inputs to this model (Section 4.1.1 through 4.1.6) are consistent with initial boundary conditions and design concepts for the Yucca Mountain site because they come from various design documents and other related model reports. The results of this model (and others) are validated by comparison with laboratory data and natural analog, as described in Section 7.2. The model abstraction, developed in Section 6.10, uses reasonable and conservative abstractions to capture the range of output.

(4) Adequate representations of the uncertainties in the characteristics of the natural systems and engineered materials is provided in the parameter development for conceptual models, process level models, and alternative conceptual models. The DOE may constrain these uncertainties using sensitivity analyses or conservative limits.

Most of the input values were treated as variables and were varied over a wide range of possible values for the base case (Section 6.3.1 and 6.3.2). In addition, many sensitivity analyses (Section 6.6) were run, which varied additional parameters, as well as expanded range of values that were used in the base case. The results of these analyses were used in the abstraction of the model to ensure that the model output captured the full possible range of variability (Section 6.10). In this way, the results of this model capture the expected behavior and the most extreme possibilities, so do not under-represent the estimate of risk.

(5) If criticality is included in the TSPA, then the DOE uses an appropriate range of input parameters for calculating the effective neutron multiplication factor;

The subject of possible criticality either in or external to the waste package is not discussed in this report. As such, this criterion is considered not to be applicable to this report.

(6) Where sufficient data do not exist, the definition of parameter values and conceptual models is based on appropriate use of other sources, such as expert elicitation conducted in accordance with NUREG-1563.

The information contained in this report is based on established physical models, justified assumptions and previously qualified input data. As a result, there was no need to use other sources, such as expert elicitation conducted in accordance with NUREG-1563, in the development of this report.

Acceptance Criterion 4 – Model Uncertainty is Characterized and Propagated Through the Model Abstraction

Uncertainties in inputs and in the methodology are considered in the development of the conceptual model (Sections 6. 3) and output parameters (Section 6.10). Discussion of uncertainty in the conceptual model appears in various places throughout the report. The model uncertainty is consistent with available site data, laboratory experiments and natural analog as described in Section 7. The use of the output of the model by TSPA-LA was analyzed and the uncertainty treatment was altered so as to lead to the most conservative results with respect to colloid stability and radionuclide solubility and thus resulted in over-representation of the risk estimate.

(1) Alternative modeling approaches of FEPs are considered and are consistent with available data and current scientific understanding; and the results and limitations are appropriately considered in the abstraction.

This report explicitly addresses alternative conceptual models related to the makeup of the water entering the waste package after postulated breach and the manner in which this water transits the waste package following entry in Section 6.4. Although due consideration was given to the use of these models, it was determined that the selected “base-case” model and selected input data is sufficient, robust and adequate for its intended purpose and that the use of the alternative conceptual models was not warranted.

(2) Alternative modeling approaches are considered and the selected modeling approach is consistent with available data and current scientific understanding. A description that includes a discussion of alternative modeling approaches not considered in the final analysis and the limitations and uncertainties of the chosen model is provided

This report explicitly addresses alternative conceptual models related to the make-up of the water entering the waste package after postulated breach and the manner in which this water transits the waste package following entry in Section 6.4. Although due consideration was given to the use of these models, it was determined that the selected “base-case” model and selected input data is sufficient, robust and adequate for its intended purpose and that the use of the alternative conceptual models was not warranted.

(3) Consideration of conceptual model uncertainty is consistent with available site characterization data, laboratory experiments, field measurements, natural

analog information, and process-level modeling studies; and the treatment of uncertainty does not result in under-representation of the risk estimate.

Each of the models developed in this report use parameter values, assumed ranges, probability distributions or bounding assumptions that are consistent with available site characterization data, technically defensible, reasonably account for uncertainties and variabilities, and do not result in underrepresentation of the risk estimate. In each situation, discussion and consideration of the uncertainties associated with specific data are addressed in detail. The treatment of uncertainty is explicitly addressed in Section 6.10.8 and specific sections throughout the document, including Sections 6.6 and 6.10.2..

(4) Adequate consideration is given to the effects of thermal-hydrologic-mechanical-chemical coupled processes in the assessment of alternative conceptual models.

Alternative Conceptual Model I (Section 6.4.1) considers the effects of a nonhomogenous system as a result of coupled processes, such as the effects of gas and changes in boundary conditions. Alternative Conceptual Model II (Section 6.4.2) considers the composition of seepage entering a waste package is likely to vary as a function of changing conditions in the unsaturated zone and drift environments.

(5) If the DOE uses an equivalent continuum model for the TSPA abstraction, the models produce conservative estimates of the effects of coupled THMC processes on calculated compliance with post-closure public health and environmental standards.

The subject of an equivalent continuum model is not discussed in this report. As such, this criterion is considered not to be applicable to this report.

Acceptance Criterion 5 – Model Abstraction Output is Supported By Objective Comparisons

The output of the model abstraction was compared with laboratory experiments and natural analog as described in Section 7.2. There are no confirmatory actions, such as compliance runs, additional sensitivity runs, and neutralization runs associated with this report.

(1) The models implemented in this TSPA abstraction provide results consistent with output from detailed process level models and/or empirical observations (laboratory and field testings and/or natural analogues);

The results of models implemented in this total system performance assessment abstraction were compared with results from detailed process-level models and or empirical observations (laboratory and field testing, or natural analogs) to ensure consistency. The results of such comparisons are addressed in Section 7 and Appendix E.

(2) Abstracted models for coupled THMC effects on seepage and flow and the waste package chemical environment, as well as on the chemical environment for radionuclide release, are based on the same assumptions and approximations

demonstrated to be appropriate for process level models or closely analogous natural or experimental systems.

The model abstraction is validated in Section 7.3, and is based on the same assumptions and approximations used in other reports.

(3) Accepted and well-documented procedures are used to construct and test the numerical models that simulate coupled THMC effects on seepage and flow, waste package chemical environment, and the chemical environment for radionuclide release. Analytical and numerical models are appropriately supported. Abstracted model results are compared with different mathematical models to judge robustness of results.

Accepted and well-documented procedures contained in the quality assurance program (Section 2) governed the development of this report and the work it documents. This model was constructed, supported, and documented according to LP-SIII.2Q-BSC. Test and validation methods (Section 7), including comparison of abstracted output with that of other relevant models, also comply with LP-SIII.2Q-BSC, as well as with applicable written guidance. This report was generated according to the requirements of *Technical Work Plan for Postclosure Waste Form Modeling* (BSC 2005 [DIRS 173246]) as directed by LP-2.29Q-BSC, *Planning for Science Activities*.

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9.2 CODES, STANDARDS, REGULATIONS, AND PROCEDURES

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9.3 SOFTWARE CODES

- 153964 *Software Code: EQ3/6. V7.2b. UCRL-MA-110662 (LSCR198).*
- 159731 *Software Code: EQ6. 7.2bLV. PC. 10075-7.2bLV-02. Windows NT, 2000.*
- 173680 *Software Code: GetEQData. V1.0.1. PC w/Windows 2000. 10809-1.0.1-00.*
- 157837 *Software Code: PHREEQC. V2.3. PC. 10068-2.3-01.*
- 155029 *Software Code: transl. V2.0. PC Windows98. 10251-2.0-00.*

9.4 SOURCE DATA, LISTED BY DATA TRACKING NUMBER

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- 161677 LL020709923142.023. Aqueous Geochemistry of Borehole Waters Collected in the Heating Phase of the DST. Submittal date: 07/26/2002.
- 173475 LL980710551022.012. Unsaturated Drip Condition Testing of Spent Fuel and Unsaturation Dissolution Tests of Glass. Submittal date: 08/11/1998.
- 151029 MO0006J13WTRCM.000. Recommended Mean Values of Major Constituents in J-13 Well Water. Submittal date: 06/07/2000.
- 160229 MO0210SPATHDYN.000. Thermodynamic Data Input Files - DATA0.YMP.R2. Submittal date: 10/02/2002.
- 161756 MO0302SPATHDYN.000. Thermodynamic Data Input Files - Data0.YMP.R2. Submittal date: 02/05/2003.
- 162385 MO0302PNLDUFTD.000. Flowthrough Dissolution Data. Submittal date: 02/28/2003.
- 162551 MO0303MWDSCMAB.000. THC Seepage Chemistry Model Abstraction Binning EQ3 Input, Pickup and Output Files. Submittal date: 03/06/2003.
- 170760 MO0407SEPFELA.000. LA FEP List. Submittal date: 07/20/2004.
- 170618 MO0407SEPUDISR.000. Uranium Dissolution Rates from Uranium Dioxide. Submittal date: 07/01/2004.
- 172097 MO0407SPAPCEML.005. EBS P&CE Model Longevity of Materials Evaluation. Submittal date: 07/02/2004.

- 172059 MO0409SPAACRWP.000. Aqueous Corrosion Rates For Non-Waste Form Waste Package Materials. Submittal date: 09/16/2004.
- 172244 MO0411SPAMOICA.000. Model Output for In-Package Chemistry Abstraction. Submittal date: 11/04/2004.
- 172712 SN0410T0510404.002. Thermodynamic Database Input File for EQ3/6 - DATA0.YMP.R4. Submittal date: 11/01/2004.

9.5 OUTPUT DATA, LISTED BY DATA TRACKING NUMBER

MO0502SPAINPCA.000. In-Package Chemistry Abstraction. Submittal date: 02/25/2005.

MO0403SPASURCR.000. Surface complexation reactions in PHREEQC format from the surface complexation demonstration (Section 6.8).

MO0403SPAYMPR3.000. Surface complexation demonstration (Section 6.8) thermodynamic database for PHREEQC (YMP_R3.dat) with abridged surface complexation reactions.

MO0403MWDSRFCC.000. Surface complexation demonstration (Section 6.8) input and output PHREEQC files, and the expanded PHREEQC database (YMP_R3mr.dat), which contains surface complexation reactions.

MO0312SPAICSCR.002. Developed in Appendix B, and documents the effect of radiolysis on the in-package chemistry.

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APPENDIX A
ADDITIONAL IN-PACKAGE CHEMISTRY DISCUSSION AND PLOTS

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A. ADDITIONAL IN-PACKAGE CHEMISTRY DISCUSSION AND PLOTS

A.1 CSNF WATER CHEMISTRY EVOLUTION

This section provides a detailed discussion of the chemistry evolution of the CSNF base case. This discussion is specific to the base case and the time steps, degradation rates and quantities of materials used. However, the trends observed in this base case are representative of the CSNF waste package in general.

The discussion begins at 31.7 years when A516 has been completely degraded. The oxidation of S and P as A516 degrades rapidly are the principle acid-producing reactions of waste package degradation. Additional acid is produced by the oxidation of S, P and Mo in the NiGd and 316 components of the package, but these degrade at lower rates. The initial, rapid degradation of A516 drives the pH down and it has reached its minimum value, 4.39, at 31.7 years when A516 degradation ceases. Note that this value is modeled without considering the effects of H^+ sorption on Fe reaction products. This sorption buffers the pH at higher values as discussed in Section 6.3.2.

After 31.7 years, the pH begins to increase from its minimum value. With increasing pH, the concentrations of several elements also change. For example, elements including Ti (below concentrations shown in figure), Zr, Fe, Cr, Mn, Zn, Al, Np and U that are controlled by oxide and hydroxide solids begin to decrease: Gd also decreases because it is controlled by $GdPO_4 \cdot 10H_2O$ solubility. With increasing pH, phosphate hydrolysis decreases, increasing the activity of PO_4^{3-} and lowering Gd activity to maintain $GdPO_4 \cdot 10H_2O$ equilibrium. Ba enters the solution as CSNF continues to degrade precipitating barite ($BaSO_4$) and lowering the total S concentration. With both the decreasing total S and Al and increasing pH, the previously-precipitated alunite ($KAl_3(OH)_6(SO_4)_2$) begins to dissolve increasing the K concentration. At this time, the Mo concentration is controlled by $Fe_2(MoO_4)_3$. As Fe decreases, total Mo increases to maintain equilibrium with this solid.

During this gradual pH increase, several additional solids begin to precipitate. At 37.7 years, $PuO_2(OH)_2 \cdot H_2O$ precipitation begins causing the Pu concentration to decrease. At 42.1 years, spinel-Co (Co_3O_4) begins to precipitate and the Co concentration falls. At 43.6 years, powellite ($CaMoO_4$) begins to form, which causes the Ca concentration to drop. At 47.0 years, nontronite-Mg ($Mg_{0.165}Fe_2Al_{0.33}Si_{3.67}H_2O_{12}$) forms and the Mg concentration decreases. At 72.6 years, the dissolving alunite is exhausted so the K concentration levels off.

At 78.0 years the $Fe_2(MoO_4)_3$, which has begun to dissolve as Mo is taken up by powellite, is exhausted. The pH then rises sharply to 5.82 at 83.7 years accompanied by decreases in Fe, Cr, Mn, Zn, Gd, Co, Al, Np, U and Mg. During this period, precipitation of antlerite ($Cu_3(SO_4)(OH)_4$) begins at 81.3 years causing Cu to drop. At 83.7 years, precipitation of trevorite ($NiFeO_4$) begins and the pH becomes virtually constant at 5.82. At 115 years, kaolinite ($Al_2Si_2O_5(OH)_4$), which has been dissolving since nontronite-Mg precipitation began at 47.0 years, is exhausted and the pH begins to rise once more.

From 115 years the pH increases gradually reaching 6.14 at 312 years. With the increasing pH, the concentrations of Fe, Cr, Mn, Zn, Co, Al, Np, U, Cu and Ni decrease. At 220 years, tenorite

(CuO) begins to precipitate taking up Cu and causing antlerite to dissolve. At 226 years, the antlerite is exhausted.

The uranium solids schoepite ($\text{UO}_3 \cdot 2\text{H}_2\text{O}$) and $(\text{UO}_2)_3(\text{PO}_4)_2 \cdot 6\text{H}_2\text{O}$ form early in the waste degradation process. At 31.7 years as the pH begins to rise, $(\text{UO}_2)_3(\text{PO}_4)_2 \cdot 6\text{H}_2\text{O}$ begins to dissolve in favor of schoepite. Between 226 and 312 years with the rising pH, the total P decreases until at 312 years, $(\text{UO}_2)_3(\text{PO}_4)_2 \cdot 6\text{H}_2\text{O}$ is exhausted. Because of the decreasing total P the Gd concentration rises steeply to maintain equilibrium with $\text{GdPO}_4 \cdot 10\text{H}_2\text{O}$.

The pH rises sharply beginning at 312 years until it levels off at 7.31 at 388 years. At 322 years with the pH at 6.22, the precipitation of $\text{Gd}_2(\text{CO}_3)_3$ begins. As the pH increases, equilibrium with this solid sharply lowers the Gd concentration which in turn increases the total P content to maintain equilibrium with $\text{GdPO}_4 \cdot 10\text{H}_2\text{O}$.

At 388 years, precipitation of the saponite-tri solid solution $(\text{Ca}_{0.5}, \text{H}, \text{K}, \text{Mg}_{0.5}, \text{Na})_{0.33} \text{Mg}_3 \text{Al}_{0.33} \text{Si}_{3.67} \text{O}_{10}(\text{OH})_2$ begins. This solid persists until 984 years and maintains the pH between 7.31 and 7.32 until it disappears. During this period, concentrations of all elements are also virtually constant except those in the solid solution (*e. g.* K) and Ba and total S. Ba continues to increase as CSNF degrades causing total S to decrease to maintain equilibrium with barite. At 634 years, the Al_6061 component of the package is exhausted. In the absence of a continuing source of Al to the solution, the saponite-tri solid begins to dissolve to maintain gibbsite ($\text{Al}(\text{OH})_3$) equilibrium. This continues until 984 years when the saponite-tri is exhausted.

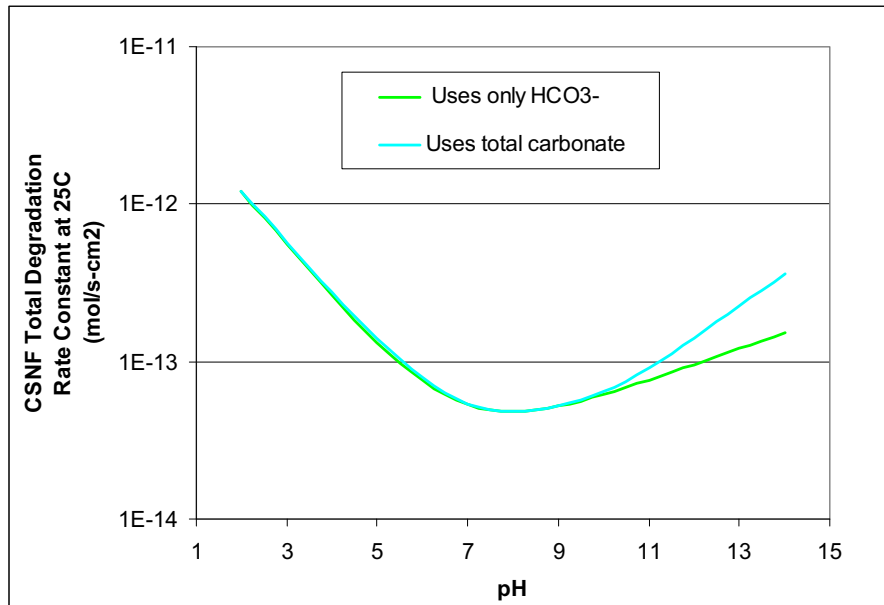
The pH drops from 984 to 2221 years during which time no changes in reaction mechanisms occur. The diagrams show points at 1112, 1659 and 2189 years during this period, but these result from the printing constraints imposed on EQ6, not because of any change in reactions.

At 2221 years, the degrading CSNF is exhausted. Ba is no longer being added to the solution, but with continuing NiGd and 316 dissolution, S continues to be oxidized adding SO_4^{-2} to the solution. With increasing SO_4^{-2} , Ba decreases to maintain equilibrium with barite. Mg decreases with continuing nontronite-Mg precipitation until 2336 years when kaolinite begins to precipitate and Mg stabilizes.

From 2336 until the end of the calculation is reached at 18752 years, the pH remains close to 5.9. No changes in reactants or products occur until 17786 years when $\text{Gd}_2(\text{CO}_3)_3$ is exhausted. At this point, Gd decreases and total P increases until, at 18492 years, $(\text{UO}_2)_3(\text{PO}_4)_2 \cdot 6\text{H}_2\text{O}$ begins to form once again.

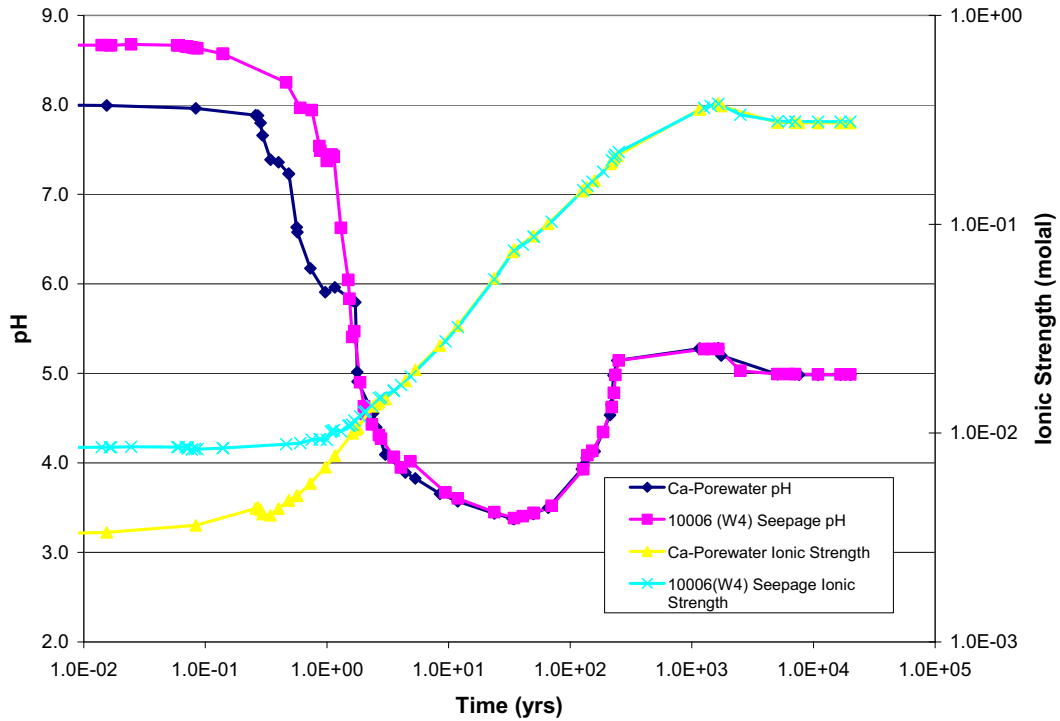
A.2 ADDITIONAL PLOTS

The computer files for the following plots are included in DTN: MO0411SPAMOICA.000 [DIRS 172244]. These plots were not updated from Rev 03 of this report, so for these cases Cr(VI) was allowed to form instead of Cr(III). The impact of using Cr(III) versus Cr(VI) is examined in Section 6.6.3. Because the purpose of these plots is to observe the model response to varying certain inputs, as long as all of the cases being compared are consistent in their treatment of chromium, the cases from Rev 03 using Cr(VI) are still valuable for showing the model response. The source for the input to these sensitivities is documented in Rev 03 of this report.



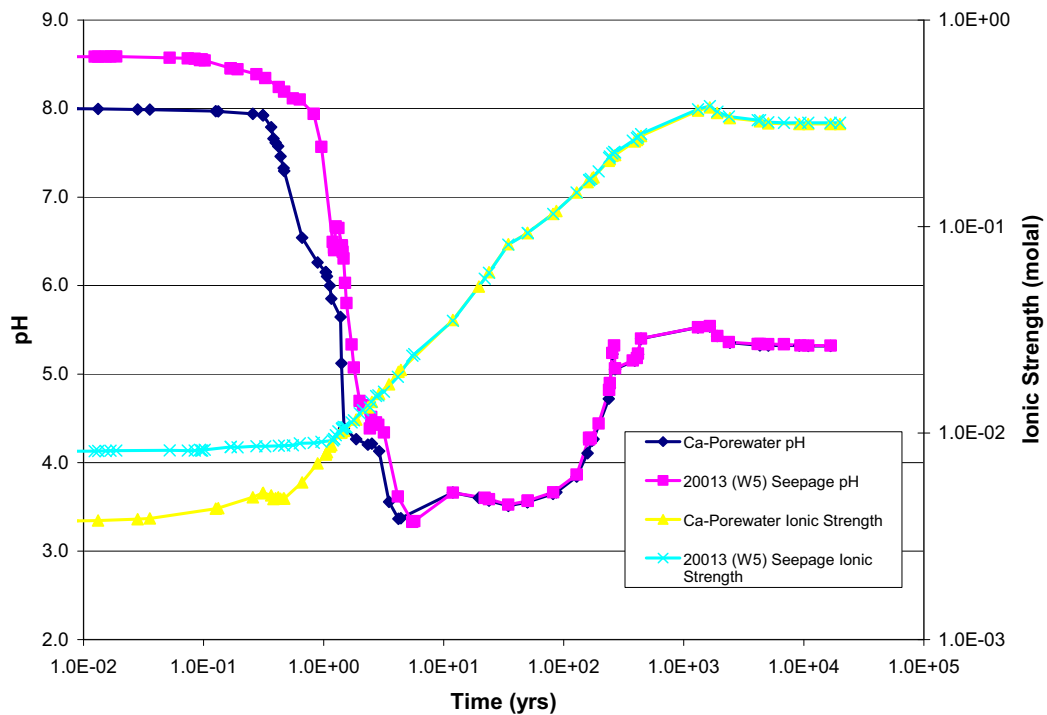
Source: CSNF.xls (Appendix F \Spreadsheets)

Figure A-1. CSNF Total Degradation Rate Constant as a Function of pH



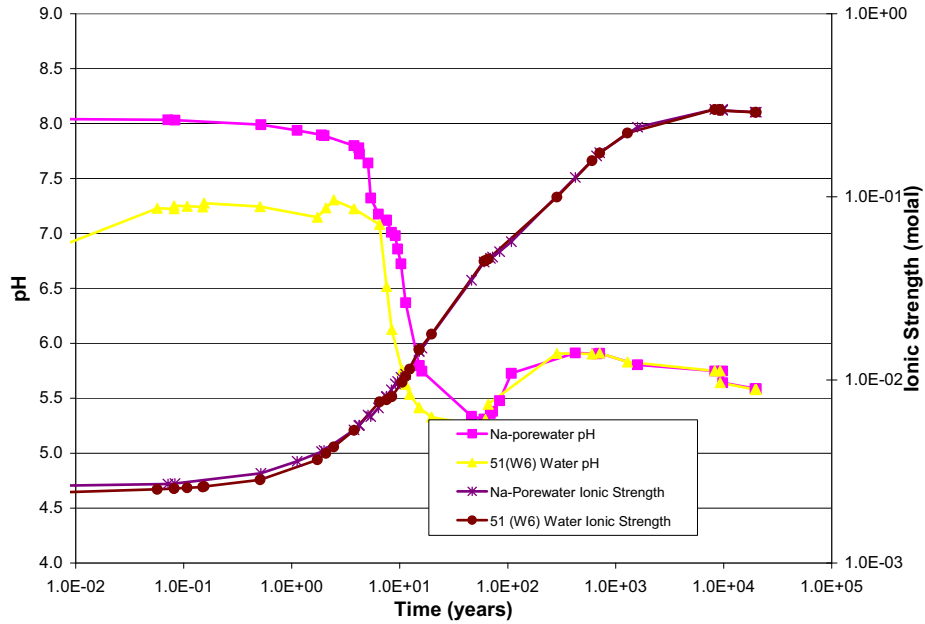
Source: CSNF_P&CE_fluids.xls in DTN: MO0411SPAMOICA.000 [DIRS 172244].

Figure A-2. Thermally Perturbed Composition 10006 (W4) with Ca-porewater (W5) for CSNF



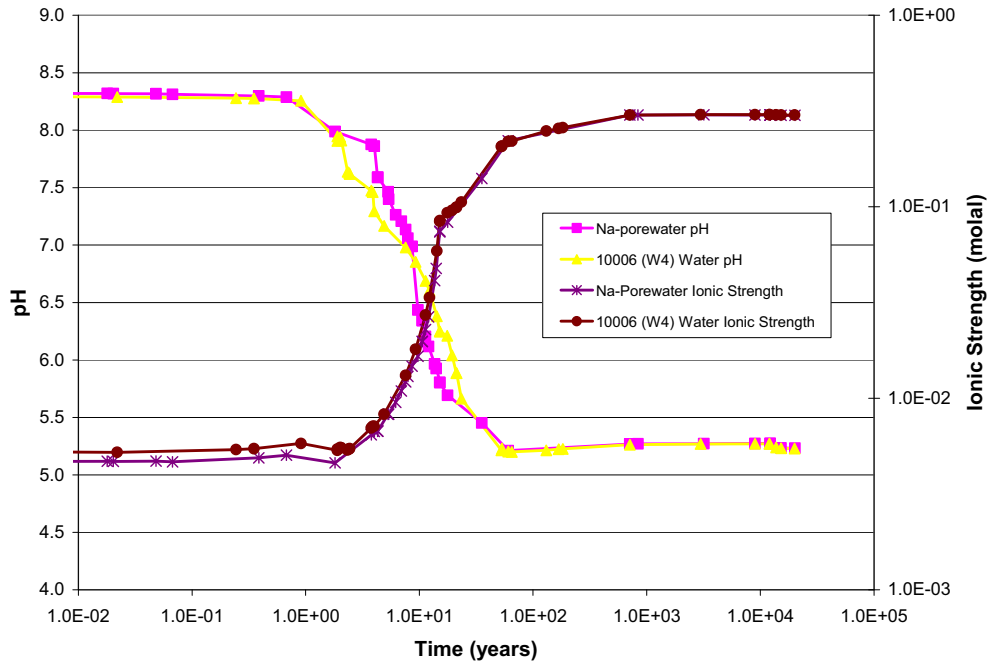
Source: CSNF_P&CE_fluids.xls in DTN: MO0411SPAMOICA.000 [DIRS 172244].

Figure A-3. Thermally Perturbed Composition 20013 (W5) with Ca-porewater (W5) for CSNF



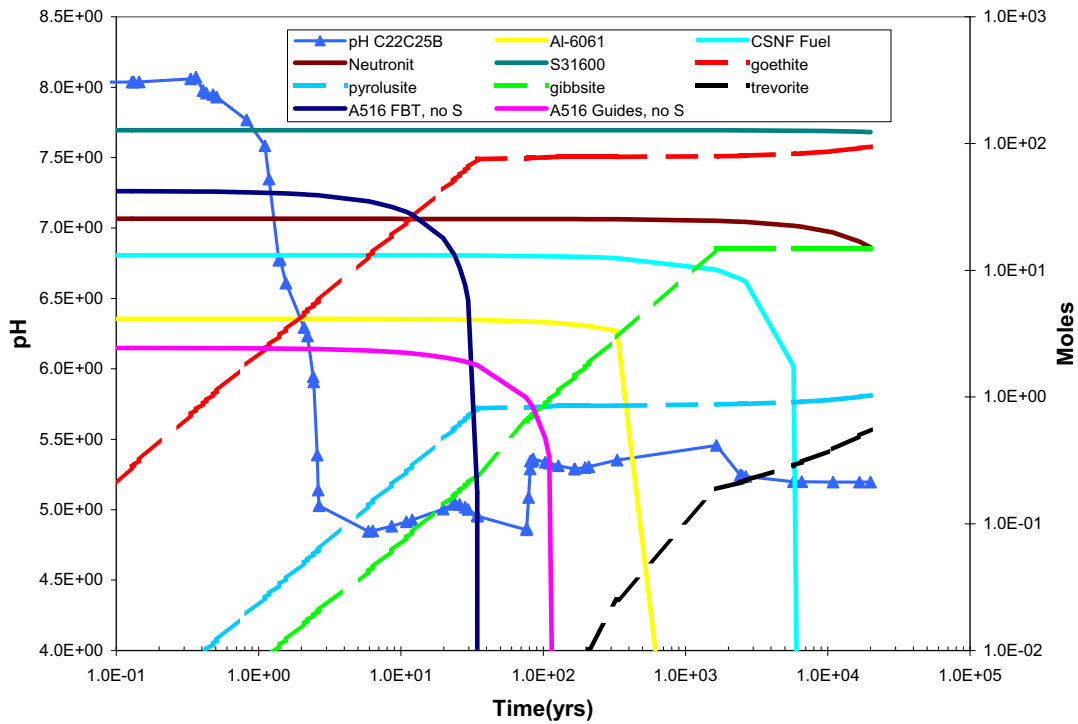
Source: *CDSP_P&CE_fluids.xls* in DTN: MO0411SPAMOICA.000 [DIRS 172244].

Figure A-4. Thermally Perturbed Composition 51 (W6) with Na-porewater (W5) for CDSP



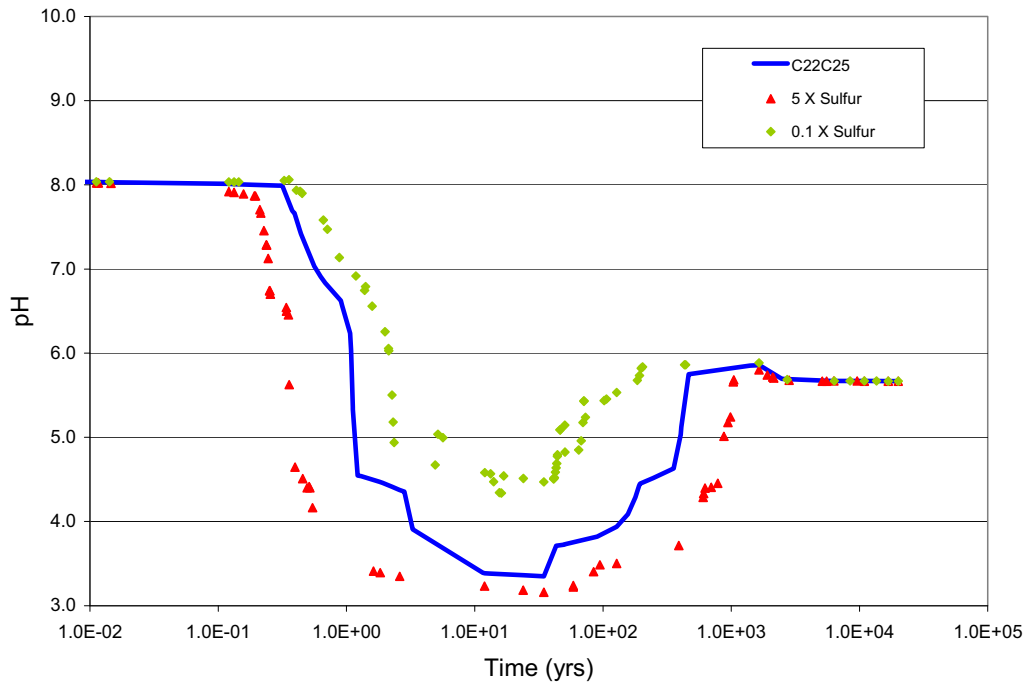
Source: *CDSP_P&CE_fluids.xls* in DTN: MO0411SPAMOICA.000 [DIRS 172244].

Figure A-5. Thermally Perturbed Composition 10006 (W4) with Na-porewater (W5) for CDSP



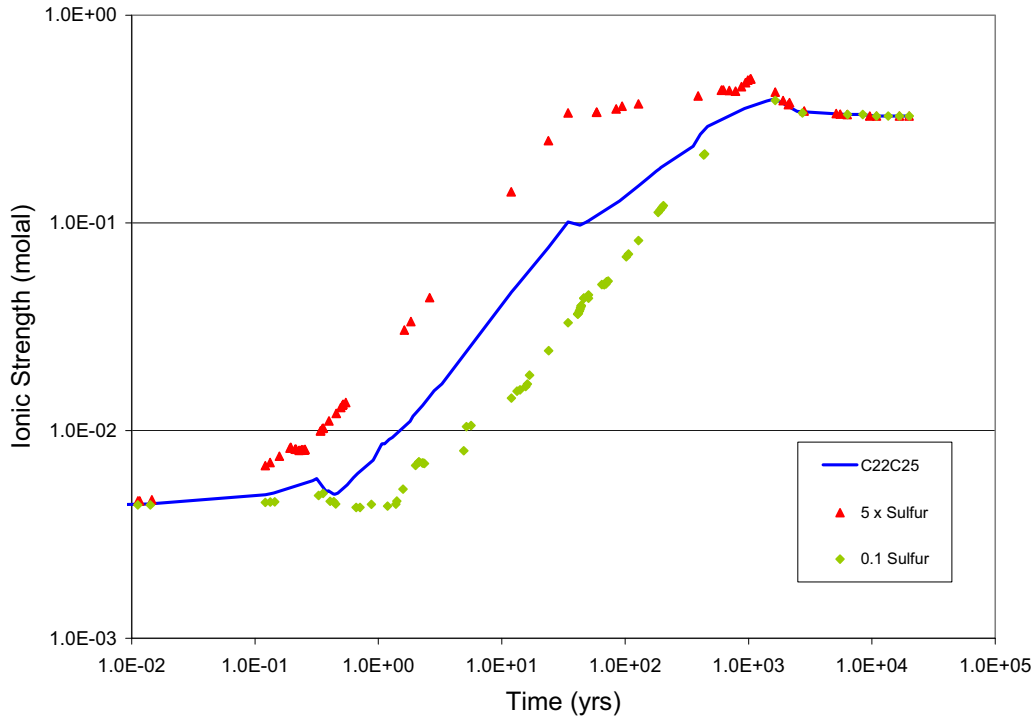
Source: CSNF_Water Influx_25.xls in DTN: MO0411SPAMOICA.000 [DIRS 172244].

Figure A-6. Corrosion Products and pH with no Sulfur in the Carbon Steel Type A516 Composition



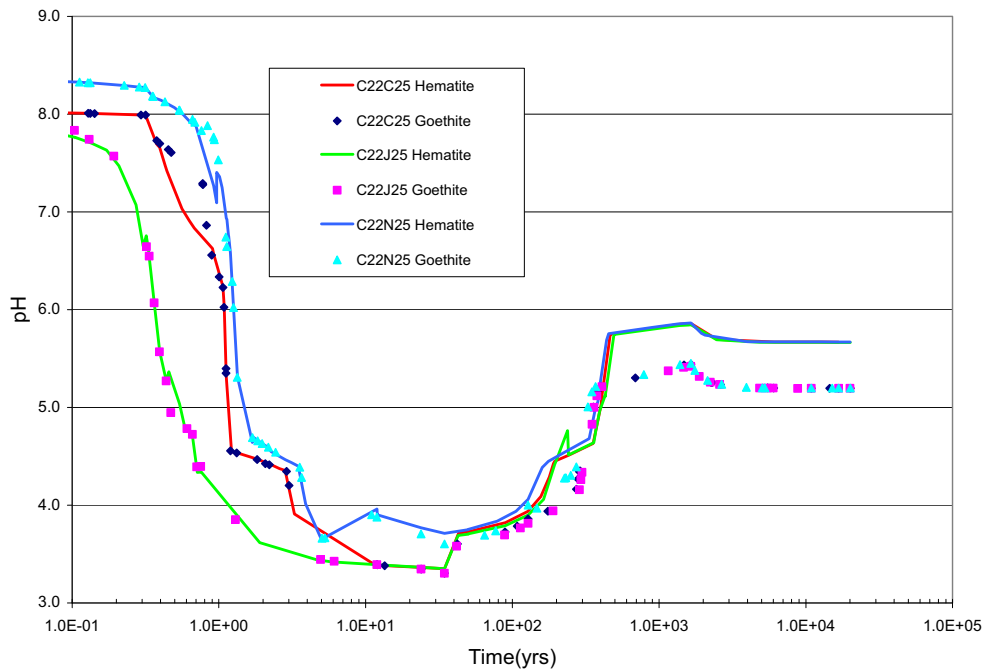
Source: DTN: MO0411SPAMOICA.000 [DIRS 172244], CSNF_Sensitivity.xls.

Figure A-7. Effect of Varying Sulfur Content in Carbon Steel Type A516 on pH



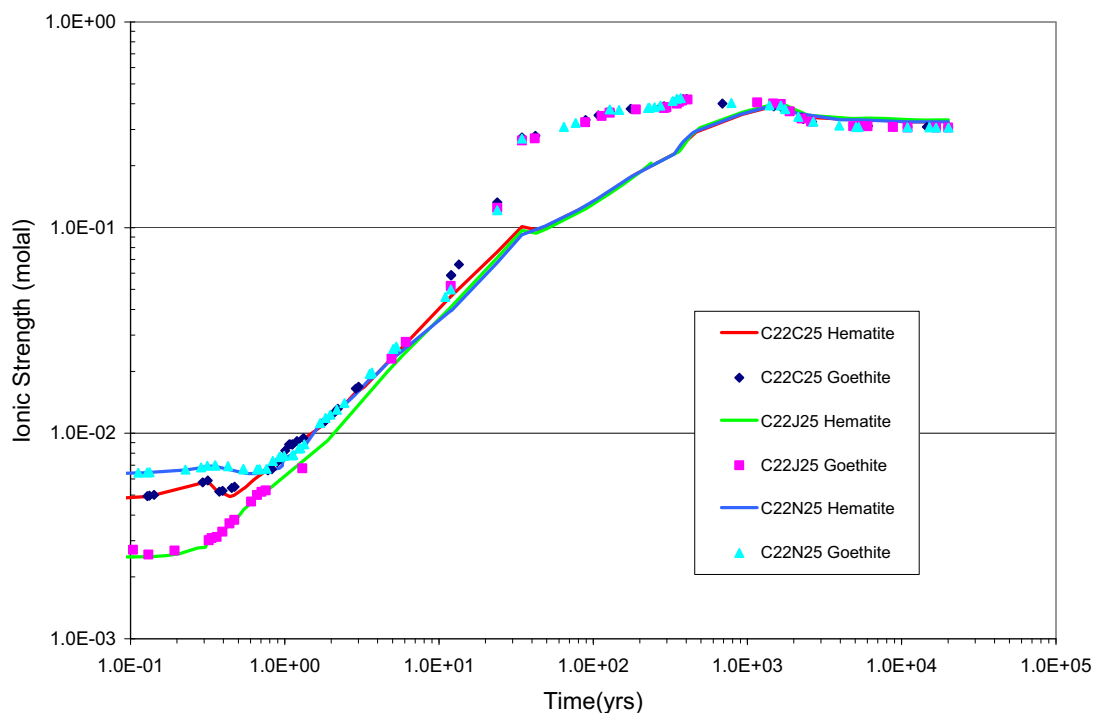
Source: DTN: MO0411SPAMOICA.000 [DIRS 172244], *CSNF_Sensitivity.xls*.

Figure A-8. Effect of Varying Sulfur Content in Carbon Steel Type A516 on Ionic Strength



Source: DTN: MO0411SPAMOICA.000 [DIRS 172244], *CSNF_Water Influx_25.xls*.

Figure A-9. Effect of Goethite and Hematite on pH



Source: DTN: MO0411SPAMOICA.000 [DIRS 172244], CSNF_Water Influx_25.xls.

Figure A-10. Effect of Goethite and Hematite on Ionic Strength

Table A-1. Minor Input Inconsistencies in the EQ3 and EQ6 Input Files

| Input | Base Value | Alternate Value | Justification |
|----------------------|---|----------------------------|--|
| Log fO_2 value | -0.7 | -0.818 (for J-13.3i files) | The difference is due to the number of significant figures copied over from previous input files, but is so minor that it will not impact the results. |
| N-Reactor Rate | 10× higher rate for higher temperatures | Used base-case rate. | The base-case rate is already high, so the difference is so minor, it will not impact the results. |
| Molar Volume of NiGd | 11.42 | 12.88 | The molar volume of Neutronit was used in the input file, but this value is only used to calculate the volume of material, which is not used in this report. |
| Soddyite | Not suppressed | Suppressed | This mineral was suppressed in the runs used to develop the ionic strength abstraction, but was not suppressed in the base-case runs. This difference is justified because the mineral did not form in the base-case runs, indicating that suppressing it has no impacts on the results. |

APPENDIX B
IN-PACKAGE RADIOLYSIS ANALYSES

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B. IN-PACKAGE RADIOLYSIS ANALYSES

B.1 RADIOLYSIS CONTRIBUTION TO IN-PACKAGE CHEMISTRY

Radiolysis is an important process that could influence the results of the in-package chemistry model. A nonmechanistic treatment in the form of a sensitivity analysis is provided here to examine how radiolysis could affect the pH and concentration of certain aqueous species. All of the pertinent input files may be found in DTN: MO0411SPAMOICA.000 [DIRS 172244], and the output files in DTN: MO0312SPAICSCR.002.

This appendix was not updated from Rev 03 of this report, so for these cases Cr(VI) was allowed to form instead of Cr(III). The impact of using Cr(III) versus Cr(VI) is examined in Section 6.6.3. The source for the input to this appendix is documented in Rev 03 of this report.

B.2 DATA AND PARAMETERS

- *Gamma and Neutron Radiolysis in the 21-PWR Waste Package from Ten to One Million Years* (BSC 2004 [DIRS 172017]). Nitric acid (HNO₃) production rate at 500 years (assuming waste package breach) is 3.60E+13 molecules/yr-cm³ (air) (BSC 2004 [DIRS 172017], Table 21, p. 33). This time was selected because it is the earliest breach, representing juvenile failure of the waste package. Figure 5 of this reference shows that the production is constant after 500 years, so the selection of this time is not critical.
- Geometry of CSNF waste package: Volume = 4,968,751 cm³, 10% fuel exposed surface area = 3,297,160 cm², void volume with 0.25-cm thick water layer = 4,144,461 cm³, from *CSNF.xls*, DTN: MO0411SPAMOICA.000 [DIRS 172244].
- Surface area normalization factor for 10% fuel exposure = 1.213E-03, from *CSNF.xls*, DTN: MO0411SPAMOICA.000 [DIRS 172244].
- G value for nitric acid production is 1.5 molecules/100 ev energy deposition (BSC 2004 [DIRS 172017], p. 27). The G value represents the number of molecules formed from radiolysis in a material and is linear with energy deposition. This number is used to calculate hydrogen peroxide (H₂O₂) generation by scaling (first bullet).
- Density of moist air (40% relative humidity, 30°C) is 1.16E-3 gm/cm³ (DTN: MO0312SPAICSCR.002, *csnf_radiolysis.xls*, Sheet MoistAir, Cell H14). This number is used to calculate hydrogen peroxide generation by scaling the first bullet, above.
- G value for hydrogen peroxide production is 0.72 molecules/100 ev energy deposition. This value is presented in *Waterside Corrosion of Zirconium Alloys in Nuclear Power Plants* (IAEA 1998 [DIRS 150560], Table 8.2, p. 214) and corroborated by Christensen (1995 [DIRS 161921], p. 375, Table II), Christensen et al. (1994 [DIRS 161919], p. 6, Table 1) and Christensen and Bjergbakke (1982 [DIRS 161918], Table 1, p.16). G value is 0.72 molecules/100 ev energy.

Other accepted engineering numbers:

- Avogadro constant = 6.02E+23 molecules/Mole
- Density of water at 27°C = 1 gm/cm³.

B.3 ASSUMPTIONS

The following assumptions are made for this report:

- Nitric acid production and hydrogen peroxide production is represented by the production rates at 500 years. This assumption is conservative because 500 years is the earliest waste package breach time because of early failures and radiolysis generally decreases with time. Figure 5 of *Gamma and Neutron Radiolysis in the 21-PWR Waste Package from Ten to One Million Years* (BSC 2004 [DIRS 172017]) shows that gamma dose and, therefore, most of the radiolysis is nearly constant after 500 years. This assumption needs no further validation.
- Nitric acid produced in the humid air by radiolysis is absorbed into the layers of water on the fuel and does not escape the waste package through the corrosion patches. This assumption is conservative because it is the upper limit of the amount of nitric acid that can be absorbed on the fuel. This assumption is necessary because modeling of chemical diffusion rates on this scale is not available. This assumption is conservative and needs no further validation.
- Radiolysis production in the outer eleven fuel assemblies of the 21-PWR design can be approximated by the radiolysis in the middle nine assemblies. *Gamma and Neutron Radiolysis in the 21-PWR Waste Package from Ten to One Million Years* (BSC 2004 [DIRS 172017]) provides analysis of the production in the center assembly of the waste package only. Radiation fluxes and, therefore, radiolysis production rates are less in the outer assemblies. This assumption is conservative and needs no further validation.
- Other than the generation of nitric acid and hydrogen peroxide, there are no other collateral chemical effects on the in-package environment. This assumption is justified because HNO₃ and H₂O₂ have the greatest potential to alter the in-package chemistry.

B.4 MODEL DISCUSSION

When radiation passes through a material, some energy is deposited in the medium and chemical reactions can occur from the local deposition of energy (radiolysis). The G value represents the number of molecules of a substance that is formed when 100 ev of energy is deposited in the media. When gamma and fast neutron energy pass through moist air, nitric acid is produced. *Gamma and Neutron Radiolysis in the 21-PWR Waste Package from Ten to One Million Years* (BSC 2004 [DIRS 172017]) predicts the amount of nitric acid that is produced per cm³ of humid air in the waste package (3.6E+13 molecules/yr-cm³ at 500 years). The amount of nitric acid formed in the waste package is calculated in EQ6 compatible input units via:

$$\text{HNO}_3 \text{ production rate} = 3.6\text{E}+13 \text{ molecules/yr-cm}^3 \times (\text{void volume waste package})$$

$$\begin{aligned}
&= 3.6\text{E}+13 \text{ molecules/yr-cm}^3 \times (4,144,461 \text{ cm}^3) \\
&= 1.49\text{E}+20 \text{ molecules/yr-waste package} \times (1 \text{ mole}/6.02\text{E}+23 \text{ molecules}) \\
&= 2.48\text{E}-04 \text{ mol/yr-waste package} \times (1.213\text{E}-03) \times (1 \text{ year}/3.16\text{E}+07 \text{ seconds}) \\
&= 9.53\text{E}-15 \text{ moles/second}
\end{aligned}$$

where 1.213E-03 is the normalization factor from *CSNF.xls*, DTN: MO0411SPAMOICA.000 [DIRS 172244].

When gamma and fast neutron energy pass through water, hydrogen peroxide is produced. The production rate for hydrogen peroxide (H₂O₂) in water is calculated by adjusting the nitric acid production rate by the ratio of G values and densities and converting to EQ6 compatible input units:

$$\begin{aligned}
\text{H}_2\text{O}_2 \text{ Production Rate} &= \text{PR}_{\text{HNO}_3} \times (\text{G}_{\text{H}_2\text{O}_2} / \text{G}_{\text{HNO}_3}) \times (\text{density water}/\text{density air}) \\
&= 3.6\text{E}+13 \text{ molecules/yr-cm}^3 \times (0.72/1.5) \times (1.0 \text{ gm/cm}^3 / 1.16\text{E}-3 \text{ gm/cm}^3) \\
&= 1.5\text{E}+16 \text{ molecules/yr-cm}^3 \times (4,144,461 \text{ cm}^3) \\
&= 6.22\text{E}+22 \text{ molecules/yr-waste package} \times (1 \text{ mol}/6.02\text{E}+23 \text{ molecules}) \\
&= 1.03\text{E}-01 \text{ mol/yr-waste package} \times (1.213\text{E}-03) \times (1 \text{ year}/3.16\text{E}+07 \text{ seconds}) \\
&= 3.97\text{E}-12 \text{ mol/second}
\end{aligned}$$

where 1.213E-03 is the normalization factor from *CSNF.xls*, DTN: MO0411SPAMOICA.000 [DIRS 172244].

B.5 EQ6 IMPLEMENTATION

The effect of radiolysis on the in-package chemistry was handled in a nonmechanistic manner (i.e., EQ6 does not have the facility to model the radiolysis process directly; therefore, only the products of radiolysis are included as inputs in EQ6 simulations). However, EQ6 does have the capability to model the effect of H₂O₂ and HNO₃ on the in-package chemistry.

A series of runs were performed where nitric acid and hydrogen peroxide were included as inputs in EQ6 runs and the results of these simulations were compared to their nonradiolysis counterparts.

Two base-case runs (C12C25 and C22C25, Section 6.3.1) were used to test the effects of radiolysis. These runs were chosen because they represent the median fuel exposure (clad failure) value and the low and median flux values. It would be expected that the effects of nitric acid and hydrogen peroxide additions would be greater at the low end of the flux range.

Two simulations were performed for each file, the first using the base-case nitric acid and hydrogen peroxide rates listed above, and a second where the HNO₃ and H₂O₂ rates were multiplied by a factor of ten. An additional run was completed using the C12C25 input file where the HNO₃ and H₂O₂ rates were multiplied by a factor of ten and the sulfur content of the Carbon Steel Type A516 waste package components was increased by a factor of 20. The EQ6 inputs are listed in Table B-1.

Table B-1. EQ6 File Names and Nitric Acid and Hydrogen Peroxide Production Rates

| EQ6 Input File | HNO ₃ Rate (mol/s) | H ₂ O ₂ Rate (mol/s) |
|----------------|-------------------------------|--|
| C22C25 | 0.0 | 0.0 |
| C22CBC | 9.53E-15 | 3.97E-12 |
| C22BC10 | 9.53E-14 | 3.97E-11 |
| C12C25 | 0.0 | 0.0 |
| C12CBC | 9.53E-15 | 3.97E-12 |
| C12BC10x | 9.53E-14 | 3.97E-11 |
| C12C10x_S20x | 9.53E-14 | 3.97E-11 |

B.6 RESULTS

The results of the simulations are displayed in Figure B-1, where the pH profiles for the various runs are displayed versus time. These results show that neither the base case nor the 10× base-case generation rates of HNO₃ and H₂O₂ had an impact on the in-package pH. Therefore, it may be concluded that if radiolysis only affects the chemistry via HNO₃ and H₂O₂ generation, then it will not be a significant process with regard to influencing the in-package chemistry.

The additional run (C12C10x_S20x) with twenty times the sulfur in the carbon steel drives the pH of the system to a low of about pH = 2.2 (Figure B-1).

Table B-2 provides the minimum pH, chloride, nitrate, hydrogen peroxide, and ferric iron maximum concentrations for the simulations. Again, the runs with HNO₃ and H₂O₂ input show little deviation compared to the runs without HNO₃ and H₂O₂.

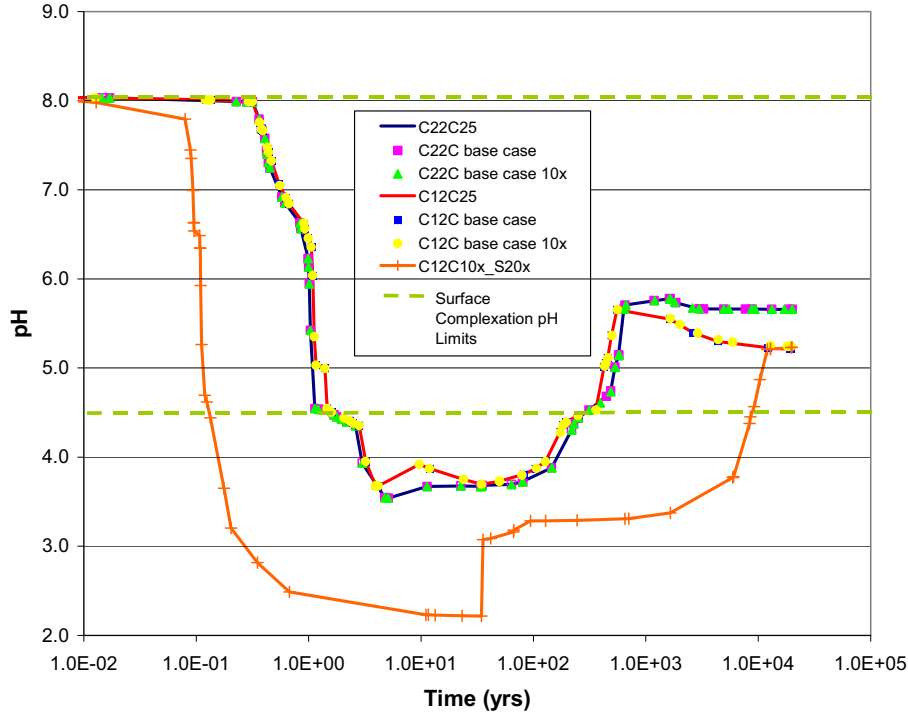
Table B-2. Chloride, Ferric Iron, and Hydrogen Peroxide Molality

| EQ6 Input File | Minimum pH | Cl ⁻ | NO ₃ ⁻ | H ₂ O ₂ (aq) | Fe ³⁺ |
|----------------|------------|-----------------|------------------------------|------------------------------------|------------------|
| C22C25 | 3.7 | 6.8E-04 | 8.5E-04 | 3.7E-19 | 5.1E-11 |
| C22CBC | 3.5 | 6.6E-04 | 1.5E-03 | 3.7E-19 | 8.5E-11 |
| C22BC10x | 3.5 | 6.5E-04 | 3.0E-03 | 3.7E-19 | 8.5E-11 |
| C12C25 | 3.7 | 9.7E-04 | 5.5E-03 | 3.7E-19 | 5.3E-11 |
| C12CBC | 3.7 | 9.6E-04 | 7.8E-03 | 3.7E-19 | 5.3E-11 |
| C12BC10x | 3.7 | 9.2E-04 | 1.9E-02 | 3.7E-19 | 5.4E-11 |
| C12C10x_S20x | 2.2 | 9.7E-04 | 1.6E-02 | 3.7E-19 | 3.8E-06 |

Source: Output DTN: MO0312SPAICSCR.002.

Based on this nonmechanistic approach to modeling the effects of radiolysis on in-package chemistry it may be stated that radiolysis will not affect the in-package chemistry in a manner

that would impact TSPA-LA. The results of these analyses are compiled in DTN: MO0312SPAICSCR.002. Surface complexation, which is not accounted for by this radiolysis analyses, would further moderate the pH of this system in a manner similar to that discussed in Section 6.8.



Source: *csnf_radiolysis_ICN01.xls*, Output DTN: MO0312SPAICSCR.002.

Figure B-1. pH Profiles Showing Radiolysis Impacts on pH



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APPENDIX C
DATA QUALIFICATION PLAN

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C. DATA QUALIFICATION PLAN

| | | |
|------------|--------------------------------|-------------|
| BSC | DATA QUALIFICATION PLAN | QA:QA |
| | | Page 1 of 1 |

| | | |
|--|--|-------------|
| Section I. Organizational Information | | |
| Qualification Title: | | |
| Qualification of Data that Supports In-Package Chemistry Abstraction (ANL-EBS-MD-000037, Rev. 03) | | |
| Requesting Organization: | | |
| Engineered Systems | | |
| Section II. Process Planning Requirements | | |
| <i>1. List of Unqualified Data to be evaluated:</i> | | |
| Sources of data that provide direct input to ANL-EBS-MD-000037 and that have not been qualified for all uses will be evaluated. The Document Input Reference System (DIRS) assigns to these sources the following numbers: 106164, 100051, 163015, 168168, and 105483. Authoritative sources will not be qualified. BSC/OCWRM documents will be justified within the text of the document. | | |
| <i>2. Type of Data Qualification Method(s) [Including rationale for selection of method(s) (Attachment 3) and qualification attributes (Attachment 4)]</i> | | |
| <p>The method of technical assessment was chosen because documentation or proof of proper information acquisition in unavailable for review for most data inputs</p> <p>The method of corroborating data was chosen for one of the methods of qualification because corroborating data are available for comparison and inferences can be clearly identified, justified, and documented</p> <p>The qualification attribute: extent to which the data demonstrate the properties of interest, and one or more of the following qualification attributes (taken from AP-SIII.2Q, Attachment 4) will be considered:</p> <ul style="list-style-type: none"> • Qualifications of personnel or organizations generating the information. • Prior peer or other professional reviews of the information and their results. • Extent and quality of corroborating information (for use in conjunction with technical assessment). | | |
| <i>3. Data Qualification Team and Additional Support Staff Required</i> | | |
| Emma Thomas – Chairman William Downs | | |
| <i>4. Data Evaluation Criteria</i> | | |
| <p>The following data evaluation criteria are based on and in this case identical to the attributes in block 2:</p> <ul style="list-style-type: none"> • The extent to which the data demonstrate the properties of interest. • Qualifications of personnel or organizations generating the information. • Prior peer or other professional reviews of the information and their results. • Extent and quality of corroborating information (for use in conjunction with technical assessment). | | |
| <i>5. Identification of Procedures Used</i> | | |
| AP-SIII.2Q, AP-SIII.9Q, and AP-3.15Q | | |
| Section III. Approval | | |
| <i>Qualification Chairperson Printed Name</i> | <i>Qualification Chairperson Signature</i> | <i>Date</i> |
| Emma Thomas |  | 9/17/04 |
| <i>Responsible Manager Printed Name</i> | <i>Qualification Manager Signature</i> | <i>Date</i> |
| Neil Brown |  | 9/17/04 |

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APPENDIX D
QUALIFICATION OF DATA THAT SUPPORTS IN-PACKAGE CHEMISTRY
ABSTRACTION

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D. QUALIFICATION OF DATA THAT SUPPORTS IN-PACKAGE CHEMISTRY ABSTRACTION

D.1 PURPOSE

This document relies on data that may not have been collected under an approved quality assurance program that meets the requirements of 10 CFR Part 63, Subpart G [DIRS 173273] or its predecessor, 10 CFR Part 60 (48 FR 28194 [DIRS 100475]). This appendix qualifies the data that this model report uses as direct input. The appendix was prepared according to the LP-SIII.2Q-BSC, *Qualification of Unqualified Data*. This qualification is being performed under the plan presented in Appendix C.

D.2 METHODS AND CRITERIA

LP-SIII.2Q-BSC offers five methods by which data may be qualified. As many of the methods were used as could be justified by the available supporting records. Technical assessment was considered as the strongest argument as documentation or proof of the quality and care of data acquisition is unavailable for review for most data inputs (e.g., laboratory quality plan unavailable for most older data collection activities). According to LP-SIII.2Q-BSC (Attachment 3), this confirmation comprises 1) “a discussion and justification that the data collection methodology used was appropriate for the type of data under consideration,” 2) “a discussion and justification that the data acquisition and/or subsequent data development discussed in source documentation was appropriate for the type of data under consideration,” or 3) “discussion and documentation that the data have been used in applications that are similar to those for which the data will be used.” Option 1 was used for the qualification process in this appendix.

In addition to corroborating data and technical assessment, the qualification process considers the extent to which the data demonstrate the properties of interest and at least one of the qualification process attributes of LP-SIII.2Q-BSC (Attachment 4).

- Qualifications of personnel or organizations generating the information
- Prior peer or other professional reviews of the information and their results
- Extent and quality of corroborating information (for use in conjunction with technical assessment).

These criteria also serve as the “data evaluation criteria” on which the data qualification team will provide judgments (LP-SIII.2Q-BSC, Section 5.1.2 (b)(4)).

The team comprises two members: Emma Thomas, model originator, chairs the team and William Downs, Ph.D., who is technically competent in data collection and qualification, assists. As the analysis originator, Emma Thomas is not independent of the information to be qualified (LP-SIII.2Q-BSC, Section 5.1.2 (b)(3)). However, the procedure requires that the originator chair the team “... when the qualification is performed within the Analysis or Model Report” (LP-SIII.2Q-BSC, Section 5.1.1). William Downs did not participate in the acquisition, but was a reviewer of the analysis.

D.3 QUALIFICATION OF DATA

LP-SIII.2Q-BSC does not apply to “established fact” or numerical data obtained from an established or authoritative data source. Because some of the data comes from authoritative sources (other than the U.S. Department of Energy, Office of Civilian Radioactive Waste Management), it does not need to be qualified. The remaining data are qualified only for its intended use.

Table D-1 lists the sources of data that are being qualified for their use in this report and designates the attributes or criteria that are used, in conjunction with the corroborating data and technical assessment, to qualify the information.

Table D-1. Sources of Data and Criteria Used to Qualify the Information

| Source | Criteria |
|--|--|
| Data Sources to be Qualified | |
| <i>Defense Waste Processing Facility Wasteform and Canister Description</i> (Baxter 1988 [DIRS 106164]) | Technical assessment/acceptable methodology Corroborating Data Personnel/organization qualifications Demonstrate properties of interest |
| <i>Aqueous Environmental Geochemistry</i> (Langmuir 1977 [DIRS 100051]) | Technical assessment/acceptable methodology Corroborating Data Personnel/organization qualifications Demonstrate properties of interest |
| “Critical Evaluation and Selection of Standard State Thermodynamic Properties for Chromium Metal and its Aqueous Ions, Hydrolysis Species, Oxides, and Hydroxides” (Ball and Nordstrom 1998 [DIRS 163015]) | Technical assessment/acceptable methodology Corroborating Data Personnel/organization qualifications Demonstrate properties of interest |
| “Surface Complexation of Ferrous Iron and Carbonate on Ferrihydrite and the Mobilization of Arsenic.” (Appelo et al. 2002 [DIRS 168168]) | Technical assessment/acceptable methodology Personnel/organization qualifications Demonstrate properties of interest |
| <i>Surface Complexation Modeling, Hydrous Ferric Oxide</i> (Dzombak and Morel 1990 [DIRS 105483]) | Technical assessment/acceptable methodology Personnel/organization qualifications Demonstrate properties of interest |

The discussions above and in the next section fulfill requirements of LP-SIII.2Q-BSC to discuss the data for qualification, the method of qualification, and the evaluation criteria (LP-SIII.2Q-BSC, Section 5.3.1, paragraphs (a)(1) through (3)).

A discussion of the remaining requirements follows:

- LP-SIII.2Q-BSC, Section 5.3.1 (a)(4)—the team considers the information technically correct because it has undergone a technical assessment and it fulfills at least one of the qualification process attributes of LP-SIII.2Q-BSC (Attachment 4).
- LP-SIII.2Q-BSC, Section 5.3.1 (a)(5)—no data are generated by this evaluation.

- LP-SIII.2Q-BSC, Section 5.3.1 (a)(6)—the team finds that the information is qualified for its use within *In-Package Chemistry Model Abstraction* ANL-EBS-MD-000037, Rev. 03.
- LP-SIII.2Q-BSC, Section 5.3.1 (a)(7)—the team recommends that the information be given the status of “qualified data.”
- LP-SIII.2Q-BSC, Section 5.3.1 (a)(8)—the team did not “abandon” any qualification method.
- LP-SIII.2Q-BSC, Section 5.3.1 (a)(9)—the data are qualified only for use in this document. It is not qualified for other potential users.
- LP-SIII.2Q-BSC, Section 5.3.1 (a)(10)—supporting or corroborating information is identified in Appendix C when used in the qualification effort.
- LP-SIII.2Q-BSC, Section 5.3.1 (a)(11)—the qualification was performed under the data qualification plan presented in Appendix C.

D.3.1 Qualification of Baxter, R.G. 1988 [DIRS 106164]

Technical Assessment/Acceptable Methodology: This report describes the physical and chemical stability characteristics of the defense high-level waste form borosilicate glass that was developed at the DOE Savannah River National Laboratory, Aiken, South Carolina. The value of interest for this report is the density of the waste glass at 25°C is 2.85 g/cm³. This value is the mean of several calculated densities based on percentages of radionuclide containing sludges and glass constituents.

Personnel/organization qualifications: The author and his organization are nationally recognized as the developers and testors of the borosilicate glass waste form. These data are comparable to “vendor data” for qualification purposes.

Corroborating Data: In order to check the validity of the density of the borosilicate glass waste form, the data were compared to similar data measured on other borosilicate glass samples by scientists at the Oak Ridge National Laboratory, Oak Ridge, Tennessee. The data agreed within 6% at 25°C (Notz et al. 1990 [DIRS 119707]).

Demonstrate properties of interest: The estimates were made for materials selected for use in the repository.

D.3.2 Qualification of Langmuir 1997 [DIRS 100051]

Technical Assessment/Acceptable Methodology: This publication is a textbook that contains empirical data for numerous thermodynamic and physicochemical parameters. The original source for these data are publications in peer-reviewed journals that have been used in numerous investigations and modeling efforts. These data have been included in virtually all of the equilibrium geochemistry simulation codes (e.g., PHREEQEC, MINTEQA2, etc.) since they were originally published and have been accepted by the scientific community.

Personnel/organization qualifications: The author is a Professor Emeritus at the Colorado School of Mines and has dozens of publications in the area of thermodynamics of dissolved species and mineral phases.

Corroborating Data: The table, from which the goethite surface data (sites per unit area and surface area per unit mass) were taken, contained data from several sources that demonstrated that the data selected were well within the range of measured data. In addition, similar data in agreement were calculated from basic crystal chemistry models and solvation theory.

Demonstrate properties of interest: The corrosion models for the iron bearing alloys used as either structural elements or waste casks have specified that the corrosion products include goethite – γ -FeOOH. This selection is considered conservative as the corrosion products include ferrihydrite (Fe[OH]₃), goethite (γ -FeOOH), and hematite (α -Fe₂O₃) as aging of the corrosion products proceeds with time. Ferrihydrite has a much larger surface area and would suggest adsorption of more elevated levels of species from solution but will dehydrate to goethite.

D.3.3 Qualification of Ball and Nordstrom 1998 [DIRS 163015]

Technical Assessment/Acceptable Methodology: This article comes from a peer-reviewed journal, *Journal of Chemical & Engineering Data*, that is a major source for thermodynamic data. The authors selected a method to achieve consistency and minimize error that include maintaining thermodynamic relationships, determine the most direct path for derived thermodynamic quantities, and compare derived thermodynamic quantities with empirical data where ever possible. For compounds and species where multiple measurements have been made, statistical analyses were conducted to determine the aleatory dispersion about the mean. The rigor of the methods used to develop these data is compatible with the requirements of the OCRWM quality program.

Personnel/organization qualifications: The authors of this article are nationally recognized specialists in the field of geochemistry and both of them have numerous publications in the field of geochemistry. The junior author, Dr. Kirk Nordstrom, is the coauthor (with J.L. Munoz) of *Geochemical Thermodynamics*, a text used by numerous universities for advanced courses in the field.

Corroborating Data: The article contains comparisons of the solubilities of key minerals calculated from the generated thermodynamic data with empirical data. In addition, the thermodynamic data for chromium species and compounds were compared and evaluated with data that had been generated and published by other researchers.

Demonstrate properties of interest: Because the various waste packages contain stainless steel as structural members and hexavalent chromium, Cr⁶⁺, represents a risk to human health and the environment, the solubility and sorption characteristics of chromium dissolved species and solids are necessary for assessing the performance of the repository.

D.3.4 Qualification of Appelo et al. 2002 [DIRS 168168]

Technical Assessment/Acceptable Methodology: This article comes from a peer-reviewed journal, *Environmental Science & Technology*, which is a major source for thermodynamic data.

The authors selected a method to achieve consistency and minimize error that include maintaining thermodynamic relationships, determine the most direct path for derived thermodynamic quantities, and compare derived thermodynamic quantities with empirical data where ever possible. The rigor of the methods used to develop these data is compatible with the requirements of the OCRWM Quality Program.

Personnel/organization qualifications: Dr. C A J Appelo received his PhD from the Vrije Universiteit, Amsterdam, NL, in 1978. He is a hydrochemical consultant providing consulting and educating for water resources, water quality and water pollution, and has presented short courses in hydrogeochemistry and hydrogeochemical transport modeling in Finland, France, the Netherlands, Portugal, South Africa, and Switzerland. He has over 35 publications in peer-reviewed technical journals and is an author of a widely used textbook on groundwater geochemistry and pollution and geochemical modeling (Appelo and Postma 1999 [DIRS 172038]). His collaboration with U. S. Geological Survey research scientists led to the development of the geochemical modeling code, PHREEQC, and its database.

Dr. Cristophe Tournassat recently received his PhD in environmental geochemistry from the University of Grenoble-I, FR. He is now a research scientist with the French Government environment organization BRGM in Orleans.

Dr. Laurent Charlet is a Full Professor in the Earth and Planetary Sciences Department at the University of Grenoble-I, FR. He received his PhD in 1986 from the University California at Riverside in Soil and Environmental Sciences, was a Postdoctoral Fellow at the Swiss Institute of Aquatic Sciences (EAWAG), and a Research Associate in the Department of Chemistry, University of Bern, Switzerland. His present research interests include mineral-water interface chemistry, atomic-scale environmental chemistry, biogeochemistry and the relationship between contamination and public health. He is author of more than 60 refereed publications in technical journals.

Professor Charlet's professional activity includes service in the Fundamental Environmental Chemistry section of the International Union of Pure and Applied Chemistry (IUPAC); membership in a review panel on nuclear waste confinement for the CNWRA, San Antonio; direction of the European Union (EU) project on "Mercury in French Guyana;" and service as associate editor of *Aquatic Chemistry* and *Advances in Environmental Science*, and as guest editor of *Geochimica et Cosmochimica Acta*.

Demonstrate properties of interest: The value given by Appelo et al. (2002 [DIRS 168168], Table 1, p. 3,097) is the value for the equilibrium constant for reactions given in Table 4-9. These are precisely the data that are required for parameterizing the surface complexation model. The surface complexation model (Section 6.7) is used to calculate the pH values fixed by surface complexation reactions on ferric oxyhydroxides (hydrous ferric oxides) formed within the waste package as a function of CO₂ fugacity ($f\text{CO}_2$). In its development, this model requires values for the equilibrium constants of several chemical reactions. Some of these reactions are between dissolved species or between gases and dissolved species. Equilibrium constants for these reactions are taken from the qualified thermodynamic database *data0.ymp.R4* (DTN: SN0410T0510404.002 [DIRS 172712]). Equilibrium constants for surface complexation reactions are also required for the surface complexation model but are not included in the

database *Data0.YMP.R4*. Appelo et al. (2002 [DIRS 168168], Table 1, p. 3,097) provide values for one of the constants needed.

D.3.5 Qualification of Dzombak and Morel 1990 [DIRS 105483]

Technical Assessment/Acceptable Methodology: This publication is a textbook that contains data for surface complexation parameters. This textbook is considered the handbook for this type of material by those in the field. The original source for these data are publications in peer-reviewed journals that have been used in numerous investigations and modeling efforts. These data have been included in virtually all of the equilibrium geochemistry simulation codes (e.g., PHREEQEC, MINTEQA2, etc.) since they were originally published and have been accepted by the scientific community.

Personnel/organization qualifications: Dr. David A Dzombak is professor in the Department of Civil and Environmental Engineering at Carnegie Mellon University, Pittsburgh, PA and a registered Professional Engineer in Pennsylvania. His research and professional interests include: aquatic chemistry, especially interactions of aqueous chemical species with mineral surfaces; fate and transport of chemicals in surface and subsurface waters; water and wastewater treatment; in situ and ex situ soil treatment; hazardous waste site remediation; abandoned mine drainage remediation; river and watershed restoration; and public communication of environmental science. In addition to this book containing the data to be qualified, he has published more than 50 articles in leading, peer-reviewed, environmental engineering and science journals and numerous book chapters.

Professor Dzombak's professional service activity has included service on the EPA Science Advisory Board (2002 to present); the Committee on Bioavailability of Contaminants in Soils and Sediments, National Research Council (2000 to 2002); editorial board of *Water Environment Research* (1993 to 1998) and *Ground Water* (1991 to 1993); Board of Directors and Officer (Treasurer) of the Association of Environmental Engineering and Science Professors (1996 to 1999); chair of committees for the American Academy of Environmental Engineers, American Society of Civil Engineers, and Water Environment Federation; and advisory committees for Allegheny County and the Commonwealth of Pennsylvania.

Dr. François M. M. Morel is the Albert G. Blanke Professor of Geosciences and Princeton University, Director, Princeton Environmental Institute and Director, Center for Environmental Bioinorganic Chemistry. His professional activities include teaching, as instructor of undergraduate and graduate subjects in aquatic chemistry element cycles, surface phenomena and biogeochemistry, and research as: Principal Investigator of projects on: (i) coordination chemistry, surface chemistry and photochemistry of trace elements in natural waters; (ii) interactions between the chemistry and microbiota in aquatic systems, (iii) trace metal nutrition and toxicity in phytoplankton; and (iv) environmental bioinorganic chemistry. He is author of four books, 14 book chapters, and more than 135 articles in peer-reviewed engineering and science journals.

Professor Morel's professional service activity has included service on the National Research Council Panel on Marine Mineral Technology (1976 to 1977), Committee on Ocean Waste Transportation (1982 to 1984) and Committee on Irrigation-Induced Water Quality Problems

(1985 to 1988). He has also served on the Advisory Board of *Environmental Science and Technology* (1978 to 1983) and as associate editor (1980), on the Editorial Board of *Limnology and Oceanography* (1980 to 1983) and as special editor (1990) and is on the editorial board of *Biogeochemistry*, *Geobiology*, *Aquatic Sciences* and *Environmental Microbiology*. He serves Institut National de l'Environnement Industriel et des Risques, as Membre du Conseil Scientifique (1991 to present), and Nancie Centre International De L'eau as membre du Conseil d'Orientation (1995 to present).

Since its publication in 1990, this book by Dzombak and Morel has become a standard source for equilibrium constants surface complexation reactions. Data from it are cited in widely used textbooks on aqueous geochemistry including those by Langmuir (1997 [DIRS 100051]) and Stumm and Morgan (1996 [DIRS 125332]). These data are also included in the databases accompanying the broadly distributed, U. S. Geological Survey geochemical modeling code PHREEQC (BSC 2001. [DIRS 154877]); Parkhurst and Appelo (1999 [DIRS 159511]) and the widely used U.S. Environmental Protection Agency geochemical assessment model MINTEQA2 (Allison et al. 1991 [DIRS 171707]); HydroGeoLogic; Allison Geoscience Consultants (1999 [DIRS 158974]).

Demonstrate properties of interest: Values provided by Dzombak and Morel (1990 [DIRS 105483], Table 5.7) are for the equilibrium constants for reactions in Table 4-9. These are precisely the data that are required for parameterizing the surface complexation model. The surface complexation model (Section 6.7) is used to calculate the pH values fixed by surface complexation reactions on ferric oxyhydroxides (hydrated ferric oxides) formed within the waste package as a function of CO₂ fugacity (f_{CO_2}). In its development, this model requires values for the equilibrium constants of several chemical reactions. Some of these reactions are between dissolved species or between gases and dissolved species. Equilibrium constants for these reactions are taken from the qualified thermodynamic database *data0.ymp.R4* (DTN: SN0410T0510404.002 [DIRS 172712]). Equilibrium constants for surface complexation reactions are also required for the surface complexation model but are not included in database *Data0.YMP.R4*. Dzombak and Morel (1990 [DIRS 105483], Table 5.7) provide values for two of the needed constants.

D.4 CONCLUSION

In conclusion, all of the data from the articles or books listed above are qualified for their intended use in this model report.

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APPENDIX E
EVALUATION OF IN-PACKAGE CHEMISTRY SURFACE COMPLEXATION
MODEL

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E. EVALUATION OF IN-PACKAGE CHEMISTRY SURFACE COMPLEXATION MODEL

The following evaluation was performed by Yifeng Wang, Principal Member of Technical Staff, Sandia National Laboratories, a recognized expert in the field of geochemistry, with a Ph.D. from the Department of Geological Sciences, Indiana University. This review was conducted on 7/23/2004 in accordance with the requirements of *Technical Work Plan for Postclosure Waste Form Modeling* (BSC 2005 [DIRS 173246]). In his report on his evaluation, Dr. Wang raised several questions. The report and the responses by Patrick Brady to the raised questions are provided below:

The in-package chemistry model is developed to predict the chemistry (e.g., pH and ionic strength) of inside a breached waste package. The surface complexation model is included as a part of model to provide realistic estimates of in-package pH values by accounting for the effect of surface protonation and deprotonation on solution pH. This review is focused only on the surface complexation model.

The corrosion of Carbon Steel Type A516 and the oxidation of elemental sulfur contained in the steel will generate a certain quantity of H₂SO₄ acid that could potentially lead to a low in-package pH. In reality, this low pH is likely to be buffered by surface protonation on corrosion products, given the factors that only a trace amount of elemental sulfur (1000 times less than Fe on a molal basis) is contained in Carbon Steel Type A516 and that Fe oxyhydroxides as a corrosion product are practically insoluble and have a high surface adsorption capability with the point of zero charge close to neutral pH. Therefore, the general concept of using a surface complexation mechanism to constrain a more realistic in-package pH is scientifically sound.

It is generally recognized that there are a large uncertainties associated with the equilibrium constants of surface complexation reactions, which are usually model-dependent. However, the equilibrium constants used in the in-package chemistry model seem reasonable. First of all, the surface complexation constants are relatively well established for Fe oxides as compared to other minerals. Second, the actual values used in the model are from a carefully compiled and widely accepted data sources (e.g., Dzombak and Morel 1990 [DIRS 105483]).

A simplified version of surface complexation model, described by an analytic equation, is used in the package chemistry. I have checked the derivation of the equation, and no error has been found. The prediction of the equation has been confirmed by PHREEQC calculations.

In general, the surface complexation model developed for in-package chemistry is adequate for its intended use (i.e., to provide a more realistic prediction of in-package pH).

Specific comments:

- In the model abstraction, the in-package chemistry model implies a well-mixed cell for a whole waste package. This assumption is not consistent with the EBS transport model, which assumes two cells in a waste package. Steel corrosion is assumed to be present only in cell 1. It seems to me that the surface complexation model should be applied only to cell 1.

Response—Spent fuel will reside inside of a matrix of steel and corrosion products. In the EBS transport model the fuel is referred to as domain 1. Steel and its corrosion products are referred to as domain 2. Water that moves into domain 1 must move first through domain 2 where the corrosion products reside and where the SCM is used to calculate the pH. Water carrying fuel components out of domain 1 must move through domain 2 as well. It is, therefore, reasonable to apply the SCM in domains 2 and 1.

- The technical basis for selection of goethite as a representative corrosion product is not fully justified.

Response—Corrosion product formation is complex and depends upon time, temperature, relative humidity, salt identity and concentrations and so on. It is impossible to fully justify the selection of a single corrosion product such as goethite. For this reason, goethite – a corrosion product that is commonly observed, with mid-range site density and surface areas – was chosen to represent what in all likelihood will be an evolving collection of iron corrosion products. For the purposes of this calculation, the technical basis is believed sufficient.

- Low pH could be also caused by the release of Cr(VI) from Stainless Steel Type 304L. But I am not sure the same surface complexation model can apply to this condition.

Response—Stainless Steel Type 304L can, in theory, cause low pH values. Because the molal dissolution rate of Stainless Steel Type 304L is quite low, the resulting proton production occurs over long time spans and will tend to be diluted by the steady influx of seepage or condensate.

- There are other uncertainties associated with the surface complexation model. The temperature effect on surface complexation has not been well established in general.

Response—Yes, the temperature effect on surface complexation remains an area of active study. There is no evidence in the evolving subject literature that temperature effects on surface complexation will lead to materially different pH trajectories though. The competition of other ions on surface protonation and deprotonation could affect on the predicted pH, although the PHREEQC calculations have shown such an effect may be not significant. Competition could affect the pH, but again the PHREEQC provides sufficient confidence that the large-scale features are captured in the analytical mode.

APPENDIX F
CD-ROM

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APPENDIX G
LISTING OF FILES ON COMPACT DISC

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G. LISTING OF FILES ON COMPACT DISC

This attachment contains the name and size of the zip file placed on the electronic media (Attachment I). Winzip 7.0 was used to compress the files.

Volume in drive D is 050707_1657
Volume Serial Number is E32F-2BC4

Directory of D:\

| | | |
|-------------------|-------------|----------------------------------|
| 07/07/2005 08:51a | 1,313,984 | Alternative Conceptual Model.zip |
| 07/07/2005 09:09a | 296,094,110 | EQ6 Runs.zip |
| 07/07/2005 08:51a | 2,648,657 | SCM.zip |
| 07/07/2005 04:19p | 7,720,991 | Spreadsheets.zip |
| 4 File(s) | 307,777,742 | bytes |

Total Files Listed:
4 File(s) 307,777,742 bytes
0 Dir(s) 0 bytes free

The zip files contain files of various types:

Excel files (extensions = xls).

Mathcad files (extensions = mcd).

EQ3/6 input files (extension = 3i or 6i).

ASCII text file: provides input parameters for EQ3/6.

EQ3/6 output files (extension = 3o or 6o).

ASCII text file: provides detailed information about the system at each print point, which is specified by the user in the input file.

EQ3/6 pickup files (extension = 3p or 6p).

ASCII text file: provides a description of the system at the end of that run to be used as an input file for a continuation run.

EQ6 Tab-delimited text files (extension = txt).

*.elem_aqu: total aqueous moles of elements.

*.elem_min: total moles of elements in minerals.

*.elem_tot: total moles of elements (aqueous + mineral).

*.min_info: moles of each mineral.

EQ6 binary output file (extension = bin).

Binary file: provides detailed information about the system at the full numerical precision for *every* time step.

EQPT input files (*data0.cr3* and *data0.ert*)

EQPT output files (*data1.cr3* and *data.ert*)

transl input file (*data0.ymp.R2*)

transl output file (extension = dat)

PHREEQC input file (no extension)

PHREEQC output file (extension = out)

Batch files (extension = bat) used to start EQ6 runs.

Winzip files (extension = zip).

The following figure illustrates the directory structure within the zip files listed above.

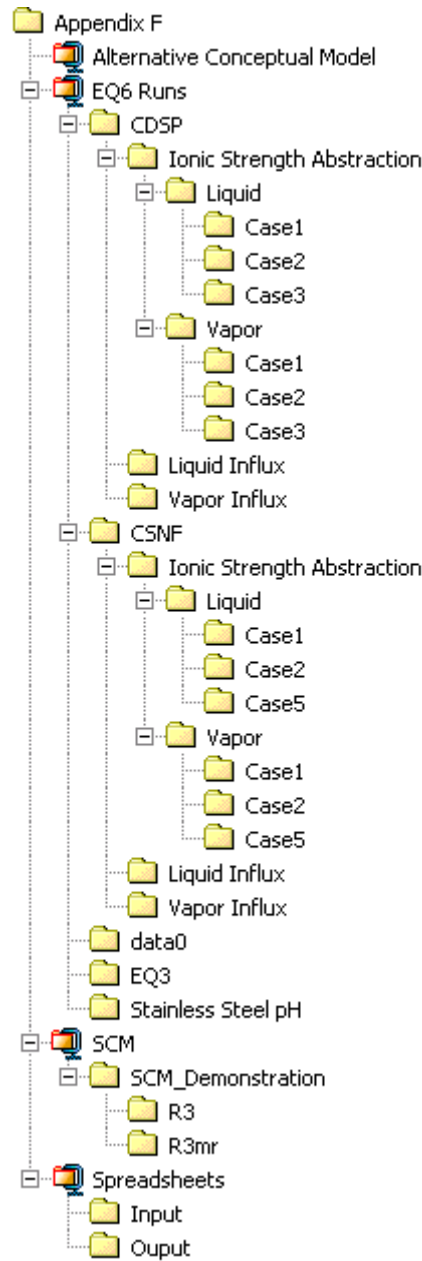


Figure G-1. Directory Structures for Appendix F Zip Files

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APPENDIX H
WATER-VAPOR FLUX INTO THE WASTE PACKAGE BASED ON
FICK'S FIRST LAW AND STEFAN TUBE EQUATIONS

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H. WATER-VAPOR FLUX INTO THE WASTE PACKAGE BASED ON FICK'S FIRST LAW AND STEFAN TUBE EQUATIONS

H.1 WATER VAPOR DIFFUSION FLUX FICK'S FIRST LAW CALCULATION

This sheet calculates the water-vapor flux (q) into the waste package through the stress corrosion cracks in the waste package outer lid based on Fick's First Law. The technical basis for this calculation was taken from *EBS Radionuclide Transport Abstraction* (BSC 2004 [DIRS 169868], Section 6.6.2). The flux is calculated over a temperature range (specified by the index, i) for a specified relative humidity. Fick's First Law is based on the assumption that the calculated flux is due to a concentration gradient, and that the change in the gradient is constant with regard to distance between the source and sink. In the case presented, it is also assumed that the relative humidity is zero inside the waste package. The justification of this assumption is that water vapor will be consumed in the formation of corrosion products and secondary minerals. In addition, this assumption is conservative as it maximizes the vapor diffusion flux into the waste package. Fick's First Law does not evaluate the contribution of the advective (convective) component to flux.

The mathematical expression of Fick's First Law is (BSC 2004 [DIRS 169868], Equation 6.6.2-8):

$$q = D_{AB} A_{SCC} \frac{\partial C_{wv}}{\partial x}$$

(Eq. H.1-1)

where q = flux, D_{AB} = binary diffusion coefficient, A_{SCC} = cross-sectional area of the stress corrosion cracks through which water vapor can diffuse, ∂C_{wv} = the change in water vapor concentration from the source to the sink, and ∂x is the distance of the travel path (thickness of the waste package outer lid = 2.54 cm).

This appendix was prepared using Mathcad, Version 11 (Table 3-1). This sheet calculates the flux ($qTemp_i$) in an iterative manner for the temperature range 30°C to 96°C, for a selected value of relative humidity (RH). The temperature range from 30°C to 96°C was selected as 30°C is approximately room temperature, and 96°C is when water will stop boiling at the site elevation.

In Equation H.1-2, adapted from *EBS Radionuclide Transport Abstraction* (BSC 2004 [DIRS 169868], Equation 6.2.2-2), the vector $i = 1 .. 67$ defines the range of the iteration, $Temp_i$ sums 29 with the vector i to define the temperature range 30°C to 96°C, and $pwfTemp$ defines a function that calculates the saturated partial pressure of water vapor over the temperature range 30°C to 96°C. Equations for $pwfTemp$ are taken from the report by Singh et al. (2002 [DIRS 161624], Equations 1a and 1b).

$$i := 1..67$$

$$Temp_i := 29 + i$$

$$pwfTemp(Temp) := \begin{cases} 610.78 \exp \left[\frac{17.269(Temp)}{237.3 + (Temp)} \right] & \text{if } (0 \leq Temp) \cdot (Temp < 63) \\ 610.78 \exp \left[\frac{17.269(Temp)}{236.3 + 1.01585(Temp)} \right] & \text{if } (63 \leq Temp) \cdot (Temp < 110) \end{cases}$$

(Eq. H.1-2)

Note: The symbol := denotes a definition in Mathcad.

From the saturated partial pressure of water vapor ($pwfTemp$) the partial pressure of water dependent on the relative humidity (RH) can be determined (BSC 2004 [DIRS 161624], Equation 6.6.2-1):

$$pwTemp_i := pwfTemp(Temp_i)$$

$$RH := 0.95$$

$$pTemp_i := (pwTemp_i) \cdot RH$$

(Eq. H.1-3)

where $pwTemp_i$ is vapor pressure of water calculated in the function $pwfTemp$ for the temperature range 30 to 96°C. RH is the relative humidity. By changing the value for RH the flux is calculated over the range of temperatures specified by the vector i . $pTemp_i$ is the partial pressure of water dependent on the relative humidity.

Next, the molal humidity is calculated. In this example it is calculated for 50°C and 95 percent relative humidity (BSC 2004 [DIRS 161624], Equation 6.6.2-4):

$$HmTemp_i := \frac{pTemp_i}{101325 - pTemp_i}$$

(Eq. H.1-4)

where $HmTemp_i$ is the molal humidity (mol H₂O/mol wet air). Pressure for dry air is 1 atm = 101,325 Pa, from equation H.1-2 at 50°C the partial pressure of water (p_w) was calculated $p_w = 12,344$ Pa. So the partial pressure at $RH = 0.95$ is $p_w = 11,717$ Pa. Then the molal humidity of water vapor in wet air (molar fraction of water vapor in wet air) can be calculated by dividing 11,717 Pa/101,325 Pa = 0.1156 mol H₂O mol⁻¹ wet air.

The concentration of water vapor at 50°C and 95 percent relative humidity can then be calculated (BSC 2004 [DIRS 161624], Equation 6.6.2-5):

$$C_{wv}Temp_i := \frac{HmTemp_i}{22414 \cdot \left(\frac{273.15 + Temp_i}{273.15} \right)}$$

(Eq. H.1-5)

where $C_{wv}Temp_i$ is the concentration of water vapor in the air mol H₂O/cm³.

To calculate the binary diffusion coefficient the following equation is used (Bird et al. 1960 [DIRS 103524] Equation 16.3-1; BSC 2004 [DIRS 161624], Equations 6.6.2-6 and 6.6.2-7):

$$DAB_{Temp} := 3.64 \cdot 10^{-4} \cdot \left(\frac{273.15 + Temp_i}{\sqrt{647.25 \cdot 132}} \right)^{2.334} \cdot (218.3 \cdot 36.4)^{\frac{1}{3}} \cdot (647.25 \cdot 132)^{\frac{5}{12}} \cdot \left(\frac{1}{18.01528} + \frac{1}{28.964} \right)^{\frac{1}{2}} \cdot (1)^{-1}$$

(Eq. H.1-6)

DAB is the binary diffusion coefficient (cm² s⁻¹), a = 3.64 × 10⁻⁴ for H₂O, Temp_i = temperature (C) converted into absolute temperature (K), TcA = 647.25 K, TcB = 132 K, pcA = 218.3 atm, pcB = 36.4 atm, MA = 18.01528 g/mol, MB = 28.964 g/mol, and p-1 = 1 atm.

Then the water-vapor flux (qTemp) that enters through the aperture of the stress corrosion cracks can be calculated (BSC 2004 [DIRS 161624], Equation 6.6.2-8):

$$q_{Temp} := \frac{DAB_{Temp} \cdot (1.93) \cdot C_{wvTemp}}{2.54}$$

(Eq. H.1-7)

The value 1.93 cm² is the stress corrosion crack cumulative aperture as it is expected that there will be 25 stress corrosion cracks (BSC 2004 [DIRS 169996]) and each will have an aperture of 7.7 × 10⁻⁶ m²/scc (BSC 2004 [DIRS 169868] Section 6.3.3.1.2.1). Therefore, to calculate the cumulative aperture (7.7 * 10⁻⁶ m² / scc) * (25 scc) * (100 cm)² / m² = 1.93 cm². The waste package outer lid (BSC 2004 [DIRS 167394] Detail A) thickness is 2.54 cm.

This calculates the vapor flux per second that enters the waste package. However, the vapor flux per year into the package is the value of interest. The following converts the flux from mol/sec to mol/year:

$$Q_{Temp} := q_{Temp} \cdot 60 \cdot 60 \cdot 24 \cdot 365.25$$

(Eq. H.1-8)

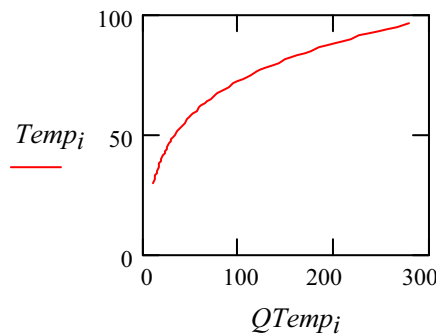


Figure H.1-1. Water Vapor Flux (QTemp_i) That Enters the Waste Package, Calculated Based on Fick's First Law and for 95 Percent Relative Humidity

Temp_i =

| |
|----|
| 30 |
| 31 |
| 32 |
| 33 |
| 34 |
| 35 |

$pwTemp_i =$

| |
|--------------------|
| $4.242 \cdot 10^3$ |
| $4.492 \cdot 10^3$ |
| $4.754 \cdot 10^3$ |
| $5.029 \cdot 10^3$ |
| $5.318 \cdot 10^3$ |
| $5.622 \cdot 10^3$ |

$QTemp_i =$

| |
|-------|
| 10.75 |
| 11.46 |
| 12.22 |
| 13.01 |
| 13.86 |
| 14.76 |

Table H.1-1. Water Vapor Flux (mol/yr) Calculated from Fick's First Law

| Relative Humidity | 0.30 | 0.40 | 0.50 | 0.60 | 0.70 | 0.80 | 0.90 | 0.95 | 1.00 |
|-------------------|--------------|-------|-------|-------|-------|-------|-------|-------|-------|
| Temp. (°C) | Flux(mol/yr) | | | | | | | | |
| 30 | 3.26 | 4.35 | 5.43 | 6.52 | 7.61 | 8.69 | 9.78 | 10.32 | 10.87 |
| 31 | 3.47 | 4.62 | 5.78 | 6.93 | 8.09 | 9.25 | 10.40 | 10.98 | 11.56 |
| 32 | 3.69 | 4.91 | 6.14 | 7.37 | 8.60 | 9.83 | 11.06 | 11.67 | 12.29 |
| 33 | 3.92 | 5.22 | 6.53 | 7.83 | 9.14 | 10.44 | 11.75 | 12.40 | 13.05 |
| 34 | 4.16 | 5.55 | 6.93 | 8.32 | 9.71 | 11.09 | 12.48 | 13.17 | 13.86 |
| 35 | 4.42 | 5.89 | 7.36 | 8.83 | 10.30 | 11.77 | 13.25 | 13.98 | 14.72 |
| 36 | 4.69 | 6.25 | 7.81 | 9.37 | 10.93 | 12.49 | 14.06 | 14.84 | 15.62 |
| 37 | 4.97 | 6.63 | 8.28 | 9.94 | 11.60 | 13.25 | 14.91 | 15.74 | 16.57 |
| 38 | 5.27 | 7.03 | 8.78 | 10.54 | 12.30 | 14.05 | 15.81 | 16.69 | 17.57 |
| 39 | 5.59 | 7.45 | 9.31 | 11.17 | 13.03 | 14.89 | 16.76 | 17.69 | 18.62 |
| 40 | 5.92 | 7.89 | 9.86 | 11.84 | 13.81 | 15.78 | 17.75 | 18.74 | 19.73 |
| 41 | 6.27 | 8.36 | 10.44 | 12.53 | 14.62 | 16.71 | 18.80 | 19.85 | 20.89 |
| 42 | 6.63 | 8.85 | 11.06 | 13.27 | 15.48 | 17.69 | 19.90 | 21.01 | 22.11 |
| 43 | 7.02 | 9.36 | 11.70 | 14.04 | 16.38 | 18.72 | 21.06 | 22.23 | 23.40 |
| 44 | 7.43 | 9.90 | 12.38 | 14.85 | 17.33 | 19.80 | 22.28 | 23.52 | 24.75 |
| 45 | 7.85 | 10.47 | 13.09 | 15.70 | 18.32 | 20.94 | 23.56 | 24.87 | 26.17 |
| 46 | 8.30 | 11.07 | 13.83 | 16.60 | 19.37 | 22.13 | 24.90 | 26.28 | 27.67 |
| 47 | 8.77 | 11.69 | 14.62 | 17.54 | 20.46 | 23.39 | 26.31 | 27.77 | 29.23 |
| 48 | 9.26 | 12.35 | 15.44 | 18.52 | 21.61 | 24.70 | 27.79 | 29.33 | 30.87 |
| 49 | 9.78 | 13.04 | 16.30 | 19.56 | 22.82 | 26.08 | 29.34 | 30.97 | 32.60 |
| 50 | 10.32 | 13.76 | 17.20 | 20.64 | 24.08 | 27.52 | 30.96 | 32.69 | 34.41 |
| 51 | 10.89 | 14.52 | 18.15 | 21.78 | 25.41 | 29.04 | 32.67 | 34.48 | 36.30 |
| 52 | 11.49 | 15.31 | 19.14 | 22.97 | 26.80 | 30.63 | 34.46 | 36.37 | 38.28 |
| 53 | 12.11 | 16.15 | 20.18 | 24.22 | 28.25 | 32.29 | 36.33 | 38.35 | 40.36 |
| 54 | 12.76 | 17.02 | 21.27 | 25.53 | 29.78 | 34.03 | 38.29 | 40.41 | 42.54 |
| 55 | 13.45 | 17.93 | 22.41 | 26.89 | 31.38 | 35.86 | 40.34 | 42.58 | 44.82 |
| 56 | 14.16 | 18.88 | 23.60 | 28.32 | 33.05 | 37.77 | 42.49 | 44.85 | 47.21 |
| 57 | 14.91 | 19.88 | 24.85 | 29.82 | 34.79 | 39.76 | 44.73 | 47.22 | 49.70 |
| 58 | 15.69 | 20.93 | 26.16 | 31.39 | 36.62 | 41.85 | 47.08 | 49.70 | 52.31 |
| 59 | 16.51 | 22.02 | 27.52 | 33.03 | 38.53 | 44.03 | 49.54 | 52.29 | 55.04 |
| 60 | 17.37 | 23.16 | 28.95 | 34.74 | 40.53 | 46.32 | 52.11 | 55.00 | 57.90 |
| 61 | 18.26 | 24.35 | 30.44 | 36.53 | 42.61 | 48.70 | 54.79 | 57.83 | 60.88 |

Table H.1-1. Water Vapor Flux (mol/yr) Calculated from Fick's First Law (Continued)

| Relative Humidity | 0.30 | 0.40 | 0.50 | 0.60 | 0.70 | 0.80 | 0.90 | 0.95 | 1.00 |
|-------------------|--------------|--------|--------|--------|--------|--------|--------|--------|--------|
| Temp. (°C) | Flux(mol/yr) | | | | | | | | |
| 62 | 19.20 | 25.60 | 32.00 | 38.39 | 44.79 | 51.19 | 57.59 | 60.79 | 63.99 |
| 63 | 20.17 | 26.90 | 33.62 | 40.35 | 47.07 | 53.79 | 60.52 | 63.88 | 67.24 |
| 64 | 21.19 | 28.25 | 35.31 | 42.37 | 49.44 | 56.50 | 63.56 | 67.09 | 70.62 |
| 65 | 22.25 | 29.66 | 37.08 | 44.49 | 51.91 | 59.32 | 66.74 | 70.44 | 74.15 |
| 66 | 23.35 | 31.13 | 38.92 | 46.70 | 54.48 | 62.27 | 70.05 | 73.94 | 77.83 |
| 67 | 24.50 | 32.67 | 40.83 | 49.00 | 57.17 | 65.34 | 73.50 | 77.59 | 81.67 |
| 68 | 25.70 | 34.27 | 42.84 | 51.40 | 59.97 | 68.54 | 77.10 | 81.39 | 85.67 |
| 69 | 26.95 | 35.94 | 44.92 | 53.90 | 62.89 | 71.87 | 80.86 | 85.35 | 89.84 |
| 70 | 28.25 | 37.67 | 47.09 | 56.51 | 65.93 | 75.35 | 84.76 | 89.47 | 94.18 |
| 71 | 29.61 | 39.48 | 49.35 | 59.22 | 69.10 | 78.97 | 88.84 | 93.77 | 98.71 |
| 72 | 31.03 | 41.37 | 51.71 | 62.05 | 72.39 | 82.74 | 93.08 | 98.25 | 103.40 |
| 73 | 32.50 | 43.33 | 54.16 | 64.99 | 75.83 | 86.66 | 97.49 | 102.90 | 108.30 |
| 74 | 34.03 | 45.37 | 56.71 | 68.06 | 79.40 | 90.74 | 102.10 | 107.80 | 113.40 |
| 75 | 35.62 | 47.50 | 59.37 | 71.24 | 83.12 | 94.99 | 106.90 | 112.80 | 118.70 |
| 76 | 37.28 | 49.71 | 62.13 | 74.56 | 86.99 | 99.41 | 111.80 | 118.10 | 124.30 |
| 77 | 39.00 | 52.00 | 65.01 | 78.01 | 91.01 | 104.00 | 117.00 | 123.50 | 130.00 |
| 78 | 40.80 | 54.39 | 67.99 | 81.59 | 95.19 | 108.80 | 122.40 | 129.20 | 136.00 |
| 79 | 42.66 | 56.88 | 71.10 | 85.32 | 99.54 | 113.80 | 128.00 | 135.10 | 142.20 |
| 80 | 44.59 | 59.46 | 74.32 | 89.19 | 104.10 | 118.90 | 133.80 | 141.20 | 148.60 |
| 81 | 46.60 | 62.14 | 77.67 | 93.21 | 108.70 | 124.30 | 139.80 | 147.60 | 155.30 |
| 82 | 48.69 | 64.92 | 81.15 | 97.39 | 113.60 | 129.80 | 146.10 | 154.20 | 162.30 |
| 83 | 50.86 | 67.82 | 84.77 | 101.70 | 118.70 | 135.60 | 152.60 | 161.10 | 169.50 |
| 84 | 53.11 | 70.82 | 88.52 | 106.20 | 123.90 | 141.60 | 159.30 | 168.20 | 177.00 |
| 85 | 55.45 | 73.93 | 92.42 | 110.90 | 129.40 | 147.90 | 166.30 | 175.60 | 184.80 |
| 86 | 57.87 | 77.17 | 96.46 | 115.70 | 135.00 | 154.30 | 173.60 | 183.30 | 192.90 |
| 87 | 60.39 | 80.52 | 100.60 | 120.80 | 140.90 | 161.00 | 181.20 | 191.20 | 201.30 |
| 88 | 63.00 | 84.00 | 105.00 | 126.00 | 147.00 | 168.00 | 189.00 | 199.50 | 210.00 |
| 89 | 65.70 | 87.60 | 109.50 | 131.40 | 153.30 | 175.20 | 197.10 | 208.10 | 219.00 |
| 90 | 68.51 | 91.34 | 114.20 | 137.00 | 159.80 | 182.70 | 205.50 | 216.90 | 228.40 |
| 91 | 71.41 | 95.22 | 119.00 | 142.80 | 166.60 | 190.40 | 214.20 | 226.10 | 238.00 |
| 92 | 74.42 | 99.23 | 124.00 | 148.80 | 173.70 | 198.50 | 223.30 | 235.70 | 248.10 |
| 93 | 77.54 | 103.40 | 129.20 | 155.10 | 180.90 | 206.80 | 232.60 | 245.60 | 258.50 |
| 94 | 80.77 | 107.70 | 134.60 | 161.50 | 188.50 | 215.40 | 242.30 | 255.80 | 269.20 |
| 95 | 84.12 | 112.20 | 140.20 | 168.20 | 196.30 | 224.30 | 252.40 | 266.40 | 280.40 |
| 96 | 87.58 | 116.80 | 146.00 | 175.20 | 204.40 | 233.60 | 262.70 | 277.30 | 291.90 |

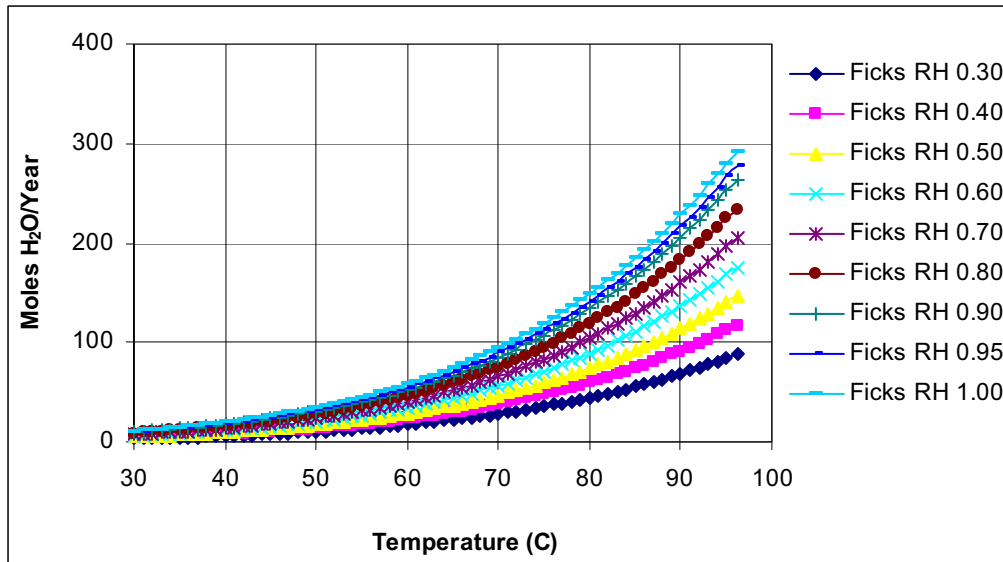


Figure H.1-2. Water Vapor Flux Calculated Based on Fick's First Law

H.2 WATER VAPOR DIFFUSION FLUX STEFAN TUBE ANALYTIC SOLUTION

The purpose of this sheet is to calculation of the water vapor flux into the waste package through the Stress Corrosion Cracks on the outer lid based on the Stefan Tube equations. In contrast to Fick's First Law that calculates the flux solely due to diffusion a Stefan Tube calculation determines flux based on diffusion and advection (Cutlip and Shacham 2000 [DIRS 163339], Equation 7-3).

$$N_A = D_{AB} C \frac{dx_A}{dz} + \frac{C_A}{C} (N_A + N_B) \quad (\text{Eq. H.2-1})$$

flux = diffusion + bulk flow (convection)

where D_{AB} is the molecular diffusivity of A in B (cm^2/s), C is the total concentration (mole/cm^3), C_A is the concentration of A (mole/m^3), N_A is the flux of A relative to stationary coordinates ($\text{mole}/\text{m}^2 \cdot \text{s}$) and N_B is the flux of B ($\text{mole}/\text{m}^2 \cdot \text{s}$). For this problem, the total gas concentration C is constant and component B is stagnant. Thus N_B is zero. The mole fraction of A in the gas mixture, x_A , can be used to replace C_A/C . The modified expression for this equation can be written as (Cutlip and Shacham 2000 [DIRS 163339], Equation 7-4):

$$N_A = -D_{AB} \frac{dC_A}{dz} + x_A N_A \quad (\text{Eq. H.2-2})$$

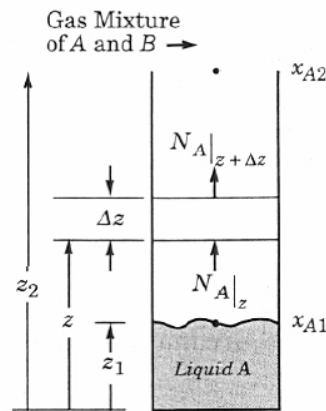
Solving for N_A and rearranging Equation H.2-2 yields (Cutlip and Shacham 2000 [DIRS 163339], Equation 7-5):

$$\frac{dx_A}{dz} = \frac{(1-x_A)N_A}{D_{AB}C} \quad (\text{Eq. H.2-3})$$

At constant temperature and pressure for ideal gases, the total concentration C and binary diffusivity D_{AB} may be considered constant. Based on this information an analytical solution can be derived and is provided below (Cutlip and Shacham 2000 [DIRS 163339], Equations 7-7 and 7-8):

$$N_{Az|z=z_1} = \frac{D_{AB}C}{(z_2 - z_1)(X_B)_{lm}} (X_{A1} - X_{A2}) \quad (\text{Eq. H.2-4})$$

$$(X_B)_{lm} = \frac{(X_{A1} - X_{A2})}{\ln[(1 - X_{A2}) / (1 - X_{A1})]} \quad (\text{Eq. H.2-5})$$



Source: Cutlip and Shacham 2000 [DIRS 163339], Figure 7-1.

Figure H.2-1. Gas Phase Diffusion of A through Stagnant B

where z_1 is the location above the liquid interface, z_2 is the location at the end of the tube, X_{A1} is the mole fraction of A at location 1, and X_{A2} is the mole fraction of A at location 2. The geometry of the locations and the corresponding mole fractions are illustrated in Figure 7-1 of the study by Cutlip and Shacham (2000 [DIRS 163339]). This figure is reproduced above. It should be remembered that a flux is dependent on the aperture or area of the opening. It is implicitly assumed (although not stated) by Cutlip and Shacham (2000 [DIRS 163339]) that the area of opening is 1 meter square. This can be shown by performing a dimensional analysis by assigning and calculating with the dimensional units. This was done for “Diffusion flux - Stefan Tube Anal. Soln (indexed- units).mcd.”

In Equation H.2-6, as adapted from *EBS Radionuclide Transport Abstraction* (BSC 2004 [DIRS 169868], Equation 6.2.2-2), the vector $i=1..67$ defines the range of iteration, $Temp_i$ sums 29 with the vector i to define the temperature range of the calculation $pwfTemp$ (30°C to 96°C), and $pwfTemp$ defines a function that calculates the saturated partial pressure of water

vapor over the temperature range (Temp) of 30°C to 96°C. Equations for $pwfTemp$ are from Singh et al. (2002 [DIRS 161624], Equations 1a and 1b).

$$i := 1.67$$

$$Temp_i := 29 + i$$

$$pwfTemp(Temp) := \begin{cases} 610.78 \exp \left[\frac{17.269(Temp)}{237.3 + (Temp)} \right] & \text{if } (0 \leq Temp) \cdot (Temp < 63) \\ 610.78 \exp \left[\frac{17.269(Temp)}{236.3 + 1.01585(Temp)} \right] & \text{if } (63 \leq Temp) \cdot (Temp < 110) \end{cases}$$

(Eq. H.2-6)

Again, from the saturated partial pressure of water vapor ($pwfTemp$) the partial pressure of water dependent on the relative humidity (RH) can be determined (BSC 2004 [DIRS 169868], Equation 6.6.2-1):

$$pwTemp_i := pwfTemp(Temp_i)$$

$$RH := 0.95$$

$$pTemp_i := (pwTemp_i) \cdot RH$$

(Eq. H.2-7)

where $pwTemp_i$ (Pa) the saturation vapor pressure of water. It is calculated in the function $pwfTemp$ over the temperature range 30°C to 96°C. $Pa = N/M^2$. RH is the relative humidity. Changing the RH calculate the flux over the previously defined temperature range. $pTemp_i$ (Pa) is the partial pressure of water dependent on the relative humidity.

Next, the molal humidity is calculated. In this example it is calculated for 50°C and 95 percent relative humidity (BSC 2004 [DIRS 169868], Equation 6.6.2-4):

$$HmTemp_i := \frac{pTemp_i}{101325 - pTemp_i}$$

(Eq. H.2-8)

where $HmTemp_i$ is the molal humidity (mol H₂O/mol wet air). Pressure for dry air is 1 atm = 101325 Pa, from equation H.1-2 at 50°C the partial pressure of water (p_w) was calculated $p_w = 12,344$ Pa. So the partial pressure at RH = 0.95 is $p_w = 11,717$ Pa. Therefore, the molal humidity (mole fraction) of water vapor under these conditions is 0.1156 mol H₂O mol⁻¹ wet air.

Next, the concentration of water vapor at 50°C and 95 percent relative humidity can be calculated (BSC 2004 [DIRS 169868], Equation 6.6.2-5):

$$C_{wvTemp_i} := \frac{HmTemp_i}{22414 \cdot \left(\frac{273.15 + Temp_i}{273.15} \right)} \quad (\text{Eq. H.2-9})$$

where C_{wvTemp_i} is the concentration of water vapor in the air ($mol\ H_2O\ cm^{-3}$).

To calculate the binary diffusion coefficient, the following equation is used (Bird et al. 1960 [DIRS 103524] Equation 16.3-1; BSC 2004 [DIRS 169868], Equations 6.6.2-6 and 6.6.2-7):

$$D_{ABTemp_i} := 3.64 \cdot 10^{-4} \cdot \left(\frac{273.15 + Temp_i}{\sqrt{647.25 \cdot 132}} \right)^{2.334} \cdot (218.3 \cdot 36.4)^{\frac{1}{3}} \cdot (647.25 \cdot 132)^{\frac{5}{12}} \cdot \left(\frac{1}{18.01528} + \frac{1}{28.964} \right)^{\frac{1}{2}} \cdot (1)^{-1} \quad (\text{Eq. H.2-10})$$

where D_{AB} is the binary diffusion coefficient ($cm^2\ s^{-1}$), $a = 3.64 \times 10^{-4}$ for H_2O , $Temp_i$ = temperature (C) converted into absolute temperature (K), $T_{cA} = 647.25\ K$, $T_{cB} = 132\ K$, $p_{cA} = 218.3\ atm$, $p_{cB} = 36.4\ atm$, $M_A = 18.01528\ g/mol$, $M_B = 28.964\ g/mol$, and $p^{-1} = 1\ atm$.

XB_i is one of the two equations for the analytical Stefan Tube equations and is based on the concentrations of gas A and B and the configuration illustrated in Figure H.2-1. Note: XB_i is dimensionless as its units are (Pa/Pa) (Cutlip and Shacham 2000 [DIRS 163339], Equation 7.8):

$$XB_i := \frac{\left(\frac{pTemp_i}{101325} - 0 \right)}{\ln \left[\frac{(1 - 0)}{\left(1 - \frac{pTemp_i}{101325} \right)} \right]} \quad (\text{Eq. H.2-11})$$

Once the value for XB_i has been determined, the water vapor flux into the waste package can be determined by solving the following equation (Cutlip and Shacham 2000 [DIRS 163339], Equation 7.8):

$$n_{Az_i} := \frac{D_{ABTemp_i} \cdot C_{wvTemp_i}}{(2.54 - 0) \cdot XB_i} \cdot \left(\frac{pTemp_i}{101325} - 0 \right) \cdot 1.93 \quad (\text{Eq. H.2-12})$$

1.93 cm^2 is the aperture of the SCC from *EBS Radionuclide Transport Abstraction* (BSC 2004 [DIRS 169868], p. 6-181).

$$N_{Az_i} := n_{Az_i} \cdot 60 \cdot 60 \cdot 24 \cdot 365.25$$

$$N_{Az_{21}} = 4.466 \blacksquare$$

N_{Az_i} converts n_{Az} from mol/sec to mol/yr.

| $Temp_i =$ | $n_{Az_i} =$ | $N_{Az_i} =$ |
|------------|-----------------------|--------------|
| 30 | $1.473 \cdot 10^{-8}$ | 0.465 |
| 31 | $1.661 \cdot 10^{-8}$ | 0.524 |
| 32 | $1.871 \cdot 10^{-8}$ | 0.59 |
| 33 | $2.106 \cdot 10^{-8}$ | 0.665 |
| 34 | $2.369 \cdot 10^{-8}$ | 0.747 |
| 35 | $2.662 \cdot 10^{-8}$ | 0.84 |
| 36 | $2.99 \cdot 10^{-8}$ | 0.944 |
| 37 | $3.356 \cdot 10^{-8}$ | 1.059 |
| 38 | $3.763 \cdot 10^{-8}$ | 1.188 |
| 39 | $4.217 \cdot 10^{-8}$ | 1.331 |
| 40 | $4.723 \cdot 10^{-8}$ | 1.491 |
| 41 | $5.286 \cdot 10^{-8}$ | 1.668 |
| 42 | $5.912 \cdot 10^{-8}$ | 1.866 |
| 43 | $6.607 \cdot 10^{-8}$ | 2.085 |
| 44 | $7.38 \cdot 10^{-8}$ | 2.329 |
| 45 | $8.238 \cdot 10^{-8}$ | 2.6 |
| 46 | $9.191 \cdot 10^{-8}$ | 2.9 |

Table H.2-1. Water Vapor Flux (mol/yr) Calculated by the Stefan Tube Equation

| Relative Humidity | 0.30 | 0.4 | 0.50 | 0.60 | 0.70 | 0.80 | 0.90 | 0.95 | 1.00 |
|-------------------|--------------|------|------|------|------|------|------|------|------|
| Temp. (°C) | Flux(mol/yr) | | | | | | | | |
| 30 | 0.04 | 0.07 | 0.12 | 0.17 | 0.23 | 0.30 | 0.38 | 0.42 | 0.47 |
| 31 | 0.05 | 0.08 | 0.13 | 0.19 | 0.26 | 0.33 | 0.42 | 0.47 | 0.52 |
| 32 | 0.05 | 0.09 | 0.15 | 0.21 | 0.29 | 0.38 | 0.48 | 0.53 | 0.59 |
| 33 | 0.06 | 0.11 | 0.16 | 0.24 | 0.32 | 0.42 | 0.54 | 0.60 | 0.67 |
| 34 | 0.07 | 0.12 | 0.18 | 0.27 | 0.36 | 0.48 | 0.60 | 0.67 | 0.75 |
| 35 | 0.07 | 0.13 | 0.21 | 0.30 | 0.41 | 0.54 | 0.68 | 0.76 | 0.84 |
| 36 | 0.08 | 0.15 | 0.23 | 0.34 | 0.46 | 0.60 | 0.76 | 0.85 | 0.94 |
| 37 | 0.09 | 0.17 | 0.26 | 0.38 | 0.51 | 0.67 | 0.86 | 0.95 | 1.06 |
| 38 | 0.10 | 0.19 | 0.29 | 0.42 | 0.58 | 0.76 | 0.96 | 1.07 | 1.19 |
| 39 | 0.12 | 0.21 | 0.33 | 0.47 | 0.65 | 0.85 | 1.07 | 1.20 | 1.33 |
| 40 | 0.13 | 0.23 | 0.37 | 0.53 | 0.72 | 0.95 | 1.20 | 1.34 | 1.49 |
| 41 | 0.15 | 0.26 | 0.41 | 0.59 | 0.81 | 1.06 | 1.35 | 1.50 | 1.67 |
| 42 | 0.16 | 0.29 | 0.46 | 0.66 | 0.90 | 1.18 | 1.51 | 1.68 | 1.87 |
| 43 | 0.18 | 0.33 | 0.51 | 0.74 | 1.01 | 1.32 | 1.68 | 1.88 | 2.09 |
| 44 | 0.20 | 0.36 | 0.57 | 0.82 | 1.13 | 1.48 | 1.88 | 2.10 | 2.33 |
| 45 | 0.23 | 0.40 | 0.63 | 0.92 | 1.26 | 1.65 | 2.10 | 2.34 | 2.60 |
| 46 | 0.25 | 0.45 | 0.71 | 1.02 | 1.40 | 1.84 | 2.34 | 2.61 | 2.90 |
| 47 | 0.28 | 0.50 | 0.79 | 1.14 | 1.56 | 2.05 | 2.60 | 2.91 | 3.23 |

Table H.2-1. Water Vapor Flux (mol/yr) Calculated by the Stefan Tube Equation (Continued)

| Relative Humidity | 0.30 | 0.4 | 0.50 | 0.60 | 0.70 | 0.80 | 0.90 | 0.95 | 1.00 |
|-------------------|--------------|-------|-------|-------|-------|--------|--------|--------|--------|
| Temp. (°C) | Flux(mol/yr) | | | | | | | | |
| 48 | 0.31 | 0.56 | 0.87 | 1.27 | 1.73 | 2.28 | 2.90 | 3.24 | 3.60 |
| 49 | 0.35 | 0.62 | 0.97 | 1.41 | 1.93 | 2.54 | 3.23 | 3.61 | 4.01 |
| 50 | 0.38 | 0.69 | 1.08 | 1.57 | 2.15 | 2.82 | 3.59 | 4.02 | 4.47 |
| 51 | 0.43 | 0.76 | 1.20 | 1.74 | 2.38 | 3.14 | 4.00 | 4.47 | 4.97 |
| 52 | 0.47 | 0.85 | 1.33 | 1.93 | 2.65 | 3.48 | 4.44 | 4.97 | 5.52 |
| 53 | 0.52 | 0.94 | 1.48 | 2.14 | 2.94 | 3.87 | 4.93 | 5.52 | 6.14 |
| 54 | 0.58 | 1.04 | 1.64 | 2.38 | 3.26 | 4.29 | 5.48 | 6.13 | 6.82 |
| 55 | 0.64 | 1.15 | 1.81 | 2.63 | 3.61 | 4.76 | 6.08 | 6.80 | 7.57 |
| 56 | 0.71 | 1.27 | 2.01 | 2.92 | 4.00 | 5.28 | 6.74 | 7.54 | 8.40 |
| 57 | 0.79 | 1.41 | 2.22 | 3.23 | 4.43 | 5.85 | 7.47 | 8.37 | 9.32 |
| 58 | 0.87 | 1.56 | 2.46 | 3.57 | 4.91 | 6.47 | 8.28 | 9.27 | 10.33 |
| 59 | 0.96 | 1.72 | 2.71 | 3.95 | 5.43 | 7.17 | 9.17 | 10.27 | 11.45 |
| 60 | 1.06 | 1.90 | 3.00 | 4.36 | 6.00 | 7.93 | 10.15 | 11.38 | 12.68 |
| 61 | 1.17 | 2.09 | 3.31 | 4.82 | 6.64 | 8.77 | 11.23 | 12.59 | 14.04 |
| 62 | 1.28 | 2.31 | 3.65 | 5.32 | 7.33 | 9.70 | 12.43 | 13.94 | 15.54 |
| 63 | 1.41 | 2.55 | 4.03 | 5.87 | 8.10 | 10.71 | 13.74 | 15.42 | 17.20 |
| 64 | 1.56 | 2.80 | 4.44 | 6.47 | 8.93 | 11.83 | 15.19 | 17.04 | 19.02 |
| 65 | 1.71 | 3.09 | 4.89 | 7.14 | 9.85 | 13.06 | 16.78 | 18.83 | 21.03 |
| 66 | 1.88 | 3.39 | 5.38 | 7.86 | 10.86 | 14.41 | 18.52 | 20.81 | 23.25 |
| 67 | 2.07 | 3.73 | 5.92 | 8.66 | 11.97 | 15.89 | 20.45 | 22.98 | 25.69 |
| 68 | 2.27 | 4.10 | 6.51 | 9.53 | 13.19 | 17.52 | 22.57 | 25.38 | 28.38 |
| 69 | 2.49 | 4.51 | 7.16 | 10.48 | 14.52 | 19.31 | 24.90 | 28.01 | 31.34 |
| 70 | 2.74 | 4.95 | 7.86 | 11.53 | 15.99 | 21.28 | 27.47 | 30.92 | 34.62 |
| 71 | 3.00 | 5.43 | 8.64 | 12.68 | 17.59 | 23.45 | 30.30 | 34.13 | 38.23 |
| 72 | 3.29 | 5.95 | 9.48 | 13.93 | 19.35 | 25.82 | 33.41 | 37.66 | 42.21 |
| 73 | 3.60 | 6.53 | 10.41 | 15.30 | 21.29 | 28.44 | 36.84 | 41.55 | 46.61 |
| 74 | 3.94 | 7.16 | 11.42 | 16.81 | 23.41 | 31.31 | 40.62 | 45.84 | 51.47 |
| 75 | 4.32 | 7.84 | 12.52 | 18.46 | 25.73 | 34.47 | 44.78 | 50.58 | 56.83 |
| 76 | 4.72 | 8.58 | 13.73 | 20.26 | 28.28 | 37.94 | 49.37 | 55.81 | 62.76 |
| 77 | 5.16 | 9.40 | 15.05 | 22.23 | 31.08 | 41.75 | 54.43 | 61.58 | 69.33 |
| 78 | 5.64 | 10.28 | 16.49 | 24.39 | 34.15 | 45.95 | 60.01 | 67.97 | 76.59 |
| 79 | 6.17 | 11.25 | 18.06 | 26.75 | 37.52 | 50.57 | 66.17 | 75.03 | 84.65 |
| 80 | 6.73 | 12.30 | 19.77 | 29.34 | 41.21 | 55.65 | 72.98 | 82.85 | 93.58 |
| 81 | 7.35 | 13.45 | 21.65 | 32.17 | 45.27 | 61.25 | 80.51 | 91.51 | 103.51 |
| 82 | 8.02 | 14.69 | 23.69 | 35.26 | 49.72 | 67.43 | 88.85 | 101.12 | 114.56 |
| 83 | 8.75 | 16.05 | 25.92 | 38.65 | 54.61 | 74.23 | 98.08 | 111.81 | 126.88 |
| 84 | 9.54 | 17.53 | 28.35 | 42.36 | 59.98 | 81.75 | 108.33 | 123.70 | 140.64 |
| 85 | 10.40 | 19.14 | 31.01 | 46.43 | 65.89 | 90.05 | 119.72 | 136.97 | 156.04 |
| 86 | 11.33 | 20.89 | 33.91 | 50.88 | 72.40 | 99.23 | 132.41 | 151.80 | 173.34 |
| 87 | 12.35 | 22.79 | 37.08 | 55.76 | 79.56 | 109.40 | 146.56 | 168.41 | 192.83 |
| 88 | 13.44 | 24.86 | 40.54 | 61.11 | 87.45 | 120.68 | 162.39 | 187.09 | 214.87 |

Table H.2-1. Water Vapor Flux (mol/yr) Calculated by the Stefan Tube Equation (Continued)

| Relative Humidity | 0.30 | 0.4 | 0.50 | 0.60 | 0.70 | 0.80 | 0.90 | 0.95 | 1.00 |
|-------------------|--------------|-------|-------|--------|--------|--------|--------|--------|--------|
| Temp. (°C) | Flux(mol/yr) | | | | | | | | |
| 89 | 14.64 | 27.12 | 44.31 | 66.99 | 96.16 | 133.21 | 180.14 | 208.17 | 239.91 |
| 90 | 15.93 | 29.57 | 48.44 | 73.43 | 105.77 | 147.16 | 200.11 | 232.06 | 268.52 |
| 91 | 17.33 | 32.24 | 52.95 | 80.51 | 116.40 | 162.73 | 222.68 | 259.26 | 301.45 |
| 92 | 18.85 | 35.15 | 57.87 | 88.29 | 128.18 | 180.15 | 248.30 | 290.45 | 339.68 |
| 93 | 20.49 | 38.31 | 63.26 | 96.85 | 141.24 | 199.70 | 277.54 | 326.49 | 384.55 |
| 94 | 22.28 | 41.75 | 69.16 | 106.28 | 155.77 | 221.72 | 311.17 | 368.55 | 438.02 |
| 95 | 24.21 | 45.50 | 75.61 | 116.69 | 171.96 | 246.63 | 350.17 | 418.29 | 503.02 |
| 96 | 26.31 | 49.58 | 82.68 | 128.18 | 190.04 | 274.98 | 395.93 | 478.14 | 584.37 |

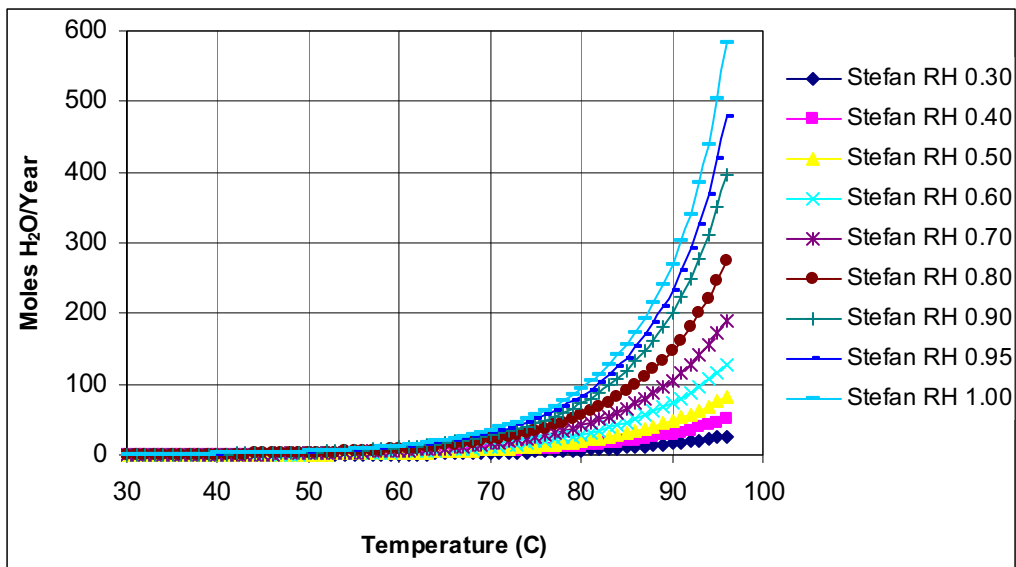


Figure H.2-2. Water Vapor Flux Calculated Based on Stefan Tube Equations



Addendum Cover Page

Complete only applicable items.

QA: QA

1. Total Pages: 184

| | | | |
|--|---|--------------------------|---------|
| 2. Addendum to (Title): In-Package Chemistry Abstraction | | | |
| 3. DI (including Revision and Addendum No.): ANL-EBS-MD-000037 REV 04 AD 01 | | | |
| | Printed Name | Signature | Date |
| 4. Originator | Paul Mariner, William Downs | <i>Paul Mariner</i> | 8/8/07 |
| | | <i>William Downs</i> | 8/8/07 |
| 5. Independent Technical Reviewer | David Sassani | <i>David Sassani</i> | 8/9/07 |
| 6. Checker | Patricia Bernot | <i>Patricia Bernot</i> | 8/9/07 |
| 7. QCS / Lead Lab QA Reviewer | Robert E. Spencer | <i>Robert E. Spencer</i> | 8/9/07 |
| 8. Responsible Manager / Lead | Patrick Brady | <i>Patrick V. Brady</i> | 8/10/07 |
| 9. Responsible Manager | Geoff Freeze | <i>Geoff Freeze</i> | 8/10/07 |
| 10. Remarks | | | |
| Change History | | | |
| 11. Revision and Addendum No. | 12. Description of Change | | |
| REV 04 AD 01 | This addendum develops, validates, and documents an in-package chemistry (IPC) model designed for individual waste form cells instead of entire waste packages. It produces abstractions for pH, ionic strength, and fluoride for use in TSPA-LA. The primary objectives of this work are to improve consistency between the IPC model and TSPA-LA models, to improve the functionality of the IPC abstractions, and to address condition reports. The condition reports addressed or partially addressed in this addendum are: CR-5384, CR-6731, CR-6857, CR-7233, CR-7464, CR-7542, CR-7756, CR-7763, CR-8555, and CR-8656. | | |

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ACRONYMS AND ABBREVIATIONS

| | |
|-------------------|---|
| ANC | acid neutralizing capacity |
| ANL | Argonne National Laboratory |
| BNC | base neutralizing capacity |
| CDSP | codisposal (waste package) |
| CR | condition report |
| CSNF | commercial spent nuclear fuel |
| DHLW | defense high-level (radioactive) waste |
| DIRS | Document Input Reference System |
| DOE | U.S. Department of Energy |
| DSNF | DOE spent nuclear fuel |
| DTN | data tracking number |
| Eh | reduction/oxidation (redox) potential |
| HLW | high-level (radioactive) waste |
| <i>I</i> | ionic strength |
| IPC | in-package chemistry |
| kg | kilogram |
| L | liter |
| LA | license application |
| MCO | multicanister overpack |
| mg | milligram |
| mmol | millimole |
| NFE | Near-Field Environment |
| pCO ₂ | negative logarithm of the partial pressure of CO ₂ (g) in bars |
| pH | negative logarithm of the activity of hydrogen ion in water |
| pH _{zpc} | pH zero point of charge |
| ppm | parts per million |
| QA | Quality Assurance |
| RH | relative humidity |
| SRL | Savannah River Laboratory |

ACRONYMS AND ABBREVIATIONS (CONTINUED)

| | |
|------|--|
| TAD | transportation, aging, and disposal (canister) |
| TDIF | technical data input form |
| TMI | Three-Mile Island |
| TSPA | total system performance assessment |
| TWP | technical work plan |
| USGS | U.S. Geological Survey |
| VA | validation activity |
| wt % | weight percent |
| YMP | Yucca Mountain Project |
| µm | micrometer |

1[a]. PURPOSE

This addendum develops and documents an additional in-package chemistry (IPC) conceptual model and a new set of IPC abstractions for pH, ionic strength, and fluoride. The purpose of the work is to improve consistency between the IPC and total system performance assessment (TSPA) models and to improve the functionality and precision of the IPC abstractions. This addendum does not replace work documented in the parent report. Rather, it augments the original work with additional analyses, assessments, and abstractions.

For clarity, the general outline of the parent report is preserved in this addendum. In sections where there are no changes relative to the parent report, the text from the parent report is not repeated. Instead, the section simply indicates that there are no changes, implicitly referring the reader to the parent report.

Most of the activities performed in this addendum are based on activities identified in *Technical Work Plan for Waste Form Testing and Modeling* (BSC 2006 [DIRS 177389]). Included among these activities is the addition of validation methods to provide a high level of confidence in the abstractions. In addition, as a deviation from the technical work plan (TWP), certain supplemental activities were performed to enhance the integration of the IPC abstractions with the TSPA models. Table 1-1[a] lists the activities addressed in this addendum, their justification, and the locations in the addendum where the activities are addressed.

The IPC conceptual model in this addendum redefines the control volume as a waste form cell instead of a waste package cell. This new definition is consistent with the waste form cells defined and used in TSPA for the license application (LA). Hence, the conceptual model reported in this addendum is called the waste form IPC model to distinguish it from the previous waste package IPC model.

The TSPA-LA conceptual model divides the waste packages into multiple cells. In the commercial spent nuclear fuel (CSNF) waste package, the cells consist of the following:

1. Cell 1 contains the fuel assemblies, the fuel basket tubes, and the fuel basket including the neutron absorber and thermal shunt plates. This waste form cell is called the CSNF cell in this addendum.
2. Cell 2 contains the fuel basket guides, the transportation, aging, and disposal (TAD) canister and its end plug, the inner vessel and its lid, and the outer corrosion barrier and its lid.

The codisposal (CDSP) waste packages, represented by 2MCO/2DHLW CDSP waste packages in this addendum, are divided into three cells:

1. Cell 1a contains two glass pour canisters and their high-level waste (HLW) glass (also referred to herein as HLWG). This waste form cell is called 2DHLW in this addendum (where “DHLW” stands for “defense high-level waste”).

2. Cell 1b contains two multi-canister overpack (MCO) units containing N-Reactor fuel. This waste form cell is called 2MCO in this addendum.
3. Cell 2 contains the divider plates, the fuel support plate assemblies, the inner vessel and its lid, and the outer corrosion barrier and its lid.

This addendum is concerned only with the interactions of fluids and materials within CSNF Cell 1 and CDSP Cells 1a and 1b. Cell 2 processes are addressed in *EBS Radionuclide Transport Abstraction* (SNL 2007 [DIRS 177407]).

The processes included in the waste form IPC model are the same as in the waste package IPC model except that surface complexation is not included. Surface complexation is not included because the substitution of stainless steel for carbon steel means that there will be a much lower rate of steel degradation in the waste form cell (Table 6-5 of parent report) and therefore a much lower abundance of iron alteration products (e.g., oxides and hydroxy-oxides) during the early history of the waste package. The pH buffering capacities of the surface reactions were used in the previous IPC model to restrict the pH ranges in the abstractions.

Waste form IPC abstractions for pH, ionic strength, and fluoride are generated. These abstractions differ from the previous waste package IPC abstractions in a number of ways (e.g., Table 1-1[a]). The unchanged abstractions are the total carbonate concentration, Eh, and chloride abstractions. The total carbonate concentration abstraction is independent of the IPC conceptual model. The Eh and chloride abstractions are not changed because they are not used in TSPA-LA.

The IPC abstractions are used in the TSPA-LA model when the waste package has been breached and the interior temperature of the waste package is below the boiling temperature of water. References to time within this addendum refer to the time after waste package breach and after the waste package cools to 100°C or lower. Liquid water only persists in appreciable quantities when the drift environment has cooled to below approximately 100°C. While the boiling point of pure water at the repository elevation is approximately 96°C, the value of 100°C is used to reflect possible higher boiling points of liquids containing dissolved solids. The waste form IPC model is a combination of a liquid influx model, applied over a liquid influx range of 0.1 to 1,000 L/yr per cell, and a vapor influx model, applied over a relative humidity range of 95% to 100%. The applied temperature range is 25°C to 100°C, and the applied range of carbon dioxide partial pressure is 10^{-4} to $10^{-1.5}$ bar. The applicability of the waste form IPC model is limited to the waste form cells.

Limitations of this model include: (1) limited availability of data on thin film chemistry (Assumption 5.1); (2) limited availability of long-term metal alloy corrosion information such as the identities of the long-term corrosion products and the evolution of the surface area of the metal waste package components as a function of time (Assumption 5.2); (3) simplifications used to define the important processes in the waste form cells (Assumptions 5.3 and 5.4[a]); (4) an assumption involving the pH under low water conditions (Assumption 5.5[a]); and (5) the approach used to simulate the effects of evaporation (Section 6.3.1.1[a]).

Table 1-1[a]. Activities Addressed in Addendum

| Activity | Description | Justification | How Activity Addressed | Section(s) Where Addressed |
|------------------------------|--|--|--|--------------------------------|
| Waste Package Design Changes | Model the effects of incorporating TAD canisters | Planned activity in TWP (BSC 2006 [DIRS 177389], pp. 6 and 22) | The carbon steel and NiGd alloy of the basket materials in the CSNF Cell 1 were changed to 316 and 304B4 stainless steels in accordance with the new TAD canister design. | 6.3.1.3[a] |
| Basalt Water | Model the effects of inflow of "basalt water," following an igneous intrusion event | Planned activity in TWP (BSC 2006 [DIRS 177389], pp. 6, 22, and 23) | Basalt water inflow simulations were performed. Results were evaluated in the quantification of uncertainty in abstractions. | 6.3.1.3.5[a], 6.6.2[a] |
| Model Validation | Enhance model validation using available laboratory data | Planned activity in TWP (BSC 2006 [DIRS 177389], pp. 6 and 23) | Validation is address with available relevant data. In addition, a PHREEQC simulation was performed to validate the EQ6 execution of the IPC process model. | 7[a] |
| Phosphate Data | Address issues (CR-6731) related to phosphate data in thermodynamic database | Planned activity in TWP (BSC 2006 [DIRS 177389], pp. 6 and 23) | An upgraded thermodynamic database (<i>data0.ymp.R5</i>) that resolves the phosphate issues was used in all simulations. | 4.1.1[a] |
| Waste Form Degradation | Address issues (CR-6857) related to inconsistencies in modeled degradation rates | Planned activity in TWP (BSC 2006 [DIRS 177389], pp. 6 and 23) | Range of degradation rates determined based on pH feedback mechanisms and long-term temperature history. | 6.3.1.3.4[a] |
| Ionic Strength | Enhance the scientific basis for calculations at ionic strengths greater than 1 molal | Planned activity in TWP (BSC 2006 [DIRS 177389], pp. 6, 7, 23, and 24) | The IPC simulations in this addendum do not exceed an ionic strength of ~1 molal. To predict ionic strength greater than 1 molal, Pitzer calculations from a separate report are used. | 6.3.1.1[a] |
| IPC Cell Definition | Address integration and consistency issues regarding the physical control volume simulated by the IPC model | Planned activity in TWP (BSC 2006 [DIRS 177389], pp. 7 and 24) | The conceptual IPC model was changed from a waste package mixing cell model to a waste form mixing cell model. | 1[a], 5.4[a], 6.3[a], 6.3.1[a] |
| Water Balance | Assess effects related to water balance including evaporation, vapor transport, water consumption, and relative humidity | Planned activity in TWP (BSC 2006 [DIRS 177389], pp. 7 and 24) | The ionic strength abstraction at low or zero liquid influx was modified to be a function of relative humidity. Evaporation is not explicitly included in the process model or abstractions, but its effects on solution composition are implicitly included when relative humidity controls the ionic strength. Consumption of water by degradation reactions has the same effect on solution composition as evaporation and is included in the process model and abstractions. | 6.3[a], 6.3.1[a] |

Table 1-1[a]. Activities Addressed in Addendum (Continued)

| Activity | Description | Justification | How Activity Addressed | Section(s) Where Addressed |
|-------------------------------------|--|--|---|--|
| Surface Complexation | No longer include the effects of surface complexation in the IPC model | Planned activity in TWP (BSC 2006 [DIRS 177389], p. 7) | The effects of surface complexation were removed from waste form IPC abstractions. Surface complexation is considered in Cell 2 of the waste package, which is addressed in <i>EBS Radionuclide Transport Abstraction</i> (SNL 2007 [DIRS 177407]). | 4.1.6[a], 5.4[a], 6.7[a], 6.8[a], 6.9[a] |
| pH Buffering Reactions | Utilize pH buffering reactions to derive minimum and maximum pH values for the pH abstractions | To enhance clarity and rigor of pH abstraction (not a planned activity in TWP) | Acid and base titrations were simulated in the waste form IPC cells to determine pH buffering capacities of the degradation products and to abstract minimum and maximum pH values related to the controlling buffering reactions. | 6.3.4[a] |
| Effects of CO ₂ Fugacity | Include CO ₂ fugacity as independent variable in pH abstractions | To enhance resolution of pH abstraction and eliminate related out-of-bounds issues in TSPA-LA (not a planned activity in TWP) | CO ₂ fugacity was included as an independent variable in the acid and base titrations used to produce the pH abstractions. This relationship was previously included via the surface complexation model. | 6.10.1[a] |
| Effect of Time | Include the effects of time on ionic strength abstractions | To capture the effects of time for an increased regulatory time period (not a planned activity in TWP) | Time was included as an independent variable in liquid influx ionic strength abstractions up to one million years. Other waste form IPC abstractions are only a weak function of time. | 6.10.2.1[a] |
| Fluoride Abstraction | Produce a fluoride abstraction consistent with the waste form IPC model | To feed <i>Dissolved Concentration Limits of Elements with Radioactive Isotopes</i> (SNL 2007 [DIRS 177418]) (not a planned activity in TWP) | Because high fluoride concentrations can potentially increase radionuclide solubilities, new fluoride abstractions were produced to reflect changes in the IPC conceptual model. | 6.10.3[a] |
| Additional Condition Reports | Address all CRs that cite or apply to the IPC model | To facilitate closure of these CRs (not a planned activity in TWP) | In addition to CR-6731 and CR-6857 identified above, the following CRs are addressed: 5384, 7233, 7464, 7542, 7756, 7763, 8555, and 8656. | 6.11[a] |

NOTE: CR = condition report.

2[a]. QUALITY ASSURANCE

2.1[a] QA PROGRAM APPLICABILITY

The output for this addendum is developed under the Yucca Mountain Project (YMP) Quality Assurance (QA) Program. This activity is subject to requirements listed in *Quality Assurance Requirements and Description* (DOE 2007 [DIRS 182051]). All waste package configurations have been determined to be important for waste isolation in accordance with LS-PRO-0203, *Preparation and Maintenance of Q-List*, and *Q-List* (BSC 2005 [DIRS 175539]).

The inputs are documented according to SCI-PRO-004, *Managing Technical Product Inputs*. Appropriate control of the electronic management of data as required by IM-PRO-002, *Control of the Electronic Management of Information*, is accomplished in accordance with *Technical Work Plan for Waste Form Testing and Modeling* (BSC 2006 [DIRS 177389]). The output data are developed using accepted standard modeling techniques and are validated in accordance with the relevant sections of SCI-PRO-006, *Models*. Uncertainties resulting from measured data and application of models are evaluated, propagated, and included in specification of outputs. Because the model inputs are obtained from controlled and qualified sources and the mathematical models are validated per applicable quality assurance procedures, the output parameters to be provided to TSPA-LA are adequate for their intended use for the prediction of the composition of fluids in the vicinity of the waste forms.

2.2[a] ELECTRONIC MANAGEMENT OF INFORMATION

The TWP contains the process control evaluation used to evaluate the control of electronic management of information (BSC 2006 [DIRS 177389], Appendix A) during the modeling and documentation activities of the work presented in this addendum. This evaluation determined that the methods in the implementing procedures are adequate; therefore, no deviations from these methods are necessary.

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3[a]. USE OF SOFTWARE

The software used in this addendum is listed in Table 3-1[a] along with the operating systems of the computers used to execute the software. In accordance with Section 2.0 of IM-PRO-003, *Software Management*, the single software program in this list that is exempt from the qualification requirement is Microsoft Excel, a commercially available standard software program used in this addendum exclusively for charting results and performing simple calculations using standard functions. Each nonexempt software item in Table 3-1[a] is appropriate for use in geochemical modeling and uses a thermodynamic database qualified for YMP work. There are no limitations on software outputs. Use of the software was consistent with the intended use and within documented validation ranges. No nonexempt software was used prior to qualification. The PHREEQC software was used only for validation.

Table 3-1[a]. Computer Software and Operating Systems Used

| Software Name | Version | Software Tracking Number | Operating System Used | Description and Components Used | Input and Output Files |
|-----------------|----------|--|-----------------------------|---|--|
| EQ3/6 | 7.2b | UCRL-MA-110662 [DIRS 153964] (Qualified on Windows 95) | Windows 95 | EQ3NR: Speciation-solubility code | Input: *.3i Output: *.3o |
| | | | | EQPT: Thermodynamic database file preprocessor | Input: data0.* Output: data1.* |
| EQ6 | 7.2bLV | 10075-7.2bLV-02 [DIRS 159731] (Qualified on Windows 2000 and NT) | Windows 2000 | EQ6: Reaction path code that simulates water-material interaction or fluid mixing in either a pure reaction progress mode or an elapsed time mode | Input: *.6i Pickup: *.6p Output: *.6o *.elem_aqu.txt *.elem_min.txt *.elem_tot.txt *.min_info.txt *.bin |
| GetEQData | 1.0.1 | 10809-1.0.1-00 [DIRS 173680] (Qualified on Windows-95, -98, -2000, and NT) | Windows 2000 | Microsoft Excel macro used to post-process EQ3/6 output information | Input: *.3o, *.6o Output: *.xls |
| PHREEQC | 2.11 | 10068-2.11-01 [DIRS 175698] (Qualified on Windows 2000) | Windows 2000 | Reactive transport code that simulates hydrogeochemical reactions and processes | Input: *. (no extension) Output: *.out |
| Microsoft Excel | 2003 SP2 | Commercial off-the-shelf software (exempt) | Windows 2000 and Windows XP | Used for graphical representation and arithmetical manipulations | Input: *.3o, *.6o, *.min_info.txt Output: *.xls |

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4[a]. INPUTS

4.1[a] DIRECT INPUT

The sources of direct inputs used in this addendum are summarized in Table 4-1[a]. These data are presented and justified for use in the subsections that follow. The direct input data are not used to validate the model.

4.1.1[a] Thermodynamic Database

The simulations documented in this addendum use the thermodynamic database *data0.ymp.R5*, archived under data tracking number (DTN) SN0612T0502404.014 [DIRS 178850], as a necessary component in the execution of the EQ3/6 software. This database extends to temperatures of 300°C, but only the data for 25°C are used in this addendum. It is appropriate for the waste form IPC model because it includes the elements that constitute the waste package, waste form, seepage, and gas compositions at the temperature of application.

Details of the development of the source *data0.ymp.R5* database are documented in *Qualification of Thermodynamic Data for Geochemical Modeling of Mineral-Water Interactions in Dilute Systems* (SNL 2007 [DIRS 177409]). Two improvements documented in that report that are important to the IPC model are:

- CR-6731 noted issues with various phosphate data in a previous version of the database. As part of the response to CR-6731, phosphate data were recalculated for consistency with the phosphate key data recommended by Committee on Data for Science and Technology (CODATA) (Cox et al. 1989 [DIRS 150874]). In addition, several rare earth phosphates, most importantly gadolinium phosphate, were mistakenly listed as having 10 waters of hydration ($\text{GdPO}_4 \cdot 10\text{H}_2\text{O}$). This was found to be a misinterpretation of the term “ $x\text{H}_2\text{O}$ ” as an indicator of a variable number of waters of hydration. These compounds were corrected and their dissolution reactions were rewritten to reflect the appropriate dissociation constants. It should be noted, however, that gadolinium is no longer a major component in the waste package design.
- The thermodynamic data for several new mineral and aqueous species were added to the database. They include data for nickel carbonate (NiCO_3), nickel molybdate (NiMoO_4), kogarkoite ($\text{Na}_3\text{SO}_4\text{F}$), amorphous antigorite ($\text{Mg}_3\text{Si}_2\text{O}_5(\text{OH})_4$), amorphous sepiolite ($\text{Mg}_4\text{Si}_6\text{O}_{15}(\text{OH})_2 \cdot 6\text{H}_2\text{O}$), and several uranium minerals that precipitate in UO_2 degradation experiments and occur in the vicinity of natural UO_2 ore deposits. Many of these additions were in response to CRs (e.g., CR-6489, CR-7542, and CR-7756) (SNL 2007 [DIRS 177409], pp. 1-14 to 1-15).

Prior to use in IPC simulations, *data0.ymp.R5* was modified as described in Section 6.3.1.1[a].

4.1.2[a] Input Water Compositions and Rates

Tables 4-2[a] through Table 4-5[a] give the compositions of the various seepage waters listed in Table 4-1[a] and used as direct input in the IPC simulations.

Five water compositions were used in the liquid influx model for the seismic scenario. They include the four compositions in Table 4-2[a] and the mean J-13 well water composition in Table 4-3[a]. The pore water samples in Table 4-2[a] represent the four major water types found in the Topopah Spring welded tuff (TSw) (SNL 2007 [DIRS 177412], Section 6.6.5). The composition of each TSw pore water sample considered in the analysis of pore water types is documented in file *TSw_Porewater_Data.xls* in DTN: SN0705PAEBSPCE.015 [DIRS 182269]. For the fluoride abstraction, the maximum composition of fluoride in these samples was used (Section 6.10.3[a]). Well J-13 has its well screen in the saturated zone of Yucca Mountain. The J-13 composition is the mean water composition of the J-13 well as documented in *Report of the Committee to Review the Use of J-13 Well Water in Nevada Nuclear Waste Storage Investigations* (Harrar et al. 1990 [DIRS 100814], Section 4).

The qualification of the J-13 well water data from Harrar et al. (1990 [DIRS 100814], Section 4) is provided in Appendix A[a]. Comparisons of J-13 well water to water samples collected from other wells at Yucca Mountain and to water samples from cores of similar tuff units collected near Yucca Mountain show that J-13 water composition is generally representative of water compositions in the saturated and unsaturated zones in the vicinity of Yucca Mountain (Harrar et al. 1990 [DIRS 100814], Sections 5 and 6). Moreover, simulated evaporation of the mean J-13 well water composition (SNL 2007 [DIRS 177411], Figure 6-4) and experimental and simulated evaporations of synthetic J-13 well water (SNL 2007 [DIRS 177411], Figure 7-69) suggest, based on pH and Ca trajectories, that J-13 water is the same water type as SD9, the most common water type (Group 1) in TSw core samples (SNL 2007 [DIRS 177412], Figure 6.6-17).

The field pH and down-hole temperature measurements of J-13 well water documented on page 4.9 of the report by Harrar et al. (1990 [DIRS 100814]) are used in this addendum. These field values are appropriate because water samples for pH are difficult to preserve and pH is affected by temperature. Use of these field measurements is supported by aqueous speciation calculations that show excellent charge balance when these field values are used (file *j13a.3o* of Output DTN: SN0702PAIPC1CA.001 shows that the mean measured alkalinity is reduced by approximately 1% to achieve charge balance when the field pH and down-hole temperature are used.). As explained on page 4.9 of the report by Harrar et al. (1990 [DIRS 100814]), the higher laboratory pH measurements are likely a result of carbon dioxide degassing.

Table 4-1[a]. Summary of Direct Inputs

| Identifier | Input | Source | Section(s) Where Used: |
|------------------------------|--|---|--------------------------------------|
| Periodic Table | Atomic Weights of Elements and Isotopes | No change | Not applicable |
| data0.ymp.R5 | Thermodynamic Data | DTN: SN0612T0502404.014 [DIRS 178850] | 4.1.1[a], 6.3.1.1[a] |
| 21-PWR CSNF Waste Package | Waste Package Component Dimensions | SNL 2007 [DIRS 179394], Table 4-1 | 4.1.4[a], 6.3.1.3.1[a], 6.3.1.3.2[a] |
| 2-MCO / 2-DHLW Waste Package | Waste Package Component Dimensions | SNL 2007 [DIRS 179567], Table 4-10 | 4.1.4[a], 6.3.1.3.1[a], 6.3.1.3.2[a] |
| TAD Canister | Waste Package Component Dimensions and Composition | SNL 2007 [DIRS 179394], Table 4-1 | 4.1.4[a], 6.3.1.3.1[a], 6.3.1.3.2[a] |
| Stainless Steel Type 316 | Corrosion rates | No change | Not applicable |
| | Density | No change | Not applicable |
| | Composition | No change | Not applicable |
| Stainless Steel Type 304B4 | Corrosion rates | DTN: MO0409SPAACRWP.000 [DIRS 172059], file: <i>Data Summary.doc</i> | 4.1.4.2[a], 6.3.1.3.3[a] |
| | Density | CRS ALLOY Data NeutroSorb PLUS (Carpenter Technology 2003 [DIRS 179642], Table "Physical Properties") | 4.1.4.3[a], 6.3.1.3.1[a] |
| | Composition | ASTM A 887-889 [DIRS 154062], Table 1 | 4.1.4.1[a], 6.3.1.3.3[a] |
| Stainless Steel Type 304L | Corrosion rates | No change | Not applicable |
| | Density | No change | Not applicable |
| | Composition | No change | Not applicable |
| Aluminum Alloy 6061 | Corrosion rates | No change | Not applicable |
| | Density | No change | Not applicable |
| | Composition | No change | Not applicable |
| Aluminum Alloy 1100 | Corrosion rates | No change | Not applicable |
| | Density | No change | Not applicable |
| | Composition | No change | Not applicable |

Table 4-1[a]. Summary of Direct Inputs (Continued)

| Identifier | Input | Source | Section(s) Where Used: |
|--|--|---|--|
| CSNF | Degradation Rates | No change | Not applicable |
| | Density | Barner 1985 [DIRS 109194], Table 4.1 | 4.1.3.1[a], 6.3.1.3.1[a] |
| | Composition | No change | Not applicable |
| HLW Glass | Degradation Rates | No change | Not applicable |
| | Density | No change | Not applicable |
| | Composition | Allison 2004 [DIRS 168734], Table 3 (SRL HLW Glass) | 4.1.3.3[a], 6.3.1.3.4[a] |
| N-Reactor fuel | pH of Leachate | Bates et al. 1993 [DIRS 100705], p. 51 | 4.1.3.3[a], 5.5[a] |
| | f_{exposure} | No change | Not applicable |
| | Degradation Rates | No change | Not applicable |
| Schoepite | Composition | No change | Not applicable |
| | Mineral Solubility | DTN: MO0707DISVALID.000 [DIRS 182270], file: <i>U solubility tables.xls</i> | 6.3.4.1[a] |
| | Activity of Water at High Ionic Strength | *.6o files in Salts25.zip in DTN: MO0701EQ36IDPS.000 [DIRS 179290] | 6.5.2[a], 6.10.2.2[a], 6.10.8.2[a] |
| Water Film Thickness | Water Monolayers and RH | SNL 2007 [DIRS 177407], Section 6.5.2.2.1, Table 6.5-8 | 6.10.9.1[a] |
| | Chemical Compositions | DTN: SN0705PAEBSPCE.015 [DIRS 182269], file: <i>TSw_Porewater_Data.xls</i> | 4.1.2[a], 6.10.3[a] |
| Determination of Representative TSw Pore Water Samples | Chemical Compositions | SNL 2007 [DIRS 177412], Section 6.6.5 | 4.1.2[a], 6.10.1.1[a], 6.10.2.1[a] |
| | Chemical Composition | DTN: GS030408312272.002 [DIRS 165226], file: <i>314666PoreWater.xls</i> , pore water sample SD-9/1184.7-1184.8/UC | 4.1.2[a], 6.5[a], 6.6[a], 6.10.1[a] (SD9 water only) |
| P33 Water | Chemical Composition | DTN: MO0005PORWATER.000 [DIRS 150930], pore water sample ESF-HD-PERM-3/34.8-35.1 | |
| P35 Water | Chemical Composition | DTN: GS060908312272.004 [DIRS 179065], file: <i>GS060908312272_004.xls</i> , pore water sample HD-PERM-3/56.7-57.1/UC | |

Table 4-1[a]. Summary of Direct Inputs (Continued)

| Identifier | Input | Source | Section(s) Where Used: |
|------------|----------------------|---|--------------------------|
| T17 Water | Chemical Composition | DTN: GS031008312272.008 [DIRS 166570], file: <i>PW_data_package.xls</i> , pore water sample ESF-THERMALK-017/26.5-26.9/UC | 4.1.2[a], 6.5[a], 6.6[a] |
| J-13 Water | Chemical Composition | No change in major ion concentrations; field measured pH values and temperature from Harrar et al. 1990 [DIRS 100814], p. 4.9 | |
| BI Water | Chemical Composition | Ground water sample from Iceland; Gislaason and Eugster 1987 [DIRS 168716], Table 3, Sample SP 01 | |
| BS Water | Chemical Composition | Ground water sample from the Columbia Basin, Saddle Mountain/Wanapum formations; Turney 1986 [DIRS 179852], pp. 74 to 78, Sample 14/31E-19B01 | |
| BW Water | Chemical Composition | Ground water sample from the Columbia Basin, Wanapum/Grande Ronde formations; Turney 1986 [DIRS 179852], pp. 49 to 53, Sample 19/31E-27G01D1 | |

NOTE: SRL = Savannah River Laboratory.

The decision to use the five water compositions in Tables 4-2[a] and 4-3[a] was based on several lines of reasoning. J-13 well water was included as a potentially relevant seepage water composition because the well is screened across the Repository horizon – the Topopah Spring Tuff middle non-lithophysal zone – and has been used in previous IPC calculations. The four host-rock pore water compositions in Table 4-2[a] were included because they were obtained from core samples near the repository. An extensive analysis of Yucca Mountain groundwater samples in *Engineered Barrier System: Physical and Chemical Environment* (SNL 2007 [DIRS 177412], Section 6.6.5) concluded that these four samples represent the four general types of Yucca Mountain pore waters. The analysis began by identifying which samples showed the best evidence for not having been altered during sampling and storage. It ended with a set of statistical analyses that defined four basic water types represented by the four samples in Table 4-2[a]. The five waters are intended to represent seepage compositions that could enter a breached waste package over the long term. Effects of high temperatures and boiling on ground water compositions were not considered because these effects will be temporary.

The igneous scenario requires consideration of groundwater that has been in contact with the basalt surrounding the waste packages. Three groundwater samples from large fractured basalt reservoirs were selected for these simulations. Two of the samples are from the Columbia River Plateau basalts at the Hanford Reservation in Washington, and one is from the Iceland basalts (Tables 4-4[a] and 4-5[a]). The samples from the Columbia Basin represent the two predominant water types observed there: a sodium-rich ground water (BW) from a deep well screened in the Wanapum and Grande Ronde formations and a calcium-rich ground water (BS) from a shallower well screened in the Saddle Mountain and Wanapum formations.

The three basalt water compositions in Tables 4-4[a] and 4-5[a] come from two publications. The Icelandic basalt water comes from an article by Gislason and Eugster (1987 [DIRS 168716]). The Columbia River Plateau (Hanford) water compositions are taken from Water Resources Investigation Report 85-4320 (Turney 1986 [DIRS 179852]), published by the U.S. Geological Survey (USGS). The data from these sources are qualified in Appendix A[a].

Pure water is used to represent vapor condensation inside the waste package and vapor condensation on the drip shield and drift wall that collects and flows into the waste package. Because water vapor is a gas and contains no dissolved constituents, vapor condensation and influx of condensate can be represented by a liquid influx of pure water.

Liquid influx rate into the waste form cell is not included in this section because it is not a direct input to the model. Instead, it is an independent variable in the model abstractions. To determine the functional relationship between the liquid influx rate and the model outputs, the rate is varied in the liquid influx process model simulations between 0.1 and 1,000 L/yr.

Table 4-2[a]. Yucca Mountain Pore Water Compositions

| Parameter | Units | SD9 | T17 | P35 | P33 |
|------------------|-------|-----|------|------|------|
| Ca | mg/L | 19 | 62 | 59.9 | 97 |
| Mg | mg/L | 0.7 | 7.9 | 16.7 | 17.4 |
| Na | mg/L | 59 | 45 | 123 | 62 |
| K | mg/L | 4.8 | 14.4 | 13.8 | 9 |
| SiO ₂ | mg/L | 42 | 52 | N/M | 75 |
| NO ₃ | mg/L | 16 | 44 | 57.4 | 10 |
| HCO ₃ | mg/L | 142 | 126 | 149 | N/M |
| Cl | mg/L | 23 | 67 | 146 | 123 |
| F | mg/L | 2.2 | 1.4 | 1.3 | 0.76 |
| SO ₄ | mg/L | 16 | 82 | 126 | 120 |
| pH | pH | 8.2 | 7.7 | N/M | 8.31 |

Source: SD9: Sample SD-9/1184.7-1184.8/UC, Unit: Tptpln (DTN: GS030408312272.002 [DIRS 165226], file: 314666PoreWater.xls).

T17: Sample ESF-THERMALK-017/26.5-26.9/UC, Unit: Tptpmn (DTN: GS031008312272.008 [DIRS 166570], file: PW data package.xls).

P35: Sample HD-PERM-3/56.7-57.1/UC (DTN: GS060908312272.004 [DIRS 179065], file: GS060908312272_004.xls).

P33: Sample ESF-HD-PERM-3/34.8-35.1 (DTN: MO0005PORWATER.000 [DIRS 150930]).

NOTE: N/M = not measured.

Table 4-3[a]. J-13 Water Composition

| Parameter | Units | Value |
|--|----------------|-------------|
| Ca | mg/L | No change |
| Mg | mg/L | No change |
| Na | mg/L | No change |
| K | mg/L | No change |
| Si | mg/L | No change |
| NO ₃ ⁻ | mg/L | No change |
| Alkalinity (as HCO ₃ ⁻) | mg/L | No change |
| Cl ⁻ | mg/L | No change |
| F ⁻ | mg/L | No change |
| SO ₄ ²⁻ | mg/L | No change |
| Field pH | Standard units | 6.9 and 7.1 |
| Temperature | Celsius | 31 |

Source: Harrar et al. 1990 [DIRS 100814], p. 4.9.

Table 4-4[a]. A Basalt Ground Water Sample (BI) from Iceland

| Parameter | Units | BI |
|------------------|------------------------|------|
| Na | ppm | 15.8 |
| SiO ₂ | ppm | 18.1 |
| Ca | ppm | 4.35 |
| K | ppm | 1.11 |
| Mg | ppm | 2.44 |
| F | ppm | 0.28 |
| Cl | ppm | 2.30 |
| C | ppm as CO ₂ | 35.6 |
| SO ₄ | ppm | 8.6 |
| pH | Standard units | 9.00 |
| Temperature | Celsius | 4.4 |

Source: BI: Sample SP 01 (Gislason and Eugster 1987 [DIRS 168716], Table 3).

NOTE: ppm = parts per million.

Table 4-5[a]. Two Basalt Water Compositions from the Columbia Basin, Washington

| Parameter | Units | BW | BS |
|-----------------------------------|----------------|--------------------|------|
| pH | Standard units | 8.6 | 8.0 |
| Temperature | Celsius | 25.9 | 17.7 |
| Ca | mg/L | 3.9 | 36 |
| Mg | mg/L | 0.5 | 21 |
| Na | mg/L | 76 | 22 |
| K | mg/L | 8.0 | 6.4 |
| HCO ₃ | mg/L | 166 | 193 |
| SO ₄ | mg/L | 21 | 55 |
| Cl | mg/L | 13 | 17 |
| F | mg/L | 1.9 | 0.4 |
| SiO ₂ | mg/L | 73 | 58 |
| NO ₂ + NO ₃ | mg/L as N | < 0.1 ^a | 2.0 |

Source: BW: Sample19/31E-27G01D1 from Wanapum/Grande Ronde formations (Turney 1986 [DIRS 179852], pp. 49 to 53).

BS: Sample 14/31E-19B01 from Saddle Mountain/Wanapum formations (Turney 1986 [DIRS 179852], pp. 74 to 78).

^a One half of the detection limit is used in the simulations.

4.1.3[a] Waste Forms

One mole of a given waste form is defined to weigh 100 grams. The mass of one mole is by definition 100 grams because the chemical formula for a waste form is defined by the number of moles of each major element in 100 grams of the waste form.

4.1.3.1[a] CSNF

The CSNF composition is the same as in the parent report.

The density of CSNF is 10.08 g/mL (Barner 1985 [DIRS 109194] Table 4.1). Table 4.1 in the reference is a compilation of fuel assembly data supplied by the vendor of the fuels for the Carolina Power and Light Co., H.B. Robinson Pressurized Water Reactor Unit 2. It is qualified as established fact vendor supplied data.

Schoepite is the primary degradation product of CSNF and will control the pH in the CSNF waste form cell under low water conditions. Low water conditions are defined in this addendum as a relative humidity less than 95% and a liquid influx rate into the cell less than 0.1 L/yr. Under these conditions, the pH is approximated based on the pH zero point of charge (pH_{zpc}) for schoepite. The pH_{zpc} of a mineral is indicated by the pH of minimum mineral solubility. Calculations of schoepite solubility as a function of pH at different carbon dioxide fugacities are provided in spreadsheet *U solubility tables.xls* in DTN: MO0707DISVALID.000 [DIRS 182270], which is output of *Dissolved Concentration Limits of Elements with Radioactive Isotopes* (SNL 2007 [DIRS 177418]). Section 6.3.4.1[a] uses these calculations to chart schoepite solubility as a function of pH. Section 6.10.9.1[a] uses them to establish a pH range in the CSNF cell under low water conditions.

4.1.3.2[a] N-Reactor Fuel

No change.

4.1.3.3[a] HLW Glass

The HLW glass composition used in the waste form IPC simulations is derived from the compositions presented in Table 3 of *Projected Glass Composition and Curie content of Canisters from the Savannah River Site (U)* (Allison 2004 [DIRS 168734]). These compositions are presented in Table 4-6[a]. Qualification of these data is provided in Appendix A[a].

The HLW glass degradation rates used in this addendum are from *Defense HLW Glass Degradation Model* (BSC 2004 [DIRS 169988], Section 8.1, Equations 50 and 51, Table 8-1). They are the same rates described in the parent report. The use of these equations is discussed in Section 6.3.1.3.4[a].

Assumption 5.5[a] states that the pH of water on HLW glass at a relative humidity below 95% will have a pH of approximately 10. This pH is based on the results of static leach tests on simulated HLW glass at 90°C that show that leachate has a pH ranging from 8 to 12, depending on the ratio of surface area to volume (Bates et al. 1993 [DIRS 100705], p. 51). Qualification of the data used from Bates et al. (1993 [DIRS 100705]) is provided in Appendix A[a].

Table 4-6[a]. Major Components of Three SRL High-Level Waste Glasses

| Oxide | Mass % Batch 1A | Mass % Batch 1B | Mass % Batch 2 |
|--------------------------------|--------------------|--------------------|-------------------|
| Al ₂ O ₃ | 4.60 | 5.37 | 4.22 |
| B ₂ O ₃ | 8.71 | 8.18 | 7.31 |
| CaO | 1.27 | 1.39 | 1.39 |
| CuO | 0.56 | 0.025 | 0.07 |
| Fe ₂ O ₃ | 12.10 | 10.5 | 12.3 |
| Li ₂ O | 3.68 | 3.53 | 3.29 |
| Na ₂ O | 12.27 | 11.5 | 11.4 |
| NiO | 0.26 | 0.158 | 0.54 |
| MgO | 2.12 | 2.16 | 2.35 |
| MnO | 1.37 | 1.76 | 2.14 |
| P ₂ O ₅ | 0.46 | 0.63 | N/R |
| SiO ₂ | 50.33 | 52.4 | 48.7 |
| U ₃ O ₈ | 1.08 | 1.06 | 3.57 |

Source: Allison 2004 [DIRS 168734], Table 3.

NOTE: N/R = not reported.

4.1.3.4[a] Degradation Rates

The sources of the degradation rates of the waste forms are the same as in the parent report. Specific rates used in this addendum are presented and discussed in Section 6.3.1.3.4[a].

4.1.4[a] Waste Package Material Compositions and Rates

Table 4-7[a] contains the names and shorthand names for the steels and alloys in the waste form cells simulated in this addendum. It also shows which steels and alloys are present in each waste form cell.

Table 4-7[a]. Cell Materials and Nomenclature

| Material Nomenclature | Shorthand | CSNF | 2DHLW | 2MCO |
|--|----------------------------|------|-------|------|
| SA-240 (UNS S31603) Stainless Steel | Stainless Steel Type 316L | X | | |
| SA-240 (UNS S30403) Stainless Steel | Stainless Steel Type 304L | | X | X |
| SS 304B4 (UNS S30464) Borated Stainless Steel | Stainless Steel Type 304B4 | X | | |
| SB-209 6061 (UNS A96061) | Aluminum Alloy Type 6061 | X | | |
| Aluminum Alloy-1100 (UNS 91100) | Aluminum Alloy Type 1100 | | | X |

Source: SNL 2007 [DIRS 179394], Table 4-1; SNL 2007 [DIRS 179567], Table 4-10.

The CSNF waste package contains three nested canisters: (1) an outer corrosion barrier of Alloy 22; (2) an inner vessel of Stainless Steel Type 316L; and (3) the TAD canister, also made of Stainless Steel Type 316L. The interior of the TAD canister contains fuel basket guides and fuel basket tubes made of Stainless Steel Type 316L, fuel baskets made of Stainless Steel Type 304B4, thermal shunts made of Aluminum Alloy Type 6061, and CSNF assemblies containing UO₂ fuel in a zirconium alloy type Zircaloy-4 cladding. Cell 1 of the CSNF waste

package contains all of the materials within the TAD canister with the exception of the Stainless Steel Type 316L fuel basket guides. Thus, the materials in CSNF Cell 1 are Stainless Steel Type 304B4, Aluminum Alloy Type 6061, Stainless Steel Type 316L, and the CSNF and its cladding. The cladding is not included in Table 4-7[a] because it degrades at a negligible rate and therefore does not affect the chemistry of the water (see Section 6.3.1.1 of parent report). It is presumed to split open early in the regulatory period. Presuming early failure of the cladding is justified because it is a conservative simplification.

There are several configurations for the CDSP waste packages that contain the various forms of U.S. Department of Energy (DOE) spent nuclear fuel (DSNF). The configuration modeled in this addendum is the 2MCO/2DHLW waste package with N-Reactor fuel. N-Reactor fuel accounts for the majority of actinide mass of CDSP waste packages, justifying its selection as the representative CDSP waste package. As explained in Section 1, the waste form cells of CDSP waste package are simulated separately. Cell 1a consists of two Stainless Steel Type 304L glass pour canisters with lids and their HLW glass. Cell 1b contains two MCOs, which consist of the outer canister and five interior fuel baskets made of Stainless Steel Type 304L. Each fuel basket contains a fuel guide made of Aluminum Alloy Type 1100 and the N-Reactor uranium metal fuel. The effects of various fuel types and a more common CDSP waste package configuration (5DHLW/1DSNF) on the waste form IPC model abstractions are discussed in Section 6.6.6[a].

4.1.4.1[a] Material Compositions

The compositions of Stainless Steel Types 316L and 304L and Aluminum Alloy Types 6061 and 1100 in the waste form cells are unchanged from the parent document. Stainless Steel Type 304B4 is new. The borated stainless steel (304B4) is substituted for the Ni-Gd Alloy 22. The composition of 304B4 is presented in Table 4-8[a]. The source is Table 1 of ASTM 887-89 [DIRS 154062] and is qualified as established fact.

The chemical formula of each material is arbitrarily defined by the number of moles of each major element in 100 grams of the material. One mole of a given material is then by definition 100 grams.

Table 4-8[a]. Composition of Stainless Steel Type 304B4

| Element | wt % |
|---------|----------------|
| B | 1.00 to 1.24 |
| C | 0.08 |
| N | 0.10 maximum |
| Si | 0.75 |
| P | 0.045 |
| S | 0.030 |
| Cr | 18.00 to 20.00 |
| Mn | 2.00 |
| Fe | Balance |
| Ni | 12.00 to 15.00 |

Source: ASTM A 887-89 [DIRS 154062], Table 1.

4.1.4.2[a] Reaction Rates

Table 4-9[a] lists minimum, maximum, and mid-range corrosion rate measurements for the steels and alloys in the waste form cells. The source of these measurements is file *Data Summary.doc* in DTN: MO0409SPAACRWP.000 [DIRS 172059], from *Aqueous Corrosion Rates for Waste Package Materials* (BSC 2004 [DIRS 169982]). To ensure that the entire potential ranges of rates were considered in the IPC calculations, all experimental conditions documented in the source were included when determining the minimum and maximum values in Table 4-9[a]. The mid-range values are selected median measurements reported for low temperature conditions (“ambient” or ~25°C). The low temperature conditions are used for the mid-range measurements because low temperatures will prevail over the regulatory period. The minimum and maximum rate measurements for a given material differ by two to five orders of magnitude; thus, the mid-range measurements were chosen such that the minimum, maximum, and mid-range values are roughly representative of lognormal distributions. Because it is not clear whether measurements from salt water or fresh water experiments are more appropriate for the application, measurements from all water types were considered.

A more sophisticated approach to selecting degradation rates is unjustified in this addendum. In the current set of waste form IPC abstractions, degradation rates only have a substantial effect on the ionic strength abstraction under liquid influx conditions (Section 6.10.2.1[a]). Adjusting the degradation rates to minimum and maximum values (Section 6.6.5[a]) provides the basis for uncertainty in the liquid influx ionic strength abstraction (Section 6.10.8.2[a]). Degradation rates do not have a major effect on the pH abstraction because the pH abstraction only determines the minimum and maximum pH values. The effect of degradation rates on the pH range is small and indirect because the pH range is only slightly affected by ionic strength (Section 6.10.1[a]).

Table 4-9[a]. Steel and Alloy Corrosion Rate Measurements

| Corrosion Rate Measurement | Aluminum Alloy | Stainless Steel Type 316L | Stainless Steel Type 304L | Stainless Steel Type 304B4 |
|----------------------------|----------------|---------------------------|---------------------------|----------------------------|
| Minimum (µm/yr) | No change | 0.0007 | No change | 2.94 |
| Maximum (µm/yr) | No change | No change | No change | 1058.4 |
| Mid-Range (µm/yr) | 9.5 | 0.7362 | 0.1285 | 20.58 |

Source: DTN: MO0409SPAACRWP.000 [DIRS 172059], file: *Data Summary.doc*.

4.1.4.3[a] Material Properties and Inventories

The surface areas and volumes of the various materials within each waste form cell are estimated from the published dimensions.

The dimensions for the TAD canister in the CSNF waste package come from *TDIP for Requirements Analysis for TAD canister and Related Waste Package Overpack Physical Attributes Basis for Performance Assessment* (SNL 2007 [DIRS 179394], Table 4-1), which was compiled from a series of design drawings (BSC 2005 [DIRS 174147]; BSC 2005 [DIRS 174148]; BSC 2005 [DIRS 174150]; BSC 2005 [DIRS 174151]; BSC 2005 [DIRS 174152]). These drawings specify components of Carbon Steel Type A516 and neutron absorber plates of Ni-Gd Alloy 22. A decision by the Technical Management Review Board specified that these materials will be replaced with Stainless Steel Type 316L and Stainless Steel

Type 304B4, respectively (BSC 2006 [DIRS 178592]). It was assumed that the components would be of the same volume. The masses of each material in each cell were estimated from these data based on the material densities in Table 4-10[a].

For the CDSP waste package, the direct inputs related to the dimensions of the cell components are the same as in the parent report.

Table 4-10[a]. Material Properties and Inventories

| Material | Density (g/cm³) | Source |
|----------------------------|-----------------------------------|---|
| Stainless Steel Type 316L | No change | Not applicable |
| Stainless Steel Type 304L | No change | Not applicable |
| Stainless Steel Type 304B4 | 7.81 | CRS ALLOY Data NeutroSorb PLUS (Carpenter Technology 2003 [DIRS 179642], Table "Physical Properties") |
| Aluminum Alloy 1100 | No change | Not applicable |
| Aluminum Alloy 6061 | No change | Not applicable |

4.1.5[a] Atomic Weights

No change.

4.1.6[a] Surface Complexation Inputs

Not used in addendum.

4.2[a] CRITERIA

No change.

4.3[a] CODES, STANDARDS, AND REGULATIONS

Standard used: ASTM A 887-89 [DIRS 154062], *Standard Specification for Borated Stainless Steel Plate, Sheet, and Strip for Nuclear Application*.

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5[a]. ASSUMPTIONS

5.1[a] BULK WATER CHEMISTRY

No change.

5.2[a] CONSTANT CORROSION OF ALLOYS

No change.

5.3[a] THERMODYNAMIC EQUILIBRIUM

No change.

5.4[a] WASTE PACKAGE SIMULATION CONCEPT

Assumption: The total waste package can be simulated as two cells that represent different sets of dominant geochemical processes. Transport of radionuclides out of a waste package is simulated in TSPA-LA as transport from a waste form cell (Cell 1) through a cell of corrosion products (Cell 2). The division of the waste package into these cells is based on the geochemical processes that dominate within each cell. In Cell 1, the dominant processes are assumed to be degradation of the waste form, corrosion of Cell 1 materials, and equilibration with Cell 1 degradation products and gas fugacities. Adsorption is assumed to be a negligible process in Cell 1. In Cell 2, which is farther away from the waste form, the dominant processes are assumed to be corrosion of the Cell 2 materials, adsorption of dissolved constituents, and equilibration with Cell 2 degradation products and gas fugacities. The waste form IPC model only simulates the geochemical processes in Cell 1. Cell 2 processes are addressed in *EBS Radionuclide Transport Abstraction* (SNL 2007 [DIRS 177407]).

Within CSNF Cell 1, the primary geochemical processes are corrosion of steels and alloys and degradation of waste forms. Degradation of each of these materials contributes individual constituents into solution or into solid degradation product phases. Within CSNF Cell 2, the materials are limited to the steels and alloys and their corrosion products, so in addition to material degradation and equilibration with degradation products and gas fugacities, adsorption of dissolved constituents onto corrosion product surfaces is modeled. The fuel assemblies, the fuel basket, and the fuel basket tubes are in Cell 1, and the basket guides, the TAD canister, the inner vessel, and the outer corrosion barrier are in Cell 2.

Similarly, CDSP Cell 1 consists of the HLW glass forms and its Stainless Steel Type 304L glass-pour canisters (Cell 1a) and the N-Reactor uranium metal fuel and its associated MCOs (Cell 1b). These waste form cells are modeled independently because they are physically separated in the waste package. Cell 2 in the CDSP contains the Carbon Steel Type A-516 divider plates and support plate assemblies, the Stainless Steel Type 316L inner vessel, and the Alloy 22 outer corrosion barrier.

Rationale: Source aqueous concentrations of radionuclides depend largely on the composition of water in contact with the materials and alteration products near the waste form. Water located in Cell 1 will have an opportunity to react and equilibrate with the waste forms, steels, and alloys

that make up the structural components of Cell 1. Materials in Cell 2 are farther removed from the waste form source and therefore will have a limited effect on water composition near the waste form. Adsorption in Cell 1 is not simulated because the amount of iron corrosion products is low in Cell 1 compared to Cell 2. In addition, preventing adsorption in Cell 1 is conservative because adsorption decreases the mobility of radionuclides.

The two-cell concept is still valid after the waste package components lose their structural integrity. The geochemical processes simulated in each cell have little bearing on geometry or scale. Even in a seismic event that causes internals to degrade to a mass of granular material, the same processes will occur. Therefore, the IPC model can still be used to predict the pH, ionic strength, and composition of water in the vicinity of the waste form (Cell 1), and transport of radionuclides away from Cell 1 will still involve the processes simulated in Cell 2.

Confirmation Status: This conceptual model is consistent with the TSPA-LA conceptual model. The alternative waste package conceptual model, simulated in the parent report, is not consistent with the TSPA-LA model. As explained above, Cell 1 processes will continue even if the structure of the waste package disintegrates, and transport of radionuclides away from Cell 1 will still involve the processes simulated in Cell 2. Regarding the independency of abstractions developed for Cells 1a and 1b in the CDSP waste package, sensitivity analyses in Section 6.6[a] and evaluation of abstractions in Section 6.10[a] indicate that the composition of liquid influx has a limited effect on abstractions. The pH ranges are unaffected by liquid influx compositions, as discussed in Section 6.10.1.1[a], and ionic strength is either unaffected if the RH controls the ionic strength (Section 6.10.2.2[a]) or is captured within the range of uncertainty if ionic strength is a function of flow rate and time (Section 6.10.8.2[a]). No further confirmation is required.

Use in the Model: This assumption is used throughout this addendum.

5.5[a] THE pH OF WATER FILMS ON HLW GLASS SURFACES UNDER LOW WATER CONDITIONS

Assumption: The pH of water films on HLW glass under low water conditions is 10. Low water conditions are defined as relative humidity below 95% and a liquid influx rate of less than 0.1 L/yr. Under these conditions, the primary sources of water are vapor condensation and adsorption of water vapor.

Rationale: Static leach tests on simulated HLW glass at 90°C show that leachate has a pH ranging from 8 to 12 depending on the ratio of surface area to volume (Bates et al. 1993 [DIRS 100705], p. 51). The alkalinity is caused by the release of sodium ion (Na^+) and the generation of sodium hydroxide (NaOH) from water. The higher pH values are obtained at higher ratios of surface area to volume. A high value of this ratio is likely not sustainable in the waste form cell due to precipitation of degradation products that tend to fill fractures in the glass. A pH value of 10 is assumed because it is midway between the upper and lower ends of the range measured in the static leach tests and is in the range of the maximum pH values calculated in the IPC abstraction for the HLW glass cell (CDSP Cell 1a) (Figure 6-42[a]).

Confirmation Status: This assumption is consistent with an assumption used in the TSPA-LA model for a different type of low water condition. At the air pressures predicted within the

repository (approximately atmospheric) and temperatures above 100°C, liquid water will boil and HLW glass surfaces will essentially be dry. Under these conditions, the TSPA-LA model calculates a HLW glass degradation rate assuming a proxy pH of 10. The assumption is conservative because TSPA-LA uses it to predict a high rate of glass degradation under low water conditions. No further confirmation is required.

Use in the Model: This assumption is used in Section 6.10.9.1[a].

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6[a]. MODEL DISCUSSION

6.1[a] OBJECTIVES

The objectives of the work in this addendum are to develop and use the waste form IPC model to predict the bulk water chemistry within waste form cells after waste package breach and to provide abstractions of pH, ionic strength, and fluoride concentration. TSPA-LA uses the abstracted pH and ionic strength values in conjunction with other models to determine the stability of colloids, the solubility of radionuclides, and the degradation rates of the waste forms. The fluoride abstraction is used by *Dissolved Concentration Limits of Elements with Radioactive Isotopes* (SNL 2007 [DIRS 177418]) to ensure predicted radionuclide solubilities are valid over the range of fluoride concentrations possible in the waste form cells.

6.2[a] FEATURES, EVENTS, AND PROCESSES

No change.

6.3[a] CONCEPTUAL MODEL

The conceptual model for this effort is similar to the previously developed conceptual model. It is a mixing cell model wherein waste package components are reacted with water and atmospheric gases over time after waste package breach. The primary difference is the control volume. In this addendum, the control volumes are individual waste form cells (defined in Section 1[a]) instead of entire waste packages as in the previously developed model. To distinguish this model from the previous model, the model developed in this addendum is called the waste form IPC model. Changing the control volumes to the individual waste form cells is based on the assumption that it is reasonable and viable to model each of these cells individually (Assumption 5.4[a]).

In the waste form IPC conceptual model, cell materials degrade over time while equilibrium is maintained between aqueous complexes, atmospheric gases, and secondary mineral phases. As in the IPC model of the parent document there are two cases for water entry into the cells: vapor influx and liquid influx. Vapor influx occurs when the primary source of water in the cell is water vapor from outside the waste package. In this case, there is no advection of liquid or vapor out of the cell. Liquid influx occurs when liquid water from condensation or seepage flows into the cell. In the liquid influx case, equilibrated liquid flows out of the cell. The surface complexation model developed previously is not used in the waste form IPC model because there is a much lower abundance of iron alteration products in the waste form cells.

In all, the waste form IPC model simulates the following processes while conserving mass:

- Rate-limited degradation of waste package materials and waste forms over time
- Equilibrium aqueous complexation and oxidation/reduction reactions
- Equilibrium precipitation and dissolution of secondary mineral phases
- Equilibrium exchange of CO₂ and O₂ between the aqueous and gas phases

- Liquid influx of seepage or condensed water into the cell and liquid outflux of equilibrated liquid
- Vapor influx.

The EQ3/6 code is implemented in nearly the same way as before when simulating the processes over time. The only difference is that a different method is used for implementing vapor influx. The vapor influx process model departs from the previous process model in that vapor diffusion is not considered to be kinetically limited. The driving force is the same (water vapor from the drift diffuses into the cell where the vapor pressure is lower), but instead of defining and using an associated diffusion rate, the rate is assumed to be rapid. The vapor influx model is important because water films generated by vapor influx promote radionuclide diffusion, which is simulated in the TSPA-LA model using a diffusion model developed in *EBS Radionuclide Transport Abstraction* (SNL 2007 [DIRS 177407]).

The process model for vapor influx was changed because the osmotic potential is a primary driving force for the accumulation of condensate. Dissolved ions from material degradation reactions (e.g., Ni^{2+} , Na^+ , Mg^{2+} , SO_4^{2-} , and NO_3^-) reduce the chemical activity of water in solution. At equilibrium, the chemical activity of water in solution is equivalent to the relative humidity, expressed as a fraction (Kinsman 1976 [DIRS 100769], p. 274). Thus, given a fixed relative humidity, there will be a net movement of water molecules between vapor and liquid phases to produce an equilibrium solution with an osmotic pressure corresponding to the relative humidity. By using relative humidity to control the amount of vapor influx, there is an equilibrium endpoint that will provide the thermodynamically favored ionic strength. Assuming equilibrium with the relative humidity of the waste package is reasonable because relative humidity in the drift and waste package does not fluctuate substantially over short periods of time.

Implementation of the vapor influx and liquid influx process models is discussed in Section 6.3.1.1[a] under “Code Implementation.”

6.3.1[a] Batch Reactor Model

6.3.1.1[a] Batch Reactor Modeling Concepts and Assumptions

Thermodynamic Equilibrium – No change. The statement regarding surface complexation applies only to the IPC model of the parent document.

CO₂ and O₂ Partial Pressure – No change.

Evaporation – As in the parent report, evaporation is not simulated in the liquid influx process model. However, the new vapor influx model implicitly accounts for evaporation by allowing the relative humidity to be the primary control on ionic strength (Section 6.10.2.2[a]).

There are several reasons evaporation is not included in the liquid influx model. First, evaporation is a difficult process to integrate into the TSPA-LA model. Evaporation is limited by vapor pressure gradients and the latent heat available for vaporization. Feedback mechanisms are difficult to implement in the TSPA-LA model such as the increase in vapor pressure inside

the cell from evaporation and the resulting effect on the flux rates of vapor and liquid into and out of the cell. Second, except for a small ionic strength effect on the pH range, evaporation does not affect the pH ranges of the abstractions because the pH ranges are primarily controlled by precipitated secondary minerals (Section 6.10.1[a]). Third, if liquid is flowing into the cell, the relative humidity in the drift and in the cell will be near 100%. Evaporation decreases to zero as relative humidity increases to 100%. In addition, excluding evaporation tends to be conservative. Higher volumes of water in the cell, caused by excluding evaporation, reduce ionic strength, which increases colloid concentrations of radionuclides. Excluding evaporation in the liquid influx model is also conservative because excluding it would increase the liquid flux out of the cell. As liquid flux out increases, the advective flux of dissolved and colloidal radionuclides out of the cell increases (the source term radionuclide concentrations, which are based on equilibrium solubilities, are unaffected by the liquid flux rate). These conservative effects can be partially offset by other effects because lower ionic strength caused by excluding evaporation also affects abstracted pH ranges and radionuclide solubilities. However, when a commensurate degree of evaporation is compared, these additional effects, which are sometimes conservative, are small or negligible compared to the effects on colloid concentrations and volumetric liquid fluxes.

Well-Mixed Batch Reactor – The reactants in the batch reactor are well mixed. The well-mixed concept is justified because the cell components are in proximity to one another, and an important aspect of the model is to investigate the effects of the interactions of the various components. The waste package will be cooling slowly over time, so the temperature gradients within the cell will contribute to convective flow that will mix the contents of the cell. Mixing by diffusion will also be substantial, especially when the turnover rates of liquid in the cell are low (liquid influx case) or there is no turnover at all (vapor influx case).

The two CDSP waste form cells are modeled separately because they are physically separated and their initial distribution within the waste package is highly heterogeneous. After the waste form canisters fail, any interactions between waste forms in the CDSP waste package would tend to reduce the predicted pH ranges and moderate the ionic strength. For example, the high pH values achieved in Cell 1a from the glass waste form would not be achievable if schoepite, the primary Cell 1b degradation product, is also present. Because interactions tend to moderate model outputs and are limited by the initial physical separation of the waste forms, interactions are not simulated.

Material Volumes and Surface Areas – Material volumes in the waste form cells are provided by or calculated from design documents based on the cell definitions in Section 1[a].

The surface areas of the CSNF and N-Reactor fuel are treated as the geometric surface area because the rates reported in Sections 4.1.3.1[a] and 4.1.3.2[a] are given in terms of geometric surface area. Similarly, the measured rates of the steels and alloys are based on geometric surface areas internal to the waste form cells. The surface area of the HLW glass is the geometric surface area multiplied by the most probable glass exposure factor of 4 (BSC 2004 [DIRS 169988]), as justified in Section 6.3.1.3.4[a].

Degradation rates in a cell or control volume are directly proportional to the geometric surface area in the cell or control volume. Degradation rates used in this addendum factor in the surface areas, which explains why the values calculated and used in this addendum are simply in

units of moles per time (e.g., Sections 6.3.1.3.3[a] and 6.3.1.3.4[a]). A sensitivity study in Section 6.6.5[a] addresses the relationship between specific surface area and degradation rates.

Void Volume – A portion of the waste form cell volume is occupied by the waste form and structural components. The rest of the cell volume is referred to as the void volume and can fill with water. Although the void volume will likely decrease over time as materials degrade to alteration products that occupy more space due to oxidation and hydration, this change is not explicitly included in the IPC model. The void volume in each cell is in contact with all materials in the cell and is spread throughout the cell. This spatial configuration is consistent with a well-mixed cell model.

The potential for degradation products to fill or partially fill void space in the cell is not explicitly included in the IPC model because it either enhances waste isolation or its effects are already captured in the IPC simulations. Filling void spaces in a waste form cell could enhance waste isolation by reducing exposure of the waste form to water and oxygen. In this way, using a fixed void volume that cannot be filled with degradation products is conservative. Alternatively, filling void spaces can cause a reduction of the volume of water in the cell, which can affect IPC model calculations by decreasing the ratio of water to reactants. However, as explained in Section 6.6.1[a], this effect is small.

Volume of Water in Cell – For the liquid influx calculations presented in this addendum, the amount of water in the cell is defined to be half of the void volume. Because the void volume is held constant (above), the volume of water is held constant in the cell. The effect of the volume of water on IPC abstractions is addressed in Section 6.6.1[a].

In the liquid influx case, when the waste package breaches, the full quantity of water required to fill half of the void volume is instantly present. In actuality, after the waste package breaches, water would enter the waste package and take a certain amount of time to accumulate. This amount of time for accumulation is difficult to quantify and would only result in a delay in radionuclide transport, so for the purposes of this model, the water is instantly available.

In the vapor influx case, there is no established volume of water in the cell because the volume of water in the cell does not affect the vapor influx waste form IPC abstractions. However, to produce the abstractions, simulations have to begin with an initial water volume, so a volume of half of the void volume (as in the liquid influx case) was used. Implementation of the vapor influx submodel is discussed in more detail later in this section.

Scaling to One Liter – The waste form cell is represented by amounts, compositions, and degradation rates of the major structural materials and waste forms within the cell. Each material is assigned a degradation rate that is a function of the surface area of the material in the cell. The water flux rate is calculated to provide a representative quantity of water to interact with the cell components at a rate corresponding to the rate at which water could enter the cell.

EQ6 is designed to perform reaction path and flow-through calculations on one liter of water. Consequently, the waste form cells were scaled to one liter of water for the liquid influx case. This was accomplished in the liquid influx case by dividing the total amounts of waste forms and materials in a cell by a scaling factor equal to the total volume of water in the cell. The degradation rates were also reduced proportionately by reducing the total surface areas of the

materials by the same scaling factors. In addition, water influx rates were reduced by the scaling factors to maintain a constant turnover rate.

In the vapor influx case, the volume of liquid water in the cell is variable and is a function only of the cumulative amount of degradation. Implementation of the vapor influx submodel is discussed in more detail later in this section.

Waste Form Cell Materials Not Included in the EQ6 Input File – The Zircaloy cladding is not included as explained in Section 4.1.4[a].

Codisposal Waste Packages – No change.

Corrosion – No change.

Code Implementation – The EQ6 code simulates a titration in which reactants are added to the cell according to their kinetic reaction rates. Implementation of the vapor influx and liquid influx cases differs only in how water is added to and removed from the cells.

For the liquid-influx case, water is entered as a “displacer” reactant to implement the solid-centered flow-through capability of EQ6. The displacer is entered such that a “mole” of the incoming seepage or condensate is equivalent to one liter. Thus, a displacer rate of 3.17×10^{-8} “mole” per second is equivalent to 1 L/yr. Because there is one liter of water in the cell, a displacer rate of 1 L/yr implies a mean residence time of 1 year (i.e., a turnover rate of 1 year^{-1}). During a given EQ6 time step, which is always much less than the mean residence time in EQ6 simulations, a portion of the equilibrated fluid in the cell is displaced by an equal volume of seepage or condensate entering the cell. The displaced fluid and its dissolved components constitute the advective flux out of the cell.

The vapor influx case uses the relative humidity in the waste package to determine the volume and composition of the equilibrated solution in the cell. At equilibrium, the relative humidity, expressed as a fraction, is equivalent to the activity of water (Kinsman 1976 [DIRS 100769], p. 274). By assuming condensed vapor in the cell is at equilibrium with the local relative humidity, no vapor influx rates are required. As a result, the only variables are the degradation rates.

For the vapor influx case, the simulations are divided into two relative humidity ranges. The high range, corresponding to an ionic strength of one molal or below, is simulated using the EQ6 batch reactor and the B-dot equation for the calculation of activity coefficients (discussed later this section). This range is simulated by degrading small amounts of cell materials (proportional to their relative reaction rates) into one liter of water and then incrementally removing H₂O from the reactor. Water (as pure H₂O) is removed independent of time, so additional degradation is not allowed during H₂O removal. As H₂O is removed, the activity of water decreases and the ionic strength increases. The simulations end when the ionic strength reaches one molal. Regardless of the amount of degradation allowed in the initial liter of water, the same water compositions are obtained for a given equilibrium activity of water.

Above one molal ionic strength, EQ6 simulations of the IPC model are not performed. Instead, the IPC vapor influx case relies on calculations from a separate report to generate the high ionic strength IPC abstractions (Section 6.10.2.2[a]).

Applicability of the B-dot Equation – The B-dot equation is defined and carefully examined in *Dissolved Concentration Limits of Elements with Radioactive Isotopes* (SNL 2007 [DIRS 177418], Section 6.3.3). For the purposes of the waste form IPC model, EQ3/6 results generated using the B-dot activity coefficient equation for solutions with ionic strength up to 1 molal are sufficiently accurate for the intent of this application (mainly to estimate pH). Experimental data (in sulfate, nitrate, and chloride solutions) show that EQ3/6 results using the B-dot activity coefficient equation can be used qualitatively up to an ionic strength of about 4 molal to indicate the general nature of the reactions that would actually occur (CRWMS M&O 1997 [DIRS 100222], Appendix D). All IPC ionic strength calculations are true ionic strengths from model output, i.e., they are calculated from the complete set of aqueous species at equilibrium.

Thermodynamic Database – The *data0.ymp.R5* database (DTN: SN0612T0502404.014 [DIRS 178850]) was used in a modified form in all simulations in this addendum. First, to prevent chromium from oxidizing to anything but Cr(III), non-Cr(III) species and minerals were removed from the database or inactivated using comment flags. Preventing the formation of all but Cr(III) species is justified in Section 6.6.3 of the parent report. Second, phases representing CSNF and HLW glass were added based on the compositions defined in Section 6.3.1.3.4[a]. Finally, a number of gases had to be removed from the database to obtain a binary working file that is compatible with the versions of EQ3/6 (versions 7.2b and 7.2bLV) used in this addendum. The gases chosen for removal were those with metal components. These gases have no effect on the IPC abstractions. The modified thermodynamic database and its binary file are *data0.R5f* and *data1.R5f* and are provided in Output DTN: SN0702PAIPC1CA.001 in compressed file *IPC_IO_files.zip*.

6.3.1.2[a] Batch Reactor Base Case Simulations

The base case simulations for the vapor influx and liquid influx simulations are defined by the input parameter values listed in Table 6-1[a]. The base case values are mid-range values with high probability relative to minimum and maximum values for these parameters. Consequently, simulations performed using the base case values are expected to produce mid-range IPC outputs. For ionic strength abstractions, base case simulations are used to predict the median log value of ionic strength over time under liquid influx conditions (Section 6.10.2.1[a]). Thus, abstracted ionic strength values are a function of the base case values used. For pH, because only the minimum and maximum values of pH are abstracted, there is little sensitivity in the pH abstractions to base case values (Section 6.10.1[a]). The fluoride abstraction also has little sensitivity to base case values because only extreme values of ionic strength, resulting from sensitivity analyses in which non-base case input values were used, affect the fluoride abstraction (Section 6.10.3[a]).

Values of the parameters in Table 6-1[a] are varied across their possible ranges in the sensitivity analyses presented in Section 6.6[a]. The results of these sensitivity analyses are used to approximate the uncertainty in the liquid influx ionic strength abstractions (Section 6.10.8.2[a]).

and the maximum fluoride concentrations (Section 6.10.3[a]). They are also used to verify that the pH remains within the ranges specified in the pH abstractions (Section 6.10.8.1[a]). Base case input and output files are documented in Output DTN: SN0702PAIPC1CA.001 as described in file *IPC file key revision 2.xls* in the DTN.

Table 6-1[a]. Base Case Input Parameter Values for Vapor Influx and Liquid Influx Simulations

| Input Parameter | Vapor Influx | Liquid Influx |
|---|----------------------------------|----------------------------------|
| Partial Pressure of Oxygen | 0.2 bar | 0.2 bar |
| Partial Pressure of Carbon Dioxide | 0.001 bar | 0.001 bar |
| Temperature | 25°C | 25°C |
| Water Composition of Influx | Pure water | SD9 (Table 4-2[a]) |
| Influx Rate | Not applicable | 1 L/yr |
| Percentage of Void Volume Filled with Water | Not applicable | 50% |
| Volume of Liquid Water in Cell (scaled) | Not applicable | 1 L |
| Alloy and Steel Corrosion Rates (scaled) | Base Case Values in Table 6-5[a] | Base Case Values in Table 6-5[a] |
| Waste Form Degradation Rates (scaled) | Base Case Values in Table 6-7[a] | Base Case Values in Table 6-7[a] |
| Cladding Failure | 100% | 100% |
| Material Amounts and Surface Areas | Table 6-2[a] | Table 6-2[a] |
| Mineral Controls | Section 6.3.1.3.6[a] | Section 6.3.1.3.6[a] |

6.3.1.3[a] Batch Reactor Model Inputs

This section documents the values of inputs used in batch reactor simulations and other calculations documented in this addendum. Occasionally, values in the simulations differ from the sources or spreadsheet calculations due to transcription errors, unincorporated updates, or rounding. Because the ranges and uncertainties in these inputs are large, usually covering orders of magnitude, and the differences are small, usually less than 5%, these differences have a negligible effect on waste form IPC abstractions. Therefore, only differences greater than 5% will be identified and addressed in this addendum.

6.3.1.3.1[a] Total Surface Areas and Moles of Cell Materials

The total calculated moles and surface areas of materials in the waste form cells are presented in Table 6-2[a]. These calculations are based on the definitions of the waste form cells (Section 1[a]) and the dimensions, densities, and molecular weights given in Sections 4.1.3[a], 4.1.4.3[a], and 4.1.5[a]. A most probable fracture factor of 4 is used to calculate the surface area of the HLW glass.

Table 6-2[a]. Total Moles and Surface Areas of Materials in Waste Form Cells

| Waste Form Cell | Material | Moles | Surface Area (cm ²) |
|-----------------|----------------------------------|----------|---------------------------------|
| CSNF | Aluminum Alloy 6061 (SB-209) | 6.21E+03 | 5.79E+05 |
| | Stainless Steel Type 316 | 6.18E+04 | 1.95E+06 |
| | Stainless Steel Type 304B4 | 3.82E+04 | 8.81E+05 |
| | CSNF | 1.10E+05 | 5.03E+06 |
| 2DHLW | Stainless Steel Type 304L | 1.49E+04 | 3.57E+05 |
| | HLW Glass (exposure factor of 4) | 6.91E+04 | 6.80E+05 |
| 2MCO | Aluminum Alloy 1100 | 1.30E+03 | 8.55E+04 |
| | Stainless Steel Type 304L | 4.72E+04 | 3.90E+05 ^a |
| | N-Reactor fuel | 1.37E+05 | 9.58E+06 |

^a The surface area of 304L in the 2MCO cell is incorrect in the source cited below. However, the correct value presented here and calculated in file *CDSP - 2MCO Cell 1.xls* of Output DTN: SN0702PAIPC1CA.001 is carried into the IPC simulations. This value is only used to calculate the 304L degradation rates in the 2MCO cell. The degradation rates used in the IPC simulations are presented and discussed in Section 6.3.1.3.3[a].

NOTE: These values are summarized in *Rates CSNF Cell 1.xls*, *Rates 2DHLW cell.xls*, and *Rates 2MCO Cell 1b.xls* in Output DTN: SN0702PAIPC1CA.001. They are calculated in *CSNF WP TAD Design Cell 1.xls* and *CDSP - 2MCO Cell 1.xls* of the same DTN.

6.3.1.3.2[a] Solid, Void, and Liquid Volumes

The total solid volume of the materials in a cell is calculated either from the dimensions of the materials or from the densities and total masses of materials in the cell. The dimensions and/or masses of materials in the waste form cells are obtained from appropriate design-related sources (SNL 2007 [DIRS 179394], Table 4-1; SNL 2007 [DIRS 179567], Table 4-10). The void volume is the void space within the outer dimensions of the cell. It is calculated by subtracting the total solid volume from the volume defined by the outer dimensions. These volumes are summarized in Table 6-3[a].

The liquid volume is the volume of liquid in the void volume. For the liquid influx base case, the void space is half filled with water; thus, the liquid volume is half of the void volume. Because flow-through EQ6 calculations require one liter of water, the liquid volumes are equivalent to the scaling factors; i.e., the displacer rates and material volumes and degradation rates are divided by the total liquid volumes to obtain rates that are scaled to one liter of water.

Table 6-3[a]. Total Solid, Void, and Base Case Liquid-Influx Liquid Volumes in Waste Form Cells

| Waste Form Cell | Total Solid Volume (L) | Total Void Volume (L) | Total Base Case Liquid-Influx Liquid Volume (L) (Base Case Liquid-Influx Scaling Factor) |
|-----------------|------------------------|-----------------------|--|
| CSNF | 2,589 | 4,737 | 2,369 |
| 2DHLW | 2,256 | 309 | 154 |
| 2MCO | 1,369 | 793 | 396 |

NOTE: These values are calculated or summarized in *Rates CSNF Cell 1.xls*, *Rates 2DHLW cell.xls*, and *Rates 2MCO Cell 1b.xls* in Output DTN: SN0702PAIPC1CA.001. Supporting calculations are provided in *CSNF WP TAD Design Cell 1.xls* and *CDSP - 2MCO Cell 1.xls* of the same DTN.

6.3.1.3.3[a] Metal Alloy Compositions and Corrosion Rates

The original metal alloy compositions relevant to this addendum are unchanged. An additional alloy is the Stainless Steel Type 304B4 . Its composition is calculated in *304B4 Comp and Reaction.xls* in Output DTN: SN0702PAIPC1CA.001 using the input data from Table 4-8[a]. Approximate maximum concentrations of the various elements in Table 4-8[a] were used.

Generation and heat treatment of 304B4 produces a borocarbide in the form of $(Cr,Fe)_{23}(B,C)_6$ where the ratio of Cr to Fe is approximately 2 (Moreno et al. 2004 [DIRS 179295]). This phase is not included in the thermodynamic database, so the corrosion reaction was modified to account for this reaction. The reaction does not release an appreciable amount of boron or carbon to solution, so all boron and carbon remain in the borocarbide. A stoichiometrically equivalent amount of Cr and Fe are also incorporated in the borocarbide at a 2:1 ratio of Cr to Fe.

As in the other metal alloys, one mole of 304B4 is defined to be 100 grams. The mass of one mole is by definition 100 grams because the chemical formula for 304B4 is defined by the number of moles of each major element in 100 grams of 304B4. The first column of numbers in Table 6-4[a] lists the calculated moles of the various elements in one mole of 304B4 assuming the maximum quantities of these elements in Table 4-8[a]. Accounting for the low- solubility borocarbide formation, none of the boron or carbon and not all of the iron and chromium are released to solution when one mole of 304B4 corrodes. This is accounted for in the second column of numbers in Table 6-4[a]. The values in the final column in Table 6-4[a] are the actual values used in the EQ6 simulations for the overall corrosion reaction with borocarbide formation. Based on the calculations in column 2, these values overstate the release of elements to solution by approximately 36%. However, this discrepancy has a negligible effect on the IPC abstractions because it is small compared to the range of reaction rates for 304B4 (Table 6-5[a]).

The base case, minimum, and maximum corrosion rates used in the liquid influx EQ6 calculations are presented in Table 6-5[a]. These rates are derived from the rates in Table 4-9[a] and are scaled to one liter of water and factor in the total surface areas in contact with one liter of water. The mid-range rates in Table 4-9[a] are used as the base case rates. For the 2DHLW cell, the maximum corrosion rate of 304L was reduced from the maximum rate in Table 4-9[a] by a factor of 4 to prevent consumption of all water in the cell. The maximum corrosion rate is limited by the amount of water available for the corrosion reaction.

For the vapor influx simulations, corrosion rates from Table 6-5[a] are used without scaling to one liter of water. This is appropriate because vapor influx simulations are not time-dependent and do not simulate flow through. The absolute values in Table 6-5[a] are only important to the vapor influx case in that they indicate the relative corrosion rates.

Table 6-4[a]. Molar Composition of Stainless Steel Type 304B4 and Molar Releases Due to Chromium-Iron Borocarbide Formation

| Element | Moles per 100 grams 304B4 | Moles Released to Solution per 100 grams 304B4 Due to Borocarbide Formation | Simulated Moles Released to Solution per 100 grams 304B4 |
|---------|---------------------------|---|--|
| B | 1.17E-01 | 0.00E+00 | 0.00E+00 |
| C | 6.66E-03 | 0.00E+00 | 0.00E+00 |
| N | 7.14E-03 | 7.14E-03 | 9.74E-03 |
| Si | 2.67E-02 | 2.67E-02 | 3.64E-02 |
| P | 1.45E-03 | 1.45E-03 | 1.98E-03 |
| S | 9.36E-04 | 9.36E-04 | 1.28E-03 |
| Cr | 3.65E-01 | 4.85E-02 | 6.61E-02 |
| Mn | 3.64E-02 | 3.64E-02 | 4.97E-02 |
| Fe | 1.13E+00 | 9.74E-01 | 1.33E+00 |
| Ni | 2.30E-01 | 2.30E-01 | 3.14E-01 |

Source: Output DTN: SN0702PAIPC1CA.001, file: 304B4 Comp and Reaction.xls.

Table 6-5[a]. Steel and Alloy Corrosion Rates Scaled to One Liter of Water for Liquid Influx Base Case

| Corrosion Rate | Aluminum Alloy 6061 (CSNF Cell) | Stainless Steel Type 316L (CSNF Cell) | Stainless Steel Type 304B4 (CSNF Cell) | Stainless Steel Type 304L (2DHLW Cell) | Aluminum Alloy 1100 (2MCO Cell) | Stainless Steel Type 304L (2MCO Cell) ^a |
|---------------------|---------------------------------|---------------------------------------|--|--|---------------------------------|--|
| Minimum (mole/s) | 8.36E-12 | 1.46E-13 | 2.71E-10 | 5.83E-13 | 7.39E-12 | 2.48E-13 |
| Maximum (mole/s) | 2.32E-09 | 3.08E-09 | 9.74E-08 | 2.28E-08 ^b | 2.05E-09 | 1.69E-10 ^b |
| Base Case (mole/s) | 1.99E-10 | 1.53E-10 | 1.89E-09 | 7.50E-11 | 1.75E-10 | 3.18E-11 |
| Minimum (mole/yr) | 2.64E-04 | 4.60E-06 | 8.54E-03 | 1.84E-05 | 2.33E-04 | 7.82E-06 |
| Maximum (mole/yr) | 7.32E-02 | 9.72E-02 | 3.07E+00 | 7.20E-01 ^b | 6.46E-02 | 5.33E-03 ^b |
| Base Case (mole/yr) | 6.27E-03 | 4.84E-03 | 5.98E-02 | 2.37E-03 | 5.54E-03 | 1.00E-03 |

^a The minimum and base case Stainless Steel Type 304L corrosion rates used in the 2MCO simulations are approximately 5.6% higher than the values shown here. This difference is insignificant considering that the range of measured 304L corrosion rates spans more than four orders of magnitude (Table 4-9[a]).

^b The maximum corrosion rate used in this simulation was reduced as indicated to prevent the reaction from consuming too much water in the cell. The reaction rate is constrained by the availability of water and by the valid range of ionic strength calculations. The maximum value was determined iteratively using EQ6.

NOTE: Except for footnoted maximum values, the values in this table are calculated from the following equation:

$$r = \frac{cA\rho}{V_w M_w}$$

where c is the corrosion rate from Table 4-9[a], A is the surface area from Table 6-2[a], ρ is the density from Table 4-10[a], V_w is the volume of water from Table 6-3[a], and M_w is the molecular weight, which is defined to be 100 g/mole for each of these materials. Unit conversions are required.

6.3.1.3.4[a] SNF and HLW Glass Compositions and Reaction Rates

The only waste form composition that changed from the parent report was that of the SRL HLW glass. The new composition used in this addendum is presented in Table 6-6[a] based on

information in Section 4.1.3.3[a]. The stoichiometry of one mole of HLW glass, defined as 100 grams, is presented in Table 6-6[a]. It is the average composition of the analyses in Table 4-6[a] converted to units of moles per 100 grams. File *CDSP_HLWGlass_Sensitivity_r2.xls* of compressed file *IPC_PP_files_rev2.zip* of Output DTN: SN0702PAIPC1CA.001 contains the calculation of the degradation reaction of HLW glass included in the thermodynamic database.

The same sources of reaction rate inputs for the waste forms in the waste package IPC model are used in this addendum. The actual values used are slightly different but still within the same ranges. They are presented in Table 6-7[a].

The CSNF reaction rates are plotted in Figure 6-1[a] based on the equations in the original source (Section 4.1.3.1 of the parent report). The discontinuities at pH 6.8 are due to the fitting functions used in the source. The values marked by large solid squares on the 25°C lines are used to represent the minimum, maximum, and base case rates. These constant rates were used in place of the rate equations to ensure that minimum, maximum, and base case rates were maintained during the simulations. Because the pH range is generally 5 to 9 for the CSNF cell (Section 6.10.1[a]), the extremes are well represented by the minimum and maximum values chosen.

This approach was also taken to select the minimum, maximum, and base case rates for HLW glass. Figure 6-2[a] shows the relationship of pH to the glass degradation rate for a glass exposure factor of 4. The lines in this figure are summations of the rate equations for acidic and alkaline conditions as defined in *Defense HLW Glass Degradation Model* (BSC 2004 [DIRS 169988], Section 8.1). The dashed sections of the lines in the figure are unlikely because high degradation rates produce alkaline conditions in the cell and low degradation rates result in slightly acidic conditions in the cell.

The glass exposure factor is a multiplier used to calculate the surface area of the glass. The distribution of this factor is triangular with a minimum and most probable value of 4 and a maximum value of 17 (BSC 2004 [DIRS 169988], Table 8-1). A value of 4 is used in this addendum because (1) values near 4 have a much higher probability than values near 17 and (2) the factor difference between the most probable and maximum values is only about one quarter of the factor difference between the base case and maximum degradation rates used (Table 6-7[a]).

If the pH is known, the probability distribution for the degradation rate of HLW glass can be determined from the equations in the source document. However, because pH is not known and is affected by the degradation rate, an iterative process involving the IPC model would be required to determine the degradation rate probability distribution. In addition, because pH is a function of other input variables, such as the fugacity of carbon dioxide, flow rate, and incoming water composition, obtaining probability distributions for the HLW glass degradation rate to propagate into IPC abstractions becomes impractical. For this reason, the base case rate for IPC abstractions is simply approximated from Figure 6-2[a]. The base case value chosen from this figure is $6\text{E-}16$ mol/s-cm². Considering the pH feedback mechanisms caused by low and high degradation rates mentioned above (i.e., high degradation rates cause high pH and low degradation rates result in low pH) and the generally low temperatures over a one-million-year

modeling period, the general range of degradation rates was approximated to be 1E-16 to 1E-14 mol/s-cm². This rationale addresses CR-6857.

The N-Reactor fuel degradation rate measurements are too high for a base case flow rate of 1 L/yr. To prevent the reaction from consuming all water in the cell, the base case degradation rate was reduced from 7.88E-7 to 5.00E-9 mol/s per liter of water (Output DTN: SN0702PAIPC1CA.001, file: *Rates 2MCO Cell 1b.xls*). This is reasonable because the reaction can only proceed as fast as the supply of water will allow.

As is shown in Section 6.10[a], the IPC abstractions are fairly insensitive to the degradation rates of the waste forms, steels, and alloys. The pH abstractions are primarily controlled by the secondary minerals that precipitate, not the rates at which these minerals precipitate. The ionic strength abstractions are more sensitive to the degradation rates but not as much as they are to liquid influx rate and time. The sensitivity to degradation rates is reflected in the quantification of ionic strength uncertainty.

Table 6-6[a]. HLW Glass Composition

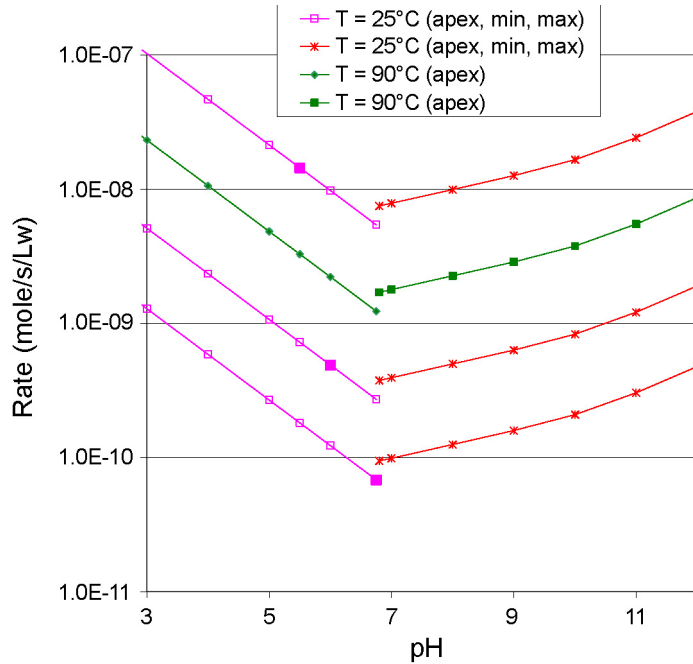
| Element | Moles/100 g |
|---------|-------------|
| Al | 9.43E-02 |
| B | 2.35E-01 |
| Ca | 2.45E-02 |
| Cu | 2.80E-03 |
| Fe | 1.48E-01 |
| Li | 2.38E-01 |
| Na | 3.84E-01 |
| Ni | 4.30E-03 |
| Mg | 5.57E-02 |
| Mn | 2.52E-02 |
| P | 7.80E-03 |
| Si | 8.54E-01 |
| U | 6.90E-03 |
| O | 2.89E+00 |

Source: Output DTN: SN0702PAIPC1CA.001, file:
CDSP_HLWGlass_Sensitivity r2.xls.

Table 6-7[a]. Waste Form Degradation Rates Scaled to One Liter of Water for Liquid Influx Base Case

| Degradation Rate | CSNF | HLW Glass | N-Reactor fuel |
|---------------------|----------|-----------|----------------|
| Minimum (mole/s) | 6.79E-11 | 4.40E-13 | Not available |
| Maximum (mole/s) | 1.44E-08 | 4.40E-11 | Not applicable |
| Base Case (mole/s) | 4.87E-10 | 2.64E-12 | 5.00E-09 |
| Minimum (mole/yr) | 2.14E-03 | 1.39E-05 | Not available |
| Maximum (mole/yr) | 4.53E-01 | 1.39E-03 | Not applicable |
| Base Case (mole/yr) | 1.54E-02 | 8.33E-05 | 1.58E-01 |

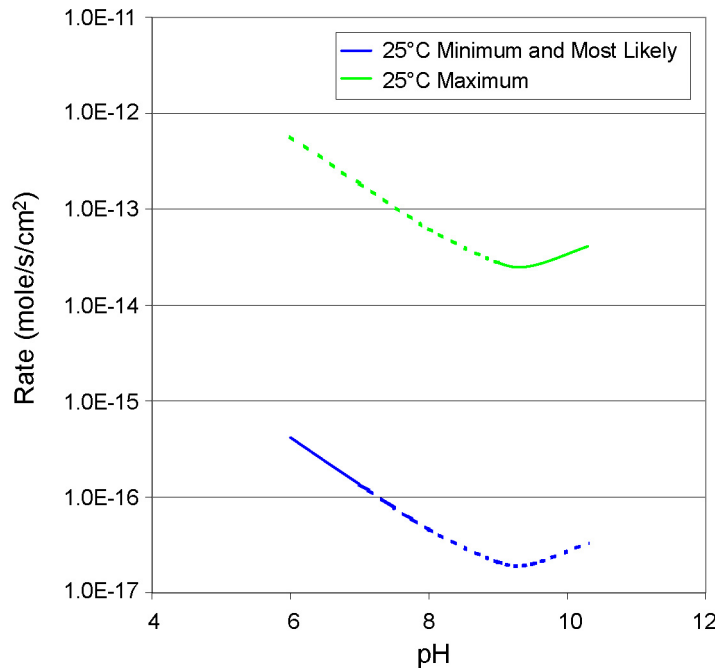
Source: Output DTN: SN0702PAIPC1CA.001, files: *Rates CSNF Cell 1.xls*, *Rates 2DHLW cell.xls*, and *Rates 2MCO Cell 1b.xls*.



Source: Output DTN: SN0702PAIPC1CA.001, file: *Rates CSNF Cell 1.xls*.

NOTE: The minimum, maximum, and base case rates used in this addendum are marked by the large solid squares on the 25°C lines. Apex indicates most probable.

Figure 6-1[a]. CSNF Degradation Rate per Liter of Water (Lw) in the Liquid Influx Model



NOTE: Data from Output DTN: SN0702PAIPC1CA.001, file: *Rates 2DHLW cell.xls*.

Figure 6-2[a]. HLW Glass Degradation Rates as a Function of pH for a Fracture Factor of 4

6.3.1.3.5[a] Water Compositions and Influx Rates

The water compositions listed in Tables 4-2[a] through 4-5[a] are used in the liquid influx simulations. Except for nitrate, the values in these tables were entered directly into the EQ3NR *.3i input files along with their units. Because the thermodynamic database structure does not permit a direct entry for nitrate, nitrate was converted to molal units and entered as “ammonia.” A basis switch (nitrate for ammonia) is used in the input file to apply the “ammonia” concentration to nitrate.

Two sets of EQ3NR *.3i input files were prepared. One set, named *.a.3i, was prepared to generate pickup files used to define displacer compositions in EQ6 files. They do not include elements in addition to those in Tables 4-2[a] through 4-5[a]. The second set of input files is identical to the *.a.3i files but includes additional elements at trace quantities (1E-16 molal). This set was needed to prepare for EQ6 simulations that include reactions involving elements not defined in Tables 4-2[a] through 4-5[a].

The waters in each input file were charge balanced by allowing EQ3NR to adjust the concentrations of H^+ and/or HCO_3^- . For P33, in which the bicarbonate concentration is missing, the dissolved carbonate concentration was set to be in equilibrium with a fugacity of carbon dioxide of 0.001 bar and the water was charge balanced using H^+ (thus, although the measured pH was not used, the equilibrated pH (8.2) is not far from the measured pH (8.31)). This is the charge balance method established for the screening and selection of representative TSw waters used in the near-field chemistry process model in *Engineered Barrier System: Physical and Chemical Environment* (SNL 2007 [DIRS 177412], Sections 6.6.3 and 6.6.5). For P35, $SiO_2(aq)$ was added by allowing EQ3NR to set it at the aqueous solubility of $SiO_2(am)$. For J-13, pH was fixed at 7.0, the mean of the field measurements (Harrar et al. 1990 [DIRS 100814], p. 4-9), and HCO_3^- was adjusted for charge balance, as justified in Section 4.1.2[a].

The EQ3NR input and output files are documented in Output DTN: SN0702PAIPC1CA.001 in directories \Waters and \Waters10 as described in *IPC file key revision 2.xls* in the DTN. The *.3p pickup files are used as the bottom portions of the *.6i input files to initialize the EQ6 simulations.

Liquid influx of each water in Tables 4-2[a] through 4-5[a] was simulated for the base case conditions in Table 6-1[a]. The selection of these waters is explained in Section 4.1.2[a]. The pore waters were selected primarily to represent the seismic scenario and the basalt waters were selected to represent the igneous scenario. However, the results indicate that the composition of these natural groundwaters has little or no effect on the IPC sensitivity analyses (Section 6.6[a]) and abstractions (Section 6.10[a]).

The base case rate liquid influx rate is 1 L/yr. This rate is approximately equivalent to 1 mL every 9 hours. The rate is varied between 0.1 and 1,000 L/yr in liquid influx sensitivity analyses documented in Section 6.6.4[a].

6.3.1.3.6[a] Mineral Controls

Table 6-8[a] lists the mineral phases suppressed in the EQ6 simulations and the justification for their suppressions. The major change is that hematite is now suppressed. The minerals removed from the original list were not found to precipitate in the simulations.

Table 6-9[a] provides a list of the major minerals that were not suppressed and were observed to precipitate in the EQ6 simulations in relatively large quantities. Because hematite is suppressed, goethite (α -FeOOH) is the major iron mineral that forms. NiMoO₄ is a new mineral in the *data0.ymp.R5* thermodynamic database (DTN: SN0612T0502404.014 [DIRS 178850]) and it was found to precipitate in large quantities.

The effects of suppressing hematite is evaluated in Section 6.6.11 of the parent document. The conclusion is that the IPC outputs are largely independent of which ferric (hydr)oxide mineral is allowed to form.

Table 6-8[a]. Suppressed Minerals

| Mineral | Chemical Formula | Justification |
|---|---|--|
| Hematite | $\alpha\text{-Fe}_2\text{O}_3$ | Hematite is more slowly formed than goethite and needs a precursor such as ferrihydrate from which it forms through dehydration and rearrangement (Schwertmann and Taylor 1995 [DIRS 105959]). Poorly crystalline solids such as ferrihydrate ($\text{Fe}_5\text{HO}_8\cdot 4\text{H}_2\text{O}$) and $\text{Fe}(\text{OH})_3$ that form during rapid oxidation of Fe and hydrolysis of Fe(III), will probably be present throughout the period of steel degradation in the waste package (Schwertmann and Cornell 1991 [DIRS 144629], Sections 1.3 and 8.1). These poorly crystalline iron oxides are unstable with respect to the more crystalline forms of iron oxides like goethite and hematite. Over time, the corrosion product assemblage is likely to be made up of some mixture of hematite, goethite, and ferrihydrate. Sensitivity simulations in Section 6.6.11 of the parent document indicate that the chemistry of the system is largely independent of whether goethite or hematite is allowed to form. |
| Quartz | SiO_2 | No change |
| Tridymite | SiO_2 | No change |
| Dolomite | $\text{CaMg}(\text{CO}_3)_2$ | No change |
| Dolomite-ord | | |
| Dolomite-dis | | |
| Talc | $\text{Mg}_3\text{Si}_4\text{O}_{10}(\text{OH})_2$ | No change |
| Zircon | ZrSiO_4 | No change |
| Sodylyte | $(\text{UO}_2)_2\text{SiO}_4\cdot 2\text{H}_2\text{O}$ | No change |
| PuO ₂ | PuO_2 | No change |
| Magnesite | MgCO_3 | Magnesite is commonly associated with metamorphic mineral assemblages such as schist. There are instances where magnesite is associated with salt deposits, yet it is uncertain that it can form under standard temperatures and pressures, as magnesite could be associated with the diagenesis of buried salt deposits. Because it is slow to form, this phase is suppressed (Klein and Hurlbut 1999 [DIRS 124293], p. 408; Langmuir 1997 [DIRS 100051], p. 195; Eugster and Hardie 1978 [DIRS 100743]). |
| Sepiolite | $\text{Mg}_4\text{Si}_6\text{O}_{15}(\text{OH})_2\cdot 6\text{H}_2\text{O}$ | No solutions simulated in this addendum become supersaturated with respect to this mineral, so suppression is immaterial. |
| Chrysotile | $\text{Mg}_3\text{Si}_2\text{O}_5(\text{OH})_4$ | No solutions simulated in this addendum become supersaturated with respect to this mineral, so suppression is immaterial. |
| Cristobalite(alpha) | SiO_2 | Cristobalite forms at temperatures greater than 1,470°C. At standard temperatures and pressures, cristobalite will slowly convert to quartz (Krauskopf 1979 [DIRS 105909], Figure 14-1). |
| $\text{Na}_2\text{U}_2\text{O}_7$ | $\text{Na}_2\text{U}_2\text{O}_7$ | Not a known alteration phase of synthetic or natural UO_2 (Wronkiewicz and Buck 1999 [DIRS 169286] Figure 3) |
| ZnCr_2O_4 | ZnCr_2O_4 | Not a known natural mineral. This compound, chromate yellow, has been used as a corrosion inhibitor on carbon steel, but has not been reported in nature. There is little evidence of this phase forming at the temperatures expected in the repository. |
| $(\text{UO}_2)_3(\text{PO}_4)_2\cdot 6\text{H}_2\text{O}$ | $(\text{UO}_2)_3(\text{PO}_4)_2\cdot 6\text{H}_2\text{O}$ | $(\text{UO}_2)_3(\text{PO}_4)_2\cdot 4\text{H}_2\text{O}$ is a stable phase under oxidizing conditions, low temperature, and pH values relevant for natural water systems (Sandino 1991 [DIRS 113307], pp. 16 to 17). While $(\text{UO}_2)_3(\text{PO}_4)_2\cdot 6\text{H}_2\text{O}$ is associated with a wide range of weathered U deposits (Finch and Murakami 1999 [DIRS 145442]), $(\text{UO}_2)_3(\text{PO}_4)_2\cdot 4\text{H}_2\text{O}$ is the phase selected to represent the precipitation of uranyl phosphate in the simulations. |

Table 6-9[a]. Major Minerals Formed in Simulations

| Mineral | Chemical Formula | Justification |
|--|---|--|
| Goethite | $\alpha\text{-FeOOH}$ | See justification for hematite suppression (Table 6-8[a]). |
| Pyrolusite | MnO_2 | No change |
| Trevorite | NiFe_2O_4 | No change |
| Powellite | CaMoO_4 | No change |
| Schoepite | $\text{UO}_3 \cdot 2\text{H}_2\text{O}$ | No change |
| Gibbsite | $\text{Al}(\text{OH})_3$ | No change |
| Nontronite | $(\text{Ca}_{0.5}\text{H}_1\text{K}_1\text{Mg}_{0.5}\text{Na})_{0.33}\text{Fe}_2(\text{Si}_1\text{Al})_4\text{O}_{10}(\text{OH})_2$ | No change |
| $\text{GdPO}_4 \cdot 2\text{H}_2\text{O}$ | $\text{GdPO}_4 \cdot 2\text{H}_2\text{O}$ | Although the hydration number was corrected from 10 to 2 in the thermodynamic database, the correction has no effect on the justification for suppression. |
| $(\text{UO}_2)_3(\text{PO}_4)_2 \cdot 4\text{H}_2\text{O}$ | $(\text{UO}_2)_3(\text{PO}_4)_2 \cdot 4\text{H}_2\text{O}$ | See justification for $(\text{UO}_2)_3(\text{PO}_4)_2 \cdot 6\text{H}_2\text{O}$ suppression (Table 6-8[a]). |
| Chalcedony | SiO_2 | No change |
| Calcite | CaCO_3 | Calcite is a common mineral in arid-climate soils and tends to dominate the chemistry of the soil or groundwater (Langmuir 1997 [DIRS 100051], p. 193). |
| Eskolaite | Cr_2O_3 | Because Cr-substituted goethite can be synthesized by aging a Cr-ferrhydrite precipitate (Schwertmann and Cornell 1991 [DIRS 144629], Chapter 5), it is likely that Cr-substituted ferrhydrite or $\text{Fe}(\text{OH})_3$ minerals which form during waste package degradation will eventually transform to Cr-substituted goethite or hematite. Eskolaite is allowed to form as a Cr sink because the EQ6 database does not contain Cr-substituted hematite or many other Cr phases. |
| NiMoO_4 | NiMoO_4 | This mineral is analogous to powellite (CaMoO_4), which often occurs as a secondary mineral in the oxidation zones of ore deposits (above). |
| $\text{PuO}_2(\text{OH})_2 \cdot \text{H}_2\text{O}$ | $\text{PuO}_2(\text{OH})_2 \cdot \text{H}_2\text{O}$ | Hydrated plutonyl hydroxide may be a component of the poorly crystalline, green degradation products observed in plutonium precipitation experiments at Los Alamos National Laboratory (Efurd et al. 1998 [DIRS 108015]; Rundle et al. 2002 [DIRS 168432]). It is a thermodynamically stable compound that would be expected to form from the degradation of CSNF. |
| Saponite | $(\text{Ca}_{0.5}\text{H}_1\text{K}_1\text{Mg}_{0.5}\text{Na})_{0.33}\text{Mg}_3(\text{Si}_1\text{Al})_4\text{O}_{10}(\text{OH})_2$ | Trioctahedral Mg-rich smectites (saponite or stevensite) have precipitated in saline and alkaline lakes and lake margins (Hover and Ashley 2003 [DIRS 169212]; Akbulut and Kadir 2003 [DIRS 169213]). These conditions are similar to those expected in evaporated solutions formed during the degradation of HLW glass. |
| Petalite | $\text{LiAlSi}_4\text{O}_{10}$ | Petalite is a lithium aluminosilicate that is found along with quartz and cleavelandite (flaky albite) in the subsolidus pegmatite deposits of the Pala District, San Diego Co., California (Roberts et al. 1990 [DIRS 107105], p. 665). It is a reasonable mineral to expect in the presence of degrading HLW glass. |
| Boltwoodite | $(\text{Na}_1\text{K})\text{UO}_2\text{SiO}_3\text{OH} \cdot 1.5\text{H}_2\text{O}$ | A known alteration phase of synthetic or natural UO_2 (Wronkiewicz and Buck 1999 [DIRS 169286], Figure 3) |

6.3.2[a] Surface Complexation Model

Not used in this addendum, as discussed in Section 1[a].

6.3.3[a] Constraints on the Waste Package Inner Environment

The migration of radionuclide and non-radioactive constituents from the repository into the environment can only be accomplished via the breaching of the engineered barriers that make up a waste package and the drip shield. Numerical simulations of the migration of radionuclides from the waste package have conservatively assumed that once a breach occurs, the entire contents of the waste package are exposed to oxygen and water. This assumption, in itself, applies a level of conservatism to all of the material degradation numerical simulations.

If a waste package is not breached catastrophically, the most probable mechanism is via a stress-corrosion crack or via pitting corrosion. Both of these mechanisms produce small openings that should be self-sealing or become filled with alteration products to the extent that movement of either vapor or liquid into or out of the waste package is trivial. Because the molar volume of metallic corrosion products is greater than that of the metal (iron metal to FeOOH expansion is ~36%), the corrosion products that form along the sides of the crack or pit should fill it. Without the flow obstruction caused by corrosion products, the stress corrosion cracks and localized corrosion pits would still provide much smaller infiltration pathways than would be available from the catastrophic failure via either seismic or igneous intrusion destructive events.

6.3.4[a] Calculation of pH Range

The primary objective of the IPC model with respect to pH is to determine the pH ranges within the waste form cells. To meet this objective, pH buffer capacities in the cells are determined as described below. The process model described in Section 6.3.1[a] simulates the effects of cell processes over time for liquid influx or at equilibrium for vapor influx, but the outputs of those simulations do not explicitly calculate the minimum and maximum pH values of the system based on buffer capacities. The following subsections explain the approach used to calculate the minimum and maximum pH values in the waste form cells.

The surface complexation model developed for the waste package IPC model is not used in the calculation of pH range in the waste form IPC model because there is a much lower abundance of iron alteration products in the waste form cells. In addition, there is no need to account for the effects of surface complexation on the pH range because rigorous and sufficient pH ranges can be established based on the pH buffering capacities of the degradation products.

6.3.4.1[a] pH Buffer Reactions

The pH of water in a cell will change over time as materials degrade, seepage enters and exists, and/or relative humidity increases or decreases. Many of the processes in the cell directly or indirectly add or subtract protons, which in turn affects pH. How easily the pH changes as a result of the various processes depends on the system pH buffer intensity. The limit of the pH change depends on the system pH buffer capacity in relation to the capacity of cell processes to exhaust the buffer capacity.

The pH limits in this addendum are defined by pH buffer reactions that cannot be overcome by potential sources of acid or base. These reactions in turn define the acid neutralizing capacity (ANC) and base neutralizing capacity (BNC) of the waste form cells.

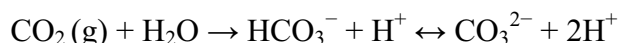
A good example of a pH buffering reaction that often defines the BNC of a cell is the dissolution of schoepite at pH values above neutral. At a fixed partial pressure of CO₂ (g) of approximately 10⁻³ bar, the dissolution reaction is represented by:



The functional relationship of schoepite solubility to pH, shown in Figure 6-3[a], indicates that at pH 7 or 8 schoepite will dissolve as pH increases. However, according to the chemical reaction above, this dissolution generates protons that work against a pH increase. Thus, schoepite dissolution buffers pH by counteracting processes that tend to increase pH.

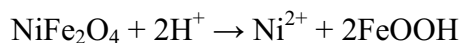
This buffering of schoepite and the associated pH maximum can be quantified by titrating the system with base. While schoepite is present, the pH will slowly rise for each aliquot of base added due to the generation of a proton for each uranyl carbonate complex formed. Only after schoepite is completely dissolved by the base will the pH rise quickly again with additional aliquots of base. The maximum pH that can be reached while schoepite is present is the pH immediately prior to complete dissolution of schoepite. The BNC is the total amount of base titrated when the schoepite becomes completely dissolved.

Another reaction that contributes to BNC is the dissolution of CO₂ (g):



As pH increases, CO₂ (g) becomes more soluble in water. However, when the pH is neutral or alkaline, the dissolution of CO₂ (g) into water releases one proton (or two when pH is above about 10.3) per CO₂ (g) molecule, which works against the pH increase.

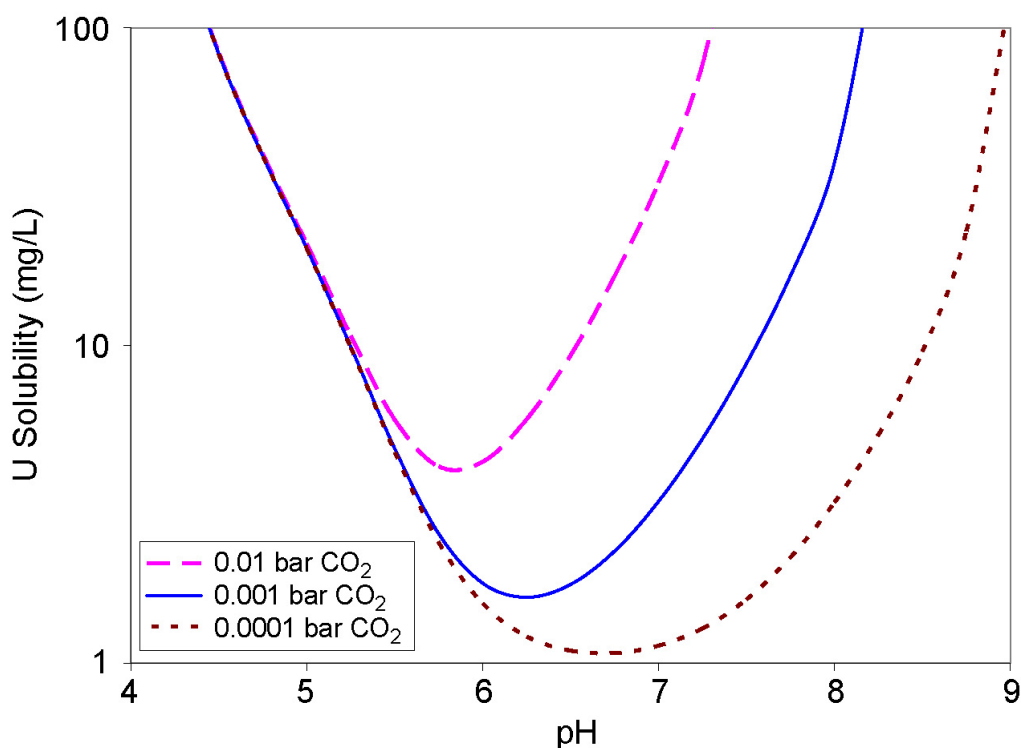
ANC in the IPC model is generally provided by trevorite. As stainless steel degrades, trevorite is allowed to precipitate to represent accumulation of Ni/Fe oxides. If processes cause pH to decrease below approximately 5.5, trevorite converts to goethite, releasing Ni as Ni²⁺ and consuming two protons for each Ni released:



This reaction is consistent with the increase in Ni solubility as pH falls below 6. This reaction buffers pH because the consumption of protons resists the pH decrease.

The buffering of this reaction and the associated pH minimum can be quantified by titrating the system with acid. Below a pH of 6 while trevorite is present, the pH will slowly decrease for each increment of acid added due to the consumption of two protons for each Ni²⁺ generated as shown in the reaction above. Only after trevorite is completely dissolved by the acid will the pH decrease quickly again with additional aliquots of acid. The minimum pH that can be reached while trevorite is present is the pH immediately prior to complete dissolution of trevorite. The ANC is the total amount of acid titrated when the trevorite becomes completely dissolved.

Another type of pH buffering reaction not included in the waste form IPC model (but included in the waste package IPC model in Sections 6.7, 6.8, and 6.9 of the parent report) is the hydrolysis of mineral surfaces. Surfaces of hydrous iron oxide minerals tend to buffer pH by donating protons to solution as pH increases and by adsorbing protons as pH decreases (Stumm and Morgan 1996 [DIRS 125332], pp. 534 to 540). Consequently, at low pH the surfaces of these minerals are positively charged and at high pH they are negatively charged. Buffering by these surface reactions tends to resist pH changes in the opposite direction of the surface's point of zero charge (pH_{zpc}). The pH_{zpc} is the pH where the net surface charge is zero. For goethite and hydrous ferric oxide, the pH_{zpc} is approximately 8 (Stumm and Morgan 1996 [DIRS 125332], p. 539).



Source: DTN: MO0707DISVALID.000 [DIRS 182270], file: *U solubility tables.xls*.

Figure 6-3[a]. Solubility of Schoepite as a Function of pH for Three Fugacities of Carbon Dioxide

6.3.4.2[a] Distinguishing between Reactions That Promote and Resist pH Change

The objective of the IPC model in terms of pH is to determine the maximum and minimum pH values for a given waste form cell. These limits are determined by calculating the capacities of the cells to buffer pH when exposed to processes that promote pH change (Section 6.10.1[a]). Although surface reactions are excluded from the waste form IPC model, there remain numerous reactions and processes that occur simultaneously in the process model that affect pH.

System processes that promote pH change in an equilibrium solution must change the mass of at least one component in solution. The system in this case is the waste form cell contents and their interactions as defined by the process model. These processes include degradation of waste

forms and structural materials, equilibration with open system gas fugacities, and fluxes of H₂O into and out of the cell. The slow, non-equilibrium degradation of materials provides a flux of components into the system. Likewise, equilibration of the system with a fixed gas fugacity of carbon dioxide and water vapor changes the total mass of carbonate and water in the system. Fluxes of seepage into and out of the cell also change the total masses of components in the system.

The processes that resist pH change (i.e., buffer pH) are equilibrium reactions that do not change the masses of components in the system. These processes include aqueous complexation and the precipitation and dissolution of secondary minerals. These reactions resist pH change due to Le Chatelier's principle. This principle states that if a chemical system at equilibrium experiences a change in concentration (from a process that changes the total masses of components in the system), the equilibrium will shift in order to minimize the change (Mahan 1975 [DIRS 125331], p. 188).

6.3.4.3[a] Acid and Base Titrations

To calculate the pH limits, acid and base titrations are simulated on the scaled waste form cells (i.e., scaled to one liter of water). The titrations begin after initially simulating ten years of cell processes to accumulate degradation products in the cell. Ten years is a sufficient amount of time to accumulate sufficient buffer capacity in the secondary minerals to render negligible the buffer capacities of aqueous reactions. It is also enough time to determine the rate at which pH-buffering minerals accumulate. The accumulated assemblage is then titrated with acid and base. Only minerals with high buffer capacities (and high abundances due to high accumulation rates) are used to set the pH limits.

The titrants in the simulated titrations are HCl and LiOH solutions. To prevent large changes in ionic strength during the titrations, the molarities of these solutions are adjusted to the ionic strength of the water in the cell being titrated. Titrations are performed at low and high ionic strengths to determine the effect of ionic strength on the upper and lower pH limits. They are also performed at different fugacities of carbon dioxide to determine the effects of carbon dioxide on these limits. Titrations typically end after 0.5 moles of acid or base are added. This is an extreme amount of acid or base for a cell containing one liter of water.

The use of hypothetical acid and base titrations is justified for determining pH ranges for the IPC model because this approach focuses on the reactions that buffer pH and it quantifies pH buffer capacities for the defined pH limits. This approach is further justified by the base case and sensitivity analyses in Sections 6.5[a] and 6.6[a] that show that the abstracted pH limits are reasonable and are never exceeded. Additional confidence is provided by the calculated accumulation rates of buffer capacities, which are much larger than the rates of any potential source of acid or base in the cell. The results of the titrations are presented in Section 6.10.1[a].

6.4[a] ALTERNATIVE CONCEPTUAL MODELS

No change.

6.5[a] BATCH REACTOR BASE CASE MODEL OUTPUT

This section provides IPC process model outputs for the base case scenarios. Liquid-influx simulations are presented in Section 6.5.1[a]. Liquid influx refers to seepage water or water that condenses outside the cell prior to flowing into the cell. It can enter only when the waste package is breached and the temperature is below 100°C (Section 1[a]). Section 6.5.2[a] presents vapor-influx simulations. Base case input values are specified in Table 6-1[a].

6.5.1[a] Liquid-Influx Model Results

This section is divided into three subsections corresponding to the three IPC waste form cells. The EQ6 simulations performed for the base case simulations are listed in Table 6-10[a]. The EQ3NR simulations used to generate the pickup files for the EQ6 simulations and all other EQ6 simulations performed in this addendum are listed in Table 6-11[a]. More detailed descriptions of these files are provided in the spreadsheet *IPC file key revision 2.xls* in Output DTN: SN0702PAIPC1CA.001.

The base case simulations do not represent the most likely outcomes. Rather, they represent potential mid-range process model realizations. Values of input parameters such as the degradation rates, liquid influx rates, and partial pressures of carbon dioxide are varied in Section 6.6[a] to determine functional relationships and uncertainty contributions of each input parameter important to the IPC abstractions.

Convergence problems associated with vanishingly small concentrations of trace elements in the cell frequently caused early termination of the liquid influx process model simulations. The extremely low concentrations are due to extensive flushing of the cell with fluids that do not contain all of the chemical components initially included in the cell. The terminations occur as the system reaches a long-term steady-state condition. These early terminations do not affect the pH abstractions because they are calculated separately in Section 6.10.1[a]. The ionic strength abstractions, however, do depend on time-dependent simulations. Specifically, they depend on the simulations described in Section 6.6.4[a]. The effect of early terminations on the ionic strength abstractions is discussed in more detail in Sections 6.6.4[a] and 6.10.2[a].

Table 6-10[a]. EQ6 Input and Output Files of Base Case Liquid Influx Process Model Simulations

| Cell | Path | Input Files | Output Files |
|-------|---------------|-------------|--------------|
| CSNF | 1milyr\CSNF\ | dpr*.6i | dpr*.6o |
| 2DHLW | 1milyr\2DHLW\ | dpr*.6i | dpr*.6o |
| 2MCO | 1milyr\2MCO\ | dpr*.6i | dpr*.6o |

Source: Output DTN: SN0702PAIPC1CA.001.

NOTE: An asterisk (*) is a wildcard for an alphanumeric string.

Table 6-11[a]. EQ3NR Input and Output Files Used in Addendum

| Water | Path | Input Files | Output Files |
|------------------|----------------------|---------------------------|---|
| SD9 | Waters\ or Waters10\ | <i>pSD9.3i, pSD9a.3i</i> | <i>pSD9.3o, pSD9.3p, pSD9a.3o, pSD9a.3p</i> |
| P33 | Waters\ or Waters10\ | <i>pP33.3i, pP33a.3i</i> | <i>pP33.3o, pP33.3p, pP33a.3o, pP33a.3p</i> |
| P35 | Waters\ or Waters10\ | <i>pP35.3i, pP35a.3i</i> | <i>pP35.3o, pP35.3p, pP35a.3o, pP35a.3p</i> |
| T17 | Waters\ or Waters10\ | <i>pT17.3i, pT17a.3i</i> | <i>pT17.3o, pT17.3p, pT17a.3o, pT17a.3p</i> |
| J-13 | Waters\ or Waters10\ | <i>J13trc.3i, J13a.3i</i> | <i>J13trc.3o, J13trc.3p, J13a.3o, J13a.3p</i> |
| BI | Waters\ or Waters10\ | <i>Bi.3i, Bia.3i</i> | <i>Bi.3o, Bi.3p, Bia.3o, Bia.3p</i> |
| BS | Waters\ or Waters10\ | <i>Bs.3i, Bsa.3i</i> | <i>Bs.3o, Bs.3p, Bsa.3o, Bsa.3p</i> |
| BW | Waters\ or Waters10\ | <i>Bw.3i, Bwa.3i</i> | <i>Bw.3o, Bw.3p, Bwa.3o, Bwa.3p</i> |
| H ₂ O | Waters\ or Waters10\ | <i>H2Otrc.3i, H2Oa.3i</i> | <i>H2Otrc.3o, H2Otrc.3p, H2Oa.3o, H2Oa.3p</i> |

Source: Output DTN: SN0702PAIPC1CA.001.

NOTE: There are two paths for two different execution dates.

6.5.1.1[a] CSNF

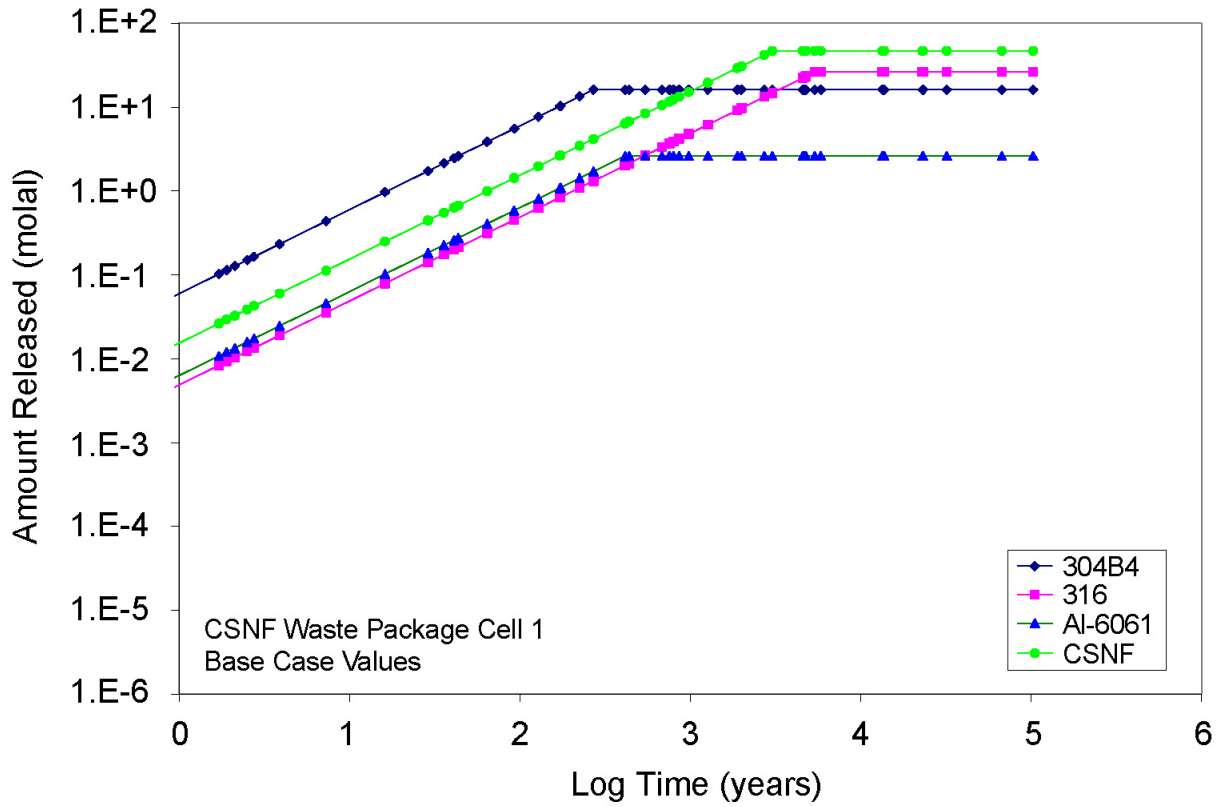
Figure 6-4[a] shows the total moles of materials in the CSNF cell degraded over time per kilogram of water in the liquid influx base case scenario defined in Table 6-1[a]. This figure indicates that under base case conditions, the borated stainless steel will completely degrade in approximately 270 years after waste package breach. Complete degradation of the aluminum shunts follows at about 440 years. Then after approximately 3,000 years and 5,400 years, the waste form and the Stainless Steel Type 316L, respectively, are fully degraded.

The pH is plotted as a function of time in Figure 6-5[a] for each seepage water defined in Section 4.1.2[a]. The evolution of pH in the cell is complex and is controlled by the various buffer reactions and degradation reactions. The pH reaches a minimum around 5.3 immediately before the Stainless Steel Type 304B4 is completely degraded. The degradation of 304B4 acidifies the solution, so when the 304B4 degradation stops, the pH rises.

The evolution of pH is almost identical regardless of the composition of the liquid influx. This implies that at a liquid influx rate of 1 L/yr, the reactions in the cell control the pH regardless of the type of groundwater that enters.

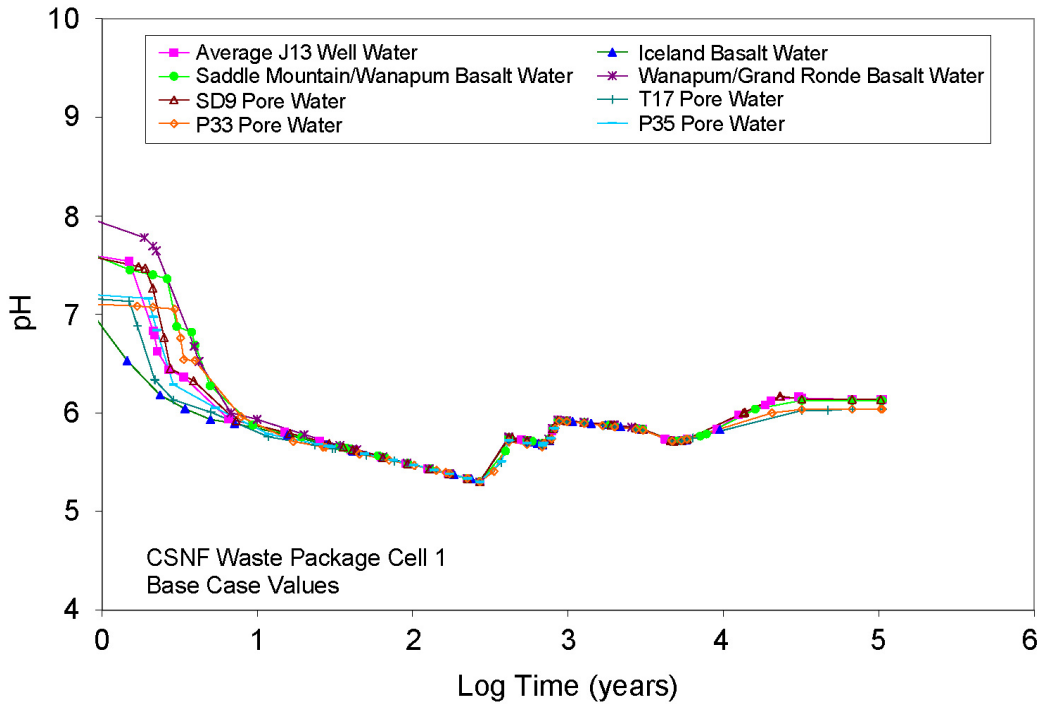
Figure 6-6[a] shows a plot of ionic strength as a function of time. The increase in ionic strength with time is driven by the most rapidly degrading material, Stainless Steel Type 304B4. As soon as the 304B4 is completely degraded, ionic strength levels off and is maintained by the remaining degradation reactions until they too end. After all degradation reactions end, the ionic strength returns to an ionic strength somewhat above the incoming ionic strength. Although degradation reactions have ceased, dissolution of the secondary minerals continues to add solutes to the water.

As in the case of pH, ionic strength is controlled primarily by the degradation reactions in this base case scenario. At 1 L/yr, the type of seepage water entering the cell has little effect on ionic strength.



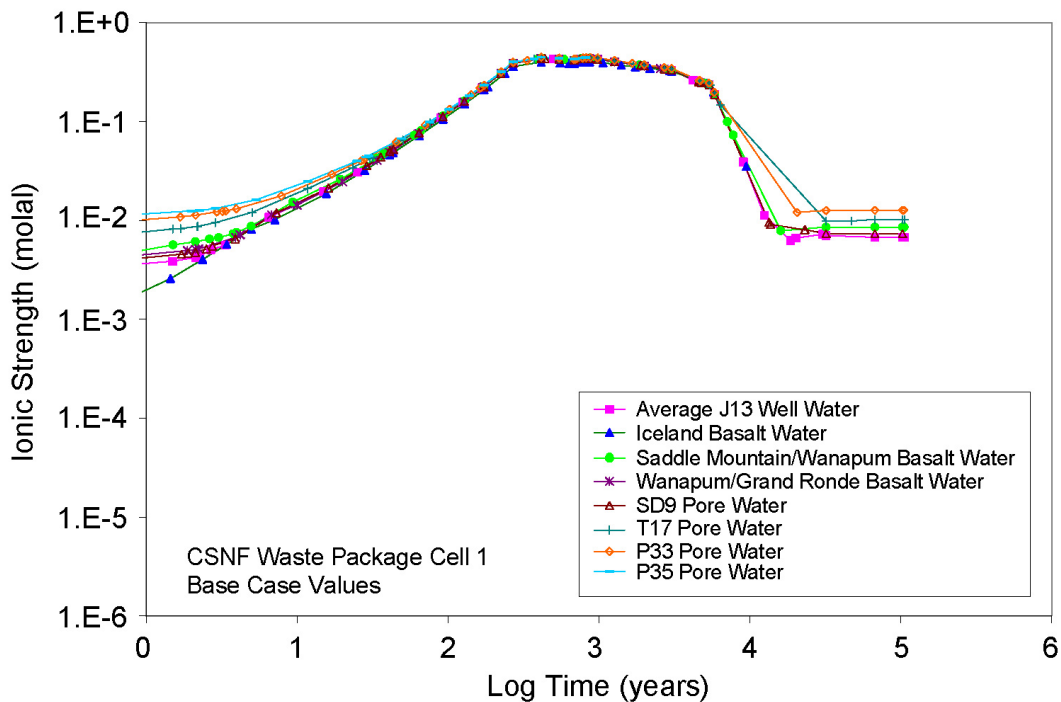
Source: Output DTN: SN0702PAIPC1CA.001, file: CSNF bc.xls.

Figure 6-4[a]. CSNF Liquid-Influx Base Case Materials Reacted over Time



Source: Output DTN: SN0702PAIPC1CA.001, file: CSNF bc.xls.

Figure 6-5[a]. CSNF Liquid-Influx Base Case pH over Time for Various Seepage Compositions



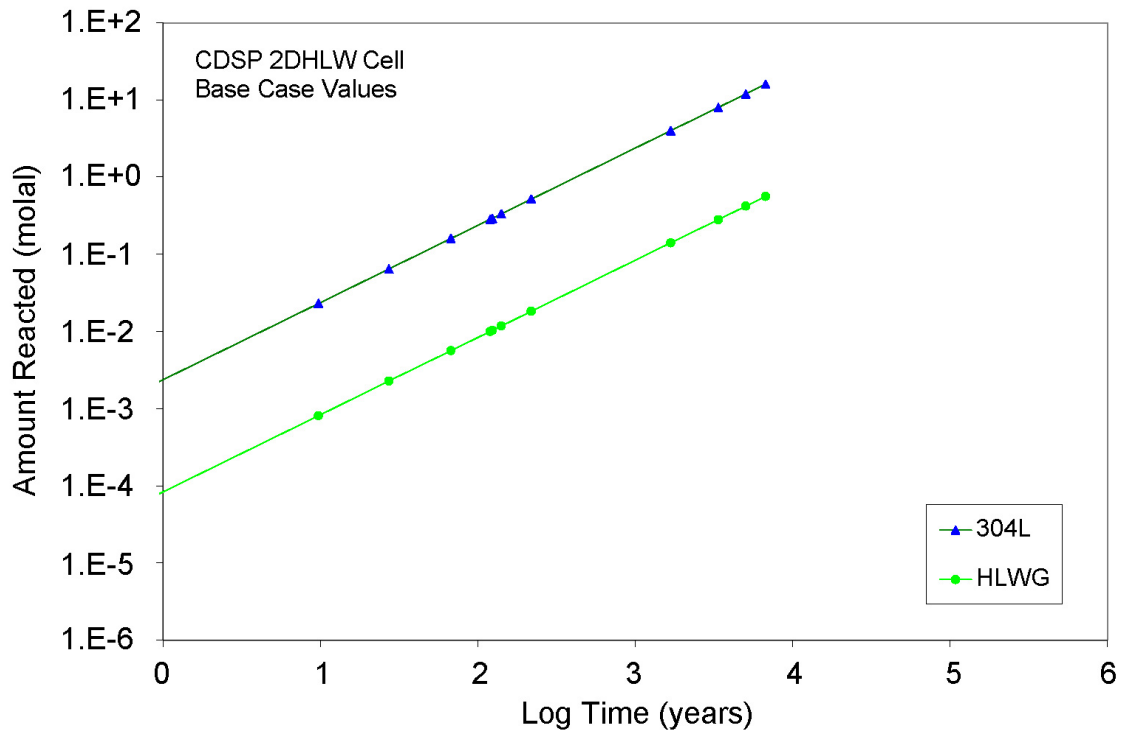
Source: Output DTN: SN0702PAIPC1CA.001, file: CSNF bc.xls.

Figure 6-6[a]. CSNF Liquid-Influx Base Case Ionic Strength over Time for Various Seepage Compositions

6.5.1.2[a] 2DHLW

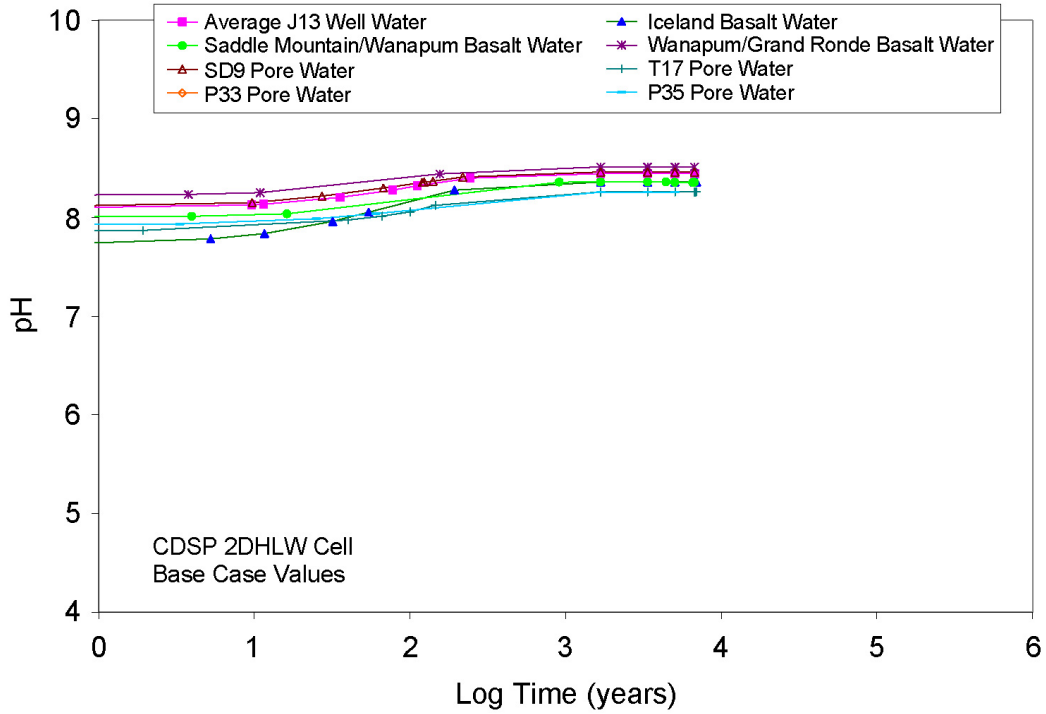
Figure 6-7[a] shows the total moles of materials in the 2DHLW cell degraded over time per kilogram of water in the liquid influx base case scenario. Under base case conditions, defined in Table 6-1[a], the 304L glass pour canisters will take more than 40,000 years to completely degrade while the HLW glass will take more than 5 million years to degrade (Output DTN: SN0702PAIPC1CA.001, file: *Rates 2DHLW cell.xls*). These simulations terminate around 7,000 years after the system reaches a steady state condition. Early termination and its relevance are discussed at the top of Section 6.5.1[a].

Figures 6-8[a] and 6-9[a] show plots of pH and ionic strength as a function of time for each seepage water defined in Section 4.1.2[a]. The degradation rates are so slow in the 2DHLW cell that there is more of an effect on pH and ionic strength by the incoming seepage at the base case liquid influx rate of 1 L/yr.



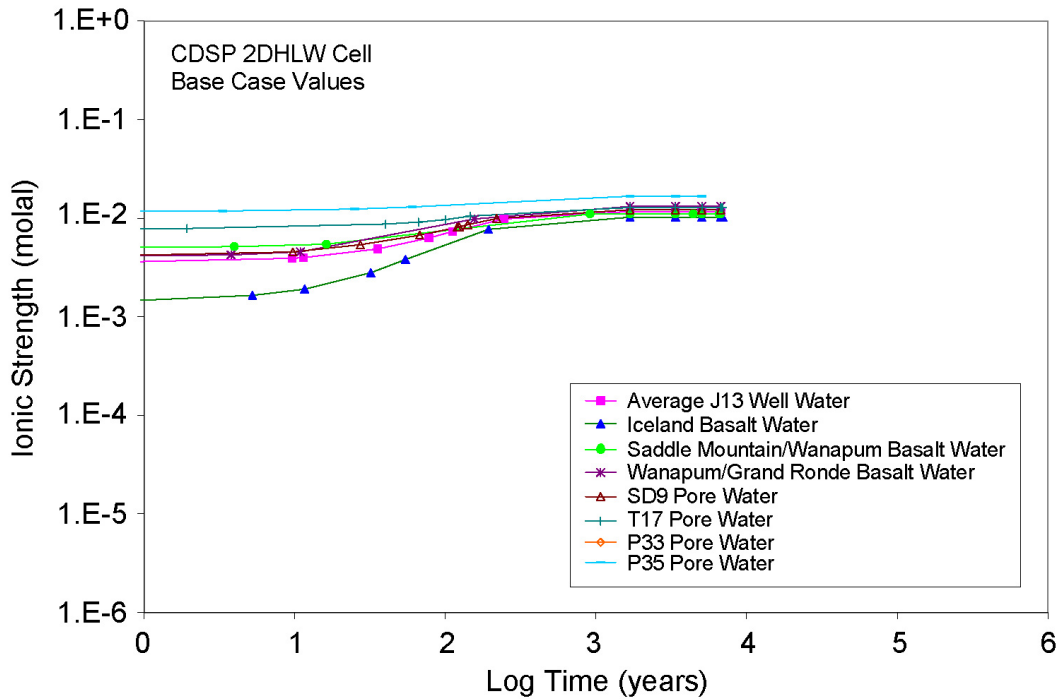
Source: Output DTN: SN0702PAIPC1CA.001, file: *2DHLW bc.xls*.

Figure 6-7[a]. 2DHLW Liquid-Influx Base Case Materials Reacted over Time



Source: Output DTN: SN0702PAIPC1CA.001, file: 2DHLW bc.xls.

Figure 6-8[a]. 2DHLW Liquid-Influx Base Case pH over Time for Various Seepage Compositions



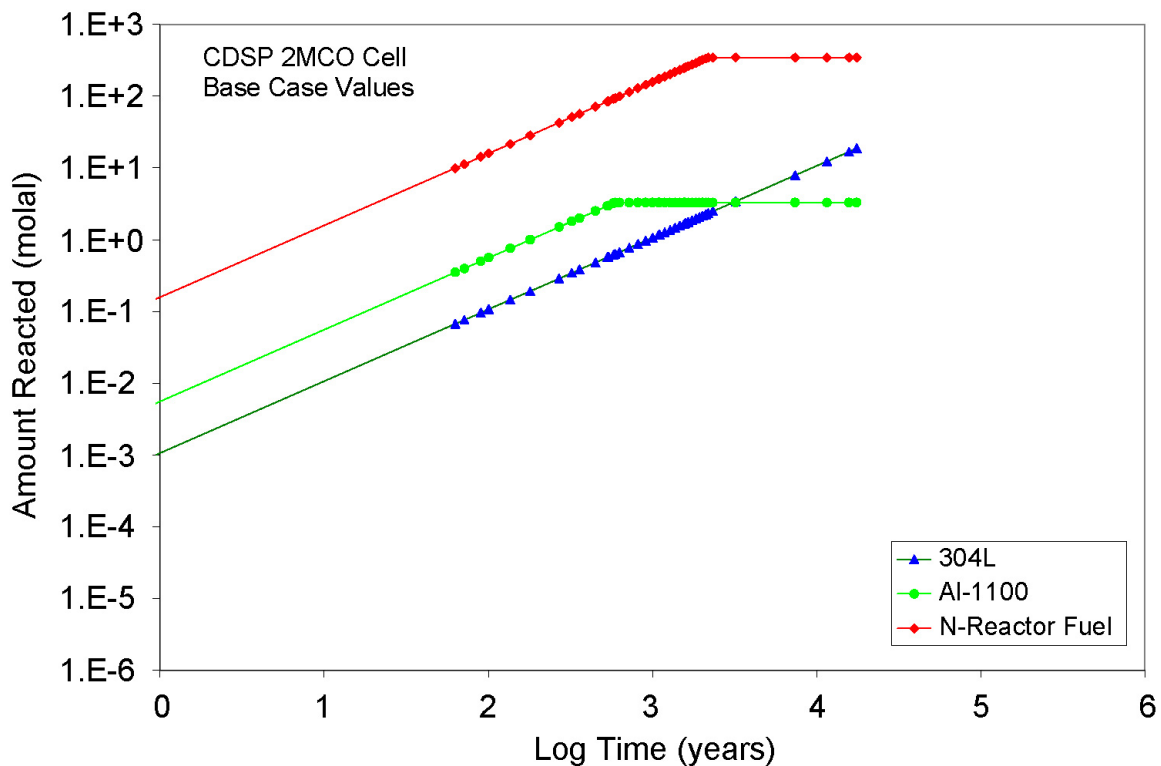
Source: Output DTN: SN0702PAIPC1CA.001, file: 2DHLW bc.xls.

Figure 6-9[a]. 2DHLW Liquid-Influx Base Case Ionic Strength over Time for Various Seepage Compositions

6.5.1.3[a] 2MCO

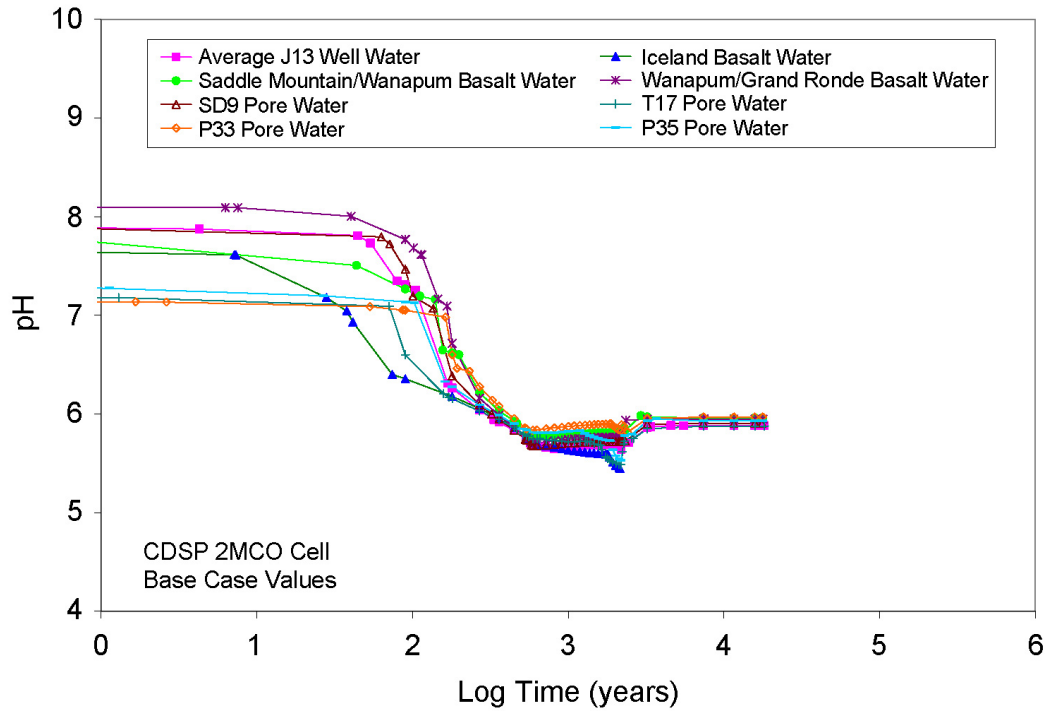
Figure 6-10[a] shows the total moles of materials in the 2MCO cell degraded over time per kilogram of water in the liquid influx base case scenario. Under base case conditions, defined in Table 6-1[a], the 304L will take more than 170,000 years to completely degrade while the waste form and aluminum fuel racks fully degrade in approximately 2,300 and 600 years, respectively (Output DTN: SN0702PAIPC1CA.001, file: *2MCO bc.xls*). These simulations terminate around 18,000 years after the system reaches a steady state condition. Early termination and its relevance are discussed at the top of Section 6.5.1[a].

Figures 6-11[a] and 6-12[a] show plots of pH and ionic strength as a function of time for the waters defined in Section 4.1.2[a]. The degradation rate of the N-Reactor fuel is rapid and until it is fully degraded it consumes much of the water that enters the cell. After the racks and fuel are fully degraded, the ionic strength decreases to values that are slightly above values in the incoming fluids. Although the rapid degradation reactions have ceased, dissolution of the secondary minerals and degradation of 304L continue to affect ionic strength and pH.



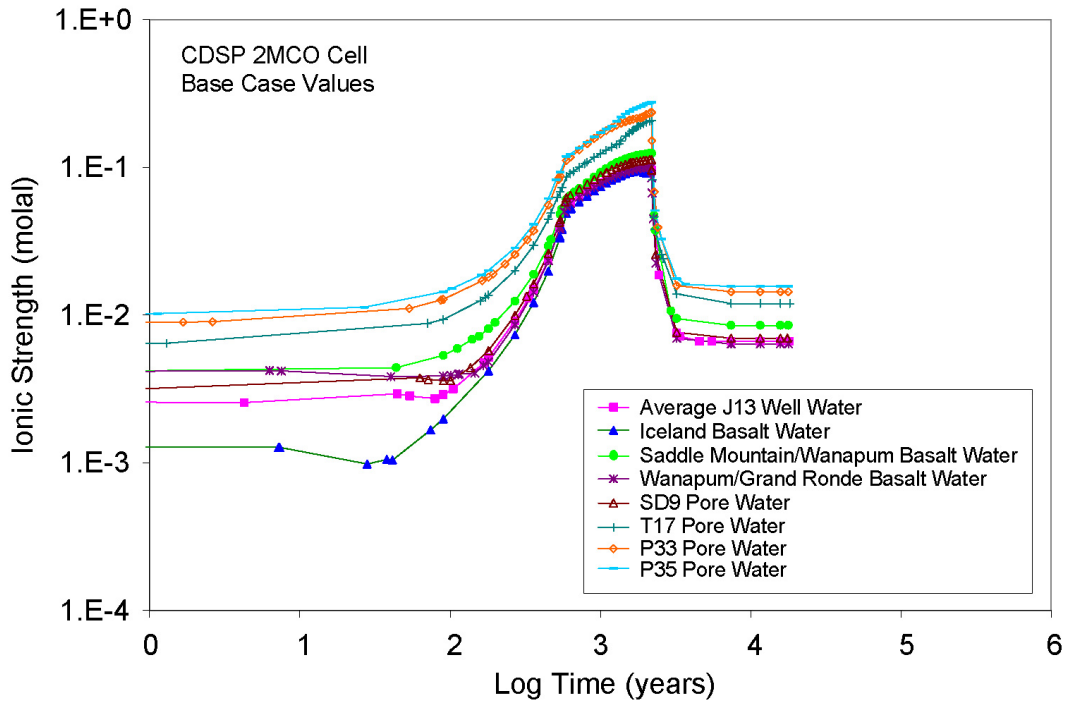
Source: Output DTN: SN0702PAIPC1CA.001, file: *2MCO bc.xls*.

Figure 6-10[a]. 2MCO Liquid-Influx Base Case Materials Reacted over Time



Source: Output DTN: SN0702PAIPC1CA.001, file: 2MCO bc.xls.

Figure 6-11[a]. 2MCO Liquid-Influx Base Case pH over Time for Various Seepage Compositions



Source: Output DTN: SN0702PAIPC1CA.001, file: 2MCO bc.xls.

Figure 6-12[a]. 2MCO Liquid-Influx Base Case Ionic Strength over Time for Various Seepage Compositions

6.5.2[a] Vapor-Influx Model Results

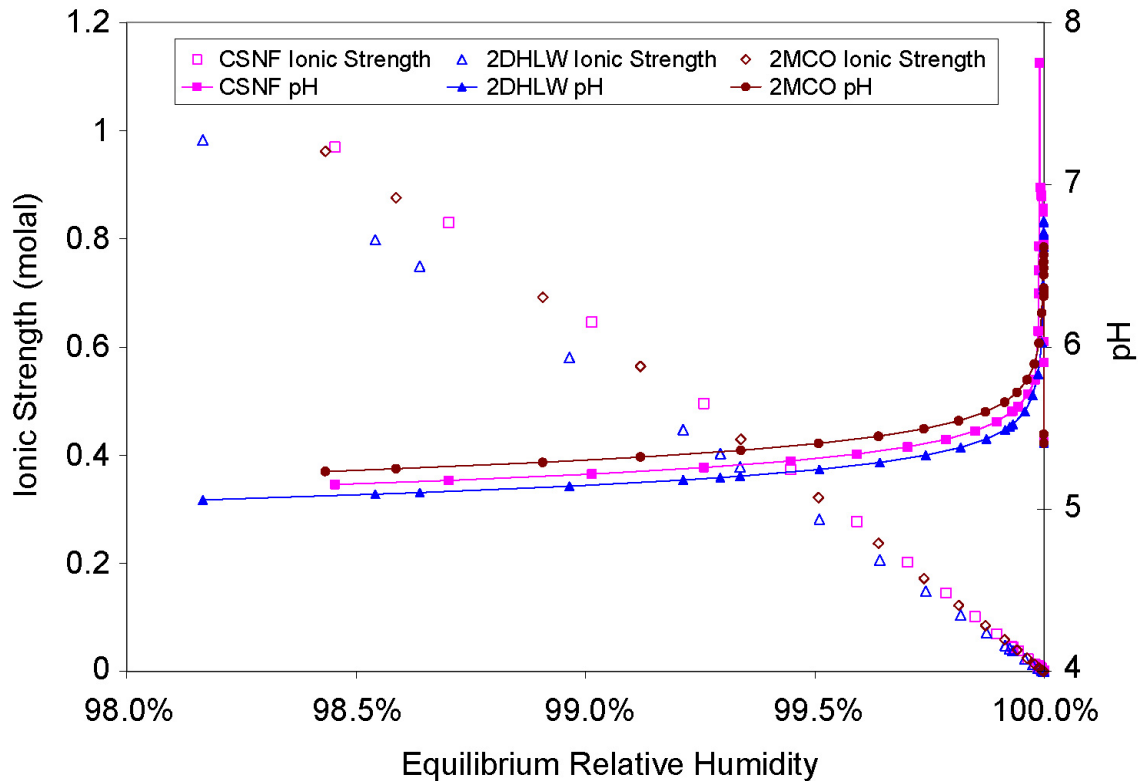
The EQ6 simulations performed for base case vapor influx simulations are listed in Table 6-12[a]. These simulations are described in Section 6.3.1.1[a] under “Code Implementation.” More detailed descriptions of these files are provided in *IPC file key revision 2.xls* in Output DTN: SN0702PAIPC1CA.001.

The results of these simulations are plotted in Figure 6-13[a] as a function of equilibrium relative humidity. The base case conditions are defined in Table 6-1[a].

Table 6-12[a]. EQ6 Input and Output Files of Base Case Vapor Influx Process Model Simulations

| Cell | Path | Input Files | Output Files |
|-------|-------------|-------------|------------------|
| CSNF | RH\ndCSNF\ | adpr.6i | adpr.6o, adpr.6p |
| 2DHLW | RH\nd2DHLW\ | adpr.6i | adpr.6o, adpr.6p |
| 2MCO | RH\nd2MCO\ | adpr.6i | adpr.6o, adpr.6p |

Source: Output DTN: SN0702PAIPC1CA.001.



NOTE: This figure is generated by plotting calculated ionic strength versus activity of water from files *RH\ndCSNF\adpr.6o*, *RH\nd2DHLW\adpr.6o*, and *RH\nd2MCO\adpr.6o* of Output DTN: SN0702PAIPC1CA.001. The figure itself is not documented in the DTN. The equilibrium relative humidity is equivalent to the activity of water expressed as a percentage.

Figure 6-13[a]. Vapor-Influx Base Case pH and Ionic Strength versus Equilibrium Relative Humidity

As in the liquid-influx case, pH is not abstracted from these process level outputs. Instead, it is abstracted from the titration calculations documented in Section 6.10.1[a].

Ionic strength, however, is abstracted from these simulations (Section 6.10.2.2[a]). The similarity in the results of the three different cells in Figure 6.13[a] indicates that ionic strength is a strong function of relative humidity and a weak function of the various degradation reactions.

6.6[a] BATCH REACTOR SENSITIVITY ANALYSES

The sensitivity of IPC process model outputs to input parameter values is addressed in this section. These sensitivity analyses are performed to determine how individual parameters affect process model outputs and to assess uncertainties. Sensitivity analyses addressed in this addendum include the ratio of water to reactants (Section 6.6.1[a]), composition of liquid influx (Section 6.6.2[a]), carbon dioxide fugacity (Section 6.6.3[a]), liquid influx rate (Section 6.6.4[a]), degradation rates (Section 6.6.5[a]), and CDSP type (Section 6.6.6[a]).

Additional sensitivity analyses are documented in the parent document. Besides the types of analyses included in this addendum, the parent document also analyzes effects of temperature, oxidation state of chromium, oxygen fugacity, mineral suppressions, and glass compositions. These additional sensitivity analyses are not used in this addendum because the effects of varying the values of these parameters over their possible ranges are shown in the parent document to be negligible (mineral suppressions, HLW glass compositions, oxygen fugacity) or nonrealistic (chromium oxidation state).

Temperature was found to have a substantial influence on pH when it is increased to 100°C (Section 6.6.6 of parent report). However, because radionuclide solubilities decrease with increasing temperature (SNL 2007 [DIRS 177418], Section 6.3.3.3), the radionuclide solubilities conservatively used in TSPA-LA are those calculated at 25°C from *Dissolved Concentration Limits of Elements with Radioactive Isotopes* (SNL 2007 [DIRS 177418], Section 6.3.3.3). To be consistent with the solubility model, all IPC abstractions in this addendum are based on calculations performed at 25°C. The higher confidence in thermodynamic data at 25°C provides additional justification for choosing 25°C for the IPC simulations.

The sensitivity analyses below indicate which parameters are important to include directly in IPC abstractions and which can be included in the quantification of uncertainties. Only liquid influx sensitivity simulations are performed in this section because most of the sensitivity relationships are the same in the vapor influx simulations. The additional sensitivity in the vapor influx simulations is the ionic strength relationship with relative humidity. This sensitivity is addressed in Section 6.10.2.2[a].

6.6.1[a] Ratio of Water to Reactants

Sensitivity analyses on the volumetric ratio of water to reactants are documented in Section 6.6.1 of the parent document. They indicate a negligible effect on the range of pH but a distinct effect on the calculated pH and ionic strength. As the ratio of water to reactant decreases, the calculated pH and ionic strength shift in time.

This effect is also shown in a sensitivity analysis documented in Appendix C of *Geochemistry Model Validation Report: External Accumulation Model* (SNL 2007 [DIRS 181395]). That model simulates the same processes as the waste form IPC model. In the analysis, water saturation (i.e., percentage of void space filled with water) is varied between 3% and 100%. Because the flow rates and degradation rates are held constant, varying the water saturation is equivalent to varying the ratio of water to reactants. Figure C-1 in that report (SNL 2007 [DIRS 181395], Appendix C) shows that the predicted pH values for the various water saturations are very similar over time but that lower water saturations approach steady-state conditions more quickly. Steady-state conditions are indicated in the figure by an unchanging pH over a period of time. Sharp changes in pH are caused by the disappearance of a reactant due to complete degradation or dissolution of the reactant. When a reactant disappears, the rate of degradation or dissolution instantly changes from a constant value to zero. Afterwards, as the system approaches a new steady-state condition, systems with lower water saturations approach steady state more quickly.

Steady state is approached more quickly in a system with lower water saturation because lower water saturation implies a higher turnover rate. The turnover rate is the rate at which the entire volume of water in the cell is replaced. For the model simulations, it is calculated by dividing the liquid influx rate q by the volume of water in the cell V_w . The inverse of the turnover rate is the mean residence time. Thus, higher turnover rates flush residual components from the cell more quickly than lower turnover rates.

The steady-state concentrations in the cell are independent of the water saturation (or ratio of water to reactants). Mass balance dictates that the mass of a dissolved component in the cell at the next time step equals the mass of the dissolved component in the cell in the current time step plus the mass that enters in the liquid influx plus the amount that enters from degradation minus the mass that flows out of the cell. The corresponding equation is:

$$C_{i+1}V_w = C_iV_w + C'q\Delta t + k\Delta t - C_iq\Delta t$$

where C_{i+1} and C_i are the dissolved concentrations (mass per volume) at the end and beginning of the time step, respectively, C' is the dissolved concentration in the liquid influx (mass per volume), Δt is the time increment (time), and k is the overall rate (mass per time) at which degradation and dissolution reactions add component mass to the water. Rearranging gives:

$$C_{i+1} - C_i = (C' - C_i)\frac{q\Delta t}{V_w} + \frac{k\Delta t}{V_w}$$

Dividing by Δt and taking the limit as Δt goes to zero implies:

$$\frac{\partial C}{\partial t} = \frac{(C' - C)q + k}{V_w}$$

This equation shows that as V_w decreases, the rate of change in the dissolved concentration increases, as discussed above. It also shows that at steady state, when $\partial C/\partial t$ equals zero, the numerator on the right is zero, implying the following relationship:

$$C_{ss} = C' + \frac{k}{q}$$

where C_{ss} is the concentration at steady state. Note that C_{ss} is not a function of V_w and thus it is not a function of the ratio of water to reactant. At high q relative to k , C_{ss} is approximately equivalent to the concentration in the liquid influx.

For the liquid influx simulations in this addendum, a water volume that is half of the void volume is used (Section 6.3.1.1[a]), i.e., a water saturation of 50%. Sensitivity analyses involving this ratio are not included in this addendum because they are included elsewhere (Appendix C of SNL 2007 [DIRS 181395] and Section 6.6.1 of the parent report). The water saturation has no effect on the pH abstractions because the pH abstractions only define the pH ranges (Section 6.10.1[a]). For the ionic strength abstractions, a low water saturation (e.g., less than 10%) does have an effect. Based on the cited simulations and the equations above, the effect would be most noticeable after a sudden and major change in the flow rate or degradation/dissolution rate. The effect would be within the range of uncertainty, except perhaps during the first several flushes. However, a conceptual model in which all surfaces are in contact with flowing water while the water saturation is less than 10% is more difficult to justify. A more reasonable model for a low water saturation is that not all surfaces are in contact with the flowing water. This would imply lower overall degradation rates affecting the flowing water, which in turn would counteract the effects of low water saturation. Moreover, after several flushes of the cell, the system approaches a new steady state and the differences in ionic strength predicted for base case and low water saturations would be negligible.

6.6.2[a] Composition of Liquid Influx

The sensitivity of the waste form IPC model to the composition of liquid influx is assessed in this section based on the results presented in other sections of this addendum. The base case simulations in Section 6.5.1[a] include five different types of ground waters at Yucca Mountain and three different ground waters from basalt aquifers. Other simulations in which the water composition is varied include the simulations performed for the sensitivity analyses in Sections 6.6.3[a] through 6.6.5[a].

In general, the composition of the liquid influx has little effect on the IPC model results and abstractions because the degradation reactions and the secondary minerals that precipitate have a large influence on the water composition in the cell. The largest effects of liquid influx composition are observed when the liquid influx rate is high. At high influx rates, the turnover rate is high and the mean residence time is low, which implies that there is less time for the composition of the liquid influx to be changed by degradation reactions. These effects are discussed in Section 6.6.4[a].

Additional sensitivity analyses involving the liquid influx of “thermally perturbed” fluids are documented in the parent document. They indicate that these fluids have little effect on waste package IPC pH and ionic strength abstractions (Section 6.6.2 of the parent report).

6.6.3[a] Carbon Dioxide Fugacity

The sensitivity simulations performed for carbon dioxide fugacity are listed in Table 6-13[a]. The $p\text{CO}_2$ variable shown here and throughout the addendum represents the negative logarithm of the partial pressure of carbon dioxide in bars, much like pH represents the negative logarithm of the activity of the hydrogen ion in water. More detailed descriptions of these files are provided in spreadsheet *IPC file key revision 2.xls* in Output DTN: SN0702PAIPC1CA.001.

Table 6-13[a]. EQ6 Input/Output Files for Carbon Dioxide Sensitivity Simulations

| Cell | $p\text{CO}_2$ | Path | Input Files | Output Files |
|-------|----------------|------------------|-------------|--------------|
| CSNF | 4 | 1milyr\CSNFC-4\ | dpr*.6i | dpr*.6o |
| | 2 | 1milyr\CSNFC-2\ | | |
| 2DHLW | 4 | 1milyr\2DHLWc-4\ | dpr*.6i | dpr*.6o |
| | 2 | 1milyr\2DHLWc-2\ | | |
| 2MCO | 4 | 1milyr\2MCOc-4\ | dpr*.6i | dpr*.6o |
| | 2 | 1milyr\2MCOc-2\ | | |

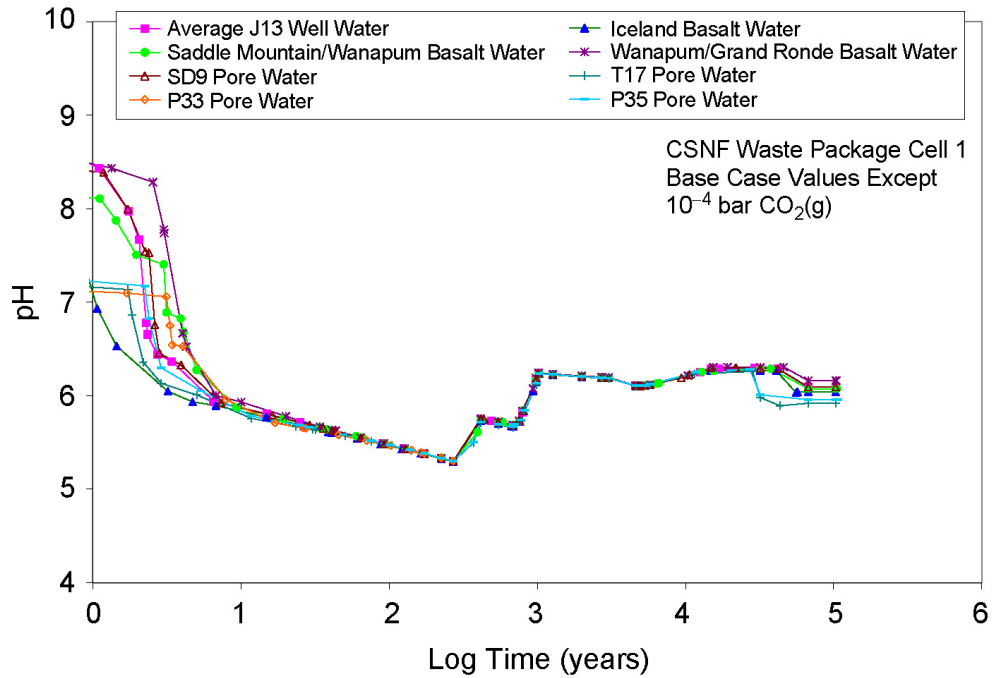
Source: Output DTN: SN0702PAIPC1CA.001.

NOTE: An asterisk (*) is a wildcard for an alphanumeric string.

The effects of carbon dioxide fugacity on IPC process model simulations are shown in Figures 6-14[a] through 6-25[a]. Effects on pH are shown in the first six figures and effects on ionic strength are shown in the latter six figures.

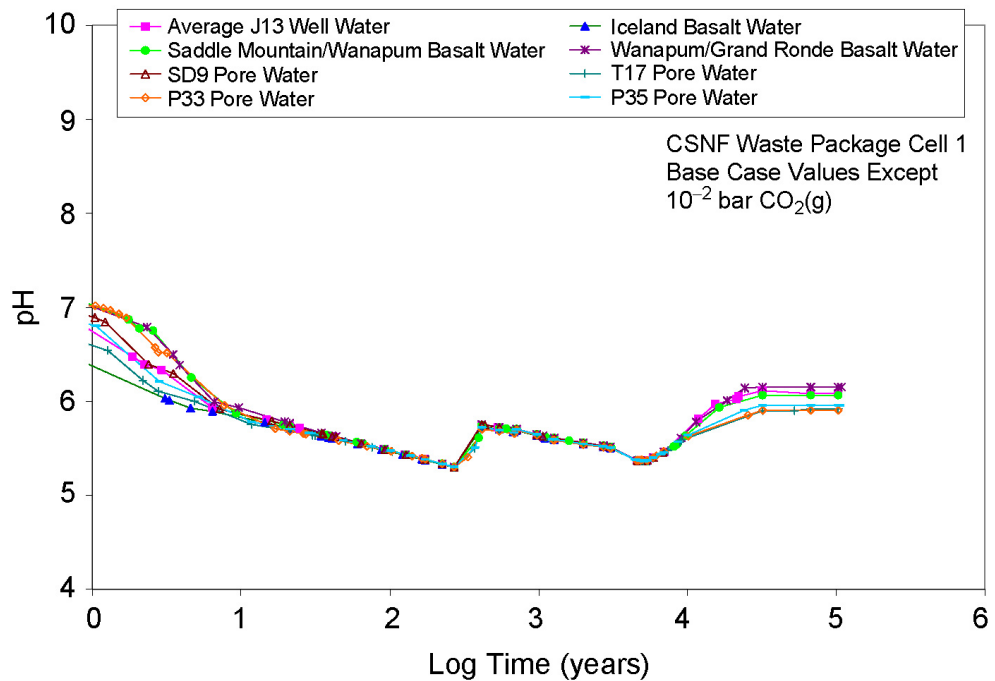
Carbon dioxide fugacity can have a significant effect on pH calculations, especially when degradation rates are low like they are in the 2DHLW cell under base case conditions (Figures 6-16[a] and 6-17[a]). However, the objective of the IPC abstraction is not to determine the most likely pH. Rather, it is to define the minimum and maximum pH values. Section 6.10.1[a] shows that the pH range varies depending on the carbon dioxide fugacity. Consequently, this functionality is built into the pH abstractions. The figures in this section provide realizations that do not fully explore the entire pH ranges, but they are consistent with the ranges defined in the abstractions.

Figures 6-20[a] through 6-25[a] show that ionic strength is only weakly affected by the fugacity of carbon dioxide. Thus, the ionic strength abstractions are not dependent on carbon dioxide fugacity.



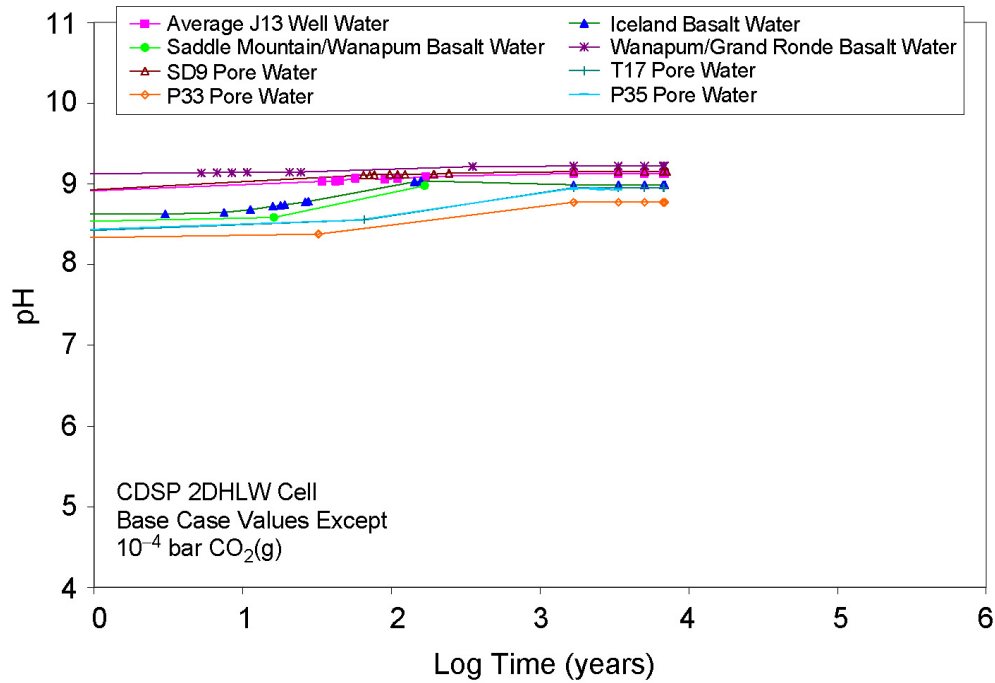
Source: Output DTN: SN0702PAIPC1CA.001, file: CSNF low co2.xls.

Figure 6-14[a]. CSNF Liquid-Influx pH over Time for Various Seepage Compositions at 10^{-4} bar $\text{CO}_2(\text{g})$



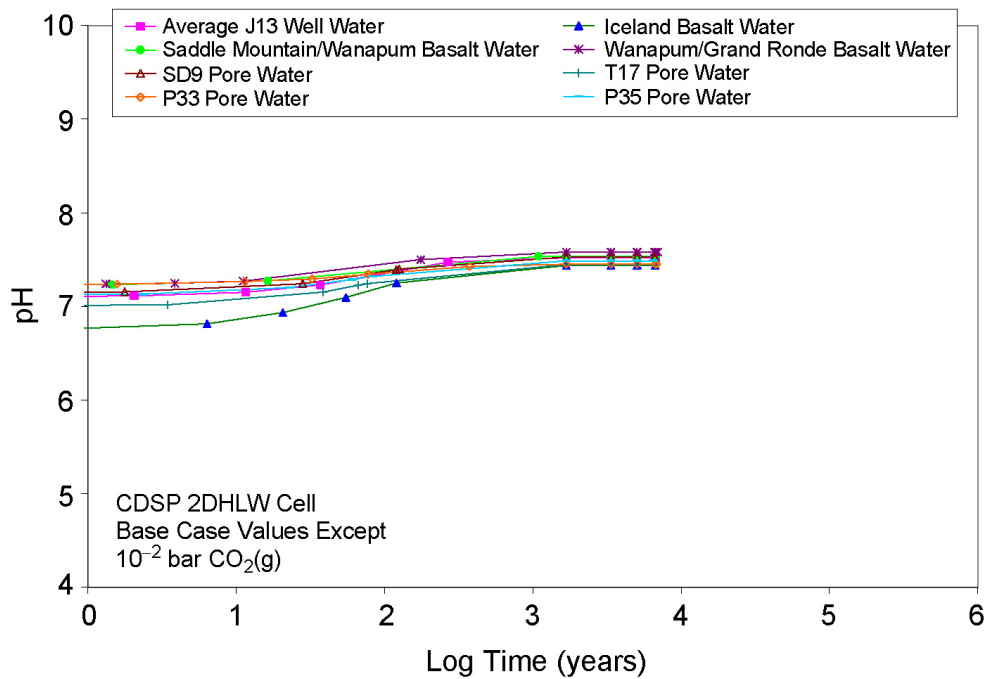
Source: Output DTN: SN0702PAIPC1CA.001, file: CSNF high co2.xls.

Figure 6-15[a]. CSNF Liquid-Influx pH over Time for Various Seepage Compositions at 10^{-2} bar $\text{CO}_2(\text{g})$



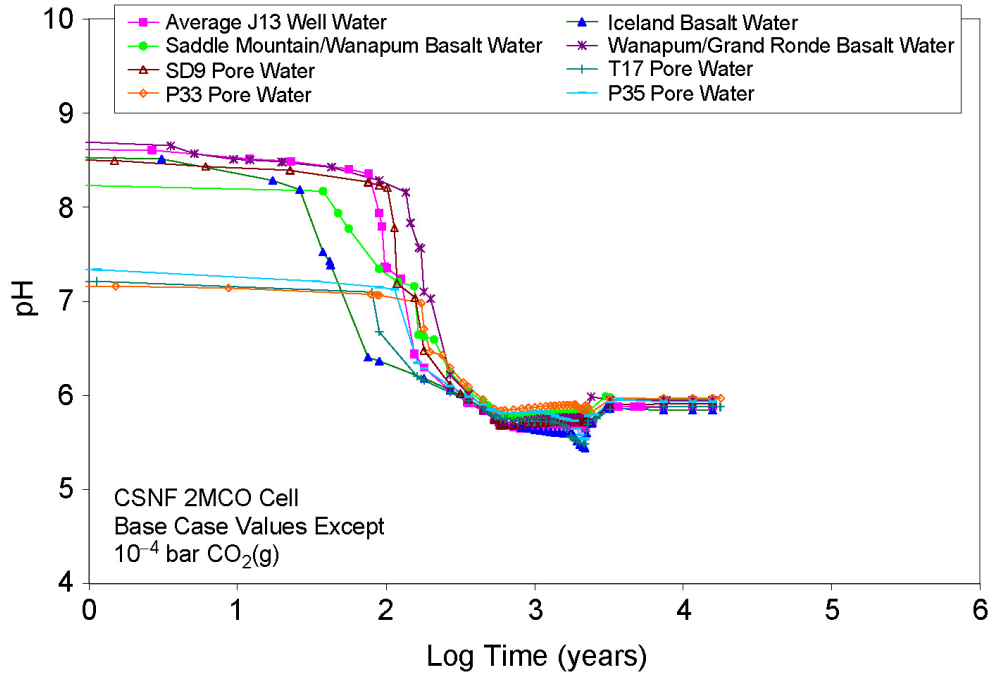
Source: Output DTN: SN0702PAIPC1CA.001, file: 2DHLW low co2.xls.

Figure 6-16[a]. 2DHLW Liquid-Influx pH over Time for Various Seepage Compositions at 10^{-4} bar $\text{CO}_2(\text{g})$



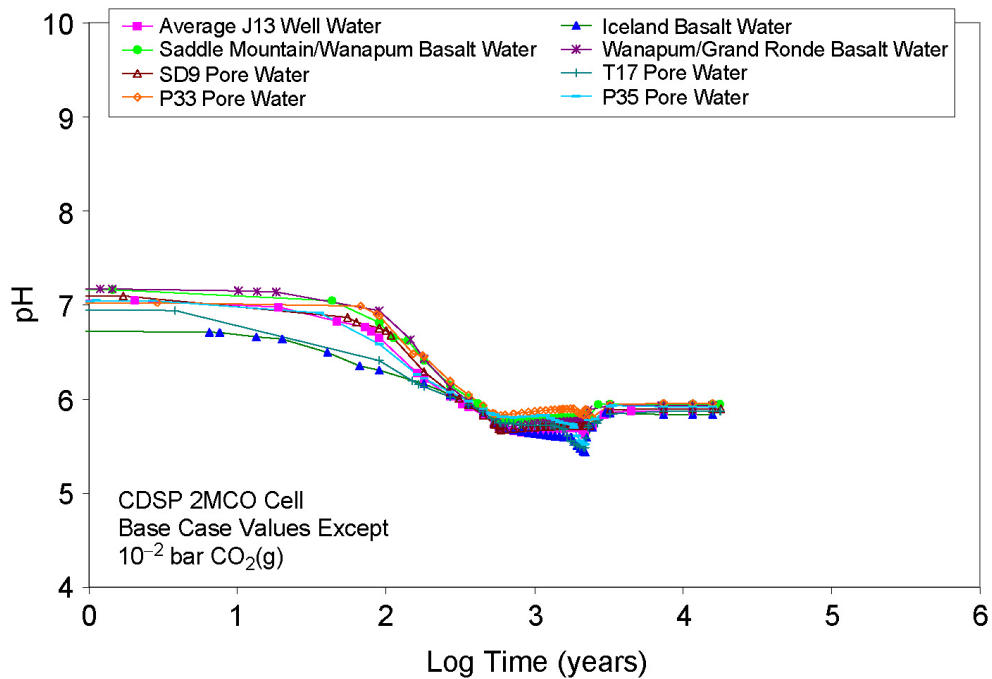
Source: Output DTN: SN0702PAIPC1CA.001, file: 2DHLW high co2.xls.

Figure 6-17[a]. 2DHLW Liquid-Influx pH over Time for Various Seepage Compositions at 10^{-2} bar $\text{CO}_2(\text{g})$



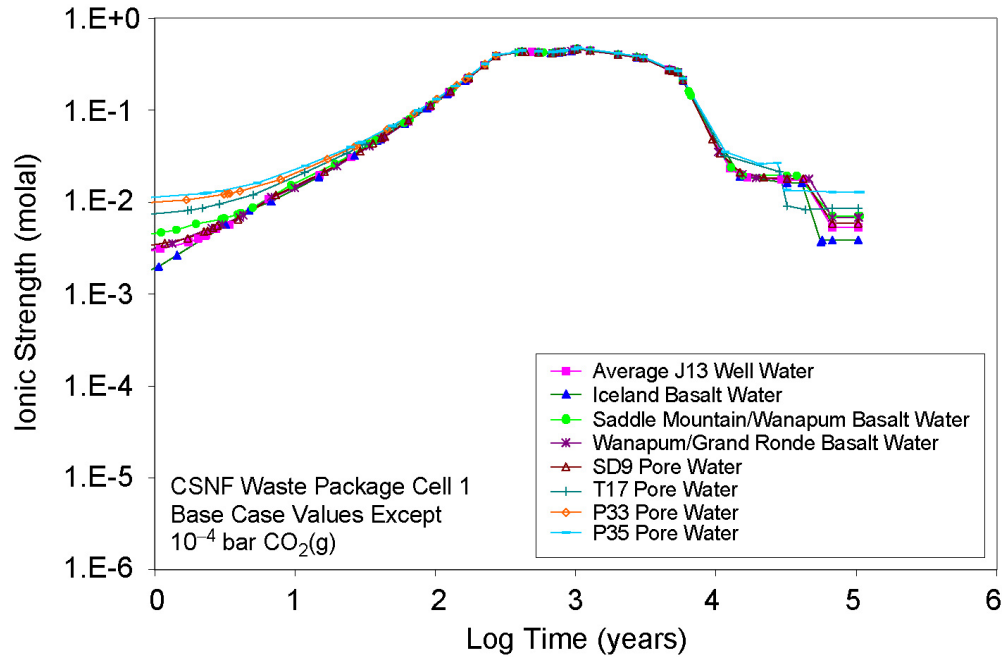
Source: Output DTN: SN0702PAIPC1CA.001, file: 2MCO low co2.xls.

Figure 6-18[a]. 2MCO Liquid-Influx pH over Time for Various Seepage Compositions at 10^{-4} bar $\text{CO}_2(\text{g})$



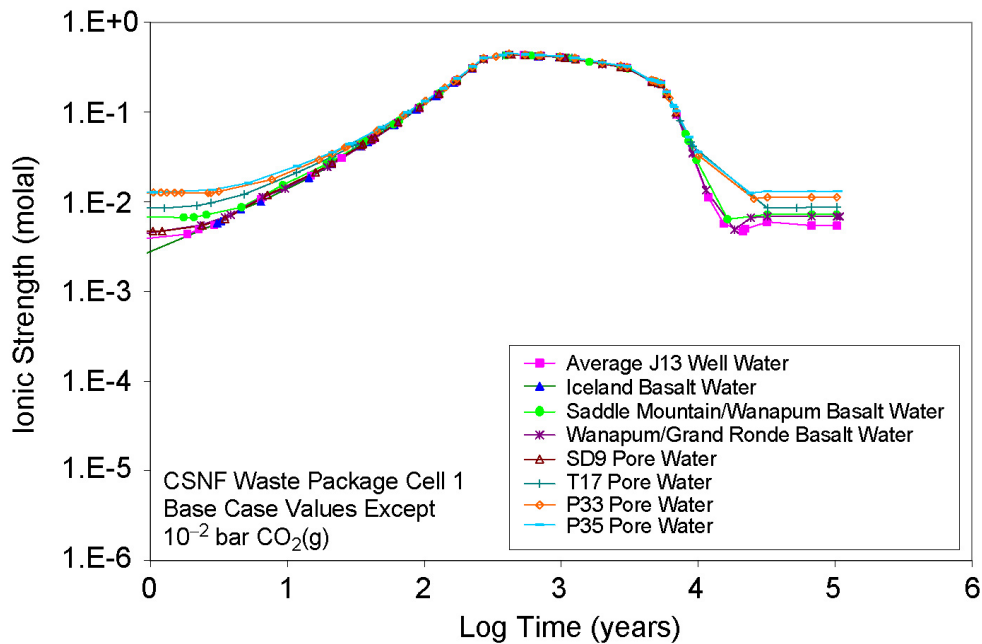
Source: Output DTN: SN0702PAIPC1CA.001, file: 2MCO high co2.xls.

Figure 6-19[a]. 2MCO Liquid-Influx pH over Time for Various Seepage Compositions at 10^{-2} bar $\text{CO}_2(\text{g})$



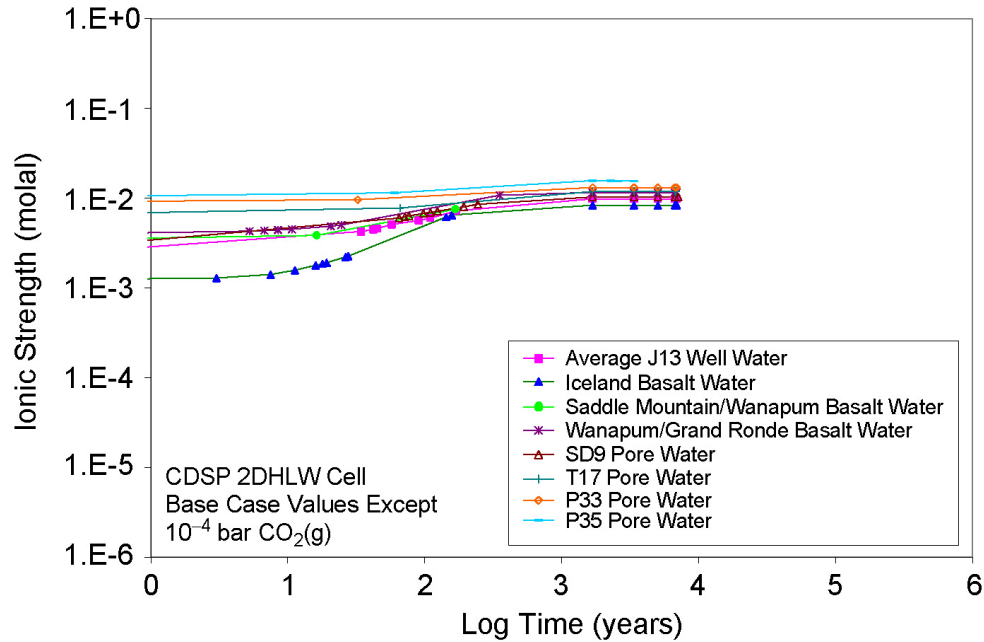
Source: Output DTN: SN0702PAIPC1CA.001, file: CSNF low co2.xls.

Figure 6-20[a]. CSNF Liquid-Influx Ionic Strength over Time for Various Seepage Compositions at 10^{-4} bar $\text{CO}_2(\text{g})$



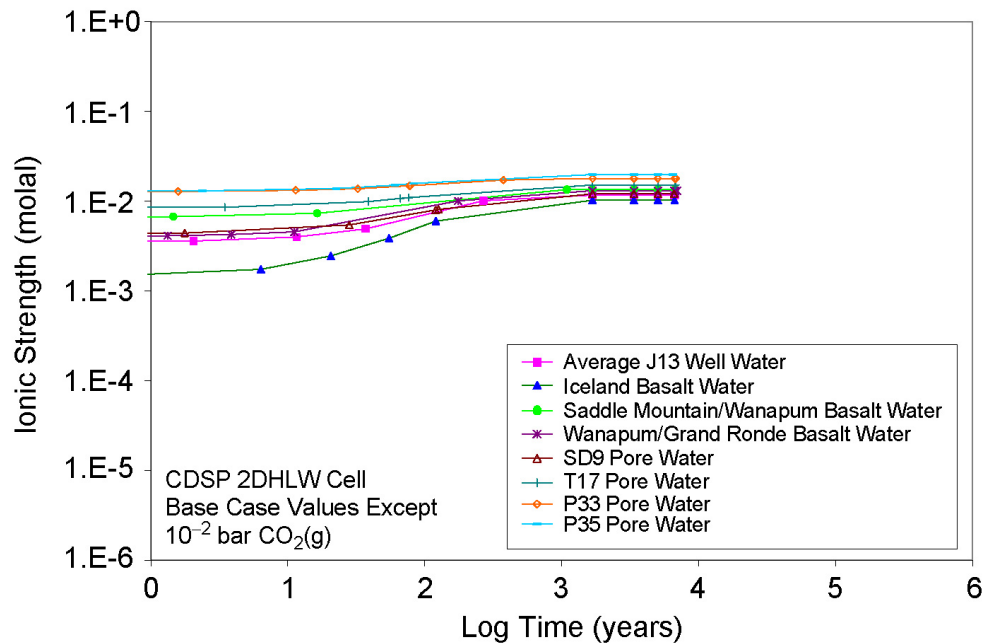
Source: Output DTN: SN0702PAIPC1CA.001, file: CSNF high co2.xls.

Figure 6-21[a]. CSNF Liquid-Influx Ionic Strength over Time for Various Seepage Compositions at 10^{-2} bar $\text{CO}_2(\text{g})$



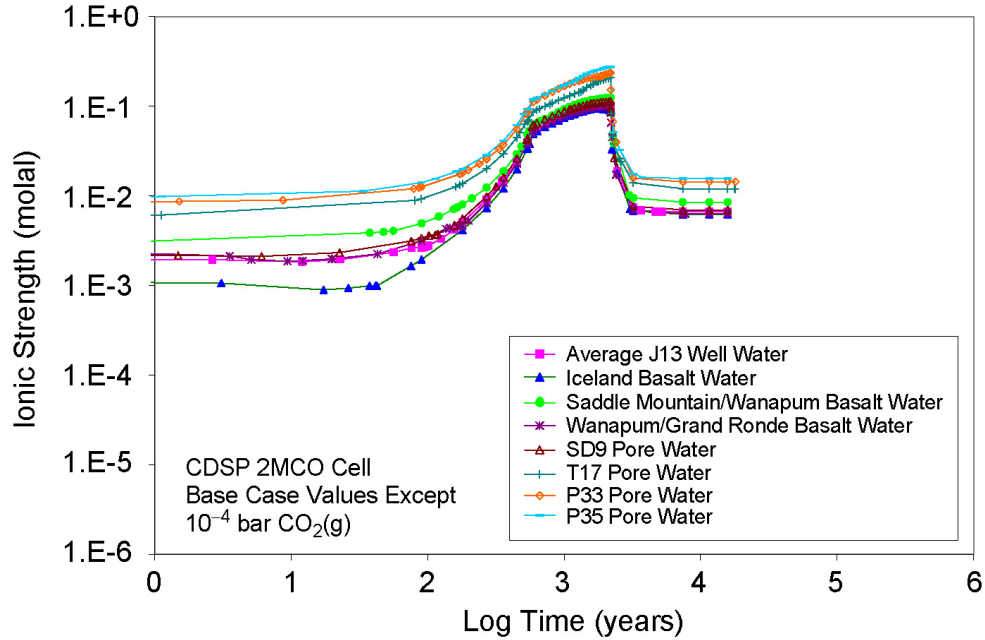
Source: Output DTN: SN0702PAIPC1CA.001, file: 2DHLW low co2.xls.

Figure 6-22[a]. 2DHLW Liquid-Influx Ionic Strength over Time for Various Seepage Compositions at 10⁻⁴ bar CO₂(g)



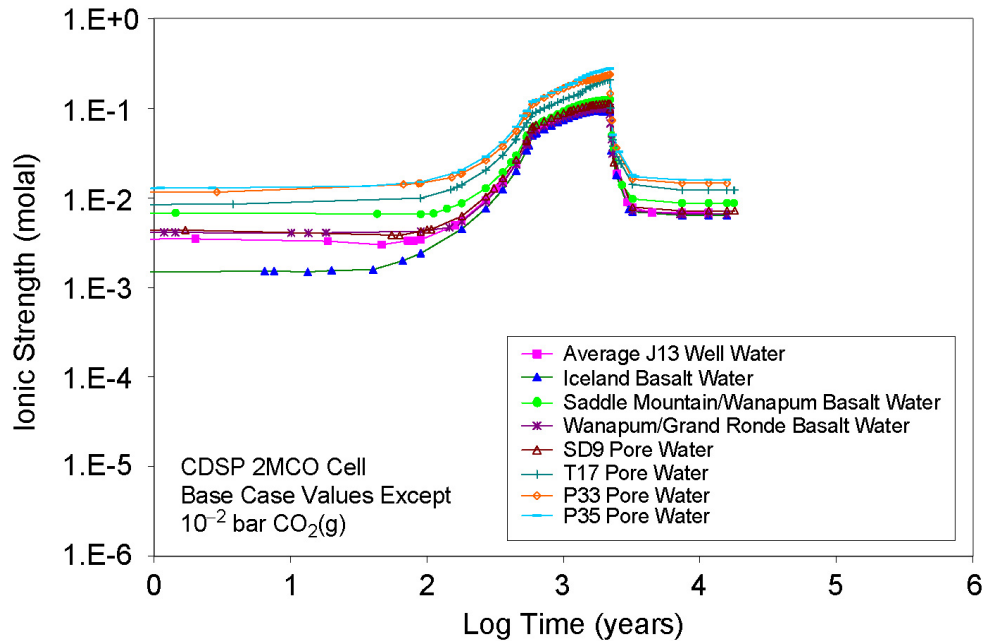
Source: Output DTN: SN0702PAIPC1CA.001, file: 2DHLW high co2.xls.

Figure 6-23[a]. 2DHLW Liquid-Influx Ionic Strength over Time for Various Seepage Compositions at 10⁻² bar CO₂(g)



Source: Output DTN: SN0702PAIPC1CA.001, file: 2MCO low co2.xls.

Figure 6-24[a]. 2MCO Liquid-Influx Ionic Strength over Time for Various Seepage Compositions at 10^{-4} bar $\text{CO}_2(\text{g})$



Source: Output DTN: SN0702PAIPC1CA.001, file: 2MCO high co2.xls.

Figure 6-25[a]. 2MCO Liquid-Influx Ionic Strength over Time for Various Seepage Compositions at 10^{-2} bar $\text{CO}_2(\text{g})$

6.6.4[a] Liquid Influx Rate

The sensitivity simulations for liquid influx rate are listed in Table 6-14[a]. More detailed descriptions of these files are provided in *IPC file key revision 2.xls* in Output DTN: SN0702PAIPC1CA.001.

Table 6-14[a]. EQ6 Input/Output Files for Liquid Influx Rate Sensitivity Simulations

| Cell | Liquid Influx Rate (L/yr) | Path | Input Files | Output Files |
|-------|---------------------------|------------------|-------------|--------------|
| CSNF | 1,000 | 1milyr\CSNFflow\ | adpr*.6i | adpr*.6o |
| | 100 | 1milyr\CSNFflow\ | bdpr*.6i | bdpr*.6o |
| | 10 | 1milyr\CSNFflow\ | cdpr*.6i | cdpr*.6o |
| 2DHLW | 1,000 | 1milyr\2DHLWflw\ | adpr*.6i | adpr*.6o |
| | 100 | 1milyr\2DHLWflw\ | bdpr*.6i | bdpr*.6o |
| | 10 | 1milyr\2DHLWflw\ | cdpr*.6i | cdpr*.6o |
| | 0.1 | 1milyr\2DHLWflw\ | ddpr*.6i | ddpr*.6o |
| 2MCO | 1,000 | 1milyr\2MCOflow\ | adpr*.6i | adpr*.6o |
| | 100 | 1milyr\2MCOflow\ | bdpr*.6i | bdpr*.6o |
| | 10 | 1milyr\2MCOflow\ | cdpr*.6i | cdpr*.6o |

Source: Output DTN: SN0702PAIPC1CA.001.

NOTE: An asterisk (*) is a wildcard for an alphanumeric string.

Figures 6-26[a] through 6-31[a] show the effects of liquid influx rate on pH and ionic strength. Influx rates were varied from 0.1 L/yr to 1000 L/yr for 2DHLW waste form cells and from 1 L/yr to 1000 L/yr for CSNF and 2MCO waste form cells. Rates considerably lower than the minimum rates simulated would produce ionic strengths above 1 molal, which is beyond the ionic strength range of the waste form IPC process model.

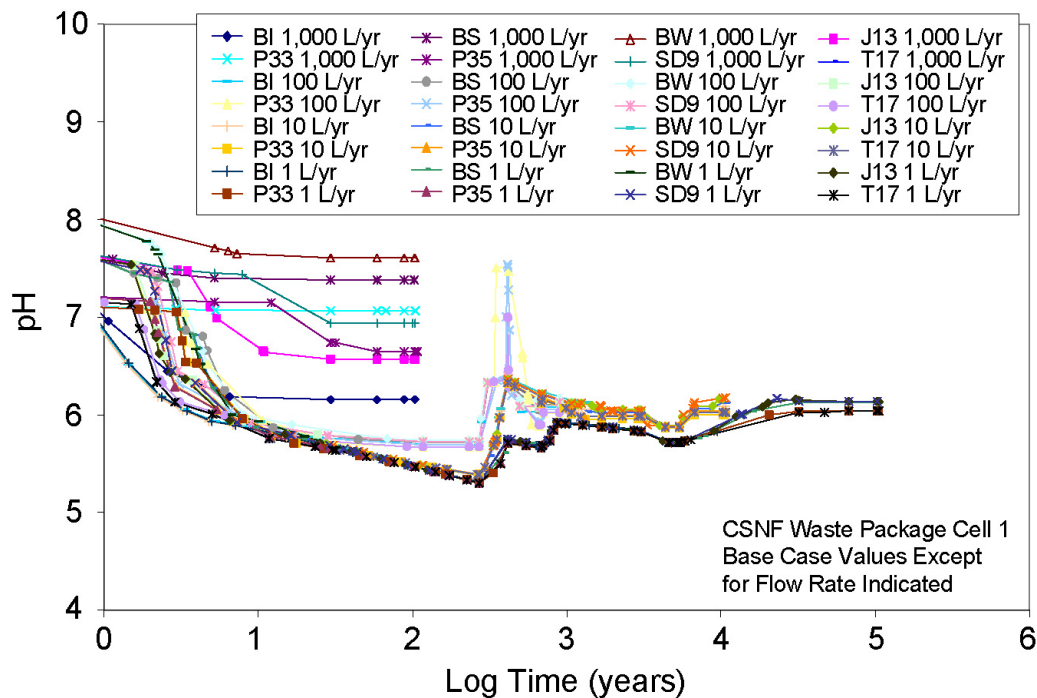
As discussed in Section 6.5.1[a], convergence problems associated with vanishingly small concentrations of trace elements in the cell frequently caused early termination of the liquid influx process model simulations. The extremely low concentrations are due to extensive flushing of the cell with fluids that do not contain all of the chemical components initially included in the cell. These terminations, which are particularly early at high liquid influx rates, do not prevent adequate interpretation of the simulations, however, because long time periods are not required to reveal the effects of liquid influx rates over time. Early terminations do not affect the pH abstractions because pH abstractions are calculated separately in Section 6.10.1[a]. The ionic strength abstractions, however, do depend on time-dependent simulations. The effect of early terminations on the ionic strength abstractions is discussed in Section 6.10.2[a].

The figures show two major dependencies for ionic strength: liquid influx rate and time. As predicted by the relationships identified in Section 6.6.1[a], increasing the liquid influx rate (or turnover rate) diminishes the effects of the degradation reactions. Consequently, as liquid influx rate increases, the reactions in the cell have a diminished effect on the incoming fluid. That is why the spread in the calculated pH and ionic strength looks more like the spread in the incoming fluid when liquid influx rates are high.

The spike in pH in the 100 L/yr simulations around 300 years for the CSNF cell occurs because there is a short period after 304B4 degradation ends and before Al-6061 degradation ends. Steel degradation causes pH to fall and aluminum alloy degradation causes it to rise (Section 6.6.5[a]). At the higher and lower flow rates simulated, the pH is not nearly as highly affected around 300 years because either the incoming liquid composition controls the pH or the degradation reactions control the pH.

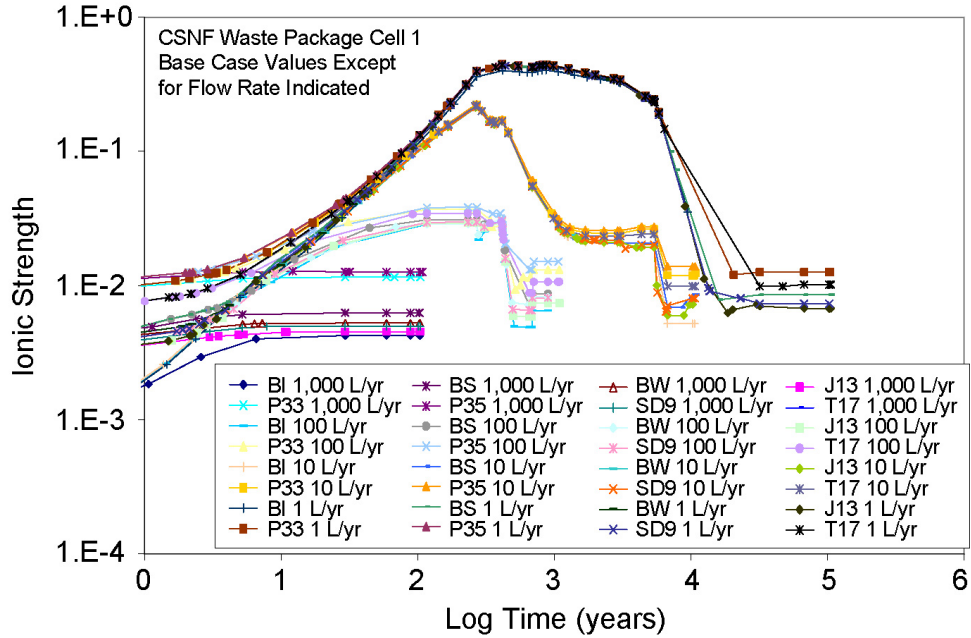
Time is an important factor because degradation reactions cease when the materials become fully degraded. When this happens, the ionic strength generally decreases to a level somewhat above the ionic strength of the incoming fluid. Although degradation reactions may have ceased, the liquid still must equilibrate with secondary mineral phases in the cell.

The functionality of liquid influx rate and time is included in the ionic strength abstractions as explained in Section 6.10.2.1[a]. They are not included in the pH abstractions. The pH abstractions define only the range of pH, so while this sensitivity analysis shows that there is a relationship between pH calculations and liquid influx rate and time, the pH ranges (minimum and maximum pH values) are not a function of these parameters.



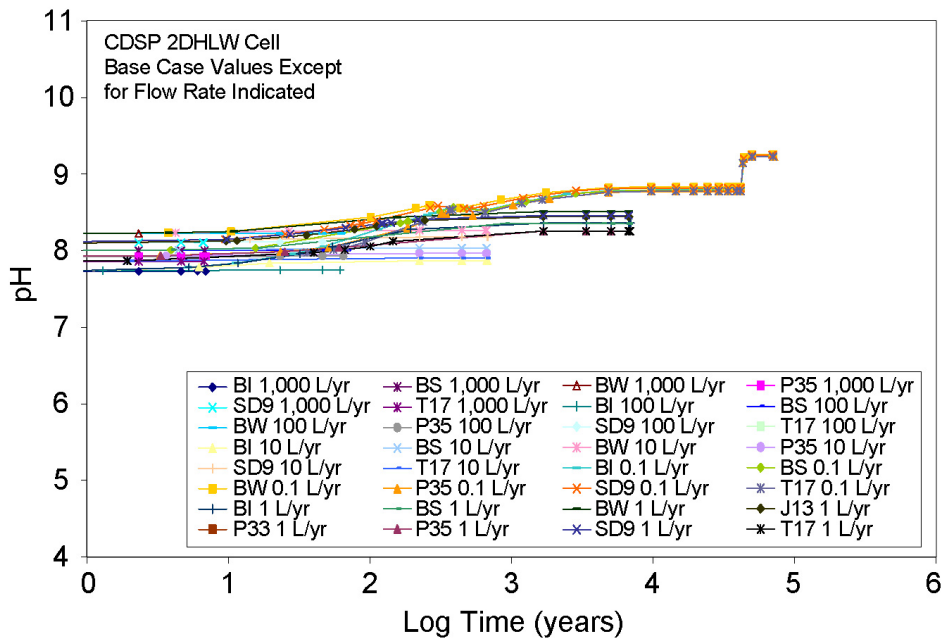
Source: Output DTN: SN0702PAIPC1CA.001, file: CSNF flow.xls.

Figure 6-26[a]. CSNF Liquid-Influx pH over Time for Various Seepage Compositions and Flow Rates



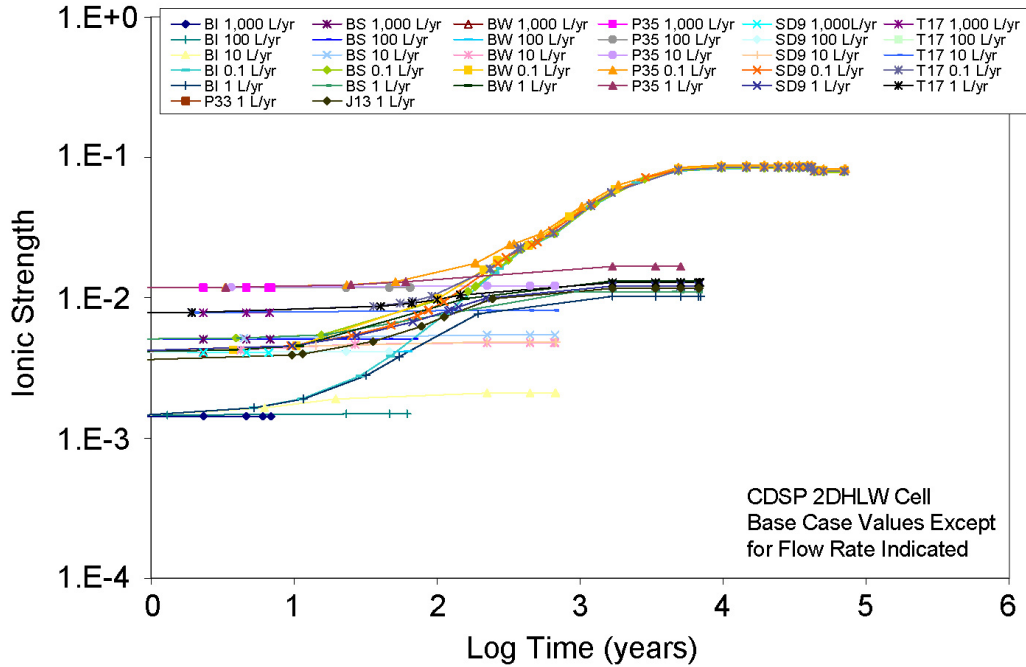
Source: Output DTN: SN0702PAIPC1CA.001, file: CSNF flow.xls.

Figure 6-27[a]. CSNF Liquid-Influx Ionic Strength over Time for Various Seepage Compositions and Flow Rates



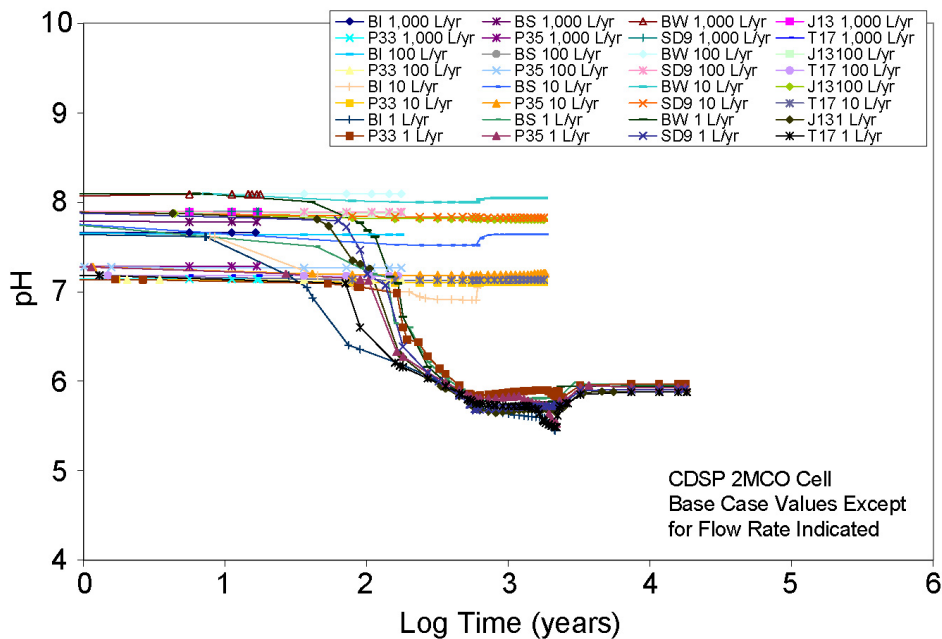
Source: Output DTN: SN0702PAIPC1CA.001, file: 2DHLW flow.xls.

Figure 6-28[a]. 2DHLW Liquid-Influx pH over Time for Various Seepage Compositions and Flow Rates



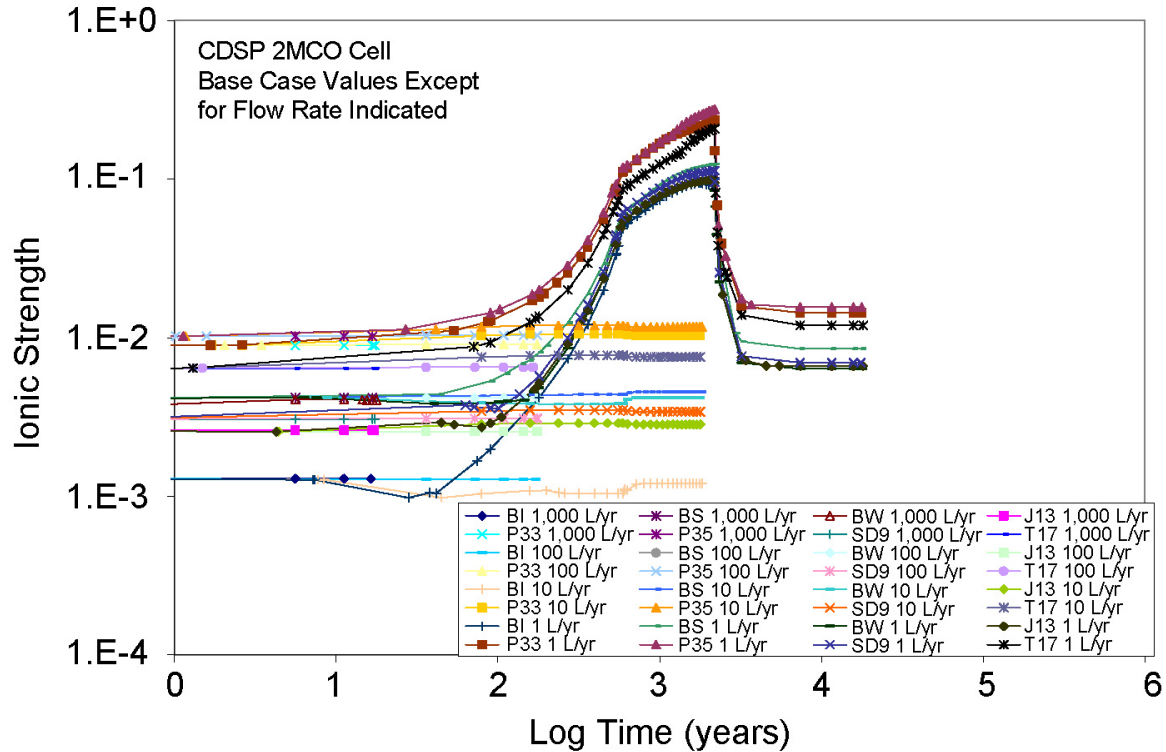
Source: Output DTN: SN0702PAIPC1CA.001, file: 2DHLW flow.xls.

Figure 6-29[a]. 2DHLW Liquid-Influx Ionic Strength over Time for Various Seepage Compositions and Flow Rates



Source: Output DTN: SN0702PAIPC1CA.001, file: 2MCO flow.xls.

Figure 6-30[a]. 2MCO Liquid-Influx pH over Time for Various Seepage Compositions and Flow Rates



Source: Output DTN: SN0702PAIPC1CA.001, file: 2MCO flow.xls.

Figure 6-31[a]. 2MCO Liquid-Influx Ionic Strength over Time for Various Seepage Compositions and Flow Rates

6.6.5[a] Degradation Rates

To assess the effects of variation in the degradation rates of cell materials, simulations using minimum and maximum degradation rate measurements were performed for each cell. For a given simulation, the degradation rates for all but one of the components were set at base case values while the degradation rate of the remaining component was set at a minimum or maximum value from Table 6-5[a] or Table 6-7[a]. Because the uncertainty in the specific surface areas of these materials is small in comparison to the range of reported degradation rates and because the degradation rates are bulk measurements that implicitly include the effects of specific surface areas, the degradation rate sensitivity analyses in this section include the effects of specific surface area. The sensitivity analysis simulations are listed in Table 6-15[a]. More detailed descriptions of these files are provided in spreadsheet *IPC file key revision 2.xls* in Output DTN: SN0702PAIPC1CA.001.

Table 6-15[a]. EQ6 Input/Output Files for Degradation Rate Sensitivity Simulations

| Cell | Adjusted Degradation Rate | Path | Input Files | Output Files |
|-------|---------------------------|-----------------|-------------|--------------|
| CSNF | Low CSNF | 1milyr\CSNFdr\ | adpr*.6i | adpr*.6o |
| | High CSNF | 1milyr\CSNFdr\ | bdpr*.6i | bdpr*.6o |
| | Low 304B4 | 1milyr\CSNFdr\ | cdpr*.6i | cdpr*.6o |
| | High 304B4 | 1milyr\CSNFdr\ | ddpr*.6i | ddpr*.6o |
| | Low 316 | 1milyr\CSNFdr\ | edpr*.6i | edpr*.6o |
| | High 316 | 1milyr\CSNFdr\ | fdpr*.6i | fdpr*.6o |
| | Low Al-6061 | 1milyr\CSNFdr\ | gdpr*.6i | gdpr*.6o |
| | High Al-6061 | 1milyr\CSNFdr\ | hdpr*.6i | hdpr*.6o |
| 2DHLW | Low HLWG | 1milyr\2DHLWdr\ | adpr*.6i | adpr*.6o |
| | High HLWG | 1milyr\2DHLWdr\ | bdpr*.6i | bdpr*.6o |
| | Low 304L | 1milyr\2DHLWdr\ | cdpr*.6i | cdpr*.6o |
| | High 304L | 1milyr\2DHLWdr\ | ddpr*.6i | ddpr*.6o |
| 2MCO | Low Al-1100 | 1milyr\2MCOdr\ | adpr*.6i | adpr*.6o |
| | High Al-1100 | 1milyr\2MCOdr\ | bdpr*.6i | bdpr*.6o |
| | Low 304L | 1milyr\2MCOdr\ | cdpr*.6i | cdpr*.6o |
| | High 304L | 1milyr\2MCOdr\ | ddpr*.6i | ddpr*.6o |

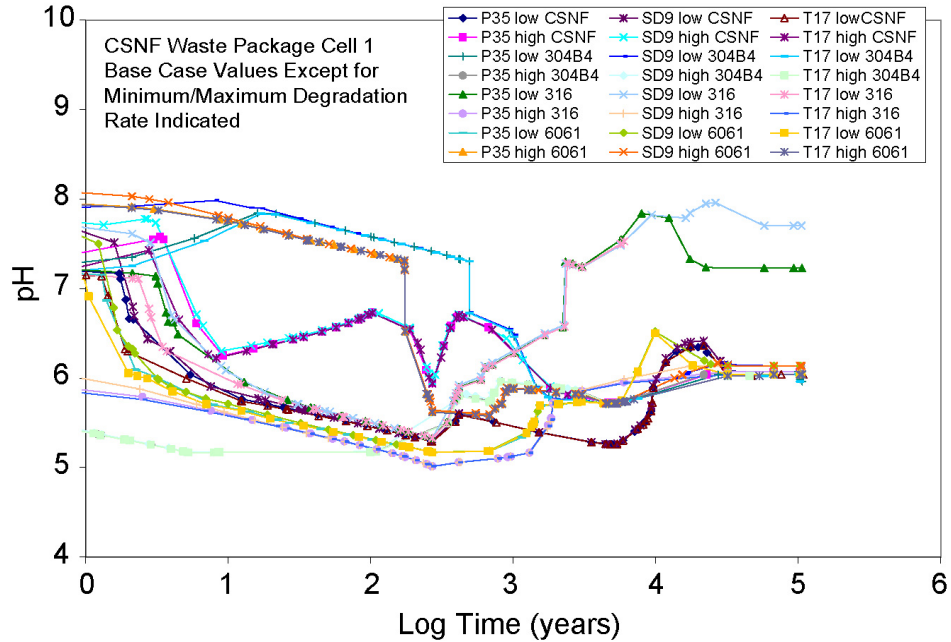
Source: Output DTN: SN0702PAIPC1CA.001.

NOTE: An asterisk (*) is a wildcard for an alphanumeric string.

Figures 6-32[a] through 6-37[a] display the results. These figures indicate that degradation rates affect both the pH and the ionic strength. The pH value can change by several units depending on the dominant degradation reaction. Ionic strength variation is generally less pronounced in the CSNF simulations, but in the 2DHLW and 2MCO simulations, ionic strength can vary by more than two orders of magnitude.

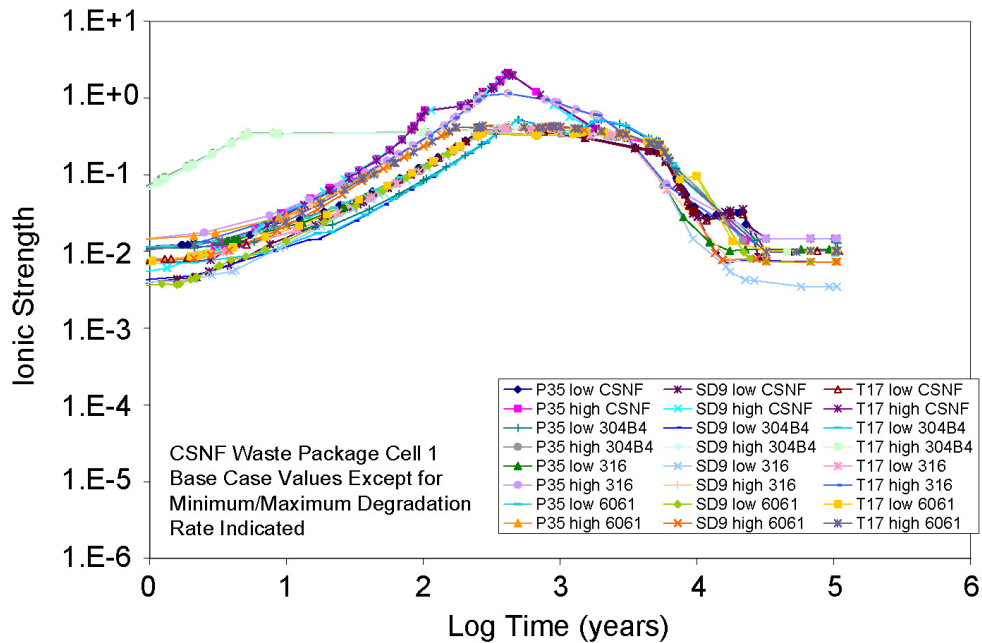
The pH tends to be below neutral when stainless steel degradation rates are high relative to the degradation rates of the waste forms and the aluminum alloys. Alternatively, pH tends to be above neutral when stainless steel rates are low relative to the other rates. This relationship is evident at early times and also at later times when materials are fully degraded. For example, in the “high 304L” simulations in the 2DHLW cell (Figure 6-34[a]), the Stainless Steel Type 304L degrades at a maximum rate and is completely degraded after approximately 500 years. Before the steel is gone, the relative degradation rate of steel to waste form is high and the pH is low. Afterwards, the ratio falls to zero, causing the pH to rise dramatically.

The ionic strength abstractions for the liquid influx case intrinsically include the effects of degradation rates because the results of base case IPC simulations are used to produce the abstractions (Section 6.10.2.1[a]). The pH abstractions, however, have little dependency to degradation rates because they only define the pH ranges (minimum and maximum pH values), which are primarily controlled by major buffering reactions and $p\text{CO}_2$. The small effect that degradation rates have on the pH range is captured by the effects of degradation rates on ionic strength (Section 6.10.1[a]).



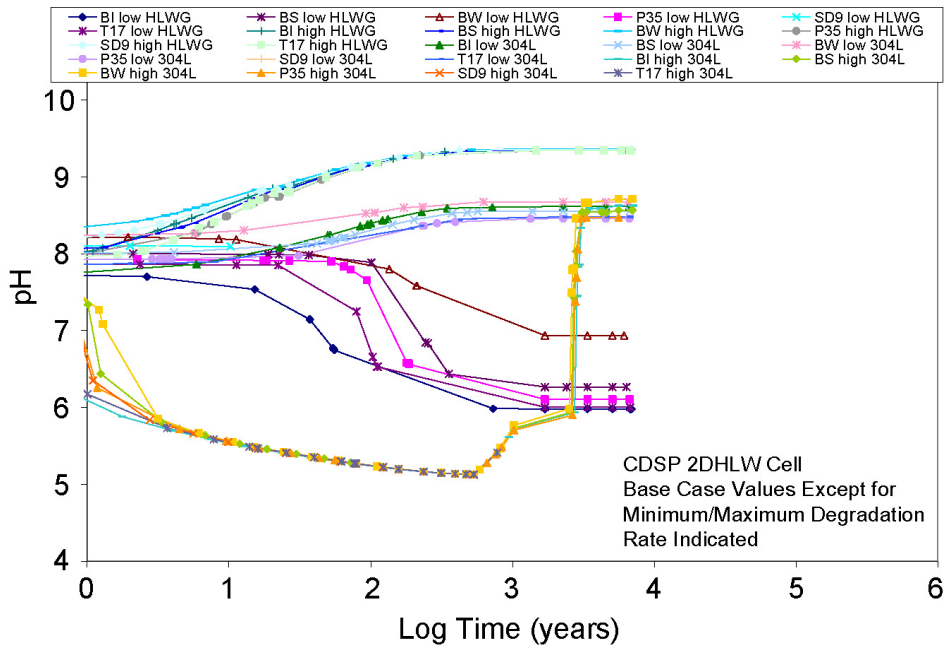
Source: Output DTN: SN0702PAIPC1CA.001, file: CSNF dr.xls.

Figure 6-32[a]. CSNF Liquid-Influx pH over Time for Various Seepage Compositions and Minimum or Maximum Degradation Rate Indicated



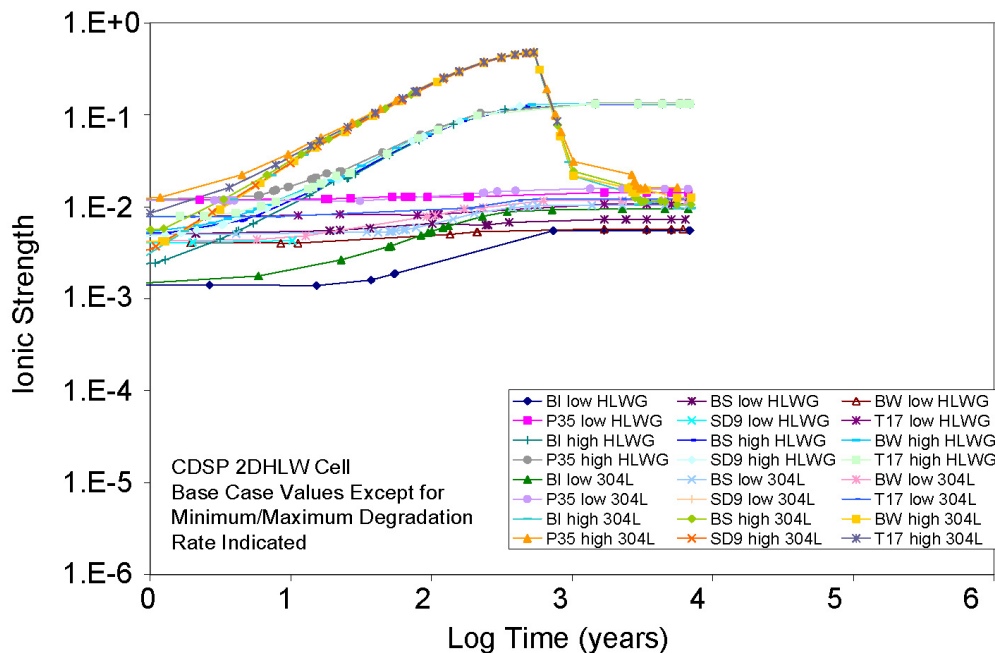
Source: Output DTN: SN0702PAIPC1CA.001, file: CSNF dr.xls.

Figure 6-33[a]. CSNF Liquid-Influx Ionic Strength over Time for Various Seepage Compositions and Minimum or Maximum Degradation Rate Indicated



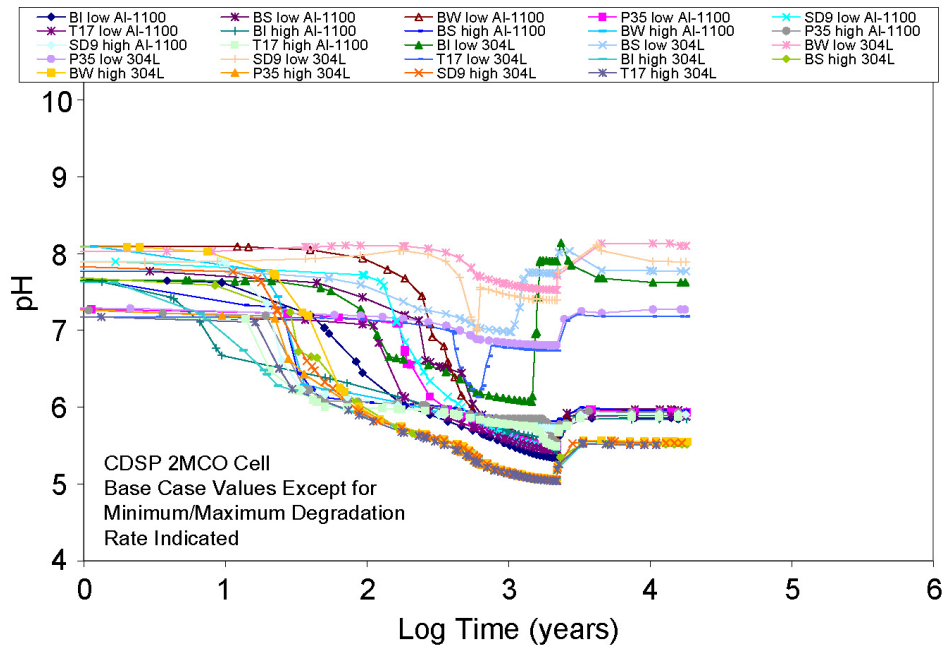
Source: Output DTN: SN0702PAIPC1CA.001, file: 2DHLW dr.xls.

Figure 6-34[a]. 2DHLW Liquid-Influx pH over Time for Various Seepage Compositions and Minimum or Maximum Degradation Rate Indicated



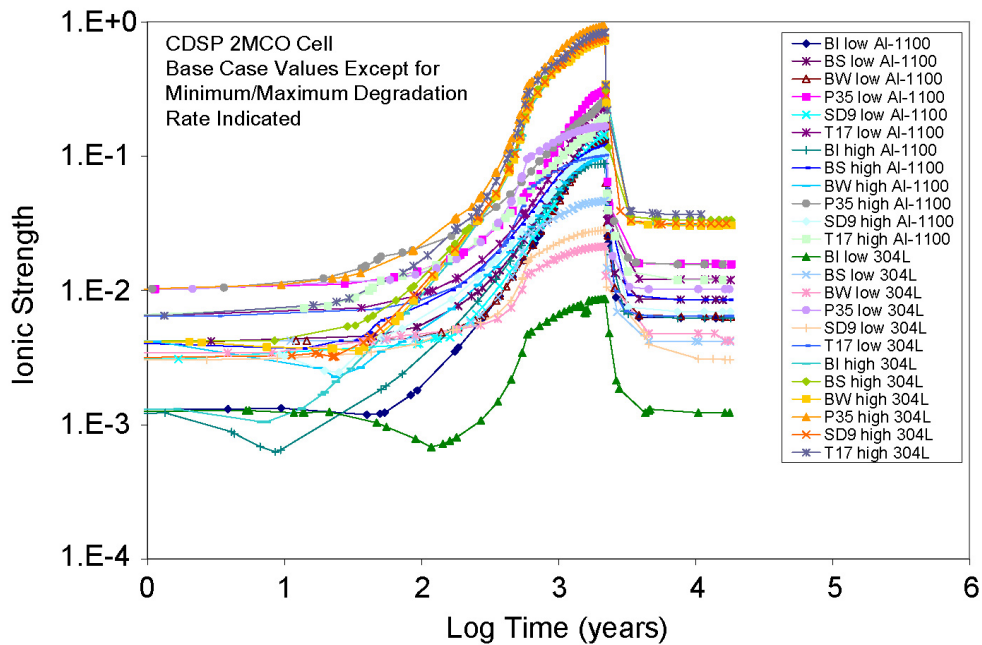
Source: Output DTN: SN0702PAIPC1CA.001, file: 2DHLW dr.xls.

Figure 6-35[a]. 2DHLW Liquid-Influx Ionic Strength over Time for Various Seepage Compositions and Minimum or Maximum Degradation Rate Indicated



Source: Output DTN: SN0702PAIPC1CA.001, file: 2MCO dr.xls.

Figure 6-36[a]. 2MCO Liquid-Influx pH over Time for Various Seepage Compositions and Minimum or Maximum Degradation Rate Indicated



Source: Output DTN: SN0702PAIPC1CA.001, file: 2MCO dr.xls.

Figure 6-37[a]. 2MCO Liquid-Influx Ionic Strength over Time for Various Seepage Compositions and Minimum or Maximum Degradation Rate Indicated

6.6.6[a] CDSP Configuration

The most numerous CDSP waste package in the design for the repository is the 5DHLW/1DSNF Long. This waste package contains five HLW glass canisters identical to the glass canisters in the 2MCO/2DHLW waste package. These canisters are placed radially around a core compartment that contains DSNF. Although this waste package is the most numerous of the CDSP waste packages, the total mass of uranium in all of the 2MCO/2DHLW waste packages exceeds the total uranium mass in all of the 5DHLW/1DSNF Long waste packages. That is the reason the IPC model simulates the 2MCO/2DHLW waste package.

For radionuclide transport calculations, the TSPA-LA model simulates the 5DHLW/1DSNF Long waste package containing Three Mile Island (TMI) spent nuclear fuel canisters (SNL 2007 [DIRS 177407]). The major differences between this waste package and the 2MCO/2DHLW waste package are that Cell 1a contains five HLW glass canisters instead of two, and Cell 1b contains UO_2 instead of uranium metal. Cell 2, which is modeled by *EBS Radionuclide Transport Abstraction* (SNL 2007 [DIRS 177407]), contains the Stainless Steel Type A-516 dividers and fuel support plates outside the waste form canisters.

The waste form IPC abstractions in this addendum are for pH, ionic strength, and fluoride. Because Cell 1a of the 5DHLW/1DSNF Long contains the same components in the same proportion as the 2MCO/2DHLW, there would be no differences in the Cell 1a scaled degradation rates. The only difference in the conceptual model would be the total volumes of materials and water, which would be 2.5 times larger (5 canisters instead of 2). This is important only with respect to the turnover rate, which affects ionic strength. For a given liquid influx rate, the turnover rate would be 40% (2/5) of the turnover rate of the 2MCO/2DHLW waste package. As explained in Section 6.6.1[a], a lower turnover rate increases the effects of degradation rates, which in turn affect ionic strength. However, a reduction of 60% in the turnover rate is well within the order-of-magnitude uncertainty of the ionic strength abstraction (Section 6.10.8.2[a]). The fluoride abstraction, which is only a function of ionic strength (Section 6.10.3[a]), is therefore also negligibly affected. The pH abstraction is not affected at all because it is not a function of turnover rate.

Like the N-Reactor fuel and CSNF, the TMI fuel will quickly degrade to schoepite in the presence of water. Schoepite is the dominant alkalinity buffer in Cell 1b of the 2MCO/2DHLW waste package (Section 6.10.1[a]) and will also be the dominant alkalinity buffer in Cell 1b of the 5DHLW/1TMI Long waste package. Equilibrium dissolution of schoepite prevents pH from rising much above 7 at high carbon dioxide fugacity and much above 9 at low carbon dioxide fugacity (Sections 6.3.4[a] and 6.10.1[a]). This is true even in the presence of low density concrete, which is also included in the TMI canisters. The amount of buffer capacity from schoepite in Cell 1b far exceeds the alkalinity that the concrete could provide. On the acid side, degradation of the stainless steel structural materials, which are present regardless of the spent nuclear fuel type, produce oxides such as trevorite that define the pH minimum (Section 6.10.1[a]). Thus, the pH abstractions for N-Reactor fuel (2MCO) can be used for TMI fuel.

Ionic strength and fluoride abstractions for Cell 1b of the 5DHLW/1TMI Long waste package can also be approximated by the 2MCO abstractions because the degradation products are

similar. Other effects are small compared to the order of magnitude uncertainty of the ionic strength abstraction. The fluoride abstraction, which is a simple function of ionic strength (Section 6.10.3[a]), is therefore also negligibly affected.

Another fuel that accounts for a large number of DSNF canisters in CDSP waste packages is mixed-oxide fuel. This fuel consists of a mixture of uranium and plutonium oxide pellets. Because it contains large quantities of uranium oxide, which will rapidly degrade to schoepite, the pH range will be limited by the same pH buffers as the 2MCO cell. This pH range will prevent plutonium from becoming highly soluble and affecting the ionic strength and fluoride concentrations. Thus, the abstractions for the 2MCO cell are appropriate for the mixed-oxide fuel DSNF.

Carbide fuels stabilized in a graphite matrix also account for a large number of DSNF canisters. Among these fuels, the fuel from the Fort St. Vrain reactor accounts for about 90% of the heavy metal mass (DOE 2001 [DIRS 154304], p. 39). These fuels are composed of carbide particles coated with pyrolytic carbon. The Fort St. Vrain fuel particles have an additional coating of silicon carbide. The particles are bonded together by a highly stable carbonaceous matrix material. The coatings and matrix will degrade very slowly (worksheet "graphite" in file *Misc_Materials.xls* in DTN: MO0409SPAACRWP.000 [DIRS 172059]) and therefore will not influence the pH. The dominant reaction that will affect pH in these fuel canisters will be the degradation of stainless steel, which directs pH to the 5 to 6 range when it is the dominant reaction (e.g., Figure 6-36[a]). The ANC buffer capacity produced by the same reaction will prevent the pH from going much below 5 (Section 6.10.1.1[a]). This pH is within the range predicted for Cell 1b and permits the use of the Cell 1b pH abstractions for this group of DSNF. The ionic strength and fluoride concentrations will also be similar to the Cell 1b 2MCO (N-Reactor fuel) abstractions because (1) relative humidity controls ionic strength in the vapor influx case, and (2) the corrosion of stainless steel is the primary contributor to ionic strength in the liquid influx case.

The other major group of DSNF included in a large number of CDSP waste packages is aluminum-based fuel. The fuels from this group are composed of uranium aluminide, uranium silicide, and uranium oxide dispersed in a continuous aluminum phase and clad with aluminum (DOE 2001 [DIRS 154304], p. 32). The degradation rates of this fuel will be similar to the degradation rates of aluminum alloys. The degradation of aluminum alloy 1100 is already included in the Cell 1b abstractions because it is a component of the N-Reactor waste form cell. However, its corrosion has little effect on pH (Figure 6-22 of parent report). Corrosion of alloy 6061 generates some alkalinity due to the release of Mg^{2+} , but it does not cause the pH to go much higher than 8 (Figure 6-17 of parent report). Because this pH is in the range of the Cell 1b (N-Reactor fuel) pH abstractions, the Cell 1b abstractions can also be used to represent aluminum-based DSNF waste form cells. The ionic strength and fluoride abstractions for Cell 1b can also be used for this type of DSNF because (1) relative humidity controls ionic strength in the vapor influx case, (2) the corrosion of stainless steel is still a primary contributor to ionic strength in the liquid influx case, and (3) any increase in ionic strength caused by degradation of aluminum-based fuels in the liquid influx case is expected to be within the large ionic strength uncertainty propagated to the TSPA-LA model.

6.7[a] SURFACE COMPLEXATION MODEL OUTPUT

Not used in addendum.

6.8[a] SURFACE COMPLEXATION DEMONSTRATION

Not used in addendum.

6.9[a] APPLICATION OF SURFACE COMPLEXATION MODEL TO THE BATCH REACTOR MODEL RESULTS

Not used in addendum.

6.10[a] MODEL ABSTRACTION

The methods used to abstract results from the IPC process model have been re-examined in the following ways:

- Maximum and minimum pH values are abstracted from titration simulations as a function of pCO₂ and ionic strength. Values of pCO₂ used in TSPA-LA simulations are determined from abstractions developed in *Engineered Barrier System: Physical and Chemical Environment* (SNL 2007 [DIRS 177412]). Ionic strength is determined from abstractions developed in this addendum (Section 6.10.2[a]).
- Under liquid influx conditions, ionic strength is abstracted as a function of liquid influx rate and time since waste package breach. Under vapor influx conditions, ionic strength is abstracted as a function of waste package RH. Values of liquid influx rate used in TSPA-LA simulations are determined from the Engineered Barrier System flow submodel developed in *EBS Radionuclide Transport Abstraction* (SNL 2007 [DIRS 177407]), and values of waste package RH are determined from abstractions developed in *Multiscale Thermohydrologic Model* (SNL 2007 [DIRS 181383]).
- Maximum fluoride concentration is abstracted as a function of ionic strength.

Details of these differences are described and justified in the following subsections.

6.10.1[a] pH Abstractions

As discussed in Section 6.3.4[a], the degradation of fuels and structural materials in the waste form cells generates ANC and BNC primarily in the form of secondary minerals such as schoepite and trevorite. The pH range in a waste form cell is determined by titrating the contents of a waste form cell with acid and base. These titrations are used to identify the pH buffering minerals in the cells, define the pH limits, and quantify the buffer capacities.

The liquid influx and vapor influx pH abstractions are presented and discussed in Sections 6.10.1.1[a] and 6.10.1.2[a]. Uncertainties in the pH abstractions are discussed in Section 6.10.8.1[a]. Abstraction implementation is explained in Section 6.10.9[a].

6.10.1.1[a] Liquid Influx

Titration curves were performed for a range of ionic strength and $p\text{CO}_2$ (negative log of the partial pressure of carbon dioxide in bars). Each simulation involved in the pH abstractions is listed in Table 6-16[a]. For these simulations, SD9 water was used as the base case liquid influx composition. SD9 is representative of the most frequent type of pore water sampled at Yucca Mountain (SNL 2007 [DIRS 177412], Section 6.6.5). However, after ten years of degradation, the incoming water composition has a negligible effect on the accumulation rates of the pH buffering minerals. At ten years, the amounts of the minerals are approximately the same regardless of which Yucca Mountain water or basalt water in Section 4.1.2[a] is used as the incoming liquid (Output DTN: SN0702PAIPC1CA.001, files: *Accum\CSNF\DPrates.xls*, *Accum\2DHLW\DPrates.xls*, and *Accum\2MCO\DPrates.xls*).

Figures 6-38[a] through 6-40[a] show the titration curves for each of the liquid influx base case simulations. Schoepite and trevorite are the primary mineral buffers that limit the pH range in the CSNF and 2MCO cells. Under base case conditions in the CSNF cell, schoepite provides a BNC of approximately 0.027 molal/year (0.27 molal per ten years), while trevorite provides an ANC of approximately 0.039 molal/year. In the 2MCO cell under base case conditions, schoepite provides a BNC greater than 0.05 molal/year, while trevorite provides an ANC of approximately 0.0006 molal/year. For the 2DHLW cell, the corresponding buffers are carbon dioxide and trevorite. Under base case conditions in this cell, carbon dioxide provides a BNC greater than 0.05 molal/year, while trevorite provides an ANC of approximately 0.001 molal/year.

Each of these ANC and BNC accumulation rates is far in excess of the rates of any reasonable external source of acid or base. For example, to overcome the lowest accumulation rate (0.0006 molal/year of ANC in the 2MCO cell) an extremely high flow rate (1,000 L/yr) of an acidic liquid influx (such as 0.5 millimolar HCl) would have to be maintained. In this example, the acidity of the liquid influx is calculated by multiplying the lowest ANC accumulation rate (0.0006 molal/year) by the volume of water in the cell (396 L as presented in Table 6-3[a]) and then dividing by 1,000 L/yr. The pH of a 0.5 millimolar HCl solution is approximately 3.3 ($= -\log(0.0005 \text{ molal})$). There is no realistic source for a sustained 1,000 L/yr influx rate of a liquid with this much acidity. Similarly, there is no external source of alkalinity that could overcome any of the BNC accumulation rates.

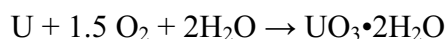
The largest internal sources of acid and base are the degradation reactions themselves, but even when their rates are increased to maximum values they also cannot overcome the rates of ANC and BNC accumulation. Corrosion of stainless steel by itself produces some acidity (as shown in Figure 6-22 of the parent report). However, it will not send the pH below the stability field of trevorite because the corrosion reaction produces trevorite. Thus, any increase in the stainless steel corrosion rate will be accompanied by an increase in the production of trevorite. In this way, the corrosion of stainless steel is self buffering.

Figure 6-22 of the parent report indicates that the corrosion of A516 carbon steel by itself can cause the pH to decrease below 2. Carbon steel is not a material in any of the waste form cells, but the acidifying effect of carbon steel is worth mentioning because it is included in the figure cited above. The primary causes of the large pH decrease in the carbon steel simulations are:

(1) the acidifying effect of the oxidation of sulfur, which is an important component the carbon steel (Table 6-4 of parent report), (2) the negligible concentration of nickel in the carbon steel, which would provide pH buffer capacity as it does for stainless steel, and (3) the exclusion of surface hydrolysis reactions in the simulations, which would have provided significant pH buffering (Section 6.7.1 of parent report).

Corrosion of aluminum alloys adds alkalinity to the solution due to the addition of cations to solution. The pH increase observed in Figure 6-17 of the parent report for the corrosion of alloy 6061 is primarily due to the oxidation of magnesium to the free divalent cation, Mg^{2+} . Alloy 1100 does not have magnesium as a major component; thus, its corrosion does not raise pH nearly as much as the corrosion of alloy 6061 (Figure 6-22 of the parent report). The rates of alkalinity produced by aluminum alloy corrosion are small in comparison to the rate at which schoepite will accumulate in these cells; therefore, corrosion of these alloys will not surpass the BNC accumulation rates in these cells.

The degradation of UO_2 (e.g., CSNF and TMI) and uranium metal (e.g., N-Reactor fuel) is defined by the following reactions:



The uranium metal of N-Reactor fuel has no significant impurities, and thus, consistent with the reaction above, simulations of N-Reactor fuel degradation show no effect on pH (Figure 6-22 of the parent report). CSNF, however, has impurities such as cesium and barium that, when released, raise pH (Table 6-6 of parent report). The oxidation of other impurities in CSNF, such as molybdenum and technetium, largely cancel this effect, but the net effect is an increase in pH (Figure 6-17 of the parent report). The concurrent precipitation of schoepite during CSNF degradation, however, prevents the pH from rising substantially. Thus, as in the case of stainless steel, the CSNF degradation reaction is self buffering in this way.

The only other degradation reaction in a waste form cell that has an important effect on pH is that of HLW glass. Degradation of HLW glass releases sodium, which causes the pH to increase. There is no schoepite precipitation in this cell to buffer the increased alkalinity; thus, only dissolution of carbon dioxide provides significant BNC (Figure 6-39[a]). The carbon dioxide BNC will never be exhausted by the alkalinity produced by HLW degradation because there is a vast reservoir of carbon dioxide.

Because there are no processes in the conceptual model that could add acid or base at rates that would overcome the sources of pH buffer capacity identified above, the minimum and maximum pH values in each cell are defined as the pH values in the titrations where these buffering phases are either exhausted (schoepite and trevorite) or exceed an accumulation rate of 0.05 mole/yr. For example, in the CSNF base case the pH minimum and maximum are approximately 5.5 and 8.3, respectively (Figure 6-38[a]). Each arrow in this figure (and in Figures 6-39[a] and 6-40[a]) indicates only the range on the x-axis (H^+ or OH^- Added) where the indicated phase buffers the pH. The vertical placement of each arrow is meaningless except that arrows near the

upper titration curve correspond to base titrations and those near the lower curve correspond to acid titrations.

The results of all titration simulations are summarized in files *Abstr CSNF Seepage pH.xls*, *Abstr 2DHLW Seepage pH.xls*, and *Abstr 2MCO Seepage pH.xls* in Output DTN: SN0702PAIPC1CA.001. These files include extrapolations to high ionic strengths and to a $p\text{CO}_2$ value (negative log of the partial pressure of carbon dioxide in bars) of 1.5. Extrapolations were performed to expand the ranges of the IPC model abstractions to accommodate the ranges of $p\text{CO}_2$ and ionic strength in the TSPA-LA model. The extrapolations are based on the trends in the simulations. Maximum values of pH decrease approximately linearly as $p\text{CO}_2$ decreases, so extrapolation to a $p\text{CO}_2$ value of 1.5 is performed using the linear forecast function of Excel. Unlike $p\text{CO}_2$ effects, the effect of ionic strength on pH limits is variable. For ionic strength, the forecast function is only used when it tends to increase the pH range; otherwise, the pH limit of the simulation with the highest ionic strength is used. All interpolations in these spreadsheets are linear interpolations between the closest values calculated in the simulations.

Plots of the pH abstractions are shown in Figures 6-41[a] through 6-43[a]. In each cell, maximum pH values decrease markedly as $p\text{CO}_2$ decreases. Because minimum pH values are below the pH range of significant carbonic acid dissociation, minimum pH values are unaffected by $p\text{CO}_2$.

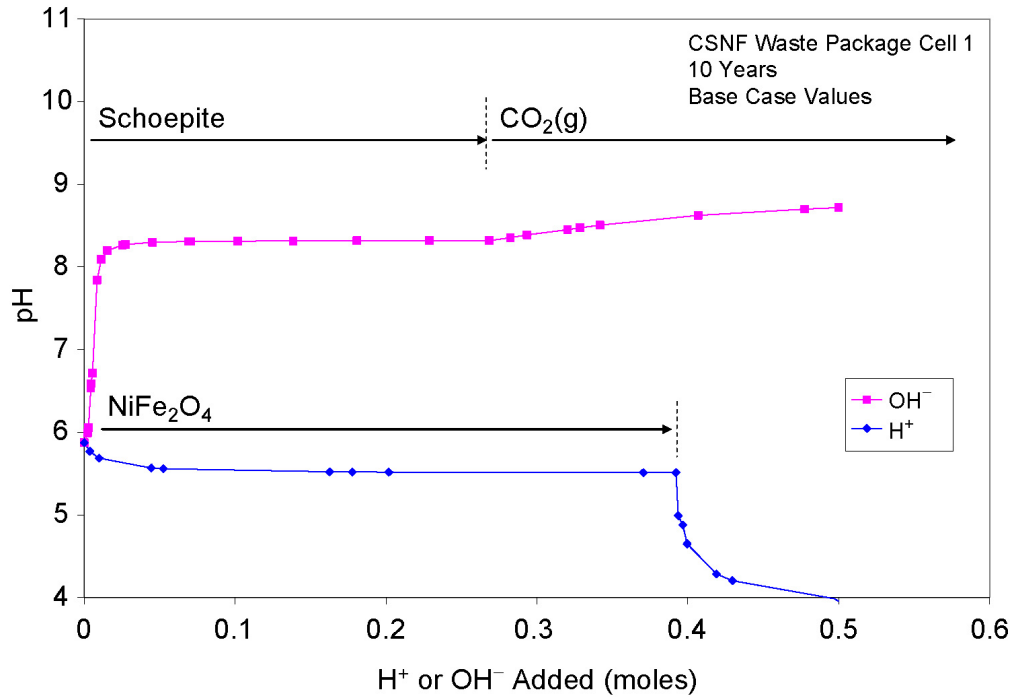
The effect of ionic strength on the pH range varies. Its largest effect is in the pH maximum values of the 2DHLW cell (Figure 6-42[a]). In this cell, the fugacity of CO_2 , not schoepite, provides the BNC (Section 6.3.4[a]), which is why the pH maximum values in the 2DHLW cell are generally much higher than in the other cells. Increasing ionic strength tends to increase the pH range, but not in all cases. The opposite trend is observed for maximum pH values in the CSNF simulations and in the higher ionic strength 2MCO simulations. When maximum pH decreases with increasing ionic strength, the pH maximum extrapolations are set at the values of the simulations with the highest ionic strengths.

Table 6-16[a]. EQ6 Input/Output Files Used in Liquid-Influx pH Abstractions

| Cell | pCO ₂ | Ionic Strength (molal) | Path | Input Files | Output Files |
|-------|-------------------------------------|------------------------|------------------------|--|--|
| CSNF | 4 | 0.03 to 0.04 | <i>Accum\CSNFC-4\</i> | <i>dpr*.6i,</i> <i>HClDpr.6i,</i> <i>LiOHdpr.6i</i> | <i>dpr*.6o, dprpSD9.6p, H*.6o, L*.6o,</i> <i>H*.elem_aqu, L*.elem_aqu,</i> <i>H*.elem_m_a, L*.elem_m_a,</i> <i>H*.elem_tot, L*.elem_tot, H*.min_info,</i> <i>L*.min_info</i> |
| | 3 | 0.03 to 0.04 | <i>Accum\CSNF\</i> | | |
| | 2 | 0.03 to 0.04 | <i>Accum\CSNFC-2\</i> | | |
| | 4 (# = a) 3 (# = b) 2 (# = c) | 0.4 to 0.9 | <i>Accum\CSNFis\</i> | <i>#dpr*.6i,</i> <i>#HClDpr.6i,</i> <i>#LiOHdpr.6i</i> | <i>#dpr*.6o, #dprpSD9.6p, #H*.6o, #L*.6o,</i> <i>#H*.elem_aqu, #L*.elem_aqu,</i> <i>#H*.elem_m_a, #L*.elem_m_a,</i> <i>#H*.elem_tot, #L*.elem_tot,</i> <i>#H*.min_info, #L*.min_info</i> |
| 2DHLW | 4 | 0.005 to 0.006 | <i>Accum\2DHLWc-4\</i> | <i>dpr*.6i,</i> <i>HClDpr.6i,</i> <i>LiOHdpr.6i</i> | <i>dpr*.6o, dprpSD9.6p, H*.6o, L*.6o,</i> <i>H*.elem_aqu, L*.elem_aqu,</i> <i>H*.elem_m_a, L*.elem_m_a,</i> <i>H*.elem_tot, L*.elem_tot, H*.min_info,</i> <i>L*.min_info</i> |
| | 3 | 0.005 to 0.006 | <i>Accum\2DHLW\</i> | | |
| | 2 | 0.005 to 0.006 | <i>Accum\2DHLWc-2\</i> | | |
| | 4 (# = d) 3 (# = e) 2 (# = f) | 0.03 to 0.04 | <i>Accum\2DHLWis\</i> | <i>#dpr*.6i,</i> <i>#HClDpr.6i,</i> <i>#LiOHdpr.6i</i> | <i>#dpr*.6o, #dprpSD9.6p, #H*.6o, #L*.6o,</i> <i>#H*.elem_aqu, #L*.elem_aqu,</i> <i>#H*.elem_m_a, #L*.elem_m_a,</i> <i>#H*.elem_tot, #L*.elem_tot,</i> <i>#H*.min_info, #L*.min_info</i> |
| | 4 (# = a) 3 (# = b) 2 (# = c) | 0.3 to 0.4 | | | |
| 2MCO | 4 | 0.003 to 0.004 | <i>Accum\2MCOc-4\</i> | <i>dpr*.6i,</i> <i>HClDpr.6i,</i> <i>LiOHdpr.6i</i> | <i>dpr*.6o, dprpSD9.6p, H*.6o, L*.6o,</i> <i>H*.elem_aqu, L*.elem_aqu,</i> <i>H*.elem_m_a, L*.elem_m_a,</i> <i>H*.elem_tot, L*.elem_tot, H*.min_info,</i> <i>L*.min_info</i> |
| | 3 | 0.003 to 0.004 | <i>Accum\2MCO\</i> | | |
| | 2 | 0.003 to 0.004 | <i>Accum\2MCOc-2\</i> | | |
| | 4 (# = d) 3 (# = e) 2 (# = f) | 0.04 | <i>Accum\2MCOis\</i> | <i>#dpr*.6i,</i> <i>#HClDpr.6i,</i> <i>#LiOHdpr.6i</i> | <i>#dpr*.6o, #dprpSD9.6p, #H*.6o, #L*.6o,</i> <i>#H*.elem_aqu, #L*.elem_aqu,</i> <i>#H*.elem_m_a, #L*.elem_m_a,</i> <i>#H*.elem_tot, #L*.elem_tot,</i> <i>#H*.min_info, #L*.min_info</i> |
| | 4 (# = a) 3 (# = b) 2 (# = c) | 0.4 | | | |

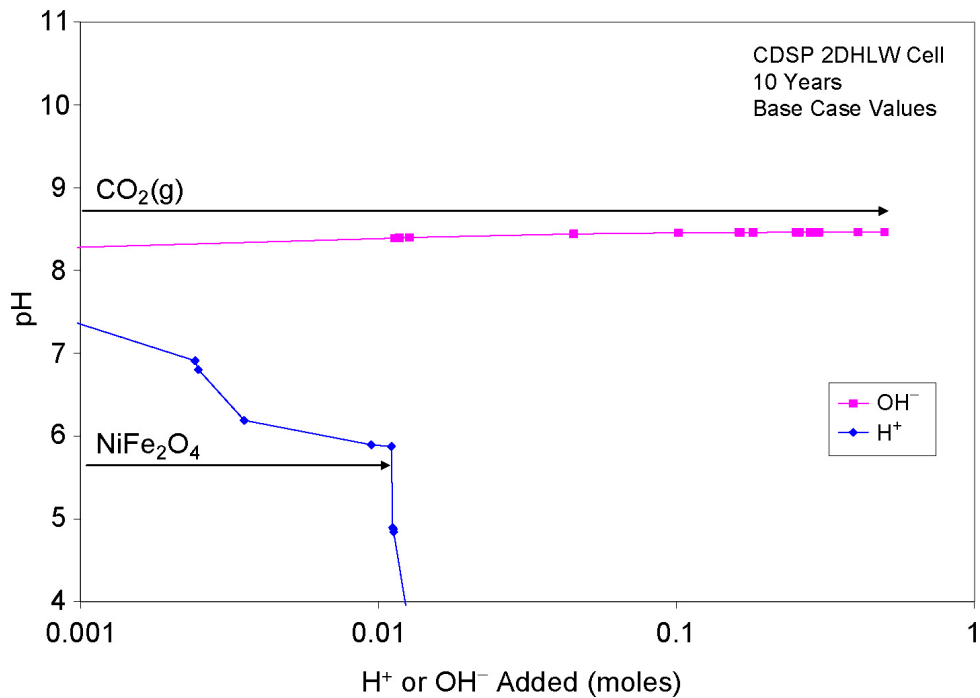
Source: Output DTN: SN0702PAIPC1CA.001.

NOTE: An asterisk (*) is a wildcard for an alphanumeric string.



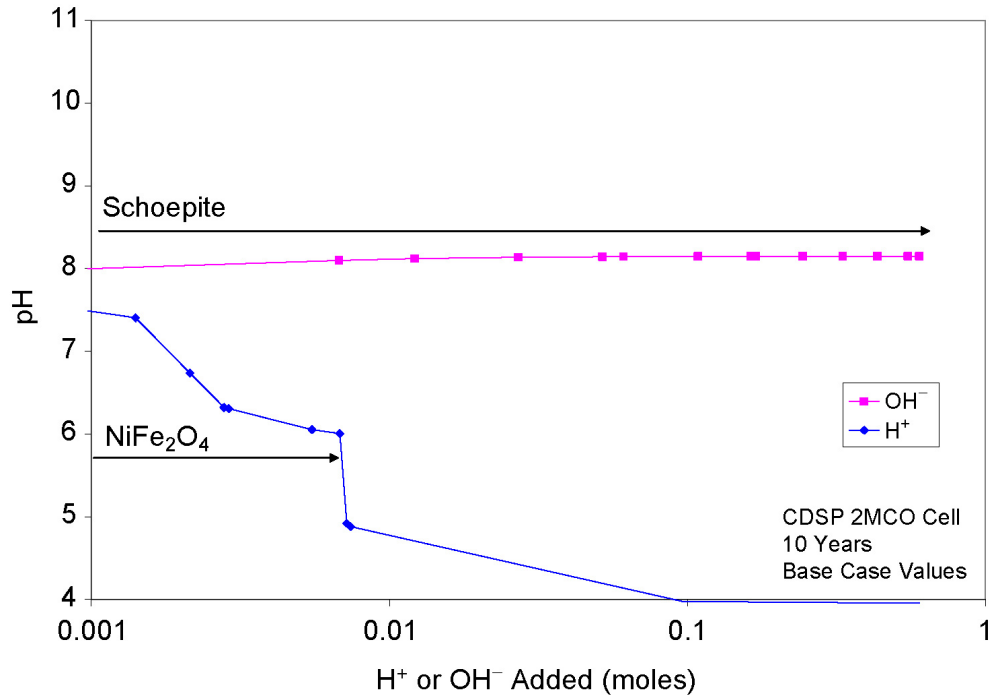
NOTE: Data from Output DTN: SN0702PAIPC1CA.001, file: *DPr pHbuf.xls*.

Figure 6-38[a]. CSNF Liquid-Influx Base Case pH Titration Curves after Ten Years of Degradation



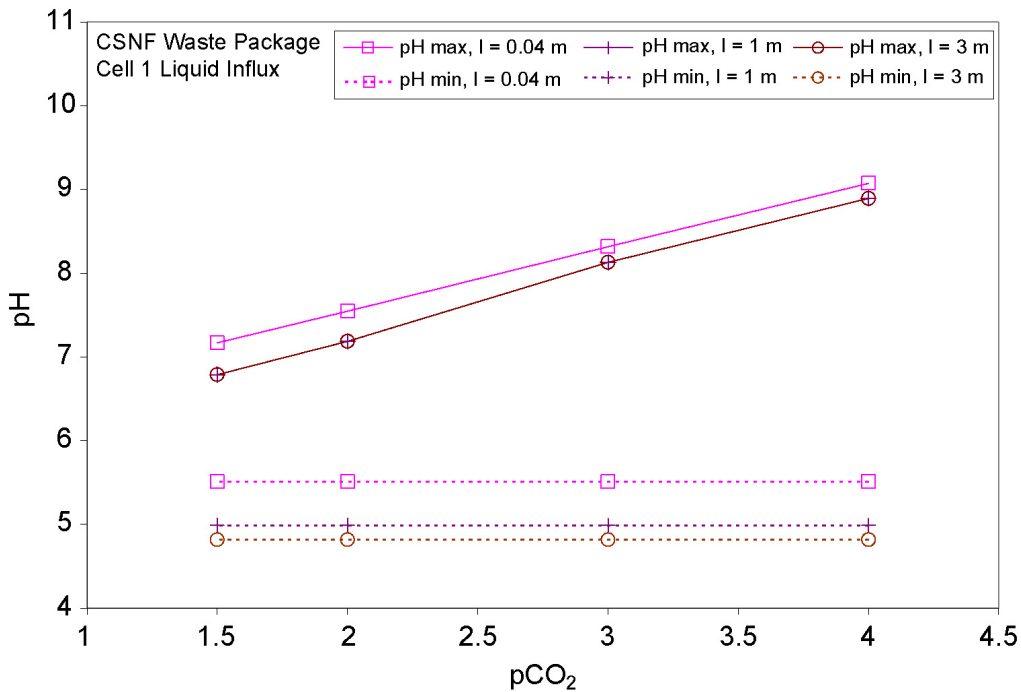
NOTE: Data from Output DTN: SN0702PAIPC1CA.001, file: *2DHLW pHbuf.xls*.

Figure 6-39[a]. 2DHLW Liquid-Influx Base Case pH Titration Curves after Ten Years of Degradation



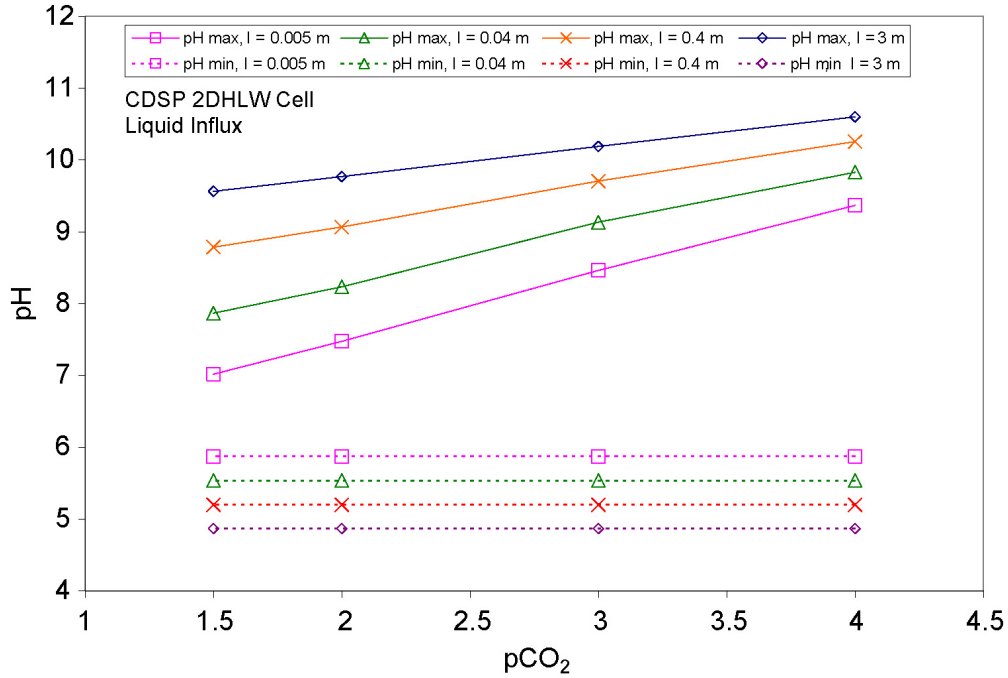
NOTE: Data from Output DTN: SN0702PAIPC1CA.001, file: 2MCO pHbuf.xls.

Figure 6-40[a]. 2MCO Liquid-Influx Base Case pH Titration Curves after Ten Years of Degradation



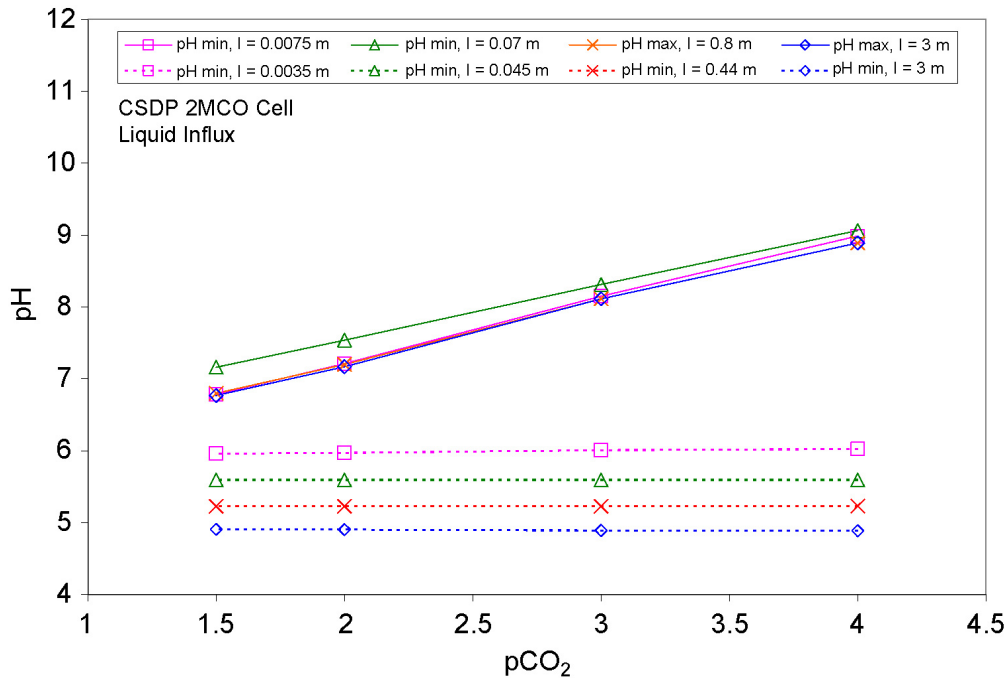
NOTE: Data from Output DTN: SN0702PAIPC1CA.001, file: Abstr CSNF Seepage pH.xls.

Figure 6-41[a]. CSNF Liquid-Influx Minimum and Maximum pH Values versus pCO_2 for Various Ionic Strengths



NOTE: Data from Output DTN: SN0702PAIPC1CA.001, file: *Abstr 2DHLW Seepage pH.xls*.

Figure 6-42[a]. 2DHLW Liquid-Influx Minimum and Maximum pH Values versus pCO₂ for Various Ionic Strengths



NOTE: Data from Output DTN: SN0702PAIPC1CA.001, file: *Abstr 2MCO Seepage pH.xls*.

Figure 6-43[a]. 2MCO Liquid-Influx Minimum and Maximum pH Values versus pCO₂ for Various Ionic Strengths

6.10.1.2[a] Vapor Influx

As in the liquid influx pH abstraction, titrations for the vapor influx case were performed for a range of pCO₂ and ionic strength. Each simulation in these abstractions is listed in Table 6-17[a]. For these simulations, pure water was used as the vapor influx and condensing liquid composition (Table 6-1[a]).

Table 6-17[a]. EQ6 Input/Output Files Used in Vapor-Influx pH Abstractions

| Cell | pCO ₂ | Ionic Strength (molal) | Path | Input Files | Output Files | | | |
|-------|-------------------------------------|-------------------------|-----------------|--|---|-------------------------------------|--------------|-----------------|
| CSNF | 4 | 0.04 to 0.05 (# = b) | Accum\ndCSNFc4\ | #dpr*.6i, HCl dpr#.6i, LiOHdpr#.6i | #dpr.6o, #dpr.6p, H*.6o, L*.6o, H*.min_info, L*.min_info | | | |
| | | 0.4 to 0.5 (# = a) | | | | | | |
| | 3 | 0.04 to 0.05 (# = b) | Accum\ndCSNF\ | | | | | |
| | | 0.4 to 0.5 (# = a) | | | | | | |
| | 2 | 0.04 to 0.05 (# = b) | Accum\ndCSNFc2\ | | | | | |
| | | 0.4 to 0.5 (# = a) | | | | | | |
| 2DHLW | 4 (# = a) 3 (# = b) 2 (# = c) | 0.005 | Accum\2DHLWnd\ | #dpr*.6i, #HCl dpr.6i, #LiOHdpr.6i | #dpr.6o, #dpr.6p, #H*.6o, #L*.6o, #H*.min_info, #L*.min_info | | | |
| | | | | | | 4 (# = d) 3 (# = e) 2 (# = f) | 0.04 | Accum\2DHLWndi\ |
| | | | | | | | | |
| | 4 (# = a) 3 (# = b) 2 (# = c) | 0.003 to 0.004 | Accum\2MCOnd\ | | | | | |
| | | | | | | 4 (# = d) 3 (# = e) 2 (# = f) | 0.04 to 0.05 | Accum\2MCOndi\ |
| | | | | | | | | |

Source: Output DTN: SN0702PAIPC1CA.001.

NOTE: An asterisk (*) is a wildcard for an alphanumeric string.

The titration results of the vapor influx case (e.g., files *pHbuf bc.xls*, *2DHLW pHbuf nd.xls*, and *2MCO pHbuf nd.xls* in Output DTN: SN0702PAIPC1CA.001) are essentially identical to those of the liquid influx case. Consequently, the pH abstractions (files *Abstr CSNF No Drip pH.xls*, *Abstr 2DHLW No Drip pH.xls*, and *Abstr 2MCO No Drip pH.xls* in Output DTN: SN0702PAIPC1CA.001) are also essentially identical to the pH abstractions of the liquid influx case. The reason for the similarity is that the secondary minerals and carbon dioxide fugacity control the pH buffering in the cells regardless of the source of water (Section 6.10.1[a]).

Because the differences are almost indiscernible from the results of the liquid influx simulations, figures of the vapor influx case titrations are not presented here. Instead, these figures can be retrieved from spreadsheets in Output DTN: SN0702PAIPC1CA.001 from the same paths as in Table 6-17[a]. Charts of the vapor influx pH abstractions as a function of $p\text{CO}_2$ are located in files *Abstr CSNF No Drip pH.xls*, *Abstr 2DHLW No Drip pH.xls*, and *Abstr 2MCO No Drip pH.xls* in Output DTN: SN0702PAIPC1CA.001.

6.10.2[a] Ionic Strength Abstractions

6.10.2.1[a] Liquid Influx

The parameters with the largest effects on ionic strength are liquid influx rate and time (Section 6.6.4[a]). To generate the ionic strength abstractions, a series of time-dependent simulations were performed at different liquid influx rates. The minimum liquid influx rates were set at the approximate minimum values that would not generate ionic strength above 1 molal, the upper range of the applicability of the B-dot equation. All IPC ionic strength calculations are true ionic strengths from model output (i.e., they are calculated from the complete set of aqueous species at equilibrium). The simulations are listed in Table 6-18[a].

For these simulations, SD9 water was used as the liquid influx composition. SD9 represents the most frequent type of pore water sampled at Yucca Mountain (SNL 2007 [DIRS 177412], Section 6.1). In addition, its ionic strength is approximately in the middle of the range of ionic strengths of the various water compositions simulated, as shown at early times in the base case simulations when the water has been minimally affected by cell processes (Figures 6-6[a], 6-9[a], and 6-12[a]). Further supporting the use of SD9 as the reference liquid influx composition is the broad uncertainty distribution defined for ionic strength in Section 6.10.8.2[a].

Figures 6-44[a] through 6-46[a] show results of the simulations in Table 6-18[a]. As described in Section 6.6.4[a], low liquid influx rates generate high ionic strengths, and high liquid influx rates reduce the departure of the ionic strength from the ionic strength of the liquid influx composition. These figures also show that ionic strength is a strong function of time.

As occurred in simulations presented in Sections 6.5[a] and 6.6[a], convergence problems associated with vanishingly small concentrations of trace elements in the cell caused early termination of model simulations. In Figures 6-44[a] through 6-46[a], the ionic strength at the termination of the simulations is extended to one million years. Because the ionic strength over the remainder of the one million years can be approximated by the ionic strength at termination, the early terminations do not prevent the generation of sufficiently accurate abstractions. There

is a large degree of uncertainty in ionic strength calculations, and this is reflected in the ionic strength distributions established in Section 6.10.8.2[a].

The time histories of ionic strength in Figures 6-44[a] through 6-46[a] represent the histories predicted for the given simulation conditions. Deviations from the degradation rates used in these simulations can alter the timing, width, and amplitude of the ionic strength peaks. The potential magnitude of these effects on the waste form cells is indicated by the degradation rate sensitivity analyses in Section 6.6.5[a]. Figures 6-33[a], 6-35[a], and 6-37[a] are used to approximate the uncertainty in ionic strength over time resulting from degradation rate uncertainty. This uncertainty is included in the quantification of ionic strength uncertainty in Section 6.10.8.2[a]. (The timing of the peaks in the 2MCO cell in Figure 6-46[a] is highly affected by liquid influx rate because at liquid influx rates of 100 L/yr and lower, the N-Reactor fuel degrades only as rapidly as the liquid influx rate allows.)

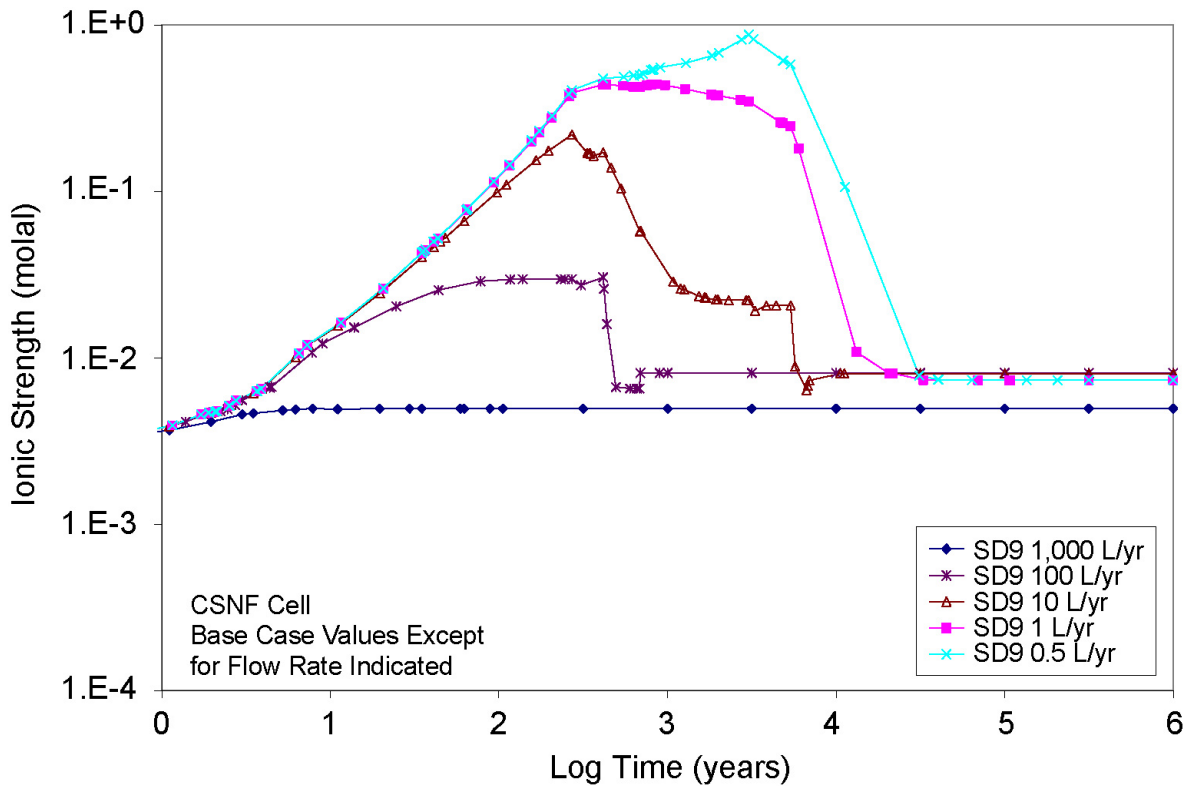
For the abstractions, a time-averaged ionic strength is determined for discrete time intervals from the various time histories. Sufficient representation of the functionality of log time since breach is obtained from half log time intervals starting at 0.01 years after breach (10^{-2} to $10^{-1.5}$, $10^{-1.5}$ to 10^{-1} , ... , $10^{5.5}$ to 10^6). The liquid influx ionic strength abstractions are documented in files *Abstr CSNF Seepage Ion Str 2.xls*, *Abstr 2DHLW Seepage Ion Str 2.xls*, and *Abstr 2MCO Seepage Ion Str 2.xls* in Output DTN: SN0702PAIPC1CA.001.

The abstractions are plotted in Figures 6-47[a] through 6-49[a]. For the CSNF and 2MCO abstractions, ionic strength was extrapolated to a liquid influx rate of 0.1 L/yr by a linear forecast based on the ionic strengths of the two lowest liquid influx rates simulated. Extrapolations were necessary because EQ6 simulations in this addendum are restricted to ionic strengths of 1 molal or below due to the range of applicability of the B-dot equation (Section 6.3.1.1[a]). This method of extrapolation introduces additional uncertainty that is reflected in the uncertainty of the extrapolations (Section 6.10.8.2[a]). In addition, uncertainty on the high end is moderated by the vapor influx abstraction, which overrides the liquid influx abstraction when the ionic strength predicted by the vapor influx abstraction is lower than that of the liquid influx abstraction (Section 6.10.9.1[a]).

Table 6-18[a]. EQ6 Input/Output Files Used in Liquid-Influx Ionic Strength Abstractions

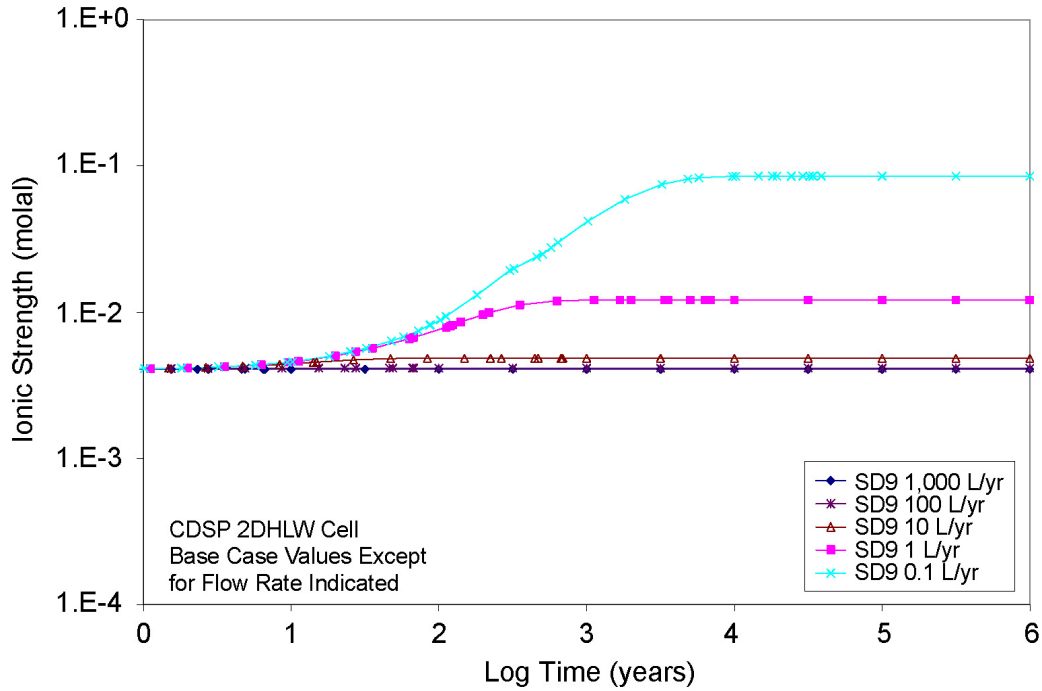
| Cell | Liquid Influx Rate (L/yr) | Path | Input Files | Output Files |
|-------|---------------------------|------------------|-------------|--------------|
| CSNF | 1,000 | 1milyr\CSNFflo2\ | adprpSD9.6i | adprpSD9.6o |
| | 100 | 1milyr\CSNFflo2\ | bdprpSD9.6i | bdprpSD9.6o |
| | 10 | 1milyr\CSNFflo2\ | cdprpSD9.6i | cdprpSD9.6o |
| | 1 | 1milyr\CSNFflo2\ | ddprpSD9.6i | ddprpSD9.6o |
| | 0.5 | 1milyr\CSNFflo2\ | edprpSD9.6i | edprpSD9.6o |
| 2DHLW | 1,000 | 1milyr\2DHLWfl2\ | adprpSD9.6i | adprpSD9.6o |
| | 100 | 1milyr\2DHLWfl2\ | bdprpSD9.6i | bdprpSD9.6o |
| | 10 | 1milyr\2DHLWfl2\ | cdprpSD9.6i | cdprpSD9.6o |
| | 1 | 1milyr\2DHLWfl2\ | ddprpSD9.6i | ddprpSD9.6o |
| | 0.1 | 1milyr\2DHLWfl2\ | edprpSD9.6i | edprpSD9.6o |
| 2MCO | 1,000 | 1milyr\2MCOflo2\ | adprpSD9.6i | adprpSD9.6o |
| | 100 | 1milyr\2MCOflo2\ | bdprpSD9.6i | bdprpSD9.6o |
| | 10 | 1milyr\2MCOflo2\ | cdprpSD9.6i | cdprpSD9.6o |
| | 1 | 1milyr\2MCOflo2\ | ddprpSD9.6i | ddprpSD9.6o |
| | 0.3 | 1milyr\2MCOflo2\ | edprpSD9.6i | edprpSD9.6o |

Source: Output DTN: SN0702PAIPC1CA.001.



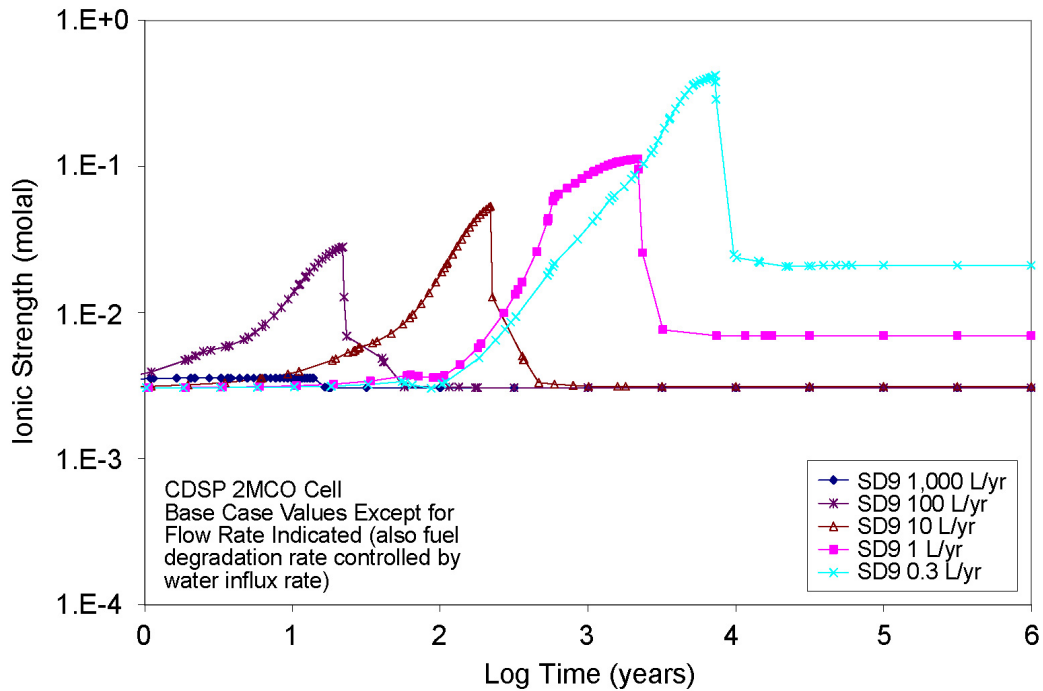
Source: Output DTN: SN0702PAIPC1CA.001, file: CSNF flow SD9.xls.

Figure 6-44[a]. CSNF Liquid-Influx Ionic Strength for Various Flow Rates



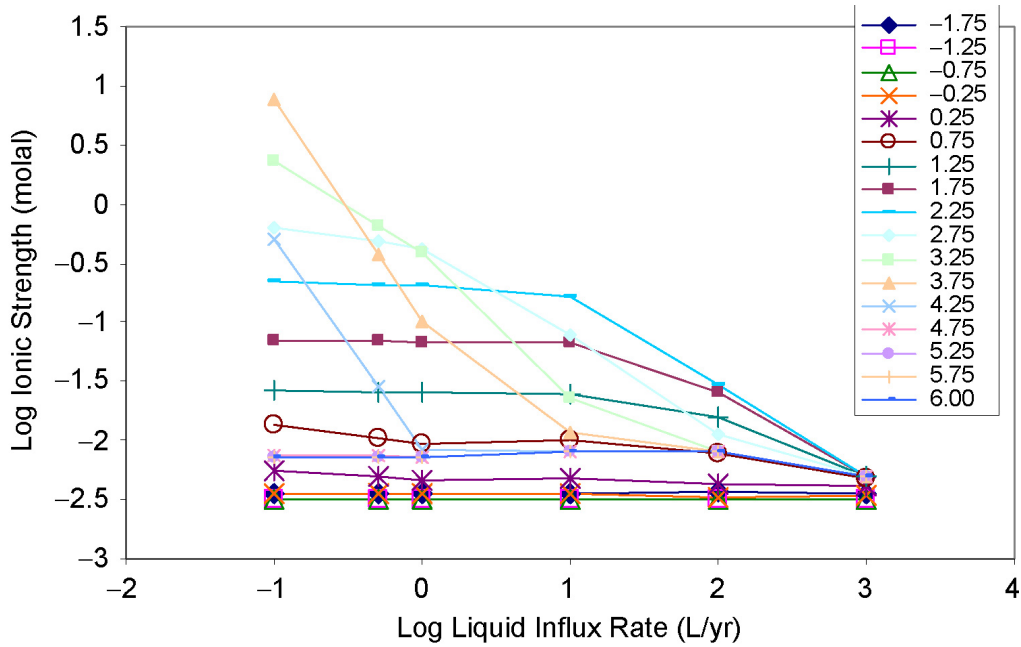
Source: Output DTN: SN0702PAIPC1CA.001, file: 2DHLW flow SD9.xls.

Figure 6-45[a]. 2DHLW Liquid-Influx Ionic Strength for Various Flow Rates



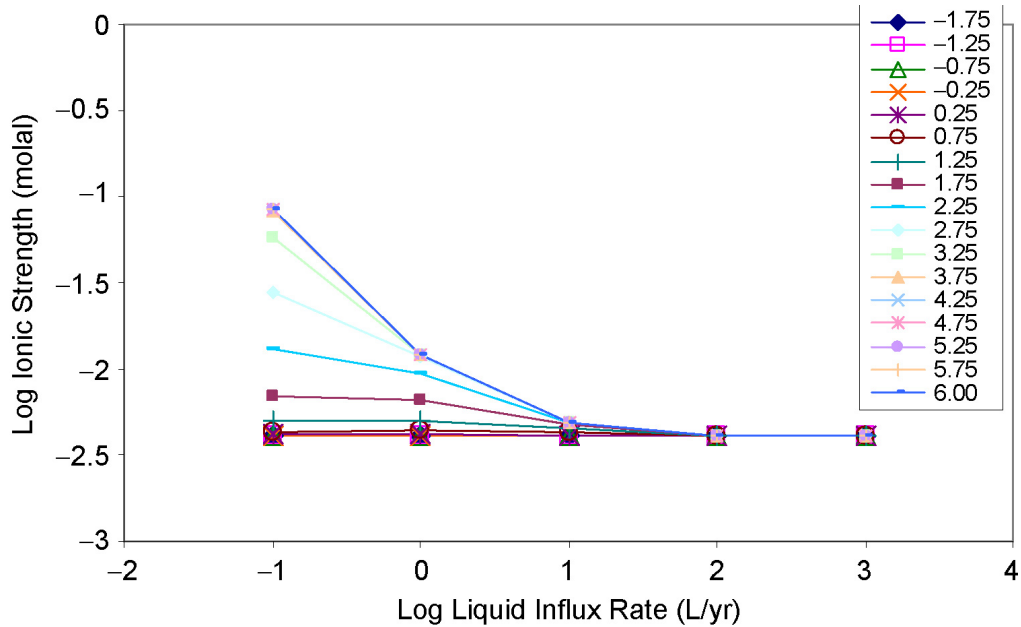
Source: Output DTN: SN0702PAIPC1CA.001, file: 2MCO flow SD9.xls.

Figure 6-46[a]. 2MCO Liquid-Influx Ionic Strength for Various Flow Rates



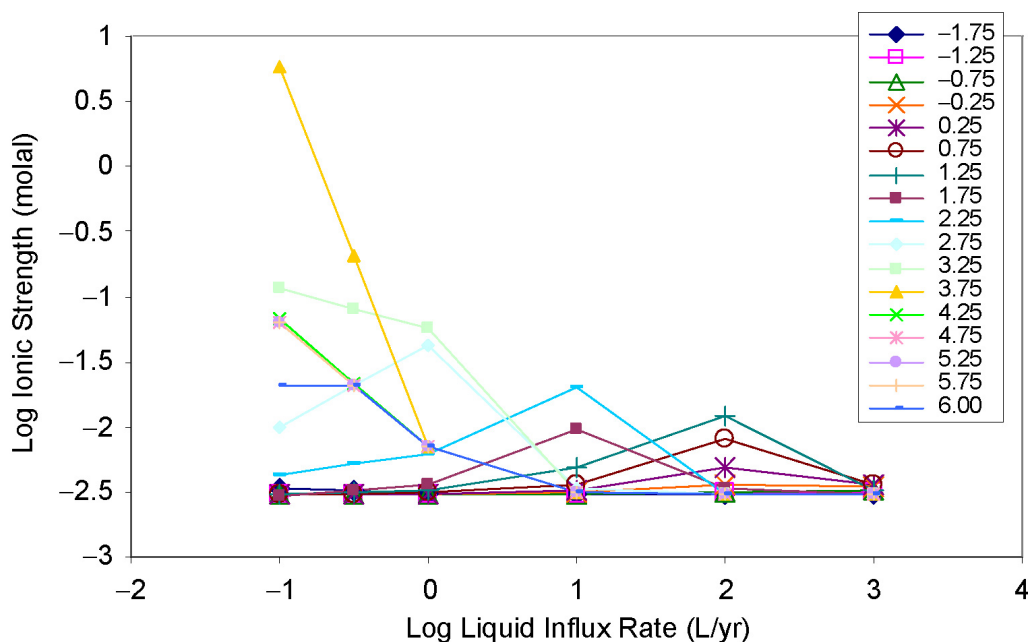
Source: Output DTN: SN0702PAIPC1CA.001, file: *Abstr CSNF Seepage Ion Str 2.xls*.

Figure 6-47[a]. CSNF Ionic Strength versus Liquid Influx Rate at Various Values of Log Time (years)



Source: Output DTN: SN0702PAIPC1CA.001, file: *Abstr 2DHLW Seepage Ion Str 2.xls*.

Figure 6-48[a]. 2DHLW Ionic Strength versus Liquid Influx Rate at Various Values of Log Time (years)



Source: Output DTN: SN0702PAIPC1CA.001, file: *Abstr 2MCO Seepage Ion Str 2.xls*.

Figure 6-49[a]. 2MCO Ionic Strength versus Liquid Influx Rate at Various Values of Log Time (years)

6.10.2.2[a] Vapor Influx

Pitzer calculations for simple salt solutions from *In-Drift Precipitates/Salts Model* (SNL 2007 [DIRS 177411]) were used to approximate the relationship between equilibrium relative humidity and ionic strength beyond the ranges simulated and plotted in Figure 6-13[a]. The in-drift precipitates/salts model is a Pitzer ion-interaction model that is qualified for use at high ionic strength. Figure 6-50[a] shows this comparison to CSNF vapor influx calculations. The Pitzer calculations and IPC model calculations are true ionic strength calculations. These comparisons are documented in the abstraction files *Abstr CSNF No Drip Ion Str.xls*, *Abstr 2DHLW No Drip Ion Str.xls*, and *Abstr 2MCO No Drip Ion Str.xls* in Output DTN: SN0702PAIPC1CA.001.

Figure 6-50[a] indicates that the trend between ionic strength and RH for the CSNF vapor influx calculations is similar to trends for salt solutions involving a divalent cation and a monovalent anion (e.g., CaBr_2 , MgCl_2). This is true for the 2DHLW and 2MCO cells as well. The reason for this correspondence is that the aqueous solutions of the three cells are also largely dominated by divalent cations (e.g., Ni^{2+} , Mg^{2+} and Zn^{2+}) and monovalent anions (e.g., NO_3^-).

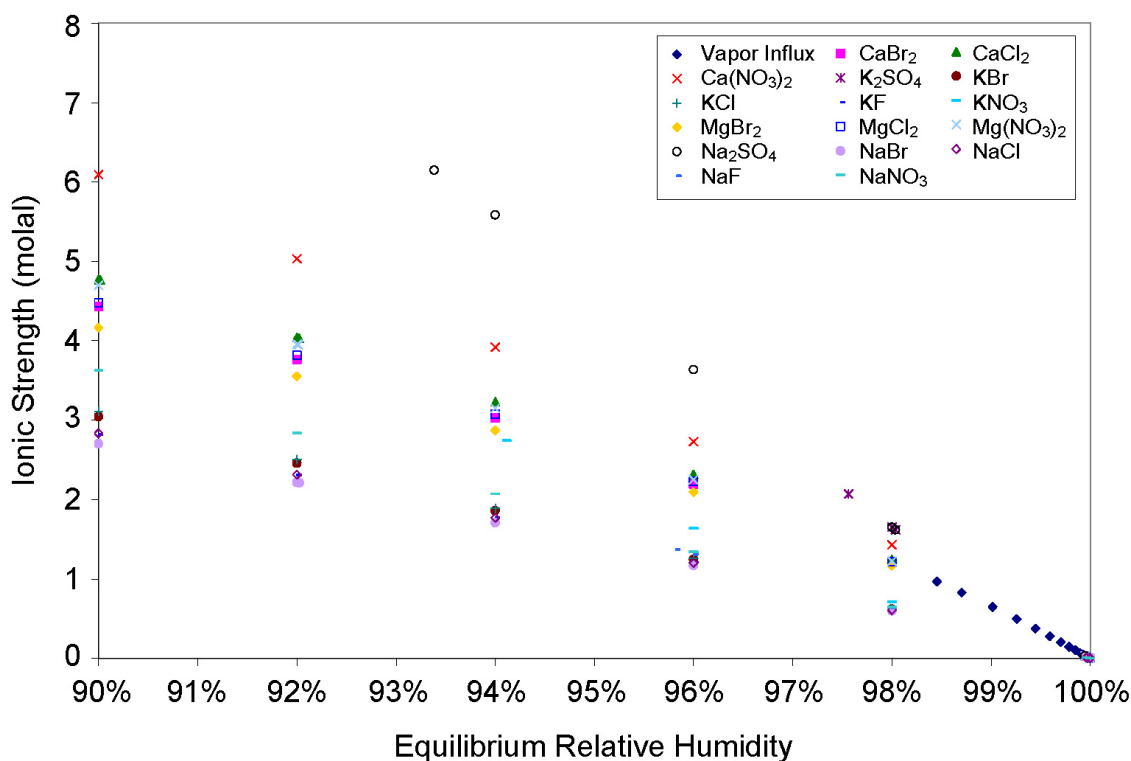
Abstraction of ionic strength to RH below the range of the vapor influx simulations is based on the trends observed for CaBr_2 (for CSNF) and MgBr_2 (for 2DHLW and 2MCO). The abstractions are summarized in Table 6-19[a].

Table 6-19[a]. Intercept and Slope of the Abstraction Equations for the Mode of Vapor Influx Ionic Strength

| Cell | RH Range | Intercept ^a (A) | Slope ^a (B) | File |
|-------|--------------------|-------------------------------|---------------------------|--|
| CSNF | 95% ≤ RH < 98.3% | 45.846 | -45.525 | <i>Abstr CSNF No Drip Ion Str.xls</i> |
| | 98.3% ≤ RH < 99.9% | 64.0629 | -64.0602 | |
| | 99.9% ≤ RH ≤ 100% | 66.4312 | -66.4308 | |
| 2DHLW | 95% ≤ RH < 97.2% | 42.992 | -42.654 | <i>Abstr 2DHLW No Drip Ion Str.xls</i> |
| | 97.2% ≤ RH ≤ 100% | 54.7803 | -54.7776 | |
| 2MCO | 95% ≤ RH < 98.33% | 42.992 | -42.654 | <i>Abstr 2MCO No Drip Ion Str.xls</i> |
| | 98.33% ≤ RH ≤ 100% | 62.4136 | -62.4112 | |

Source: Output DTN: SN0702PAIPC1CA.001.

^a Mode ionic strength (molal) = A + B(RH).



Source: Output DTN: SN0702PAIPC1CA.001, file: *Abstr CSNF No Drip Ion Str.xls*; DTN: MO0701EQ36IDPS.000 [DIRS 179290], files: *.6o in *Salts25.zip*.

Figure 6-50[a]. Ionic Strength of CSNF Vapor Influx Simulation and Simple Salt Solutions versus Equilibrium Relative Humidity

6.10.3[a] Fluoride Abstractions

High fluoride concentrations increase actinide solubility. Because the effects of fluoride are incorporated into the uncertainty calculations for solubility limits in *Dissolved Concentration Limits of Elements with Radioactive Isotopes* (SNL 2007 [DIRS 177418]), fluoride abstractions from the IPC model were developed. This was accomplished by examining the liquid influx

sensitivity analyses in Section 6.6.4[a]. The largest variation in fluoride concentrations are provided by the degradation rate sensitivity analyses (Section 6.6.5[a]). Vapor influx simulations were not examined because there are no sources of fluoride in any of the IPC cells (except possibly HLW glass, which is addressed below).

In the sensitivity simulations, the fluoride ion concentration most often varies between trace quantities (i.e., less than 10^{-6} molal) and the concentration in the liquid influx. Often fluoride precipitates in the form of fluorite (CaF_2) or sellaite (MgF_2). However, in CSNF and especially in N-Reactor (2MCO) cells, fluoride can concentrate as ionic strength increases in the absence of concentrations of calcium and magnesium that are high enough for saturation with respect to fluorite and sellaite (e.g., files *CSNF dr.xls* and *2MCO dr.xls* in Output DTN: SN0702PAIPC1CA.001).

Figures 6-51[a] through 6-53[a] show the relationship of fluoride and ionic strength in the three waste form cells for various incoming waters and degradation rates. At low ionic strength, the fluoride concentration either remains in the vicinity of the concentration in the liquid influx or falls below the liquid influx concentration due to mineral precipitation. At high ionic strength for the CSNF cell and especially for the 2MCO cell, fluoride often concentrates as water is consumed by degradation reactions. However, at high ionic strength in the 2DHLW cell, fluoride precipitation prevents the fluoride concentration from ever increasing much above the incoming concentration.

Fluorine was included in the estimated composition of the HLW glass used in the parent document. That composition was hypothetical and based on the analyses of the radionuclide sludges that had fluoride from hydrofluoric acid used to dissolve the wastes. However, including fluorine in 2DHLW IPC simulations does not result in higher aqueous fluoride concentrations. Fluorine was added to the HLW glass of this addendum at the same concentration as in the parent IPC document in the 2DHLW IPC simulations documented in *Total System Performance Assessment Model / Analysis for the License Application* (SNL 2007 [DIRS 178871]). In each simulation, despite the fluorine component added to the HLW glass, fluoride precipitates and never substantially exceeds the incoming fluoride concentration in the liquid influx simulations and is always below 10^{-5} molal in the vapor influx simulations (DTN: SN0704PMAIPCPM.001 [DIRS 180745], files: \Liquid Influx\2DHLW**.6o and \Vapor Influx\2DHLW**.6o).

To satisfy the needs of the solubility model (SNL 2007 [DIRS 177418]), single fluoride distributions were defined for discrete ionic strength intervals. These distributions were defined to be right-triangular with a zero value for the minimum and apex and a conservatively high value for the maximum. This distribution is justified based on conservative arguments as long as the maximum fluoride values are set conservatively high, which they are.

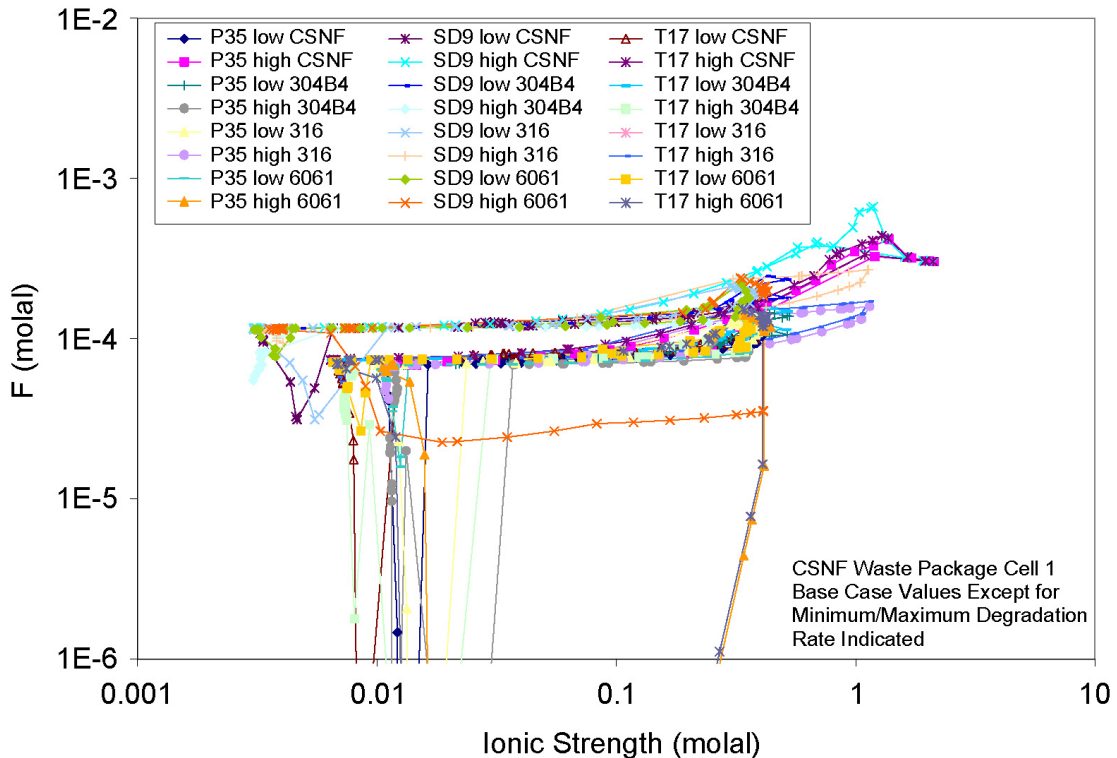
At low ionic strength the maximum fluoride concentration is set at the maximum concentration observed in qualified pore waters from the Topopah Spring welded tuff (TSw), 4.8 mg/L (0.00025 molal) (file *TSw_Porewater_Data.xls* in DTN: SN0705PAEBSPCE.015 [DIRS 182269]). This value exceeds the fluoride concentrations of each of the various waters used in the base case and sensitivity analyses in Sections 6.5[a] and 6.6[a] and is used as the maximum possible fluoride concentration that can occur in the waste package without considerable water loss due to degradation reactions or evaporation.

At high ionic strength in the CSNF and 2MCO cells, when degradation reactions are rapid enough to consume a substantial amount of water, maximum fluoride concentrations are increased as shown in Table 6-20[a]. This approach is justified by the strong correlation between ionic strength and maximum fluoride concentrations in Figures 6-51[a] and 6-53[a].

Table 6-20[a]. Maximum Fluoride Concentrations for Various Ranges of Ionic Strength

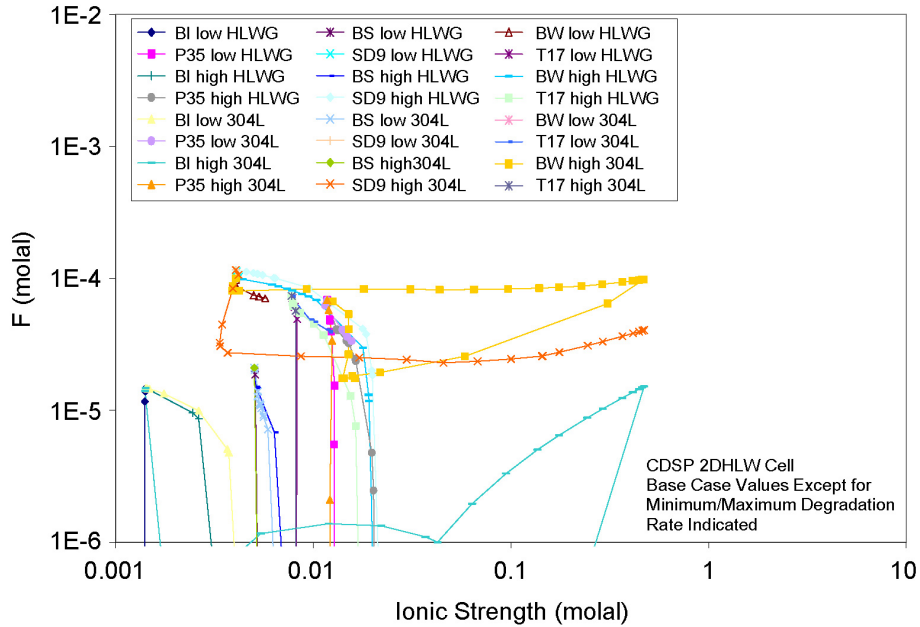
| Cell | Ionic Strength Range (molal) | Maximum Fluoride Concentration (molal) | File |
|-------|------------------------------|--|----------------------------------|
| CSNF | $I < 0.2$ | 0.00025 | <i>Abstr CSNF Seepage F.xls</i> |
| | $I \geq 0.2$ | 0.0025 | |
| 2DHLW | $I > 0$ | 0.00025 | <i>Abstr 2DHLW Seepage F.xls</i> |
| 2MCO | $I < 0.004$ | 0.00025 | <i>Abstr 2MCO Seepage F.xls</i> |
| | $I \geq 0.004$ | 0.01 | |

Source: Output DTN: SN0702PAIPC1CA.001.



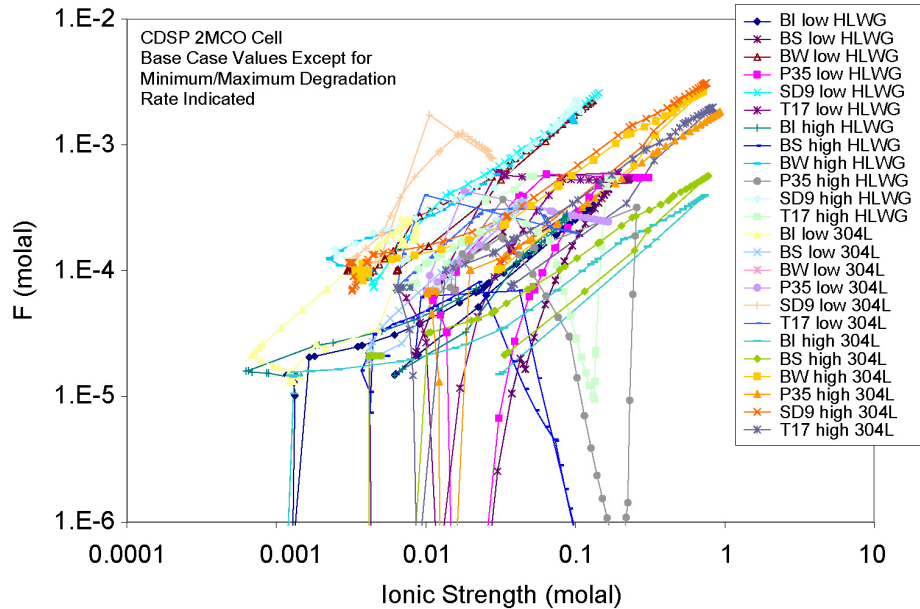
Source: Output DTN: SN0702PAIPC1CA.001, file: *CSNF dr.xls*.

Figure 6-51[a]. Plot of F Concentration versus Ionic Strength for IPC CSNF Liquid Influx Calculations for Six Different Pore Waters (BI, BS, BW, P35, SD9, and T17) Using Base-Case Degradation Rates Except for the Minimum or Maximum Degradation Rate Indicated



Source: Output DTN: SN0702PAIPC1CA.001, file: 2DHLW dr.xls.

Figure 6-52[a]. Plot of F Concentration versus Ionic Strength for IPC CDSP 2DHLW Liquid Influx Calculations for Six Different Pore Waters (BI, BS, BW, P35, SD9, and T17) Using Base-Case Degradation Rates Except for the Minimum or Maximum Degradation Rate Indicated



Source: Output DTN: SN0702PAIPC1CA.001, file: 2MCO dr.xls.

Figure 6-53[a]. Plot of F Concentration versus Ionic Strength for IPC CDSP 2MCO Liquid Influx Calculations for Six Different Pore Waters (BI, BS, BW, P35, SD9, and T17) Using Base-Case Degradation Rates Except for the Minimum or Maximum Degradation Rate Indicated

6.10.4[a] Chloride Abstractions

Not evaluated in addendum.

6.10.5[a] Total Carbonate Abstraction

No change.

6.10.6[a] Eh Abstraction

This section is not changed because the IPC Eh abstraction is not used in the TSPA-LA model. However, should this abstraction be used in the future, it should be noted that there is an assumption in the derivation of the Eh expression that is not explained in the parent report. The assumption is that the 25°C value of the Gibbs free energy of water can be used for all temperatures. This assumption causes small errors in Eh abstraction calculations at temperatures substantially different from 25°C.

6.10.7[a] Linking the Vapor-Influx and Liquid-Influx Models for TSPA-LA

This topic is discussed in Section 6.10.9.1[a].

6.10.8[a] Output Uncertainty and Correlations

The output uncertainty is determined based on uncertainties in the inputs, uncertainties in the model, and ranges observed in sensitivity simulations. Uncertainties in pH, ionic strength, and fluoride ion concentration are discussed in the following subsections. Correlations are also discussed in these sections. Uncertainties and correlations in the total dissolved carbonate concentration and Eh abstractions are unchanged from the parent document. All uncertainties defined in the IPC model are epistemic.

6.10.8.1[a] pH

The titration calculations in Section 6.10.1[a] are used to determine the minimum and maximum pH values in the vicinity of the waste form. These calculations identify and quantify the major pH-buffering reactions that cannot be overcome by possible sources of acid or base. The secondary minerals that accumulate in each IPC cell are the dominant pH buffers. The hypothetical additions of acid and base in these titrations are extreme and exceed anticipated sources (Section 6.10.1.1[a]) so that minimum and maximum pH limits are determined with a high degree of confidence.

Time-dependent base case simulations and sensitivity analyses confirm that the established minimum and maximum pH limits are reasonable (Sections 6.5[a] and 6.6[a]). Sensitivity simulations include varying the composition of the liquid influx (Section 6.6.2[a] and other subsections in Section 6.6[a]), varying the fugacity of carbon dioxide (Section 6.6.3[a]), varying liquid flux rates (Section 6.6.4[a]), and varying degradation rates of the IPC cell materials and waste forms (Section 6.6.5[a]). The minimum and maximum pH limits defined in Section 6.10.1[a] are not exceeded in these simulations.

The values of pH calculated in the time-dependent simulations are widely distributed between the minimum and maximum pH values established in the titrations. For example, the lowest and highest pH values among all CSNF degradation rate sensitivity simulations are 5.01 and 8.0 (Figure 6-32[a]; Output DTN: SN0702PAIPC1CA.001, file: *CSNF dr.xls*). In comparison, the abstracted minimum and maximum pH values at the corresponding ionic strengths (1.18 and 0.01 molal) and pCO₂ (3.0) can be interpolated from the abstractions. They are approximately 4.97 and 8.32 (Figure 6-41[a]; Output DTN: SN0702PAIPC1CA.001, file: *Abstr CSNF Seepage pH.xls*). Because of the large buffer capacities quantified at the abstracted minimum and maximum pH values, the uncertainties in the pH limits are negligible and do not require quantification.

The pH distribution for TSPA-LA is defined to be uniform between the minimum and maximum pH values. Examples of the minimum and maximum pH values as a function of pCO₂ and ionic strength are displayed in Figures 6-41[a] through 6-43[a]. A uniform distribution is chosen because sensitivity analyses indicate no clear central tendency for pH.

Correlation coefficients involving the pH range (i.e., minimum and maximum pH) and other parameters calculated for a cell are not prescribed as additional constraints on uncertainty because correlations are already built into the calculations. Correlation to ionic strength is directly included in the calculation of the pH range (Section 6.10.1[a]). Similarly, correlation to total carbonate is included in the calculations because total carbonate is a function of pH (Section 6.10.5[a]). In the case of the two CDSP cells (1a and 1b) which share a single waste package, correlations are imposed by using RH values (for vapor influx) or liquid influx rates (for liquid influx) that are reasonable for each of the cells in the same waste package at the same time. Imposing RH values or liquid influx rates that are reasonable and consistent for a given CDSP waste package ensures that the ionic strengths and pH ranges for Cells 1a and 1b are reasonable and consistent.

6.10.8.2[a] Ionic Strength

Ionic strength is determined differently for the vapor influx and liquid influx cases. In the vapor influx case, ionic strength is estimated from RH. In the liquid influx case, ionic strength is estimated from liquid influx rate and time since waste package breach. All IPC ionic strength calculations are true ionic strengths from model output.

Vapor Influx – The basis of the vapor influx model is that aqueous solutions that persist in the waste package must be in equilibrium with the local water vapor pressure. At equilibrium, RH (expressed as a fraction) is equivalent to the activity of water. Figure 6-50[a] shows that for various salt solutions and for IPC model estimates there is a fundamental relationship between ionic strength and RH. Figure 6-50[a] shows that solutions with divalent ions (e.g., SO₄²⁻, Mg²⁺, Ca²⁺) generally plot higher than solutions with only monovalent ions.

For uncertainty in the value of ionic strength in the vapor influx case, a triangular distribution is defined. The most likely (mode) values are given in Table 6-19[a]. The maximum and minimum values of ionic strength are set at ±50% of the mode based on the ranges of the ionic strengths observed in Figure 6-50[a]. The only salt solution excluded from the uncertainty range is Na₂SO₄. It is excluded because it is substantially different from the other solutions in the

figure and from the base case vapor influx calculations for the three IPC cells (Figures 6-13[a] and 6-50[a]).

The ionic strength of evaporated seawater would plot below the RH-ionic strength trajectories in Table 6-19[a], but it would remain within the $\pm 50\%$ uncertainty distribution. Pitzer simulations indicate that evaporated seawater would have ionic strengths of 0.76 and 1.5 molal at RH values of 98% and 96%, respectively (file *inagua r2.xls* in DTN: MO0701EQ36IDPS.000 [DIRS 179290]). These values are above the minimum ionic strengths in the vapor influx abstractions, which are around 0.6 and 1.1 molal, respectively, at 98% and 96% RH (worksheet "Chart Uncert" in files *Abstr CSNF No Drip Ion Str.xls*, *Abstr 2DHLW No Drip Ion Str.xls*, and *Abstr 2MCO No Drip Ion Str.xls* in Output DTN: SN0702PAIPC1CA.001). The seawater trajectory is below the trajectories calculated for the waste form cells because seawater has lower relative concentrations of divalent ions.

Liquid Influx – In the liquid influx case, the value of $\log I$ is related to the log values of the liquid influx rate ($\log q$) and the time since waste package breach ($\log t'$) (Section 6.10.2.1[a]). With $\log q$ and $\log t'$ factored into the log ionic strength, degradation rates and incoming water compositions have the next largest effects on $\log I$. Thus, uncertainty in $\log I$ under liquid influx conditions is assessed based on the results of degradation rate sensitivity simulations, which include simulations for various incoming water compositions.

Figures 6-33[a], 6-35[a], and 6-37[a] show the sensitivity of ionic strength in the three waste form cells to variations in degradation rates and liquid influx water compositions. Based on these simulations and results of the base case liquid influx simulations (Figures 6-6[a], 6-9[a], and 6-12[a]), extreme values of degradation rates tend to cause ionic strength to depart from base case predictions in both positive and negative directions, justifying an uncertainty distribution with a central tendency. This tendency is clear for the 2MCO cell but less so for the CSNF and 2DHLW cells where deviations in the negative direction are less apparent. However, because (1) there is a high degree of uncertainty in the distributions of representative degradation rates, (2) a combination of minimum or low rates in these cells (plausible especially over long periods of time) would result in lower ionic strengths, and (3) condensate water (with low ionic strength) is not included in the sensitivity analysis, it is reasonable to approximate ionic strength uncertainty using a simple symmetric log triangular distribution. The figures indicate that maximum $\log I$ deviations from base case values are approximately 1. Thus, uncertainty in the liquid influx ionic strength abstractions is represented by a log triangular distribution in which the mode (or apex) $\log I$ value is determined by the lookup tables used to generate Figures 6-47[a] through 6-49[a] and the minimum and maximum values of the distributions are ± 1 log unit, except as explained in the next paragraph when the maximum deviation is larger under low influx conditions.

For low liquid influx conditions that would cause temporary excursions above an ionic strength of 1 molal in EQ6 calculations, modes are approximated by extrapolation (Section 6.10.2.1[a]) but the distribution type (log triangular) is unchanged. Under these conditions, maximum deviations are increased in proportion to the change in ionic strength. Specifically, the deviation at 0.1 L/yr is increased by half of the difference in the mode of $\log I$ at liquid influx rates of 0.1 and 1 L/yr. For example, if mode $\log I$ increases from -0.2 to $+0.4$, then the maximum deviation at 0.1 L/yr increases to 1.3 ($1 + (0.4 - (-0.2))/2$). This approach is

justified because extrapolations increase uncertainty. Relating the increase in uncertainty to the change in the mode is justified because extrapolations that predict no change in the mode indicate that the mode is likely not a function of liquid influx rate for the conditions of the extrapolations.

Correlation coefficients involving ionic strength and other parameters calculated for a cell are not prescribed as additional constraints on uncertainty because such correlations are weak or are already built into the calculations. The effects of time, flow rate, degradation rates, and material inventories are already built into the ionic strength abstractions because the abstractions are derived from process model simulations that include these parameters. Correlation to the pH range is captured because the pH range is a function of ionic strength (Sections 6.10.1[a]). Correlation to total carbonate is included indirectly because total carbonate is a function of pH, whose distribution in turn is a function of ionic strength. In the case of the two CDSP cells (1a and 1b), which share a single waste package, correlations are imposed by using RH values (for vapor influx) or liquid influx rates (for liquid influx) that are reasonable for each of the cells in the same waste package at the same time.

6.10.8.3[a] Fluoride Ion Concentration

The fluoride abstraction is conservative (Section 6.10.3[a]). Therefore, no additional uncertainty terms are defined for fluoride.

6.10.9[a] Implementation of IPC Abstractions

Table 6-21[a] provides the names, descriptions, and forms for parameters provided to TSPA-LA by the IPC abstraction. The TSPA-LA parameter names and definitions are listed in columns 1 and 2. Parameter types, listed in column 3, are constants, variables defined in lookup tables, or variables defined by equations. Unless documented in lookup tables, values of the TSPA-LA parameters are provided in column 4. Column 5 cites sources and lookup tables. Table 6-22[a] provides the same type information for the fluoride abstraction used by *Dissolved Concentration Limits of Elements with Radioactive Isotopes* (SNL 2007 [DIRS 177418]).

The IPC abstractions represent chemical conditions within each of three waste form cells defined in Section 1[a]. For each cell, two IPC abstractions are provided, one for vapor influx and one for liquid influx. The liquid influx rate is the rate at which seepage and/or condensed water enters the breached waste package. For an intact drip shield, the only source of liquid influx into the waste package is drip shield condensation. At all times, the partial pressure of O₂ (g) is set at 0.2 bar, which is the approximate atmospheric partial pressure. This high partial pressure is applied because it approximates the partial pressure that will prevail in the drift and is conservative in the prediction of radionuclide release and mobility.

The IPC abstractions may be used for all the scenarios, including the igneous intrusion scenario. Sensitivity analyses using the various water compositions listed in Section 4.1.2[a] indicate that the composition of water entering the waste package has little effect on IPC abstractions except for ionic strength abstractions when the liquid influx rate is high (Section 6.6.4[a]). At high liquid influx rates, the abstracted IPC ionic strengths approach the values in the incoming liquid. Uncertainty in IPC ionic strength abstractions (Section 6.10.8.2[a]), however, is comparable to

the range of ionic strengths of the various water compositions in the sensitivity analyses, and therefore this uncertainty allows the IPC ionic strength abstractions to apply even at high liquid influx rates.

The choice between the liquid and vapor influx abstractions is addressed in Section 6.10.9.1[a]. Implementation of the abstractions is explained in Sections 6.10.9.2[a] (liquid influx) and 6.10.9.3[a] (vapor influx). Section 6.10.9.4[a] discusses the implementation of the fluoride abstraction.

Table 6-21[a]. TSPA-LA Parameters in IPC Abstractions

| Parameter Name | Description | Type | Independent Variable(s) or Value | TSPA-LA Roadmap: Source File in Output DTN: SN0702PAIPC1CA.001 |
|--|---|-----------|---|---|
| InPkg_Istr_Mode_CS NF_S_log InPkg_Istr_Dev_CS NF_S_log | Mode and maximum deviation of the log value of ionic strength (molal) under liquid influx conditions in Cell 1 of CSNF packages | Table | Log liquid influx rate, Log time since waste package breach | <i>Abstr CSNF Seepage Ion Str 2.xls</i> |
| InPkg_Istr_Mode_2DHLW_S_log InPkg_Istr_Dev_2DHLW_S_log | Mode and maximum deviation of the log value of ionic strength (molal) under liquid influx conditions in Cell 1a of CDSP packages | Table | Log liquid influx rate, Log time since waste package breach | <i>Abstr 2DHLW Seepage Ion Str 2.xls</i> |
| InPkg_Istr_Mode_2MCO_S_log InPkg_Istr_Dev_2MCO_S_log | Mode and maximum deviation of the log value of ionic strength (molal) under liquid influx conditions in Cell 1b of CDSP packages | Table | Log liquid influx rate, Log time since waste package breach | <i>Abstr 2MCO Seepage Ion Str 2.xls</i> |
| IStr_CS NF_NS_A1 IStr_CS NF_NS_B1 IStr_CS NF_NS_RH_1 | Intercept (A1) and slope (B1) of linear equation used to calculate the mode of the ionic strength (molal) under vapor influx (no-drip) conditions in Cell 1 of CSNF packages as a function of in-package RH when in-package RH is below IStr_CS NF_NS_RH_1 | Constants | 45.846 -45.525 98.3% (independent variable is in-package RH) | <i>Abstr CSNF Nodrip Ion Str.xls</i> |
| IStr_CS NF_NS_A2 IStr_CS NF_NS_B2 IStr_CS NF_NS_RH_2 | Intercept (A2) and slope (B2) of linear equation used to calculate the mode of the ionic strength (molal) under vapor influx (no-drip) conditions in Cell 1 of CSNF packages as a function of in-package RH when in-package RH is between IStr_CS NF_NS_RH_1 and IStr_CS NF_NS_RH_2 | Constants | 64.0629 -64.0602 99.9% (independent variable is in-package RH) | <i>Abstr CSNF Nodrip Ion Str.xls</i> |
| IStr_CS NF_NS_A3 IStr_CS NF_NS_B3 | Intercept (A3) and slope (B3) of linear equation used to calculate the mode of the ionic strength (molal) under vapor influx (no-drip) conditions in Cell 1 of CSNF packages as a function of in-package RH when in-package RH is above IStr_CS NF_NS_RH_2 | Constants | 66.4312 -66.4308 (independent variable is in-package RH) | <i>Abstr CSNF Nodrip Ion Str.xls</i> |

Table 6-21[a]. TSPA-LA Parameters in IPC Abstractions (Continued)

| Parameter Name | Description | Type | Independent Variable(s) or Value | TSPA-LA Roadmap: Source File in Output DTN: SN0702PAIPC1CA.001 |
|--|--|-----------|--|---|
| InPkg_Istr_CSNF_NS_Max_Factor InPkg_Istr_CSNF_NS_Min_Factor | Maximum and minimum ionic strength factors under vapor influx conditions in Cell 1 of CSNF packages | Constants | 1.5 0.5 | Abstr CSNF Nodrip Ion Str.xls |
| Istr_2DHLW_NS_A1 Istr_2DHLW_NS_B1 Istr_2DHLW_NS_RH_1 | Intercept (A1) and slope (B1) of linear equation used to calculate the mode of the log value of ionic strength (molal) under vapor influx (no-drip) conditions in Cell 1a of CDSP packages as a function of in-package RH when in-package RH is below Istr_2DHLW_NS_RH_1 | Constants | 42.992 -42.654 97.2% (independent variable is in-package RH) | Abstr 2DHLW Nodrip Ion Str.xls |
| Istr_2DHLW_NS_A2 Istr_2DHLW_NS_B2 | Intercept (A2) and slope (B2) of linear equation used to calculate the mode of the log value of ionic strength (molal) under vapor influx (no-drip) conditions in Cell 1a of CDSP packages as a function of in-package RH when in-package RH is above Istr_2DHLW_NS_RH_1 | Constants | 54.7803 -54.7776 (independent variable is in-package RH) | Abstr 2DHLW Nodrip Ion Str.xls |
| InPkg_Istr_2DHLW_NS_Max_Factor InPkg_Istr_2DHLW_NS_Min_Factor | Maximum and minimum ionic strength factors under vapor influx conditions in Cell 1a of CDSP packages | Constants | 1.5 0.5 | Abstr 2DHLW Nodrip Ion Str.xls |
| Istr_2MCO_NS_A1 Istr_2MCO_NS_B1 Istr_2MCO_NS_RH_1 | Intercept (A1) and slope (B1) of linear equation used to calculate the mode of the ionic strength (molal) under vapor influx (no-drip) conditions in Cell 1b of CDSP packages as a function of in-package RH when in-package RH is below Istr_2MCO_NS_RH_1 | Constants | 42.992 -42.654 98.33% (independent variable is in-package RH) | Abstr 2MCO Nodrip Ion Str.xls |
| Istr_2MCO_NS_A2 Istr_2MCO_NS_B2 | Intercept (A2) and slope (B2) of linear equation used to calculate the mode of the ionic strength (molal) under vapor influx (no-drip) conditions in Cell 1b of CDSP packages as a function of in-package RH when in-package RH is above Istr_2MCO_NS_RH_1 | Constants | 62.4136 -62.4112 (independent variable is in-package RH) | Abstr 2MCO Nodrip Ion Str.xls |
| InPkg_Istr_2MCO_NS_Max_Factor InPkg_Istr_2MCO_NS_Min_Factor | Maximum and minimum ionic strength factors under vapor influx conditions in Cell 1b of CDSP packages | Constants | 1.5 0.5 | Abstr 2MCO Nodrip Ion Str.xls |

Table 6-21[a]. TSPA-LA Parameters in IPC Abstractions (Continued)

| Parameter Name | Description | Type | Independent Variable(s) or Value | TSPA-LA Roadmap: Source File in Output DTN: SN0702PAIPC1CA.001 |
|------------------------|---|----------|--------------------------------------|---|
| InPkg_pH_CSNF_SDM_Max | Maximum pH under liquid influx conditions in Cell 1 of CSNF packages | Table | Log ionic strength, pCO ₂ | Abstr CSNF Seepage pH.xls |
| InPkg_pH_CSNF_SDM_Min | Minimum pH under liquid influx conditions in Cell 1 of CSNF packages | Table | Log ionic strength, pCO ₂ | Abstr CSNF Seepage pH.xls |
| InPkg_pH_2DHLW_SDM_Max | Maximum pH under liquid influx conditions in Cell 1a of CDSP packages | Table | Log ionic strength, pCO ₂ | Abstr 2DHLW Seepage pH.xls |
| InPkg_pH_2DHLW_SDM_Min | Minimum pH under liquid influx conditions in Cell 1a of CDSP packages | Table | Log ionic strength, pCO ₂ | Abstr 2DHLW Seepage pH.xls |
| InPkg_pH_2MCO_SDM_Max | Maximum pH under liquid influx conditions in Cell 1b of CDSP packages | Table | Log ionic strength, pCO ₂ | Abstr 2MCO Seepage pH.xls |
| InPkg_pH_2MCO_SDM_Min | Minimum pH under liquid influx conditions in Cell 1b of CDSP packages | Table | Log ionic strength, pCO ₂ | Abstr 2MCO Seepage pH.xls |
| InPkg_pH_CSNF_NDM_Max | Maximum pH under vapor influx (no-drip) conditions in Cell 1 of CSNF packages | Table | Log ionic strength, pCO ₂ | Abstr CSNF Nodrip pH.xls |
| InPkg_pH_CSNF_NDM_Min | Minimum pH under vapor influx (no-drip) conditions in Cell 1 of CSNF packages | Table | Log ionic strength, pCO ₂ | Abstr CSNF Nodrip pH.xls |
| InPkg_pH_2DHLW_NDM_Max | Maximum pH under vapor influx (no-drip) conditions in Cell 1a of CDSP packages | Table | Log ionic strength, pCO ₂ | Abstr 2DHLW Nodrip pH.xls |
| InPkg_pH_2DHLW_NDM_Min | Minimum pH under vapor influx (no-drip) conditions in Cell 1a of CDSP packages | Table | Log ionic strength, pCO ₂ | Abstr 2DHLW Nodrip pH.xls |
| InPkg_pH_2MCO_NDM_Max | Maximum pH under vapor influx (no-drip) conditions in Cell 1b of CDSP packages | Table | Log ionic strength, pCO ₂ | Abstr 2MCO Nodrip pH.xls |
| InPkg_pH_2MCO_NDM_Min | Minimum pH under vapor influx (no-drip) conditions in Cell 1b of CDSP packages | Table | Log ionic strength, pCO ₂ | Abstr 2MCO Nodrip pH.xls |
| IPC_RH_Threshold | Minimum relative humidity in an IPC cell required to predict pH from the IPC abstractions above. Below this threshold, only degradation occurs. | Constant | 95% | Section 6.10.9.1[a] |

Table 6-21[a]. TSPA-LA Parameters in IPC Abstractions (Continued)

| Parameter Name | Description | Type | Independent Variable(s) or Value | TSPA-LA Roadmap: Source File in Output DTN: SN0702PA IPC1CA.001 |
|--|--|-----------|----------------------------------|--|
| IPC_pH_CSNF_lowRH_Max IPC_pH_CSNF_lowRH_Min | Maximum and minimum proxy pH values used in calculating CSNF degradation rate when the relative humidity is below IPC_RH_Threshold | Constants | 7 6 | Section 6.10.9.1[a] |
| IPC_pH_HLWG_lowRH | Proxy pH used in calculating the HLW glass degradation rate when the relative humidity is below IPC_RH_Threshold | Constant | 10 | Section 6.10.9.1[a] |
| SeepRate_Threshold | Minimum liquid influx (L/yr) into a cell for liquid influx abstraction | Constant | 0.1 | Section 6.10.9.1[a] |

Table 6-22[a]. Non-TSPA-LA Parameters in IPC Abstractions Used in Dissolved Concentration Limits of Elements with Radioactive Isotopes (SNL 2007 [DIRS 177418])

| Parameter Name | Description | Type | Independent Variable(s) or Value | Source File in Output DTN: SN0702PAIPC1CA.001 |
|---------------------|--|----------|----------------------------------|---|
| IPC_Fmax_seep_CSNF | Maximum concentration of fluoride ion in Cell 1 of a CSNF package with a failed drip shield and seepage | Table | Ionic strength | Abstr CSNF Seepage F.xls |
| IPC_Fmax_cond_CSNF | Maximum concentration of fluoride ion in Cell 1 of a CSNF package with an intact drip shield or with a failed drip shield and condensation only | Constant | 0 molal | Not applicable |
| IPC_Fmax_seep_2DHLW | Maximum concentration of fluoride ion in Cell 1a of a CDSP package with a failed drip shield and seepage | Constant | 0.00025 molal | Abstr 2DHLW Seepage F.xls |
| IPC_Fmax_cond_2DHLW | Maximum concentration of fluoride ion in Cell 1a of a CDSP package with an intact drip shield or with a failed drip shield and condensation only | Constant | 0 molal | Not applicable |
| IPC_Fmax_seep_2MCO | Maximum concentration of fluoride ion in Cell 1b of a CDSP package with a failed drip shield and seepage | Table | Ionic strength | Abstr 2MCO Seepage F.xls |
| IPC_Fmax_cond_2MCO | Maximum concentration of fluoride ion in Cell 1b of a CDSP package with an intact drip shield or with a failed drip shield and condensation only | Constant | 0 molal | Not applicable |

6.10.9.1[a] Choice of Vapor or Liquid Influx Abstraction

The choice between the vapor and liquid influx abstractions depends on the RH of the waste package, the liquid influx rate, and the abstraction that predicts lower ionic strength. A matrix that defines the decisional process based on the values of these input and output parameters is presented in Table 6-23[a].

For the vapor influx model, approximately 8 to 20 monolayers of water are required for bulk water conditions (Section 5.1 of the parent report). Calculations in *EBS Radionuclide Transport Abstraction* (SNL 2007 [DIRS 177407], Section 6.5.2.2.1) indicate that the number of layers of water increases with RH and that there are approximately 9.1 monolayers of water at 95% RH in the CSNF corrosion products domain when water saturation is maximized. Based on these calculations and relationships, a relative humidity of 95% is established as the minimum threshold value required to predict a meaningful pH from the vapor influx model. Given the ultimately small volume of water film being considered at these RH conditions (RH < 95%), virtually all of the waste form materials degraded under these conditions will be precipitated in place and not be available for transport. The small mass of radionuclides that remains dissolved in this minuscule water volume would also not be available for transport because the water film is not expected to be interconnected under these conditions.

Below a relative humidity of 95%, the pH of water films in the waste form cells is predicted in other ways. For HLW glass, a pH value of 10 is assumed, as is done in the TSPA-LA model for temperatures greater than 100°C (Assumption 5.5[a]). This pH is in the range of the maximum pH values calculated in the IPC abstraction for the HLW glass cell (Figure 6-42[a]). For CSNF, the dominant degradation product is schoepite. The pH approximation for water films in the CSNF waste form cell is derived from the pH zero point of charge (pH_{zpc}) of schoepite, which is approximated from the pH of minimum solubility (Section 6.3.4.1[a]). At a partial pressure of CO₂ of 0.01 bar, the minimum schoepite solubility occurs around pH 6 while at 0.0001 bar it occurs around pH 7 (Langmuir 1997 [DIRS 100051], p. 499; also Figure 6-3[a]). To account for uncertainty, the pH of water films in the CSNF cell at RH below 95% is sampled uniformly from 6 to 7. The pH in CDSP Cell 1b (2MCO) at RH below 95% can be approximated in the same manner as the CSNF waste form cell because it will contain large amounts of schoepite.

For the liquid influx case, ionic strength is primarily affected by liquid influx rate, time since breach, and degradation rates, as explained in Section 6.10.2.1[a]. Under low liquid influx conditions, degradation reactions add ions to solution faster than they can be flushed out of the cell; thus, the composition of the liquid influx makes no difference at low liquid influx rates. However, because the liquid influx model does not simultaneously include the process of vapor influx, it will tend to overestimate ionic strength at low liquid influx rates. Thus, whenever a lower ionic strength is predicted by the vapor influx model, the lower ionic strength should be used. At high liquid influx rates or when degradation reactions cease, ionic strength is generally predicted to be much closer to the ionic strength of the incoming water.

Table 6-23[a] shows that whenever the liquid influx rate is less than 0.1 L/yr (and RH is 95% or higher), the vapor influx abstraction is used to calculate ionic strength and pH. This criterion is justified because a liquid influx rate less than 0.1 L/yr is negligible compared to vapor influx. At or above 0.1 L/yr (and at or above 95% RH), the liquid influx abstraction is used to determine

ionic strength and pH except when the vapor influx abstraction predicts a lower ionic strength. For this exception, the vapor influx ionic strength is used to determine the pH from the liquid influx abstraction. This exception is justified because the liquid influx model simulations exclude the process of vapor influx.

The vapor influx model sets a reasonable upper limit for ionic strength because the ionic strength should not exceed the value that would be in equilibrium with the in-package relative humidity. Degradation reactions can decrease the amount of water in the cell at a low (or zero) liquid influx rate; however, vapor influx is rapid and is driven by an essentially unlimited source, so any water consumed by the relatively slow process of degradation is quickly replenished, preventing the ionic strength from increasing above the value calculated in the vapor influx abstraction. However, in the event that there are nonequilibrium conditions such that RH is less than 95% and liquid influx is 0.1 L/yr or higher, the liquid influx model should be used to determine pH and ionic strength.

Table 6-23[a]. pH and Ionic Strength Abstraction Matrix

| RH | ≥ 95% | | < 95% | |
|-------------|-------------|-------------|-------------|--|
| q (L/yr) | ≥ 0.1 | < 0.1 | ≥ 0.1 | < 0.1 |
| V(I) < L(I) | V(I), L(pH) | V(I), V(pH) | L(I), L(pH) | CSNF: 6 ≤ pH ≤ 7 2DHLW: pH = 10 2MCO: 6 ≤ pH ≤ 7 |
| V(I) ≥ L(I) | L(I), L(pH) | | | |

NOTES: V(I) = ionic strength from vapor influx abstraction.
V(pH) = pH from vapor influx abstraction.
L(I) = ionic strength from liquid influx abstraction.
L(pH) = pH from liquid influx abstraction.

6.10.9.2[a] Liquid Influx Abstraction

The independent variables of the liquid influx abstraction are liquid influx rate (q), time since waste package breach (t'), the negative log of the partial pressure of carbon dioxide ($p\text{CO}_2$), and the sampled percentiles for IPC model output uncertainties. IPC model outputs do not change randomly from time step to time step. Therefore, while IPC model output distributions change from one time step to the next due to changes in q , t' , and $p\text{CO}_2$, sampled percentiles for these outputs for a given TSPA-LA realization are held constant for the realization.

The first parameter to determine in the IPC abstraction is the ionic strength (I). The mode of $\log I$ (e.g., IPC_logISmode_seep_CS NF in Table 6-21[a]) is determined from a lookup table (e.g., *Abstr CSNF Seepage Ion Str 2.xls* in Output DTN: SN0702PAIPC1CA.001). $\log I$ is a function of $\log t'$ and $\log q$. The lookup table defines a continuous surface such that intermediate values of the mode of $\log I$ are determined by linear interpolation of $\log t'$ and $\log q$. If $\log t'$ or $\log q$ is beyond the range provided in the lookup table, the nearest value of $\log t'$ or $\log q$ is used. Uncertainty in $\log I$ is represented by a triangular distribution having a maximum and minimum deviation from the mode (Section 6.10.8.2[a]). $\log I$ distribution is negligibly affected by the source of the liquid influx (Section 6.6.2[a]).

Once the realized $\log I$ is determined, it is used along with $p\text{CO}_2$ to determine the pH minimum and maximum values from the lookup tables. The lookup tables for minimum and maximum pH

define continuous surfaces such that intermediate values of these outputs are determined by linear interpolation of $\log I$ and $p\text{CO}_2$. If $\log I$ or $p\text{CO}_2$ is beyond the range provided in the lookup table, the nearest value of $\log I$ or $p\text{CO}_2$ is used. The pH distribution is uniform between the minimum and maximum pH values defined in lookup tables (e.g., IPC_pHmax_seep_CSNF and IPC_pHmin_seep_CSNF in *Abstr CSNF Seepage pH.xls*, Output DTN: SN0702PAIPC1CA.001). The realized pH of the water in the cell for the given time period is determined by applying the sampled percentile to the uniform distribution. Uncertainty in the realized pH value is built into the minimum and maximum values abstracted in the lookup tables (Section 6.10.8.1[a]). The pH distribution is negligibly affected by the source of the liquid influx (Section 6.6.2[a]).

6.10.9.3[a] Vapor Influx Abstraction

As in the liquid influx abstraction, sampled percentiles for IPC abstraction outputs of the vapor influx abstraction are to be held constant over time for each realization. The vapor influx abstraction requires the following independent variables: the sampled IPC output percentiles, RH, and the partial pressure of CO_2 .

Ionic strength is the first parameter to determine. The mode of I (e.g., IPC_ISmode_nodrip_CSNF) is determined from an equation in the abstraction spreadsheet (e.g., in file *Abstr CSNF nodrip Ion Str 2.xls* from Output DTN: SN0702PAIPC1CA.001). In the vapor influx abstraction, I is only a function of RH. This function covers all possible values of RH, so extrapolations are not applicable. As indicated in Section 6.10.9.1[a], a minimum of approximately 8 to 20 monolayers of water is required for the vapor influx model. Uncertainty in I is represented by a triangular distribution with maximum and minimum values that are $\pm 50\%$ of the mode (Section 6.10.8.2[a]).

Once the realized I is determined, it is used along with $p\text{CO}_2$ to determine the pH distribution. As in the liquid influx case, the lookup tables for minimum and maximum pH define continuous surfaces whereby intermediate values of these outputs are determined by linear interpolation of $\log I$ and $p\text{CO}_2$. If $\log I$ or $p\text{CO}_2$ exceeds the range provided in the lookup table, the nearest value of $\log I$ or $p\text{CO}_2$ is used. The pH distribution is uniform between the maximum and minimum pH values defined in lookup tables (e.g., IPC_pHmax_nodrip_CSNF and IPC_pHmin_nodrip_CSNF in *Abstr CSNF nodrip pH.xls* in Output DTN: SN0702PAIPC1CA.001). The realized pH of the water in the cell for the given time period is determined by applying the sampled percentile to the uniform distribution. Uncertainty in the realized pH value is built into the maximum and minimum values abstracted in the lookup tables (Section 6.10.8.1[a]).

6.10.9.4[a] Fluoride Abstraction

The fluoride abstraction is used by *Dissolved Concentration Limits of Elements with Radioactive Isotopes* (SNL 2007 [DIRS 177418]) in assessing uncertainties in radionuclide solubility. Maximum fluoride concentrations for liquid influx abstractions are documented in Table 6-20[a] and in the files identified in Table 6-22[a]. They are a function of ionic strength for liquid influx conditions in CSNF Cell 1 and CDSP Cell 1b. For vapor influx conditions and for intact drip shields, there are no significant sources of fluoride in any of the IPC cells, so the maximum fluoride concentration is set at zero.

6.11[a] CONDITION REPORTS

A total of ten condition reports (CRs) that were open during the development of this addendum cite or apply to the parent report. Each CR is addressed in this section in numerical order.

CR-5384 concerns the use of basalt waters in *Dike/Drift Interactions* (SNL 2007 [DIRS 177430]). It cites the parent IPC report but requires no action concerning it. The issue raised in the CR is being addressed in *Dike/Drift Interactions* (SNL 2007 [DIRS 177430]).

CR-6731 concerns the thermodynamic data for select phosphate species. Corrections to these data were made in the production of the *data0.ymp.R5* database documented in *Qualification of Thermodynamic Data for Geochemical Modeling of Mineral-Water Interactions in Dilute Systems* (SNL 2007 [DIRS 177409], Section 6.7.8). This updated database is the source database for all calculations in this addendum (Section 4.1.1[a]). Thus, with regard to this addendum, CR-6731 is addressed.

CR-6857 concerns the justification of the HLW glass degradation rates used in IPC simulations. This CR is addressed in Section 6.3.1.3.4[a].

CR-7233 pertains to whether Output DTN: MO0507SPAINPKG.000 was produced by the parent document. This DTN was not produced as part of the parent report (nor as part of this addendum). It was downgraded to unqualified primarily because the DTN includes inputs and sensitivity calculations that are not intended for use beyond the realm of the parent report. A “readme” file was added to the DTN to explain the downgrade, and an Impact Review Action Notice was initiated. Also, as explained in the “readme” file, qualified output from the parent document is adequately provided in Output DTN: MO0502SPAINPCA.000.

CR-7464 pertains to design changes in the mass of two components in the CSNF waste package, the Ni-Gd fuel basket plates and the 316 stainless steel spread ring. The changes amounted to differences of approximately one percent or less of the total masses of Ni-Gd alloy and 316 stainless steel in the waste package. These changes have a negligible effect on waste package IPC model outputs. The masses of components in the waste package remain uncertain. The current TAD design uses borated stainless steel in place of Ni-Gd alloy (Section 4.1.4.1[a]).

CR-7542 concerns an update of the thermodynamic data for NiCO₃. The data for this mineral was updated in the *data0.ymp.R5* database (SNL 2007 [DIRS 177409], Table 6-61). This updated database is the source database for all calculations in this addendum (Section 4.1.1[a]). Thus, with regard to this addendum, CR-7542 is addressed.

CR-7756 concerns the addition to the thermodynamic database of a number of uranium mineral phases found in nature and in UO₂ degradation experiments. In response to this CR, several uranium phases were added to the *data0.ymp.R5* database (SNL 2007 [DIRS 177409], Table 4-53). This updated database is the source database for all calculations in this addendum (Section 4.1.1[a]). Thus, with regard to this addendum, CR-7756 is addressed.

CR-7763 pertains to the prediction and experimental validation of radionuclide solubility at high pH and high ionic strength. It primarily concerns *Dissolved Concentration Limits of Elements with Radioactive Isotopes* (SNL 2007 [DIRS 177418]). The IPC abstractions are not affected by

the radionuclide solubility calculations of concern. The only such calculation that affects the waste form IPC abstractions is that of schoepite. However, when schoepite is important (in the CSNF and 2MCO cells), its pH buffering capacity prevents the pH from advancing to the high pH range where experimental solubility data are lacking. Thus, the issue involving this CR is not applicable to the IPC model. The issue will be addressed in *Dissolved Concentration Limits of Elements with Radioactive Isotopes* (SNL 2007 [DIRS 177418]).

CR-8555 concerns the title and description of the technical data input form (TDIF) for Output DTN: MO0507SPAINPKG.000. The title and description are technically correct but are not clear. To resolve this issue, a “readme” file was added to the DTN to provide a clear description of the contents. In addition, the status of this DTN was downgraded to unqualified in response to CR-7233 (see above).

CR-8656 concerns a disparity between the sources listed within DTNs of the parent report in the Automated Technical Data Tracking system and what is listed within the Document Input Reference System (DIRS). For those DTNs that are product output of the report, the sources have been removed per practice established on 06/04/2004 (Howard 2004 [DIRS 177137]), which directs the removal of sources within product output DTNs and the addition of language to the comments field of each DTN’s TDIF, directing users to the DIRS document associated with the report. For the DTNs with supersession issues that are not product output, the improper sources have been amended or removed as appropriate.

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7[a]. VALIDATION

The purpose of the in-package chemistry (IPC) model and the IPC model abstraction is to predict the bulk chemistry inside of a breached waste package and to provide simplified expressions of that chemistry for use in TSPA-LA. The in-package chemistry model consists of a batch reactor model. The batch reactor model has two water-influx conditions, liquid influx and vapor influx. These fluids sponsor fuel degradation and structural component degradation at equilibrium with local oxygen and carbon dioxide fugacities. Based on these models, output model abstractions were generated in Section 6.10[a].

The development of the IPC abstractions for TSPA-LA consisted of (1) developing a conceptual model; (2) establishing the environmental conditions within the repository as a function of time; (3) mathematically simulating the system using a geochemical simulation code and a qualified thermodynamic data base, and (4) compiling the parameter estimates into a form that can be used by TSPA-LA. The geochemical simulation codes and the thermodynamic database are individually qualified and controlled and require no additional validation. The environmental conditions are specified in various qualified YMP reports and require no additional validation. Thus, validation is limited to the conceptual model and the parameter estimates.

The TWP (BSC 2006 [DIRS 177389], Table 2-3) indicates that the IPC model requires a medium level of confidence (Level II) because of the impact of pH, one of the main outputs of the IPC model, on plutonium and neptunium solubility.

Achieving Level I criteria requires satisfying the pre-model development criteria (Section 7.1[a]) and conducting a single post-model development validation method (Section 7.2[a]). Achieving Level II criteria requires satisfying Level I criteria and conducting an additional post-model development validation method (SCI-PRO-002, Attachment 3).

The validation methods outlined in the TWP (BSC 2006 [DIRS 177389]) are considered insufficient to provide the high confidence needed for validation of solution chemistries predicted by IPC over geologic time frames. Additional methods are described in Section 7.3[a]. The validation activities presented in subsequent sections provide all validation necessary for the waste form IPC model and abstractions in this addendum.

7.1[a] DOCUMENTED DECISIONS AND ACTIVITIES IMPLEMENTED DURING MODEL DEVELOPMENT PROCESS

Activities required to build confidence in the estimates generated by the various models are specified in SCI-PRO-006 (Sections 6.3.1 and 6.3.2) and in SCI-PRO-002 (Attachment 3). The specified pre-model development activities for Level I validation are:

1. *Evaluate and select input parameters and/or data that are adequate for the model's intended use (SCI-PRO-002, Attachment 3).*

Section 4.1[a] provides the inputs used in the IPC model, which include waste package design information, material compositions, atomic weights, densities, water compositions, waste form compositions, reaction rates, and thermodynamic data. These inputs were chosen to best represent all of the components of the IPC model and were

taken from qualified (Q) sources or were qualified for their use in this document. Therefore, this criterion has been met for all of the salient features.

2. *Formulate defensible assumptions and simplifications that are adequate for the model's intended use (SCI-PRO-002, Attachment 3).*

Sections 5[a] and 6.3.1[a] provide the assumptions (Section 5[a]) and simplifications (Section 6.3.1[a]) used in the IPC model. All model assumptions and simplifications have been shown to be either conservative or captured within the uncertainty of the model. Therefore, this criterion has been met for all of the salient features of the model.

3. *Ensure consistency with physical principles, such as conservation of mass, energy, and momentum, to an appropriate degree commensurate with the model's intended use (SCI-PRO-002, Attachment 3).*

Sections 6.3[a] and 6.3.1[a] provide brief discussions of the EQ6 geochemical modeling software used for the IPC model simulations. EQ6 simulations conserve mass, abide by thermodynamic principles, and enforce charge balance; therefore, the IPC model abstractions, which are based on EQ6 calculations, are also consistent with physical principles of conservation. This criterion has been met for all of the salient features.

4. *Represent future state (aleatoric), parameter (epistemic), alternative model uncertainties in an appropriate degree commensurate with the model's intended use (SCI-PRO-002, Attachment 3).*

Unexpected random (aleatoric variation) events are outside of the realm of the IPC model. However, the IPC model was designed to be sufficiently robust to handle large changes in temperature, flux, and seepage composition. Alternative conceptual models are discussed in Section 6.4[a] and the current IPC model's robustness eliminates the need for these alternative conceptual models. Therefore, this criterion has been met for all of the salient features.

5. *Ensure simulation conditions have been designed to span the range of intended use and avoid inconsistent outputs or that those inconsistencies can be adequately explained and demonstrated to have little impact on the results.*

Section 6.3.1[a] outlines the wide range of inputs (flux, fuel exposure, temperature, reactant combinations) that were used in the IPC model. These conditions span the range of intended use in TSPA-LA for which the IPC model is used. Therefore, this criterion has been met for all of the salient features.

6. *Ensure that model predictions (performance parameters adequately represent the range of possible outcomes, consistent with important uncertainties and modeling assumptions, conceptualizations, and implementation (SCI-PRO-002, Attachment 3).*

Section 6.3.1[a] provides the input conditions that span the range of the intended use of the model; thus, logic dictates that the range of outputs (performance parameters) also represents the range of possible outcomes. Additionally, sensitivity analyses have

been performed to expand the performance parameters to values consistent with important conditional uncertainties. Therefore, this criterion has been met for all of the salient features.

In addition to the pre-model development activities, Attachment 3 of SCI-PRO-002 specifies that one post-model development activity must be conducted to attain Level I validation and two must be conducted to attain Level II validation. The pre-development methods are listed in Section 6.3.2 of SCI-PRO-006 and are discussed in Section 7.1[a] of this addendum. The post-development validation of models outlined by the TWP is discussed in Section 7.2[a] and the rationale for selection of the post-development methods is explained in Section 7.3[a].

7.2[a] POST-MODEL DEVELOPMENT VALIDATION METHODS

Level II validation includes two methods of post-model development validation consistent with a model of moderate importance to mean annual dose estimates. The TWP (BSC 2006 [DIRS 177389], Table 2-3) states that the post-development validation of the model will focus on the following issues, each associated with a specific validation activity (VA):

- A. Are the types of minerals formed and the aqueous concentrations predicted by the model corroborated by experimental results (VA 1)?
- B. Because the output will be applied over geologic time spans, are the specific outputs of this model (pH, ionic strength) corroborated by natural analogue observation and relevant experimental results (VA 1 and/or VA 3)?
- C. Is the fluoride model output corroborated by experimental data (VA 1)?
- D. Is the total carbonate model output consistent with published literature (VA 3)?
- E. Is the Eh model output corroborated by experimental data (VA 1)?
- F. Surface-complexation portion of the model will be validated by corroboration with experimental data (VA 1) and independent technical review (VA 5).
- G. Validation of the abstraction models will be accomplished by comparison of their output with the output from the original process models (VA 6).

The development of the new conceptual two-cell model and the change to an all stainless steel CSNF waste package removed the carbon steel from Cell 1 – the focus of the IPC model – and made surface complexation a minor process that is not evaluated. Thus, validation activity F is no longer applicable.

It was found that some of the proposed post-development validation methods could not be implemented. For example, the estimated equilibrium mineral assemblages can be compared to both natural analogue observations of uranium and iron ore deposits and to the published data from fuel degradation experiments. However, the estimated solution compositions – e.g., pH, ionic strength, fluoride concentration, etc. – do not have natural analogues and are usually externally imposed on solubility and corrosion experiments. This leaves only two of the

post-development validation activities outlined in Section 6.3.2 of SCI-PRO-006 available to validate solution chemistry model output: critical review and comparison of estimates from two independent models. These are the activities that will be used for validation of the solution compositions.

The reduction/oxidation potential (Eh) is not used in the TSPA-LA model, so Level I validation is sufficient.

7.3[a] RATIONALE FOR SELECTION OF VALIDATION METHODS

The TWP (BSC 2006 [DIRS 177389], Table 2-3) names three methods for validation, different methods for different aspects of the model report, and these methods are listed in Table 7-1[a]. The degradation phases and the solution chemistry properties are validated separately because the validation methods are different for the two parameter sets. The deviations from the TWP (BSC 2006 [DIRS 177389]) are for the validation of the pH, ionic strength, fluoride, chloride, and total carbonate models. The TWP lists comparison to natural, experimental, or published data for validation of these models. As explained previously, these approaches are problematic, so new validation criteria are provided for them in Table 7-1[a].

For the fuel-degradation and the steel-degradation portions of the model, the predicted degradation phases are compared to experimental data from laboratories because the laboratory experiments were conducted in various environments intended to capture the possible range of conditions anticipated to exist in the repository. Comparison of the EQ3/6 simulation estimates for degradation phases with information from either natural or experimental analogues was determined to be the most appropriate method to validate the model results regarding the types of minerals that are anticipated to form in the waste package prior to and following any postulated breaches. This selection was based on the conclusion that for long-term environmental performance (i.e., 10,000 or more years) the use of analogues presented the best opportunity to provide a meaningful comparison with the model outputs of interest. While the experimental data support the predicted degradation phases, the duration of the experiments is very short compared to the repository time frame. The second validation method for the predicted degradation phases is via a critical review.

For the solution properties (e.g., pH, ionic strength, fluoride concentration, etc.), there are no natural analogues for which one can compare the estimated values. The majority of the uranium deposits in the world are in the vadose zone, and those that are located below the water table impose a reducing environment on the groundwater and have a very limited range of pH and very low ionic strengths. The Yucca Mountain repository has been assumed to be oxidizing over the majority of the one Ma regulatory period. Because stainless steels only became widely available in the early part of the twentieth century, there are no ancient artifacts that one could consider a natural analogue. All of the natural materials have deficiencies preventing them from being considered an analogue – e.g., meteorites are seldom found in saturated environments and, unlike stainless steel, they have very low chromium contents.

Thus, for the pH, ionic strength, and fluoride portions of the model, the estimated concentrations are validated via a critical review and comparison of the EQ6 calculations to the calculations of an independent geochemical modeling code, PHREEQC.

The surface complexation model is not used in this addendum.

Table 7-1[a]. Validation Criteria, Activities, and Metrics

| Validation Criteria / Activities | Metric / Criteria |
|---|---|
| Is the fuel-degradation model consistent with results from laboratory experiments published in refereed journals or literature? (VA 1, 3) ^a | The criteria for meeting all of the corroborating validation activities will be qualitative. |
| Are the types of minerals formed and the aqueous concentrations predicted by the model consistent with experimental results? (VA 1) (Same as BSC 2006 [DIRS 177389], Table 2-3, Point A) | |
| Is the steel-degradation model consistent with results from laboratory experiments on man-made analog materials? (VA 1) | |
| Because the output will be applied over geologic time spans, are the specific outputs of this model (pH, ionic strength) will be validated via an independent technical review and by comparison of the estimates from two independent models. (VA 5) | The independent technical review will assure that the modeling is done using best scientific practices and in accordance with all applicable procedures. The estimates from the geochemical simulation codes will agree within two orders of magnitude. |
| Is the fluoride model output consistent with experimental data? (VA 5) (Same as BSC 2006 [DIRS 177389], Table 2-3, Point C) | The independent technical review will assure that the modeling is done using best scientific practices and in accordance with all applicable procedures. The estimates from the geochemical simulation codes will agree within two orders of magnitude. |
| Is the total carbonate model output consistent with published literature? (VA 5) (Same as BSC 2006 [DIRS 177389], Table 2-3, Point D) | |
| Is the Eh model output consistent with experimental data? (VA 5) (Same as BSC 2006 [DIRS 177389], Table 2-3, Point E) | The Eh model is no longer used in the TSPA-LA simulations and does not require Level II validation. |

^a The activities that are taken verbatim from BSC 2006 [DIRS 177389] are noted. The remainder are modified from those listed in Table 2-1 of BSC 2006 [DIRS 177389].

NOTE: The validation criteria, activities, and metrics as presented in this table are adapted from BSC 2006 [DIRS 177389]. VA = validation activity as defined in BSC 2006 [DIRS 177389], Table 2-1.

7.4[a] WASTE PACKAGE DEGRADATION PRODUCTS VALIDATION

The change in the conceptual model to a two-cell system effectively removed all of the Carbon Steel Type A-516 from Cell 1 and placed it into Cell 2. This addendum is only concerned with the solutions and reactions in Cell 1 of the CSNF and CDSP waste packages. Thus, Section 7.4.4[a], “Steel-Degradation Model Consistent with Results from Laboratory Experiment,” which covered both carbon and stainless steel in the parent report, is now limited to stainless steel.

7.4.1[a] pH and Ionic Strength Validation

As explained in Section 7.3[a], there are no defensible natural analogues for the compositions of the solutions within Cell 1 of either the CSNF or CDSP waste packages. Thus, the two methods available are an independent technical review and comparison of the estimated values from two independent modeling codes. The results of the critical review by Laura Price are contained in Section 7.10.2[a].

The comparison of the output estimates from EQ6 and PHREEQC are contained in Validation DTN: SN0705IPCEQPHR.001 and summarized in Section 7.10.1[a].

7.4.2[a] Degradation Rates Consistent with Results from Laboratory Experiments

The fuel and waste form degradation models were derived from laboratory experiments performed by DOE contractors and are documented and validated in YMP reports *CSNF Waste Form Degradation: Summary Abstraction* (BSC 2004 [DIRS 169987]), *DSNF and Other Waste Form Degradation Abstraction* (BSC 2004 [DIRS 172453]), and *Defense HLW Glass Degradation Model* (BSC 2004 [DIRS 169988]).

7.4.3[a] Degradation Products Consistent with Experimental Results

The primary phases predicted to form in the model are ferric oxyhydroxides (goethite, etc.) and spinels (trevorite, etc.) from stainless steel corrosion, and schoepite from the degradation of spent nuclear fuel. Iron-based alloys and minerals are observed in nature and in experiments to corrode to metal oxides whose specific identities depend upon the particular conditions of corrosion and the composition of the steel. Hematite and goethite are routinely observed as the most common iron oxides in soil (Schwertmann and Taylor 1995 [DIRS 105959]) and are observed in the corrosion products in both natural analogues (Johnson and Francis 1980 [DIRS 125291]) and laboratory experiments (Kim 1999 [DIRS 105168]). Analyses of oxide coatings on stainless steels under very aggressive conditions (e.g., low pH nitric acid solutions, induced potentials, pressurized water reactor environment, etc.) show that inverse chromium or nickel-iron spinels are the major components (Wang et al. 2001 [DIRS 178973]; Da Cunha Belo et al. 1997 [DIRS 178971]).

Schoepite is observed as an alteration product in experiments (BSC 2004 [DIRS 169987]; SNL 2007 [DIRS 177418]; Wronkiewicz et al. 1996 [DIRS 102047]) and at the Peña Blanca natural analogue (Langmuir 1997 [DIRS 100051], p. 512; Palache et al. 1944 [DIRS 163604]). The aqueous concentrations of Fe and U in fluids in contact with ferric hydroxides and schoepite, respectively, are fixed by the presence of these solids (Section 7.7 of Langmuir 1997 [DIRS 100051]; Section 7.2 of Stumm and Morgan 1996 [DIRS 125332]).

The incongruent dissolution of HLW glass prevents the formation of pure aluminum phases such as gibbsite ($\text{Al}(\text{OH})_3$) or boehmite (AlOOH) (Hsu 1995 [DIRS 105875]) that are normally found in soils. The ready availability of silica predicts the formation of clays and zeolites similar to those found with basaltic glasses from Iceland and Hawaii (Cowan and Ewing 1989 [DIRS 124396]; Luo et al. 1997 [DIRS 163271]; Grambow et al. 1986 [DIRS 163258]).

7.4.4[a] Steel-Degradation Model Consistent with Results from Laboratory Experiment

The new waste package designs (SNL 2007 [DIRS 179394], Table 4-1; SNL 2007 [DIRS 179567], Table 4-10) replace all of the Carbon Steel Type A516 with Stainless Steel Type 316L. The stainless steel degradation rates cover a broad range and are consistent with many experimental data as documented in *Aqueous Corrosion Rates for Waste Package Materials* (BSC 2004 [DIRS 169982]).

7.4.5[a] Natural Analogue Constraints on Solution Compositions

While no natural analogues exist for the waste package solutions, data from natural systems can be used to constrain solution compositions and to build confidence in the predictions. Because

the output will be applied over geologic time spans, the specific outputs of this model (pH, ionic strength) are compared to natural analogue observation (rather than experimental data) to provide confidence that the behavior of the predicted interactions between materials and solutions are appropriate.

7.4.5.1[a] Production of Alkaline Waters by Glass Dissolution

Glass dissolution causes pH values greater than 7 when protons from solution exchange with alkalis, typically Na⁺, K⁺, or Ca²⁺, present at the dissolving glass–solution interface, e.g.:



The general link between dissolution of alkali-bearing silicate minerals and glasses with acidity consumption (alkalinity production) is amply documented in the peer-reviewed scientific literature. It has been hypothesized that large-scale weathering of alkali-bearing silicates ultimately leads to high pH values in alkali lakes (Berner and Berner 1987 [DIRS 161746], pp. 280 to 281). Experiments seeking to mimic interaction of seawater with basaltic glass on the ocean floor observe alkaline pH values (note that basaltic glass is generally considered to be an appropriate natural analogue for nuclear waste glass; see Ewing and Haaker 1979 [DIRS 161749]). A comparison of the major elements in SRL HLW glass and synthetic basalt glass is presented in Table 7-2[a].

Because basaltic glass is an appropriate analogue for waste glass, the alkaline, long-term pH values calculated for glass-bearing packages should be similar to those of ground waters from basalt formation and other rock types of basaltic composition. It is recognized that glass is only a small portion of a basalt, but the amorphous nature of the glass will cause it to react first and form the initial set of alteration minerals. In his authoritative review of the chemistry of natural waters, Hem (1995 [DIRS 115670]) gives the analyses of two basalt ground waters and one water from a tuff containing the basaltic mineral olivine. The basalt waters have pH values of 7.8 and 7.9 (Hem 1995 [DIRS 115670], Table 12, Analyses 4 and 5) and water from an olivine tuff has a pH of 8.2 (Hem 1995 [DIRS 115670], Table 16, Analysis 3). It must be recognized that these waters are in equilibrium with entire suite of minerals in the basalt and the values here are less than those predicted from the degradation of the alkali metal rich HLW glass.

Besides saturated environments, such as lakes and deep groundwater mentioned above, increases in pH due to degradation of glass have also been observed in unsaturated environments that are more analogous to repository conditions. A study of the natural alteration of ancient vitreous slags (the glassy refuse that remains after the smelting of metallic ore) found that all samples (100 to 4,000 years old) had a first stage of weathering in which the solution pH increased due to the reaction of the protons of the solution in the acid environment with the vitreous slags. This was found for different chemical compositions of glass and for different weathering environments, from open-air stacks to underground conditions. If sufficient oxygen exists, such as in the case of the open-air samples, the second stage involves further increases in pH toward a neutral to moderately basic environment, as the dissolution of the glass skeleton proceeds (Mahé-Le Carlier et al. 2000 [DIRS 171509]).

In short, the elevated pH values predicted by the CDSP Cell 1a HLWG simulations in the IPC model are corroborated by natural observations documented in the peer-reviewed literature (Vernaz and Goden 1992 [DIRS 174503]). This observation provides confidence that the estimates of pH in the solutions in contact with HLW glass will be greater than those associated with CDSP Cell 1b containing DSNF. Because the DSNF degradation model is very similar to that of CSNF, the fluids exiting CDSP Cell 1b will be modified to be very similar to those exiting the CSNF Cell 1.

Table 7-2[a]. Comparison of Major Elements of Synthetic Basalt Glass and SRL Vitrification Plant Glass

| Component | Synthetic Basalt Glass (wt %) ^a | Subduction Zone Glass (wt %) ^b | Average SRL HLW Glass (wt %) ^c |
|--------------------------------|--|---|---|
| Al ₂ O ₃ | 13.62 | 13.6 to 19.8 | 4.73 |
| B ₂ O ₃ | N/R | N/R | 8.07 |
| CaO | 10.72 | 6.51 to 9.59 | 1.35 |
| Fe _x O _y | 4.08 as Fe ₂ O ₃ and 8.18 as FeO | 8.12 to 13.4 as FeO | 11.63 as Fe ₂ O ₃ |
| Li ₂ O | N/R | N/R | 3.50 |
| Na ₂ O | 2.48 | 3.42 to 4.63 | 11.72 |
| MgO | 7.35 | 2.54 to 4.94 | 2.21 |
| MnO | 0.16 | 0.07 to 0.27 | 1.76 |
| P ₂ O ₅ | 0.10 | 0.12 to 0.35 | 0.55 |
| SiO ₂ | 50.76 | 51.1 to 56.8 | 50.48 |
| TiO ₂ | 1.66 | 0.67 to 1.89 | N/R |
| U ₃ O ₈ | N/R | N/R | 1.90 |

^a Crovisier et al. 1986 [DIRS 163211].

^b Sission and Layne 1993 [DIRS 122549], Table 1.

^c Taken from Table 4-6[a].

NOTE: N/R = not reported. Minor constituents of the basaltic glasses such as K₂O are not included in the comparison as these are not used in the glass composition in the model and are only minor constituents of basalt glass.

7.4.5.2[a] Lower Limits to pH Inside the Package

The minimum pH values inside the degrading waste form cell are predicted by EQ3/6 simulations to be around 5 (Figures 6-41[a] to 6-43[a]). The chemical makeup of the waste form corrosion products will be dominated by metal oxides and hydroxides similar to natural soils. The pH values in natural waters typically only become more acidic than 5 when high levels of organic acids are present, or when reduced sulfur is oxidized. Hem (1995 [DIRS 115670], Table 13, Analyses 1 and 4; Table 14, Sample 7) gives analyses of natural waters with pH values of 4.0, 3.8, and 3.0, respectively. These waters all contain relatively high proportions of dissolved sulfate, indicating that the source of their acidity is the oxidation of reduced sulfur. Oxidation of reduced sulfur can also be a major source of acidity in waters corroding waste package materials. However, the release of reduced sulfur from stainless steels is sufficiently slow that the concentration is never very high and its oxidation will not contribute much H⁺ into a given cell volume of solution. The levels of reduced sulfur in the waste package are too low to exceed the pH-buffering capacity of the corrosion products themselves (Section 6.10.1[a]). Given a relative absence of high levels of organic acids and reduced sulfur, the IPC model predicts low-end pH values near 5.

The pH ranges in the CSNF and 2MCO waste form cell abstractions (Figures 6-41[a] and 6-43[a]) are consistent with the pH range observed in UO₂ degradation experiments conducted at Argonne National Laboratory (ANL) (Wronkiewicz et al. 1996 [DIRS 102047]). Figure 5 of the ANL report (Wronkiewicz et al. 1996 [DIRS 102047]) shows that pH measurements varied between 5.9 and 8.2. The experiments involved slowly dripping J-13 well water, pre-equilibrated with crushed tuff at 90°C, onto UO₂ pellets encased in Zircaloy-metal sleeves and maintained in 90°C unsaturated environment over a period of ten years. The initial CO₂ partial pressure in the experiments was atmospheric (i.e., pCO₂ around 3.4), but because these assemblies were closed to the atmosphere between sampling events, the CO₂ partial pressure likely increased to some degree (i.e., pCO₂ decreased).

7.4.5.3[a] Neutralization of Alkaline and Acidic Waters

This section describes the neutralization of the alkaline waters produced by the dissolution of the HLW glass. There are no sources of acidic fluids within the waste packages. This neutralization is shown by comparing the pH values generated by the single components of the CDSP waste package in Figure 6-22 of the parent document with the combined reactants shown in Figure 6-23 of the parent document. The concept of acids and bases neutralizing one another is an established fact and does not require validation.

7.4.5.4[a] Production of Extreme Ionic Strength Solutions

High ionic strength solutions (>1 molal) are predicted to form when very low volumes of fluid (i.e., few mL) remain in contact with soluble waste form and waste package components for long periods of time. This condition can be sponsored by either stagnant water or evaporation of water from relatively dilute degradation solutions. This observation is supported by the observation that older geologic fluids tend to be saltier than younger ones that have had less time to react with solids (Drever 1982 [DIRS 100725]). It must be noted that the very small volume of these concentrated solutions will not be capable of containing a very large mass of material from the degradation of either structural components or nuclear waste.

Low ionic strength solutions (<0.05 mol/L) are predicted to form when high volumes of water flow past the waste form and waste package components. These low ionic strengths are conservative for colloid stability and thus result in the greatest radionuclide release in advective flow conditions.

7.4.6[a] Relationship between Ionic Strength and Flow Rate

Freeze and Cherry (1979 [DIRS 101173], Figure 7.17(b)) show the relationship between groundwater flow rate and concentration of solution for a constant reaction rate. The plot shows that water flowing at a higher flow rate, but with the same degradation rate and flow distance, will have a lower concentration than water flowing at a lower flow rate. For the in-package system, this is equivalent to saying that if water is flowing through the waste package, the water with the highest flow rate will exit the waste package with the lowest ionic strength. This behavior is exhibited in this model as shown in Section 6.6.4[a].

7.5[a] SURFACE COMPLEXATION MODEL

This model is not used in this addendum. Thus, no change.

7.6[a] FLUORIDE VALIDATION

Consolidated into Section 7.10[a], Solution Composition Validation.

7.7[a] CHLORIDE VALIDATION

The chloride was not modified in this addendum. No change.

7.8[a] TOTAL CARBONATE VALIDATION

The total carbonate model is unchanged from the parent report.

7.9[a] EH VALIDATION

The Eh model is not used in TSPA-LA and was not modified in this addendum. No change.

7.10[a] SOLUTION COMPOSITION VALIDATION

The validation of solution composition constituents has been accomplished via comparison of the estimates from two independent models (Section 7.10.1[a]) and a Critical Review (Section 7.10.2[a]).

7.10.1[a] Comparison of Model Estimates

The mathematical simulations of the IPC Cell 1 model are performed using EQ6 version 7.2bLV in Sections 6.5[a] and 6.6[a]. As a validation exercise to build confidence in the ability of EQ6 to execute the mathematical model, the CSNF base case was executed using a separate code, PHREEQC version 2.11 (see Table 3-1[a]). PHREEQC is a USGS code that has many of the same features as EQ6. It can be used to perform a variety of aqueous geochemical calculations and has capabilities for kinetic processes, mixing reactions, and one-dimensional transport. Each of the processes simulated by EQ6 in the waste package degradation model can be simulated using PHREEQC.

Section 7.10.1.1[a] documents the development of the PHREEQC CSNF simulations. The resulting PHREEQC calculations are compared to EQ6 calculations in Section 7.10.1.2[a]. The PHREEQC validation files are documented in Validation DTN: SN0705IPCEQPHR.001.

7.10.1.1[a] PHREEQC Implementation of CSNF Base Case

Time-dependent IPC model simulations use the solid-centered flow-through mode of the EQ6 code. This mode simulates the flow of source water into and through a constant-volume batch reactor. At the beginning of each equilibration step, a small amount of source water displaces an equal amount of water already equilibrated with the phases in the reactor. The water in the reactor then mixes completely, and the water, solids, and gases within the reactor re-equilibrate. If specified, kinetically controlled reactants are added to the reactor prior to each equilibration.

This covers the case where the residence time within the reactor is sufficiently short that equilibrium cannot be reached with slowly degrading constituents – e.g., metal degradation.

PHREEQC can be used to mimic the EQ6 flow-through mode. A “cell” in PHREEQC can represent the reactor but cells in PHREEQC are generally fully flushed with solution at each equilibration step in batch reactor and transport problems. To simulate partial displacements of water in a cell using PHREEQC, a set of mixing reactions must be defined.

In the CSNF base case, the base case flow rate is 1 L/yr for the entire waste form cell (Cell 1). Because the base case volume of water in this cell is approximately 2,370 liters, the mean residence time of water flowing through the entire cell is 2,370 years and the turnover rate or flush rate is $(1 \text{ L/yr})/(2,370 \text{ L})$ or 0.000422 yr^{-1} .

EQ6 can only simulate the IPC flow-through model when the volume of water in the flow-through cell is one liter. Therefore, the CSNF cell was scaled down by a factor of 2,370, i.e., equivalent to the base case liters of water in the CSNF cell. Consequently, the liquid influx rate was set at $1.34\text{E-}11 \text{ L/s}$ to simulate a turnover rate of 0.000422 yr^{-1} .

For time steps of 0.2 years, the amount of water displaced before each equilibration in the flow-through cell is 0.0000844 L (84.4 μL), or 0.00844%. To accomplish this in PHREEQC, source water from one cell (Cell 4 in this example) is mixed with water from the reactor cell (Cell 1) in a ratio of 0.0000844:0.9999156 to produce the new combined liter in the reactor cell. This mixing is specified using the *MIX* subroutine in PHREEQC as follows:

```
MIX 1
    1    0.9999156
    4    0.0000844
END
```

The mixing approach is straightforward for simulating displacement of small portions of water in the reactor. However, it is impractical to use by itself for a system in which hundreds or thousands of incremental displacements are simulated. The solution is to use PHREEQC’s *TRANSPORT* subroutine in conjunction with the *MIX* subroutine to execute the repeated fractional displacements and provide user-friendly, tabulated output. This application of PHREEQC mimics EQ6’s solid-center flow-through mode.

Two transport cells had to be defined so that “no flow” boundaries could be defined for the first and last transport cell. Specifying “no flow” boundaries prevents displacement of the full water volume of each cell at each equilibration step and allows implementation of the “diffusion only” mode for transport between the transport cells. In this case, Cells 1 and 2 were defined as the first and final cells. In addition, stagnant cells were defined for each transport cell; Cell 4 for transport Cell 1 and Cell 5 for transport Cell 2. These stagnant cells were defined to represent the source water for the displacement via mixing and were prevented from exchanging with their associated transport cells except via the *MIX* option. Thus, Cells 4 and 5 are identical and contain the source water (e.g., simulated J-13 well water), and Cells 1 and 2 are identical and contain the degrading waste package structural materials, CSNF, and the evolving aqueous solution.

For the first 25 time intervals (“shifts”), the mixing and transport sections of the PHREEQC input files follow:

```
MIX 1
  1    0.9999156
  4    0.0000844
END
MIX 2
  2    0.9999156
  5    0.0000844
END
TRANSPORT
  cells                2
  shifts               25      # 5 years
  time_step           6311500 # seconds = 0.2 years
  flow_direction      diffusion_only
  boundary_conditions  closed closed
  stagnant            1  0  0  0
  print_cells         1
  print_frequency     40
  punch_cells         1
  punch_frequency     1
  warnings            true
END
```

Mixing between Cells 1 and 4 and between Cells 2 and 5 is identical. As a result, diffusion between transport Cells 1 and 2 has no effect. The evolving compositions in Cell 1 can be considered the evolving aqueous and mineral compositions in the waste package reactor. Further information on the inputs specified above can be found in the user’s manual (Parkhurst and Appelo 1999 [DIRS 159511]).

Overall, the PHREEQC simulations perform the same calculations as the EQ6 simulations. However, there are minor differences. One is the step frequency. Instead of EQ6’s dynamically changing time intervals, the time intervals in the PHREEQC simulations are constant. There is also a minor difference in how potential minerals are specified in the input files. In EQ6, all minerals in the database are allowed to precipitate if supersaturated with the exception of those minerals specified as “suppressed” in the input file. In PHREEQC, the convention is opposite: all minerals in the database are suppressed except those specified for equilibration in the input file. Including a potential mineral in the PHREEQC input file with a starting value of 0 moles has no effect on the calculations unless the mineral reaches saturation, whereupon it will precipitate. The minerals allowed to precipitate in the PHREEQC simulations are the minerals that formed in the EQ6 simulation. Minerals that did not form in the EQ6 simulation do not become supersaturated in the PHREEQC simulation, except for anatase (TiO₂) which is prevented from precipitating in the PHREEQC calculations for reasons discussed below.

To optimize the PHREEQC simulation such that high accuracy was maintained throughout the run while avoiding the generation of copious amounts of output, the time steps and print-to-file frequency were varied as a function of time in the PHREEQC input file (*CSNF_comp_120k.pqi*). For the first 5 years of the simulation, the time step was 0.2 years and the print frequency (*PUNCH FREQUENCY* in the input file) was 1, i.e., the output was collected for every time step. For the period from 5 to 50 years the time step was also 0.2 years but the print frequency was

increased to 10. From 50 to 450 years the time step was increased to 20 years and the print frequency was set at 1, and from 450 to 10,050 years the time step was held to 20 years and the print frequency was set at 10. Finally, from 10,050 years to 120,000 years the time step was maintained at 20 years and the print frequency was increased to 200. This prevented the printing of large amounts of slowly changing estimated concentrations.

In order to manipulate the EQ6 inputs to mimic the PHREEQC time sequence, it was necessary to develop four separate input files, each representing the same time interval as those for PHREEQC. The beginning EQ6 run (*dprpSD90_5.6i*) contained the scaled starting masses of the Cell 1 waste package components and each of the subsequent EQ6 input files were generated from the pickup file of the previous run. Thus, the time-depleted waste package material inventories were maintained from input file to input file. Using the constant time stepping in EQ6 caused the simulation to fail after 5,370 years, so no EQ6 input files were generated for the 10,050- to 120,000-year period. Rather, the relevant EQ6 output from Output DTN: SN0702PAIPC1CA.001 was extracted from the CSNF base case simulation for graphical comparison (Section 7.10.1.2[a]).

Additional modifications to the EQ6 CSNF base case simulation included the suppression of the mineral phase anatase (TiO_2). Anatase is a highly insoluble form of TiO_2 , and in the EQ6 simulation its precipitation drops the aqueous titanium to values less than 1×10^{-46} molal. In PHREEQC, the low calculated aqueous concentrations of titanium create numerical instability in the code, and as a result computations do not converge when anatase is among the potential phases allowed to precipitate. Thus, a more soluble phase of TiO_2 , rutile, in the PHREEQC and EQ6 simulations was allowed to control the aqueous titanium concentration. The presence of rutile intergrown with quartz (i.e., rutilated quartz) from Minas Gerais, Brazil, indicates that it forms from relatively low temperature aqueous solutions (Palache et al. 1944 [DIRS 163604]; Deer et al. 1966 [DIRS 102773]).

In the case of Zn_2TiO_4 , PHREEQC would not converge when this phase was among the allowed phases, probably because the aqueous Ti concentrations were already very low due to rutile formation. Thus, when Zn_2TiO_4 was suppressed in the EQ6 simulation a new phase, Zn_2SiO_4 , formed in its place. This phase was then added to the PHREEQC input file. With Zn_2SiO_4 on the list of allowed minerals in the PHREEQC input file, the code executed with no difficulty. The impact on the bulk chemistry related to the substitution of these phases was negligible because Zn, Ti, and Si were only present in very low concentrations.

The source database for the PHREEQC simulation was *phreeqcDATA0R525.dat* from DTN: MO0704PHREEQ25.000 [DIRS 181214]. This database is a translation of the 25°C thermodynamic data from the EQ3/6 *data0.ymp.R5* database (DTN: SN0612T0502404.014 [DIRS 178850]). For the PHREEQC simulation, the *phreeqcDATA0R525.dat* database was altered to be consistent with the *data0.r5f* database, developed for the EQ6 simulations (Section 6.3.1.1[a]). In the altered database, named *phreeqcDATA0R525r_Cr.dat*, all of the non-Cr(III) species and phases were inactivated using comment flags to avoid Cr^0 oxidation to Cr(II), Cr(V), and Cr(VI) phases and species. The altered *phreeqcDATA0R525r_Cr.dat* database is included in Validation DTN: SN0705IPCEQPHR.001.

The ion activity model used in each simulation follows the B-dot expression developed by Helgeson (Wolery 1992 [DIRS 100835]). PHREEQC uses the extended Debye-Hückel expression, which is identical to the B-dot expression. In the PHREEQC database, *phreeqcDATA0R525r_Cr.dat*, the B-dot parameters are identical to those in the EQ6 database and are included for each species.

The PHREEQC and EQ6 input and output files for this simulation are documented in Validation DTN: SN0705IPCEQPHR.001. The file names and descriptions are:

- PHREEQC input file: *CSNF_comp_120k*
- PHREEQC output file: *CSNF_comp_120k.out*
- Tabulated output, 0 to 120,050 years: *CSNF_comp1L-yr.xls*
- EQ6 input/output file 0 to 5 years, *dprpSD90_5.6i* and *.6o
- EQ6 input/output file 5 to 50 years, *dprpSD95_50.6i* and *.6o
- EQ6 input/output file 50 to 450 years, *dprpSD950_450.6i* and *.6o
- EQ6 input/output file 450 to 10,000 years, *dprpSD9450_10k.6i* and *.6o.

7.10.1.2[a] Comparison of PHREEQC and EQ6 Estimates

The validation activities specified in *Technical Work Plan for Waste Form Testing and Modeling* (BSC 2006 [DIRS 177389]) were amended. The amended activities included corroboration of model estimates with other model results obtained from the implementation of other independent mathematical models developed for similar or comparable intended use/purposes.

To evaluate corroboration of the PHREEQC and EQ6 results, spreadsheet *EQ6_PHRQC_Val.xls* (Validation DTN: SN0705IPCEQPHR.001) was generated for graphical comparison of the respective output. Relevant graphs from this spreadsheet are reproduced and discussed in this section.

For plotting purposes, time conversion from log years in the EQ6 output worksheets (“EQ6 Output” and “EQ6 IPC TDIP DTN”) to years was performed. In the PHREEQC output worksheet (“PHRQC Output”), time in seconds was converted to years.

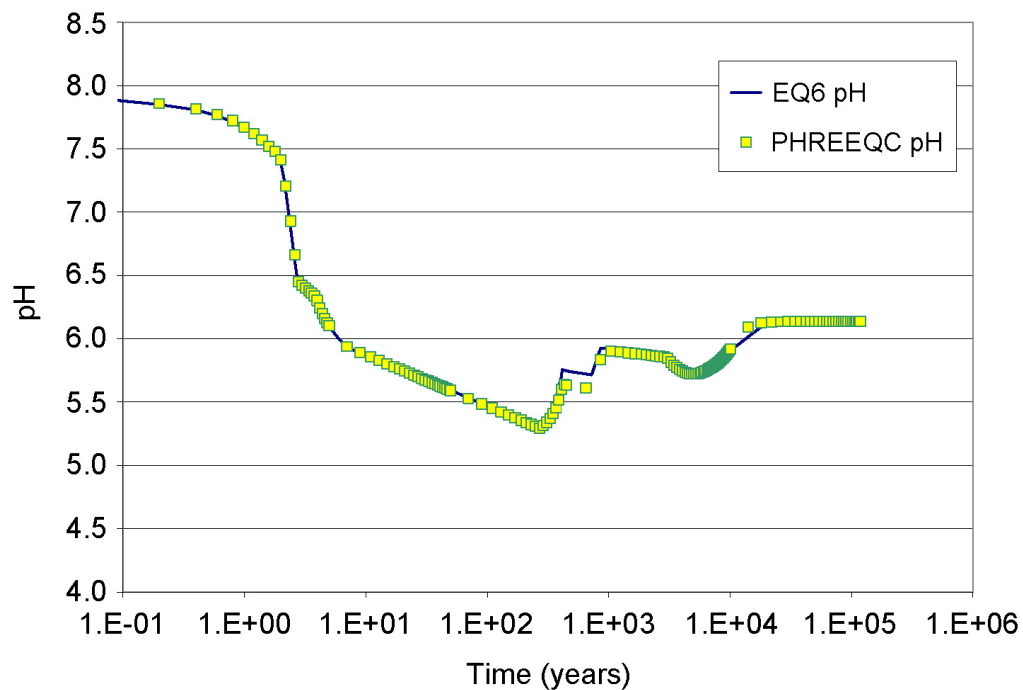
Three parameters were plotted: pH, ionic strength, and fluoride concentration as a function of time. These parameters were chosen because they are the parameters for which abstractions are generated for TSPA-LA.

Figure 7-1[a], compares the pH trajectories of the PHREEQC and EQ6 simulations. Qualitatively, the curves are essentially identical, easily satisfying the validation criterion of agreement within two orders of magnitude.

Ionic strength evolution as a function of time is plotted in Figure 7-2[a]. There is some difference between the codes above ionic strength values of about 0.37 molal. These discrepancies are minor and may be linked to the way in which the codes differ in their approach to mass balancing water in the system, or a combination of water mass balance and the different time stepping employed by the codes. Regardless of the slight variations, both codes produce similar results and virtually identical trends in ionic strength.

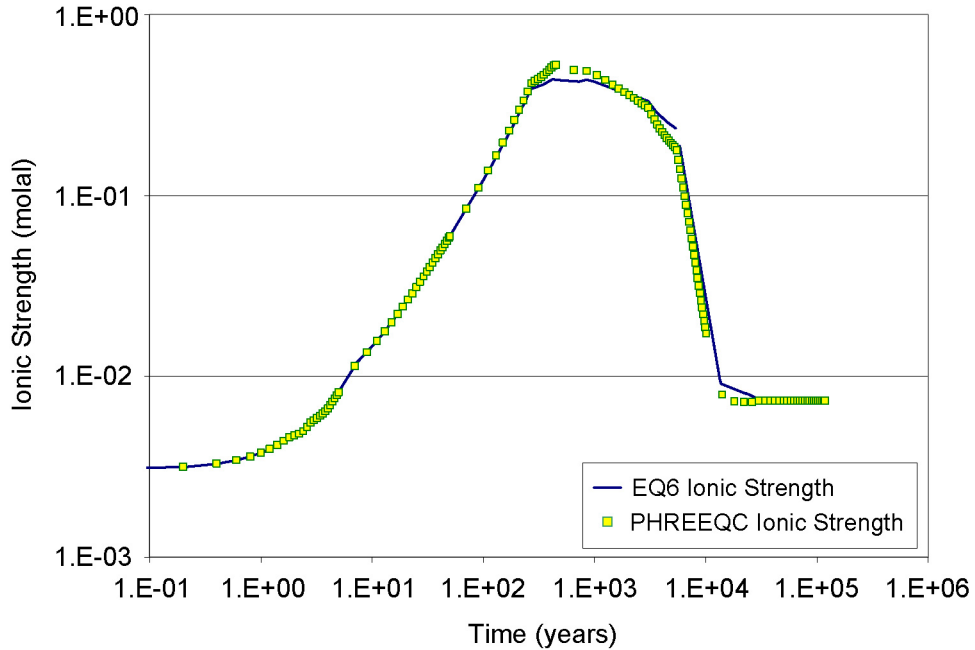
The predicted fluoride concentrations as a function of time are plotted in Figure 7-3[a]. The traces are identical until the region where the ionic strength shows it greatest variability, between about 450 to 3,000 years. Like ionic strength, the differences in the fluoride curves are minor and likely due to water mass balancing and time step variation.

In summary, although minor differences are observed between the EQ6 and PHREEQC simulations, the differences are small compared to model uncertainties and compared to the ranges of concentrations predicted for each of these components over the time period modeled. Therefore, the comparison of PHREEQC and EQ6 model simulations lends confidence to the Cell 1 IPC model.



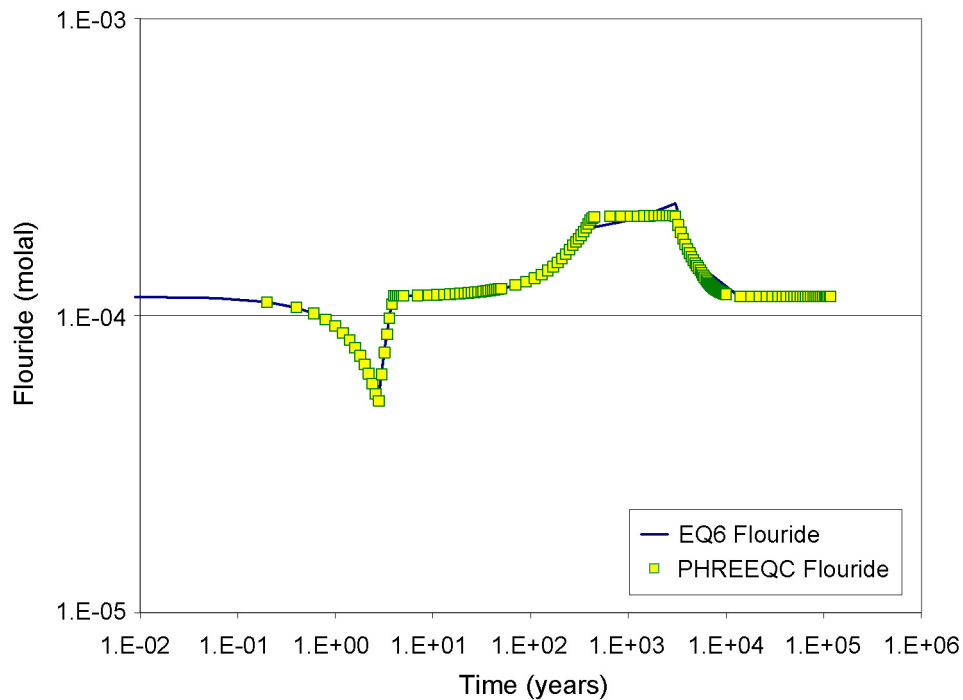
Source: Validation DTN: SN0705IPCEQPHR.001, file: *EQ6_PHRQC_Val.xls*.

Figure 7-1[a]. Comparison of Predicted pH in PHREEQC and EQ6 Simulations of the CSNF Base Case



Source: Validation DTN: SN0705IPCEQPHR.001, file: *EQ6_PHRQC_Val.xls*.

Figure 7-2[a]. Comparison of Predicted Ionic Strength in PHREEQC and EQ6 Simulations of the CSNF Base Case



Source: Validation DTN: SN0705IPCEQPHR.001, file: *EQ6_PHRQC_Val.xls*.

Figure 7-3[a]. Comparison of Predicted Fluoride Concentrations in PHREEQC and EQ6 Simulations of the CSNF Base Case

7.10.2[a] Critical Review

A critical review of the waste form IPC model and abstractions was performed by Laura Price in accordance with Attachment 4 of SCI-PRO-006. The group manager for the Near-Field Environment (NFE) team (Geoff Freeze) directed the critical review per Section 6.3.2 of SCI-PRO-006, as part of the NFE group manager's determination regarding the adequacy of model validation. The critical reviewer selected by the NFE group manager is independent of the development, checking, and review of the documentation for the IPC model, including documents providing inputs to the model, and meets the defined criteria for selection within Appendix B of the TWP (BSC 2006 [DIRS 177389]).

Laura Price has a B.S. and a M.S. in chemical engineering, and has worked for 19 years in the field of radioactive waste disposal performance assessment. She has worked on radionuclide source term models for other radioactive waste disposal sites, and is familiar with the many interrelated models that are needed to estimate the release of radionuclides from the proposed Yucca Mountain repository. She has many years experience in performance assessment, so she is familiar with performance assessment concepts, such as the appropriate use of models for the long time scales and the highly uncertain conditions that are inherent in performance assessment calculations.

The intended use of the IPC model is characterized in the TWP (BSC 2006 [DIRS 177389], Table 2-2) as:

Predict the bulk chemistry inside a failed waste package and provide simplified expressions of that chemistry for use in TSPA-LA.

The overall criterion for the critical reviewer to apply to all evaluations of the IPC model is given in Table 7-1[a]:

The independent technical review will assure that the modeling is done using best scientific practices and in accordance with all applicable procedures.

The critical reviewer found that the waste form IPC process models and abstractions are adequate for their intended purpose. The complete review is attached in Appendix B[a].

7.11[a] SUMMARY

To summarize, the IPC model is a numerical simulation of water reactions with acid-producing solids (stainless steel components), alkali-producing solids (glass), and both commercial and defense spent nuclear fuel. The outputs are ionic strength trajectories and pH distributions. Documenting that these estimates are within the bounds of pH values and ionic strengths observed in natural or experimental situations, as done in the peer-reviewed literature, where similar processes prevail, is critical to model validation of the IPC model.

Achieving a sufficient level of confidence required documenting:

1. Fuel and steel degradation models
2. Formation of corrosion products
3. Production of low-pH waters by steel degradation
4. Production of high-pH waters by glass degradation
5. Production of ionic strength extremes.

This model requires Level II validation (BSC 2006 [DIRS 177389]). In addition to satisfying Level I criteria (a) through (f), the evidence above collectively provides broad justification for assignment of a Level II validation for the IPC model. The justification of the validation methods used is provided in Section 7.3[a]. The validation of the IPC model meets the requirements for implementation in TSPA-LA.

8[a]. CONCLUSIONS

Section 8.1[a] provides a summary of the modeling activity. The abstractions are summarized in Section 8.2[a]. YMRP criteria are addressed in Section 8.3[a].

8.1[a] SUMMARY OF MODELING ACTIVITY

The in-package chemistry (IPC) model is developed and validated in this addendum to predict pH, ionic strength, and fluoride concentration within waste form cells after waste package breach. The conceptual model in this addendum redefines the control volume as a waste form cell instead of a waste package cell. This new definition is consistent with the waste form cells defined and used in TSPA-LA. The waste form cell in the CSNF waste package consists of the fuel, the fuel assemblies, the fuel basket tubes, and the fuel basket including the neutron absorber and thermal shunt plates. There are two CDSP waste form cells. They consist of the HLW glass logs and their canisters (Cell 1a) and DSNF and their canisters (Cell 1b). Cell 2, which is simulated in *EBS Radionuclide Transport Abstraction* (SNL 2007 [DIRS 177407]), consists of the waste package materials outside of the waste form cells.

The changes in this addendum include:

- New values for various inputs
- Modifications in response to condition reports
- Model validation enhancements
- Improvements in abstraction calculation methods and functionality
- Improvements in the integration of abstractions with TSPA-LA.

A detailed list of changes is provided in Table 1-1[a].

The IPC model outputs depend on a number of input parameters. The most important parameters were determined from IPC process model simulations in which input values were varied over their potential ranges. The parameters with the largest effects were incorporated as independent variables in the abstractions. For ionic strength, the independent parameters are the liquid influx rate, time since waste package breach, and relative humidity (Section 6.10.2[a]). For pH, they are the partial pressure of carbon dioxide and ionic strength (Section 6.10.1[a]). Ionic strength is also an independent parameter for the fluoride concentration abstraction (Section 6.10.3[a]). Parameters with smaller effects on the abstractions were used to help quantify uncertainty. For example, material degradation rates and liquid influx compositions were used to quantify ionic strength uncertainty in the ionic strength abstractions (Section 6.10.8.2[a]).

Abstractions for pH, ionic strength, and fluoride concentration were developed for each waste form cell for use in TSPA-LA calculations. Although the CDSP cells simulated in this addendum are based on the 2MCO/2DHLW waste package, the abstractions can be applied to the waste form cells in other CDSP waste packages such as the 5DHLW/1DSNF waste package (Section 6.6.6[a]).

Other IPC model abstractions are unchanged from the parent document. The total carbonate abstraction is unchanged because it is independent of the IPC conceptual model. The Eh and chloride abstractions are not re-examined because they are no longer used in TSPA-LA.

The ranges of applicability of the pH, ionic strength, and fluoride concentration abstractions are shown in Table 8-1[a]. For the partial pressure of oxygen, simulations in this addendum are performed at $10^{-0.7}$ bar, which is consistent with the values expected in the drift over time and with the abstractions from *Dissolved Concentration Limits of Elements with Radioactive Isotopes* (SNL 2007 [DIRS 177418]). For the range of carbon dioxide partial pressure, pH abstractions are based on simulations performed at 10^{-4} , 10^{-3} , and 10^{-2} bar. Results are extrapolated to $10^{-1.5}$ bar based on the trends observed so that the abstractions can apply to a range of 10^{-4} to $10^{-1.5}$ bar. Sections 6.10.1.1[a] and 6.10.1.2[a] provide justification for the extrapolation. Should $p\text{CO}_2$ fall outside this range, the nearest value of $p\text{CO}_2$ (either 10^{-4} or $10^{-1.5}$ bar) is used in the abstraction.

As justified in Section 6.6[a], the temperature of all IPC abstractions in this addendum is 25°C. However, the 25°C abstractions can be used for temperatures from 25 to 100°C. This is consistent with the radionuclide solubility abstractions used in TSPA-LA developed in *Dissolved Concentration Limits of Elements with Radioactive Isotopes* (SNL 2007 [DIRS 177418], Section 6.3.3.3). That report justifies the use of 25°C simulations at higher temperatures because radionuclide solubility generally decreases with increasing temperature (thus, it is conservative to use the solubilities calculated at lower temperature).

The range of RH for the vapor influx abstractions is 95 to 100%. For liquid influx abstractions, the liquid influx range is ≥ 0.1 L/yr. When the liquid influx range is 0.1 L/yr or above and the RH is 95% or above, the selection of which abstraction to use is determined as indicated in Section 6.10.9.1[a]. At liquid influx rates below 0.1 L/yr, the vapor influx model is used.

Table 8-1[a]. Range of IPC Abstraction Applicability

| Parameter | Value | Units |
|--|----------------------------|-------|
| Partial Pressure of Oxygen | $10^{-0.7}$ | bar |
| Partial Pressure of Carbon Dioxide | $10^{-4.0}$ to $10^{-1.5}$ | bar |
| Temperature | 25 to 100 | °C |
| RH for Vapor Influx Model | 95 to 100 | % |
| Liquid Influx Rate for Liquid Influx Model | ≥ 0.1 | L/yr |

8.2[a] MODEL ABSTRACTIONS

The model abstractions are developed and presented in detail in Section 6.10[a]. They are documented in Output DTN: SN0702PAIPC1CA.001 along with all process model calculations and simulations. The TSPA-LA parameters defined for the IPC abstractions are identified in Table 6-21[a]. This table includes values for the parameters or references to the files containing the associated lookup tables in the DTN. Details of TSPA-LA implementation of these abstractions are provided in Section 6.10.9[a]. The non-TSPA-LA parameters for the fluoride abstraction are identified and quantified (or referenced) in Table 6-22[a].

The subsections that follow summarize the form and function of the waste form IPC abstractions for TSPA-LA.

8.2.1[a] pH Abstractions

The pH abstractions define the pH ranges of each waste form cell. These ranges are defined by phases that provide substantial acid and base buffering capacities (Section 6.3.4[a]) and are quantified by simulated acid and base titrations (Section 6.10.1[a]). The titrations show that the major acid neutralizing capacity in each cell is provided by the Ni/Fe oxides that accumulate as steel degrades. The base neutralizing capacity depends on the cell. In the CSNF and DSNF cells the base neutralizing capacity is due to schoepite that precipitates from the degradation of the fuel. For the HLW glass cells, where schoepite does not precipitate, the base neutralizing capacity is due primarily to the dissolution of carbon dioxide from the gas phase.

The minimum and maximum pH values in the abstractions are defined in lookup tables that are a function of $p\text{CO}_2$ and ionic strength (Section 6.10.1[a]). The $p\text{CO}_2$ is defined in this addendum as the negative log of the carbon dioxide fugacity (much like pH is the negative log of the hydrogen ion activity). Although the minimum pH does not vary with $p\text{CO}_2$, the maximum pH increases as $p\text{CO}_2$ increases. The functional relationship of the pH range with ionic strength is more complicated, but it is easily determined from the titrations.

The pH distribution for TSPA-LA is defined to be uniform between the minimum and maximum pH values. A uniform distribution is chosen because sensitivity analyses indicate no clear central tendency for pH. Because of the large buffer capacities quantified at the abstracted minimum and maximum pH values (Section 6.10.1[a]), the uncertainties in the pH limits are negligible and do not require quantification. Time-dependent base case simulations and process model sensitivity analyses confirm that the established pH ranges are reasonable because minimum and maximum pH limits are never exceeded (e.g., Sections 6.5[a] and 6.6[a]).

8.2.2[a] Ionic Strength Abstractions

The ionic strength abstractions define the mode and deviation of ionic strength for each waste form cell. For the liquid influx case, these abstractions are represented by lookup tables that are a function of the liquid influx rate and time since waste package breach (Section 6.10.2.1[a]). For the vapor influx case, they are represented by equations that are a function of relative humidity (Section 6.10.2.2[a]).

The liquid influx rate is important because at high rates, ionic strength is predicted to be much closer to the ionic strength of the incoming water (Section 6.10.2.1[a]). At low liquid influx rates, degradation reactions can substantially increase the ionic strength by releasing or generating soluble ions such as Ni^{2+} , Mg^{2+} , and NO_3^- . The effect of time is included in the liquid influx abstractions because ionic strength decreases after materials are fully degraded and the ions produced from degradation reactions are mostly flushed from the cells. At very low liquid influx rates, vapor influx dominates. Under vapor influx conditions, the activity of water is controlled by relative humidity. Ionic strength is strongly related to the activity of water; thus, the ionic strength abstraction under vapor influx conditions is a function of relative humidity (Section 6.10.2.2[a]).

The ionic strength distributions are defined as triangular. This type of distribution is consistent with the central tendencies observed in sensitivity analyses. Uncertainty in the ionic strength abstraction depends on the dominant form of influx: liquid or vapor. Under liquid influx conditions, degradation rates have the largest effect on predicted ionic strength among the parameters not included directly in the ionic strength abstractions (Section 6.10.8.2[a]). Therefore, uncertainty is approximated based on the variation in ionic strength predictions in the degradation rate sensitivity analyses (Section 6.6.5[a]). For vapor influx conditions, ionic strength is a function of relative humidity. Uncertainty in the vapor influx ionic strength abstraction is based on ionic strength variations observed in cited Pitzer calculations for 16 different salt solutions (Section 6.10.8.2[a]).

8.2.3[a] Fluoride Abstractions

The fluoride abstractions are used by *Dissolved Concentration Limits of Elements with Radioactive Isotopes* (SNL 2007 [DIRS 177418]) in assessing uncertainties in radionuclide solubility. For the purposes of that report, distributions of fluoride concentration are defined to be right triangular such that the minimum and apex are zero. The right triangular distributions are retained in the waste form IPC abstractions but the maximum fluoride concentrations are allowed to increase at high ionic strength in the CSNF and 2MCO cells as described in Section 6.10.3[a]. The fluoride abstractions are based on pore water samples collected at Yucca Mountain and on IPC process model simulations. These fluoride distributions are only for liquid influx conditions involving failed drip shields because there are no sources of fluoride in any of the waste packages or waste package cells.

8.3[a] YUCCA MOUNTAIN REVIEW PLAN CRITERIA

The following acceptance criteria are from Section 2.2.1.3.3 of *Yucca Mountain Review Plan, Final Report* (NRC 2003 [DIRS 163274]) and are based on the requirements of 10 CFR 63.114 (a)-(c) and (e)-(g) [DIRS 173164]. The responses below pertain to the addendum.

Acceptance Criterion 1 – System Description and Model Integration Are Adequate

(1) TSPA adequately incorporates important design features, physical phenomena and couplings and uses consistent and appropriate assumptions throughout the quantity and chemistry of water contacting waste packages and waste forms abstraction process.

The IPC model in this addendum is exclusively developed for the waste form cells. The waste form IPC model reflects the design features of the waste package and waste forms (Sections 4.1.3[a] and 4.1.4[a]). The physical phenomena and couplings important to the model are identified and incorporated into the model (Section 6.3[a]). Assumptions used in the model are appropriate and consistent with assumptions used in the TSPA-LA model (Sections 5[a] and 6.3[a]). The quantity of water in contact with the waste forms has no effect on vapor influx in-package chemistry abstractions because relative humidity, alteration products, and the fugacity of carbon dioxide control model outputs (Sections 6.10.2.2[a] and 6.10.1.2[a]). For liquid influx, the quantity of water has a small but negligible effect because steady state conditions will predominate (Section 6.6.1[a]), but the liquid influx rate has important effects

that are captured in the ionic strength abstractions (Section 6.10.2.1[a]). The chemistry of water contacting the waste form is consistent with (a) the predominant water types identified in the Topopah Spring welded tuff (SNL 2007 [DIRS 177412], Section 6.6.5) and (b) water samples from basalt aquifers in the igneous scenario (Section 4.1.2[a]). The chemistry of liquid influx is shown to have important effects only for high liquid influx rates (Section 6.6.4[a]). These effects are incorporated in the waste form IPC ionic strength abstractions in the form of uncertainty (Section 6.10.8.2[a]).

(2) The abstraction of the quantity and chemistry of water contacting the waste packages and waste forms uses assumptions, technical bases, data and models that are appropriate and consistent with other related DOE abstractions.

As explained above, the quantity of water in contact with the waste package has little effect on the in-package chemistry abstractions, but the liquid influx rate has important effects that are captured in model abstractions. The chemistry of water contacting the waste forms from the in-package chemistry abstractions depends on other related DOE abstractions to determine the values of the independent variables, such as relative humidity, partial pressure of carbon dioxide, and liquid influx rate. The TSPA-LA model uses the values of these parameters to determine the appropriate water composition outputs from the in-package chemistry abstractions. This coupling enforces consistency between the in-package chemistry abstractions and other related DOE abstractions.

(3) Important design features, such as waste package design and material selection, backfill, drip shield, ground support, thermal loading strategy, and degradation processes are adequate to determine the initial and boundary conditions for calculations of the quantity and chemistry of water contacting waste packages and waste forms.

Throughout this model, information is presented related to the design of the waste package and waste form (Sections 4.1.3[a] and 4.1.4[a]), degradation processes (Sections 4.1.4[a] and 6.3.1.1[a]), physical phenomena (Section 6.3.1.1[a]), and couplings that may affect the degradation of the engineered barriers (Section 6.3.1.1[a]). These descriptions are consistent with the information delineated in other reports and are adequate for their intended purposes (i.e., to enable the modelers to develop and implement appropriate models and to enable the reviewer to understand the bases for the analytical activities performed by the applicant and those results).

(4) Spatial and temporal abstractions appropriately address physical couplings (thermal-hydrologic-mechanical-chemical).

The spatial abstraction for the batch reactor model is described in Section 6.3.1.1[a] and appropriately addresses physical couplings. The process model simulations use degradation rates and liquid influx rates to determine the effects of coupled processes over time. The abstractions incorporate temporal effects by requiring the calculated values of temporal input parameters to determine the values of the output parameters over time. The independent temporal input parameters incorporated in the in-package chemistry abstractions are relative humidity, liquid influx rate, and the partial pressure of carbon dioxide.

(5) Sufficient technical bases and justification are provided for TSPA assumptions and approximations for modeling coupled T-H-M-C effects on seepage and flow, the waste package chemical environment, and the chemical environment for radionuclide release. The effects of the distribution of flow on the amount of water contacting the waste package and waste form are consistently addressed in all relevant abstractions.

Sufficient technical bases and justification for assumptions and approximations are provided in Sections 5[a] and 6.3[a]. As explained above, the amount of water in contact with the waste package and waste forms has little effect on the in-package chemistry abstractions, but the liquid influx rate has important effects that are captured in model abstractions.

(6) The expected ranges of environmental conditions within the waste package emplacement drifts, inside of breached waste packages, and contacting the waste forms and their evolution with time are identified. These ranges may be developed to include: (i) the effects of drip shield and backfill on the quantity and chemistry of water; (ii) conditions that promote corrosion of engineered barriers and degradation of waste forms; (iii) irregular wet and dry cycles; (iv) gamma-radiolysis; and (v) size and distribution of penetrations of waste packages.

Relative humidity and liquid influx rates are the environmental variables incorporated into the waste form in-package chemistry ionic strength abstractions. These abstractions encompass the expected ranges of these variables (Section 6.10.2[a]). The partial pressure of carbon dioxide is incorporated into the pH abstractions. The expected range of this variable is captured in the pH abstractions (Section 6.10.1[a]). Other environmental variables have limited effects on the pH and ionic strength abstractions due to limited ranges or limited importance over their potential ranges. The effect of gamma radiolysis on water composition is evaluated in Appendix B of the parent report.

(7) The model abstraction for quantity and chemistry of water contacting the waste packages and waste forms is consistent with the detailed design information on waste package design and other engineered features.

This model uses the design of the waste package and the waste package materials as input, as described in Section 4.1.4[a], and the input files for the model are based on this design information.

(8) Adequate technical bases are provided, including activities such as independent modeling, laboratory or field data, or sensitivity studies, for inclusion of any T-H-M-C couplings and features, events and processes.

This model is validated as described in Section 7[a]. Degradation rates used in model simulations are based on laboratory and field data (Sections 7.4.2[a]). Degradation products predicted in the model are generally consistent with mineral phases observed in natural analogues (Sections 7.4.3[a] and 7.4.4[a]). Numerous sensitivity analyses are performed to evaluate the effects of the expected ranges of various parameters on model outputs

(Section 6.6[a]). In addition, results from a different geochemical code corroborate model outputs (Section 7.10.1[a]).

(9) Performance affecting processes that have been observed in thermal-hydrologic tests are included in the TSPA.

The processes included in the IPC model in this addendum are discussed in Sections 6.3[a] and 6.3.1[a]. They include processes observed in thermal-hydrologic tests, such as exchange of CO₂, O₂, and H₂O between the aqueous and gas phases, advection of seepage and condensed water, and precipitation and dissolution of secondary mineral phases.

(10) Likely modes for container corrosion are identified and considered in determining the quantity and chemistry of water entering the waste packages and contacting the waste forms.

The waste form IPC model does not identify and consider the likely modes of container corrosion that influence the quantity and chemistry of water entering the waste package. It only simulates corrosion of materials in the waste form cells. Corrosion is simulated as general corrosion because once water has entered the cell, the mode of corrosion, whether pitting or general, has little consequence. Instead, it is the rate of corrosion in the cell that can affect in-package chemistry abstractions (Section 6.6.5[a]).

(11) The abstraction of the in-package criticality or external-to-package criticality within the emplacement drifts provides an adequate technical basis for screening these events. If either event is included in the TSPA, then the DOE uses acceptable technical bases for selecting the design criteria that mitigate the potential impact of in-package criticality on the repository performance, identifies the FEPs that may increase the reactivity of the system inside the waste package, identifies the configuration classes and configurations that have the potential for nuclear criticality, and includes changes in thermal conditions and degradation of engineered barriers in the abstraction of the quantity and chemistry of water contacting waste packages and waste forms.

The subject of potential criticality is discussed in other reports. This criterion is considered not to be applicable to this report.

(12) Guidance in NUREG-1297 and NUREG-1298 . . . or other acceptable approaches is followed.

The information contained in this report is based on established physical models, justified assumptions, and qualified input data. As a result, there was no need to use the guidance of NUREG-1297 and/or NUREG-1298 in the development of this report.

Acceptance Criterion 2 – Data Are Sufficient for Model Justification

(1) Geological, hydrological and geochemical values used in the License Application are adequately justified. Adequate description of how the data were used, interpreted and appropriately synthesized into the parameters is provided.

The sources for the geological, hydrological, and geochemical data are justified in Section 4.1[a]. Section 6.3.1.3[a] describes how these data were used and converted into values appropriate for the model input files. The chemical thermodynamic database was developed specifically for the YMP (Section 4.1.1[a]).

(2) Sufficient data were collected on the characteristics of the natural system and engineered materials to establish initial and boundary conditions for conceptual models of T-H-M-C coupled processes that affect seepage and flow and the waste package environment.

The sources for the geological, hydrological, and geochemical data (Section 4.1[a]) come from a broad body of data collected on the characteristics of the natural system and engineered materials. Section 6.3.1.3[a] describes how the source data were used to establish initial and boundary conditions.

(3) Thermo-hydrologic tests were designed and conducted with the explicit objective of observing T-H processes for the temperature ranges anticipated in the repository and making measurements for mathematical models. Data are sufficient to verify that T-H conceptual models address important T-H phenomena.

The waste form IPC model does not attempt to predict thermo-hydrologic processes for TSPA-LA. Instead, it uses the ranges of parameters predicted by other models affected by thermo-hydrologic processes to produce abstractions that are a function of these parameters so that each TSPA-LA model realization can determine the appropriate outputs for predicted thermo-hydrologic conditions from the in-package chemistry abstractions. The thermo-hydrologic parameters important to the in-package chemistry abstractions include relative humidity, liquid influx rates, and the partial pressure of carbon dioxide (Section 6.10[a]).

(4) Sufficient information to formulate the conceptual approaches for analyzing water contact with the drip shield, waste packages and waste forms is provided.

The conceptual model for water contacting the waste form cells is provided in Sections 6.3[a] and 6.3.1[a]. The IPC model does not extend to the outer waste package and drip shield.

(5) Sufficient data are provided to complete a nutrient and energy inventory calculation, if it has been used to justify the inclusion of the potential for microbial activity affecting the waste package chemical environment and chemical environment for radionuclide release. As necessary, data are adequate to support determination of the probability for microbially induced corrosion and microbial effects, such as the production of organic by-products and microbially enhanced dissolution of the high-level radioactive waste glass form.

Microbially influenced corrosion is not addressed in this addendum.

Acceptance Criterion 3 - Data Uncertainty Is Characterized and Propagated through the Model Abstraction

Sensitivity analyses were performed to evaluate the effects of uncertain input parameters (Section 6.6[a]). Results of these analyses were used to approximate model uncertainty (Section 6.10.8[a]).

(1) Models use parameter values, assumed ranges, probability distributions and/or bounding assumptions that are technically defensible, reasonably account for uncertainties and variabilities, and do not result in under-representation of the risk estimate.

The model developed in this addendum uses parameter values, assumed ranges, probability distributions or bounding assumptions, or both, that are technically defensible, reasonably account for uncertainties and variabilities, and do not result in under-representation of the risk estimate (Sections 4.1[a], 5[a], 6.3[a], 6.6[a], and 6.10[a]).

(2) Parameter values, assumed ranges, probability distributions and bounding assumptions used in the TSPA calculations of quantity and chemistry of water contacting waste packages and waste forms are technically defensible and reasonable based on data from the Yucca Mountain region (e.g., results from large block and drift-scale heater and niche tests) and a combination of techniques that may include laboratory experiments, field measurements, natural analog research and process modeling studies.

The geological, hydrological, and geochemical data (Section 4.1[a]) come from a broad body of data collected on the characteristics of the natural system and engineered materials. These data are technically defensible and reasonable.

(3) Input values used in the TSPA calculations of quantity and chemistry of water contacting engineered barriers (e.g., drip shield and waste package) are consistent with the initial and boundary conditions and the assumptions of the conceptual models and design concepts for the Yucca Mountain site. Correlations between input values are appropriately established in the DOE TSPA. Parameters used to define initial conditions, boundary conditions and computational domain in sensitivity analyses involving coupled T-H-M-C effects on seepage and flow, the waste package chemical environment, and the chemical environment for radionuclide release, are consistent with available data. Reasonable or conservative ranges of parameters or functional relations are established.

The ranges of inputs to this model (Section 4.1[a]) are consistent with initial boundary conditions and design concepts for the Yucca Mountain site because they come from various design documents and other related model reports.

(4) Adequate representations of the uncertainties in the characteristics of the natural systems and engineered materials is provided in the parameter development for conceptual models, process level models, and alternative

conceptual models. The DOE may constrain these uncertainties using sensitivity analyses or conservative limits.

The waste form in-package chemistry abstractions adequately represent the uncertainties in the characteristics of the natural systems and engineered materials either by incorporating direct functional relationships to independent parameters over their expected ranges (Sections 6.10.1[a], 6.10.2[a], and 6.10.3[a]) or by assessing the effects of parameter ranges on the variability of model outputs (Section 6.10.8[a]). The former approach defers uncertainty propagation to the models generating the values of the independent parameters while the latter approach is used to approximate uncertainty in in-package chemistry abstraction outputs propagated to the TSPA-LA model.

(5) If criticality is included in the TSPA, then the DOE uses an appropriate range of input parameters for calculating the effective neutron multiplication factor.

The subject of criticality is not addressed in this addendum. As such, this criterion is considered not to be applicable.

(6) Where sufficient data do not exist, the definition of parameter values and conceptual models is based on appropriate use of other sources, such as expert elicitation conducted in accordance with NUREG-1563.

The information contained in this addendum is based on established physical models, justified assumptions, and qualified input data. There was no need to use other sources such as expert elicitation.

Acceptance Criterion 4 – Model Uncertainty Is Characterized and Propagated through the Model Abstraction

The results of sensitivity analyses were used to approximate IPC model uncertainty for propagation into the TSPA-LA model (Section 6.10.8[a]).

(1) Alternative modeling approaches of FEPs are considered and are consistent with available data and current scientific understanding; and the results and limitations are appropriately considered in the abstraction.

Several alternative conceptual models are presented in Section 6.4 of the parent report. In addition, the waste package model and abstractions developed in the parent report provide an alternative modeling approach due to important differences in the control volume and the processes modeled (Section 6.3[a]). Although consideration was given to the use of these models in assessing model uncertainty, it was determined that the model developed in this addendum is substantially more consistent and integrated with other models used in TSPA-LA. Therefore, the results of the alternative models are not considered in the estimation of uncertainties in the abstractions developed in this addendum.

(2) Alternative modeling approaches are considered and the selected modeling approach is consistent with available data and current scientific understanding. A description that includes a discussion of alternative modeling approaches not

considered in the final analysis and the limitations and uncertainties of the chosen model is provided.

Section 6.4 of the parent report explicitly addresses alternative conceptual models related to the composition of the water entering the waste package after postulated breach and the manner in which this water transits the waste package following entry. In addition, Sections 6.3[a] and 6.10[a] explain why substantial changes were made to the in-package chemistry conceptual model and abstractions.

(3) Consideration of conceptual model uncertainty is consistent with available site characterization data, laboratory experiments, field measurements, natural analog information, and process-level modeling studies; and the treatment of uncertainty does not result in under-representation of the risk estimate.

The conceptual model developed in this addendum is based on established waste package dimensions and materials, predicted ranges of thermo-hydrologic parameters, and measurements of degradation rates and water compositions. Uncertainties related to the conceptual model include the fraction of internal surfaces exposed to water after waste package breach. In the conceptual model all surfaces are exposed to ensure that degradation of the waste forms is not under-represented (Section 6.3.1.1[a]). In addition, in the conceptual model in this addendum, surface complexation is not considered. If surface complexation had been included in the model and were shown to have a substantial pH buffering effect in the waste form cells, the effect would be a narrower pH range, which would reduce the maximum radionuclide solubilities achievable.

(4) Adequate consideration is given to the effects of thermal-hydrologic-mechanical-chemical coupled processes in the assessment of alternative conceptual models.

The effect of plug flow was given adequate consideration in Alternative Conceptual Model I (Section 6.4.1 of parent report), and the effect of variable composition of liquid influx over time was given adequate consideration in Alternative Conceptual Model II (Section 6.4.2 of parent report).

(5) If the DOE uses an equivalent continuum model for the TSPA abstraction, the models produce conservative estimates of the effects of coupled THMC processes on calculated compliance with post-closure public health and environmental standards.

The subject of an equivalent continuum model is not discussed in this addendum. As such, this criterion is considered not to be applicable.

Acceptance Criterion 5 – Model Abstraction Output Is Supported by Objective Comparisons

Validation of the waste form in-package chemistry model abstractions is documented in Section 7[a]. These abstractions are supported by objective comparisons to natural analogues where possible.

(1) The models implemented in this TSPA abstraction provide results consistent with output from detailed process level models and/or empirical observations (laboratory and field testings and/or natural analogues).

The abstractions in Section 6.10[a] provide results that are consistent with process model outputs documented in Sections 6.5[a] and 6.6[a] and with observations in the natural environment (Section 7[a]).

(2) Abstracted models for coupled THMC effects on seepage and flow and the waste package chemical environment, as well as on the chemical environment for radionuclide release, are based on the same assumptions and approximations demonstrated to be appropriate for process level models or closely analogous natural or experimental systems.

The model abstraction is validated in Section 7[a] and is based on the same assumptions and approximations demonstrated to be appropriate in the development of the process model.

(3) Accepted and well-documented procedures are used to construct and test the numerical models that simulate coupled THMC effects on seepage and flow, waste package chemical environment, and the chemical environment for radionuclide release. Analytical and numerical models are appropriately supported. Abstracted model results are compared with different mathematical models to judge robustness of results.

Accepted and well-documented procedures contained in the quality assurance program (Section 2[a]) governed the development of this addendum and the work documented. This model was developed, supported, and documented according to SCI-PRO-006. Test and validation methods (Section 7[a]), which include a comparison to outputs using a different geochemical code (Section 7.10.1[a]), also comply with SCI-PRO-006.

9[a]. INPUTS AND REFERENCES**9.1[a] DOCUMENTS CITED**

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9.2[a] CODES, STANDARDS, REGULATIONS, AND PROCEDURES

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- LS-PRO-001, Rev. 03, ICN 0. *Technical Reports*. Washington, D.C.: U.S. Department of Energy, Office of Civilian Radioactive Waste Management. ACC: DOC.20070627.0005.
- LS-PRO-0203, Rev. 03, ICN 0. *Preparation and Maintenance of the Q-List*. Washington, D.C.: U.S. Department of Energy, Office of Civilian Radioactive Waste Management. ACC: ENG.20070809.0002.
- SCI-PRO-004, Rev. 4, ICN 0. *Managing Technical Product Inputs*. Washington, D.C.: U.S. Department of Energy, Office of Civilian Radioactive Waste Management. ACC: DOC.20070718.0009.
- SCI-PRO-006, Rev. 4, ICN 0. *Models*. Washington, D.C.: U.S. Department of Energy, Office of Civilian Radioactive Waste Management. ACC: DOC.20070718.0002.

9.3[a] SOURCE DATA, LISTED BY DATA TRACKING NUMBER

- 165226 GS030408312272.002. Analysis of Water-Quality Samples for the Period from July 2002 to November 2002. Submittal date: 05/07/2003.
- 166570 GS031008312272.008. Analysis of Pore Water and Miscellaneous Water Samples for the Period from December 2002 to July 2003. Submittal date: 11/13/2003.
- 179065 GS060908312272.004. Chemical Analysis of Pore Water Samples Extracted from HD-PERM, USW SD-9, and ESF-SAD-GTB#1 Core for the Period from April 29, 2006 to July 21, 2006. Submittal date: 09/14/2006.
- 150930 MO0005PORWATER.000. Perm-Sample Pore Water Data. Submittal date: 05/04/2000.
- 172059 MO0409SPAACRWP.000. Aqueous Corrosion Rates For Non-Waste Form Waste Package Materials. Submittal date: 09/16/2004.

- 179290 MO0701EQ36IDPS.000. EQ3/6 Input and Output Files Used for IDPS Model Validation Simulations of Evaporation of Dilute Salt Solutions at 25C and 100C, Seawater, Binary and Ternary Salt Solutions, Synthetic Topopah Springs Tuff Pore Water at 95C and in the Carbonate System. Submittal date: 01/16/2007.
- 181214 MO0704PHREEQ25.000. PHREEQC Thermodynamic Database from DATA0 Rev. 05 for 25 C. Submittal date: 04/20/2007.
- 182270 MO0707DISVALID.000. Dissolved Concentration Limits Files for Validated Models. Submittal date: 07/18/2007.
- 178850 SN0612T0502404.014. Thermodynamic Database Input File for EQ3/6 - DATA0.YMP.R5. Submittal date: 12/15/2006.
- 180745 SN0704PMAIPCPM.001. In-Package Chemistry Process Model Files for Performance Margin Analysis. Submittal date: 04/19/2007.
- 182269 SN0705PAEBSPCE.015. P&CE Selection of Pore Water Data. Submittal date: 05/31/2007.

9.4[a] DATA TRACKING NUMBERS NOT INCLUDED IN DIRS REPORT

Output DTN: SN0702PAIPC1CA.001. In-Package Chemistry Calculations and Abstractions. Submittal date: 04/04/2007.

Validation DTN: SN0705IPCEQPHR.001. In-Package Chemistry EQ3/6-PHREEQC Validation. Submittal date: 05/09/2007.

9.5[a] SOFTWARE CODES

- 153964 EQ3/6 V. 7.2b. 1999. UCRL-MA-110662 (LSCR198).
- 159731 EQ6 V. 7.2bLV. 2002. WINDOWS 2000, NT. STN: 10075-7.2bLV-02.
- 173680 GetEQData V. 1.0.1. 2002. WINDOWS 2000. STN: 10809-1.0.1-00.
- 175698 PHREEQC V. 2.11. 2006. WINDOWS 2000. STN: 10068-2.11-00.

APPENDIX A[a]
DATA QUALIFICATION

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Data Qualification Plan

Complete only applicable items.

QA: QA
Page 1 of 2

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| Section I. Organizational Information | |
| Qualification Title | |
| Qualification of In-Package Chemistry Input Data | |
| Requesting Organization | |
| Near Field Environment | |
| Section II. Process Planning Requirements | |
| 1. List of Unqualified Data to be Evaluated | |
| a) | The chemical composition of Defense High Level Waste glass produced at the Savannah River Laboratory's Defense Waste Processing Facility as cited from Allison 2004 [DIRS 168734]. |
| b) | The pH of DHLW glass dissolution experiments as cited from Bates et al. 1993 [DIRS 100705]. |
| c) | The composition of J-13 well water as cited from Harrar et al. 1990 [DIRS 100814]. |
| d) | The composition of water from a basalt aquifer in Iceland as cited from Gislason and Eugster 1987 [DIRS 168716]. |
| e) | The composition of water from basalt aquifers in the Columbia Basin, Washington as cited from Turney 1986 [DIRS 179852]. |
| 2. Type of Data Qualification Method(s) [Including rationale for selection of method(s) (Attachment 3) and qualification attributes (Attachment 4)] | |
| a) | The Allison data will be qualified using Method 4 – Technical Assessment Rationale: The data collection procedures are not available. Attributes: 1) Qualification of organization generating data, and 2) Extent to which data demonstrate properties of interest. |
| b) | The Bates data will be qualified using Method 1 – Equivalent QA Program. Rationale: These data were generated for the DOE Environmental Restoration and Waste Management Program (EM) under a DOE approved Quality Assurance Plan. Attributes: 1) Qualification of organization generating data, 2) Extent to which data demonstrate properties of interest, 3) Quality of measurement control program, and 4) Extent and reliability of documentation associated with the data. |
| c) | The Harrar et al. data will be qualified using Method 4 – Technical Assessment Rationale: The data collection procedures are not available. Attributes: 1) Qualification of organization generating data, 2) Extent to which data demonstrate properties of interest, and 3) Quality of measurement control program. |
| d) | The Gislason & Eugster data will be qualified using Method 4 – Technical Assessment Rationale: Rationale: The data collection procedures are not available. Attributes: 1) Qualification of organization generating data, and 2) Extent to which data demonstrate properties of interest. |
| e) | The Turney data will be qualified using Method 4 – Technical Assessment Rationale: Rationale: The data collection procedures are not available. Attributes: 1) Qualification of organization generating data, and 2) Extent to which data demonstrate properties of interest. |
| 3. Data Qualification Team and Additional Support Staff Required | |
| William F. Downs, Qualification Chairperson Paul Mariner | |
| 4. Data Evaluation Criteria | |
| a) | The Technical Review of the HLW glass will focus on the 1) Qualification of organization generating data, and 2) Extent to which data demonstrate properties of interest. |
| b) | The QA system in effect at ANL during the time that this number was generated will be assessed to determine its adequacy as a surrogate for the QA Program specified in the YMP QARD. |
| c) | The Technical Review of the composition of J-13 well water will focus on 1) Qualification of organization generating data, 2) Extent to which data demonstrate properties of interest, and 3) Quality of measurement control program. |
| d) | The Technical Review of the composition of Icelandic basalt reservoir water will focus on 1) Qualification of organization generating data, 2) Extent to which data demonstrate properties of interest, and 3) Quality of corroborating data. |
| e) | The Technical Review of the composition of Columbia Basin basalt reservoir water will focus on 1) Qualification of organization generating data, 2) Extent to which data demonstrate properties of interest, and 3) Quality of corroborating data. |

SCI-PRO-001.1-R1



Data Qualification Plan

QA: QA
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Complete only applicable items.

| | | |
|---|---|-----------------|
| 5. Identification of Procedures Used SCI-PRO-001 – Qualification of Unqualified Data SCI-PRO-006 – Models | | |
| 6. Plan coordinated with the following known organizations providing input to or using the results of the data qualification This plan is not coordinated with any organization other than Performance Assessment, Near Field Environment. | | |
| Section III. Approval | | |
| Qualification Chairperson Printed Name William F. Downs | Qualification Chairperson Signature | Date 7/24/07 |
| Responsible Manager Printed Name Geoff Freeze <i>for Ernest Hardin</i> | Responsible Manager Signature | Date 7/26/07 |

SCI-PRO-001.1-R1

Qualification of External Source Data

This section presents planning and documentation for the data qualification of unqualified external source data used as direct input only to this report. Data qualification is performed in accordance with SCI-PRO-006, *Models*.

Data for Qualification

There are five external sources of unqualified data used as direct input to this report addendum:

1. Allison, J.M. 2004. "Request for Referenceable Information on High-Level Waste (HLW) Radionuclide Inventories in Support of Preparation of the Yucca Mountain Project License Application (Your Letter, JCP-0445, 1/28/04)." Memorandum from J.M. Allison (DOE/SR) to J. Arthur, III (OCRWM), February 26, 2004, 0303040661, with attachment. ACC: MOL.20040317.0265. [DIRS 168734]
2. Bates, J.K.; Bourcier, W.L.; Bradley, C.R.; Buck, E.C.; Cunnane, J.C.; Dietz, N.L.; Ebert, W.L.; Emery, J.W.; Ewing, R.C.; Feng, X.; Gerding, T.J.; Gong, M.; Hoh, J.C.; Li, H.; Mazer, J.J.; Morgan, L.E.; Newton, L.; Nielsen, J.K.; Phillips, B.L.; Tomozawa, M.; Wang, L.; and Wronkiewicz, D.J. 1993. *ANL Technical Support Program for DOE Environmental Restoration and Waste Management. Annual Report, October 1991-September 1992*. ANL-93/13. Argonne, Illinois: Argonne National Laboratory. TIC: 244798. [DIRS 100705]
3. Harrar, J.E.; Carley, J.F.; Isherwood, W.F.; and Raber, E. 1990. *Report of the Committee to Review the Use of J-13 Well Water in Nevada Nuclear Waste Storage Investigations*. UCID-21867. Livermore, California: Lawrence Livermore National Laboratory. ACC: NNA.19910131.0274. [DIRS 100814]
4. Gislason, S.R. and Eugster, H.P. 1987. "Meteoric Water-Basalt Interactions. I: A Laboratory Study." *Geochimica et Cosmochimica Acta*, 51, (10), 2827-2840. New York, New York: Pergamon. TIC: 235295. [DIRS 168716]
5. Turney, G.L. 1986. *Quality of Ground Water in the Columbia Basin, Washington, 1983*. Water-Resources Investigations Report 85-4320. Tacoma, Washington: U.S. Geological Survey. ACC: LLR.20070321.0001. [DIRS 179852]

Method of Qualification Selected

The method for qualification for four of the external sources of data (Numbers 1, 3, 4, and 5) is the "technical assessment method." The rationale for using this method for these documents is that there is no record of the quality assurance plans under which the data were collected in the original source (i.e., scientific journal or publication). Qualification process attributes used in the technical assessment of each external source are selected from the list provided in Attachment 4 of SCI-PRO-001, *Qualification of Unqualified Data*. Attributes used specifically as data qualification attributes in this report are:

- Qualifications of personnel or organizations generating the data are comparable to qualification requirements of personnel generating similar data under an approved program that supports the YMP license application process or postclosure science.
- The extent to which the data demonstrate the properties of interest (e.g., physical, chemical, geologic, mechanical).
- The quality and reliability of the measurement control program under which the data were generated.

The method of qualification for document Number 2 (Bates et al. 1993 [DIRS 100705]) is Method 1 – Equivalent QA Program. These data were collected at Argonne National Laboratory under the Technical Support Program for DOE Environmental Restoration and Waste Management. This program had a DOE-approved quality assurance plan that is available and that can be reviewed. Attributes used specifically as data qualification attributes in this report are:

- Qualifications of personnel or organizations generating the data are comparable to qualification requirements of personnel generating similar data under an approved program that supports the YMP license application process or post closure science.
- The extent to which the data demonstrate the properties of interest (e.g., physical, chemical, geologic, mechanical).
- Quality of measurement control program.
- Extent and reliability of the documentation associated with the data.

1. Technical Assessment of External Data from Allison 2004 [DIRS 168734]

In previous versions of this document, the composition of HLW glass was estimated from the compositions of waste sludges and the estimates of the components necessary to produce a borosilicate glass waste form. The Savannah River Laboratory (SRL) Defense Waste Processing Facility generated several tons of HLW glass in batches and these were sampled and analyzed. The measured HLW glass compositions were incorporated into this report.

The following criteria were used to assess these external data:

- Qualifications of personnel or organizations generating the data are comparable to qualification requirements of personnel generating similar data under an approved program that supports the YMP license application process or postclosure science
- The extent to which the data demonstrate the properties of interest (e.g., physical, chemical, geologic, mechanical).

Justification for the appropriate use of data: The analyses were conducted in accordance with SRL analytical procedures and under the DOE-approved SRL QA program (Fellinger and Bibler 2000 [DIRS 179902]). The technical personnel are internationally recognized in the field of

nuclear waste form chemistry and have many peer-reviewed publications to their credit. The SRL QA program requires multiple analyses of the HLW glass samples as an internal quality control check and has procedures for maintenance and calibration of the instrumentation. Thus, the qualifications of the personnel and organization generating these data are comparable to those generating data specifically for the YMP license application.

These data are specifically for the chemical compositions of the HLW glasses that are slated for disposal in the Yucca Mountain repository. The applicability of the data is direct and requires no interpretation or extrapolation.

Thus, these data are qualified for their current use as direct input in this document.

2. Qualification of External Data from Bates et al. 1993 [DIRS 100705]

The data used from this document is the pH of the solutions in contact with HLW glass. Method 1 – Equivalent QA Program – is used to qualify these data.

The following criteria were used to assess these external data:

- Qualifications of personnel or organizations generating the data are comparable to qualification requirements of personnel generating similar data under an approved program that supports the YMP license application process or postclosure science.
- The extent to which the data demonstrate the properties of interest (e.g., physical, chemical, geologic, mechanical).
- The quality and reliability of the measurement control program under which the data were generated.
- The extent and reliability of the documentation associated with the data.

Justification for the appropriate use of the data: The data were measured by well-respected scientists at Argonne National Laboratories (ANL) under a DOE-approved ANL QA program – *Quality Assurance Plan for Unsaturated Glass Testing for DOE-HQ/DP Defense High Level Waste Processing* (ANL 1989 [DIRS 182277]). In addition, the program underwent several quality audits during the course of the investigations (Hartstern 1994 [DIRS 182279]; George 1994 [DIRS 182280]). The audits found that the scientists were knowledgeable and were conducting all activities in accordance with the requirements of *Quality Assurance Requirements and Description* (DOE 2007 [DIRS 182051]). The scientists conducting the experiments are internationally recognized with numerous peer-reviewed publications to their credit. The document containing the experimental data includes the ranges of measurements and discussions of the reasons for the variability of those measurements. Thus, the organization and personnel were qualified, the measurement control program was reliable, and the documentation of these facts is readily available.

The data from this source is directly relevant to estimating pH in thin films of water on HLW glass because the material used in the study is essentially identical to the material being modeled. No other data more closely match the model application.

Thus, these data are qualified for direct input in this document.

3. Qualification of External Data from Harrar et al. 1990 [DIRS 100814]

The data used from this document is the composition of groundwater monitoring well J-13. Method 4 – Technical Review – is used to qualify these data.

The following criteria were used to assess these external data:

- Qualifications of personnel or organizations generating the data are comparable to qualification requirements of personnel generating similar data under an approved program that supports the YMP license application process or postclosure science.
- The extent to which the data demonstrate the properties of interest (e.g., physical, chemical, geologic, mechanical).
- The quality and reliability of the measurement control program under which the data were generated.

Justification for the appropriate use of the data: This report is a compilation and evaluation of analytical data collected from the J-13 well over nearly 25 years and analyzed by various laboratories. These laboratories include the U.S. Geological Survey, Lawrence Livermore National Laboratory, Argonne National Laboratory, Hanford Engineering Development Laboratory, and Los Alamos National Laboratory. All of these organizations are internationally recognized and their staffs have numerous peer-reviewed publications. Over the period of sampling and between the various laboratories, the sample collection and analyses protocols were undoubtedly different. However, once the well was developed in 1963, the Committee concluded that the concentrations of the various dissolved constituents are stable and unchanging. Differences among sampling and analysis protocols that introduced bias into the measurements would have been evident during the Committee's analysis. This report provides a mean composition for the J-13 well water and the small standard deviation provides confidence that these mean compositions are representative of the J-13 well water. In addition, the J-13 data are very similar to pore-water samples from Tsw cores. These mean compositional data have been employed by the YMP in a number of performance assessment documents including *EBS Radionuclide Transport Abstraction* (SNL 2007 [DIRS 177407]), *Dissolved Concentration Limits of Elements with Radioactive Isotopes* (SNL 2007 [DIRS 177418]), and *Evaluation of Potential Impacts of Microbial Activities on Drift Chemistry* (BSC 2004 [DIRS 169991]). This widespread use demonstrates that the data are properties of interest and are appropriate for the YMP.

Based on this assessment, the data from Harrar et al. (1990 [DIRS 100814]) are qualified for use as direct input to this report.

4. Qualification of External Data from Gislason and Eugster 1987 [DIRS 168716]

The data used from this document is the composition of groundwater from a basalt aquifer in Iceland. These data were used to estimate the composition of influent water in the igneous scenario. Method 4 – Technical Review – is used to qualify these data.

The following criteria were used to assess these external data:

- Qualifications of personnel or organizations generating the data are comparable to qualification requirements of personnel generating similar data under an approved program that supports the YMP license application process or postclosure science.
- The extent to which the data demonstrate the properties of interest (e.g., physical, chemical, geologic, mechanical).

Justification for the appropriate use of the data: This journal article contains analyses of groundwater samples from springs issuing from a fractured basalt aquifer in Iceland. The researchers are professors at the University of Iceland and Johns Hopkins University, respectively, and have many publications in the area of rock-water interactions to their credit. The data are published in *Geochimica et Cosmochimica Acta*, one of the oldest and most respected periodicals in the field of geochemistry. All articles are subjected to peer-review by three researchers in the field. The data are for groundwaters in equilibrium with basalt and are used to simulate the water that would be entering a breached waste package after inundation in magma and cooling to less than the boiling temperature of the fluid. The equilibrium concentrations of those hypothesized fluids should be similar to those from basalt aquifers.

Based on this assessment, the basalt water data from Gislason and Eugster (1987 [DIRS 168716]) is qualified as direct input to this addendum.

5. Qualification of External Data from Turney 1986 [DIRS 179852]

The data used from this document is the composition of groundwater from a basalt aquifer in the Columbia Basin, Washington. These data were used to estimate the composition of influent water in the igneous scenario. Method 4 – Technical Review – is used to qualify these data.

The following criteria were used to assess these external data:

- Qualifications of personnel or organizations generating the data are comparable to qualification requirements of personnel generating similar data under an approved program that supports the YMP license application process or postclosure science.
- The extent to which the data demonstrate the properties of interest (e.g., physical, chemical, geologic, mechanical).

Justification for the appropriate use of the data: This USGS Water-Resources Investigations Report contains analyses of groundwater samples from groundwater wells in a fractured basalt aquifer in the Columbia Basin, Washington. The researcher was a member of the USGS and has many publications in the area of water resources and quality to his credit. The data are published

in *Water-Resources Investigations Report 85-4320*, one of the standard reports on water resources and quality published by the USGS. All USGS publications are subjected to peer-review by independent USGS researchers in the field and are approved for publication by the Director. The data are for groundwaters in contact with basalt and are used to simulate the water that would be entering a breached waste package after inundation in magma and cooling back to less than the boiling temperature of the fluid. The concentrations of those hypothesized fluids should be similar to those from basalt aquifers.

Based on this assessment, the basalt water data from Turney (1986 [DIRS 179852]) is qualified as direct input to this report.

APPENDIX B[a]
CRITICAL REVIEW

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A critical review of the waste form IPC model was performed as described in Section 7.10.2[a]. The results of the review are attached below.



Albuquerque, New Mexico 87185-

date: July 2, 2007

to: Geoffrey Freeze

from: Laura Price

Laura Price 7/2/07

subject: INDEPENDENT TECHNICAL REVIEW FOR MODEL VALIDATION OF THE IN-PACKAGE CHEMISTRY MODEL

The purpose of this memo is to document the findings of a critical review of *In-Package Chemistry Abstraction Addendum* (ANL-EBS-MD-000037 REV 04 AD01B) that was performed to satisfy post-development model validation requirements, as specified in SCI-PRO-006 Section 6.3.2. According to Table 7-1[a] of this model report, the particular components of the in-package chemistry model that are to be validated using independent technical review are pH, ionic strength, fluoride concentration, and total carbonate concentration.

According to the Technical Work Plan that governed the work described in the addendum (*Technical Work Plan for Waste Form Testing and Modeling*, TWP-WIS-MD-000018 REV 01, Table 2-3), the post-development model validation methods that were to be used to validate these components were comparison with published field, experimental, and literature data. However, as discussed in Section 7[a] of *In-Package Chemistry Abstraction Addendum* (ANL-EBS-MD-000037 REV 04 AD01B), these methods were subsequently found to be inadequate, and so the two post-development model validation methods adopted were independent technical review and a comparison of estimated values from two independent modeling codes. This memo documents the independent technical review.

As specified in Attachment 4 of SCI-PRO-006, which gives instructions for conducting the model review, any comments generated during the critical review should be documented on Document Review Comment Sheets (DRCS). Accordingly, specific review comments discussed below have also been entered on the accompanying DRCS, and reference is made to the corresponding comment number on the DRCS.

Assumptions and Conceptual Model

It is often necessary to make assumptions to develop and define the conceptual model of the system being modeled. Five assumptions are presented in Section 5 of the report; some are carried over from the parent report and some are new. I found all assumptions to be reasonable and appropriate given the intended purpose of the model.

I think it would be interesting to investigate the sensitivity of the dose estimated by the TSPA-LA model to the waste package simulation concept, whether the waste package is represented by one cell or by two distinct cells (Assumption 5.4). The explanation given in Assumption 5.4 as to why modeling the waste

Exceptional Service in the National Interest

package as two distinct cells is appropriate even after the cells have lost their structural integrity is reasonable, but such a sensitivity analysis might provide some insight as to important processes.

pH Abstraction

The pH abstraction is presented in Section 6.10.1[a]. It is based on the observation that the degradation of fuels and structural materials in the waste form cells generates acid neutralizing capacity (ANC) and base neutralizing capacity (BNC), primarily in the form of secondary minerals such as schoepite and trevorite, or in the form of CO_2 , as in the case of the DHLW waste form cell. To develop the abstraction, the pH range in a waste form cell is determined by titrating the contents of a waste form cell with acid and base. These titrations are used to identify the pH buffering minerals in the waste form cells, define the pH limits, and quantify the buffer capacities. The abstractions are presented in Figures 6-41[a], 6-42[a], and 6-43[a], with pH given as a function of pCO_2 and ionic strength. For a given pCO_2 and ionic strength, maximum and minimum pH values are defined, and these are assumed to represent maximum and minimum values of a uniform distribution, which is sampled in TSPA. Titrations were performed for both liquid-influx and vapor-influx conditions, using a similar approach. The results from the liquid-influx case were indistinguishable from those of the vapor-influx conditions, and so the vapor-influx results were not reported separately in the model report. Accordingly, the results of my review of the pH abstraction for the liquid-influx case and the vapor-influx case are discussed together.

The approach of recognizing and modeling the buffering capacity of the degrading components inside the waste package is technically sound and consistent with fundamental chemical principles. In general, the results of the titration are consistent with base-case output from the process model, which in turn is consistent with pH results from an independent modeling code (PHREEQC, Figure 7-1[a]) adding confidence to model results.

I was concerned with whether the sensitivities explored in Section 6.6 are adequately captured in the abstraction. The pH abstraction is given as a uniform distribution between maximum and minimum values that are determined by pCO_2 and ionic strength. As a result, other dependencies are not explicitly incorporated into the abstraction. Two particular potential sensitivities were of concern: dependence on liquid influx rate (Section 6.6.4) and on degradation rates (Section 6.6.5). These are discussed below.

While Section 6.6.4 states that the pH abstraction is not functionally dependent on liquid influx rate and time, a study of the model output reveals that the dependence of pH on liquid influx flow rate is captured to some extent via the dependence of pH on ionic strength. Figures 6-26[a], 6-28[a], and 6-30[a] show how pH varies with flow rate over time for various seepage compositions for CSNF, DHLW, and 2MCO waste form cells, respectively. Figure 6-26[a] shows that the pH in a CSNF cell does not drop below 6 for very high flow rates (1000 L/yr); this trend is more-or-less reflected in the pH abstraction for CSNF in that the minimum pH in a CSNF cell for low ionic strength (which is associated with high flow rates) is about 5.5 (Figure 6-41[a]). Figure 6-28[a] shows that the pH in a DHLW waste form cells is not highly dependent on liquid influx rate. Figure 6-30[a] shows that higher liquid influx rates are associated with higher pH values in a 2MCO waste form cell. This behavior is more-or-less reflected in the pH abstraction in that the minimum pH value increases with decreasing ionic strength; that is, the minimum pH value increases with increasing flow rate (Figure 6-43[a]). Thus, although the pH abstractions are not explicitly functions of liquid influx rate and time, they do incorporate general trends of pH with liquid influx rate, although they may not necessarily produce precise values of minimum pH.

It is stated in Section 6.6.5 that the functionality of degradation rates is not included in the pH abstraction, but no justification is given for this. Figures 6-32[a], 6-34[a] and 6-36[a] give pH in the

CSNF, DHLW, and 2MCO cells (respectively) as a function of time for various water compositions and extreme values (i.e., minimum and maximum values) of degradation rate. These figures indicate that, for CSNF and 2MCO waste form cells, pH varies over time for a given degradation rate of a given material. However, for the DHLW waste form cell, the sensitivity results in Figure 6-34[a] indicate that, for all starting waters, a low HLWG degradation rate results in a pH around 6 for all times after about 1000 years. This same figure shows that, for all starting waters, a high HLWG degradation rate results in a pH around 9 for all times after about 100 years. The HLWG degradation rate coefficient is sampled in TSPA, and given the strong influence it seems to have on pH, it appears at first glance that it might be appropriate to correlate the HLWG degradation rate coefficient and the sampled pH percentile. This comment is captured in the DRCS as comment 4.

However, as the response to this comment indicates, the HLWG degradation rate is itself a function of pH, with higher pH values leading to higher degradation rates, all other factors being equal. Therefore, higher pH values will be associated with higher HLWG degradation rates, consistent with Figure 6-34[a]. In addition, there appears to be a feedback mechanism at work, since the HLWG degradation rate is a function of pH, but the pH appears to be controlled to some extent by the HLWG degradation rate (Figure 6-34[a]). Given this feedback mechanism, it is not obvious that it would be appropriate to correlate the sampled pH percentile and the HLWG degradation rate coefficient. Finally, the response to the comment indicates that this functionality was incorporated into the Performance Margin Analysis (PMA), although the significance of this added functionality has not yet been assessed. I would expect the sensitivity analyses that will be performed as part of analyzing the PMA will provide some insights as to the relationship between the HLWG degradation rate and the pH.

My concerns regarding whether the abstraction adequately captured the sensitivities explored in Section 6.6 have been addressed. I find that this model is adequate for its intended purpose.

Ionic strength

The ionic strength abstraction is presented in Section 6.10.2. These abstractions are discussed separately (liquid influx and vapor influx) below.

Liquid Influx Ionic Strength Abstraction

The approach to modeling ionic strength in a waste form cell in a failed waste package under liquid influx conditions assumes that ionic strength is a function of liquid influx rate and time since waste package breach. This approach is reasonable and consistent with basic physical principles. Furthermore, the process model results are similar to those from an independent modeling code (Figure 7-2[a]), adding confidence to the model. The simulations were run in a reasonable fashion and were run such that the ionic strength would not exceed 1, which is the upper range of the applicability of the B-dot equation. However, two particular issues were of concern; these are discussed below.

Of the sensitivities explored in Section 6.6, the only sensitivity that is of concern is the lack of functional dependence of the ionic strength on the DHLW degradation rate. Figure 6-35 indicates that, for all times after about 100 years after waste package breach, high DHLW degradation rates result in ionic strengths of about 1×10^{-1} M, while low DHLW degradation rates result in ionic strengths between 1×10^{-3} and 1×10^{-2} M. As mentioned above, the DHLW degradation rate coefficient is sampled in TSPA, and its distribution is skewed to lower values. To avoid the possibility of modeling physically unrealizable conditions inside a failed waste package, it appears at first glance that the DHLW degradation rate coefficient should be linked to the ionic strength, perhaps via correlation. This issue is comment 5 in the DRCS.

However, linking in some fashion the ionic strength and the DHLW degradation rate coefficient could introduce unintended and unforeseen consequences in the TSPA-LA because of the complex relationships between ionic strength, pH, and DHLW degradation rate. If the ionic strength were to be a function of DHLW degradation rate (as well as liquid influx rate and time since waste package breach), then ionic strength would be, indirectly, a function of pH because the DHLW degradation rate is a function of pH. But in the abstraction the opposite is true: pH is a function of ionic strength. Although the complex relationships between ionic strength, pH, and DHLW degradation rate are not completely captured in their respective abstractions, the in-package chemistry abstraction does capture the two most important parameters for estimating ionic strength (flow rate and time since waste package breach), as pointed out in the response to DRCS comment #5. I believe that the proper way to address the issue of the effect of the DHLW degradation rate on ionic strength is to incorporate it directly into the abstraction; this was done for the PMA. Analysis of PMA results should provide some insights into the relationship between DHLW degradation rate, ionic strength, and pH.

The other concern is in how uncertainty in ionic strength is propagated through the abstraction. It is stated in Section 6.10.8.2 that extreme degradation rates cause ionic strength to both increase and decrease from base-case conditions, justifying an uncertainty distribution with a central tendency. However, comparison of the figures mentioned in Section 6.10.8.2 (Figure 6-6[a] and Figure 6-33[a]; Figure 6-9[a] and Figure 6-35[a], and Figure 6-12[a] and figure 6-37[a]) show that extreme degradation rates generally cause only an increase in ionic strength, not a decrease, as compared to base-case simulations. This observation calls into question the basis for specifying the uncertainty distribution as having a central tendency. This is comment 9 in the DRCS.

However, the response to comment 9 points out that "...simulations involving condensed water were not included in these figures. Including condensed water would have provided an incoming water with a low ionic strength, which would be important if the rates were low for the two or three degradation reactions responsible for the largest contributions to ionic strength in a cell." Although the figures have not been updated to reflect the sensitivity of ionic strength to condensed water, it is reasonable to believe that calculated ionic strengths would be lower than those currently shown on the figures. This would support an uncertainty distribution that has a central tendency.

My concerns regarding whether the abstraction adequately captured the sensitivities explored in Section 6.6, and how uncertainty in ionic strength was propagated through the abstraction have been addressed. I find that this model is adequate for its intended purpose.

Vapor Influx Ionic Strength Abstraction

The vapor influx ionic strength abstraction is based on the relative humidity, because model calculations showed that ionic strength was a strong function of relative humidity and only a weak function of the various degradation reactions. Calculations were performed for simple salt solutions to establish the relationship between equilibrium relative humidity and ionic strength. For ionic strengths below 1 M, EQ6 (which uses the B-dot equation) was used, and for ionic strengths above 1M the Pitzer database, which is qualified for use at higher ionic strengths, was used.

The approach of basing ionic strength on relative humidity is reasonable and consistent with basic physical principles, and the treatment of uncertainty appears to agree with the calculations on which the abstraction is based. I agree with the overall approach and find it to be adequate for its intended purpose.

Fluoride

The fluoride abstraction is developed and presented in Section 6.10.3. It applies only under liquid influx conditions because inflowing water is the source of fluoride, and the water source under vapor influx conditions is assumed to be pure water. The purpose of the fluoride abstraction is to provide estimates of the fluoride concentration in a waste form cell to the model that calculates radionuclide concentrations.

The model is based on examining the results of the sensitivity studies performed for Section 6.6 (Figures 6-51[a] through 6-53[a]). The abstraction presents fluoride concentration as a function of ionic strength for the three different waste form cells. The abstraction appears to overpredict fluoride concentration, which will result in higher radionuclide concentrations in the waste form cell because concentrations of dissolved radionuclides tend to increase with increasing fluoride concentrations. However, given the manner in which the fluoride concentration is incorporated into the dissolved radionuclide concentration abstraction, the effect is likely to be small. I conclude that the fluoride abstraction is adequate for its intended purpose.

Total Carbonate

The total carbonate model has not changed from the model presented in the parent document. In the parent document, the total carbonate was calculated using the equilibrium mass action expressions. This is a reasonable and valid approach for the system in question, where the partial pressure of CO₂ is constant over the modeled period and the pH and temperature are known. Uncertainty in the abstraction is accounted for via uncertainty in pH. This model is adequate for its intended use.

Summary

The models for pH, ionic strength, fluoride concentration, and total carbonate concentration inside a failed waste package are consistent with basic chemical principles, and the abstractions are reasonable representations of the process model outputs. I find that these models are adequate for their intended purpose.