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1	UNITED STATES OF AMERICA
2	NUCLEAR REGULATORY COMMISSION
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4	PUBLIC MEETING
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6	REGARDING LICENSE RENEWAL
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8	Tuesday, December 16, 2003
9	The Park of the Quad Cities
10	1201 River Drive
11	Moline, Illinois
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13	The NRC Public Meeting met at The Mark of the
14	Quad Cities, 1201 River Drive, at 7:00 p.m., Chip
15	Cameron presiding.
16	PRESENT:
17	Chip Cameron
18	John Tappert
19	Duke Wheeler
20	Bruce McDowell
21	Robert Palla
22	Kimberly Kimberley Corp
23	
24	
25	

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## P-R-O-C-E-E-D-I-N-G-S

(7:00 P.M.)

MR. CAMERON: Good evening, everyone. And welcome to the NRC's public meeting tonight. My name is Chip Cameron. I'm the Special Counsel for Public Liaison at the Nuclear Regulatory Commission. And it's my pleasure to serve as your facilitator for the meeting tonight. And in that role I'll just try to make sure that all of you have a productive meeting.

And the topic tonight is the Draft Environmental Impact Statement that the NRC has prepared to assist it in its evaluation of the license application that we got for renewal of the Quad Cities operating license from the Exelon Company. Our format for the meeting is fairly simple. We're going to give you some background information. We have a number of presentations tonight on the NRC process and also on what the conclusions and findings and analysis are that are contained in the Draft Environmental Impact Statement.

We also want to hear from any of you who want to make a more formal comment on the record for us tonight on any of the issues in the Draft Environmental Impact Statement. And ground rules, if you have any questions or whatever, just signal me.

I'll bring you this cordless microphone. Tell us who you are and your affiliation if appropriate. And we'll capture that on a transcript. Mr. Ron LeGrand is our stenographer tonight. That transcript of this meeting will be available, publicly available for anybody who wants to see it.

And the agenda is going to start out with Mr. John the Chief Tappert, who is of the Environmental Section in the Office of Nuclear Reactor Regulation back at NRC Headquarters in Rockville, Maryland. John's going to give you a formal welcome. And then we're going to move to Kimberly Kimberley Corp, who's here, who is also with the NRC at NRC in our Office of Nuclear Headquarters Regulation. Kimberly Kimberley is the backup Project Manager on the evaluation of the Quad Cities license renewal application, on the safety evaluation. you'll be hearing there's an environmental evaluation. There's a safety evaluation to aid us in making a decision on whether to grant the renewal. Kimberly Kimberley will tell us about the overall license renewal process.

We're then going to focus in on the environmental review process. And we do have the project manager for the environmental review for the

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Quad Cities license renewal and that's Mr. Duke Wheeler. He's right here. Also, Office of Nuclear Reactor Regulation. Then we'll go on to you to see if there are any questions about the license renewal process.

And then we're going to get into some substantive conclusions. We're going to have Mr. Bruce McDowell, who's right over here. And Bruce is the team leader for the group of experts that the NRC has helping us to prepare the Draft Environmental Impact Statement. Bruce is from Lawrence Livermore National Lab in California. And he leads the team of experts from labs around the country who have been looking at the environmental impact. He's going to tell you what's in the Draft Environmental Impact Statement. He's going to do a summary of that for you.

We have a short subject, so to speak, something called Severe Accident Mitigation Alternatives. That's part of the Environmental Impact Statement and Bob Palla from the NRC staff, again Office of Nuclear Reactor Regulation, is going to lead us through that. And then we'll go for questions. And then we'll go to those who might want to make a formal comment. And if you decide to make a formal comment, just let me know. We've asked people to sign

up in advance but we don't have a big crowd, so if the moment seizes you during the meeting, just let us know.

And thank you for being here tonight and we'll try to do our best to answer your questions.

And we definitely want to listen to your comments.

And I'm going to ask John to start us up.

MR. TAPPERT: Thank you, Chip. And good evening and welcome. And for those of you back from this afternoon, welcome back. My name is John Tappert and on behalf of the Nuclear Regulatory Commission I'd like to thank everyone for coming out tonight and participating in this process. I hope that you'll find the information we will share with you tonight to be useful and we look forward to receiving your comments tonight and in the future.

I'd like to start off by briefly going over the agenda and the purposes of tonight's meeting. First of all, we're going to provide a brief overview of the entire license for renewal process. This includes both a safety review as well as the environmental review, which will be the principle focus of tonight's meeting. Then we're going to present the preliminary findings of our environmental review, which assesses the impacts associated with

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extending the operating licenses, the Quad Cities Units 1 and 2, for an additional 20 years. Then we'll give you some information about the schedule for the balance of our review and how you can submit comments in the future. And then finally, really the most important part of tonight's meeting where we receive any comments that you may have tonight.

But first let me provide some general context for the license renewal process. The Atomic Energy Act gives the NRC the authority to issue operating licenses to commercial nuclear power plants for a period of 40 years. For the Quad Cities units, those licenses will expire in 2012. And our regulations also make provisions for extending those operating licenses for an additional 20 years. And Exelon has requested license renewal for both units.

As part of the NRC's review of that license renewal application, we do an environmental review to look at the impacts on the environment for 20 years of extended operation. And we held a meeting here last April to seek your input early in our review and now we've returned, as we indicated at that earlier scoping meeting, to present the preliminary results in our Draft Environmental Impact Statement. And again, the real reason we're here

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tonight is to receive any comments that you may have on that draft.

And with that brief introduction, I'd like to ask Kimberly Kimberley to give us more information on the safety review.

MS. CORP: Thank you, John. As Chip said, my name is Kimberly Kimberley Corp and I'm the NRC's Backup Project Manager supporting the safety review of the Exelon's license renewal application for both Quad Cities in Justin Dresden. Before I get into the discussion of the license renewal process, I'd like to take a minute to talk about the Nuclear Regulatory Commission in terms of what we do and what our mission is.

As John said earlier, the Atomic Energy Act of 1954 is the legislation that authorizes the NRC to regulate the civilian use of nuclear materials. In caring carrying out that authority, the NRC's mission is threefold. One is to insure adequate protection of public health and safety, two is to protect the environment, and three is to provide for common defense and security.

The NRC accomplishes its mission through a combination of regulatory programs and processes such as inspections, enforcement actions, assessment

of licensee performance and evaluation of operating experience of nuclear plants across the country. The NRC's license renewal review is similar to the original licensing process in that it involves two parts.

The safety review, which includes a safety evaluation, plant inspections and independent review by the ACRS or Advisory Committee on Reactor Safeguards, as well as an environmental review, which Duke will discuss later. First you might ask what does the safety review consider? There are two types of safety issues, current operating issues which are dealt with now and aging management issues that are dealt with in license renewal.

Under the current operating license, the NRC's regulatory oversight deals with current safety issues. We do not wait for a plant to come in for license renewal before requiring them to address any issue. Because the NRC has or is dealing with those issues such as security or emergency planning, we do not re-evaluate them in license renewal.

The license renewal safety review focuses on aging management issues and the programs that the licensee has already implemented or will implement to maintain the equipment safely. The safety evaluation

report is independently reviewed by the ACRS. The ACRS is a group of nationally recognized technical experts in the nuclear safety area that serve as a consulting body to the Commission itself. They review each license renewal application as well as the Stat staff's Safety Evaluation Report and form their own conclusions and recommendations and report them directly to the Commission.

The environmental review evaluates the impact of license renewal on a number of areas. These areas include, among others, ecology, hydrology, cultural resources and socioeconomic issues. As I said earlier, Duke will discuss the environmental review in greater detail next.

The next slide will discuss the license renewal process. You might ask, how does all this come together? This slide really gives a big picture overview of the license renewal process. And as you can see from this slide, the process involves two parallel paths; the safety review and the environmental review.

The safety review involves the NRC staff review and assessment of the technical information that's contained in the licensee's application. There's a team of about 30 NRC technical reviewers and

contractors back at the NRC Headquarters in D.C. who are conducting the safety review right now. And the team is also supported by the technical experts at three different national laboratories including Argonne, outside of Chicago, Brookhaven in Long Island New York and Pacific Northwest in Washington State. So there's a lot of expertise in the team conducting this review.

The staff safety review focuses on the effectiveness of the proposed aging management programs for these plants systems, structures and components that are within the scope of license renewal. The NRC staff reviews the effectiveness of these programs to insure that the plant safety can be maintained throughout the license renewal term.

The safety review also focuses on the application's time limited aging analysis. Each original design analysis that had assumed a 40 year life must be reevaluated to extend the 40 year term to a sixty year life term for license renewal. This safety review process also involves audits and on site inspections. These inspections have been conducted by a team of inspectors pulled together from both Headquarters as well as NRC's Regional Office in Chicago.

The results of their inspections were documented in separate inspection reports. And the results have of the staff's safety review as well as the results of inspections will be documented in the Safety Evaluation Report. And a copy of that will be provided to the ACRS for independent evaluation. Both the regional scoping and aging management review inspections have been completed. And we are in the process of writing the Safety Evaluation Report right now.

The second part of the review process involves an environmental review with scoping activities and developing a draft supplement to the GEIS, or Generic Environmental Impact Statement, for license renewal of nuclear plants. And this has been published for comment. And eventually we'll be issuing a final supplemental to the GEIS for license renewal of nuclear plants which will address the comments that we receive here today at this meeting or in the future from any written comments.

So, as you can see from the slide, the final agency decision on whether to approve or deny the application will require a number of things. A Safety Evaluation Report, which documents the results of the safety review; the final supplement of the

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Generic Environmental Impact Statement, which documents the results of the environmental review as well as inspection reports that documents the results from the Regional inspections. All three of these reports will be factored in as well as the independent review of the ACRS into the final agency decision.

And that concludes the license renewal process.

MR. WHEELER: Good evening. I'm Duke Wheeler and I'm the Environmental Project Manager responsible for the environmental review that's being performed to support the license renewal application for Exelon for license renewal of Quad Cities Units 1 and 2.

The National Environmental Policy Act of 1969 requires a systematic approach in evaluating the impacts of proposed major Federal actions. Consideration is to be given to environmental impacts of the proposed action and mitigation for any impacts believed to be significant. Alternatives to the proposed action including taking no action on the applicant's request are also to be considered. Our environmental impact statement is a disclosure tool and it does involve public participation. The NRC

regulations require that an environmental impact statement be prepared for license renewals.

Our decision standard, stated perhaps a bit more simply than what you read on a this slide, is basically, our are environmental impacts of the proposed action great enough that maintaining the license renewal option for Quad Cities Units 1 and 2 is unreasonable. And I'd like to point out at this time that we, the NRC, do not decide whether or not Quad Cities will operate for an additional 20 years. Other regulatory agencies and the licensee will actually make that decision.

Now, this slide is just an expansion of that bottom that you saw on Kimberly's Kimberley's, I think it's Slide 5 that you have, the bottom line was the path for the environmental review. This is an expansion of that. And basically where we stand in the process, the applicant did submit their application back in January the 3rd of this year through the Federal Register and other avenues. We publicized our intent to prepare an environmental impact statement.

One of the early phases of our process that's laid out by the National Environmental Policy Act was referred to as the scoping process. And there

is an opportunity there for public participation. And basically the scoping process is, it's an activity whereby we receive comments from interested members of the public that help us to scope out the bonds of the environmental review for the various disciplines that we're going to be performing.

We also had a site audit. A team of environmental experts came out and visited the site in March. And also we had a public meeting in April, as John mentioned a bit earlier, another opportunity for public participation. And that was just a part of the scoping process.

After the site audit, if it's determined that we still don't have enough information for us to prepare our environmental impact statement, then we will send a formal request for additional information out to the licensee. We did that. They responded. We now have all the information we need. And we then published a draft of our environmental impact And some of the alphabet soup here is statement. This is a Generic Environmental GEIS. Statement that we published several years ago. And it addressed, it gave common conclusions related to a lot of different environmental issues for license renewal of power plants across the country.

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As each plant comes in for license renewal, we will publish a plant specific supplement to that Generic Environmental Impact Statement. And what I have published here in November is the supplement for Quad Cities Units 1 and 2. That's Supplement 16. And this meeting here is an opportunity for the public to provide us their comments on that Draft Environmental Impact Statement.

Once we get comments in from the public, and we'll go back, and because it's a draft, we'll take a look at it ourselves to see if there's any parts of it that need to be tweaked to be put into final form. And then July of 2004 we're going to be on schedule to publish our final environmental impact statement.

And I'd like to conclude my comments at the moment at this portion of it, turn the meeting back over to Chip. And then I'll be followed by our team leader, Bruce McDowell, who will get right into the real substance of what our environmental findings are. Chip?

MR. CAMERON: Okay, thanks, Duke. And before we get into the substance, let's see if there's any questions about the NRC process or about the NRC itself. Kimberley gave us a little bit of

1 information on what our responsibilities are. And if 2 you can just give us your name and affiliation, if 3 appropriate. 4 MS. REGAN: Hi, my name is Molly Regan, 5 and I'm with ICAN. And you made a comment, Duke, that 6 other agencies and Exelon would be the ones that would 7 determine whether this license is renewed or not. 8 MR. WHEELER: Right. 9 MS. REGAN: So does that mean that the NRC 10 does not determine --11 MR. WHEELER: Right, we do not. 12 MS. REGAN: What agencies then --13 MR. WHEELER: State regulators have a give 14 say in whether or not the plant will operate and under 15 what conditions. But what other Federal 16 MS. REGAN: 17 agencies are involved in the final determination of 18 issuing a license? 19 MR. CAMERON: I think that one thing we 20 need to make clear here is that Duke didn't say that 21 other agencies were involved in the decision to renew 22 the license but whether to continue operating. In 23 other words, the company needs an approval from the 24 NRC in order to operate. But it's the company's

business decision about whether they actually will

operate and the State agencies who have an influence 1 2 on whether the company will operate and at what rates. What agency is that, Duke? 3 MR. WHEELER: I would have to defer to the 4 5 licensee, excuse me. If you're asking which State 6 agency is the one that determines whether or not they 7 can or cannot operate? 8 MR. CAMERON: Well, when you were 9 referring to the statement that Molly was asking about 10 and you said that other agencies and the licensee 11 would be involved in whether the plant actually would 12 operate. 13 MR. WHEELER: My real message was although 14 we issue the license to operate we are not the ones 15 who make the actual decision as to whether or not they 16 really do operate. It's our license that they must 17 have in order to operate. But it's not our decision 18 as to whether or not they actually will operate. That 19 decision is a very large part up to, among others, the 20 licensee. 21 MR. CAMERON: Is that clear, Molly? It's 22 a distinction perhaps between the safety aspects of 23 operation and the business economic aspects of

Do you want us to go further?

operation.

1	MS. REGAN: So is it the State where it's
2	located? So it's just Illinois that has a
3	determination in this? It's not any of it wouldn't
4	be like Iowa agencies would have anything to do with
5	that?
6	MR. WHEELER: I would ask if there is a
7	representative from Exelon here that can shed some
8	light on who you have to deal with in order to get all
9	the permissions you need to operate the plant. Can
10	anybody
11	MR. CAMERON: Fred, do you want to take a
12	shot at this or?
13	MR. STORMER: Molly, to answer your
14	question, I think the question that you're asking
15	I'm Bill Stormer, Site Communicator from Exelon
16	Nuclear. I want to clarify your question. I think
17	the question that you're asking, Molly, is who makes
18	the decision whether to renew the license or not, who
19	gives us the final permission as Exelon to operate the
20	plant. Is that the question you're asking?
21	MR. CAMERON: You're going back to the NRC
22	statement again.
23	MS. REGAN: Maybe I should read what I
24	wrote down when Duke was speaking. Other agencies and
25	Exelon will determine whether or not Quad Cities 1 and

2 will have their license renewed. And my question 1 2 was what other agencies? 3 MR. CAMERON: Duke, and just to make sure 4 that we know what you were saying, did you say that 5 other agencies would make the decision about whether 6 the license was renewed or --7 MR. WHEELER: No. 8 MR. CAMERON: -- they would operate. 9 Okay. First of all, --10 MR. WHEELER: Right. The decision on 11 whether or not to renew the license is an NRC 12 decision. Does that clarify anything? 13 MR. CAMERON: That's one thing. 14 Once the renewed license MR. WHEELER: 15 then is issued, it's up to the utility and other 16 regulators to decide what they want to do with what 17 that renewed license will allow. 18 MR. CAMERON: John, do you want to try to 19 shed some light on this for us? 20 MR. TAPPERT: Yeah, I don't know if I can 21 or not but the point we're trying to make with that, 22 the NRC is a the sole regulatory authority for issuing 23 the license, okay? So we're going to make the 24 determination some time late next year whether to 25 extend their license for another 20 years or not. The

1 distinction that we're trying to make in the 2 presentation is just because we extend that license to 3 2032, they may or may not operate during that period 4 of time. They have a license to operate. 5 have a license to drive. You may chose not to drive 6 for any number of reasons. You don't have a car. You 7 don't have, you know. They may decide for economic 8 reasons it's not appropriate to continue to operate 9 the facility but they have a license. 10 That's not to say that the NRC just issues 11 a license and then walks away. There's a continuing 12 and ongoing oversight process to make sure that if 13 they do operate they will operate safely. 14 sure if that helps at all but that was the point we 15 were trying to make there. 16 MR. CAMERON: Okay. Thanks, Molly, for at 17 least allowing us to try to clarify what we were 18 talking about there. 19 How about other questions? Anything on 20 process or the NRC before we go on to the findings? 21 And if something comes up during the meeting, a 22 question, we'll deal with it then. 23 Duke, thank you and Kimberly Kimberley and

And now we're going to go to Bruce McDowell who's going to talk about the findings in the Draft Environmental Impact Statement.

MR. MCDOWELL: Good evening, I'm Bruce McDowell from the Lawrence Livermore Laboratory and I was am the team leader for the team of experts that prepared the Supplemental Environmental Impact Statement for Quad Cities license renewal.

In the Generic Environmental Impact
Statement for license renewal, the NRC identifies 92
environmental issues that are evaluated for license
renewal. 69 of these issues are considered generic or
Category 1, which means that the impacts are the same
for all reactors or the same for all reactors with
certain features such as plants that use water from
large rivers.

For the other 23 issues, referred to as Category 2, the NRC found that the impacts were not the same at all sites and therefore a site specific analysis was needed. Only certain issues addressed in the Generic Environmental Impact Statement are applicable to the Quad Cities plant. For those generic issues that are applicable to Quad Cities, we accessed if there was any new information related to the issue that might change the conclusion in the

Generic Environmental Impact Statement. If there is no new information then the conclusions of the Generic Environmental Impact Statement are adopted.

If new information is identified and determined to be significant, then a site specific analysis would be performed. For site specific issues related to Quad Cities, site specific analyses were performed. Finally, during the scoping period, the public was invited to provide information on potential new issues. And the team, during their review, looked to see if there were any new issues that needed evaluation.

Environmental Impact Statement, which I'm going to call the GEIS, an impact level is assigned. These impact levels are consistent with the Counsel on Environmental Quality. For a small impact the effect is not detectable or too small to de-stabilize or noticeably alter any important attribute of the resource. For example, the plant may cause loss of adult and juvenile fish at the intake structure. If the loss of fish is so small that it cannot be detected in relation to the total population in the river, the impact would be small.

For a moderate impact the effect is sufficient to alter noticeably but not de-stabilize the important attributes of the resource. Using the fish example again, if losses of intake cause the population to decline and then stabilize at a lower level, the impact would be moderate. And finally for an impact to be considered large, the effect must be clearly noticeable and sufficient to de-stabilize important attributes of the resource. So if losses at the intake cause fish population to decline to the point where it cannot be stabilized and continually declines, then the impact would be large.

As Kim said earlier, there's a team with a broad expertise that wrote this supplemental environmental impact statement. And these are some of the areas, these are the areas that we addressed in our analysis. The staff has considered information from a broad range of sources during the development of this supplemental EIS. We have considered the licensee's evaluation of environmental impacts that was submitted with the license application.

We have conducted a site audit during which the staff visited the plant and interviewed staff personnel. We talked to Federal, State and local officials as well as local service agencies. In

addition, we have also considered all the comments received from the public during the scoping period. These comments are listed in Appendix A along with the NRC responses. The information received from all these sources is the basis for the analysis and a preliminary conclusions in the draft EIS that you have in front of you.

In Chapter 2 of the draft supplemental EIS, we discuss the plant and the environment around In Chapter 4 we looked at the potential the plant. environmental impacts for an additional 20 years of operation for the Quad Cities nuclear station. team looked at issues related to the cooling system, transmission lines, radiological impacts, socioeconomic impacts, ground water use and quality and threatened and endangered species. Each of these in detail are discussed in the supplemental EIS. I'll take just a few minutes to identify the highlights of our review.

One of the issues we looked closely at is the cooling system for the Quad Cities plant. This is the layout of the cooling intake and discharge canals. Although there are a number of Category 1 issues related to the cooling system, and remember that we said the Category 1 issues are those that have been

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determined to have the same significance for all plants. No new and significant information was identified during scoping by the applicant or the staff during the review of the issues.

The issues that the team looked at on a site specific basis include intrainment and impingement of fish and shellfish, heat shock and enhancement of microbiological organisms. Potential impacts in these areas were determined to be small and additional mitigation is not warranted.

Radiological impacts are a Category 1 issue. As you recall, this means that NRC has made a generic determination that the impacts resulting from radiological releases during nuclear plant operations are small. But because it is often a concern of the public I wanted to take a minute to briefly discuss it. During the site visit we looked at the effluent release and monitoring program documentation. We looked at how the gases and liquid effluence effluents were treated and released as well as how the solid waste were treated, packaged and shipped. This information is found in Chapter 2 of the Draft Supplemental EIS.

We also looked at how the applicant determines and demonstrates that they are in

compliance with regulations for a release of radiological effluence. The licensee monitors the near site and on site locations for airborne releases and direct radiation. There are other monitoring stations beyond the site boundary including locations where water, milk, fish and food products are sampled. Releases from the plant and the resulting off site potential doses are not expected to increase on a year to year basis during the 20 year license renewal period.

No new and significant information was identified during the staff's review. The public's input during the scoping process or other evaluation or the evaluation of other available information.

The generic EIS determines that the impacts of the 69 Category 1 issues were small based upon the information known at that time. As part of my team's review we looked at all information collected during the scoping process to identify any information that was both new and significant with regard to any one of these 69 issues.

We looked at the information developed by the licensee, information developed independently by my team and information received during the public comment process. We determined that none of the

information was both new and significant. Therefore, the conclusions of the generic EIS are adopted in this draft supplemental EIS.

The last issue from Chapter 4 I'd like to discuss is that of threatened and endangered species. The only Federally listed aquatic species that currently occurs in the vicinity of the Quad Cities plant is the Higgins eye pearly Muscle mussel. The essential habitat for this species is located about one mile downstream from the site.

There are a number of terrestrial species listed as threatened or endangered that could occur in the range of the Quad Cities site and transmission lines. These include the bald eagle, Indiana bat, the river otter, the Iowa Plastering pleistocene snail and the western hognose snake. During winter migration, bald eagles visit open water in the Mississippi River caused by the plant's thermo thermal discharges. They also use the area for summer nesting and there is a known nest about eight miles north of the site.

The Indiana bat, river otter, Iowa Plastering pleistocene snail and western hognose snake could occur in the counties where the plant and the transmission line are located. Since the licensee does not plan any refurbishment or construction

activities as part of relicensing, the natural area where these species would be found would not be disturbed. This would also be true for the three threatened plant species; the eastern and western prairie fringe Clover orchid and the prairie bush clover.

Therefore, the staff's preliminary determination is that the impact of the operation on the Quad Cities plant during the license renewal period on threatened and endangered species would be small.

The staff also considered cumulative These are impacts that are minor when impacts. considered individually but significant when considered with other past, present or reasonably foreseeable future actions regardless of what agency or person undertakes the other actions. considered cumulative impacts resulting from operation of the cooling system, operation of transmission releases of radiation and radiological materials, sociological impacts, ground water use and quality impacts and threatened and endangered species impacts.

These impacts were evaluated to the end of the 20 year license term, license renewal term. The

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geographical boundary of the analysis was dependent upon the resource. For instance, the area analyzed for transmission lines was of course different than the area analyzed with the cooling water system. The staff's preliminary conclusion is that any cumulative impacts resulting from the operation of the Quad Cities plant during the license renewal period would be small.

The team also looked at the uranium fuel cycle and solid waste management and decommissioning. All issues for uranium fuel cycle and solid waste management as well as decommissioning are considered Category 1. And for these issues, no new and significant information was identified and we therefore adopted the conclusions of the Generic Environmental Impact Statement.

Our team evaluated the potential environmental impact associated with the Quad Cities plant not continuing operation and replacing this generation with alternative power sources. In 2001, Quad Cities Units 1 and 2 generated 13 billion kilowatt hours of electricity. The team looked at no action alternatives, -- action alternative, new generation from coal-fired, gas-fired and new nuclear, purchased power, alternative technologies such as

wind, solar and hydro power and then a combination of alternatives.

types of issues. For example, water use, land use, ecology and socioeconomics that we looked at for the operation of Quad Cities during the license renewal term. For two alternatives, solar and wind, I'd like to describe the scale of the alternatives that we considered because scale is important in understanding our conclusions.

energy available in Illinois and the current conversion efficiencies of photopotanicks [?] panels, these cells would produce about 100 kilowatt hours per square meter per year. As such, about 120 million square meters or about 46 square miles of cells would be required to replace the generation from the Quad Cities plant.

Regarding wind power, wind turbans turbines have capacity factors of between 30 and 35 percent. As such, at least 4,200 megawatts of wind power would have to be developed to replace Quad Cities 1800 megawatts. To put this in context, in 2002, total wind power capacity in the United States was 4,500 megawatts. In other words, the total wind

power in the United States would have to double to replace the generation of the Quad Cities.

Due to these scale issues and other siting requirements of reasonable alternatives, the team's preliminary conclusion is that the environmental impacts of alternatives, at least in some impact categories, reach moderate or large significance.

So to review; in their Generic Statement, examined Environmental Impact NRC environmental issues at all sites and found that the same conclusion could be made for 69 Category 1 In our analysis we found no information that was new and significant and we adopted the GEIS conclusions. We also performed site specific analysis for Category 2 issues applicable to Quad Cities. And lastly, we found no new impacts that were not discussed in the GEIS.

Category 1 issues presented in the GEIS, again we found no information that was new and significant. Our team analyzed the remaining issues in the supplemental EIS and we found the environmental effects resulting from these issues were also a small significance with one exception. On one segment of the transmission line the induced currents were

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1	calculated to be six miliamps. Since this slightly
2	exceeds the national, the NESC standard of 5 miliamps,
3	we judge the impact to be a moderate significance.
4	Since this line is not owned by the licensee, NRC has
5	notified the owner of its findings.
6	Lastly, we found that the environmental
7	impacts of alternatives, at least in some impact
8	categories, reached moderate or low significance.
9	. Now, I turn it back to Chip, see if
10	there's any questions.
11	MR. CAMERON: Okay, thanks, Bruce. Bruce
12	talked about the number of different categories of
13	environmental impacts that the NRC looked at,
14	including alternatives. Are there any questions about
15	some of the potential impacts, findings of the Draft
16	Environmental Impact Statement?
17	Yes, sir. And if you can just give us
18	your name, please.
19	MR. BROWN: My name is Bennett Brown. I'd
20	like to know more about how the directory expense
21	system of this plant in particular was considered in
22	the plant's specific environmental impact statement.
23	MR. MCDOWELL: The which?
24	MR. CAMERON: Can you just state that
25	again for us?

MR. BROWN: The Quad Cities plants both are Mark 1 Reactors from General Electric. And they their are containment system, that the primary containment, is a concrete shell designed to contain the reactor under high pressure. The secondary containment is a one million gallon donut shaped tank of water under ground. And in the '70's, after five years of operation, these reactors were identified as having been designed incorrectly. The tank was recognized as being under sized and a recommendation was made by the NRC that modifications needed to be made to all of those 18 plants because there was a 90 percent likelihood that if called upon in the event of an accident that that secondary containment system would fail, the 90 percent likelihood that it would fail.

To address that problem the Mark 1 owners, the collaboration of companies that own Mark 1 reactors from General Electric, came up with a solution which was approved by the NRC as a patch work fix to the design of these plants to bypass containment in the event of an accident by connecting the torus, the donut shape tank of water, to the stack so that if the pressure in the cooling, the secondary cooling tank in the torus builds up above 30 PSI, the

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reactor operators operates under several hundreds of PSI, I believe. If the pressure in that donut shaped tank rises above 30 PSI, then a plug is blown and butterfly valve at the option of plant control operators can be opened to the stack. And then the emissions are released to the atmosphere directly bypassing the containment.

I'm wondering how that modification, my question then is how is that modification to the original plant taken into consideration in your analysis of the risk of radioactive release to the public?

MR. CAMERON: Okay, and thank you, And I think there's at least two parts to this and we're going to go to Bob Palla. But the first part, Bob, is comment on Bennett's to characterization of the issue. And I think the second part is is that an issue that the NRC deals with under the normal operating regulation framework or is it something that would be looked at in terms of license renewal either in the safety evaluation or the environmental impact statement.

MR. PALLA: Yes, let me explain. And it's a fairly accurate characterization of what this system is. It may be called a Torus Vent system. And it's

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true that the owners group at NRC's urging, all of the Mark 1 plants with maybe some exceptions on -- there may be an exception with an isolation condenser Mark 1 plants. But they implemented a venting system that would allow the Torus pressure to be relieved in certain events. And by relieving the pressure, in essence you have a controlled release and you avoid a catastrophic rupture of the containment, the primary containment or the torus itself.

The types of sequences that this vent was intended to address are beyond design basis. These involve multiple failures of the containment heat removal system. And the scenario that typically forms a basis for this plant improvement is a scenario in which the containment heat removal is completely lost and for an extended period of time. And I'm talking on the order of a day. It could be 24, 36 hours.

The reactor scrams. There's no heat removal from the torus where the heat would normally be drawn from and put removed through the heat exchangers in release. So over time the water in the torus would heat up, boil, create a bunch of steam and then gradually over pressurize the containment unless it was vented. And all this time the core is still being kept cool. Water's being injected to the core.

But it's being steamed into the containment and the pressure's increasing. So this vent system was installed to allow the containment to be vented without a loss of the containment function.

And if you lost the containment and if you lost the torus, you could drain the water out of the torus and lose. Tthis is the water you want to have so you can inject it to the core. So, this is, again, it's beyond design basis accident. It's picked up typically in a probablistic risk assessment study, you would look at that class of accidents.

Before the vent was installed, these accidents would go out to about 24 hours or so, leave the and lead to containment failure. The core is cooled at that time but then, as a result of the containment failure, you'd lose injection to the core and then you'd have a core melt. So, you know, at about a day, you know, a little after a day.

With this vent system installed, again, you're going to have to lose containment heat removal for over a day before you get into the situation. But with the vent, you'd vent the containment and conceivably would just keep injecting until eventually you would recover containment to your heat removal and you would not melt the core. You would just vent the

1 containment. It's a clean release, so to speak. Core 2 melt hasn't occurred yet. MR. CAMERON: And this venting system, is 3 4 this something that's dealt within the license renewal 5 aging analysis, I guess is the next question or the 6 environment impact statement. 7 MR. PALLA: It's not really an issue for, 8 it's not an aging related issue. It's a facet of the 9 design now that has been implemented, taking into 10 account risk considerations and severe accident 11 So, it's in essence a system that would analysis. 12 prevent an accident from going the to core damage as 13 a result of containment failure. That's why that 14 system was put in place. 15 MR. CAMERON: Okay, so, Bennett, I think 16 that the answer to the question is, from what I 17 understand --18 MR. PALLA: That was a short answer. MR. CAMERON: -- it won't be dealt with in 19 20 the License Renewal Aging Program or the Environmental 21 Impact Statement. And perhaps, Bob, --22 MR. PALLA: Let me just suggest how is it 23 -- it's not part of anything done in any of the 24 environmental review other than if you've looked at 25 just the severe accident mitigation alternative right

write up in Chapter 5, or back in the Appendix G, we look at the risk profile for the plan. And the risk profile would include -- it has a, every plant has a risk profile, which is in essence a number of unique ways that you can end up with core damage and releases to the environment. And each one of those is assigned a frequency of occurrence and a source term, the quantities of fission products that would be released.

And there is a class of accidents that involve this loss of containment heat removal that would be identified. There would be a certain frequency assigned and a source term would be assigned to that that would correspond to the release as it would occur. So there is a source term and a frequency assigned. And this is one of the components of the risk profile.

That's the starting point for the SAMA analysis. We look at that risk profile and then say, is there some way we can identify a further way to reduce the risk?

MR. CAMERON: You're going to talk about not this particular problem, necessarily, or issue but you're going to talk about the SAMA program in the next presentation. And Bennett, let me ask you if you have a follow up.

1 MR. BROWN: I had several questions, I guess, that arises as a result of this. But before 2 proceeding I'd like to ask a procedural question. Are 3 my questions in this section recorded in the comment 4 section that will be appended to the SEIS? 5 This is, this is on the 6 MR. CAMERON: 7 And the transcript will show this. 8 during the question and answer period someone will ask a question that implies a comment about something that 9 When that does happen, we treat 10 we should look at. 11 that as a comment. We look at that issue. So, 12 other words, it doesn't have to be made formally characterized as a comment. 13 14 Is that the question you're asking? MR. WHEELER: I think, Bennett, possibly 15 16 before you came in, it was pointed out this meeting is We have a transcriber that's 17 being transcribed. 18 getting all this down. And I'll get a copy of the 19 And first of all, I'll put those transcripts. 20 transcripts in the public record. But then in preparing the final Environmental Impact Statement 21 22 that will be issued in July, all of these comments will appear in Appendix A of that final. 23

just block feed right out of the transcript into the

To the extent that it is practical, I will

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1	Environmental Impact Statement. If it gets so long of
2	a project that the document becomes <del>unwheely</del> unwieldy,
3	then I reserve the right to summarize at least
4	somewhat. But the substantial substance of what is
5	being said here will go into the final Environmental
6	Impact Statement at Appendix A.
7	MR. CAMERON: Whether it's offered during
8	the formal comment part of the meeting or was offered
9	during the question and answer, right?
10	MR. WHEELER: Oh, absolutely, yes. Or any
11	one of other ways that I'll get into a little later.
12	MR. CAMERON: I think that's what the
13	concern is.
14	Do you want to ask okay, go ahead,
15	Bennett.
16	MR. BROWN: I'm grateful for your time
17	here and I don't want to take too much of it but it's
18	an issue that's important to me and I think it's a
19	critical question concerning this particular reactor.
20	So if you'll bear with me and dwell on the question
21	for a moment. I'm grateful for your help over the
22	last several months in understanding the process
23	related to this particular plant.
ı	
24	I have several questions that were raised

it's a gradual build up of heat. And as I understand the calculations, an 800 megawatt plant, if it fails to scram for any reason, if there were any failure to stop the reaction, and I'm not talking about a super critical event. I'm simply talking about for one reason or another the plant needs to scram, for instance, the grid were to fail and the plant has nowhere to deliver that power and therefore needs to shut down the power so that the heat that is being generated will not boil the cooling water. So the plant needs to scram.

If that scram were to fail, how many seconds will it take before the heat storage available in the torus, in this million gallon tank, is exhausted before the million gallons of water boils? And as a physicist it's a back of the envelope calculation. A million gallons of water is, you know, four times that gets you liters, which is kilograms. You multiply it by a thousand to get grams and you multiply it by four to get jewels joules. And I think it's five minutes.

So I think in the event of a failure to scram, five minutes from that point, once blow down begins, once the process of blowing pressurized reactor steam into the torus begins, it would be five

minutes before that torus boils. At that point pressures would rise very rapidly. I'm sure you'd agree and it would be a matter of seconds before the direct torus vent system, the system that directly vents the torus to the atmosphere would need to be deployed. So I wanted to clarify your characterization of gradual.

MR. CAMERON: Okay, let me, before you do that, Bob, I know that for at least some of us in the room that we are using technical terms like scram and things like that that people probably, some people, some of us don't understand exactly what that means. And, Bob, this is an important issue and to answer the question. And maybe if we do have one more follow up from you, Bennett, you can, you know, bring us back up to, you know, 50,000 feet sort of and tell us what the implications are of what you're saying so that everybody understands that.

Bob?

MR. PALLA: Now, the type of accident that you're referring to, we call it an anticipated transient without scram. And that could occur to varying degrees. It could be a complete loss or it could be a partial loss of shut down. So if you completely lost the shut down function, you would be

dumping a large quantity of heat into a pool that can only take so much. So it would be a matter of -- I'm not sure that it would be five minutes or whether it would be, you know, an hour. But it would be a relatively short term event.

Now, let me go back to, I guess, just to put this kind of an accident in the right kind of box, this is, again, an accident that involves the failure of multiple safety systems, systems that were put there deliberately to prevent that kind of an accident. And that was the ATWS rule that mandated certain things be done to address that kind of event because it would be a challenge to the containment integrity. And it would be a challenge to the integrity with or without this torus vent.

The torus vent is not large enough to, in and of itself, relieve the pressure and have everything just maintained at an adequately low pressure. Even if the torus vent actuates, you're still going to over-pressurize the containment in this scenario. And so I don't think it really affects the, ultimately time of release. It'll have some influence but it won't have a radical impact on what happens in that event. If it's a complete loss of shut down, you're going to basically pump the containment up with

steam and not be able to control the pressure in the containment with or without that vent. You'd have to have a vent that it would probably be about three foot in diameter to deal with the K decay heat levels that I think you'd have in that accident.

MR. CAMERON: Thanks, Bob. And let me ask

John to try to put all of this and Bennett's questions

and concerns in a context.

MR. TAPPERT: Yes, I just want to bring it back to what your original question was, was the vent considered in the environmental review and things of that nature. And the answer is, yes. I mean, Bob's next presentation up here is going to be about severe accident mitigation charges. And he looks at these beyond design basis. These very rare but potentially high consequence events. So that as in the Chapter 5 analysis, in the review. And in the Generic Environmental Impact Statement, we also looked at severe accidents.

So, obviously Bob can go into bone crushing detail in all these scenarios. He'd be happy to do that with you after the meeting, if you'd like. But the short answer is these scenarios were, in fact, considered and Bob's next presentation's going to cover some of that.

1 MR. CAMERON: Okay. So, after Bob's 2 presentation, if you have more questions on this, 3 let's go to those at that point, okay? 4 MR. BROWN: Thank you. 5 MR. CAMERON: All right. Yes, madam, and 6 could you just tell us your name, please? 7 MS. MONAHAND: Dorothy Monahand. I just 8 had a question about how you determine fatigue value 9 of the properties over the 20 year period. I know 10 personally that 20 years can be very debilitating. 11 MR. CAMERON: Okay. I think that's an 12 aging issue for perhaps Kimberley. And 13 Kimberly Kimberley, is the question clear to you? 14 MS. CORP: Right, she's asking about the 15 fatigue analysis. And they reevaluate all of those 16 figures and the staff does an independent calculations 17 of their own from the data that is from the plant. 18 They use actual plant data to -- they use a Reg Guide 19 of 1.99 sets out the guidelines for their fatigue 20 And the results of that will be in the analysis. 21 Safety Evaluation Report, which will be, the draft 22 will be published in March of next year. So, 23 currently that's under review right now. 24 MR. CAMERON: So, if someone wanted to see 25 more details on how we do the fatigue analysis, they

1	could look, first of all, at this regulatory guide
2	that we have
3	MS. CORP: Right.
4	MR. CAMERON: 1.99.
5	MS. CORP: Yes, that is correct. But
6	it'll be specifically in Chapter 4 of the SER when
7	it's published, the Time and Aging Analysis.
8	MR. CAMERON: Does that give you somewhat
9	of an answer? I mean, we have documents and analysis
10	that deal with it. But does that answer your question
11	or can we provide some more?
12	MS. MONAHAND: Yes.
13	MR. CAMERON: All right. And we'll make
14	sure that we talk to you after the meeting, too, to
15	make sure that we have given you as much information
16	on that as possible.
17	Anything else on the Draft Environmental
18	Impact Statement at this point?
19	Okay. We're going to go to Bob well,
20	Bennett, do you have a question that's not going to be
21	addressed in this presentation? In other words, you
22	have another question on the Environmental Impact
23	Statement?
24	MR. BROWN: Not having heard your
25	presentation yet, I have no way of being able to say

But I wanted to ask a brief that obviously. straightforward question so that you could be sure to include this information in your presentation. considering alternatives and comparing the option to continue the licensor license of this plant versus other power sources, I just want to know where the consideration of this design flow is taken into consideration. So when you look at, for instance, building a new nuclear power plant that would not have this flaw, which is, I understand, it would be easy enough to build a new one that would not have this flaw from these early nuclear power plants. Just if you would, please, point out how the risk factor of an accident and the exposure levels to the public are reduced in that model versus the existing models.

MR. PALLA: You're would probably have to come back to me with a more, you know -- I probably won't hit that enough to satisfy you.

MR. CAMERON: Just let me ask a question of the staff before we go on there. In terms of Bruce's discussion, the discussion in the Draft Environmental Impact Statement on looking at alternative sources, I think at least we can answer the question of when you look at alternatives, for example, another nuclear plant, did you consider doing

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1 that analysis any specific design issues related to 2 the nuclear plant or did you only, Bruce, look at --3 what did you look at when you look at an alternative 4 for another nuclear plant? 5 MR. MCDOWELL: The alternatives, what our 6 task was is to evaluate the environmental impacts of 7 alternatives. We analyzed the environmental impacts 8 of the operation of Quad Cities. We looked at the 9 environmental impacts of the new nuclear plant, a 10 coal-fired plant, a gas-fired plant and all the 11 different range of alternative technologies. And we 12 came to a conclusion on the environmental impacts of 13 each one of those. 14 For probably part of the reasons that Bob 15 is going to tell you about, the accident that you're 16 considering I think is dealt with to the NRC 17 satisfaction in the safety space and we didn't 18 consider that specific thing you're thinking about to 19 be an impact area. It was, it's a flaw that I think 20 Bob can talk more about how it's being addressed. 21 So, I just leave that up to him. 22 MR. PALLA: Yes, I'll try to hit on that 23 but we can talk some more if I don't. 24 Okay, you want to go to the next slide 25 there?

I'm with the My name is Bob Palla. Probablistic Safety Assessment Branch of NRC. And I'll be discussing the environmental impacts of postulated accidents. These impacts are described in of the Generic Environmental Section 5 Statement or GEIS. The GEIS evaluates two classes of design basis accidents and severe accidents: The design basis accidents are those accidents. accidents that both the licensee and the NRC staff evaluate to ensure that the plant can safely respond to a broad spectrum of postulated accidents without risk to the public.

The environmental impacts of design basis accidents are evaluated during the initial licensing process and the ability of the plant to withstand these accidents has to be demonstrated before the plant's granted a license. Most importantly, a licensee's required to maintain an acceptable design and performance capability throughout the life of the plant, including any extended life operation.

Since the licensee has to demonstrate acceptable plant performance for the design basis accidents throughout the life of the plant, the Commission has determined that the environmental impact of design basis accidents are of small

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significance. Neither the licensee nor the NRC is aware of any new and significant information on the capability of the Quad Cities plant to withstand design basis accidents. Therefore, the staff concludes there are no impacts related to design basis accidents beyond those discussed in the GEIS.

The second category of accidents evaluated in the GEIS are severe accidents. Severe accidents are, by definition, more severe than design basis accidents because they could result in substantial damage to the reactor core. The Commission found in the GEIS that the risk of severe accident in terms of atmospheric releases fall out onto bodies, open bodies of water and releases the ground water and societal impacts. These are all small for all plants.

Nevertheless, the Commission determined that alternatives to mitigate severe accidents must be considered for all plants that have not done so. We refer to these alternatives as severe accident mitigational alternatives or SAMA's, for short. The SAMA evaluation is a site specific assessment and is a Category 2 issue as explained earlier. The SAMA review for Quad Cities is summarized in Section 5.2 and described in detail in Appendix G of the GEIS supplement.

Now, the purpose of performing the SAMA evaluation is to insure that plant changes with the potential for improving severe accident safety performance are identified and evaluated. The scope of plant improvements that were considered include hardware modifications. And along that line of are things like filter vents, which would be a similar type of vent for this hardened torus vent that we're talking about. But it would include an added filter. Large vents, larger sized vents that could accommodate anticipated transients without scram. These are the kinds of things, the hardware mods that we looked at.

Also looked at procedure changes, training program improvements as well as additional changes. Basically a full spectrum of potential changes. And the scope includes SAMA's that would prevent core damage as well as SAMA's that improve containment performance given that core damage event were to occur.

The SAMA evaluation process consists of a four step process. The first step is to characterize the overall plant risk and the leading contributors to risk. This typically involves the extensive use of the plant specific probabilistic risk assessment study or PRA. The PRA is a study that identifies different

combinations of system failures and human errors that would be required for an accident to progress to either core damage or containment failure.

The second step in the process is to identify potential improvements that could further reduce risk. The information from the PRA, such as dominant accident sequences, is used to help identify plant improvements that would have the greatest impact in reducing risk. Improvements identified in other NRC and industry studies, as well as SAMA analysis for other plants are also considered.

quantify the risk reduction potential in the implementation costs for each improvement. The risk reduction and the implementation cost for each SAMA are typically estimated using a bounding analysis. The risk reduction's generally overestimated by assuming that the plant improvement is completely effective in eliminating the accident sequences it is intended to address. And the implementation costs are generally underestimated by neglecting certain cost factors such as maintenance costs and surveillance costs that would be associated with the improvement.

The risk reduction in the cost estimates are used in the final step to determine whether

implementation of any of the improvements can be justified. In determining whether an improvement is justified, the NRC staff looks at three factors. The first is whether the improvement is cost beneficial. In other words, is the estimated benefit greater than the estimated implementation cost of the SAMA. The second factor is whether the improvement provides a significant reduction in total risk. For example, does it eliminate a sequence or a containment failure mode that contributes to a large fraction of plant risk. And the third factor is whether the risk reduction is associated with aging affects during the period of extended operation. In which case, if it was, would consider implementation of improvement as part of the license renewal process.

Preliminary results of the Quad Cities' SAMA evaluation are summarized on this slide. 280 candidate improvements were identified for Quad Cities based on review of the plant specific PRA, relevant industry and NRC studies on severe accidents and SAMA analysis performed for other plants. Exelon reduced this set to a set of 15 potential SAMA's based on a multi-step screening process.

Factors considered during this screening included whether the SAMA is not applicable to Quad

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Cities due to design differences, would it involve major plant modifications that would clearly exceed the maximum obtainable benefit or would provide only a minimal risk reduction based on review of the PRA. A more detailed assessment of the conceptual design and costs was then performed for each of the 15 remaining SAMA's. This is described in detail in Appendix G of the GEIS supplement.

The cost benefit analysis shows that four of the 15 SAMA's are cost beneficial when evaluated in accordance with NRC guidance for performing regulatory analysis. All four cost beneficial SAMA's involve procedural improvements rather than hardware modifications.

As shown on the next slide, the cost beneficial SAMA's involve developing procedures to operate equipment locally following loss of 120 volt bust by using temporary connections to the second unit. The second procedure involves, that would be developed involves procedures to manually control feedwater given the loss of a 120 volt DC control power bus. 120 volt DC losses are important in the risk profile in this plant. That's why these improvements come to the top.

1 The third procedural enhancement involves 2 terminate developing procedures to reactor depressurization prior to loss of the steam driven 3 4 reactor injection pump so that core cooling can be And the fourth improvement involves 5 maintained. 6 developing procedures to control containment pressure 7 during venting in order to assure adequate suction 8 head for the pumps that are used for core injection. 9 So of these four, for all of the four, 10 none of these four SAMA's are related to aging or 11 managing the effects of plant aging. And therefore, 12 none of them are required to be implemented as part of 13 license renewal. 14 So, to summarize, the NRC's staff's 15 preliminary conclusion is that additional plant 16 improvements to further mitigate severe accidents are 17 not required at Quad Cities as part of license 18 renewal. It's necessary for me to point out, however, 19 that even though they're not required as part of 20 license renewal, the staff intends to pursue these 21 improvements further with Exelon under the current 22 operating license. 23 So, I can take any additional questions. 24 MR. CAMERON: Okay, thanks, Bob. Bennett,

with that perspective, do you want to ask some more

questions about the particular design feature that you're talking about?

MR. BROWN: I think I understand how you, to what extent you included the directory expenses to

MR. PALLA: There were some specific enhancements targeted in that area. These, wWhen one looks at the cost estimates for doing hardware fixes like that, they're hugely expensive. When you look at the probability of the accidents that you're dealing with, and let's take these atwis ATWS events, Their frequency's quite low. Like ten to example. In Appendix G there's a line is the minus eighth. listing of dominate dominant contributors and this one isn't labelled as well as it might have been. But in Table G1 on Page G3, Appendix G, manual shut down, initiating events/accident class is the heading and there's an entry Manual Shut Down. I believe this is a failure to manually shut down the reactor. It's something like basically ten to the minus seven events per year.

You have to account for the frequency in accessing what is the, you know, how much benefit are we going to derive from spending a certain amount of money. So, you've got a combination of an event that

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1	could, in fact, have a large consequence associated
2	but it's probabalistically weighted. And then the
3	costs are compared to that. And these are very
4	expensive mods. These are multi million dollar fixes.
5	So that is one of the mods that would be screened out
6	in the early phases of this process.
7	MR. CAMERON: Bob, I hate to, I hesitate
8	to ask but is there any way that when you talk about
9	a frequency of ten to the minus seven, can you give
10	the people an idea of what that means?
11	MR. PALLA: One in ten million years.
12	MR. CAMERON: Okay, thank you.
13	Any other questions on this SAMA's or on
14	Bruce's presentation on the other types of
15	environmental impacts at this point?
16	Okay, well, Duke is going to give us a few
17	words on how you submit comments. And then we're
18	going to on to you for some more formal comments.
19	Duke?
20	MR. WHEELER: Thank you. First of all, to
21	summarize what our preliminary conclusions are in this
22	
ĺ	Draft Environmental Impact Statement, first of all,
23	Draft Environmental Impact Statement, first of all, the environmental impacts of license renewal is

one exception of the North Nelson Line that Bruce had

pointed out where the induced current was 6 miliamps compared to the National Electric Safety Codes specification of 5 miliamps.

The impacts of alternatives to license renewal range anywhere from small to large and we end up with our preliminary recommendation is that the environmental impacts of license renewal for Quad Cities 1 and 2 are not so great that preserving the option of license renewal is unreasonable.

This slide just has a couple of key milestones in our schedule here that are related to the environmental review portion of our schedule. I did publish the Draft Environmental Impact Statement on November the 4th. We are now pretty much in the middle of our public comment period, which will expire on January the 27th of next year.

And by the way, one comment that I'd like to say is that I'm not going to slam the door shut the close of business on January the 27th. Anything that I do receive by that date I will include in the final environmental impact statement and the comment will be addressed in the final. If I do receive a comment after January the 27th, then I will try to address it.

But we get to a point where it becomes impractical because for me to publish by July, there's

a certain time when I have to get the manuscript over to publication. And backing up from there, there's preparation of the manuscript. Getting it staffed through all the people that need to review it and concur in it. And after January the 27th, I'll just give it my best shot but can't make any promises. And yet the last item on the slide there is that we do have a schedule that does provide for issuance of the final environmental impact statement in July.

primary point of contact with the NRC staff for matters related to the environmental impact statement and our environmental review. The slide also indicates where in the local community copies of our Environmental Impact Statement can be found. The Cordova District Library, the River Valley District Library and then also the Davenport Public Library. And after we mailed this out, I did get on the phone with all three libraries and did verify that they had received their copies of it so it's there if you want to take a look at it.

The last item on this slide also indicates how if you want to get on the Internet, you can access our Environmental Impact Statement. And that link that's on the slide is a pretty long one but it works.

I tried it. It works just fine. However, if you have any difficulties with it or for some reason just are frustrated at the keyboard, give me a call and we will go through it one small step at a time until you get what you're looking for.

Other ways that we can receive comments, you can certainly send a letter into the NRC staff that'll end up on my desk by so called snail mail. I would ask that you address that letter though to the Chief of our Rules and Directives Branch. And what that does is that guarantees that your letter will be put in the public record. Whether or not it goes to Rules and Directives or directly to me, I will nevertheless make sure that all comments that come in do get put in the public record.

It's a long shot but if by chance anybody happens to be in the Washington D.C. or the Rockville, Maryland area where our Headquarters is located, you can certainly stop by and visit with us personally. And I will receive your comments. Whether I write them down or you write them down, though, the comments, before you leave, will end up being put on paper, again, so that I can get them into the public record. Or you can send in comments to the NRC staff at the e-mail address that is at the bottom of the

slide. This address was created for the expressed purpose of providing the public another avenue of communicating with the NRC staff on this environmental review.

Now, it's an e-mail address. It is not a bulletin board. So if somebody who makes a comment wants to see what other comments have been made by other people, you wouldn't be able to get that information directly off of, you know, by coming into us at that e-mail address. There are ways, though, that you can find out what other people have said. And that is we do have a document management system that I will feed all of this into which can be accessed through our web site and all the information can be found through that system. If you want to know what's been said either by e-mail or other letters that have come in. And, of course, the transcripts of this meeting will also be on that web site. If you're not real familiar with our system and are planning to for the first play with it time, my strong recommendation is that you just call me first. might save a lot of frustration.

That concludes my prepared remarks and if there are no questions, I'll turn it back over to Chip.

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1 MR. CAMERON: Okay, thanks, Duke. And 2 that part of the meeting where we ask any of you who 3 want to make a more formal comment on the Draft 4 Environmental Impact Statement to come up and talk to 5 us. 6 And Molly, did you want to make a comment, 7 Molly Regan? You weren't sure at the beginning. 8 was just checking in with you. 9 MR. REGAN: No, I'm fine. 10 MR. CAMERON: Okay. Let's go to Bennett, 11 Bennett Brown to come up and talk to us. Or you can 12 do it from your seat. Okay, thank you, Bennett. 13 Anybody else have a comment at this point? 14 Are there any final questions about schedule or 15 commenting or anything like that that people need 16 answers to? 17 Okay, I just want to emphasize that we're 18 ending early but the NRC staff, and we're going to go 19 to Bennett in a second here, is the NRC staff will be 20 here after the meeting to talk about any of these 21 subjects informally. And talked about we 22 environmental review, we talked about the safety 23 evaluation. I just wanted to mention, just introduce 24 some people on the inspection side of the NRC staff.

course, we do have resident, resident

of

inspectors at every plant. And I wanted to introduce our residents at Quad Cities. Senior Resident Karla Studdard Stoedter and I'll probably never get that right, and Mike Kirth Kurth and they're our residents.

And we do in every region or at least in this region we do have a lead inspector for all the plants in the Region for license renewal and that's Laura Kozak, who's right here. And do we also have our Branch Chief from the Region 3 Office, Mark Ring, who is right here. And we have other NRC staff with us from Headquarters. So if you have some questions, we have the people here to answer them.

And let me go to see if Bennett has another question or comment for us. Bennett?

MR. BROWN: I do have a couple other comments. I just wanted to give other people the opportunity to speak first since I had spoken during the presentation.

My comments fall into two categories and I'm really speaking under two different hats. The first is simply as a physicist interested in energy and safe reliable energy production. I studied physics at MIT. I had the opportunity to work briefly in a reactor. I'm by no means afraid of nuclear reactors but I think it's extremely important that

they be operated safely and that safe designs, that we restrict ourselves to safe designs.

The design of the Quad Cities plant concerns me, not the design, the Quad Cities plant at this point concerns me really on two different grounds. The first one I've spoken to. It's a design issue. And I think it's simply an outdated design. I think there were mistakes made when the design was implemented. I think that the best attempt made possible has been made to correct those design problems so that the plant can live out its 40 year license period.

And I don't feel as a physicist that it's appropriate to renew the license for a plant that bypasses such a fundamental component of its containment and safety systems. To give you an example, it was just this last April there was a I found it shocking that you thought not many people in here don't know what a scram is. How many of you know what a scram is? Come on. Okay, okay, so Forgive me for the antics. And it's significant. appropriate. Everybody should be on board with the conversation.

Last April there was a scram at one of the two Quad Cities reactors. Scrams are hard on the

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plant's valves. The assert pressure transients. They're rapid changes in temperature and pressure throughout the reactor that's hard on materials just like it's hard if you heat up a piece of cookware and then stick it in the sink, it's likely to shatter under the sudden changes in temperature.

Now, the plant is designed to be able to withstand a scram. But it still ages the plant and there are a number of scrams that have occurred at this plant over the years. The most recent one that I'm aware of, though I imagine it's probably not the most recent one considering the frequency with which they occur, was in April.

And in that incident a valve that connects the reactor core to the torus, that I was speaking of earlier, was open and stuck open. I'm not privy to the reasons that that valve was open or the reason that it was stuck open. It's a couple of systems to close it, both failed and a manual attempt to close the valve, as I understand, also failed. So the reactor was scrammed because steam was venting into the torus and that torus water was heating up.

At the time that the reactor was scrammed, the torus water had already heated up from what I presume is its normal temperature of ground

temperature, which would be in the 50 Fahrenheit or 20 degree Celsius and it had already heated up to 95 degrees Celsius. Now, boiling of water occurs at 100 degrees Celsius.

boil, to not have to vent to the atmosphere as long as the scram is initiated at a temperature that's 110 degrees or less. So it was already at 95 and rising and they scrammed and they were successful in scramming. A scram, if everything goes correctly, takes just a few seconds. So no release occurred and it was business as usual and the plant returned to full power after the NRC returned to control the plant to Exelon the following day.

I believe all my details there are correct but I don't work at the plant. I see a couple of you shaking your head. Please --

MR. RING: There's probably several people that can talk to this. My name is Mark Ring and I'm the Regional Branch Chief and I think you got your Celsius and Fahrenheit values a little bit mixed up. I'd have to ask Carl or Mike probably but I think it started in the 70's somewhere, went to about 90 degrees or so. Actions were being taken and I think

1 the high point was maybe around 110, 120, something 2 like that. 3 MR. BROWN: Fahrenheit? 4 MR. RING: Right. MR. BROWN: So the scram was initiated 95 5 degrees Fahrenheit and water -- oh, this is the NRC 6 7 log of the event. Okay, so the scram was initiated at 8 95 Fahrenheit. Water boils at 212 Fahrenheit and the 9 plant is designed to be able to contain the problem as 10 long as the scram is initiated at 110 Fahrenheit or 11 And the water peaked at 118 Fahrenheit and 12 there was no problem. Now I have the details correct. 13 I'm not going to speak anymore about the 14 incident. I don't think it was a particularly unusual 15 incident. I only raise it because I think that this 16 is a serious problem with this reactor. Here we had 17 one valve that failed, stuck open. And we were within 18 15 degrees Fahrenheit of the limit at which had we 19 gone above that we would had to have vent the torus to 20 the atmosphere, as I understand it. 21 I think it's unnecessary to continue 22 operating a reactor beyond the year 2012 given that it 23 has a fundamental design flaw. So that's the first of

my objections to this particular reactor. And I would

like to see the torus vent system addressed in the SEIS.

The second concern that I have is actually more alarming to me. As I say, I'm not an alarmist about nuclear power. I worked for many years with radioactive tracers in a biology lab. And this plant is aged. It's part of a fleet of boiling water reactors that have shown unexpected stresses due to radiation. After the first surprise event at which cracking of a core shroud was observed, I believe that that was in Ohio. Does anybody know, they can fill in the blank for me there? I think it was Davis Bessie but I just don't want to be citing things. I'm not speaking off of notes on this event.

So a boiling water reactor was observed on inspections that weren't particularly routine to look for this so it was a surprise that there were cracks in the core shroud, the shroud that contains the core. And it was identified that they were of serious concern and a survey was undertaken by the NRC of other boiling water reactors in the country.

The cracking was found to be widespread in the core shrouds and was a result of radiation exposure of the metals to the radiation from the core. This particular plant, the core shroud on one of the

reactor cores exhibited severe cracking. The NRC classifies the cracking in this study as none, slight, moderate and severe. And at the Quad Cities plant the core shroud cracking was severe, in some cases with fissures up to a half of an inch in the core shroud wall and they hadn't yet penetrated through the wall but if they did, that would be a disastrous event.

The core shroud is not the only component of the core that is subjected to this radiation and it's subjected to the type of aging that I'm speaking The components that concern me the most are the plates which keep the rods, both the control rods and fuel assembly rods in place so that if sudden insertion of a control rod is necessary, as it is every time a plant scrams, if those plates are worked or have creeped or have buckled, all of these are consequences of radiation exposure of metals, then it's completely plausible that the control rods will be unable to insert as expected during a scram. If a plant fails to scram, the reaction continues and the heat has to go somewhere. That would be the torus, which brings me back to the design flaw of this particular plant.

So, to summarize, I think there are two problems with the Quad Cities plants. Number one,

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they utilize an old flawed design that should be retired. And number two, they are subject to aging. That aging will be 40 years by the time of this license expiration. And the NRC study fairly clearly showed that reactors that were greater than 20 years old exhibited an unexpected spike in their aging characteristics.

To back up, when they look at the plant and looked at whether there was none, slight, moderate or severe cracking in the core shroud and presumably in other internal components of the core that were not so easily examined without full removal of all the fuel assemblies, they found the plants that were younger of 20 years mostly exhibited no aging of this type and plants that were more than 20 years old almost all of them exhibited cracking of this type.

So I think to operate this for 40 years is ify and I think to extend the license for 20 years is unnecessary. So that's the first category of my assignments and it's the first hat I'm wearing as a physicist.

The second comment that I would like to make to the NRC and to be included in the SEIS concerns specifically alternatives considered in the impact statement. And I'd like to address

specifically Section 8.2. So, in Section 8.2.3 you consider new nuclear power generation. And I think it should be mentioned that there's a specific site being considered that Exelon has applied for an advance site permit for the construction of a new nuclear reactor in Illinois. And as you consider alternatives to this aged plant, I think it's relevant to mention that there is an alternative site already being accessed and considered by the NRC.

The second category of alternative I'd like to address is Section 8.2.5.2 and for those of you that have the appendix here, the SEIS draft, that's Pages 8-49 to 8-50. And that's about wind energy. And it's in this regard that I speak not as a physicist necessarily but as a board member and the treasurer of IWORLD [?] Renewable Energy Association. I have been monitoring wind speeds in Iowa for a number of years. I live about 50 miles west of here, slightly north. And I'm part of an NSF funded study that looks at wind correlation.

And I find a section on considering wind energy as a replacement for the Quad Cities plants incomplete and in some cases misleading. So specifically what I would like to see you include in that assessment, you cite four reasons that wind is

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not an alternative to consider for nuclear power. And I'd like to address each one of those in turn, if you will give me the time needed to address that. I won't speak for long and I will be concise.

MR. CAMERON: Go ahead.

MR. BROWN: Thank you. The four arguments against wind that are advanced in the SEIS, in the plant specific environmental impact statement, the first one is that the power required to replace the Quad Cities plant is marginally present in Illinois. Specifically, this was the one point mentioned in today's presentations. Specifically that 4,200 megawatts would be needed to replace the plant capacity at Quad Cities.

And to be clear, with wind you have to distinguish between what is name plate on the turbine, you know, at the base of the tower, a one megawatt turbine, that's not what that turbine produces all the time because wind blows at varying rates. So that's the name plate capacity. And as was stated in your presentation, it would require 4,200 megawatts of name plate capacity of turbines to replace this nuclear power plant. That would, in effect, only be about 1,000 megawatts of consistent power production on average through the year.

So, 4,200 megawatts; that's a lot of power. Illinois only has 3,000 megawatts of Class 4 Wind Sites it says in the SEIS. That's probably not part of your field of knowledge since I see that most of you are within the nuclear realm. The Patel Class, the Department of Energy has classified U.S. land by how windy it is. The higher the number, the more the wind. A Patel Class 4 right now is developable. Wind farms are being built in the United States in Class 4 sites. Illinois has only 3,000 megawatts of Class 4 capacity.

That's not enough. There are an additional 6,000 megawatts of Class 3 sites but those aren't as windy and therefore averaged over the year the cost per kilowatt hour is a little higher if you were to develop that wind. And utilities aren't interested in developing that wind for a good reason. They can put the turbine elsewhere to get more bang for their buck.

What's misleading is to use Illinois numbers. This plant, after all, is on the border of Iowa and Illinois. Illinois has a pathetic wind resource. I don't mean that to any detriment of Illinois but it's not a windy state despite Chicago's monitor.

Iowa is a windy state. In fact, Iowa has enough Class 4 and better sites to replace the Quad Cities, both of the Quad Cities plants 20 times over. Furthermore, north of Iowa, in the Dakotas, we could easily power the entire midwest on turbines. The only issue would be how do you get the power to the population centers? The areas that are easily developed in the Dakotas are not on transmission lines so part of the cost of developing those turbines would have to include transmission.

So the first point here that sufficient power is marginal I think is incorrect. There is more than enough wind power in the vicinity to replace the Ouad Cities.

Second, the NRC document mentions that it is enormously, and this is a quote, enormously expensive to develop these wind resources. I had the opportunity on Friday to attend the Midwest Regional Wind Collaborative. It was a meeting of about 15 people that included utility commissioners from Montana, the Dakotas, Minnesota. It also included legislators from as far south as Kansas. And the purpose of this meeting was to develop a regional plan for developing our wind energy resources and delivering them to market.

The subjects were broad ranging from how to develop it to how to monitor tradeable permits and so on. At that meeting were many utilities. I spoke with a person from Bason [?] Electric, a fairly large rural electric cooperative within what was formally the Map Region. It's a portion of the grid. And this fellow confided in me that a price that they were able to bring wind energy to market. So I will share with you what he gave to me as a public figure, which is that they are currently producing wind at two cents to two and-a-half cents per kilowatt hour.

That figure is flat for 20 years. So for the next 20 years they will be able to produce, and their total production is in the hundred megawatt range of wind. So it's sizeable. Two to two and-a-half cents of kilowatt hour is small when you consider that that includes capitalization of the turbine, it includes the transmission and roads necessary. It includes the interest on the capitalization. It includes the operation and maintenance. And it includes the fuel, which of course is free.

So, two to two and-a-half cents is definitely cost competitive with even a gas turbine, let alone a new nuclear power plant particularly if you omit the Price Anderson Act under which the

nuclear industry has collectively said that nuclear power would not be economically feasible to develop if the nuclear industry had to carry liability for any accidents that were to occur.

So, I think to say that it is enormously expensive to develop is only correct in a silly expense. It is expensive. Power's expensive. It takes a lot of money to build a new nuclear power plant. It takes a lot of money to operate a nuclear power plant and it takes a lot of money to develop wind. But to compare it to other fuel sources I think is simply false. It's not economically expensive to develop in comparison with other fuels. It is economically viable.

The third point that the NRC document brings up is that the land use of turbines would be significant. And I bring this up because it is, after all, an environmental impact statement. Wind may be cheaper. It may not have the risk of accidents. We may not have to deal with the tailings from uranium mining or the terrorist problems with a power plant nor the storage problems with the waste. But wind turbines will take up land. A two megawatt turbine takes up about a quarter of an acre of land that you can farm right up to the turbine.

If you were to replace the Quad Cities plants, they would take about a square mile. It's not a significant consumption of land and it is an environmentally responsible consumption of land. It is a good neighbor to the farmers. In fact, farmers are clamoring to have wind turbines on their farms. I don't see a line of farmers here clamoring to have caskets on their farms. So, I think that the NRC needs to develop that section guite a bit more.

And finally the fourth point that SEIS brings up is that wind, I forget the wording, that wind can only provide intermittent power. That the Quad Cities plants provide a base load power that simply cannot be replaced by wind. This statement is inconsistent with a variety of conclusions that utilities both within the United States and internationally have reached.

To be specific about wind, I feel like there needs to be some education on this point so I'm going to belabor it a little bit. There are three ways in which the wind fluctuates. You get the gust. That's less than one second transience. Then you get the fluctuations that are from a second to ten minutes. And then there are fluctuations that are

longer than that, the very short and medium term fluctuations.

Studies have been commissioned by the independent system operators that maintain the grid. And the conclusion is that the use of wind does not represent any change necessary to the grid of the United States as long as penetration is up to 25 percent. We could replace 25 percent of our electricity generation with wind and not have to change the grid at all. If we were to go beyond 25 percent penetration, we would have to address the fact that wind gusts.

The fluctuations in the wind, today it flows, tomorrow it doesn't, that's at one turbine. If you're talking about replacing two plants that are each hundreds of megawatts, you're talking about many, many turbines at different locations, some of them grouped in a farm so that when a gust hits one turbine it's not at another and averaged over that wind farm, it's a steady output power. And averaged over days, one farm, one wind farm is not particularly windy, another wind farm is. So the output power on a day to day basis even is fairly constant. It is a feasible base flowed production of energy.

One issue, however, is that it's not windy in August. It is windy in January. August is when we need power. It's when people turn on their air conditioners. And as such you have to design the wind production so that you build enough wind turbines so that even in low August wind power generation months you're generating enough power to service August demands. But that's simply a cost issue and when you do out the numbers, as I said, it is economically viable.

So, in conclusion, wind energy, I believe,

So, in conclusion, wind energy, I believe, is a very viable replacement for the Quad Cities plants. In neighboring Iowa, it could be done very easily. In the Dakotas it would require some transmission. The Lady Foundation has done some research on what transmission would be necessary to bring Dakota power to Chicago. It comes out to about two cents a kilowatt hour averaged over the lifetime of those transmission lines. It's not significant even to use Dakota power with new transmission.

So thank you for your attention.

MR. CAMERON: Thank you, Bennett, for those specific suggestions and comments.

Does anybody else have a comment or question before we adjourn for the night? Yes.

MS. PARRIGO PERRIGO: I'm Leslie Parrigo
Perrigo, again. I'm from ICAN and I'm actually, I'm
also on the Board of IRENEW and as a follow up to one
of Bennett's point about transmission lines and where
the power comes from, we have contacted the Iowa
Utility Board and they could not speak for the
Illinois Quad Cities. But the Iowa Quad Cities only
receives 23.6 percent of our power of total net
generation from nuclear sources. Of those nuclear
sources, they come from two separate power plants.
One is in Nebraska and the other one is the Quad
Cities plant.

MR. CAMERON: Thank you, Leslie.

Bennett, can you just make this short? I mean, your comments are very thought provoking and appreciated.

MR. BROWN: Thank you. I realize that I've taken more than my share of the air time if you divide the hours by the people in here. But the primary comment in the SEIS statement was that it would represent a doubling of U.S. wind capacity if we were to replace the Quad Cities plants with wind. That's true but it's, again, it's a irrelevant statistic. In fact, the Senate, as I'm sure you're well aware, considered a law that were required us to

bump up to ten percent of our generation from 1 2 renewables, primarily wind, by 2010. This plant 3 expires in 2012 with its existing license. So already 4 the Senate was considering mandating going from 0something percent of our capacity up to ten percent, 5 which would be like a 20 fold doubling before the 6 plant is even up for its new license period. 7 MR. CAMERON: All right, thank you. 8 9 John, do you want to close us out? 10 MR. TAPPERT: Just want to thank everyone 11 for coming out tonight and sharing your thoughts with And just to remind everyone, if you have some 12 us. 13 comments that you would like to share with us in the 14 future, our comment period does extend till January 15 27th. So, you have our e-mail addresses and our phone 16 numbers. So, please send those to us. 17 And thanks for coming out again tonight 18 and have a good evening. 19 (Whereupon, the above public hearing was

adjourned at 8:50 p.m.)